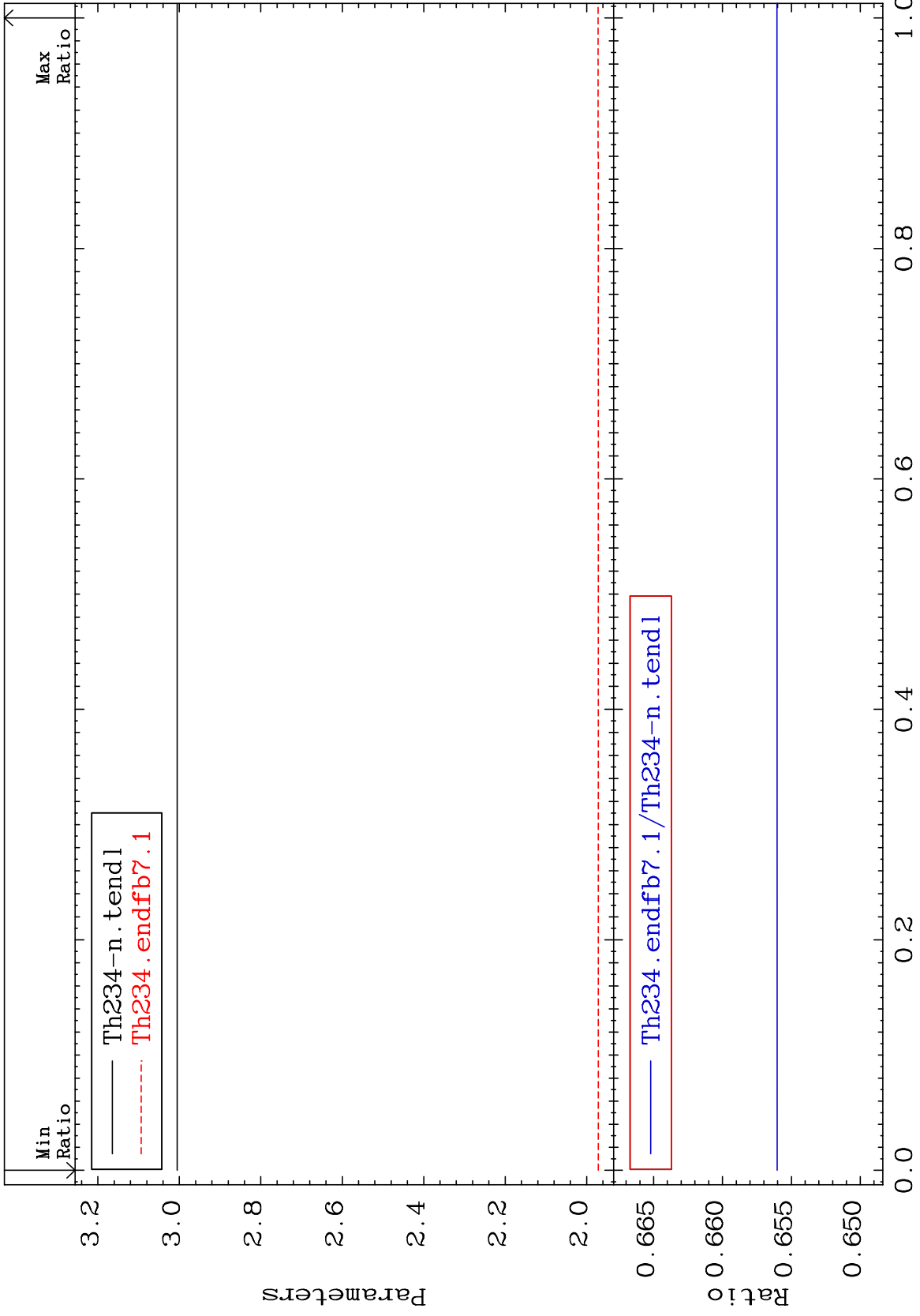


MAT 9046

Total  $\bar{\nu}$   
Parameters

90-Th-234  
-34.40 To -34.40%



Incident Energy (KeV)

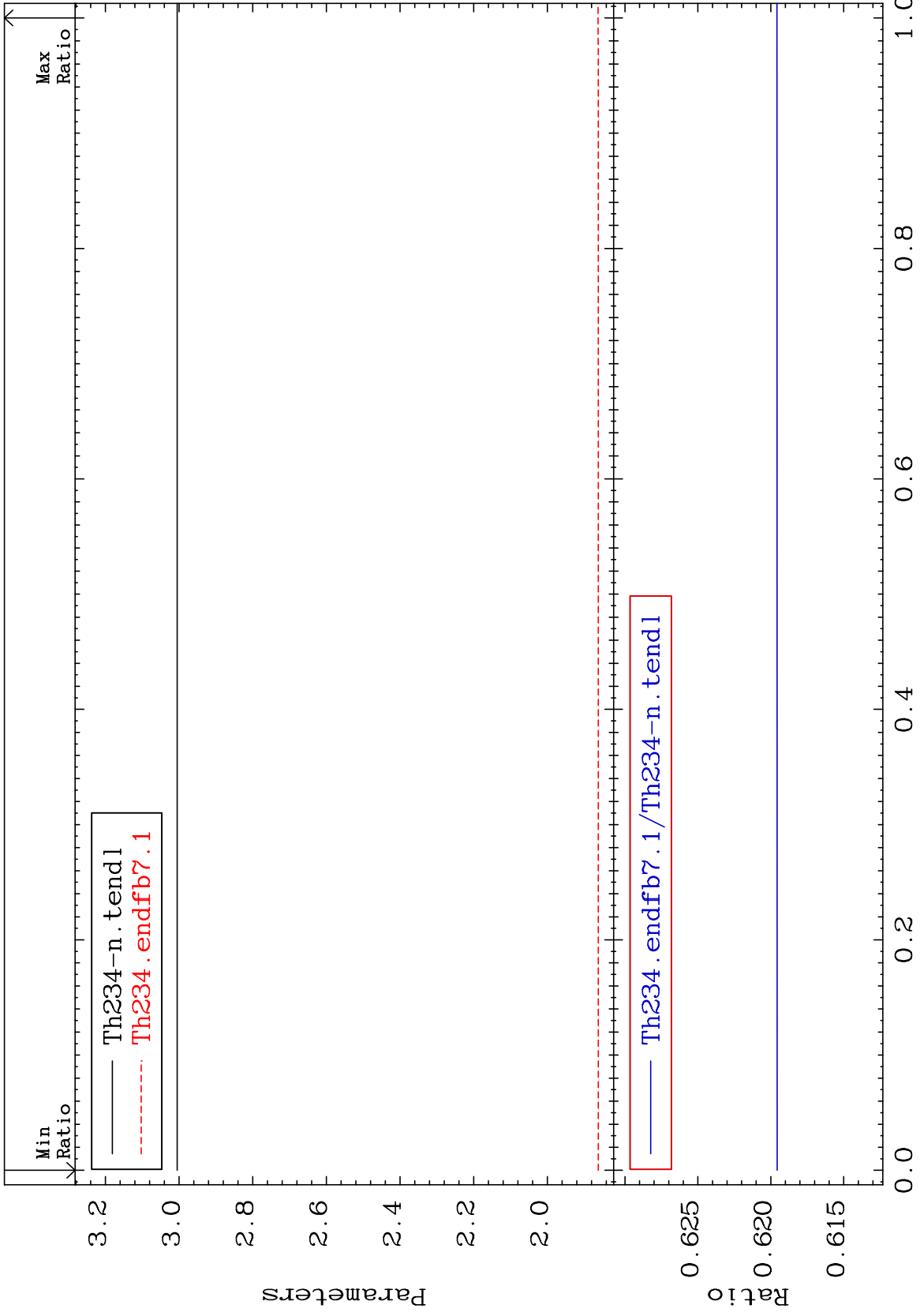
90-Th-234

1

MAT 9046

Prompt  $\bar{\nu}$   
Parameters

90-Th-234  
-38.04 To -38.04%



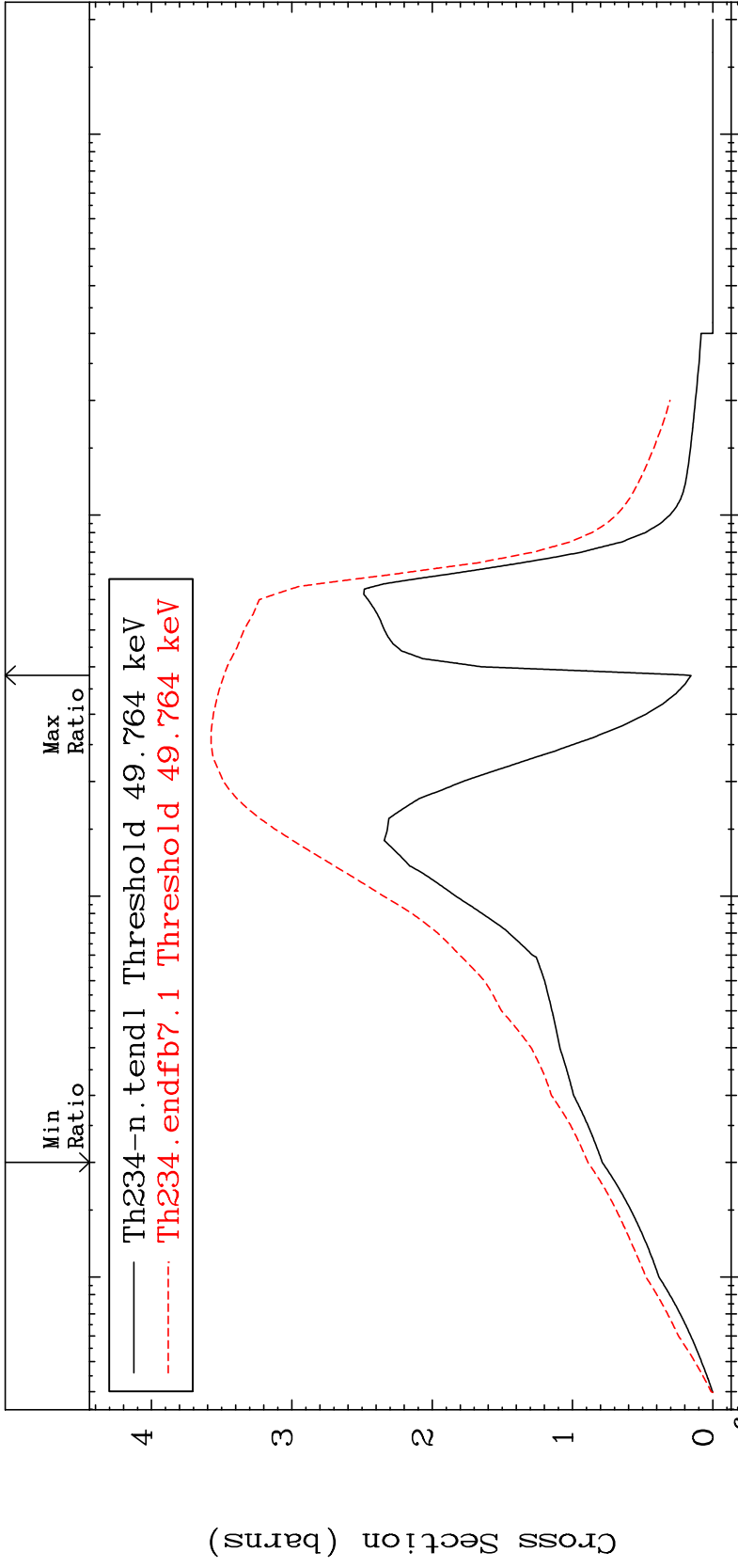
2

Incident Energy (KeV)

