

Program EVALPLOT
(Version 2015-2)

by

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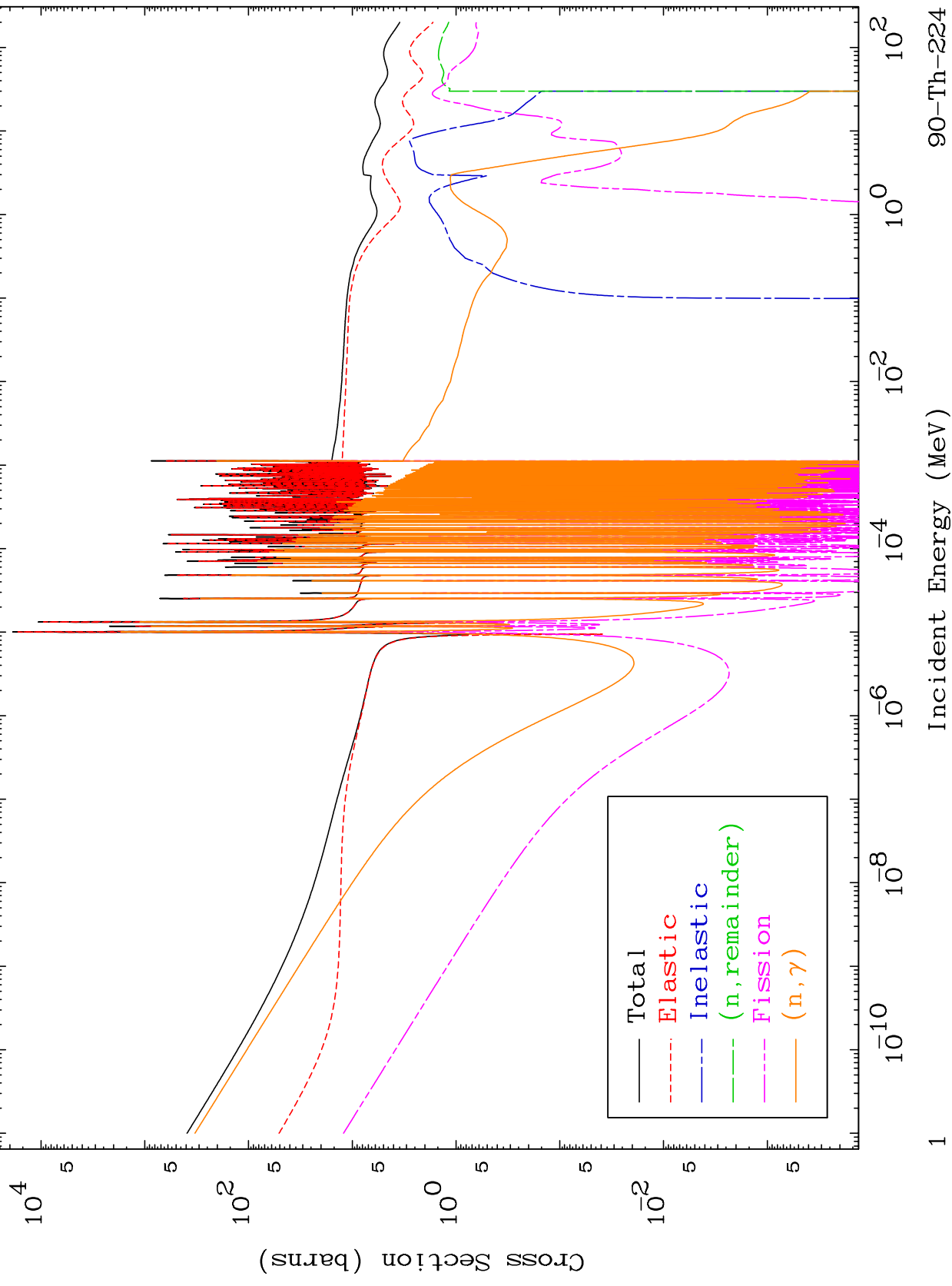
Press Mouse Button to Start

MAT 9016

Major

293 Kelvin Cross Sections

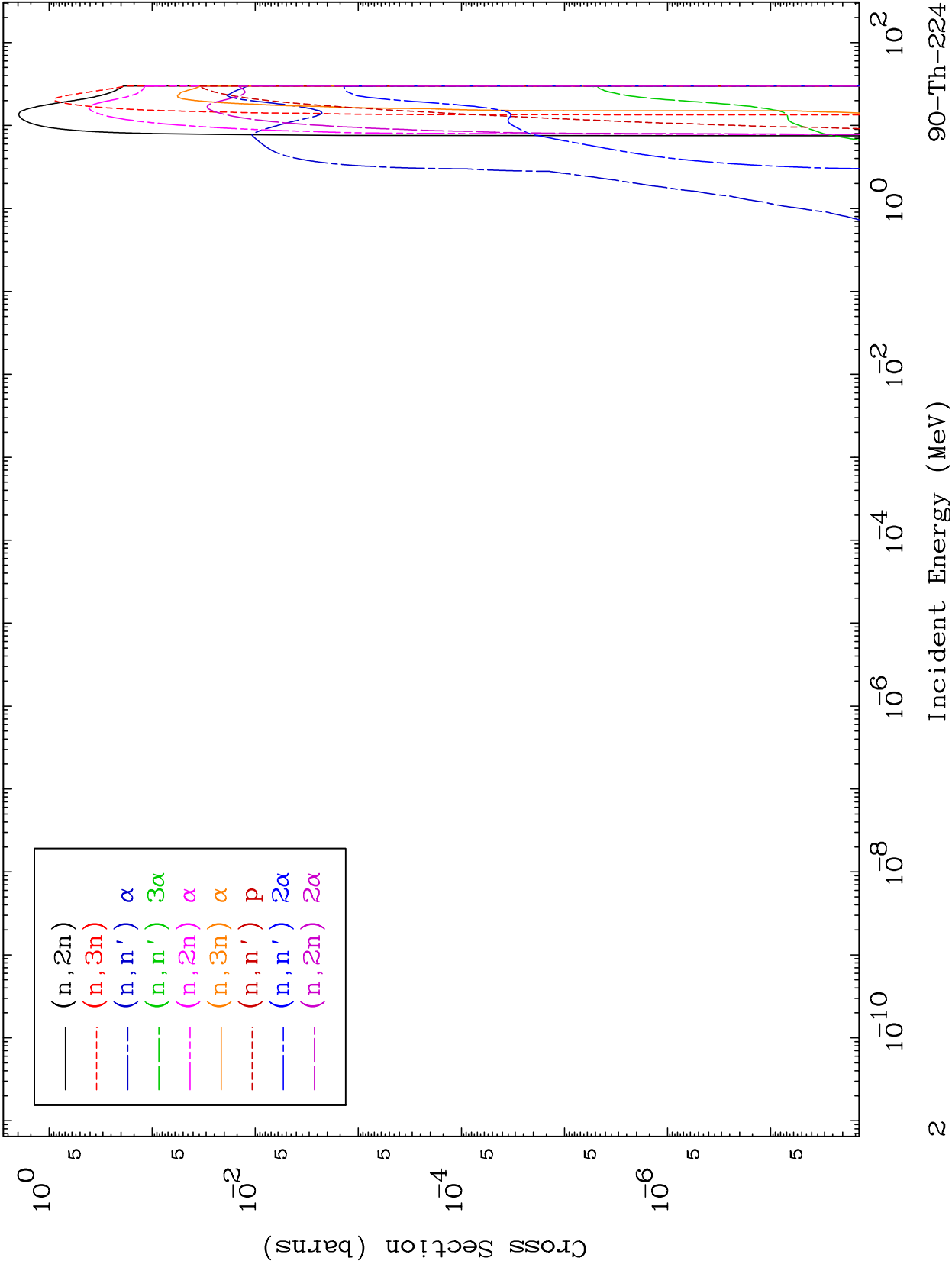
90-Th-224



MAT 9016

Neutron Production
293 Kelvin Cross Sections

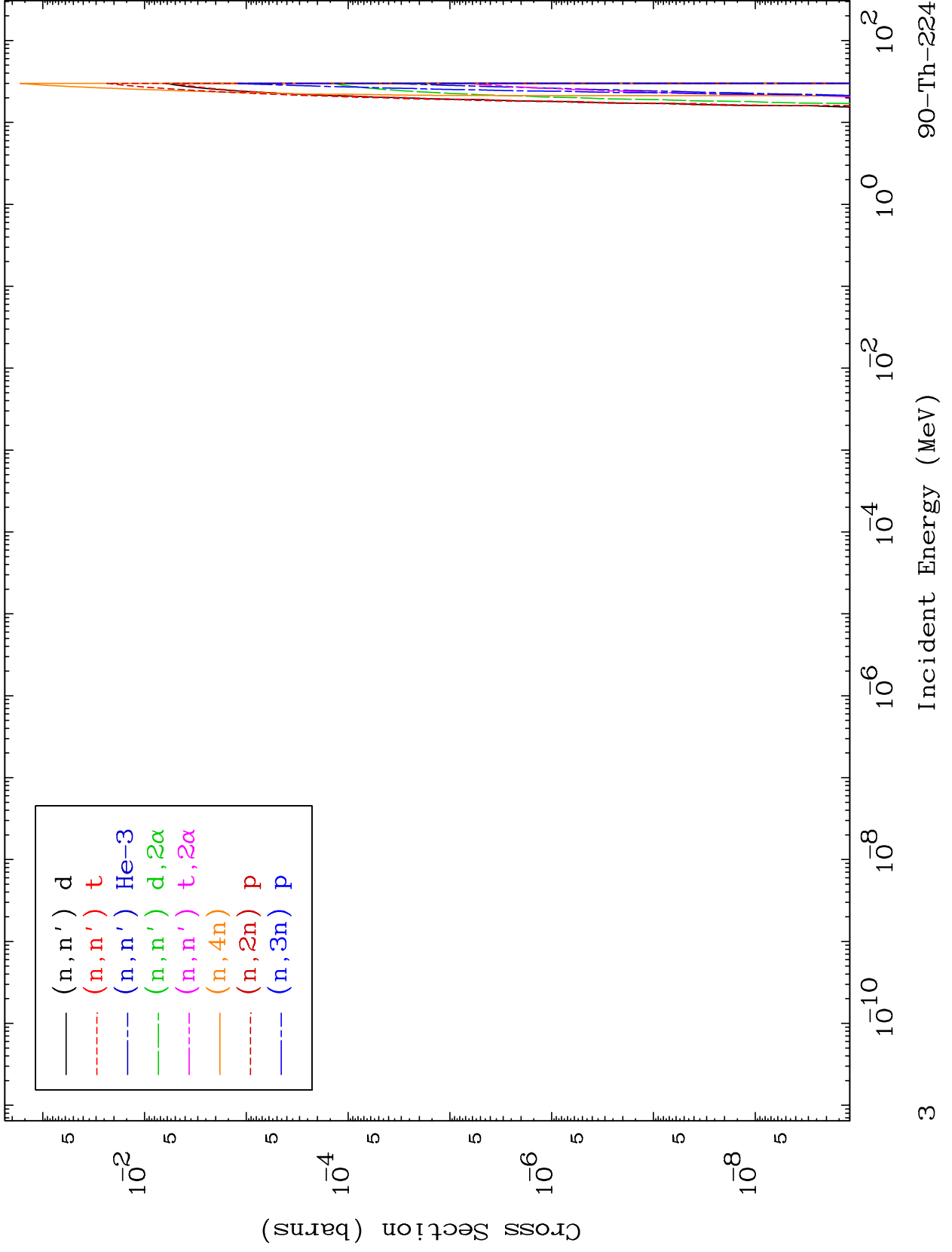
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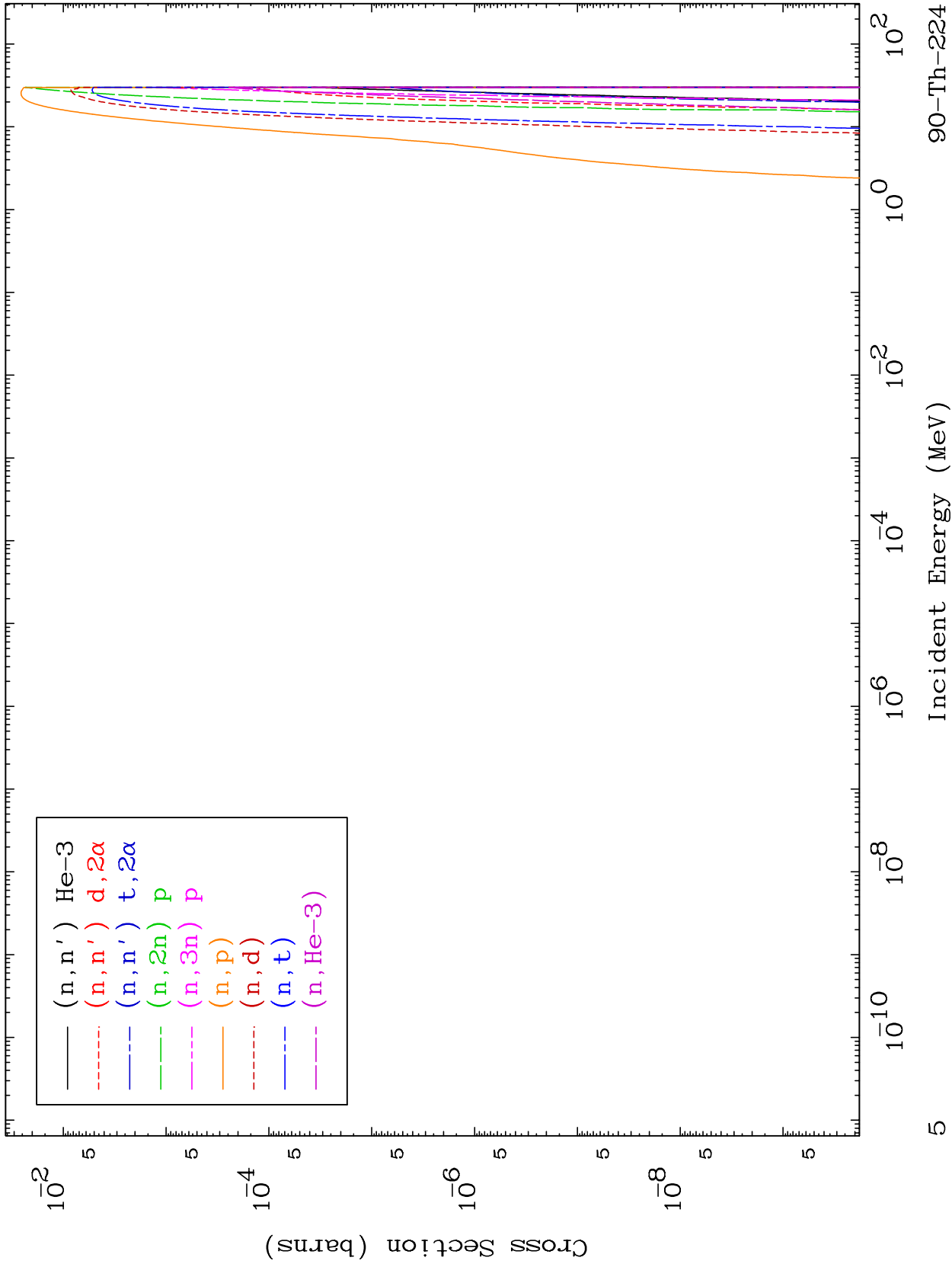


