

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

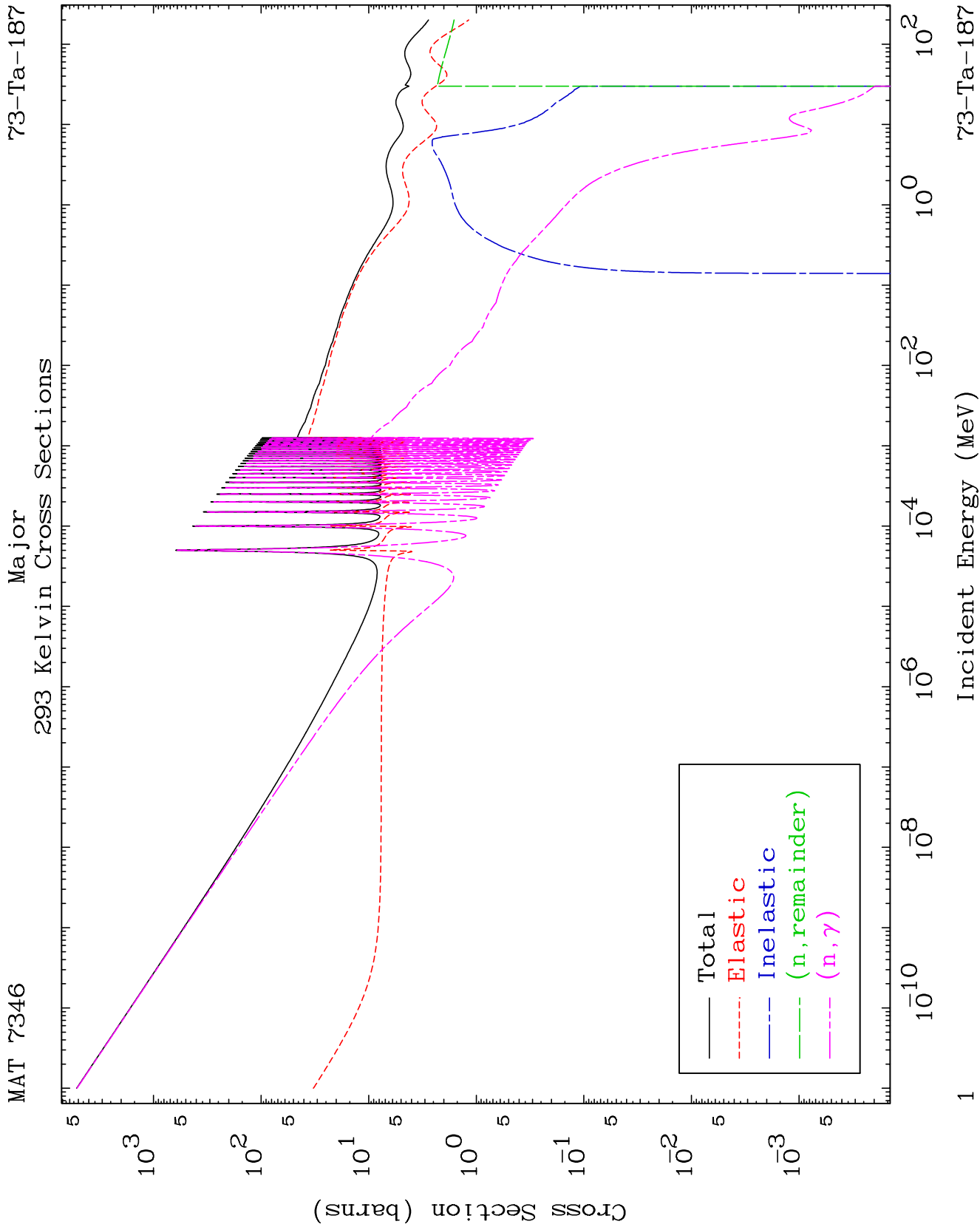
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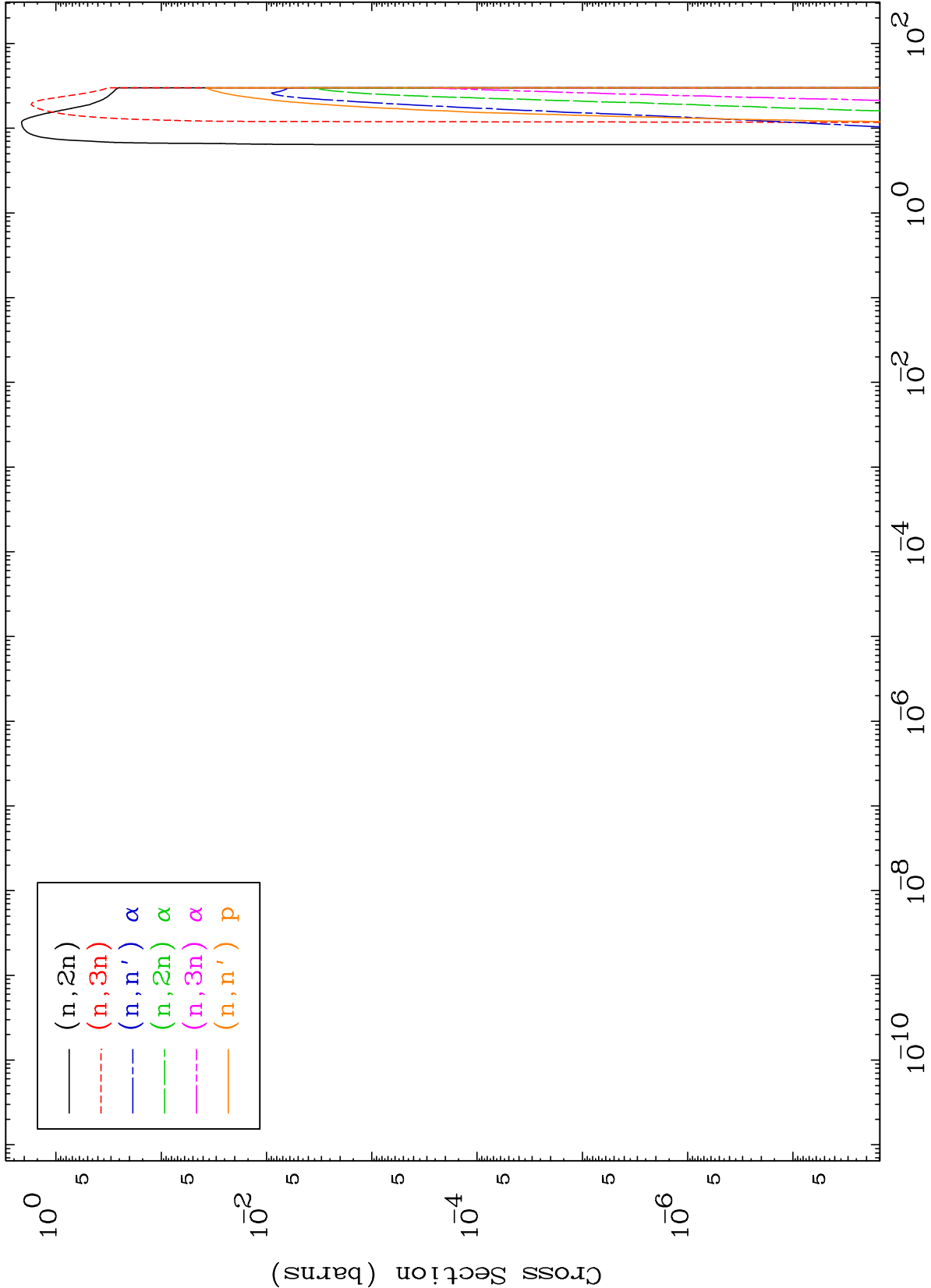
Press Mouse Button to Start