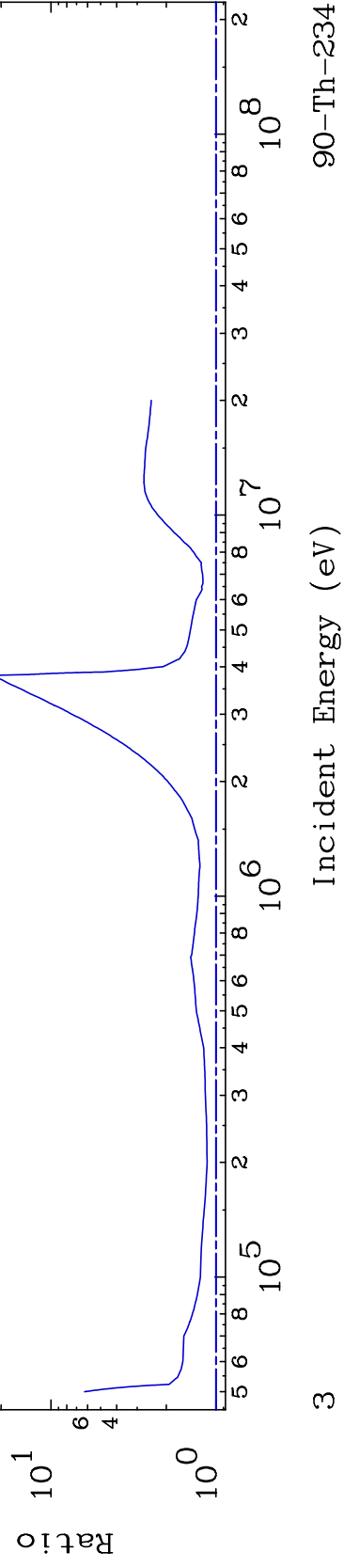
90-Th-234

MAT 9046

Inelastic Cross Section  
90-Th-234  
13.00 To 2177. %



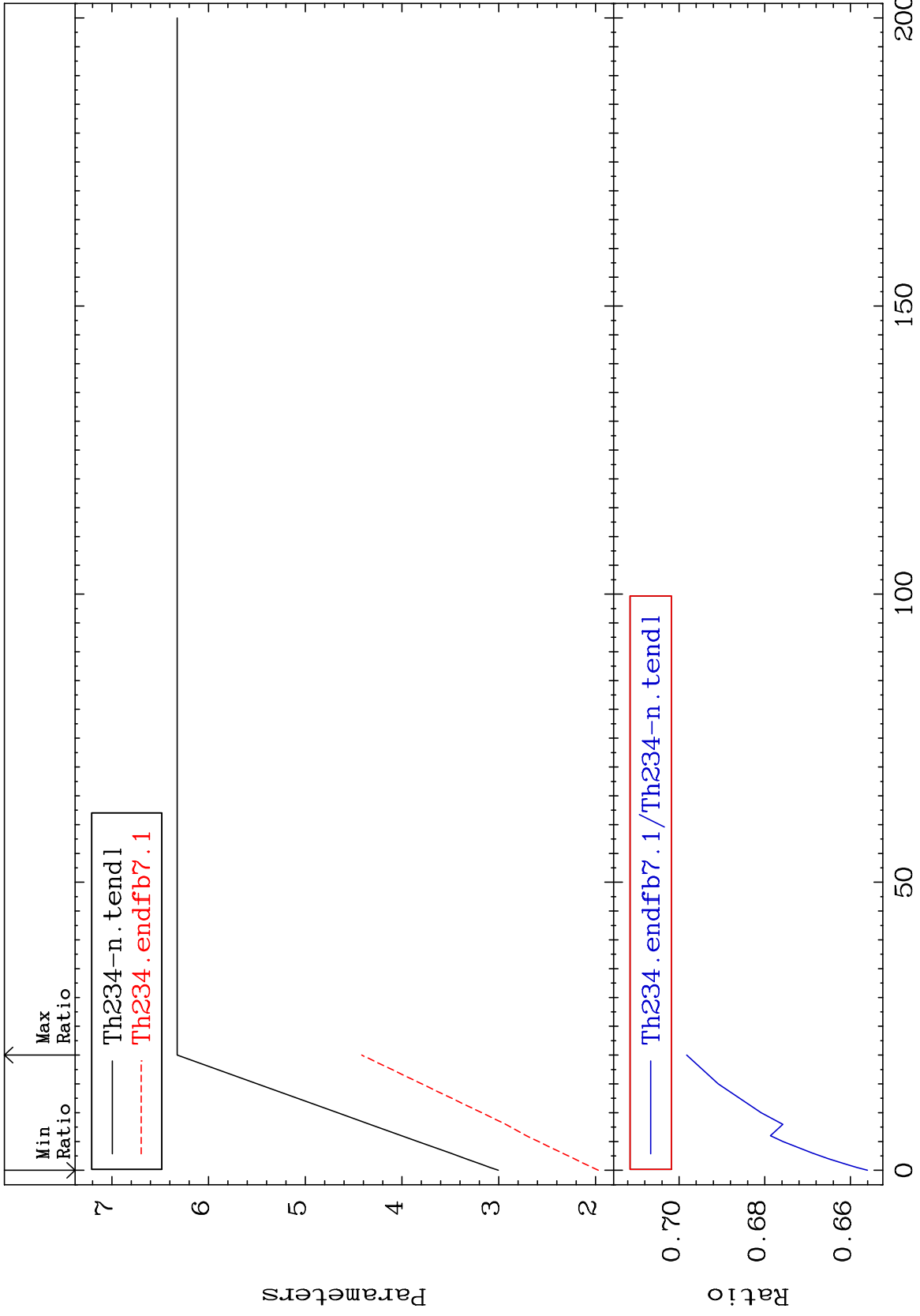
Th234.endfb7.1/Th234-n.tendl



MAT 9046

Total  $\bar{\nu}$   
Parameters

90-Th-234  
-34.40 To -30.18%



1

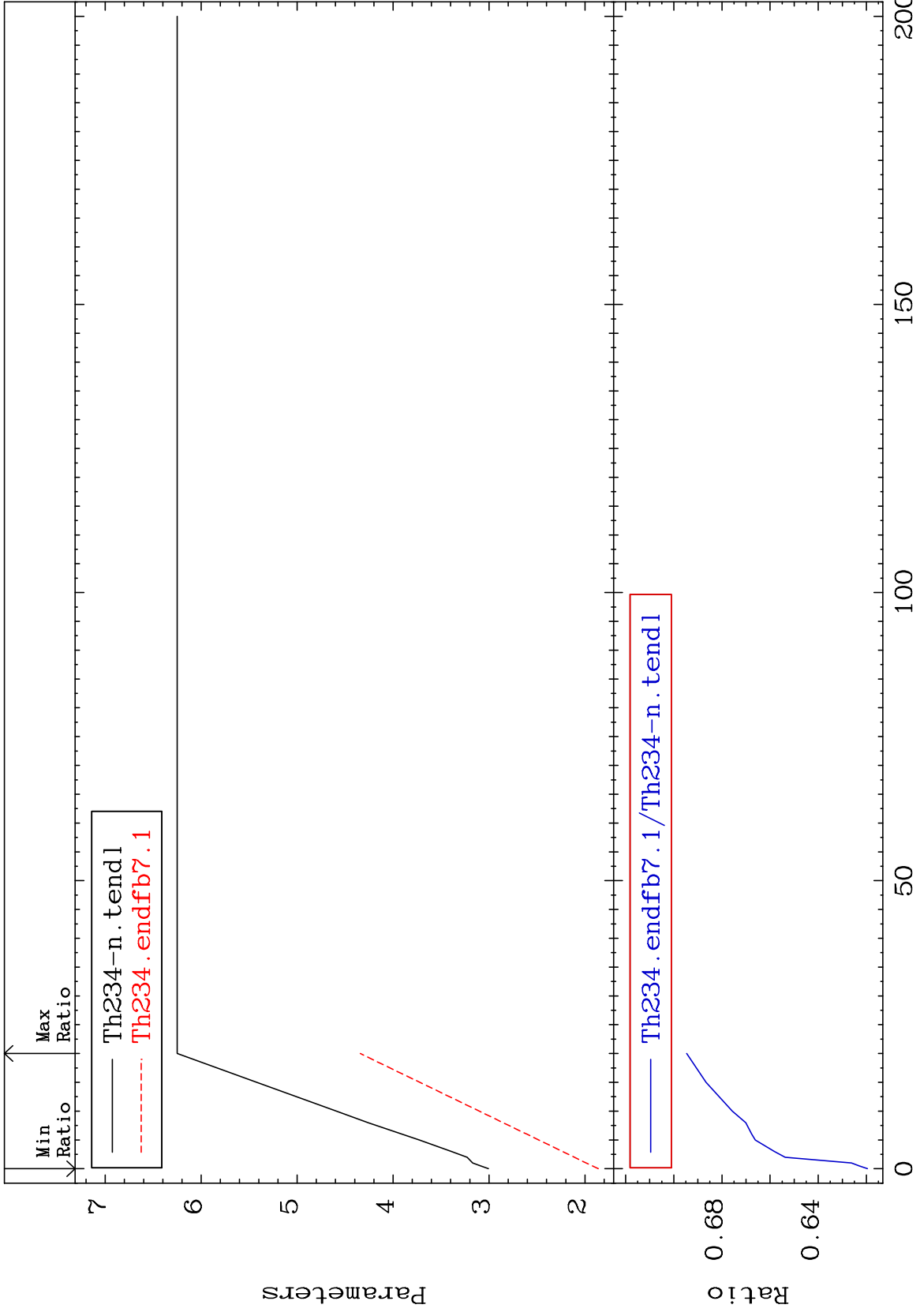
Incident Energy (MeV)

90-Th-234

MAT 9046

Prompt  $\bar{\nu}$   
Parameters

90-Th-234  
-38.04 To -30.54%



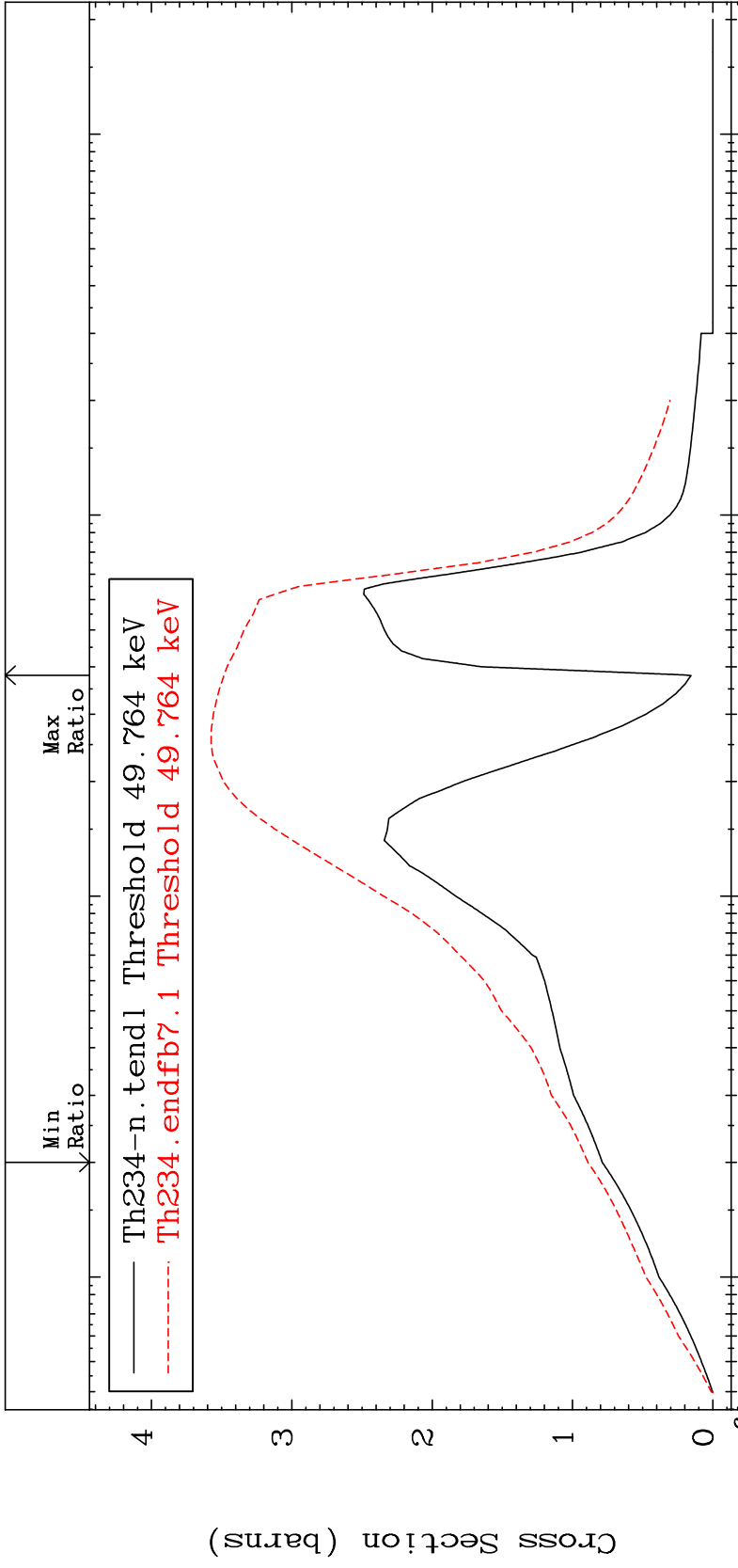
2

Incident Energy (MeV)

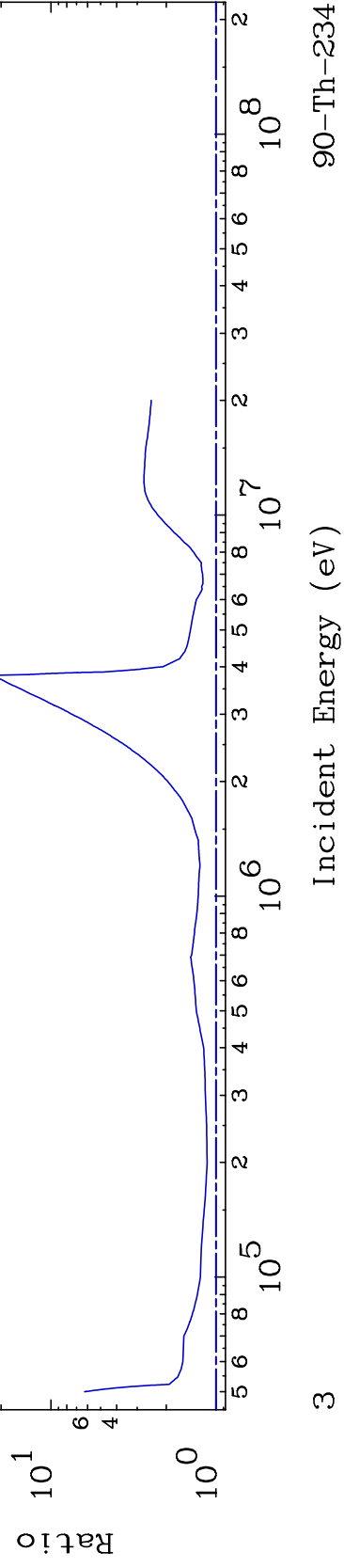
90-Th-234

MAT 9046

Inelastic Cross Section 90-Th-234 To 2177. % 13.00



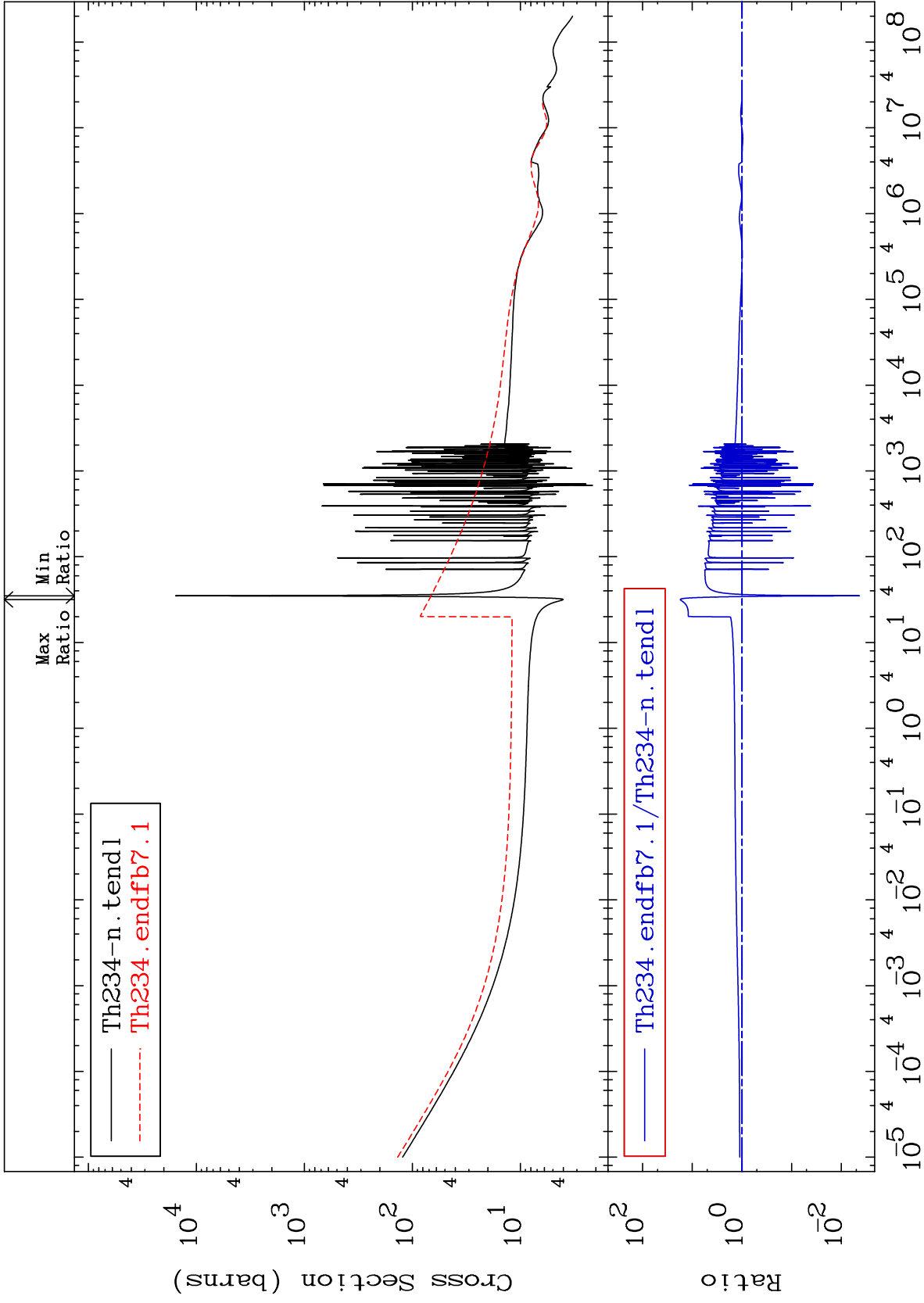
Th234.endfb7.1/Th234-n.tendl



90-Th-234

MAT 9046

Total Cross Section  
90-Th-234  
-99.57 To 1639. %



90-Th-234

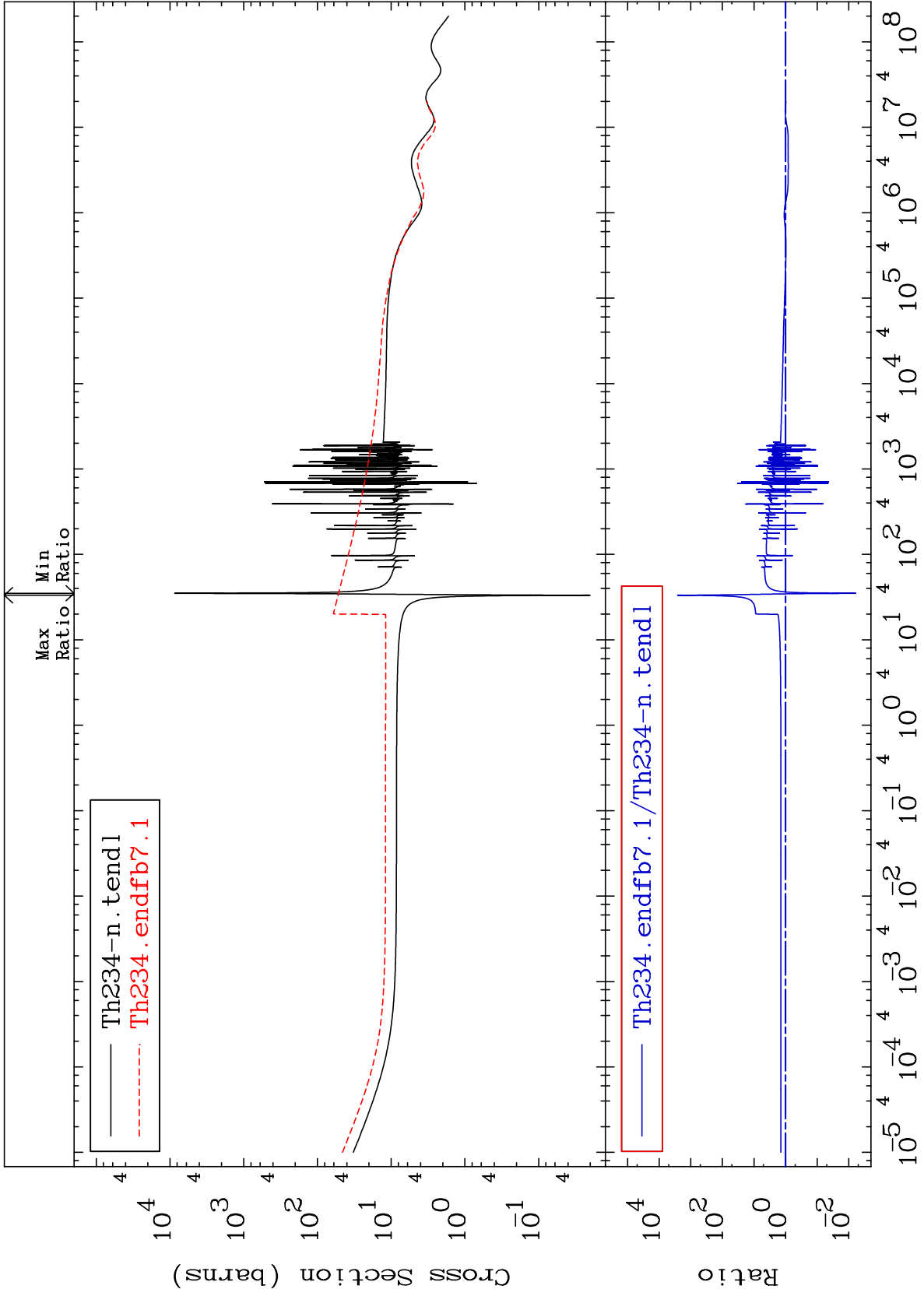
Incident Energy (eV)