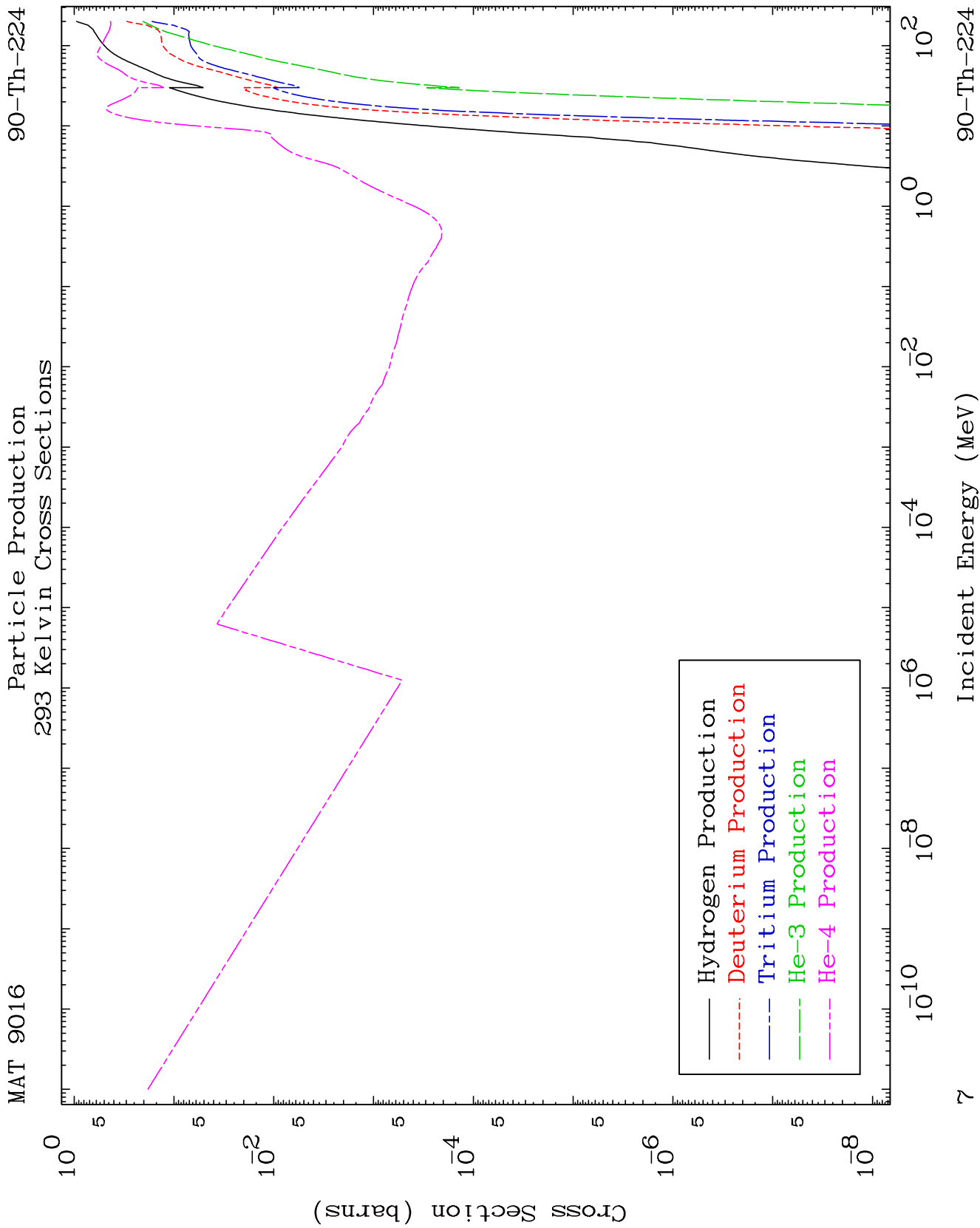
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Neutron Production
293 Kelvin Cross Sections