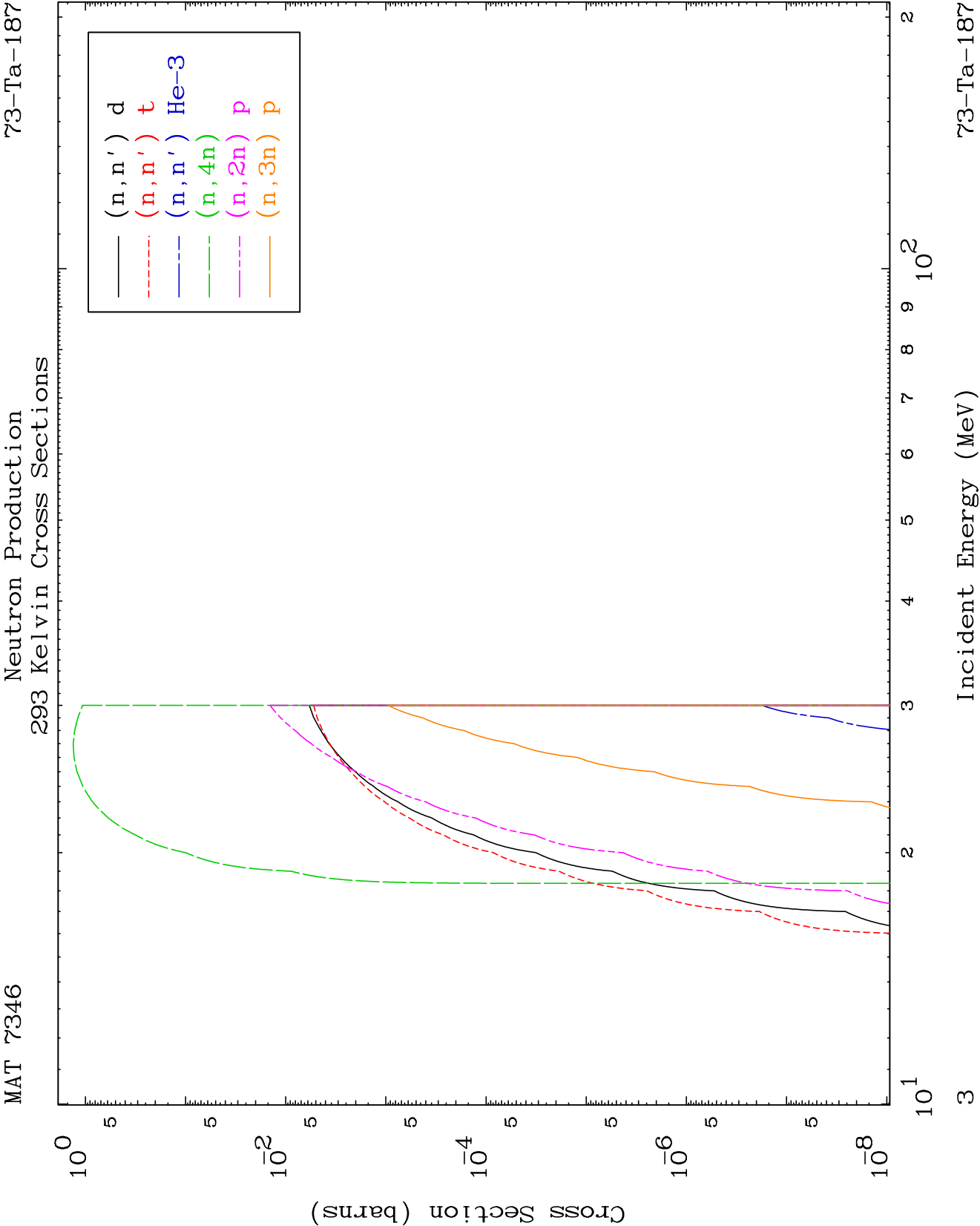


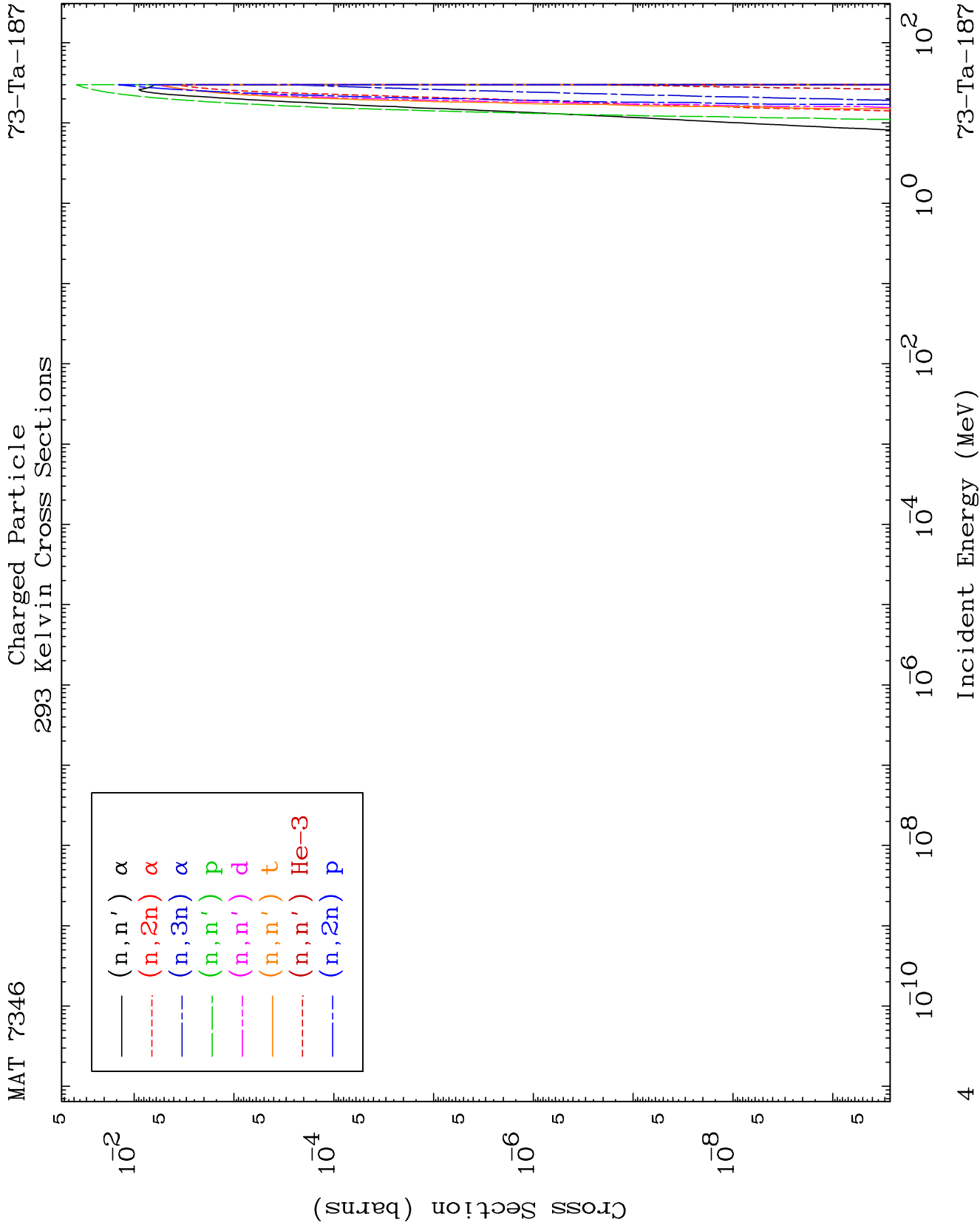
MAT 7346

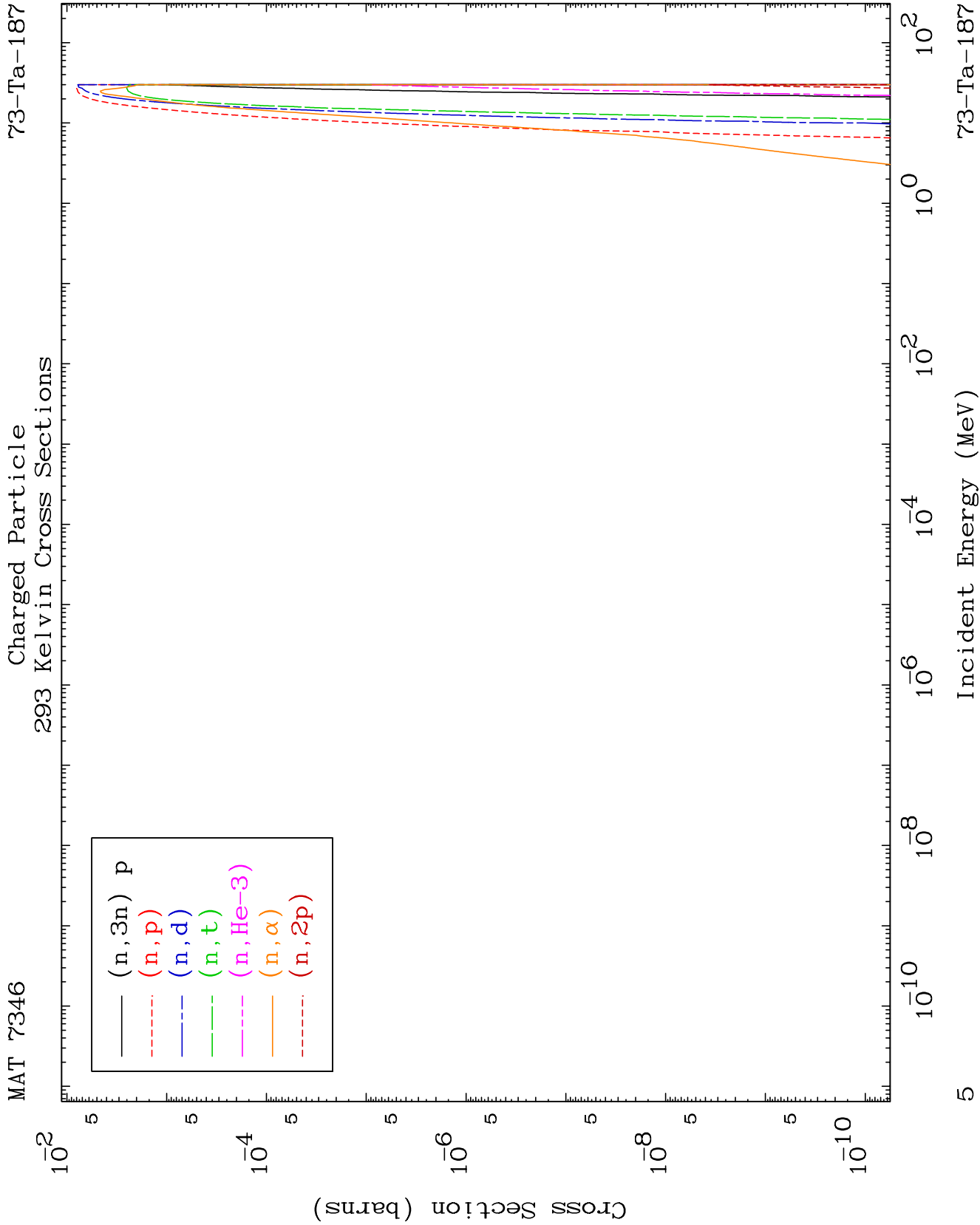
Neutron Production
293 Kelvin Cross Sections