1

MAT 9046

Elastic  
Cross Section

90-Th-234  
-99.40 To 9999. %



2

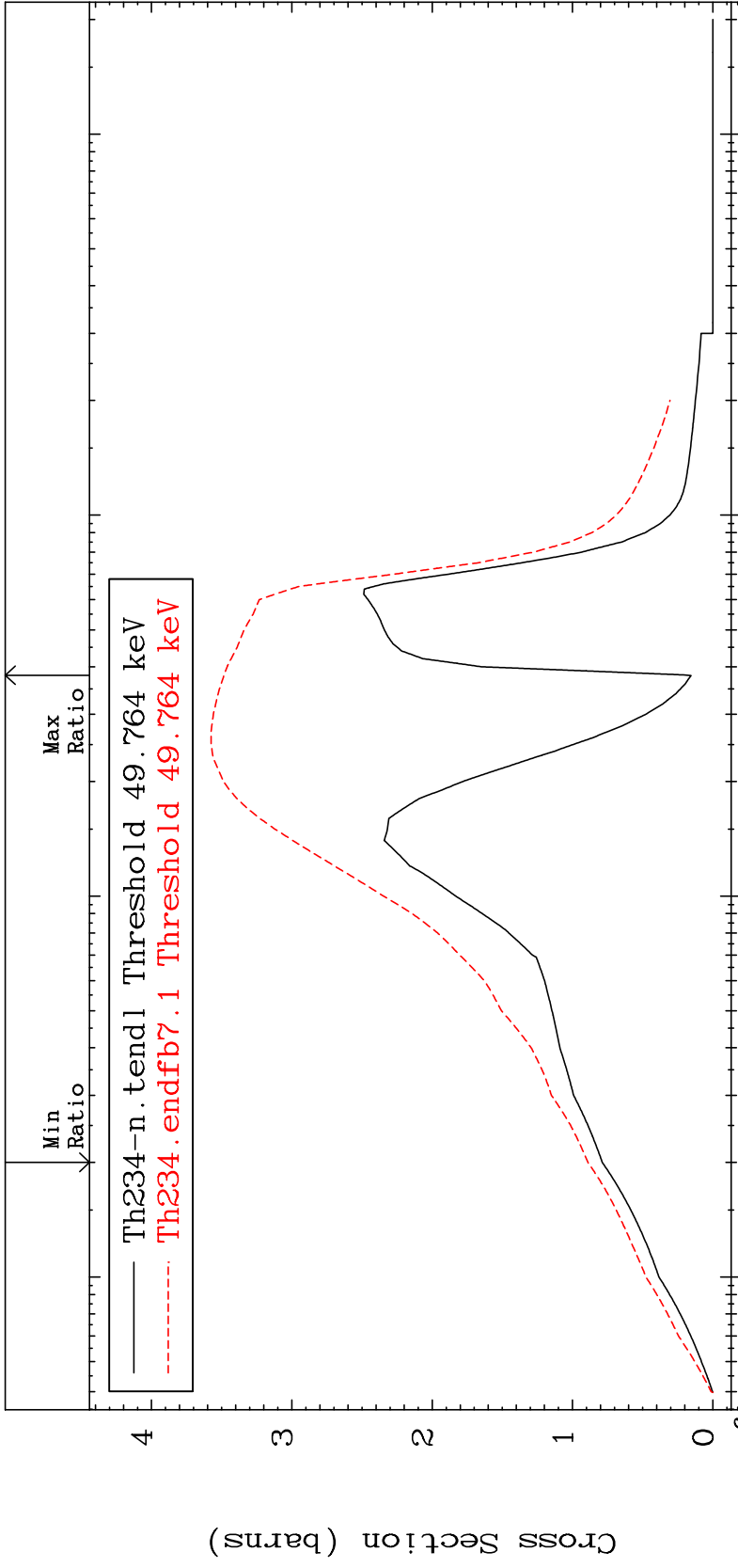
Incident Energy (eV)