90-Th-224



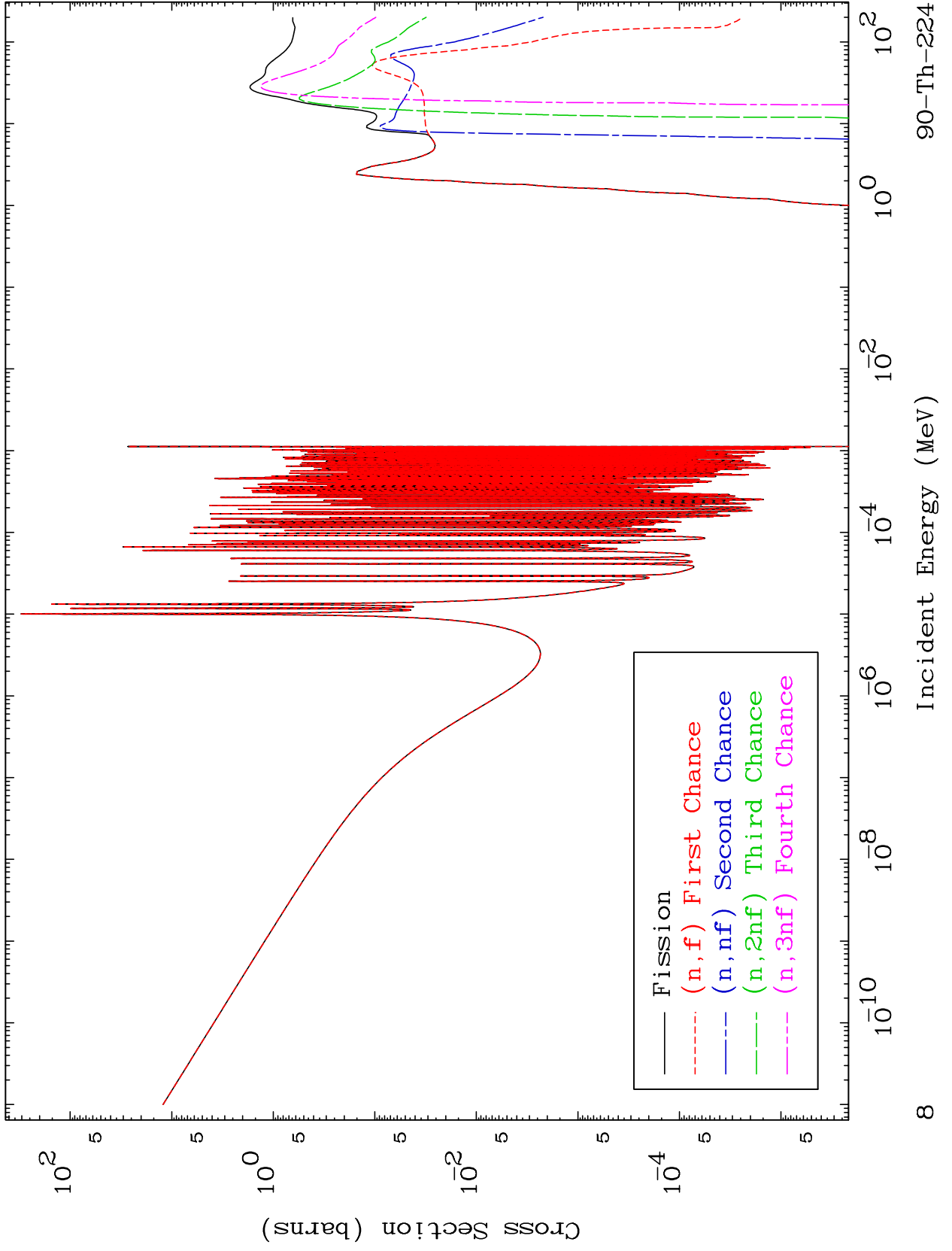




MAT 9016

Fission
293 Kelvin Cross Sections

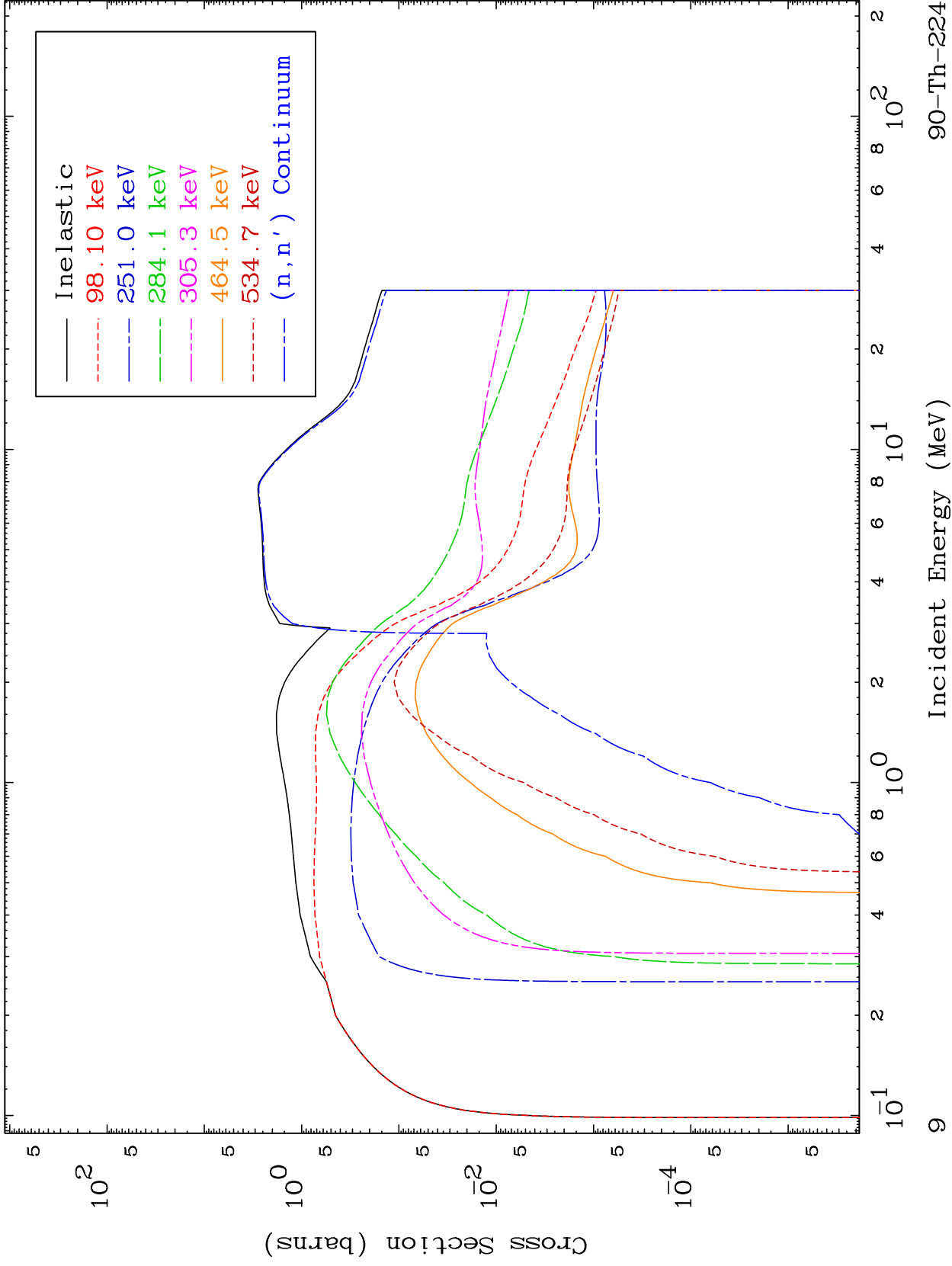
90-Th-224



MAT 9016

(n,n') Level
293 Kelvin Cross Sections

90-Th-224



90-Th-224

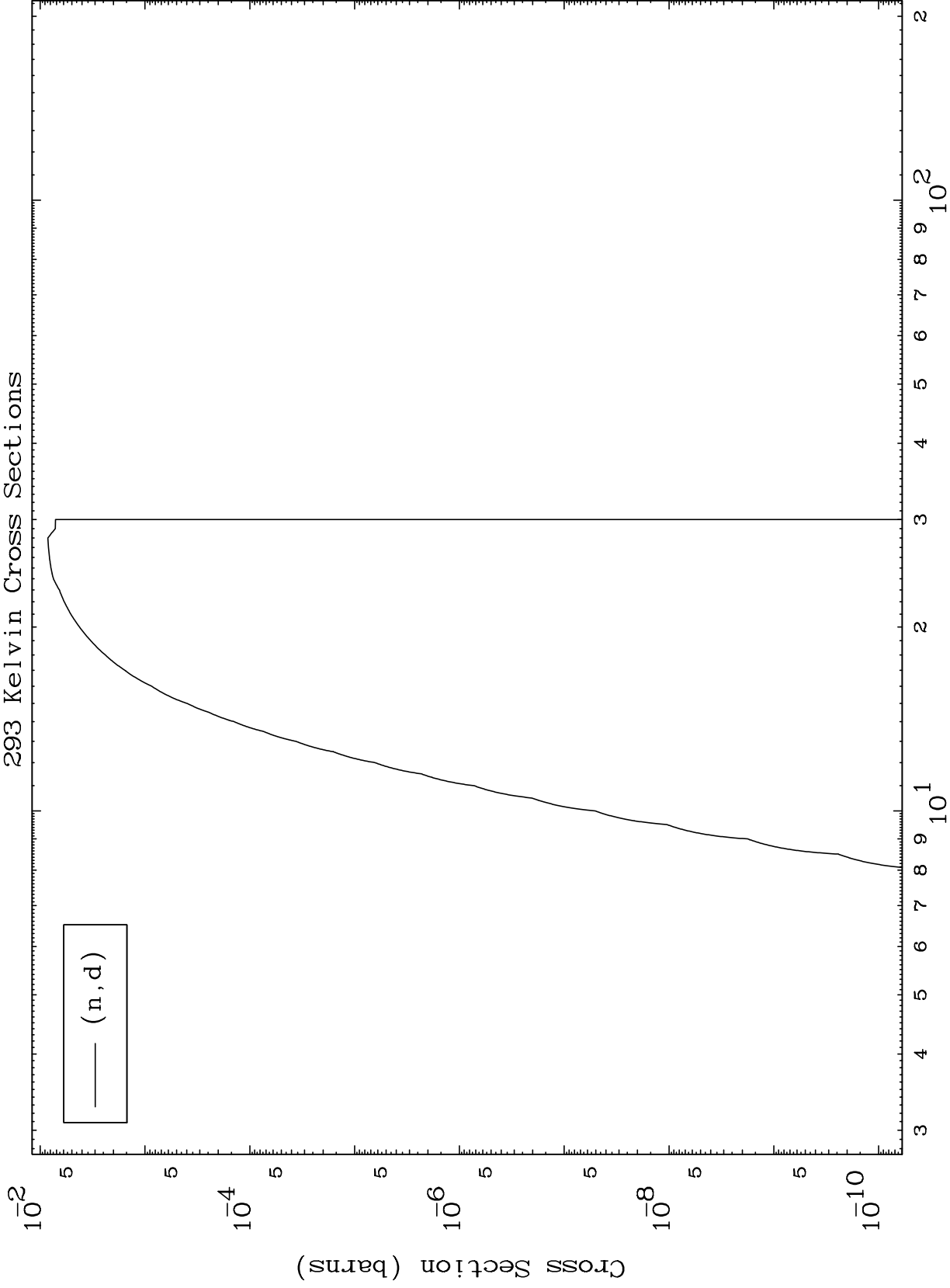
Incident Energy (MeV)

9

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(n,d) Levels
293 Kelvin Cross Sections

90-Th-224



11

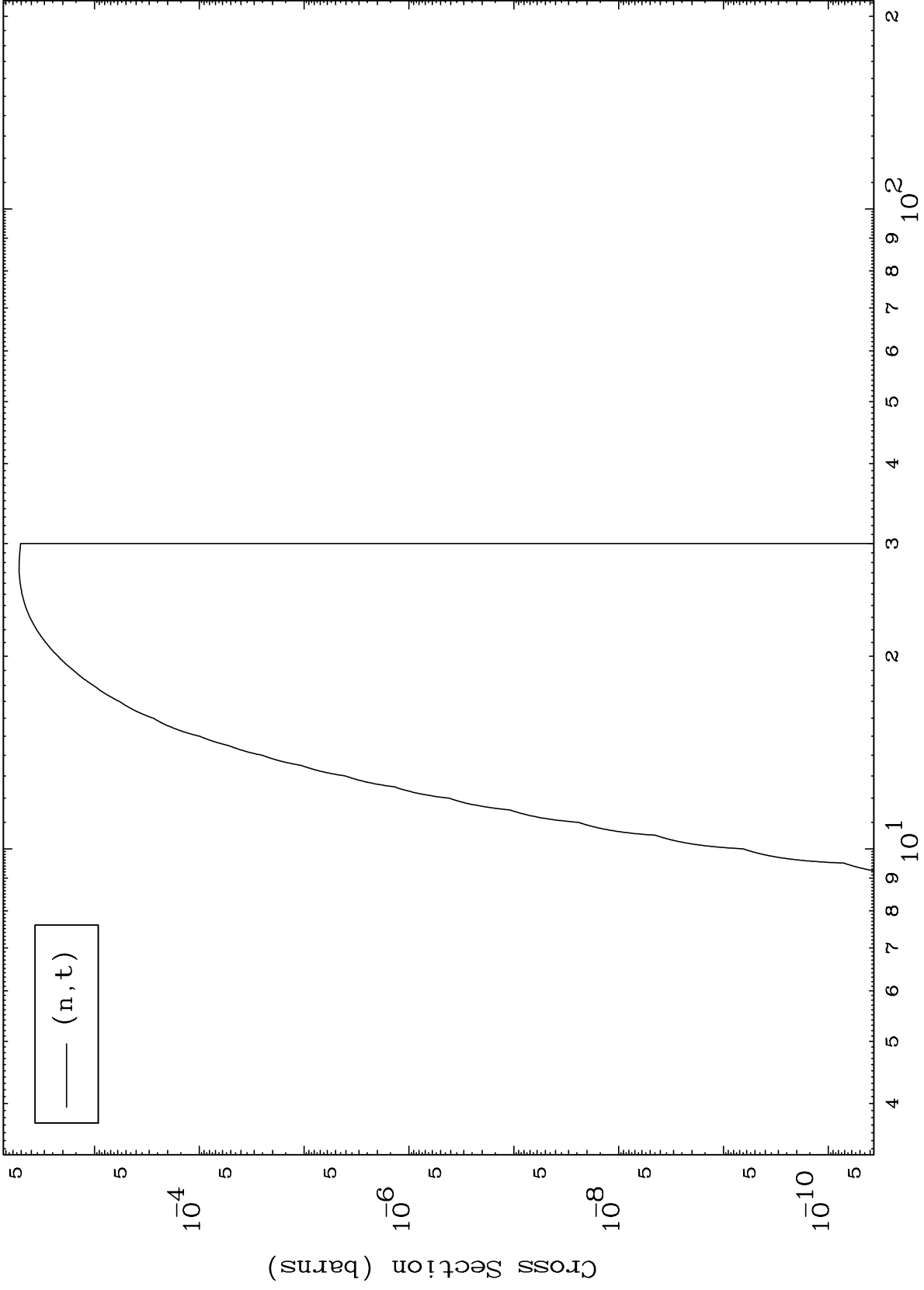
Incident Energy (MeV)

90-Th-224

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(n,t) Levels
293 Kelvin Cross Sections