73-Ta-187





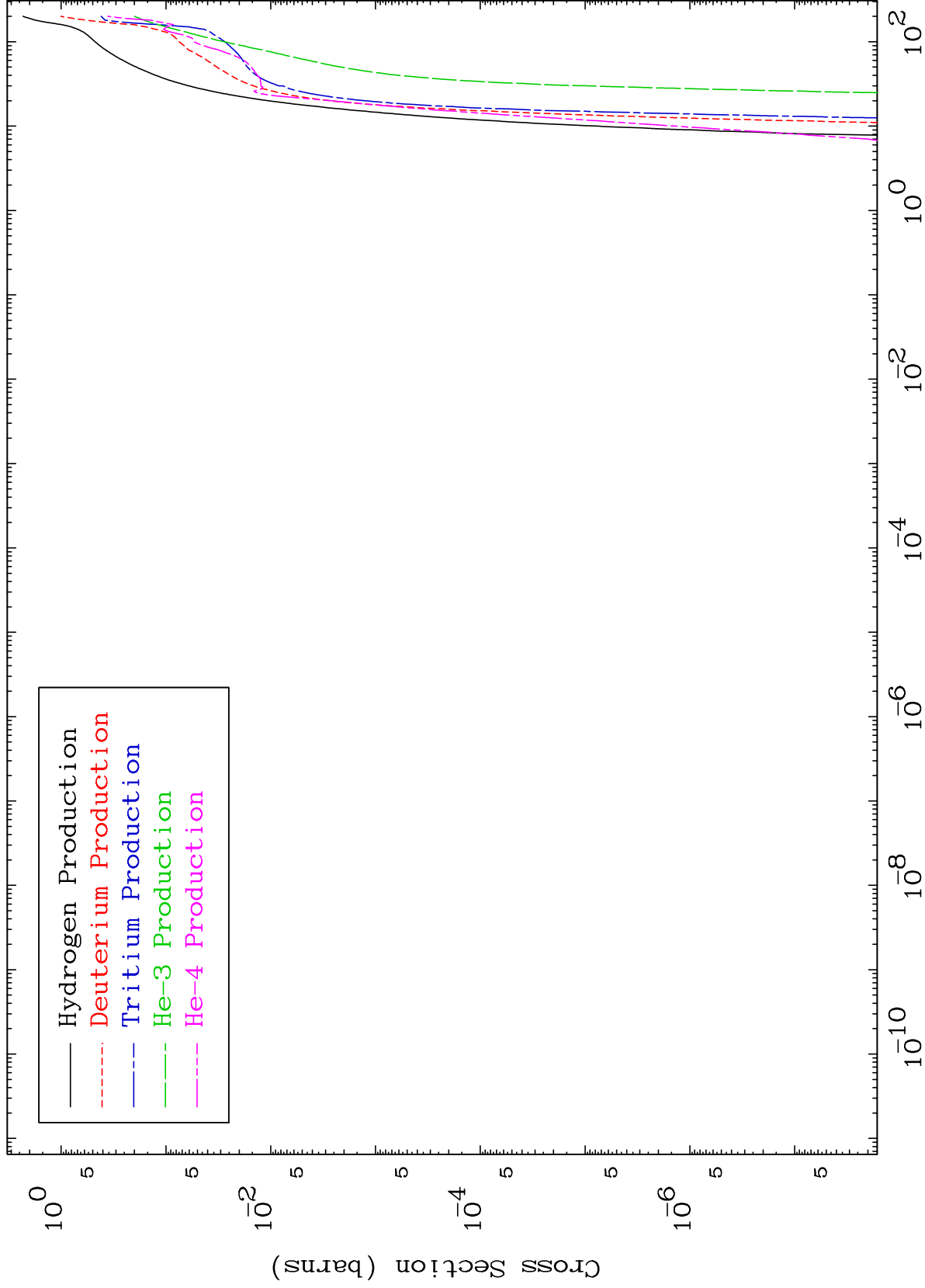


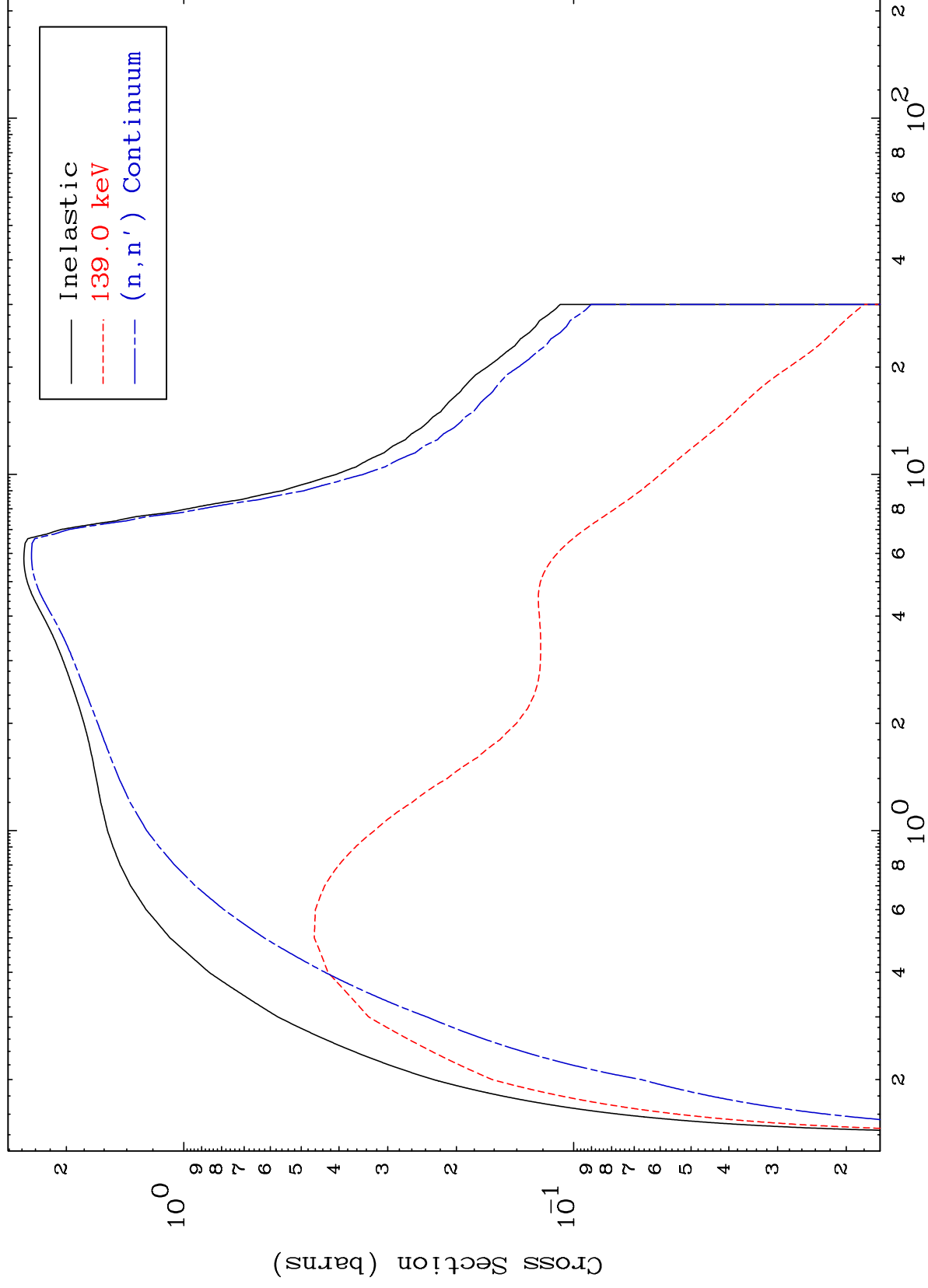


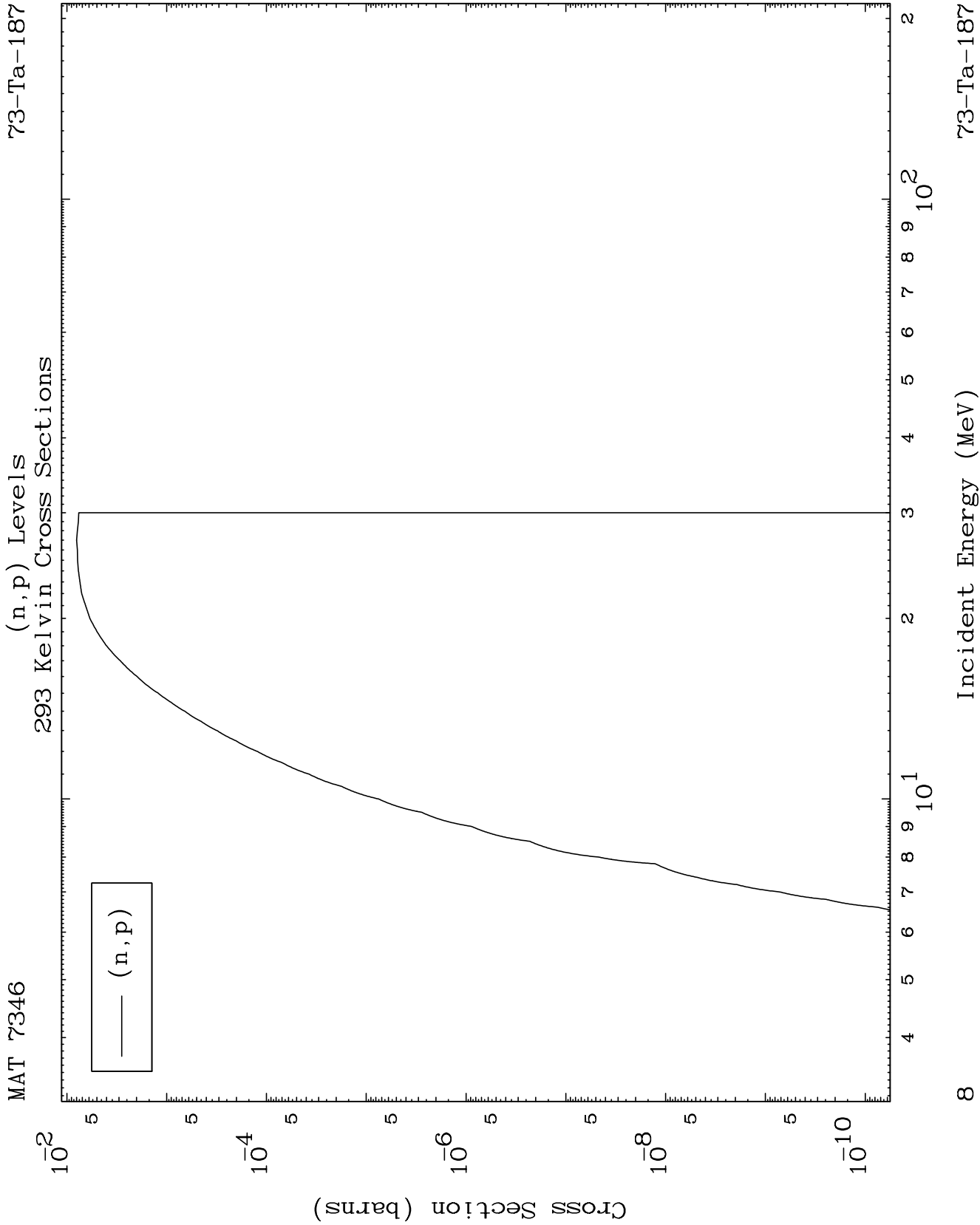
MAT 7346

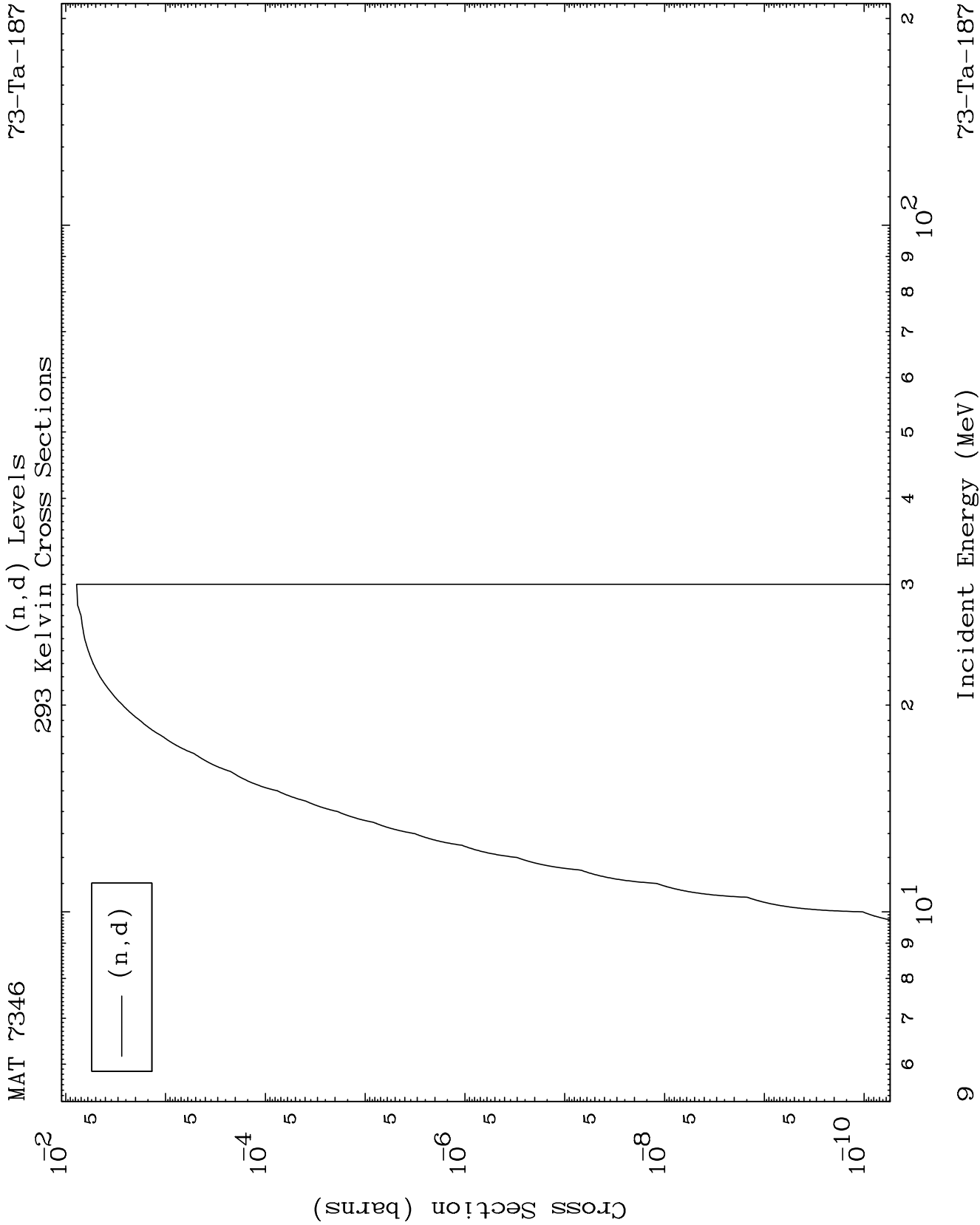