90-Th-234

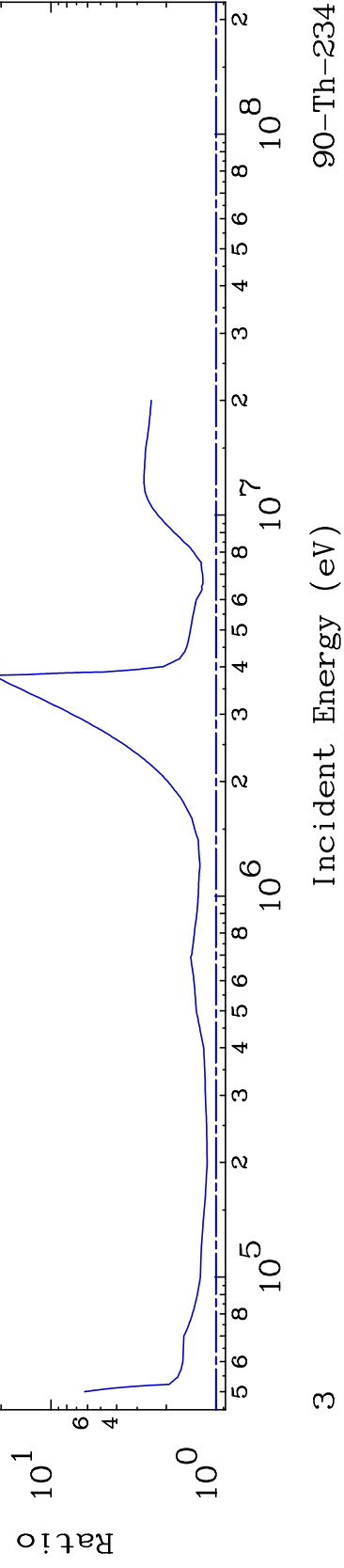


MAT 9046

Inelastic Cross Section  
90-Th-234  
13.00 To 2177. %



Th234.endfb7.1/Th234-n.tendl

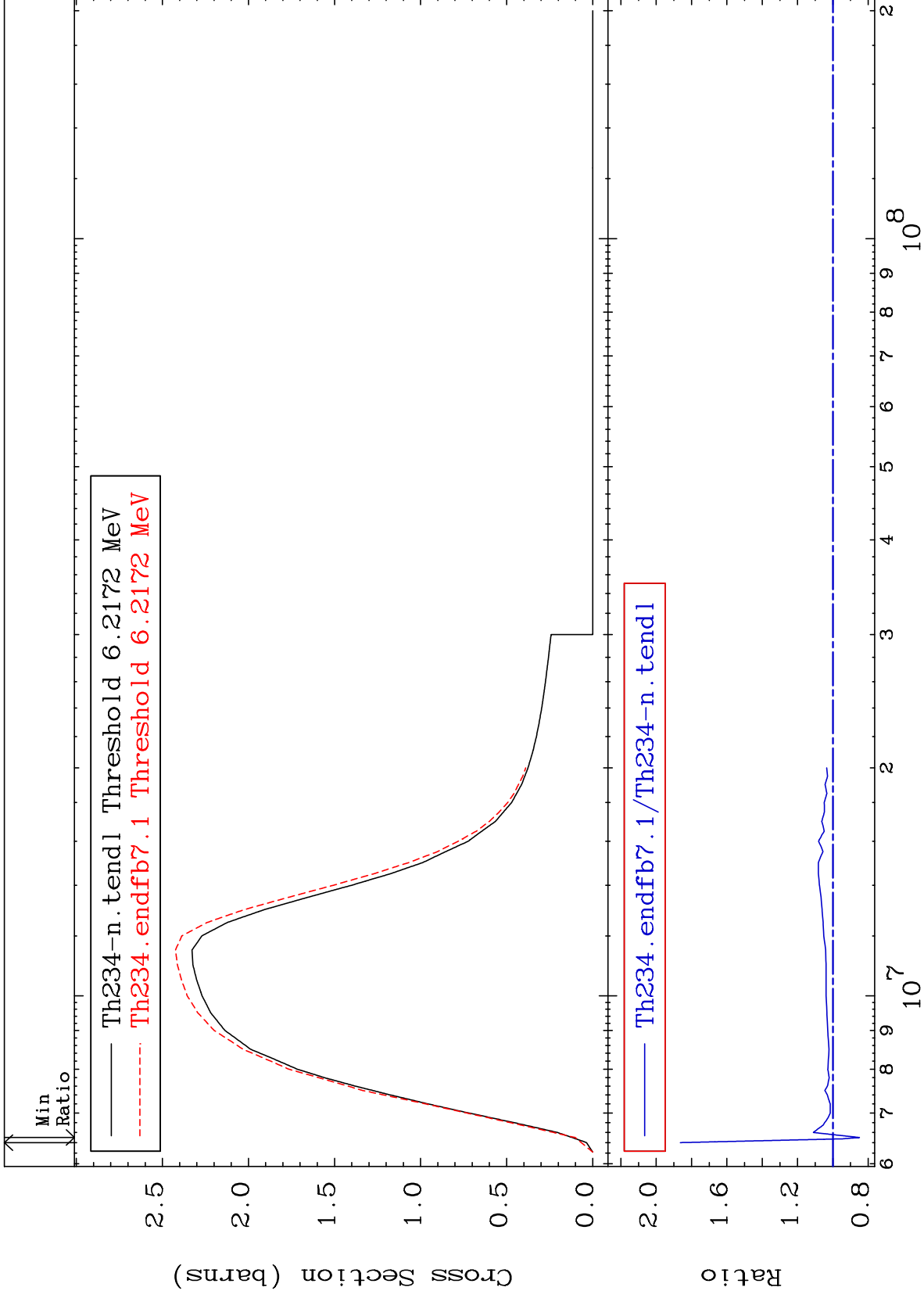


90-Th-234

MAT 9046

(n,2n)  
Cross Section

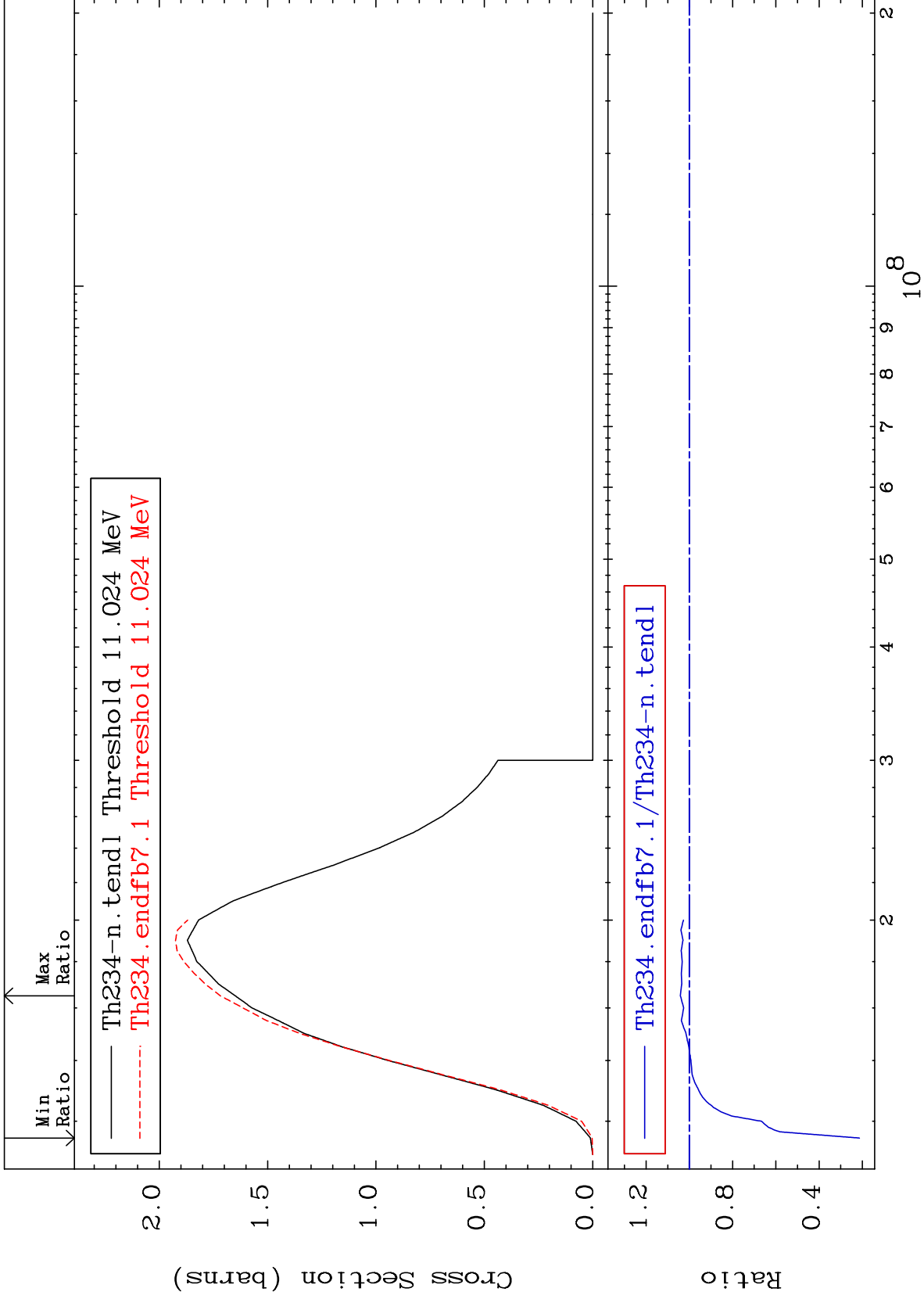
90-Th-234  
-15.03 To 86.34 %



MAT 9046

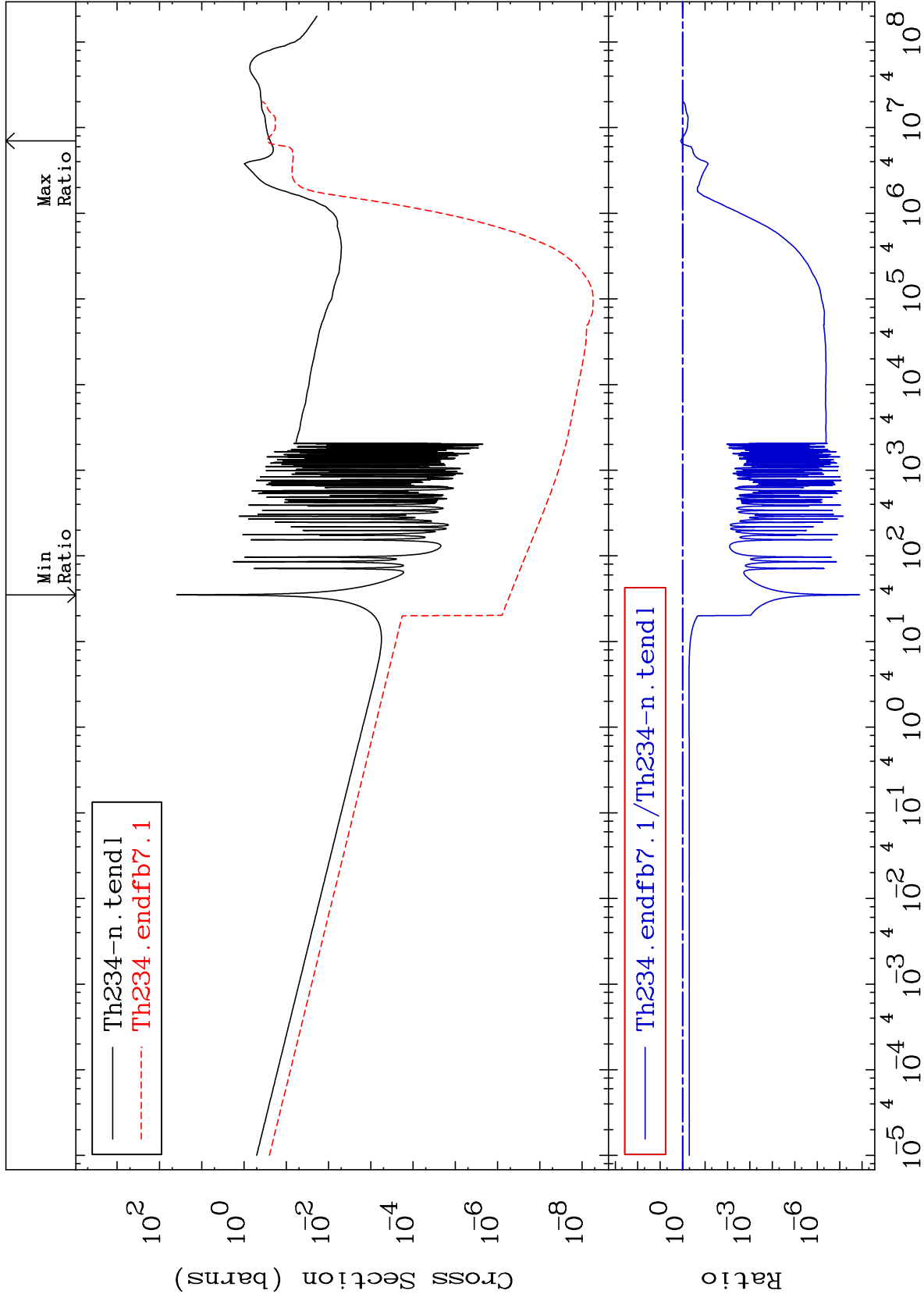
(n,3n)  
Cross Section

90-Th-234  
-78.66 To 4.158 %



MAT 9046

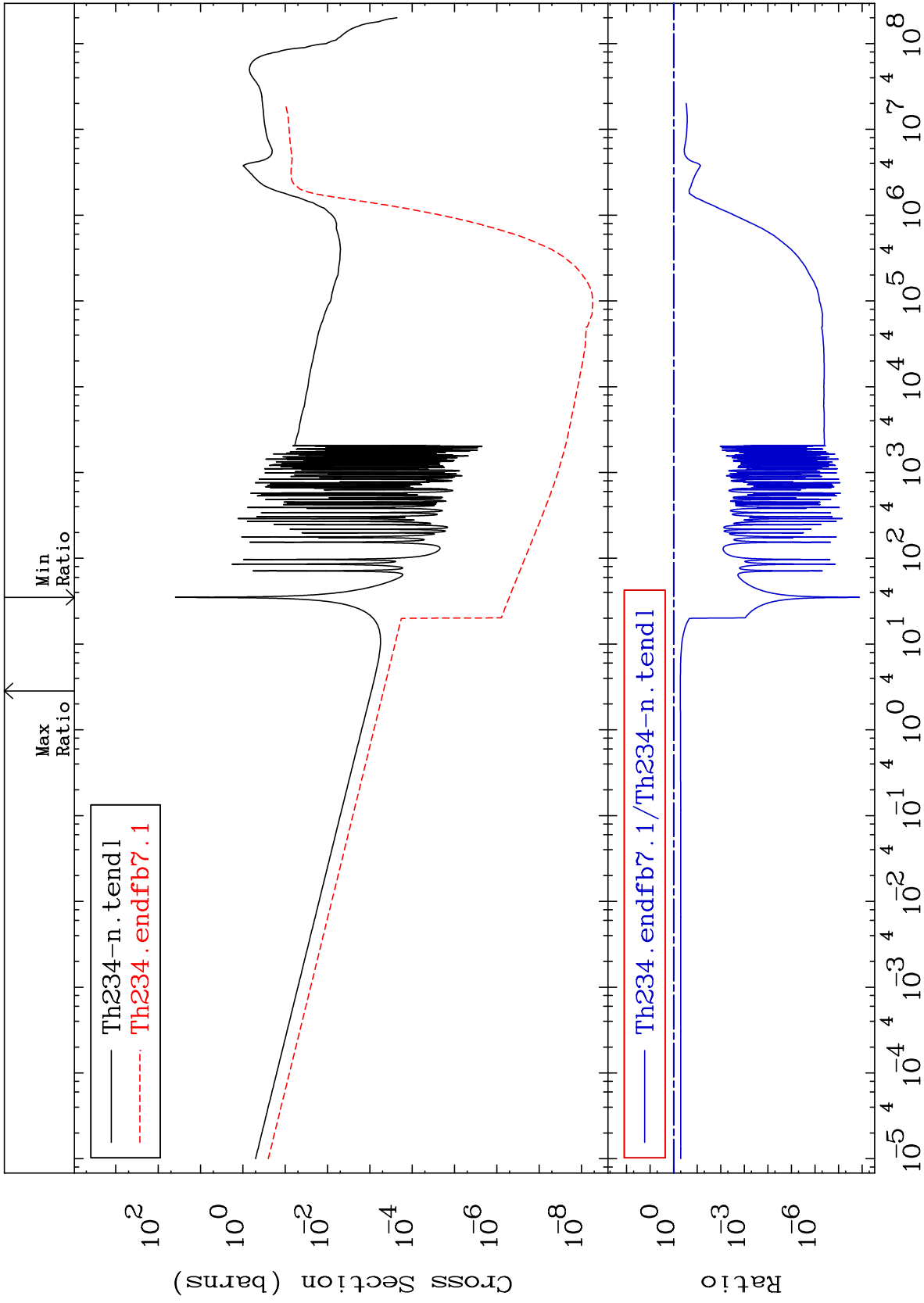
Fission Cross Section  
90-Th-234  
-100.0 To 20.78 %



MAT 9046

(n,f) First Chance  
Cross Section

90-Th-234  
-100.0 To -48.03%



7

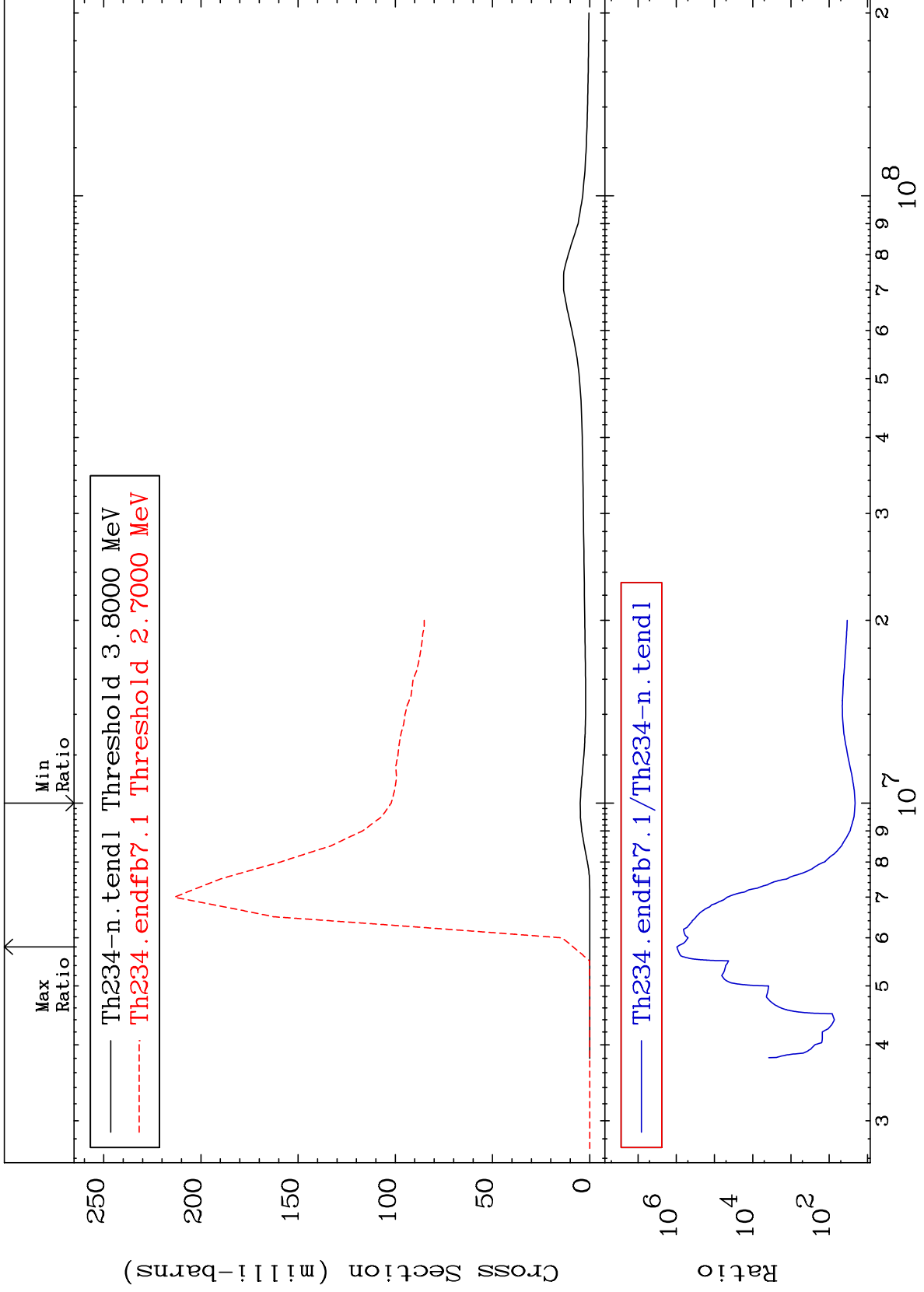
Incident Energy (eV)

90-Th-234

MAT 9046

(n, nf) Second Chance  
Cross Section

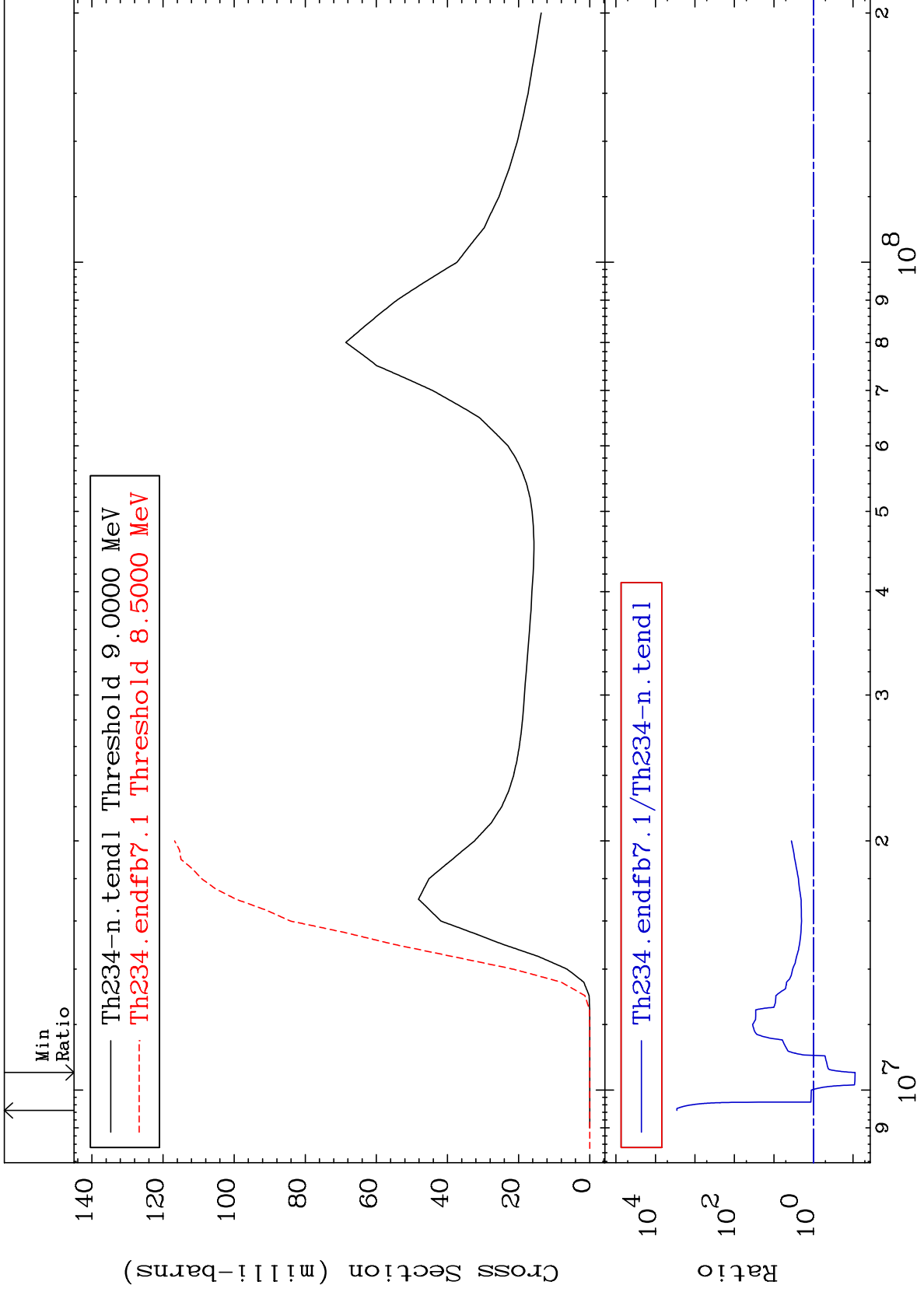
90-Th-234  
2006. To 9999. %



MAT 9046

(n,2nf) Third Chance  
Cross Section

90-Th-234  
-91.15 To 9999. %



9

90-Th-234

90-Th-234

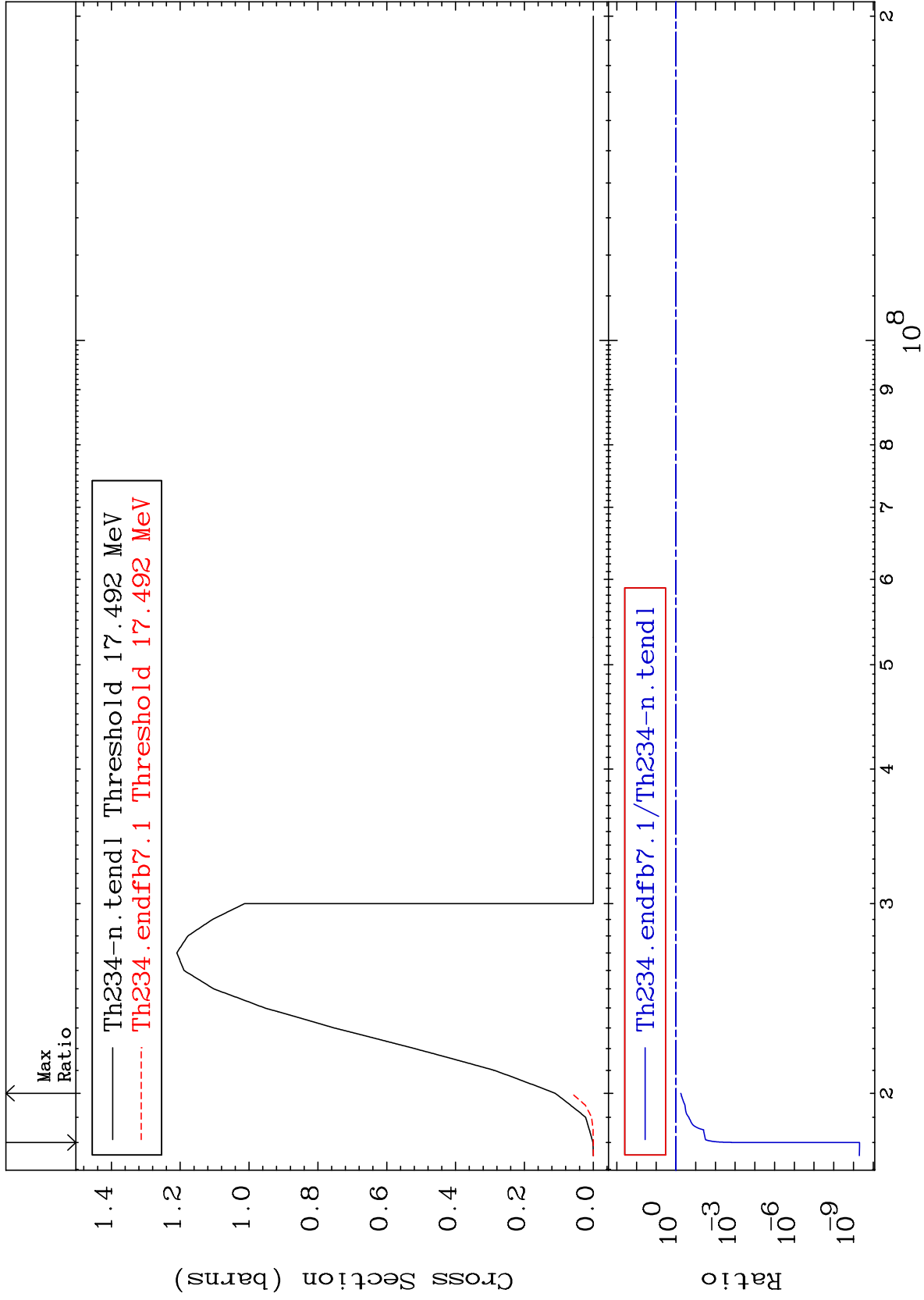
MAT 9046

(n,4n)