90-Th-224



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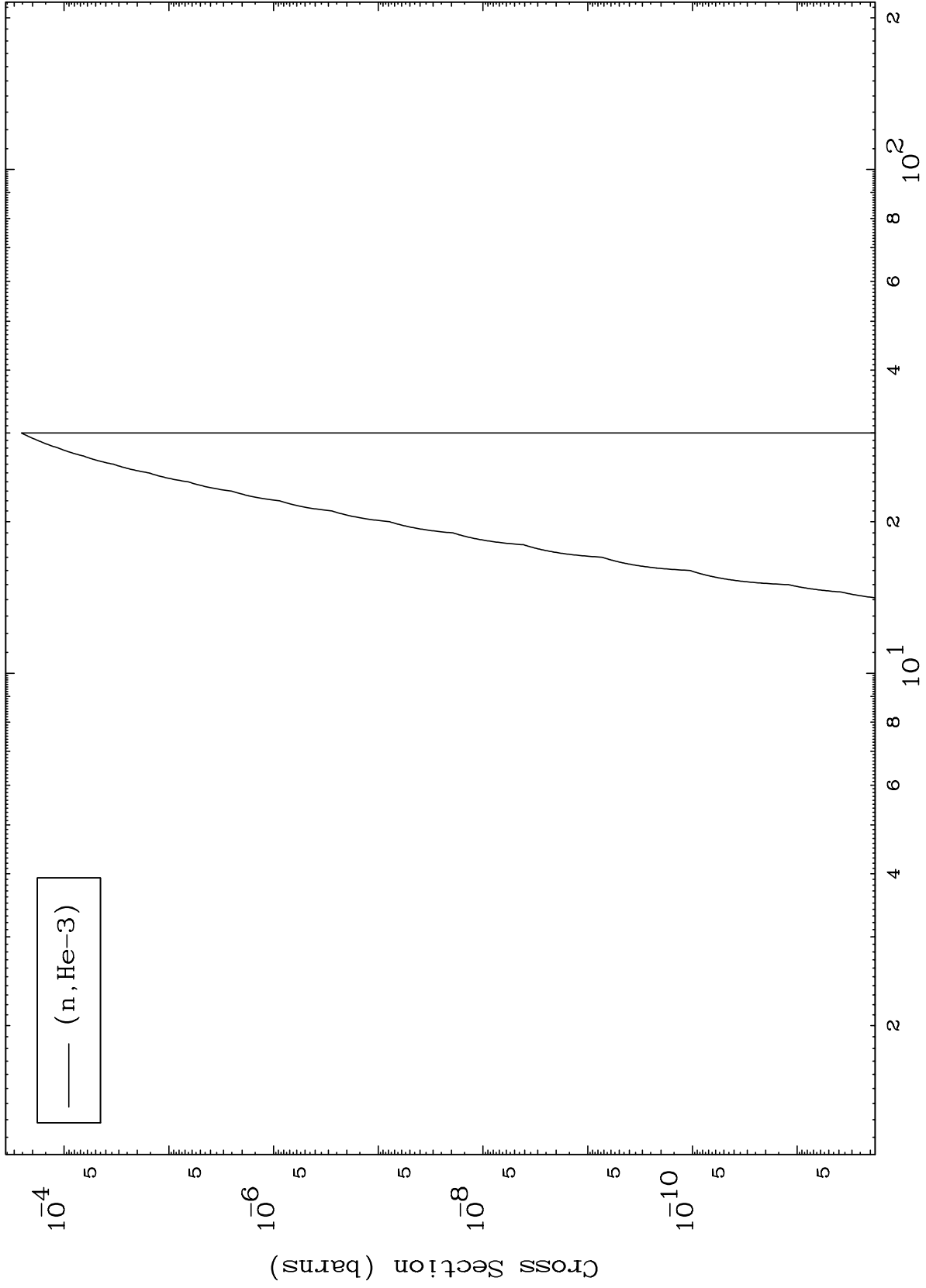
Incident Energy (MeV)

90-Th-224

MAT 9016

(n,He3) Levels
293 Kelvin Cross Sections

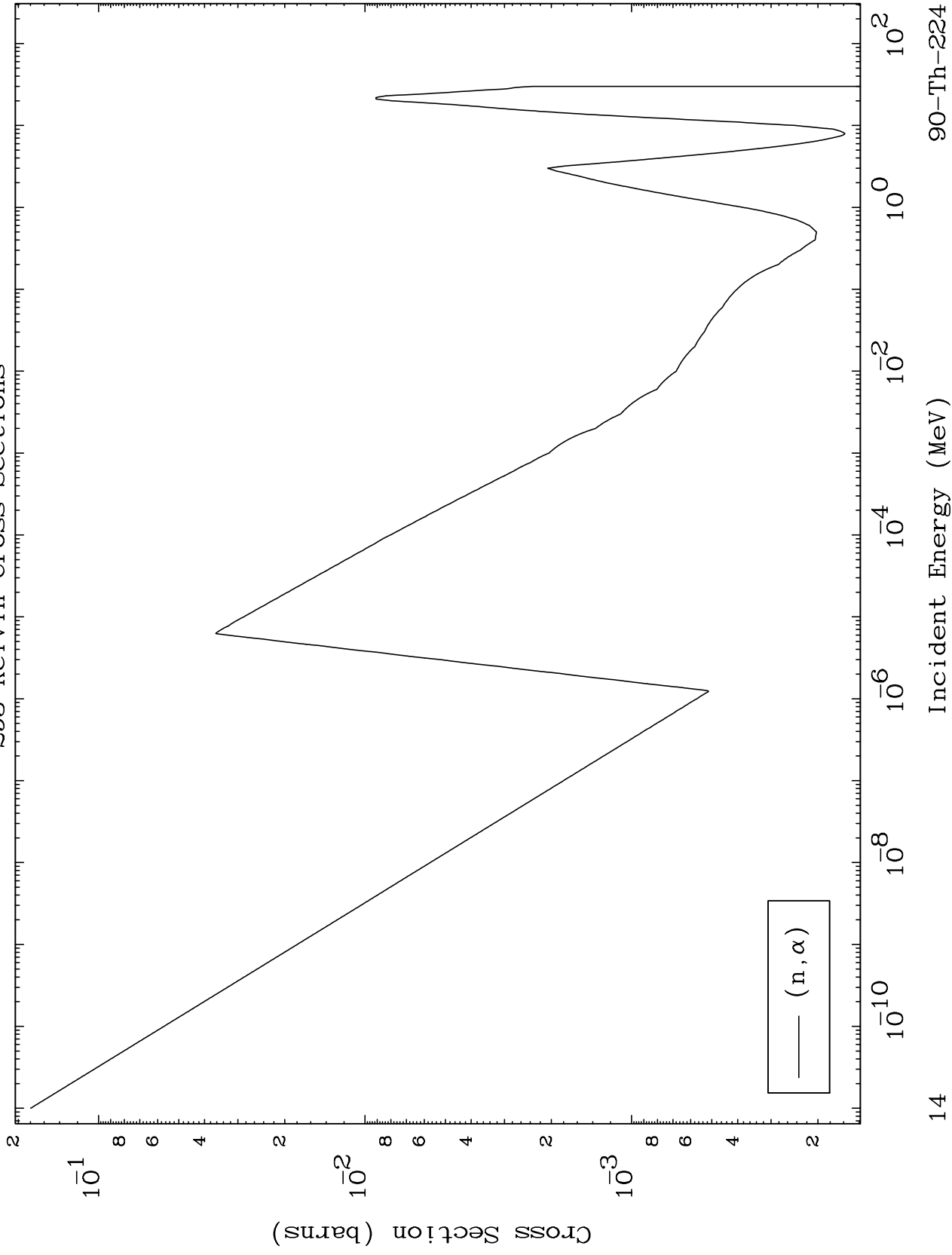
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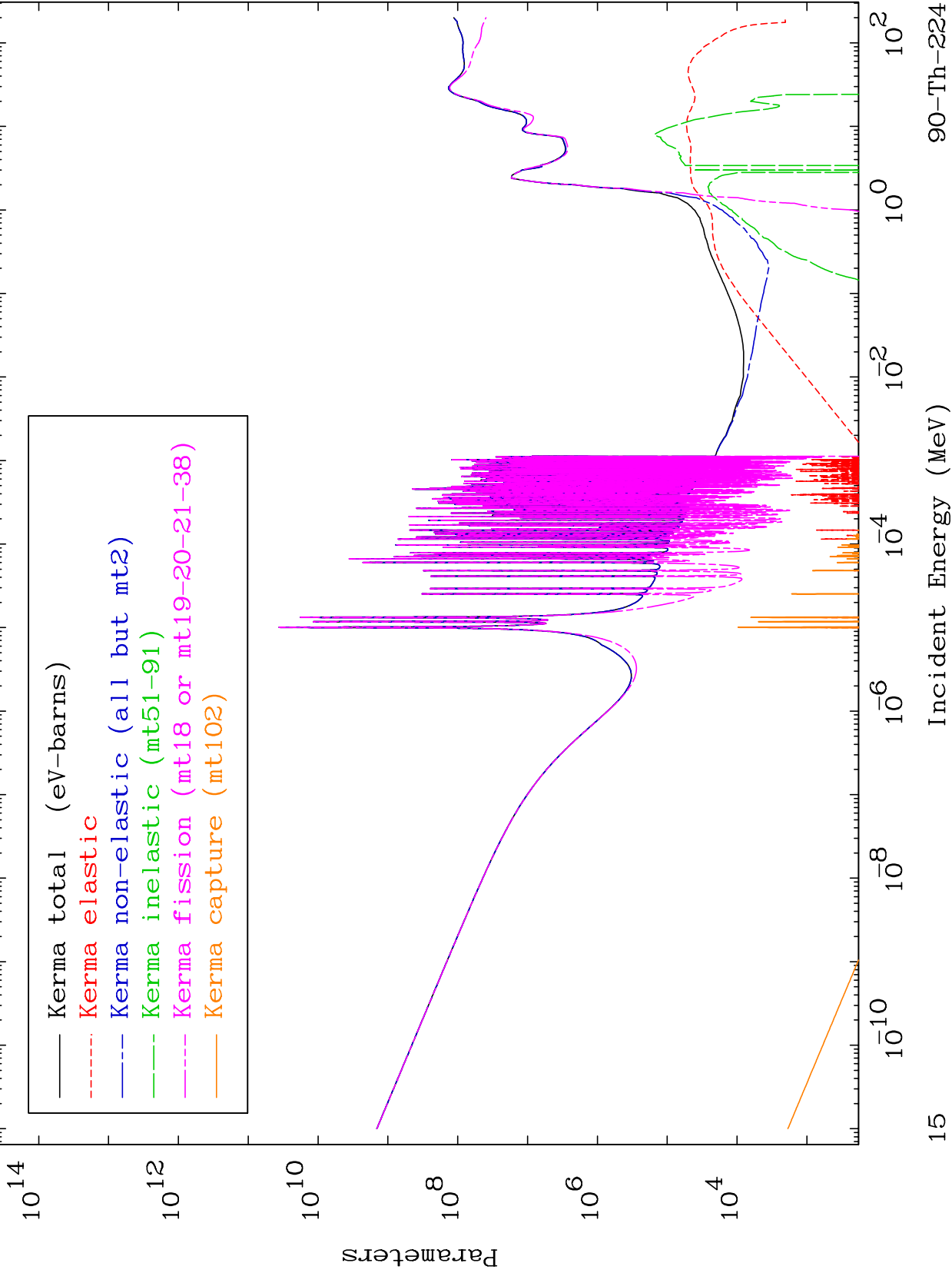


