

Program EVALPLOT  
(Version 2015-2)

by

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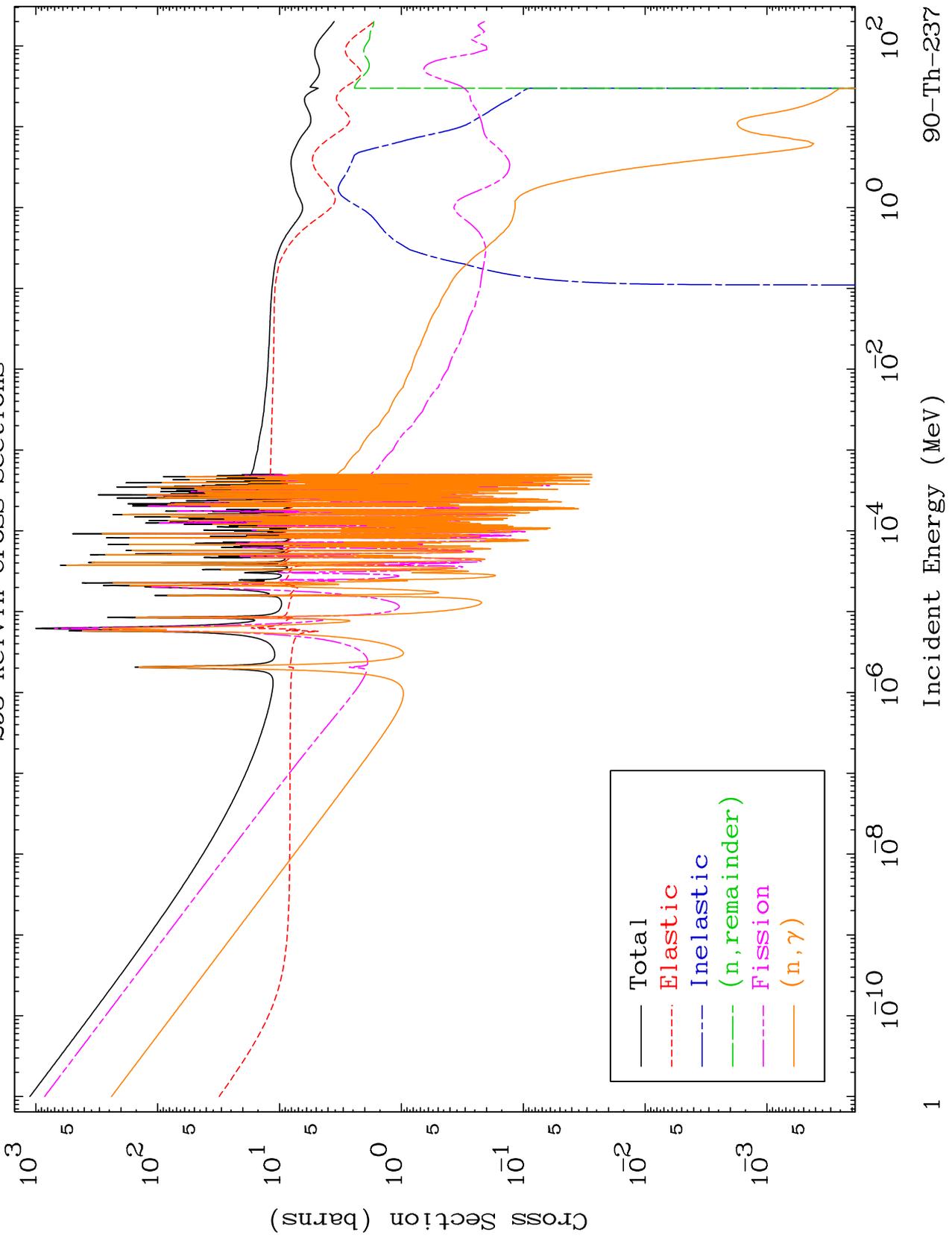
Press Mouse Button to Start