90-Th-234

Cross Section

-100.0 To -43.07%



10

Incident Energy (eV)

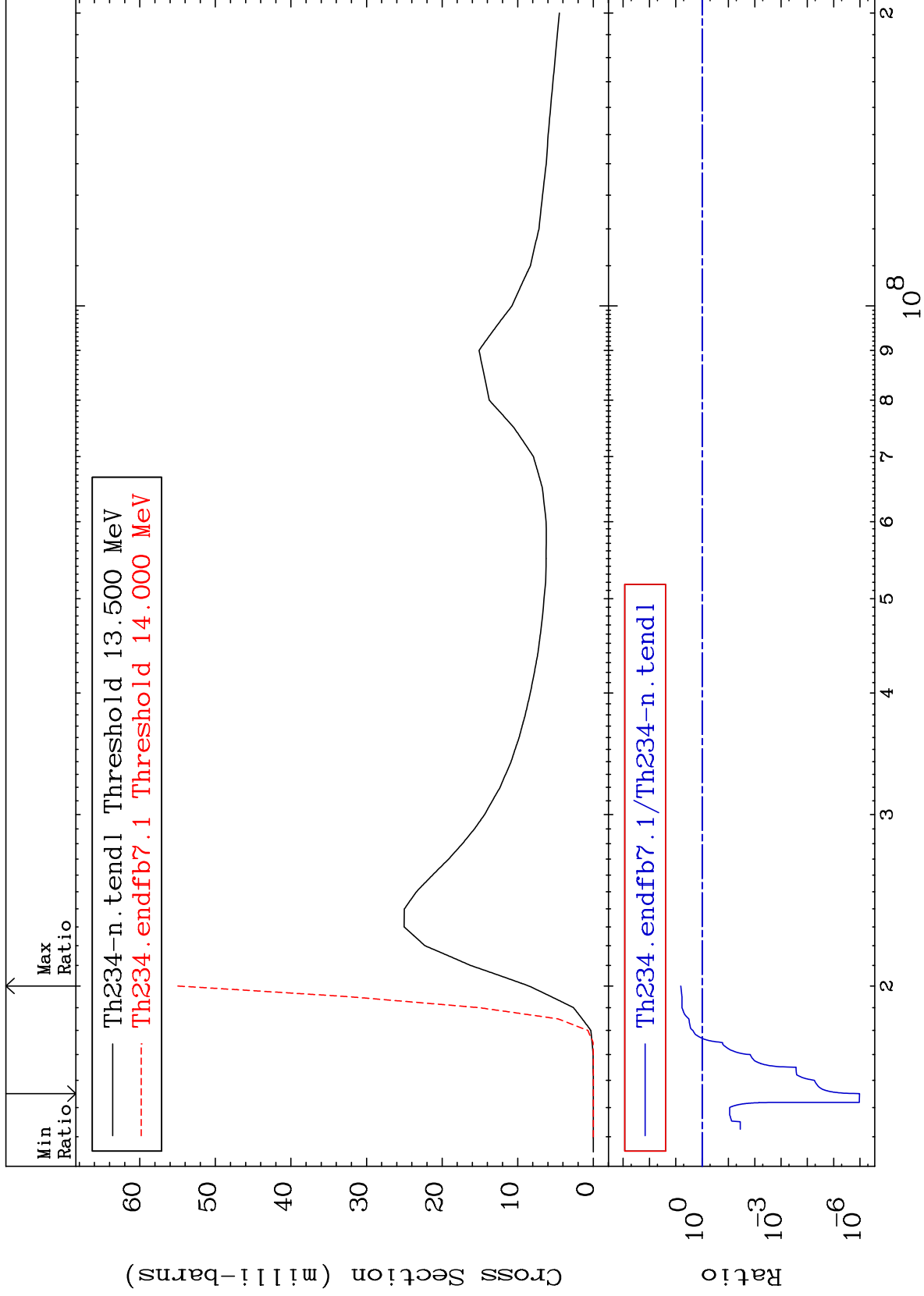
90-Th-234



MAT 9046

(n,3nf) Fourth Chance  
Cross Section

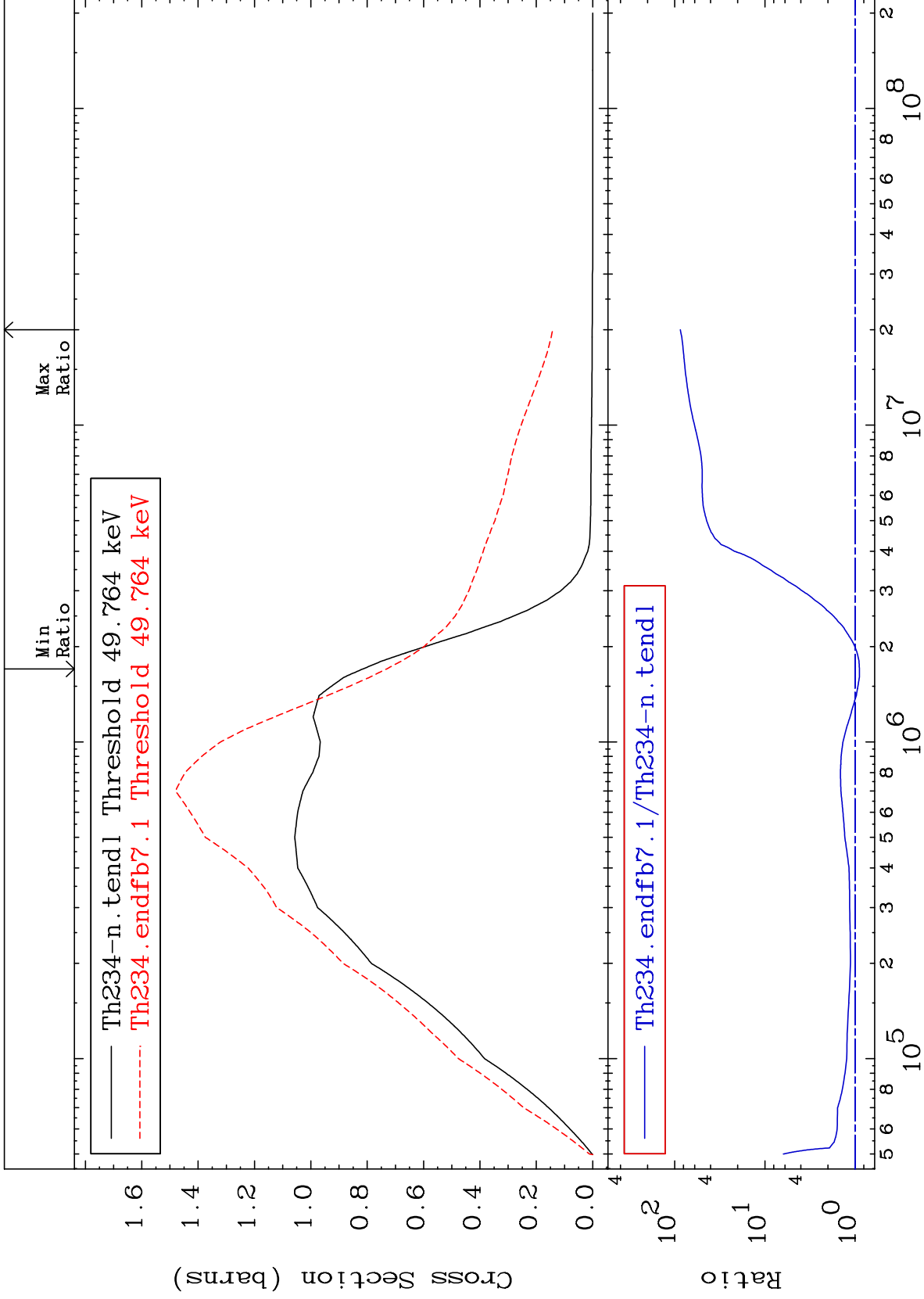
90-Th-234  
-100.0 To 546.7 %



MAT 9046

49.55 keV (n,n') Level  
Cross Section

90-Th-234  
-10.39 To 8553. %



12

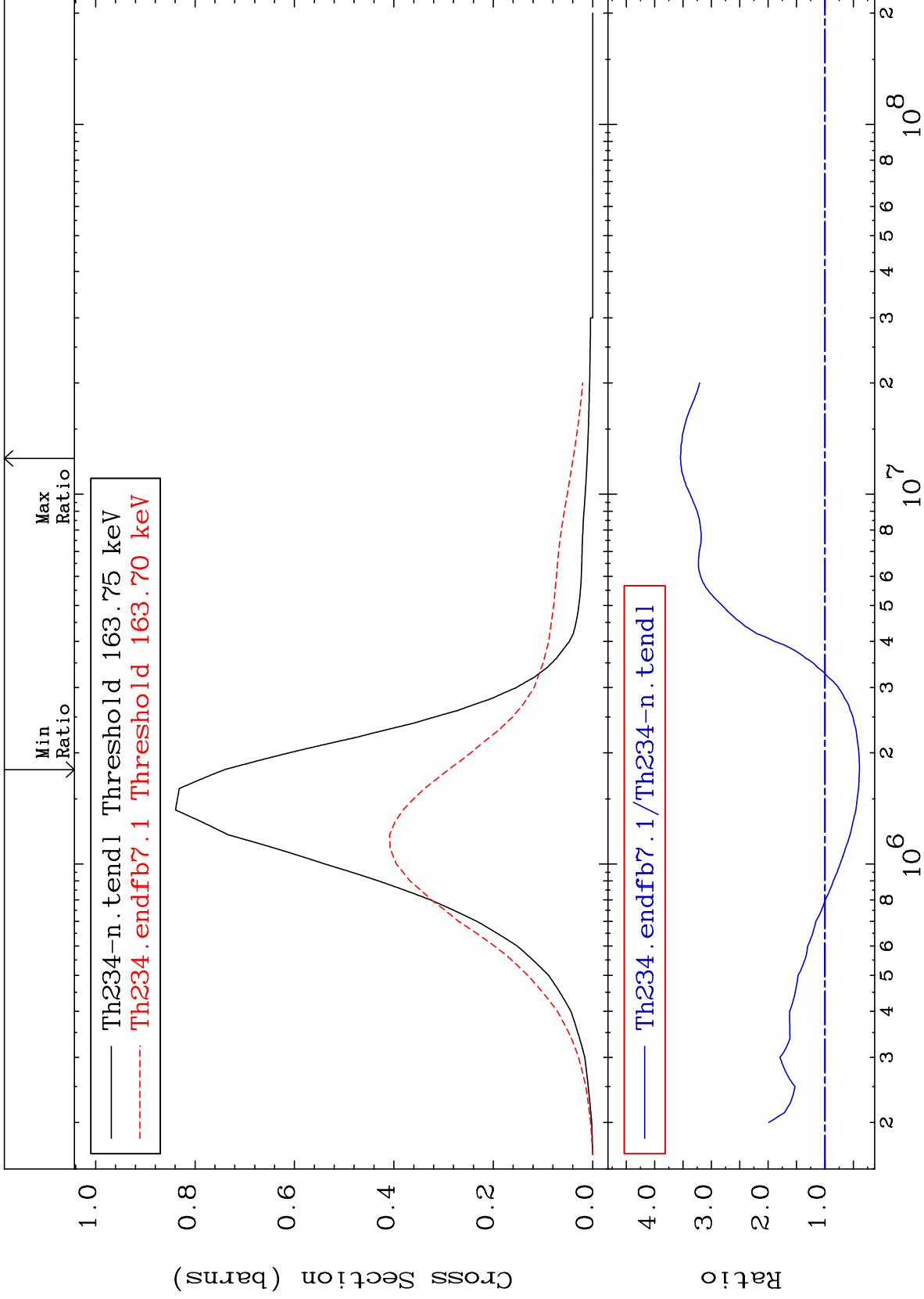
Incident Energy (eV)