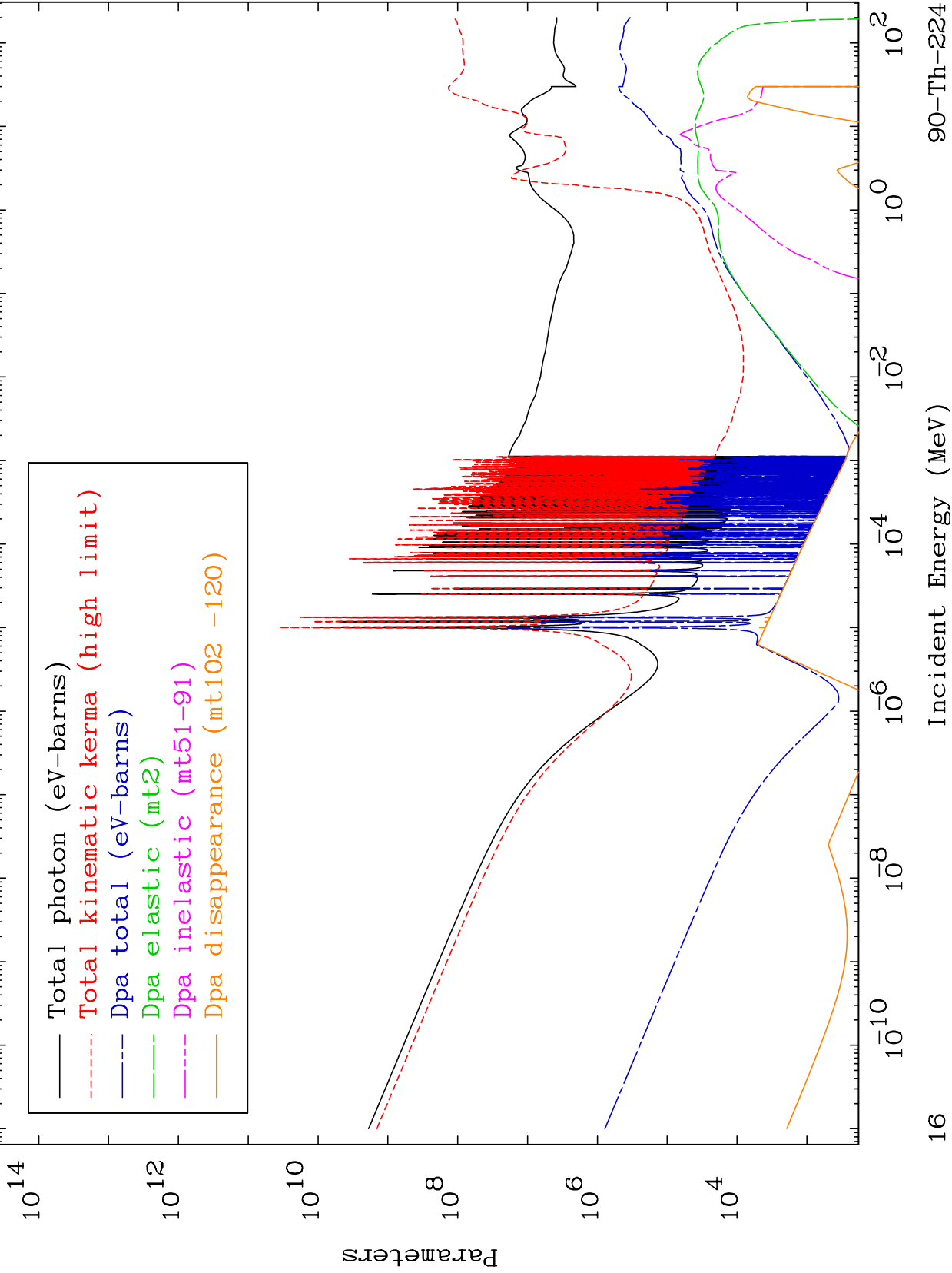
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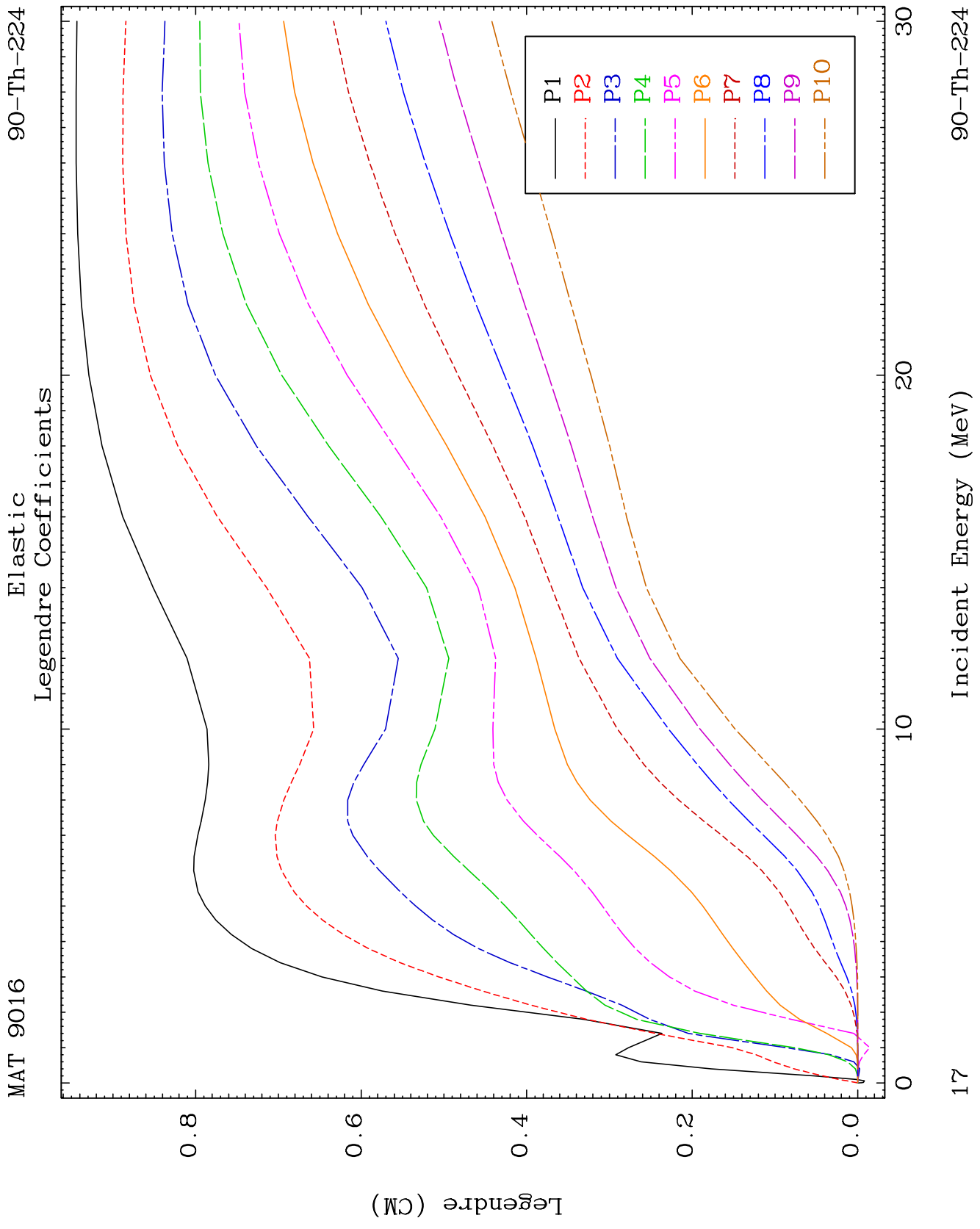
(n,α) Levels
293 Kelvin Cross Sections

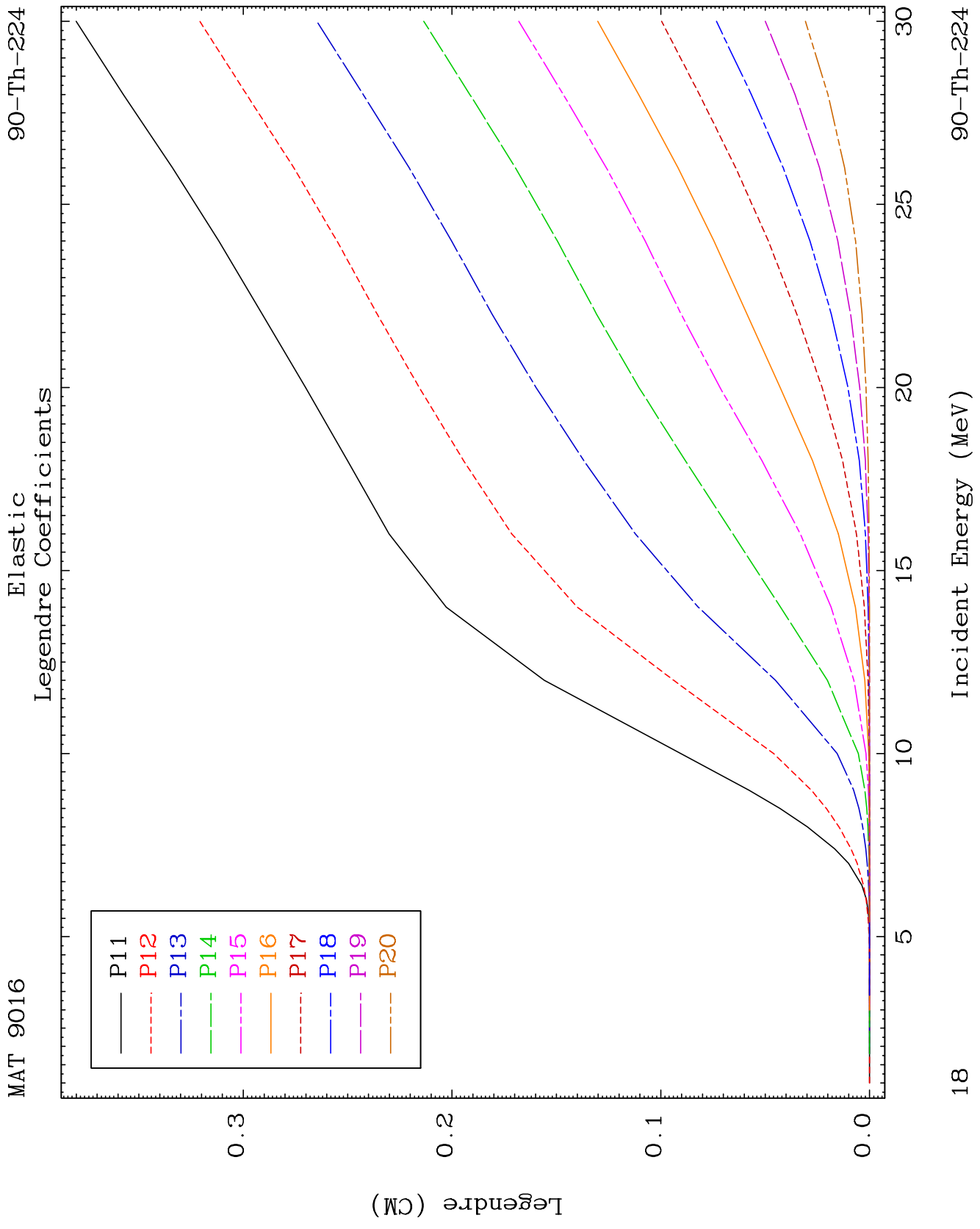
90-Th-224







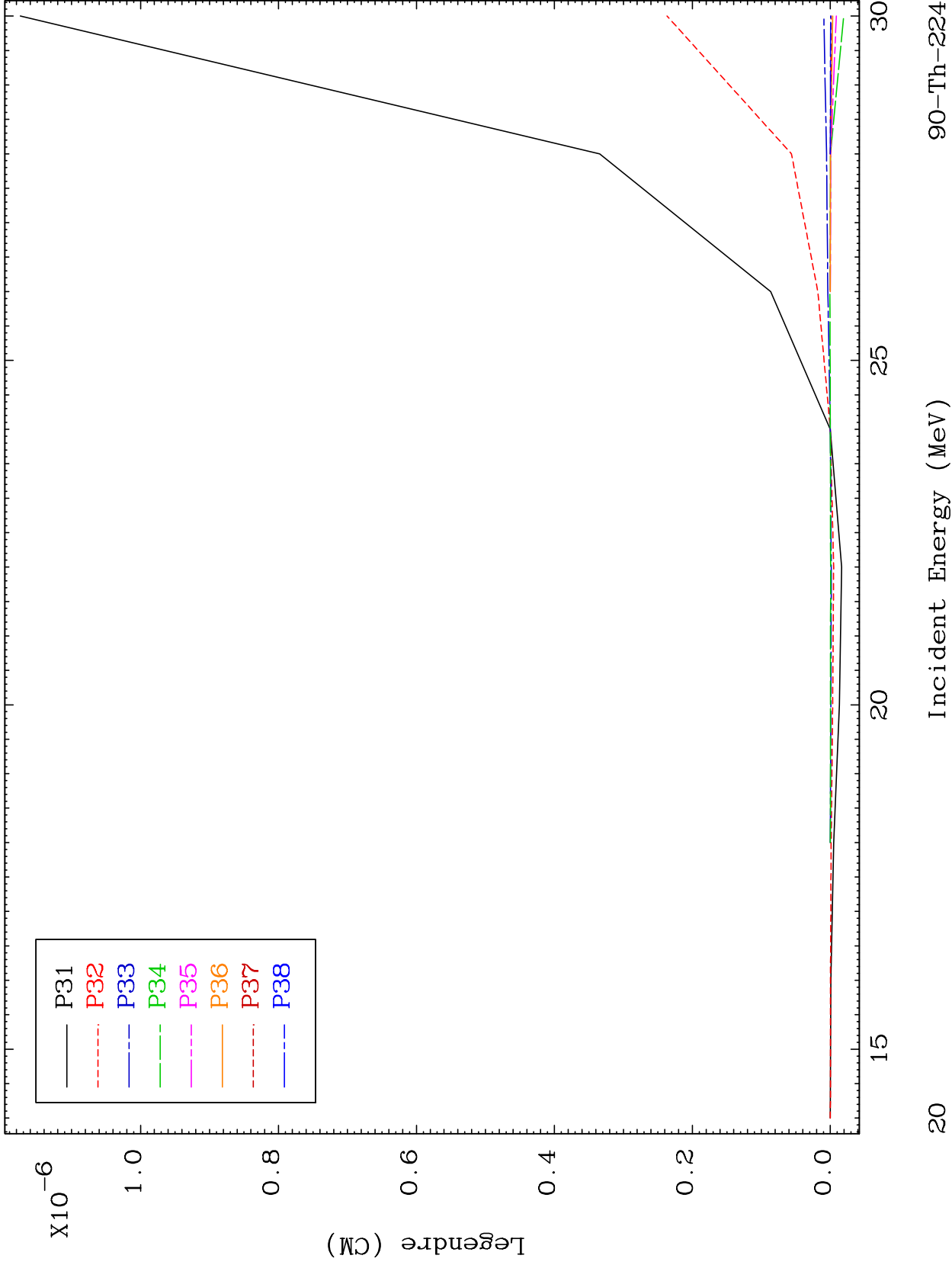




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Elastic Legendre Coefficients