Particle Production
293 Kelvin Cross Sections

73-Ta-187





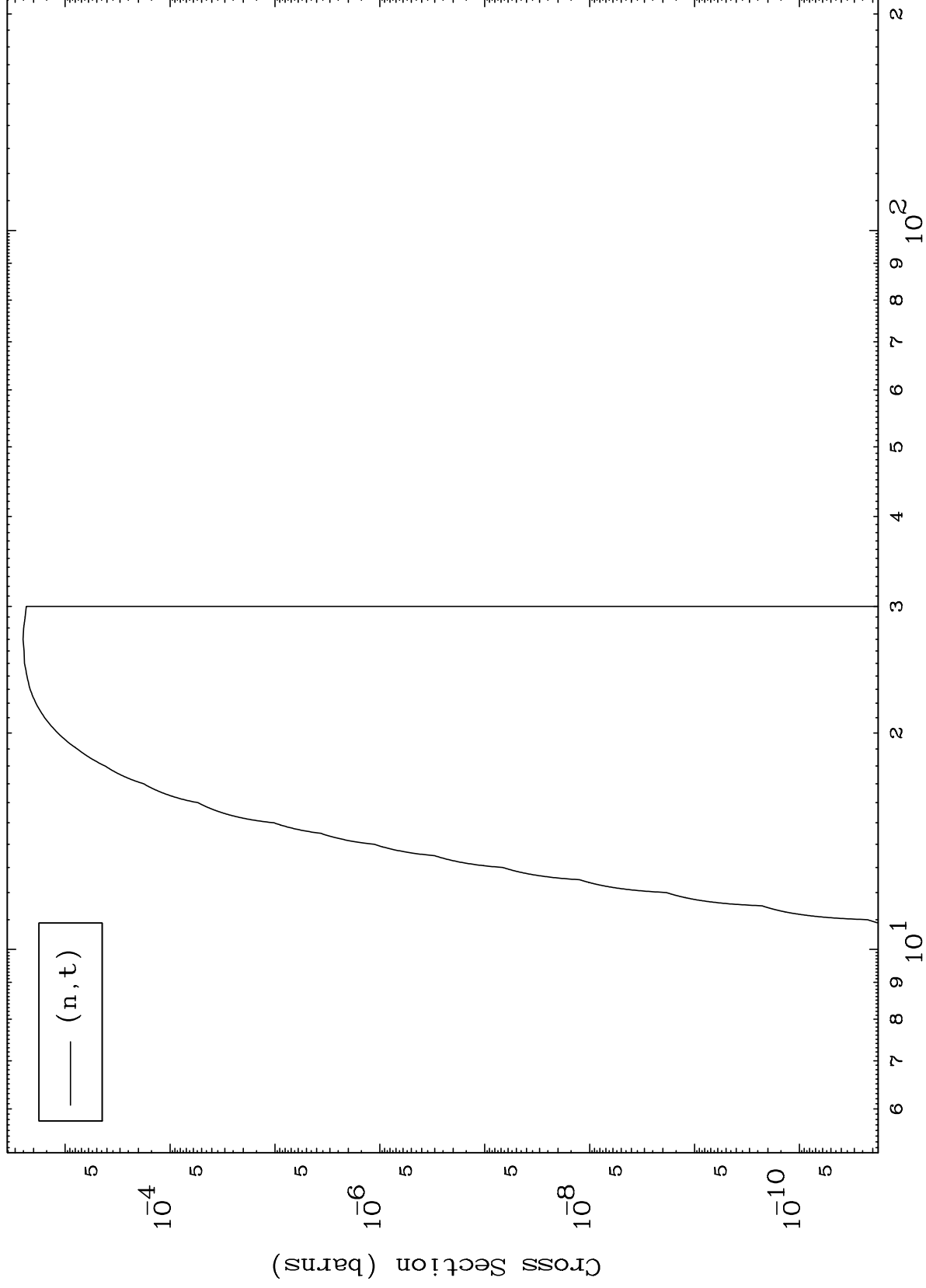




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(n,t) Levels
293 Kelvin Cross Sections