90-Th-234

MAT 9046

163.1 keV (n,n') Level  
Cross Section

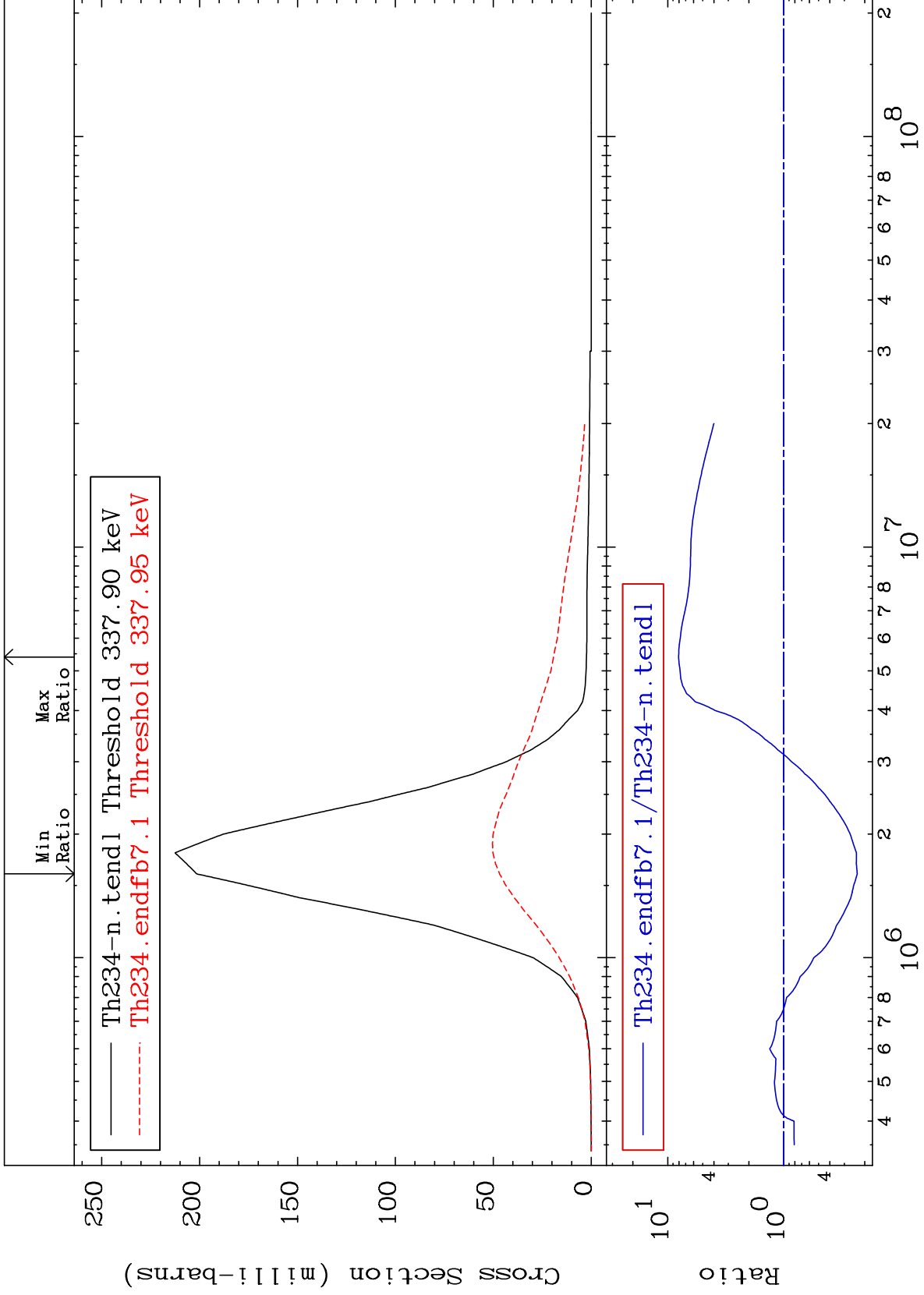
90-Th-234  
-60.80 To 254.7 %



MAT 9046

336.5 keV (n,n') Level  
Cross Section

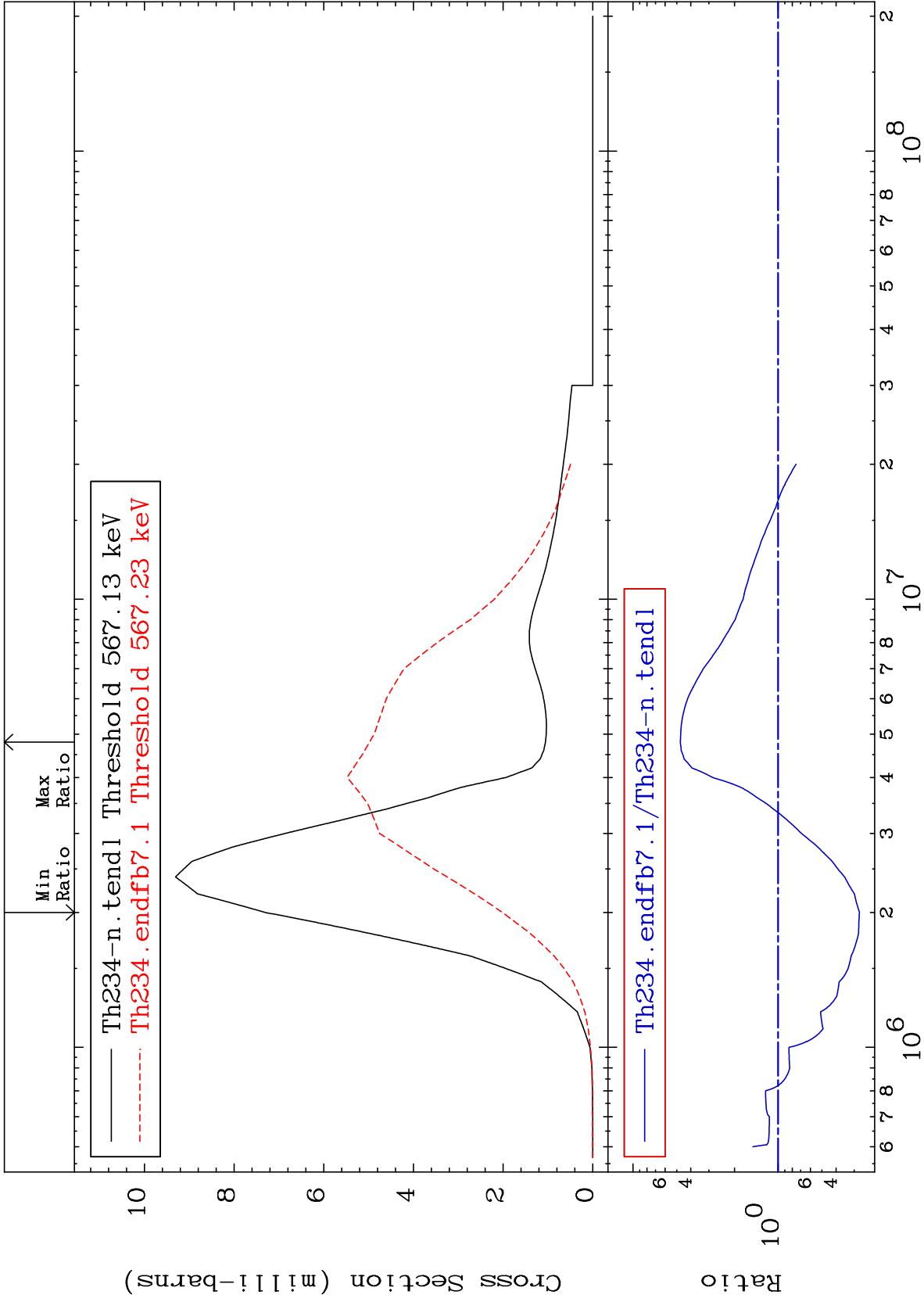
90-Th-234  
-76.79 To 705.4 %



MAT 9046

564.7 keV (n,n') Level  
Cross Section

90-Th-234  
-72.41 To 371.7 %



15

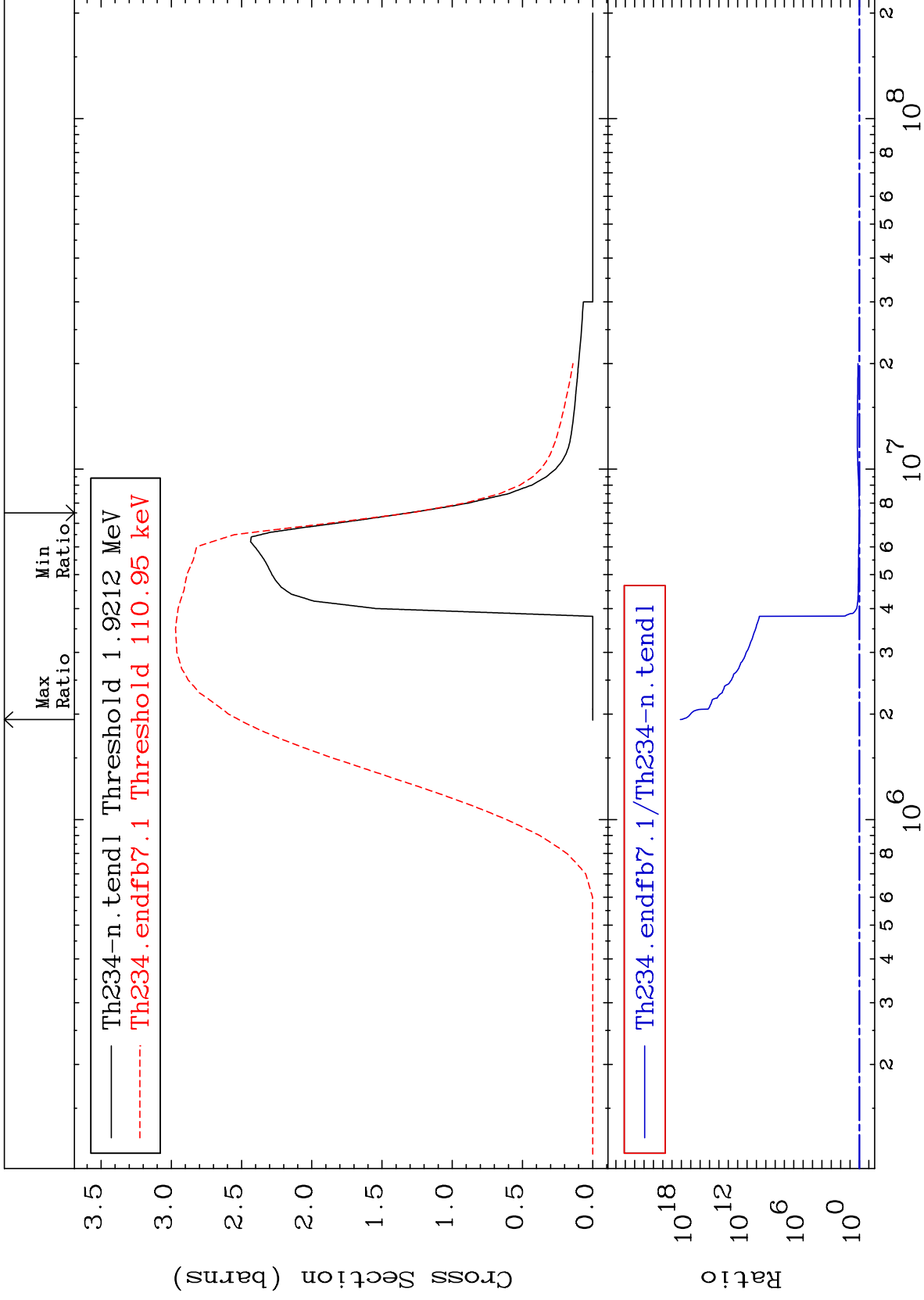
Incident Energy (eV)