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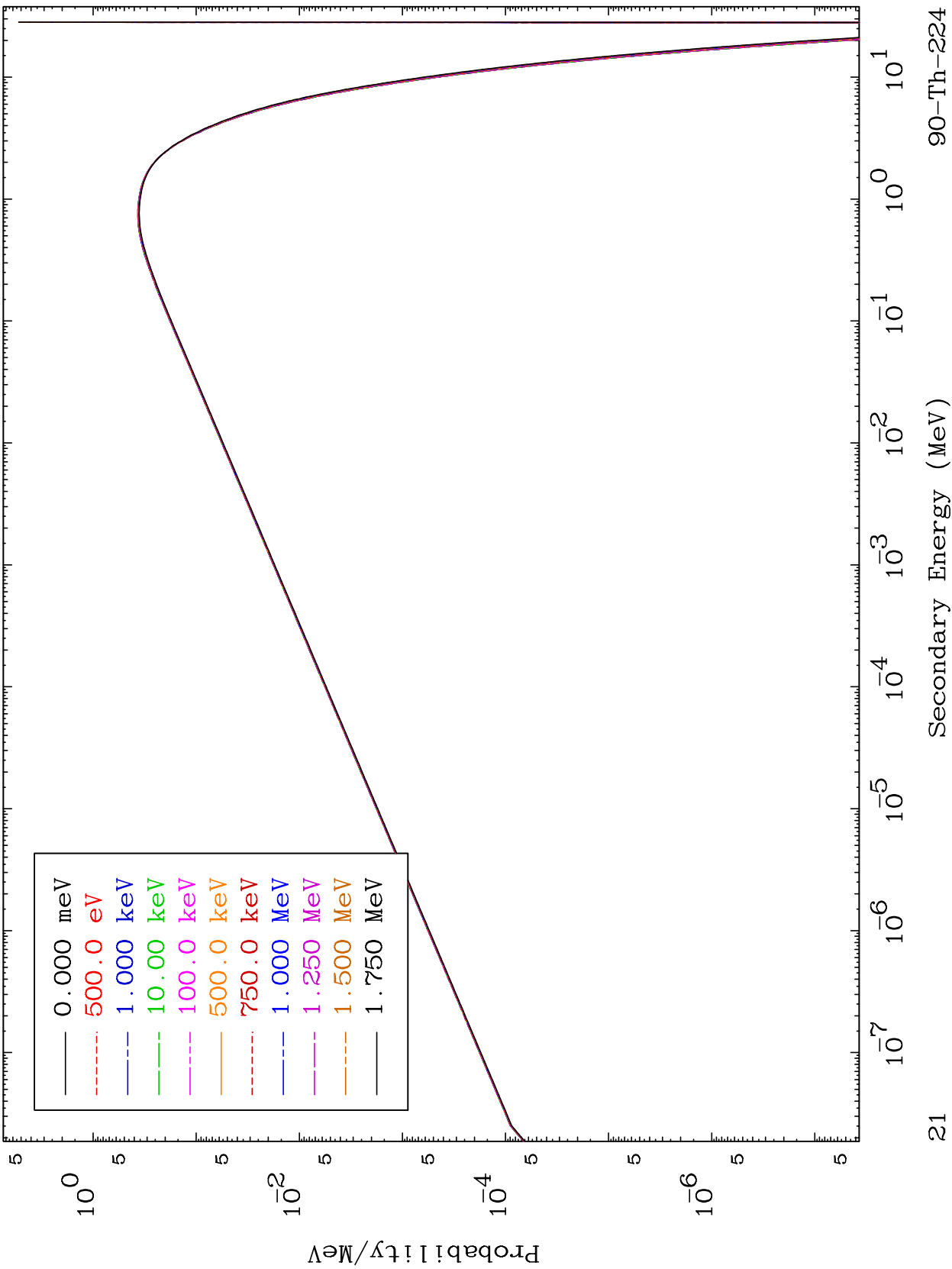
Incident Energy (MeV)