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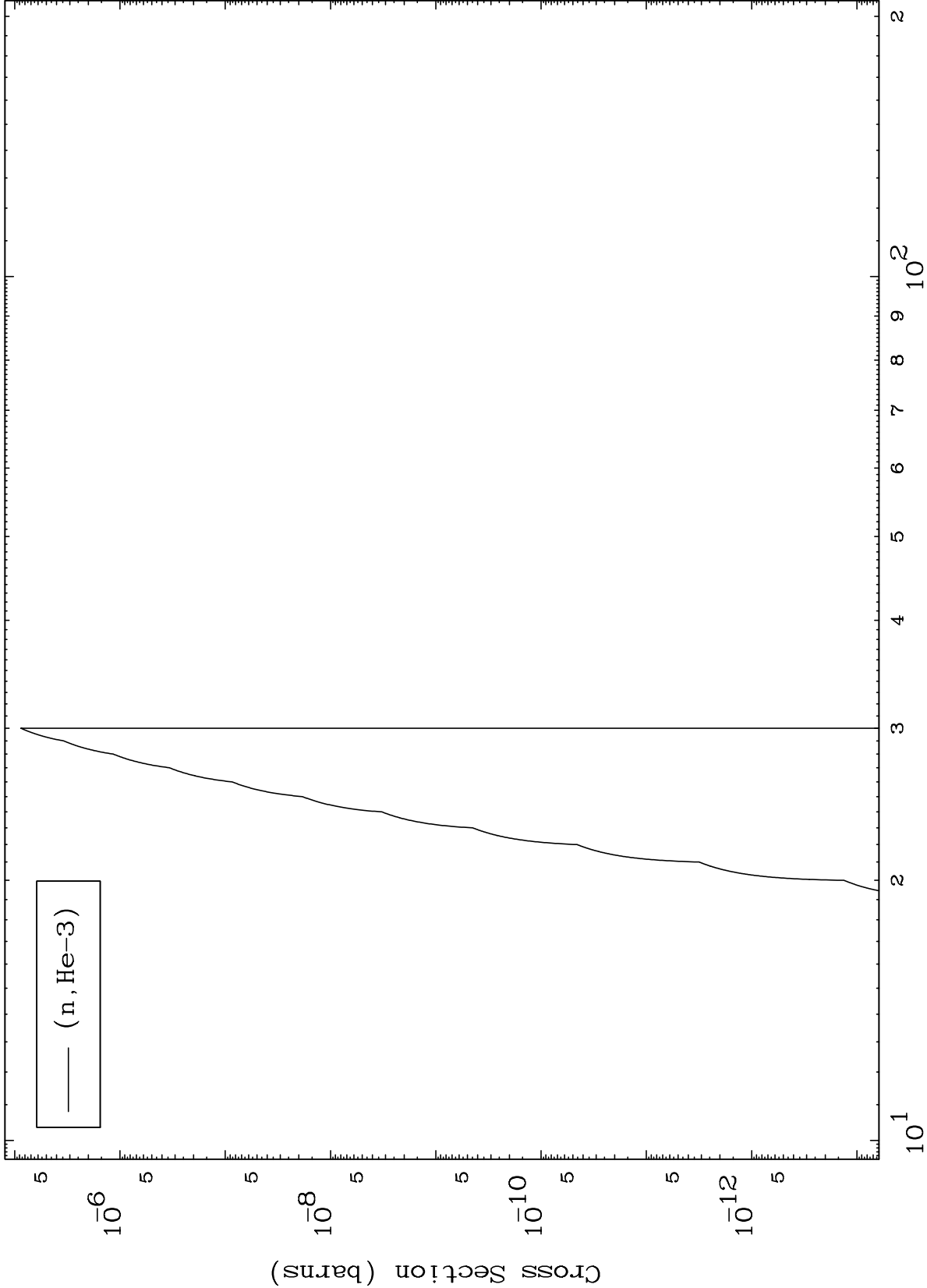
Incident Energy (MeV)

10

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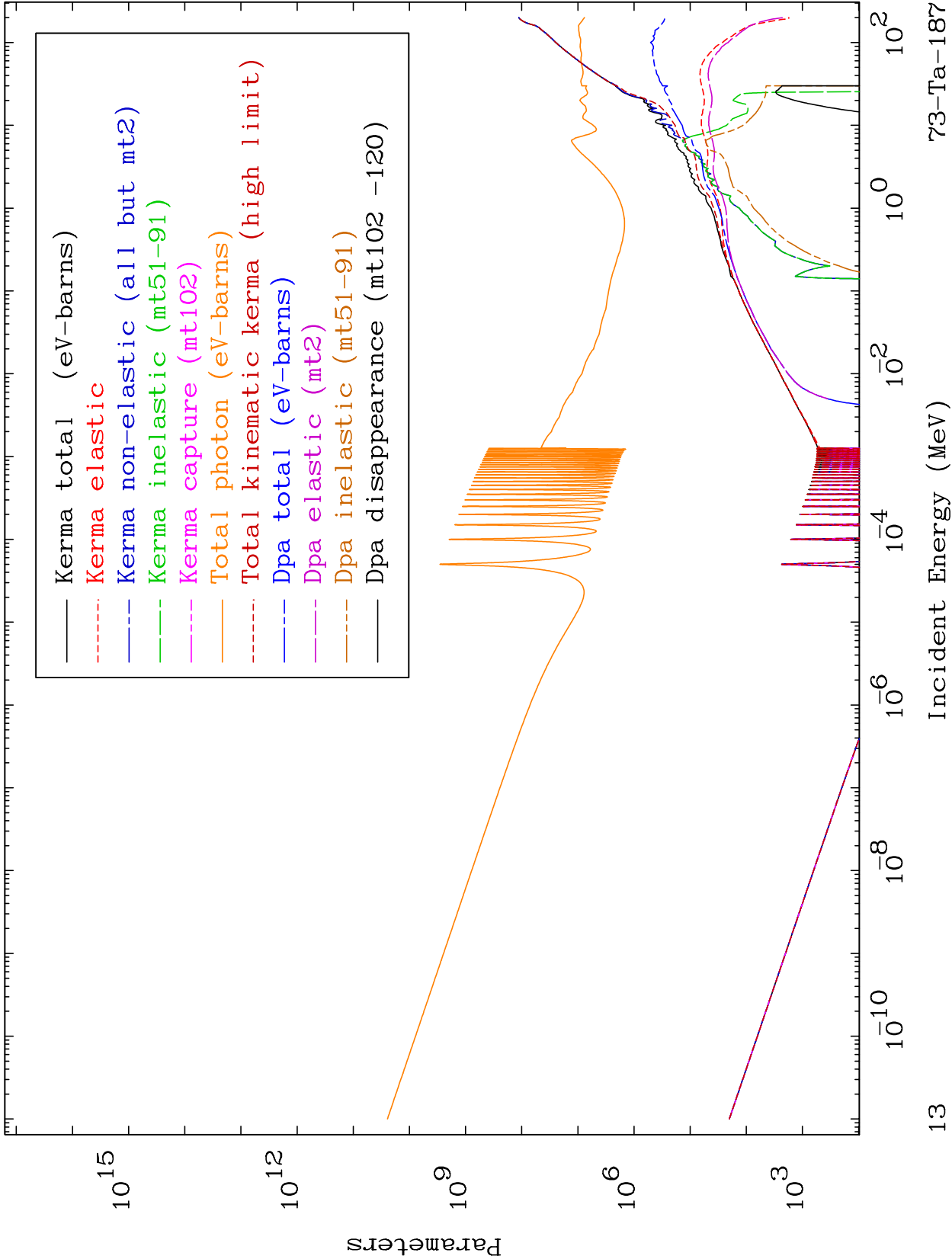
(n,He3) Levels
293 Kelvin Cross Sections

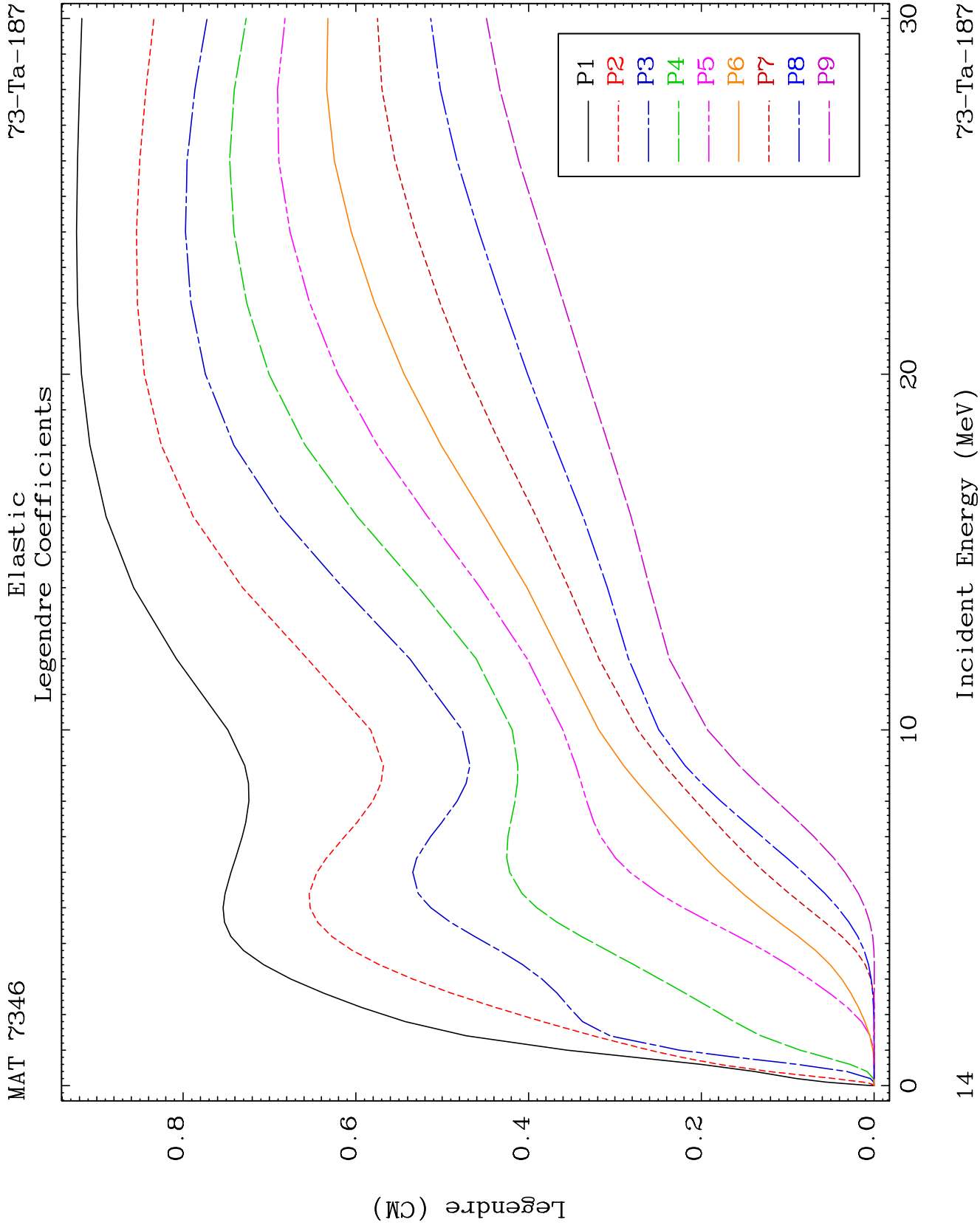
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Incident Energy (MeV)