90-Th-234

MAT 9046

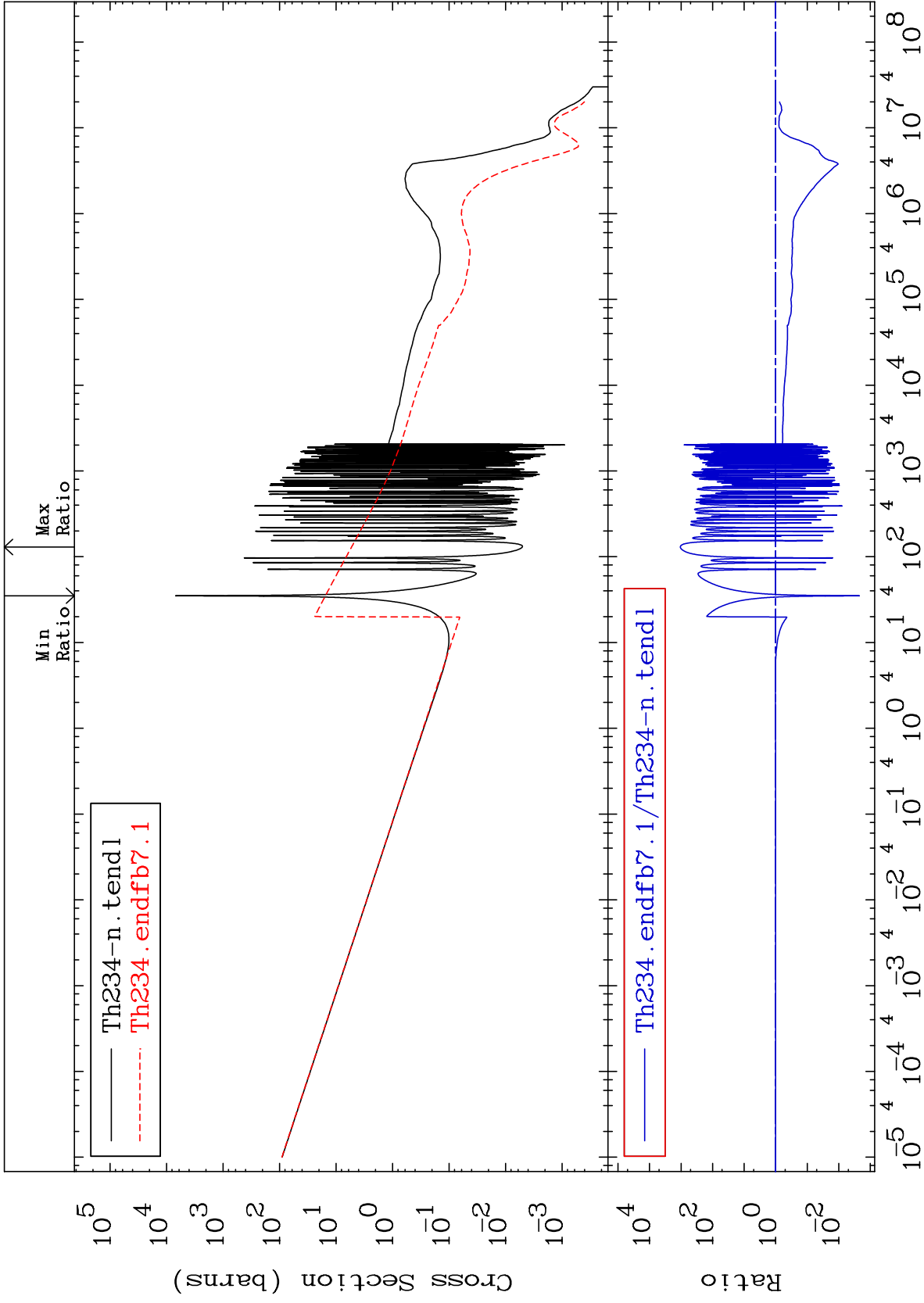
(n, n') Continuum  
Cross Section

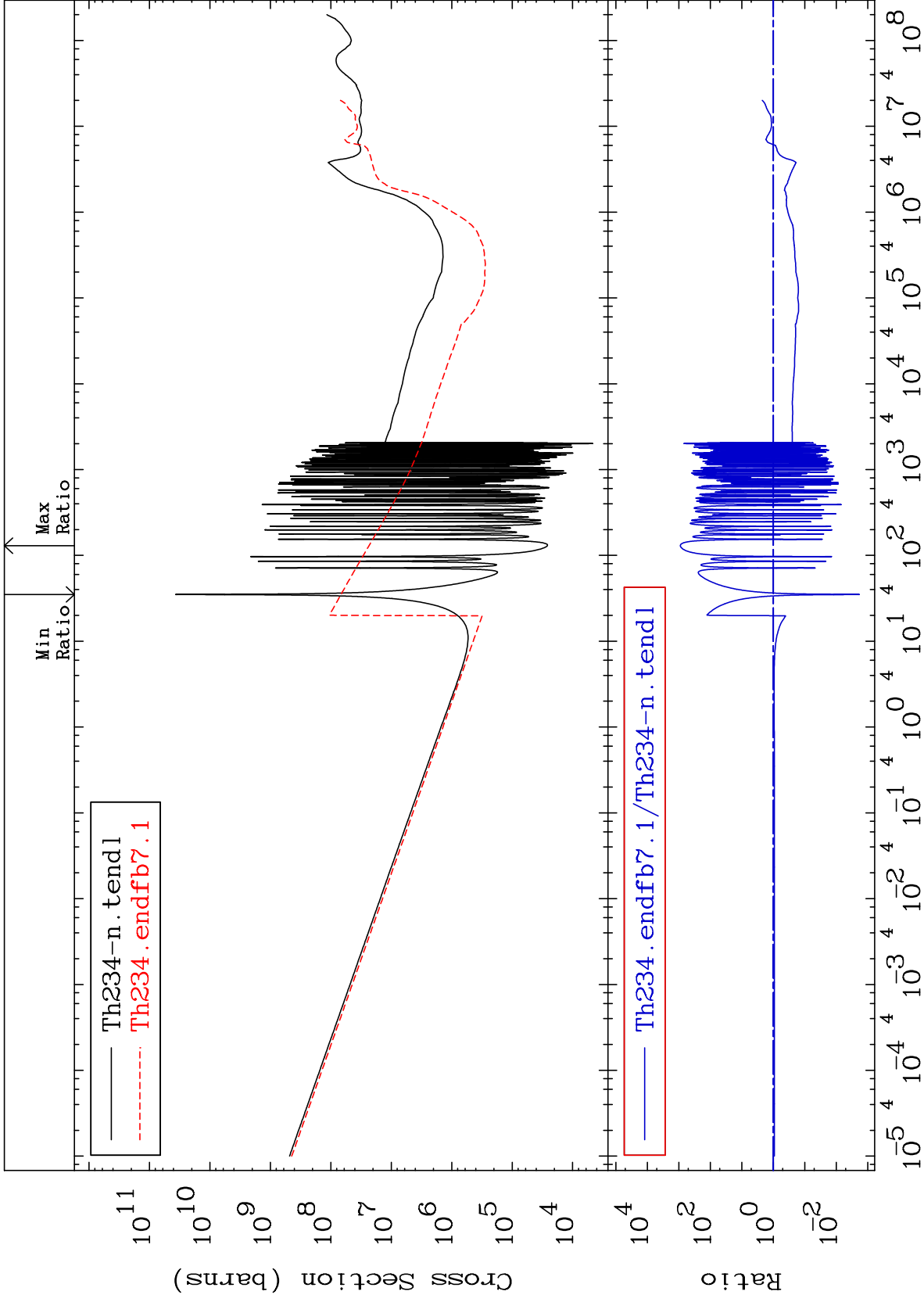
90-Th-234  
-1.474 To 9999. %



MAT 9046

(n,  $\gamma$ )  
Cross Section  
90-Th-234  
-99.78 To 9999. %



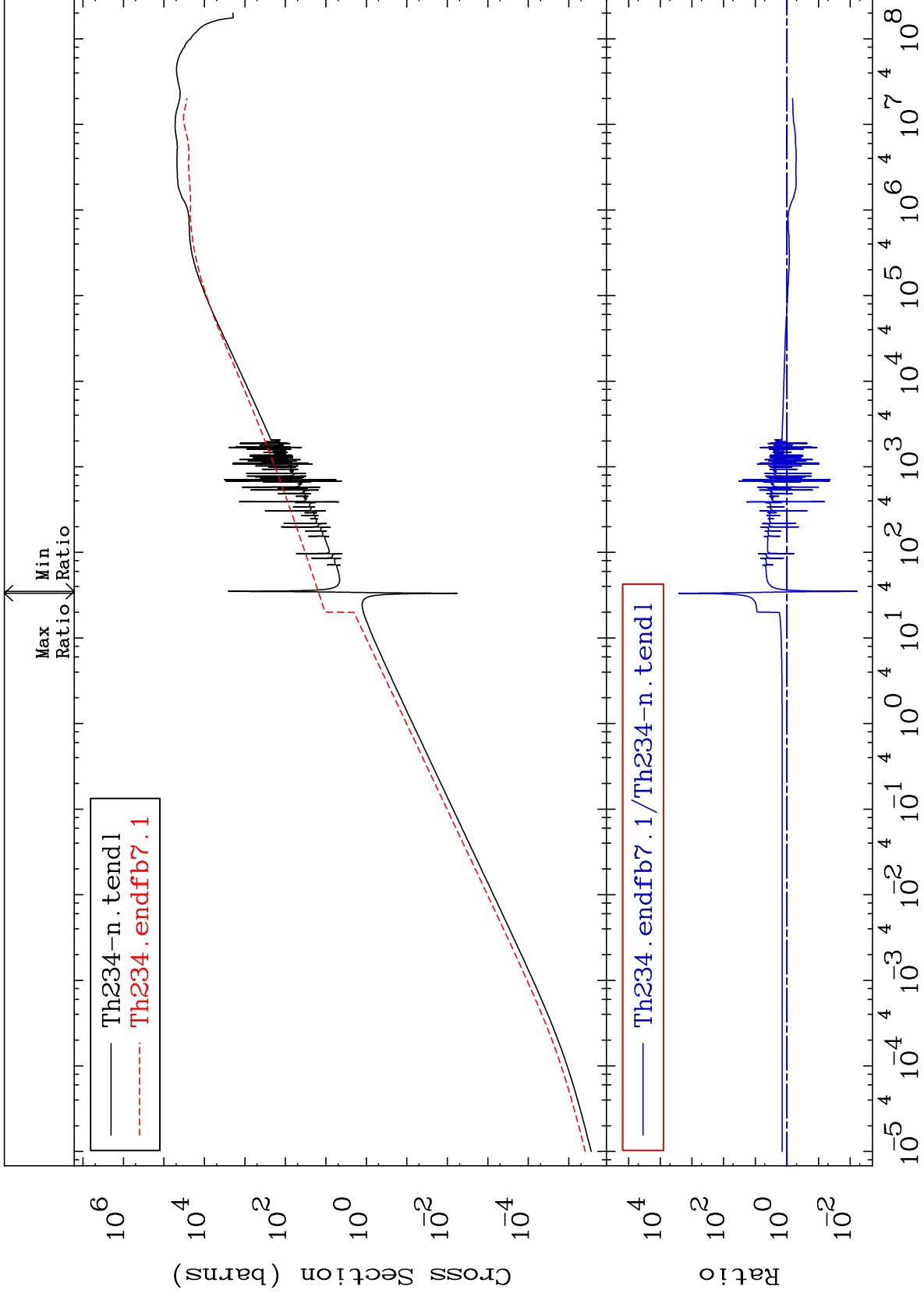


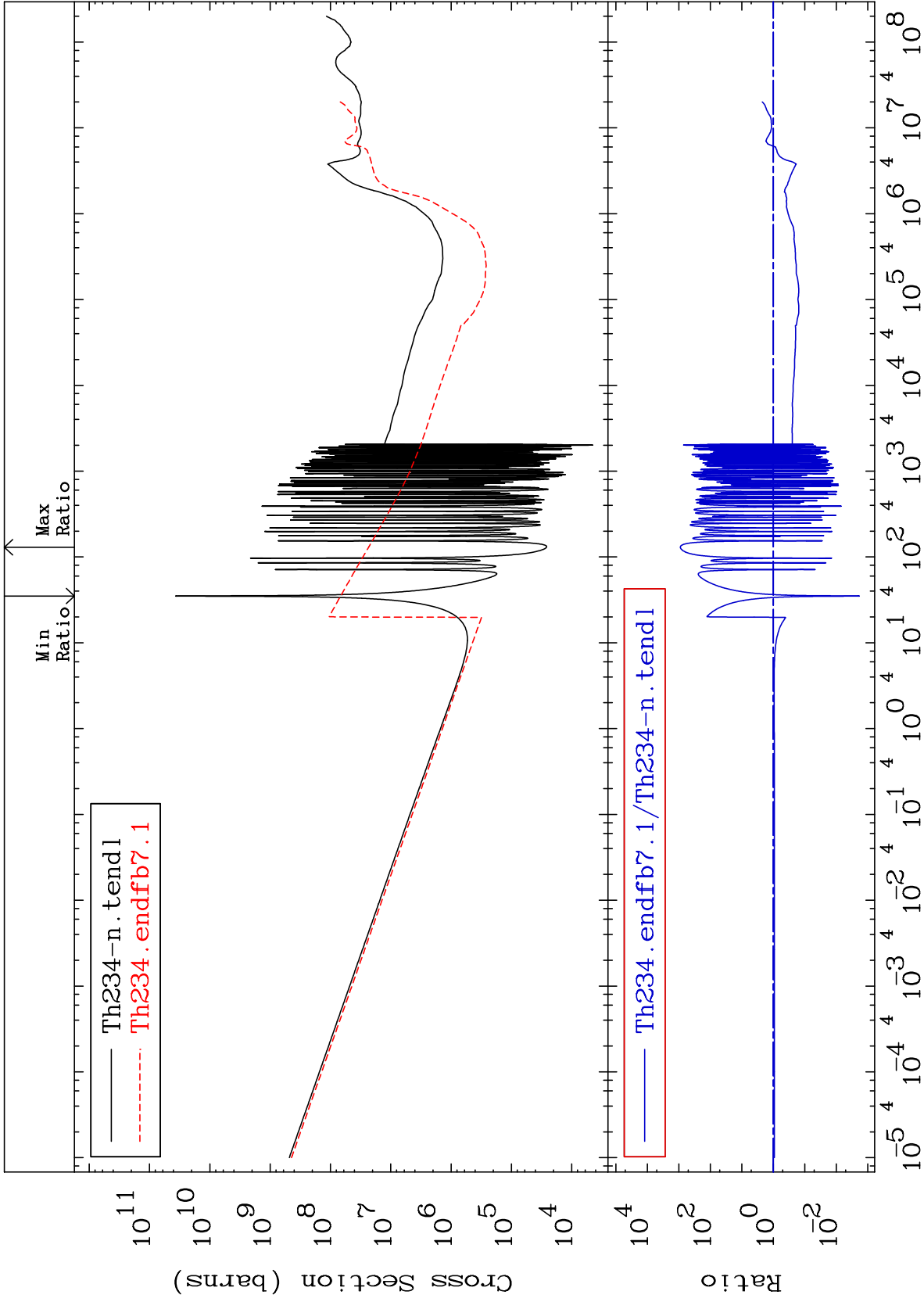


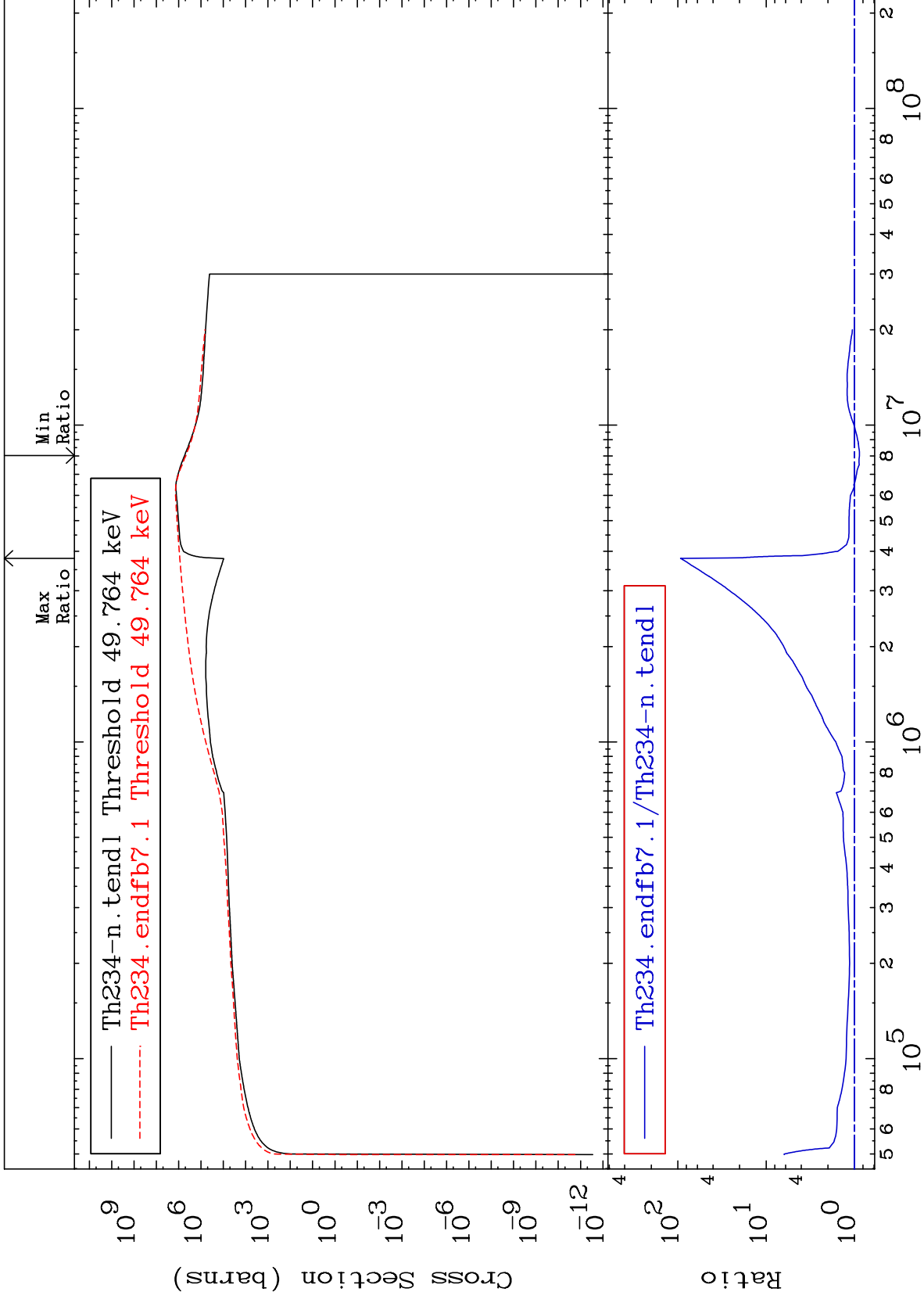
MAT 9046

Kerma elastic  
Cross Section

90-Th-234  
-99.40 To 9999. %



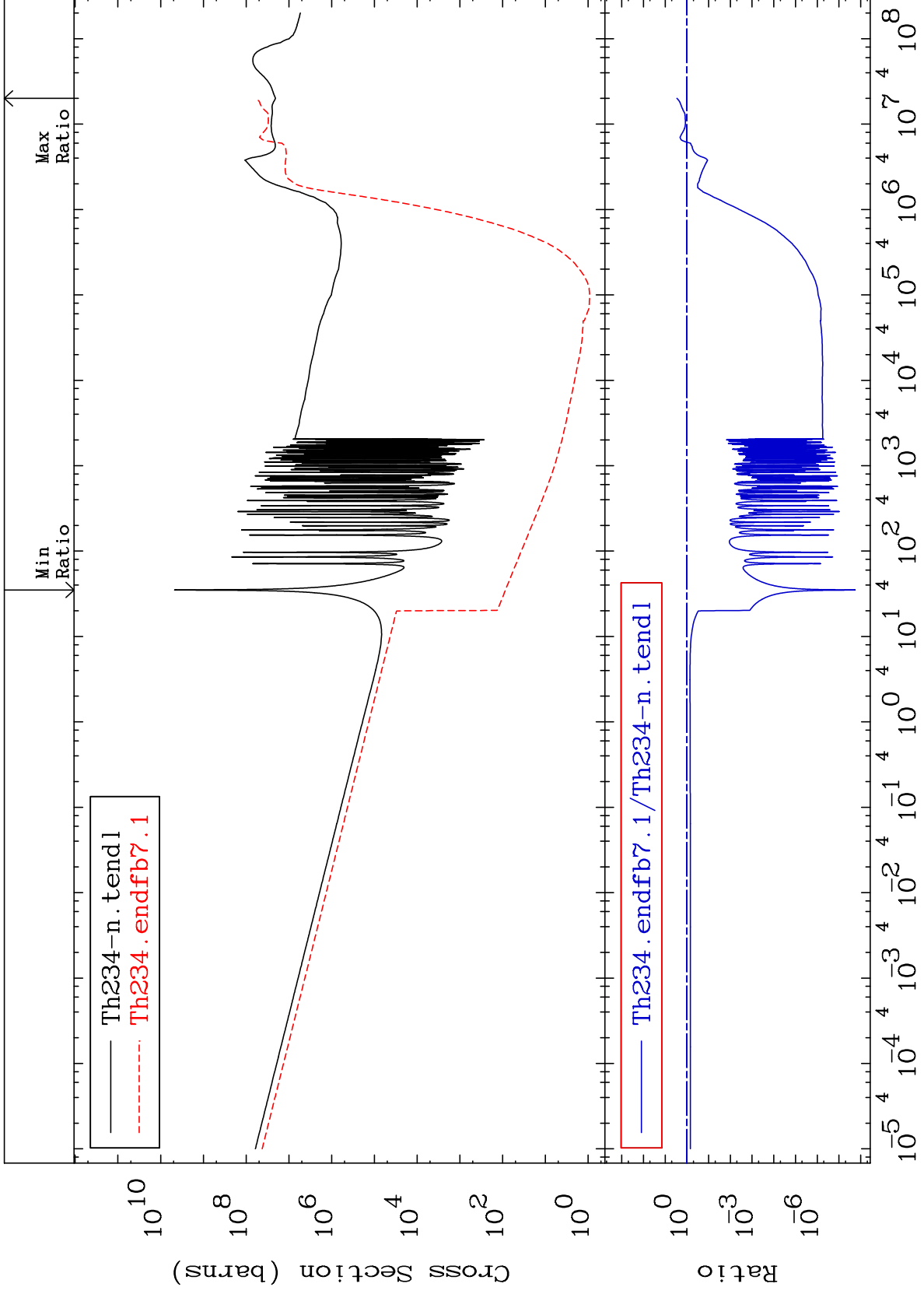


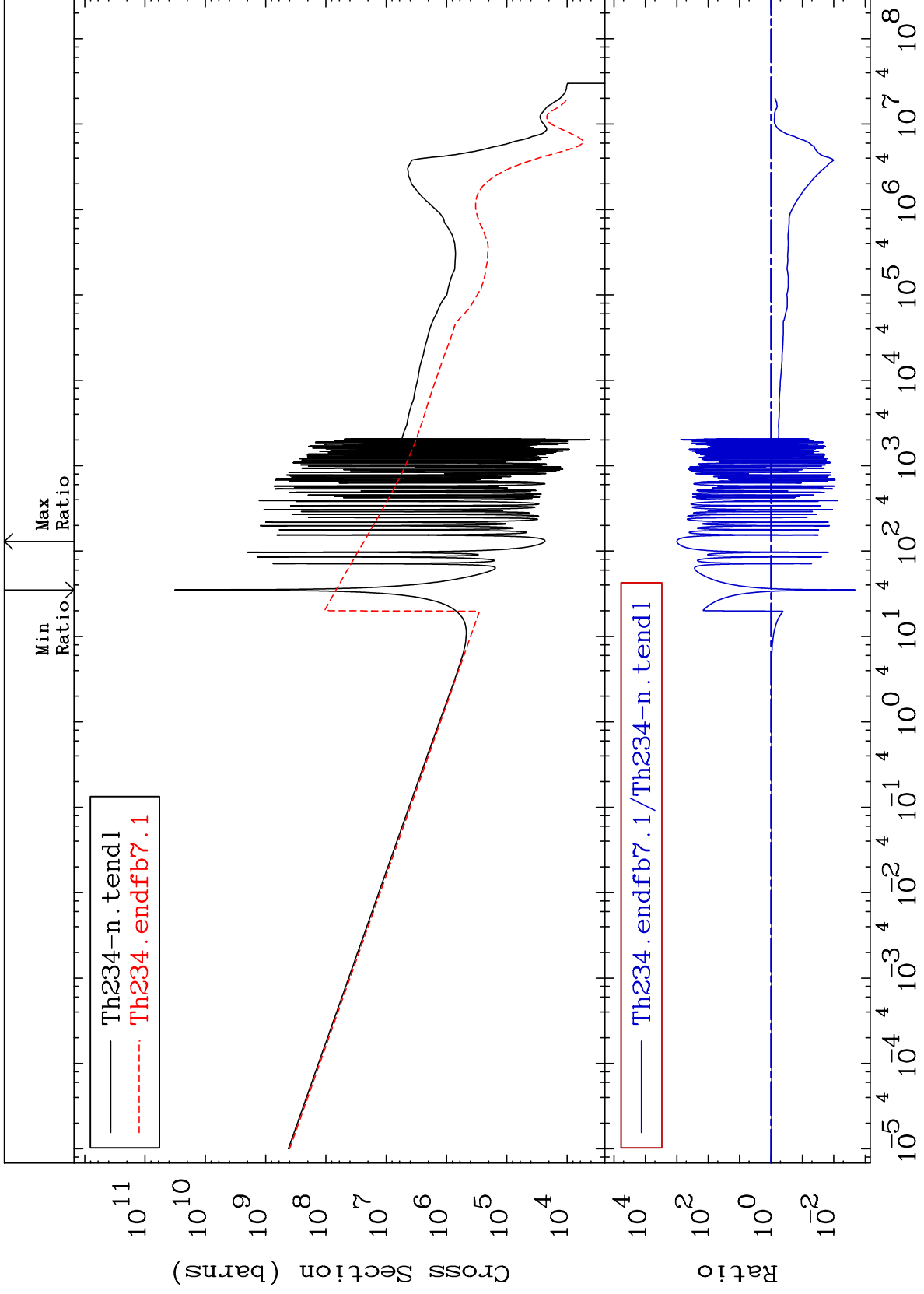


MAT 9046

Kerma fission (mt18 or mt19-20-21-38)  
Cross Section