MAT 9055

Major

293 Kelvin Cross Sections

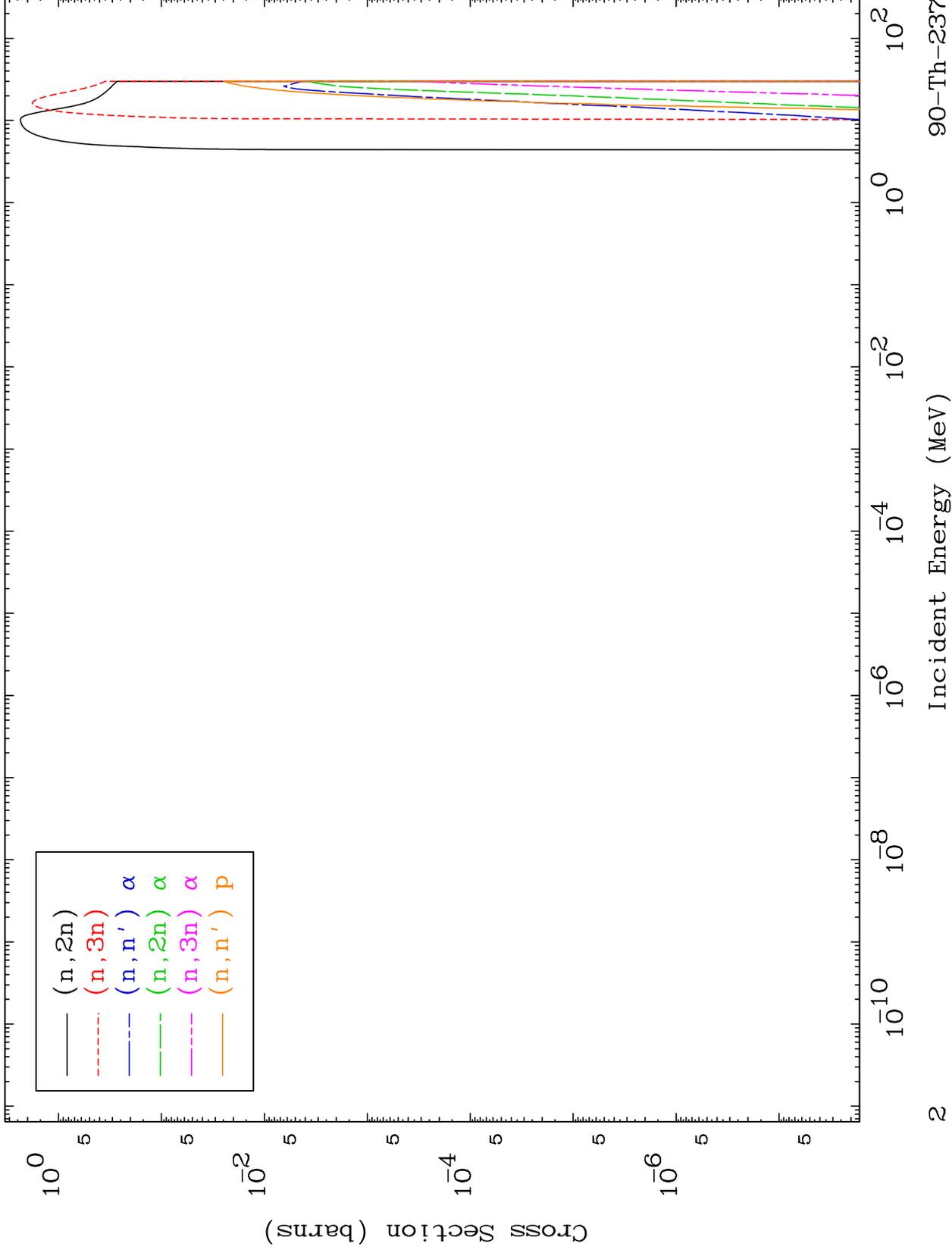
90-Th-237



MAT 90555

Neutron Production  
293 Kelvin Cross Sections