90-Th-224

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Fission Energy Distribution

90-Th-224

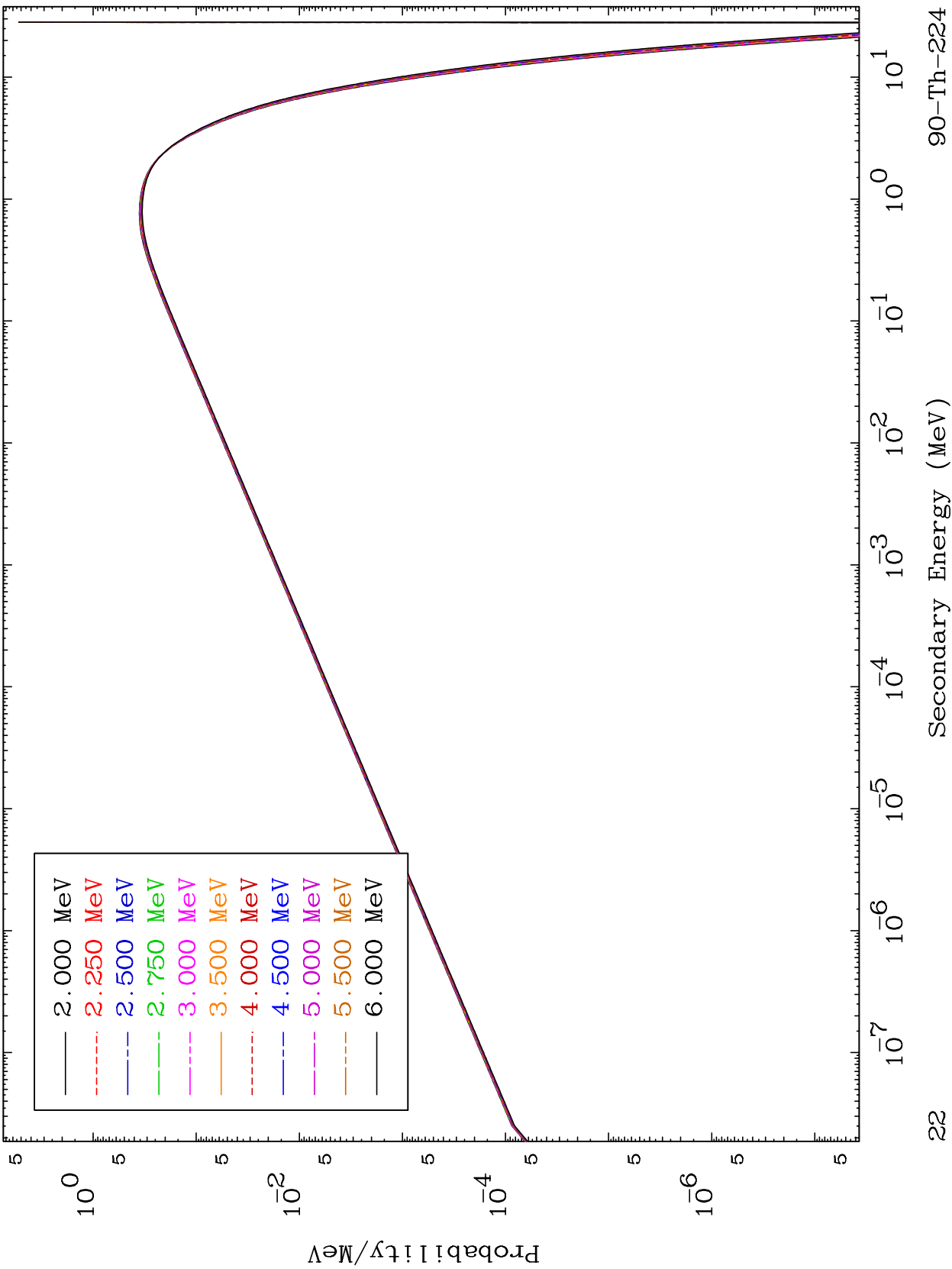


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Fission Energy Distribution

90-Th-224

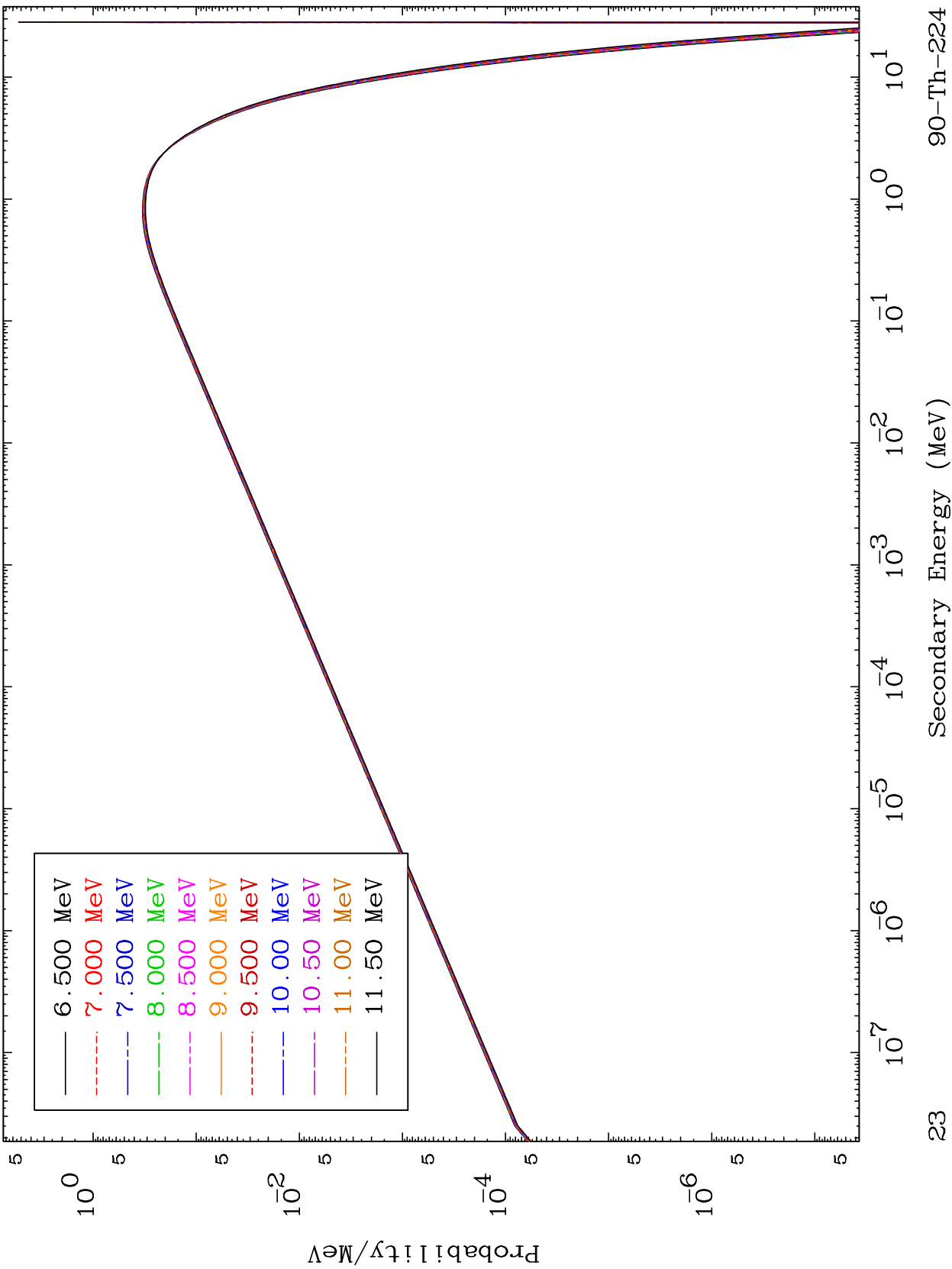


90-Th-224

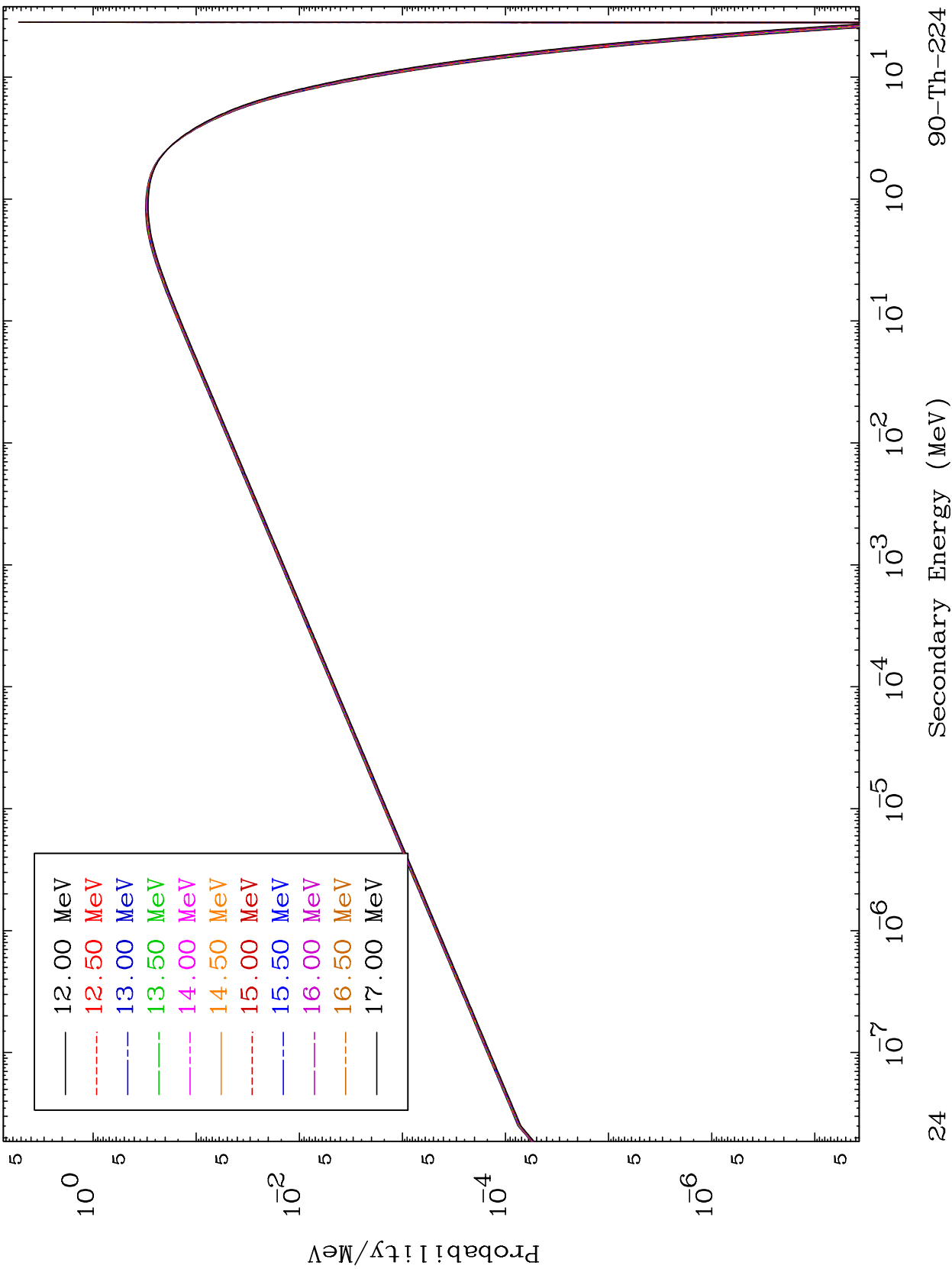
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Fission Energy Distribution

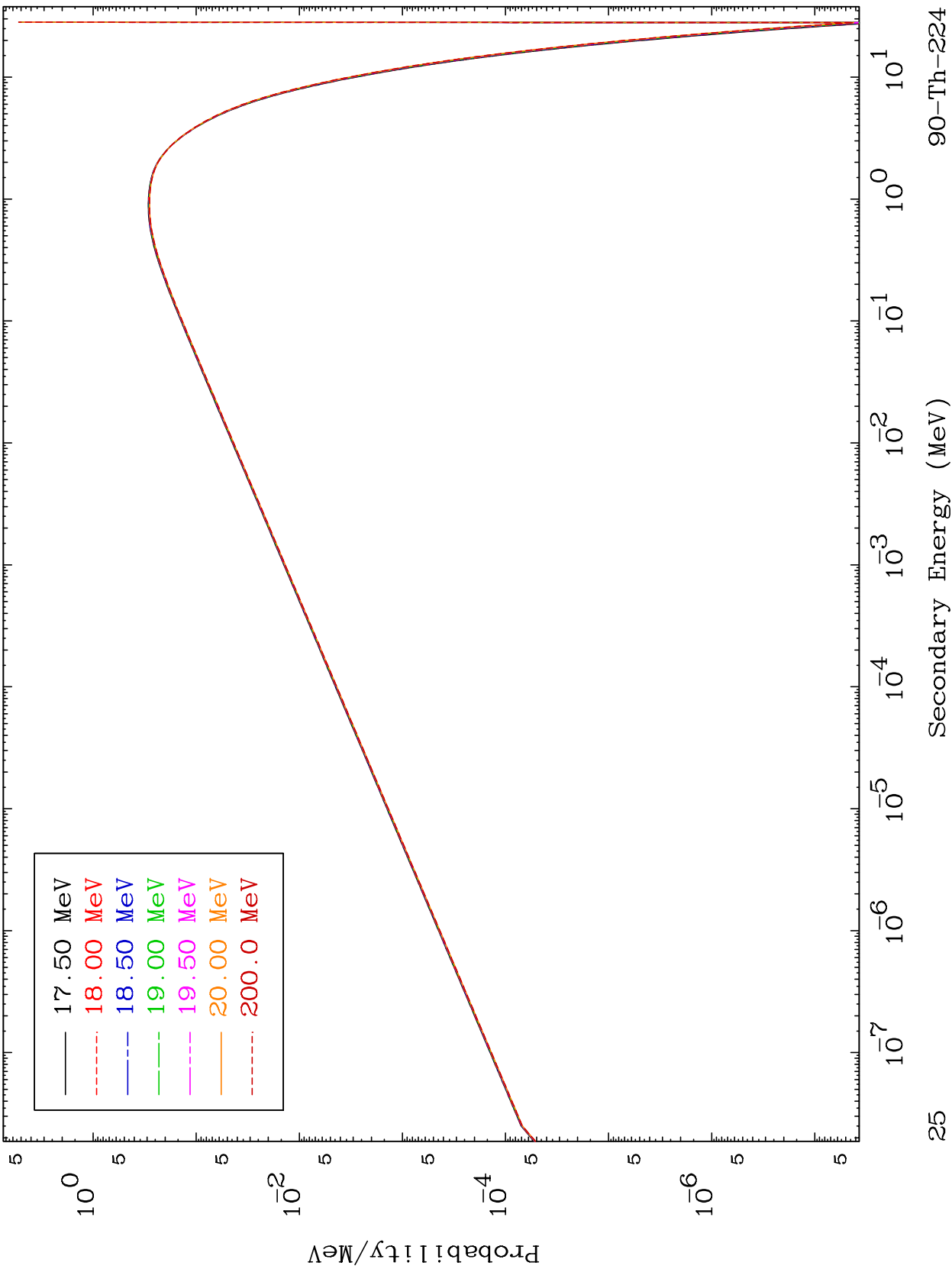
90-Th-224



Fission Energy Distribution



Fission
Energy Distribution

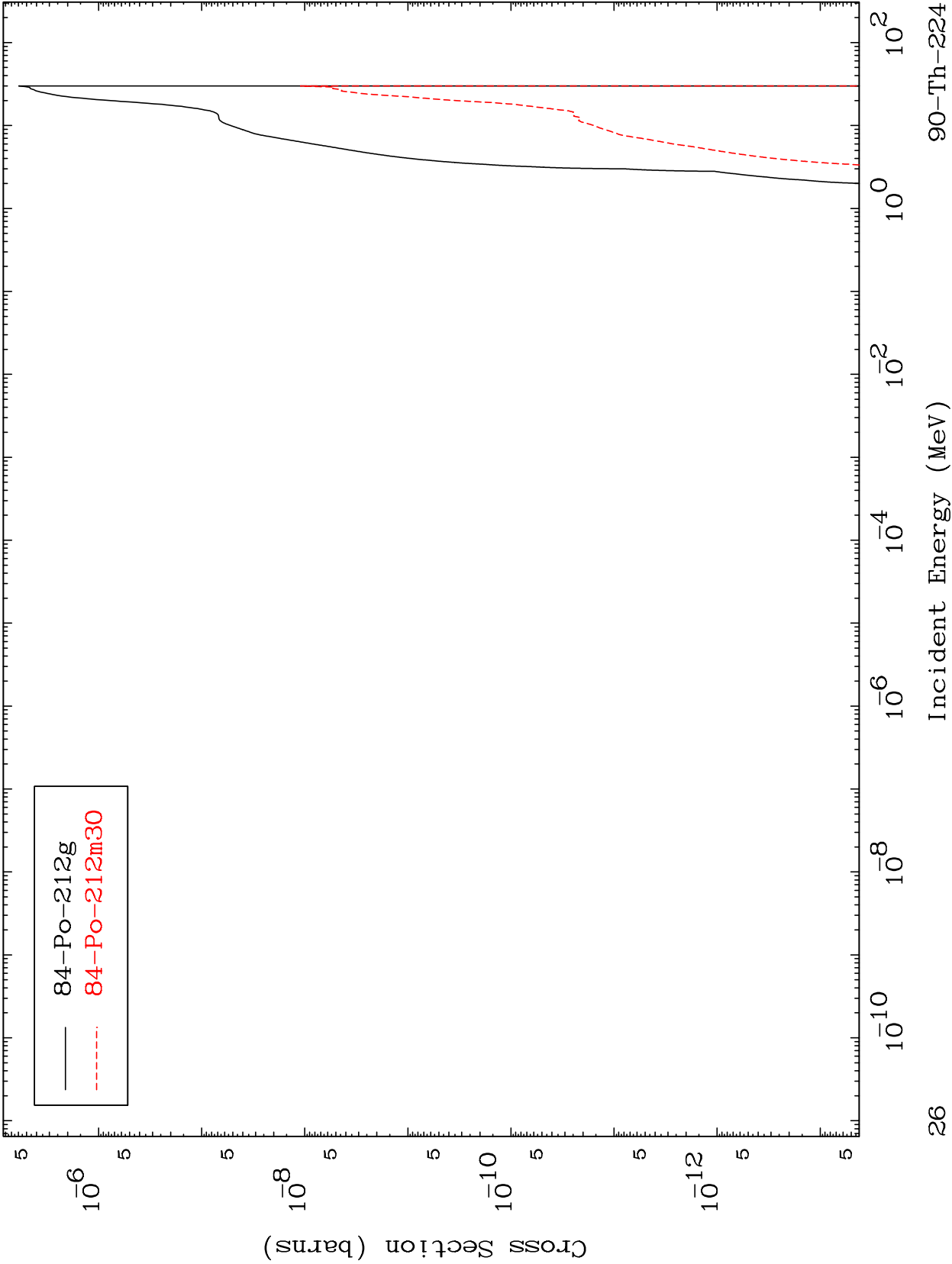


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(n,n') 3 α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