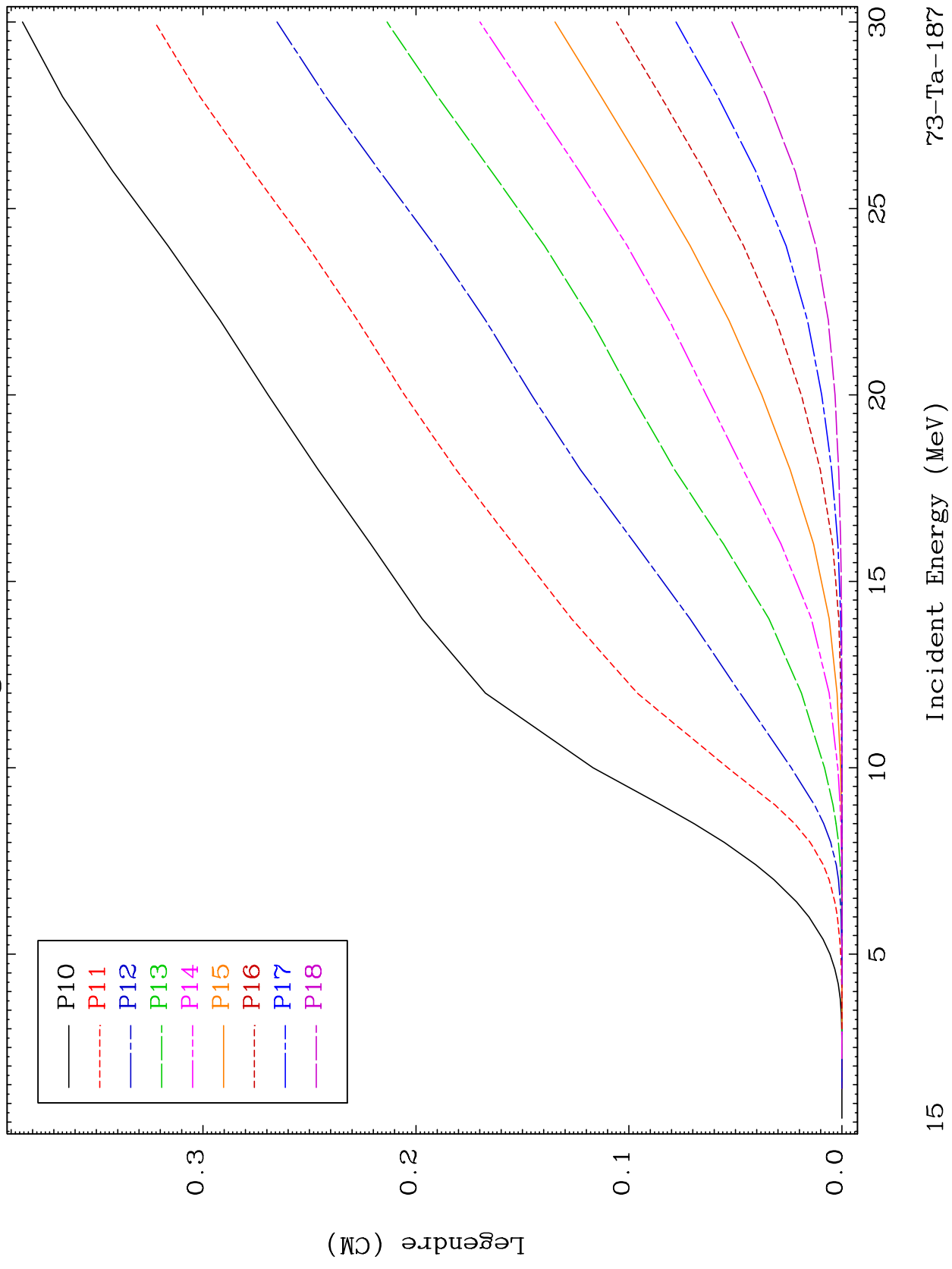
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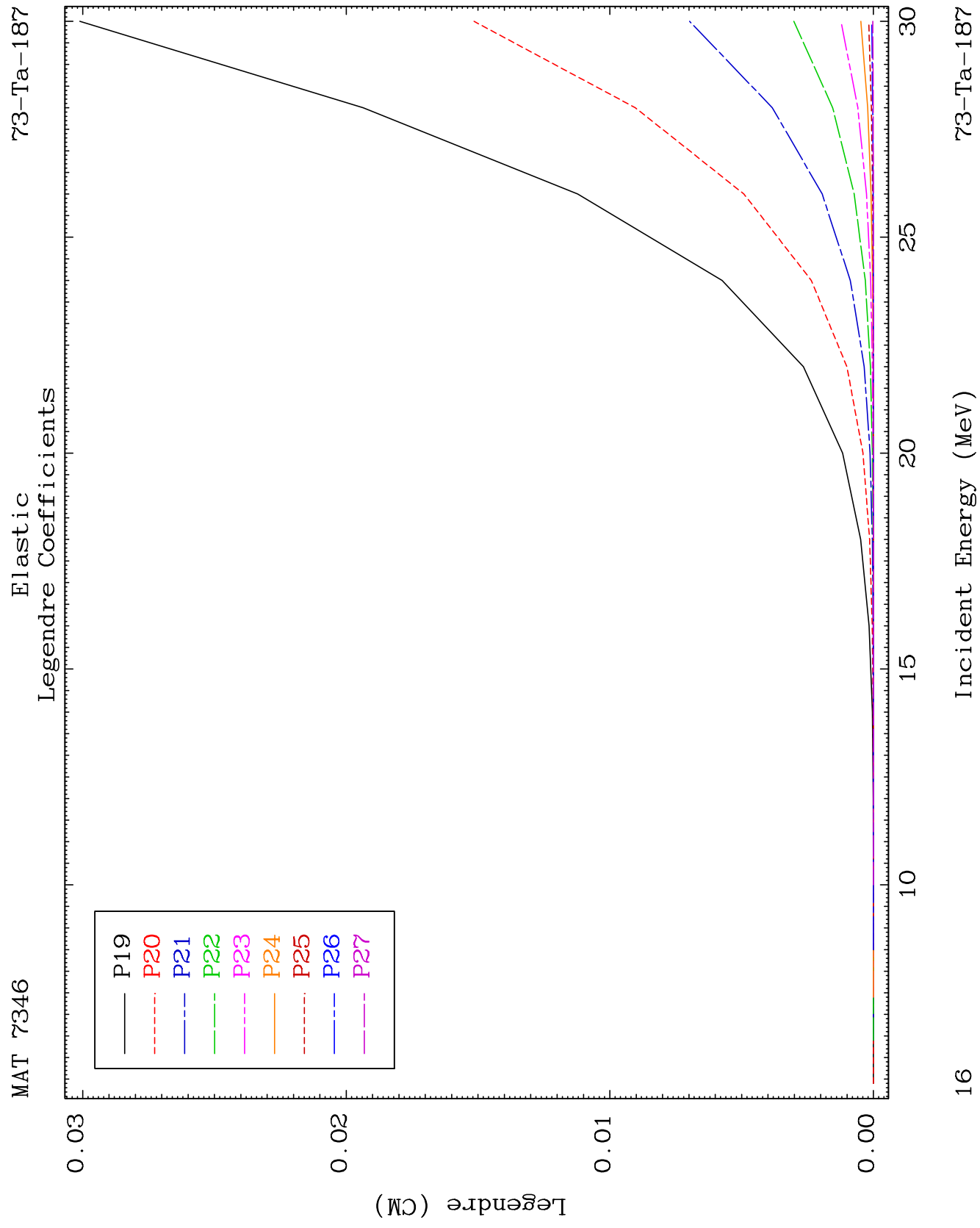