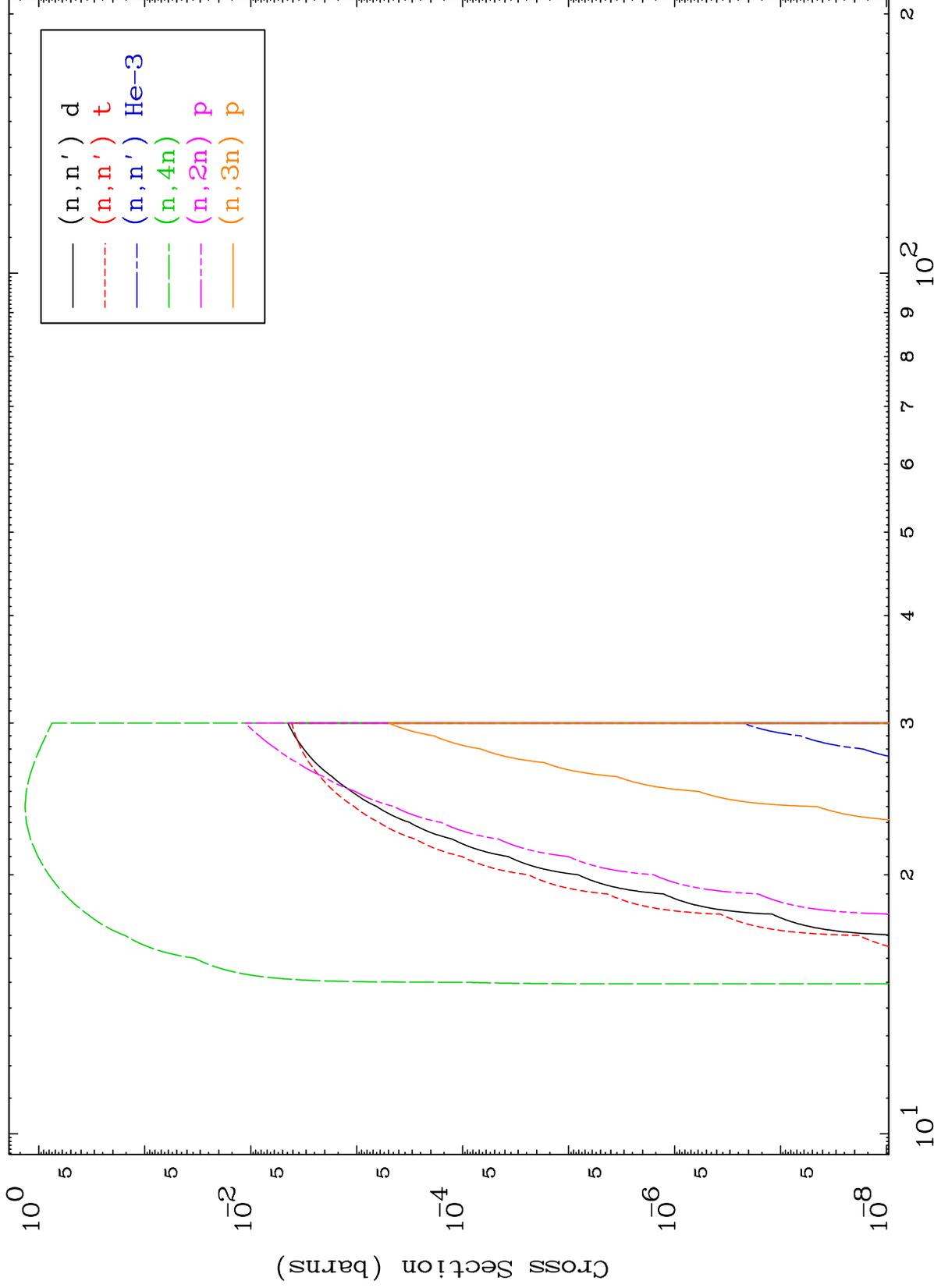
90-Th-237



MAT 9055

Neutron Production  
293 Kelvin Cross Sections

90-Th-237



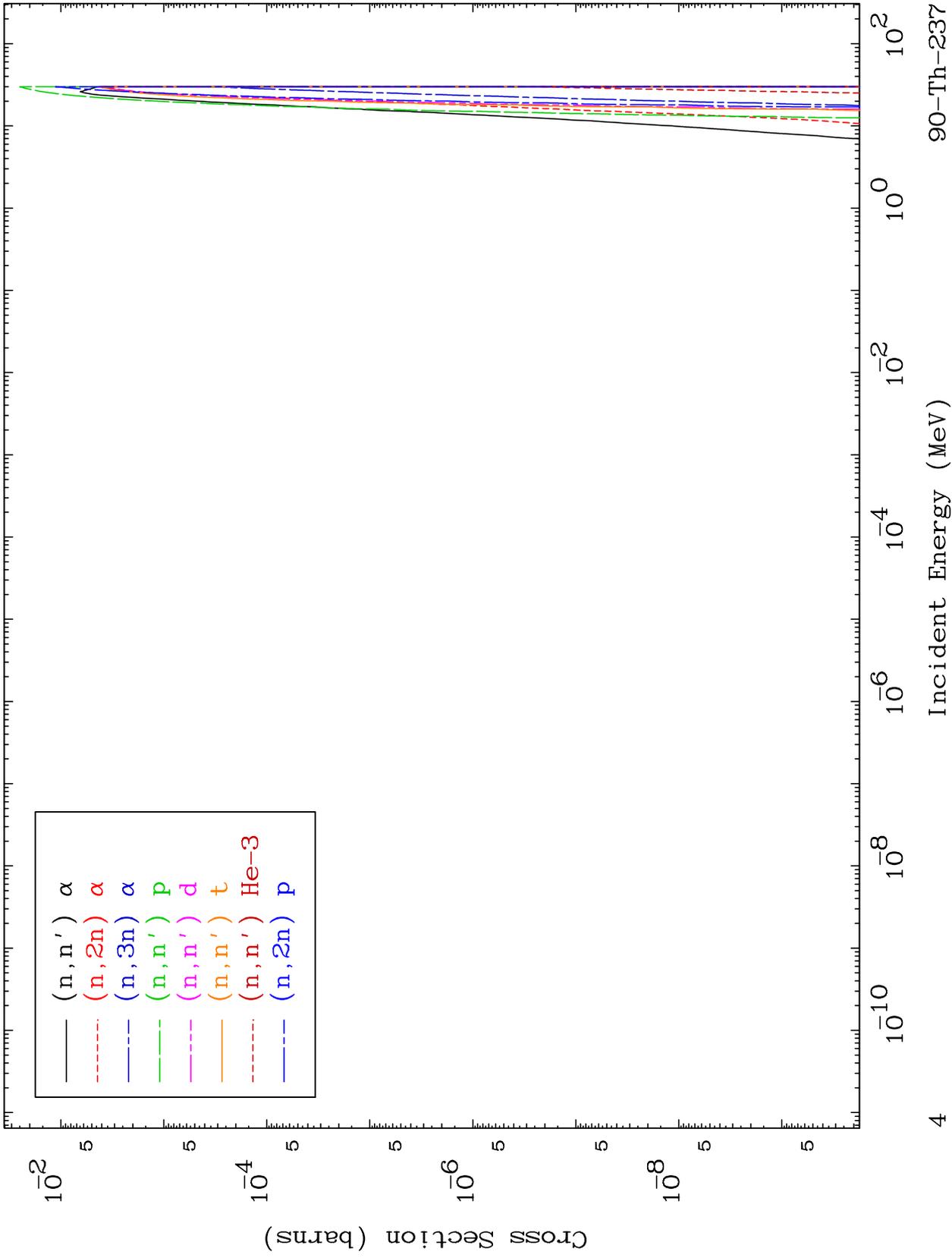
Incident Energy (MeV)