90-Th-234  
-100.0 To 188.3 %





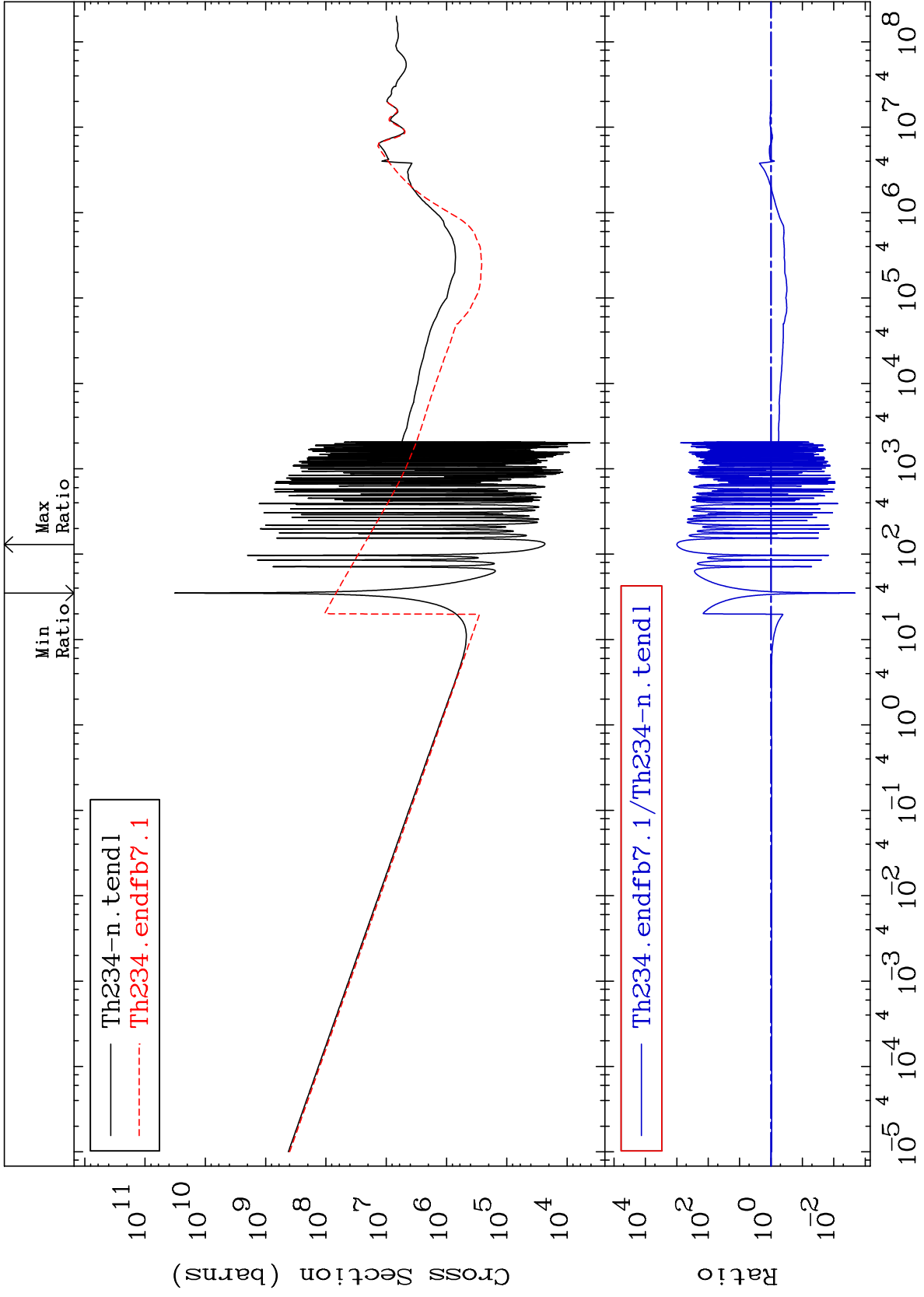
MAT 9046

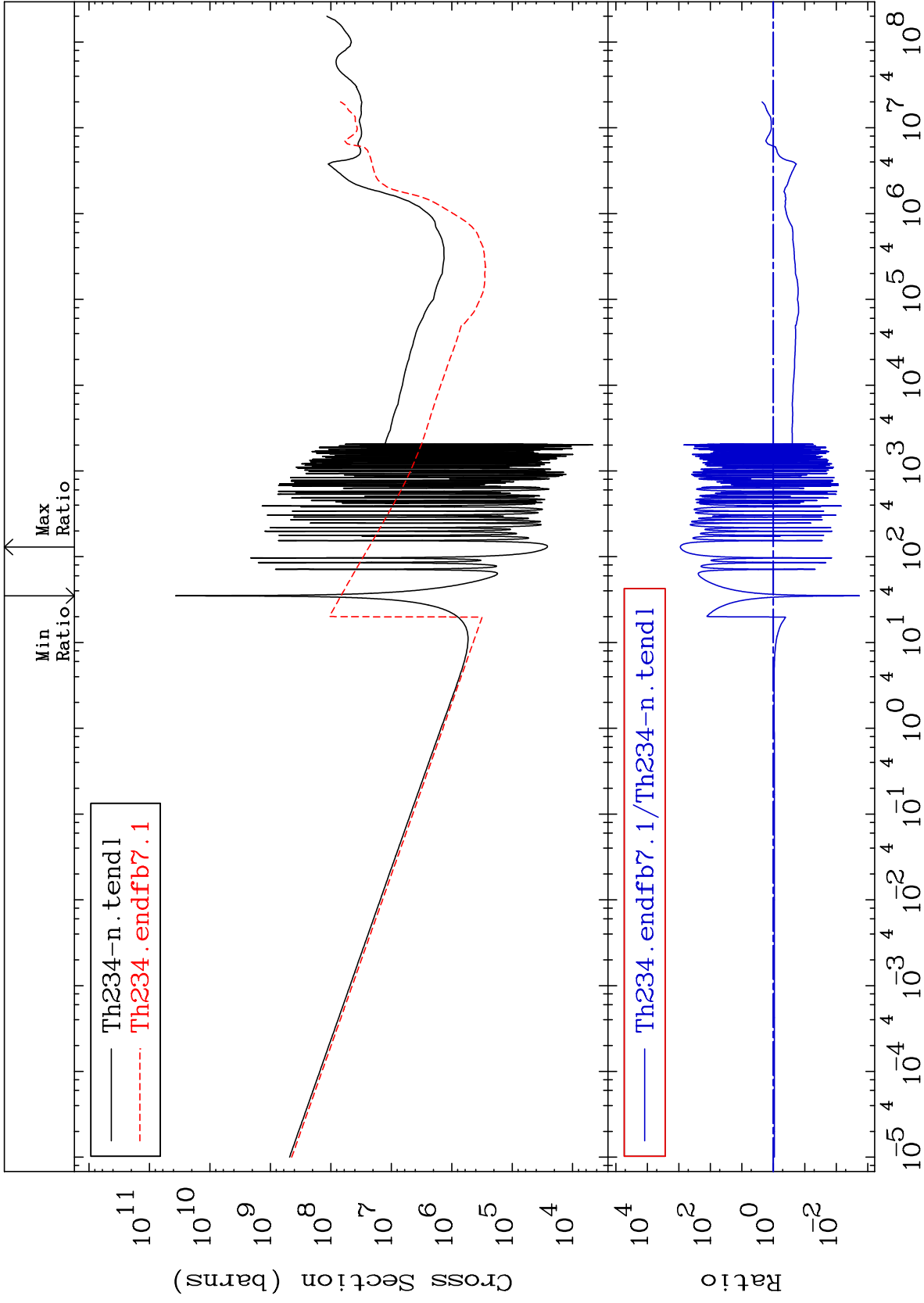
Total photon (eV-barns)

90-Th-234

Cross Section

-99.79 To 9999. %





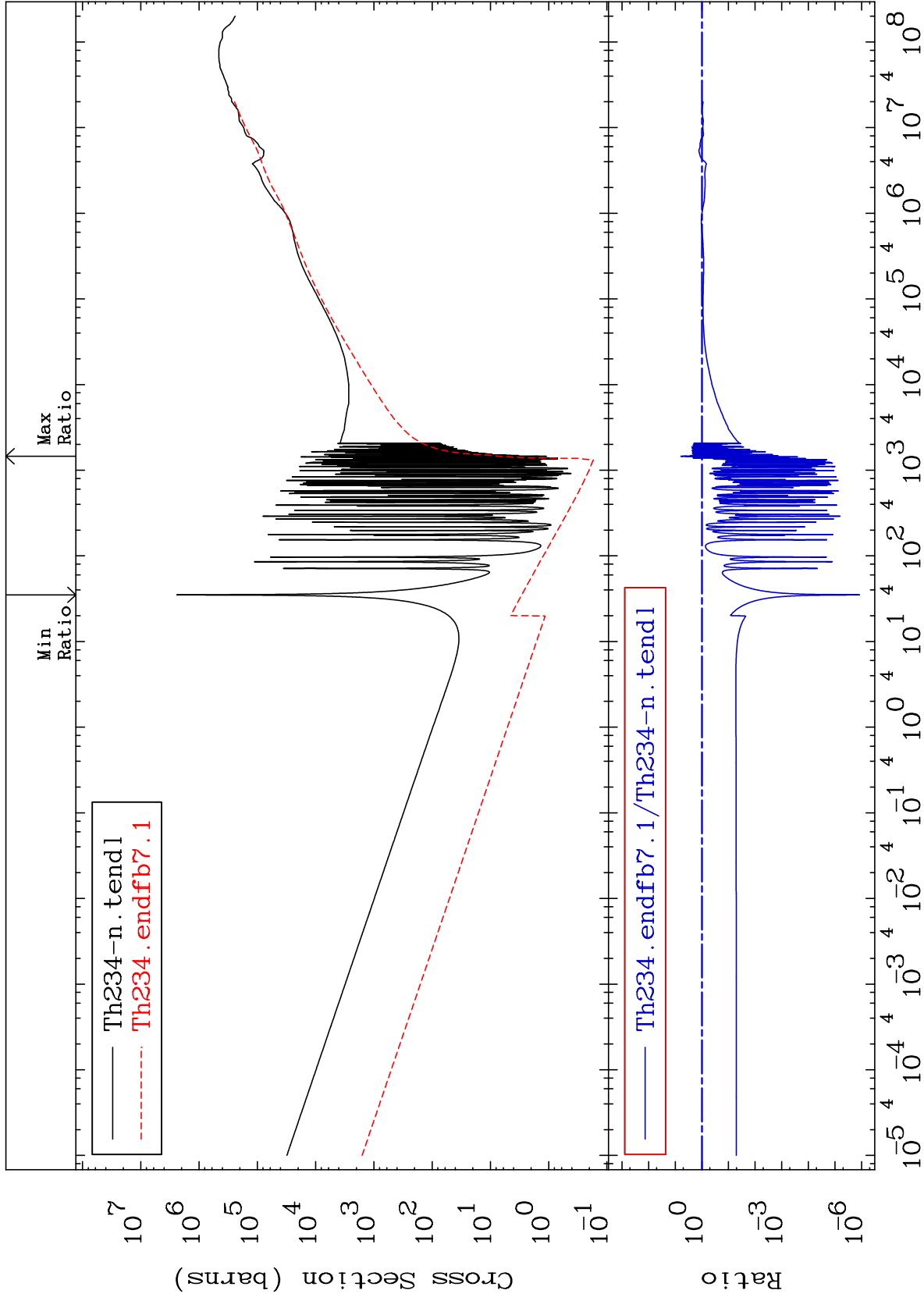
MAT 9046

Dpa total (eV-barns)

90-Th-234

Cross Section

-100.0 To 519.5 %



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Incident Energy (eV)

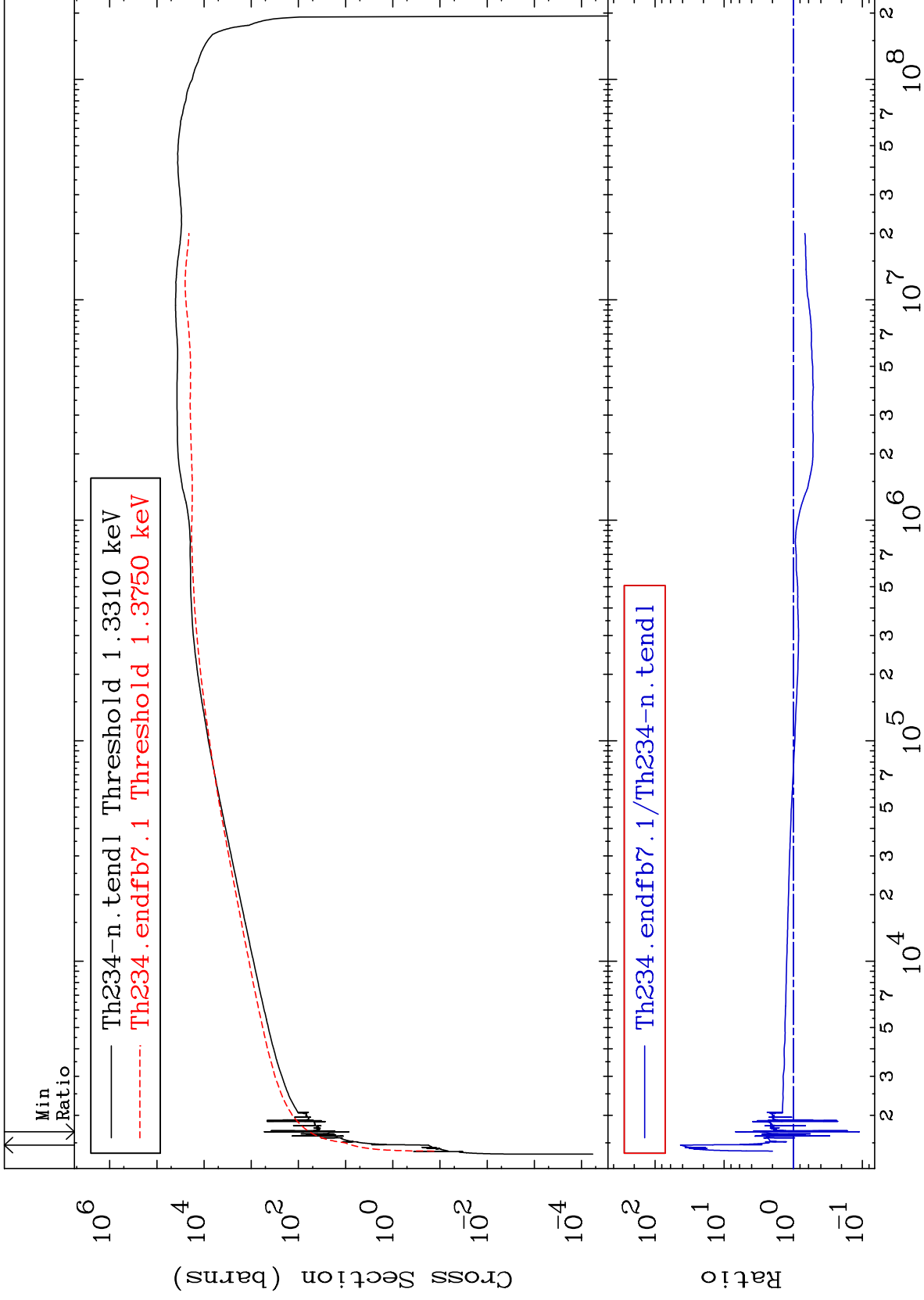
90-Th-234



MAT 9046

Dpa elastic (mt2)  
Cross Section

90-Th-234  
-88.97 To 4209. %



MAT 9046

Dpa inelastic (mt51-91)  
Cross Section

90-Th-234  
-99.84 To 1790. %