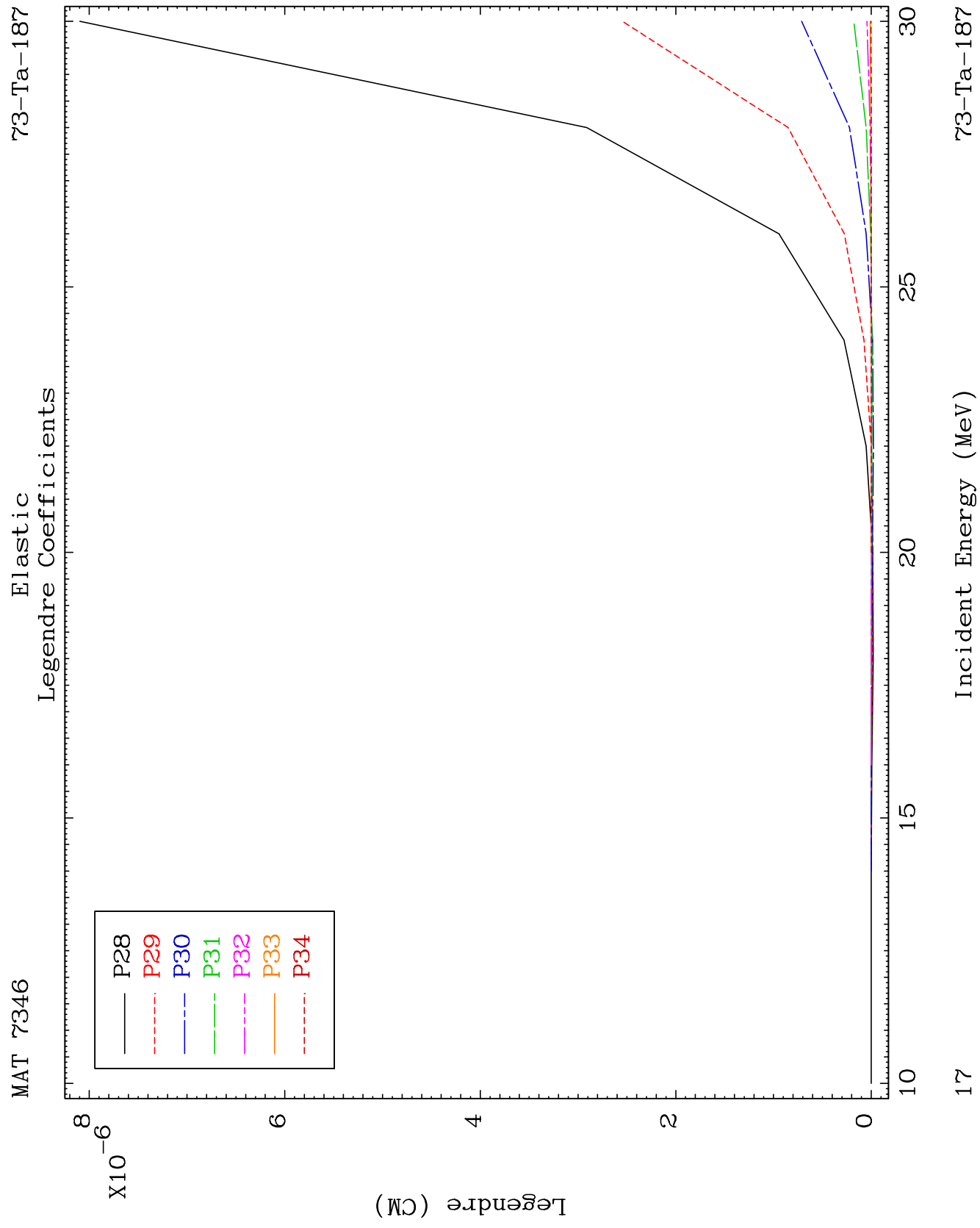


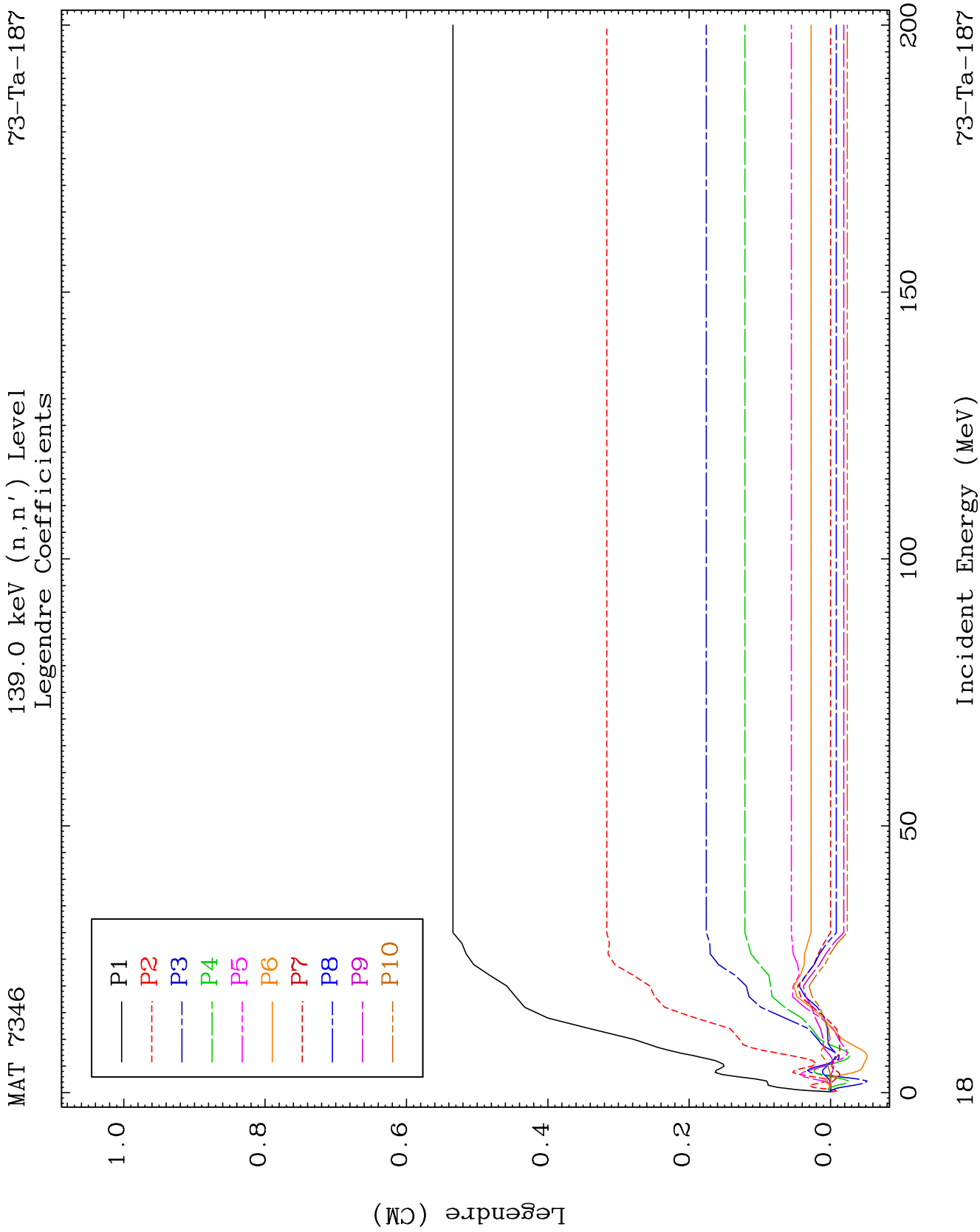
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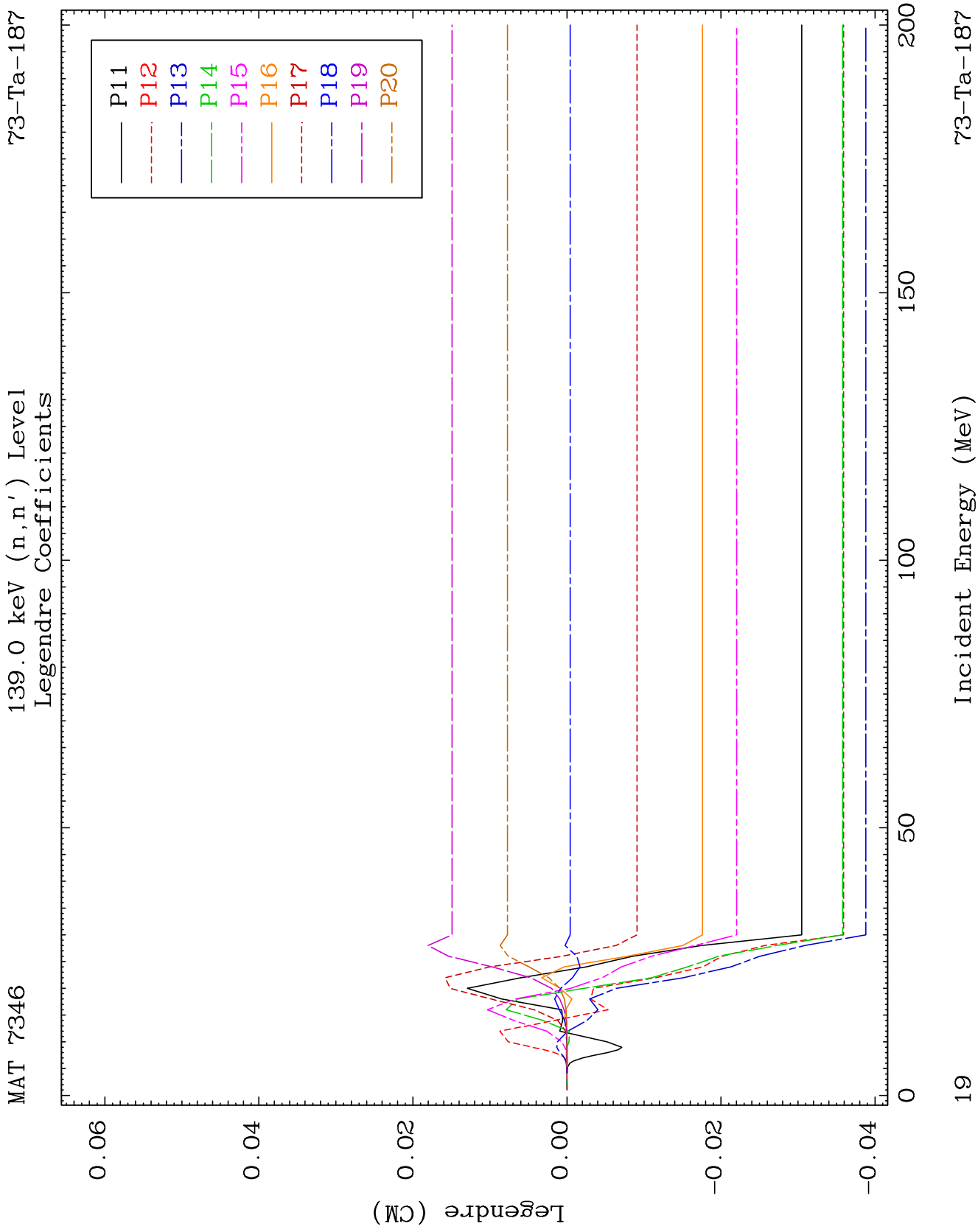
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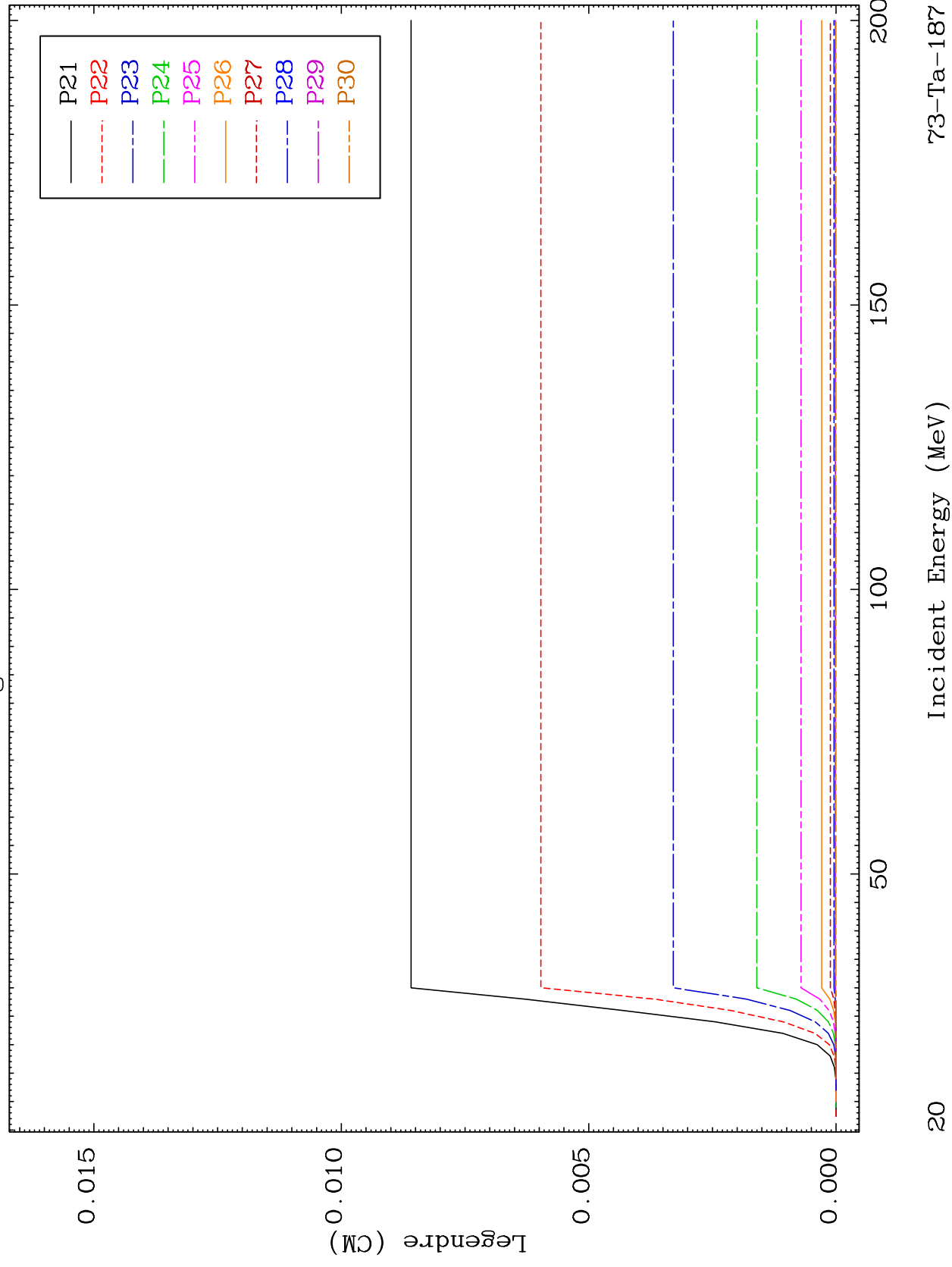