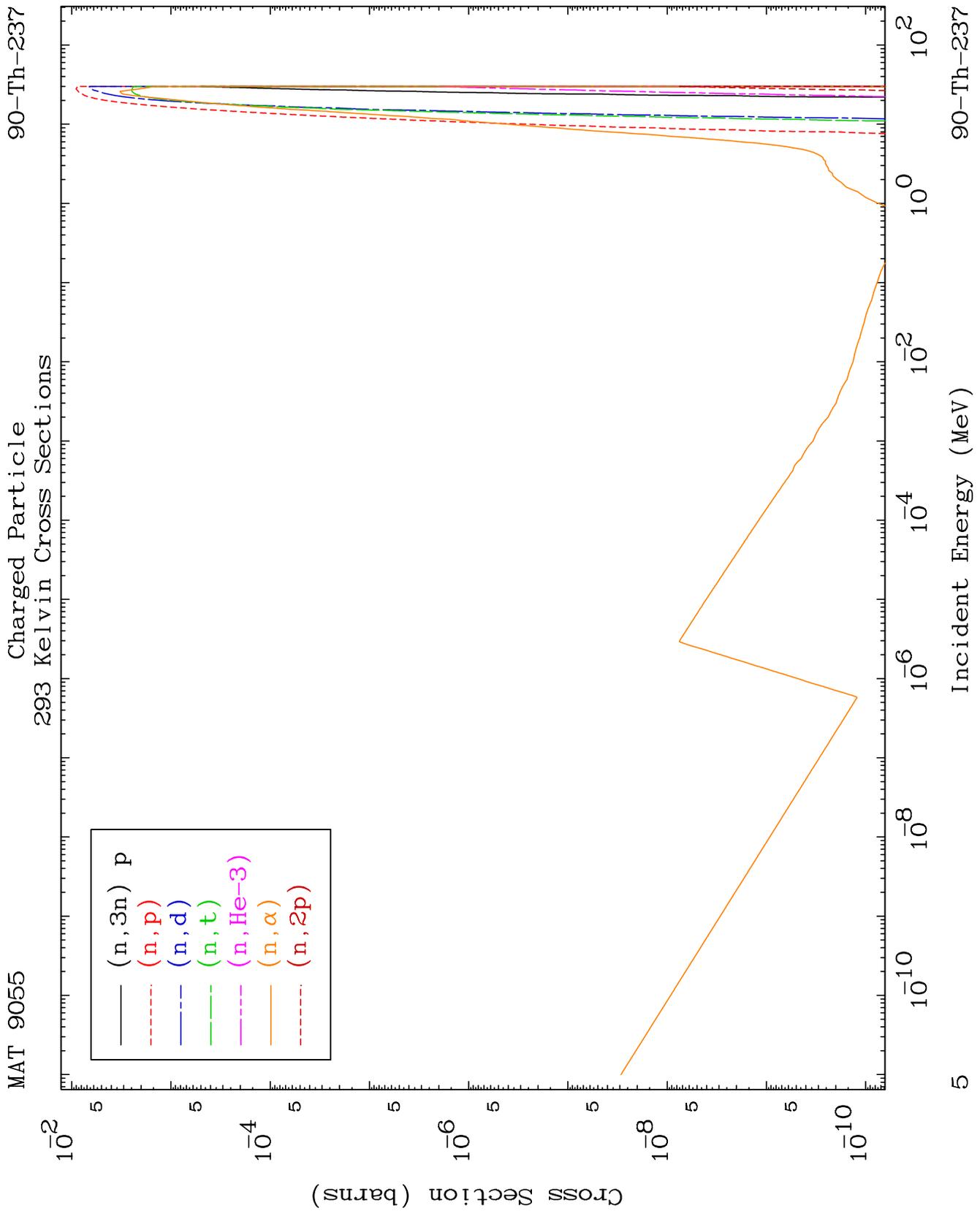
90-Th-237

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Charged Particle  
293 Kelvin Cross Sections