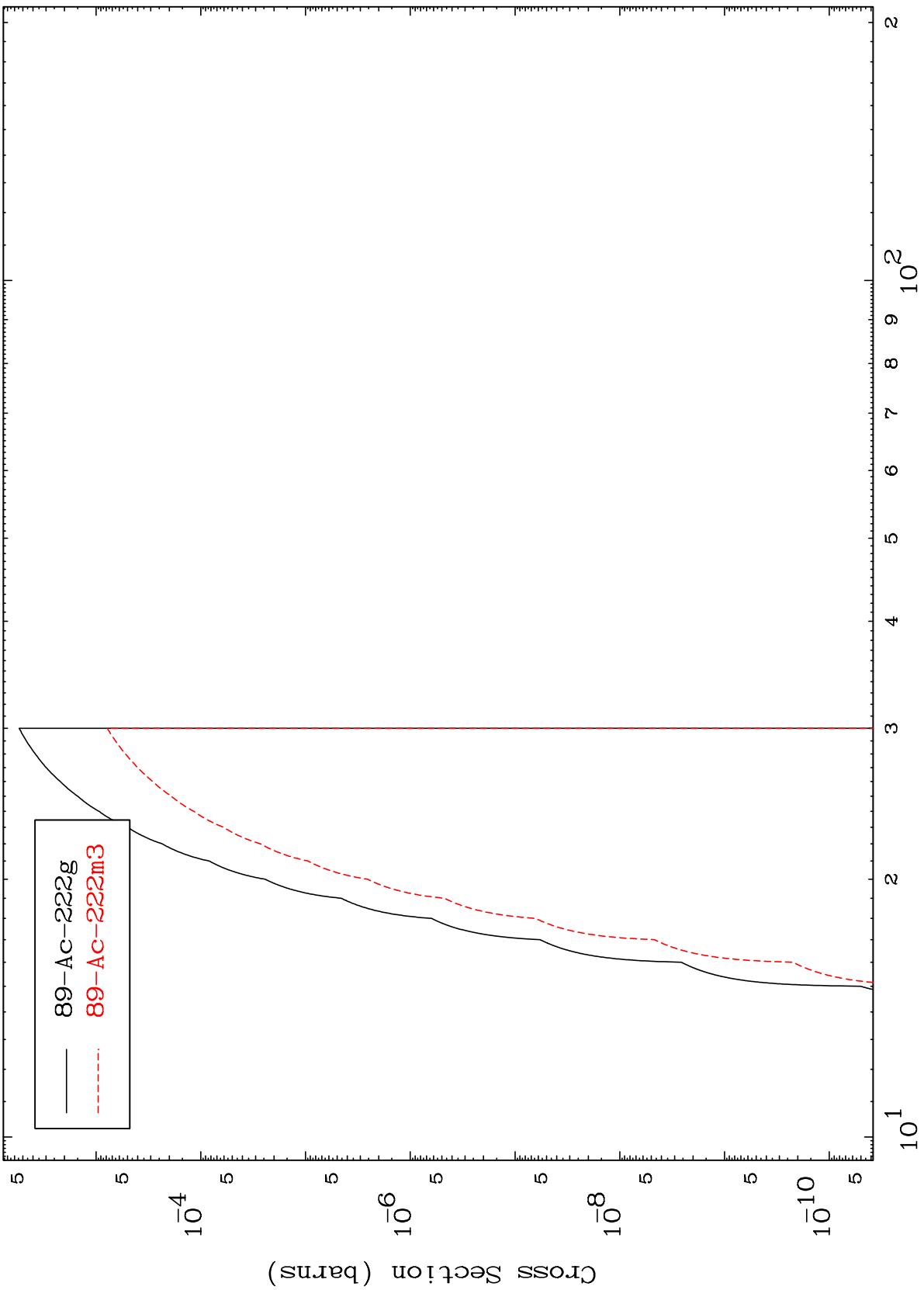
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(n,n') d

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Radionuclide Production Cross Section

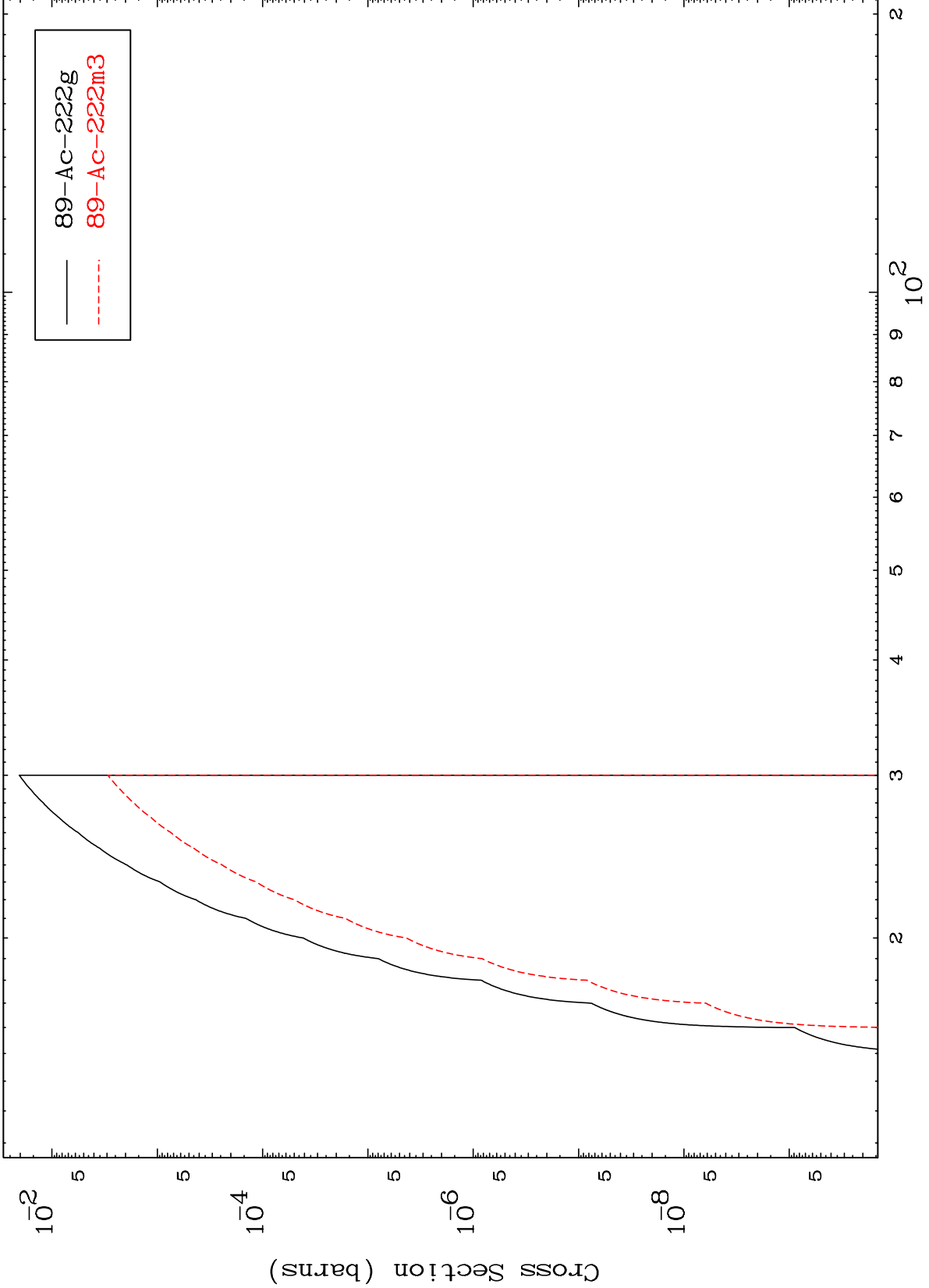


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Incident Energy (MeV)

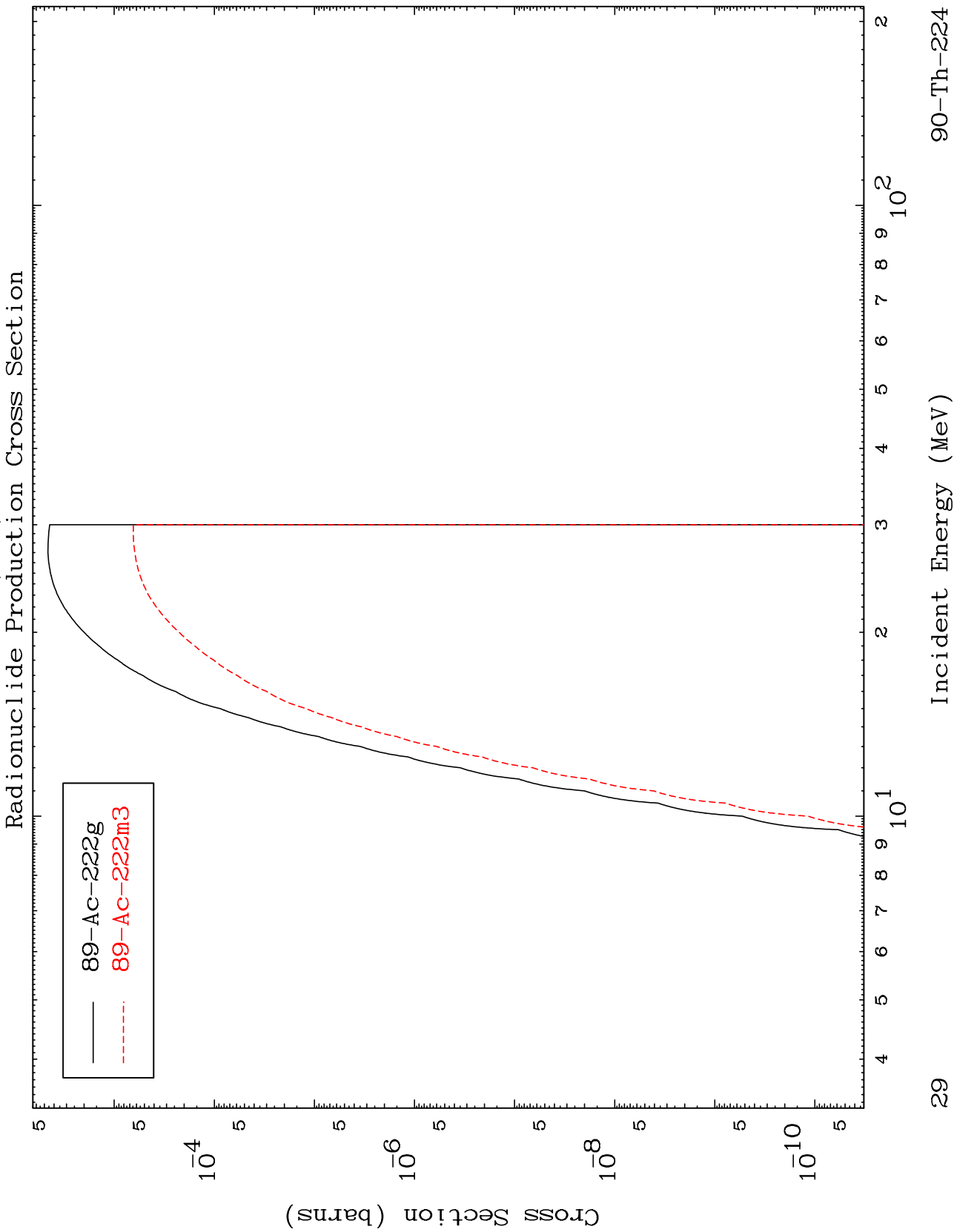
90-Th-224

Radionuclide Production Cross Section



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90-Th-224



90-Th-224

Incident Energy (MeV)

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