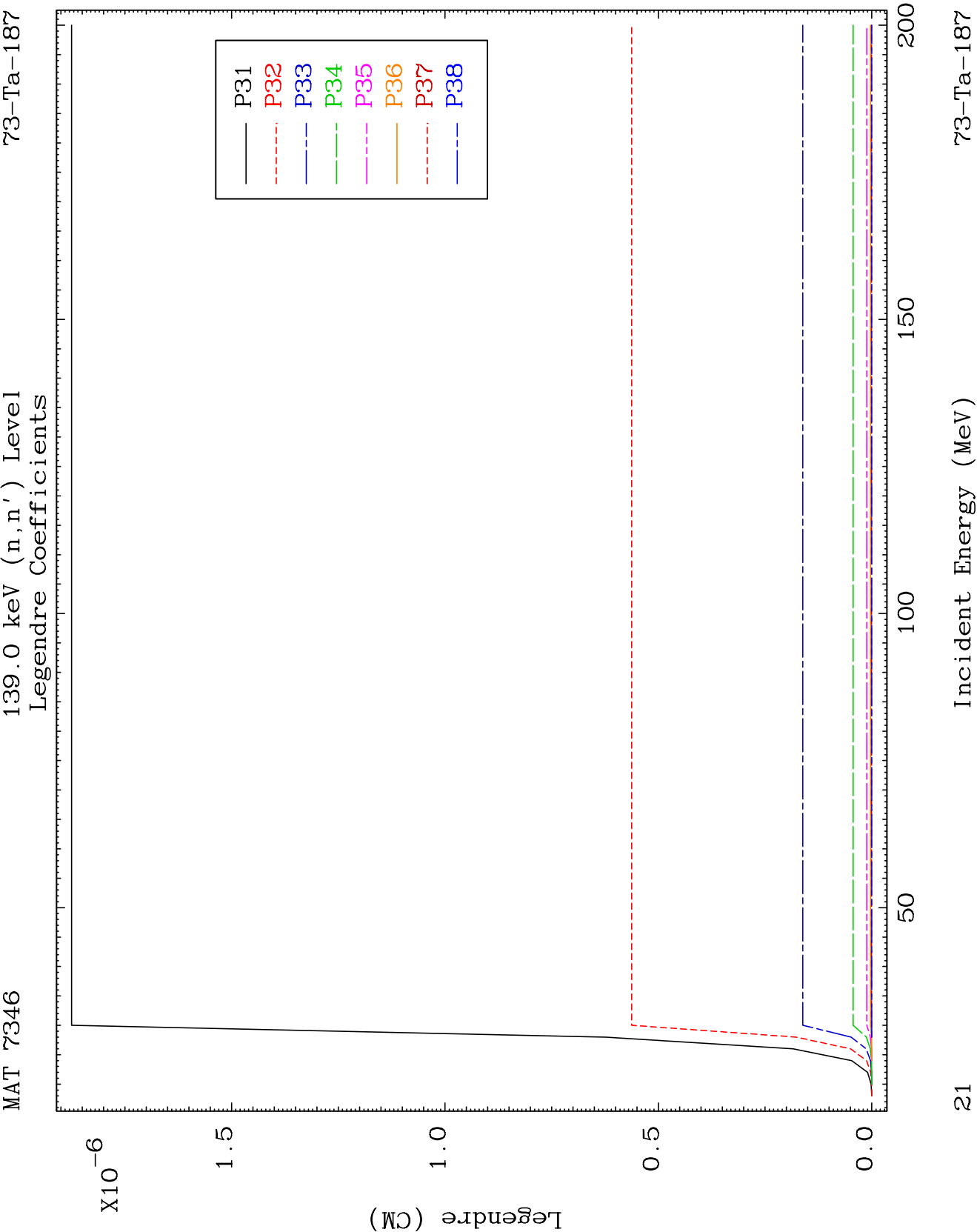


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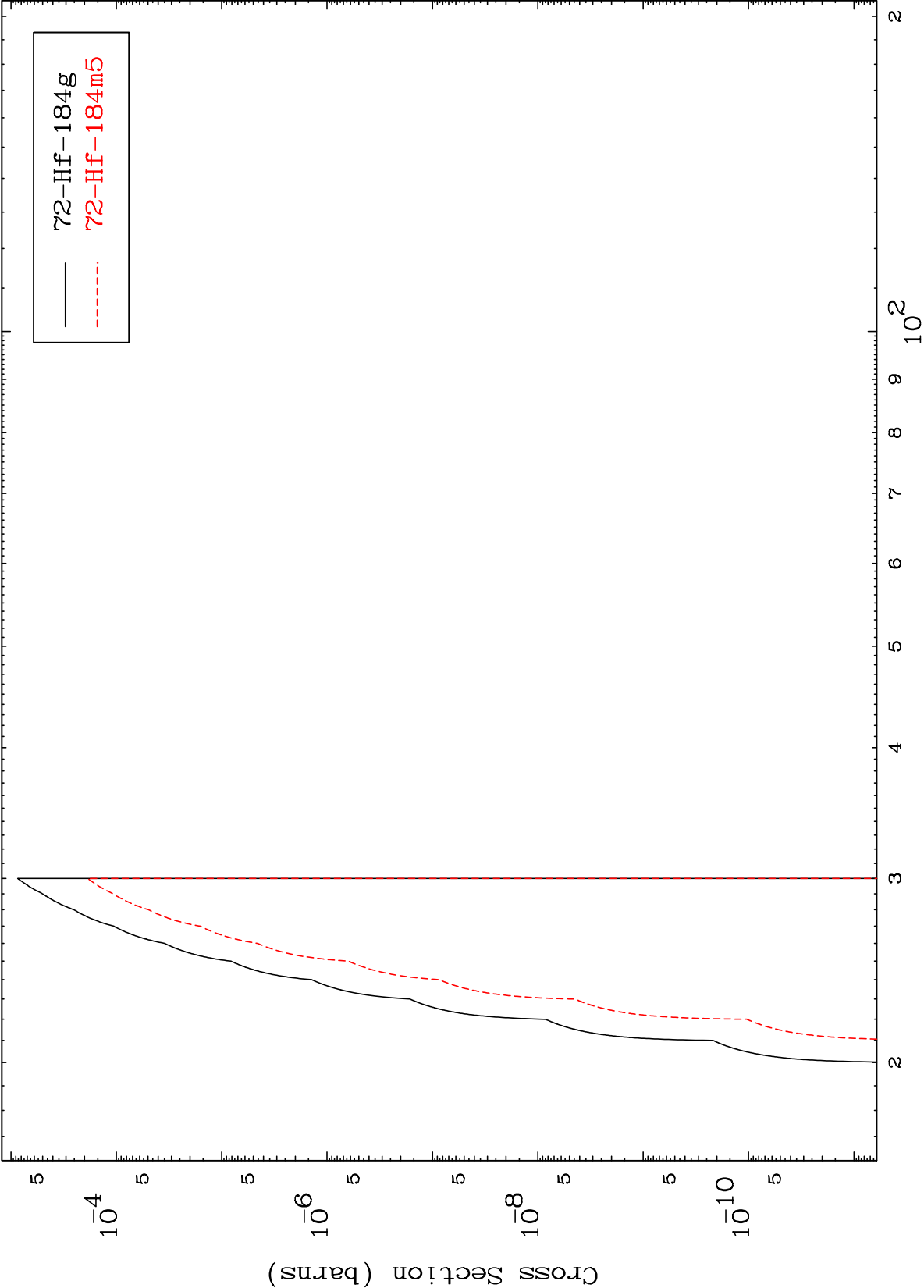
139.0 keV (n, n') Level Legendre Coefficients

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Radionuclide Production Cross Section



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Fission

⁷³Ta-187

Radionuclide Production Cross Section