90-Th-237

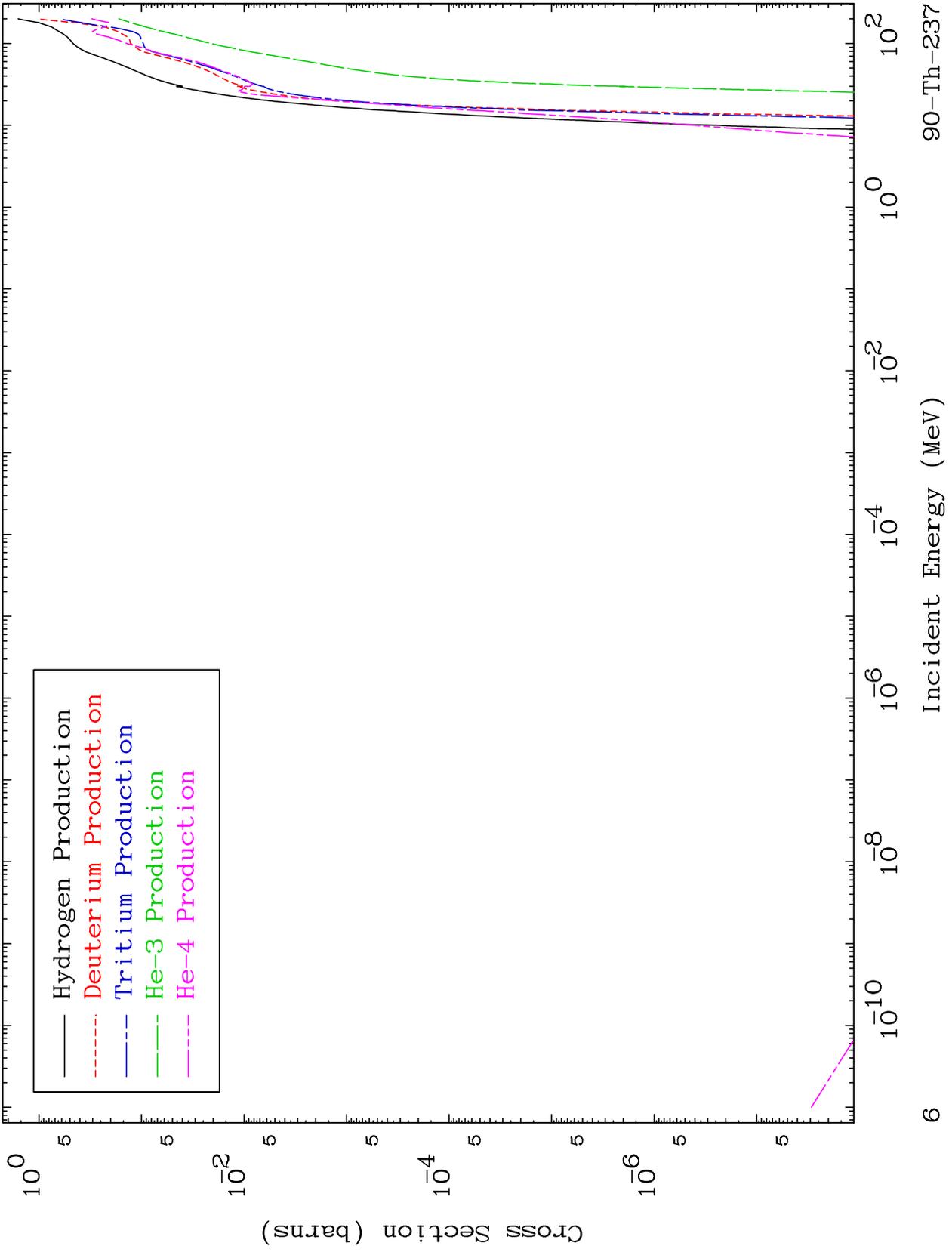




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Particle Production  
293 Kelvin Cross Sections

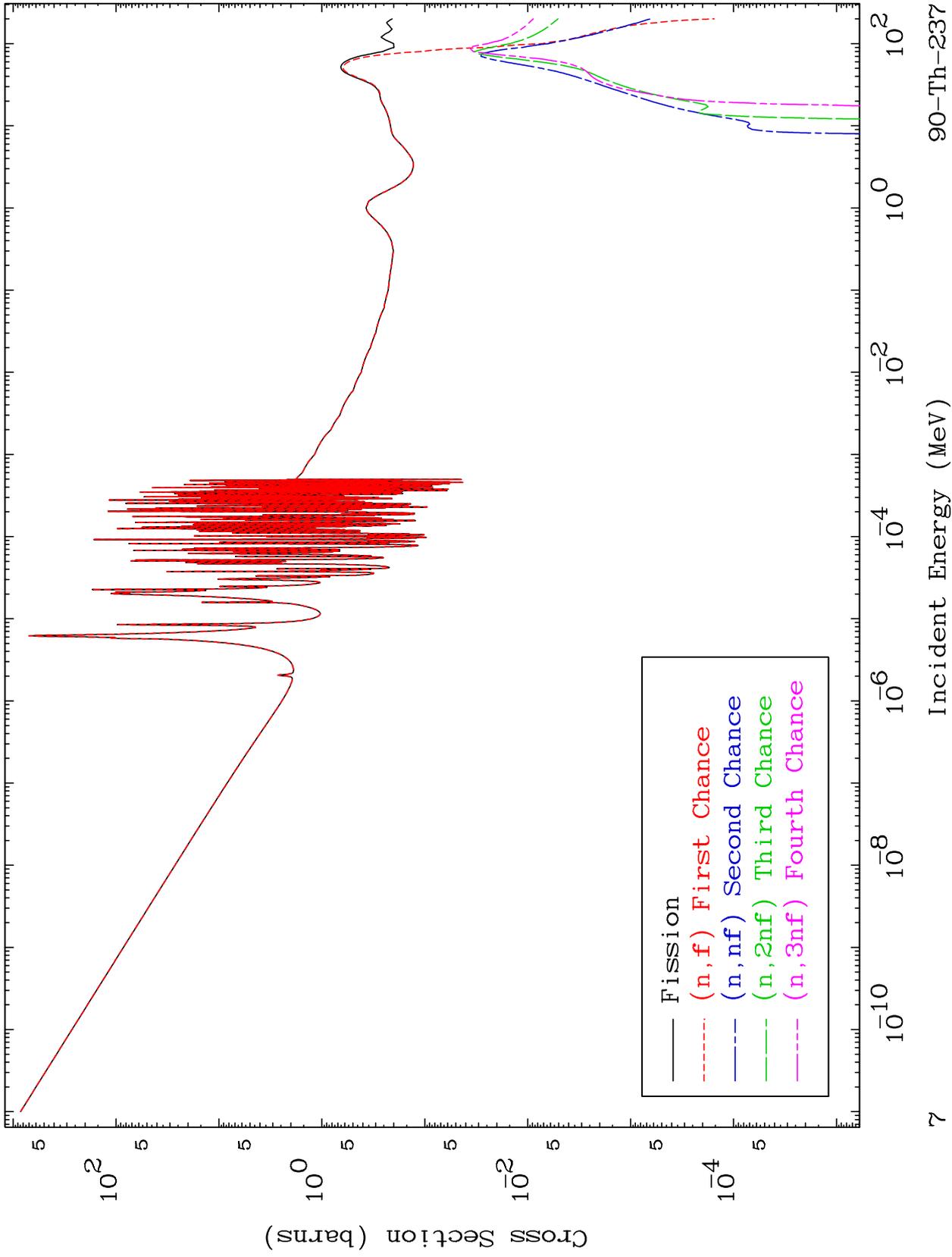
90-Th-237



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Fission  
293 Kelvin Cross Sections

90-Th-237



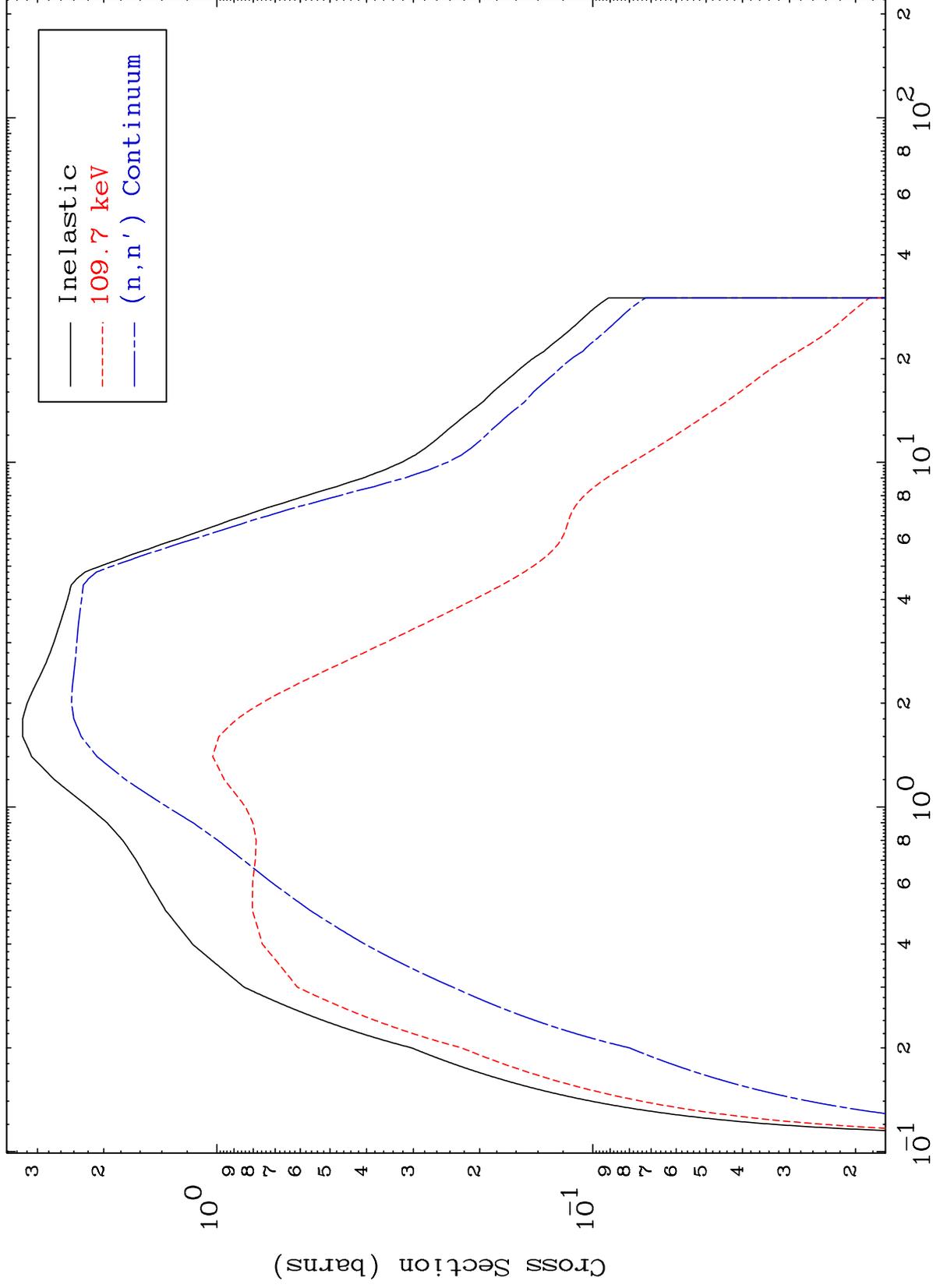


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(n,n') Level

90-Th-237

293 Kelvin Cross Sections



Incident Energy (MeV)

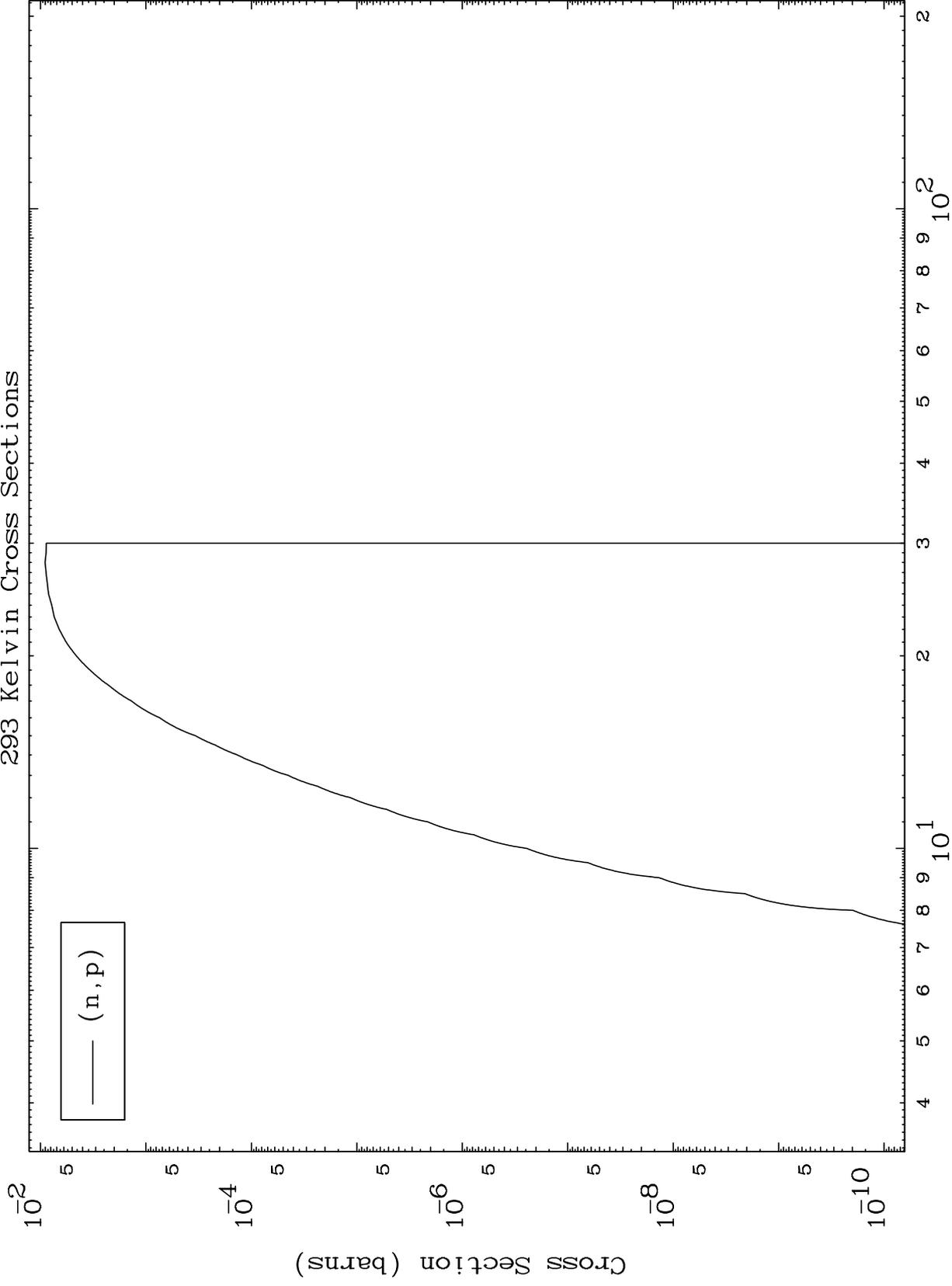
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(n,p) Levels  
293 Kelvin Cross Sections

90-Th-237



9

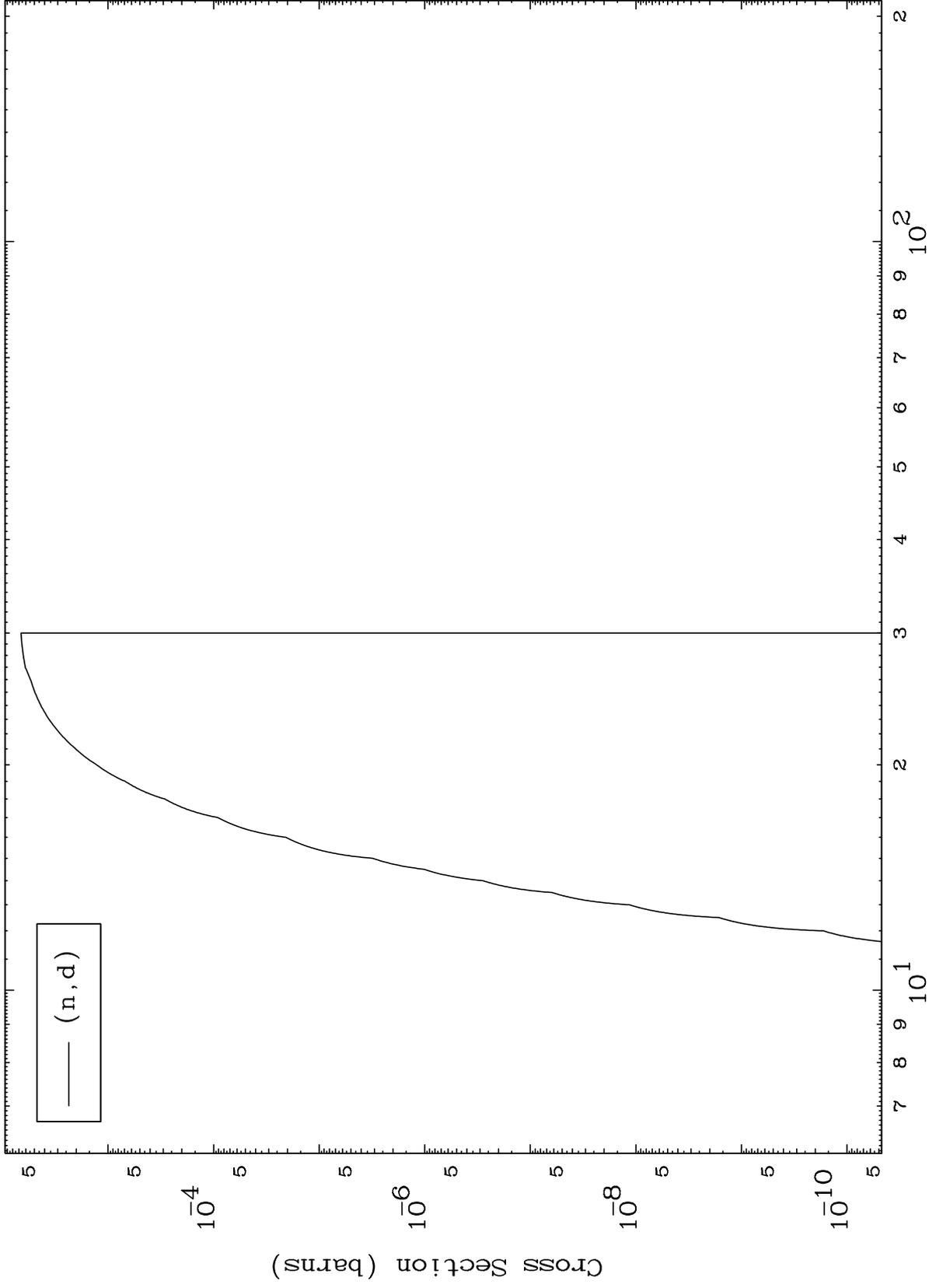
Incident Energy (MeV)

90-Th-237

MAT 9055

(n,d) Levels  
293 Kelvin Cross Sections

90-Th-237



10

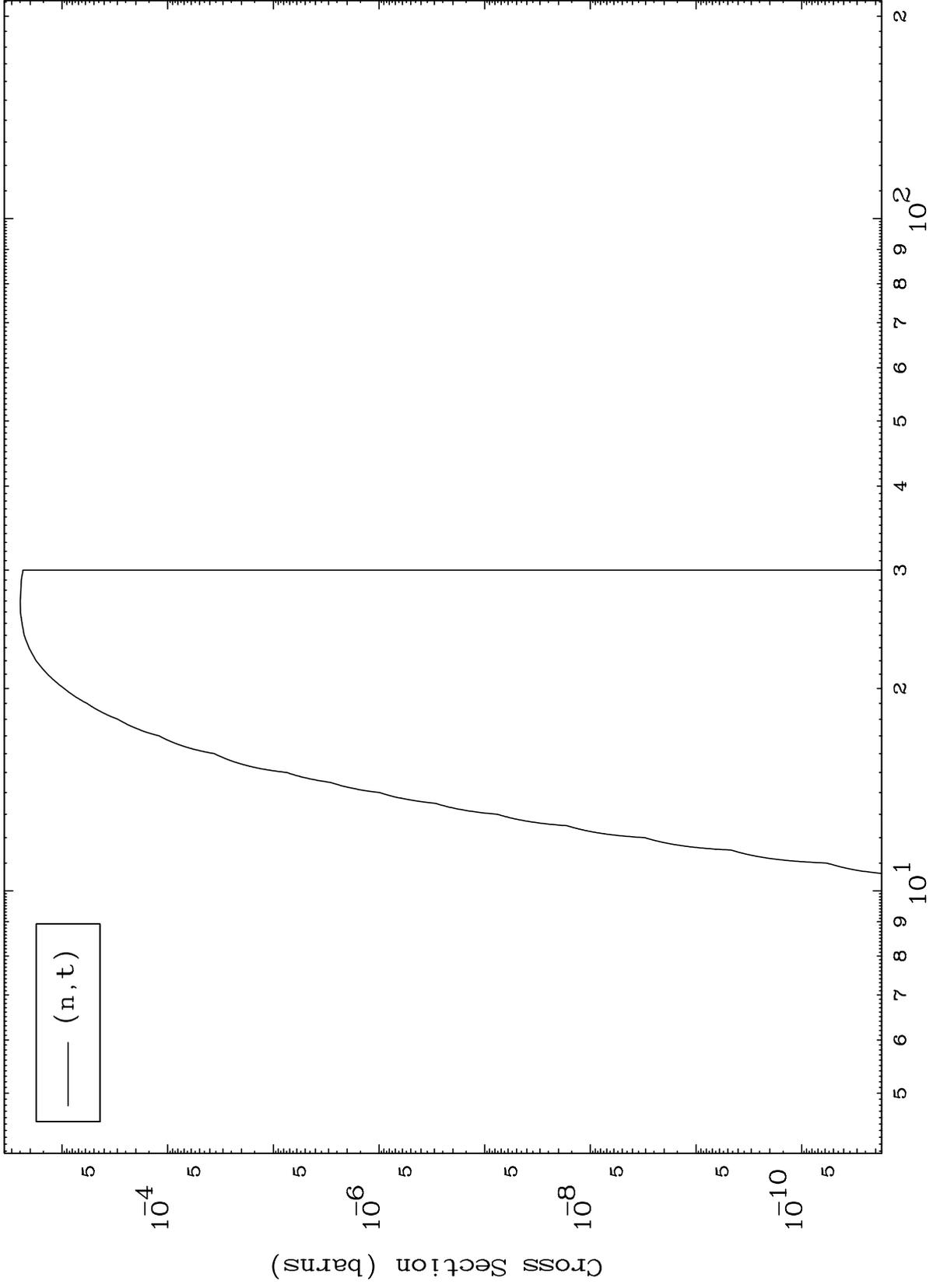
Incident Energy (MeV)

90-Th-237

MAT 9055

(n,t) Levels  
293 Kelvin Cross Sections

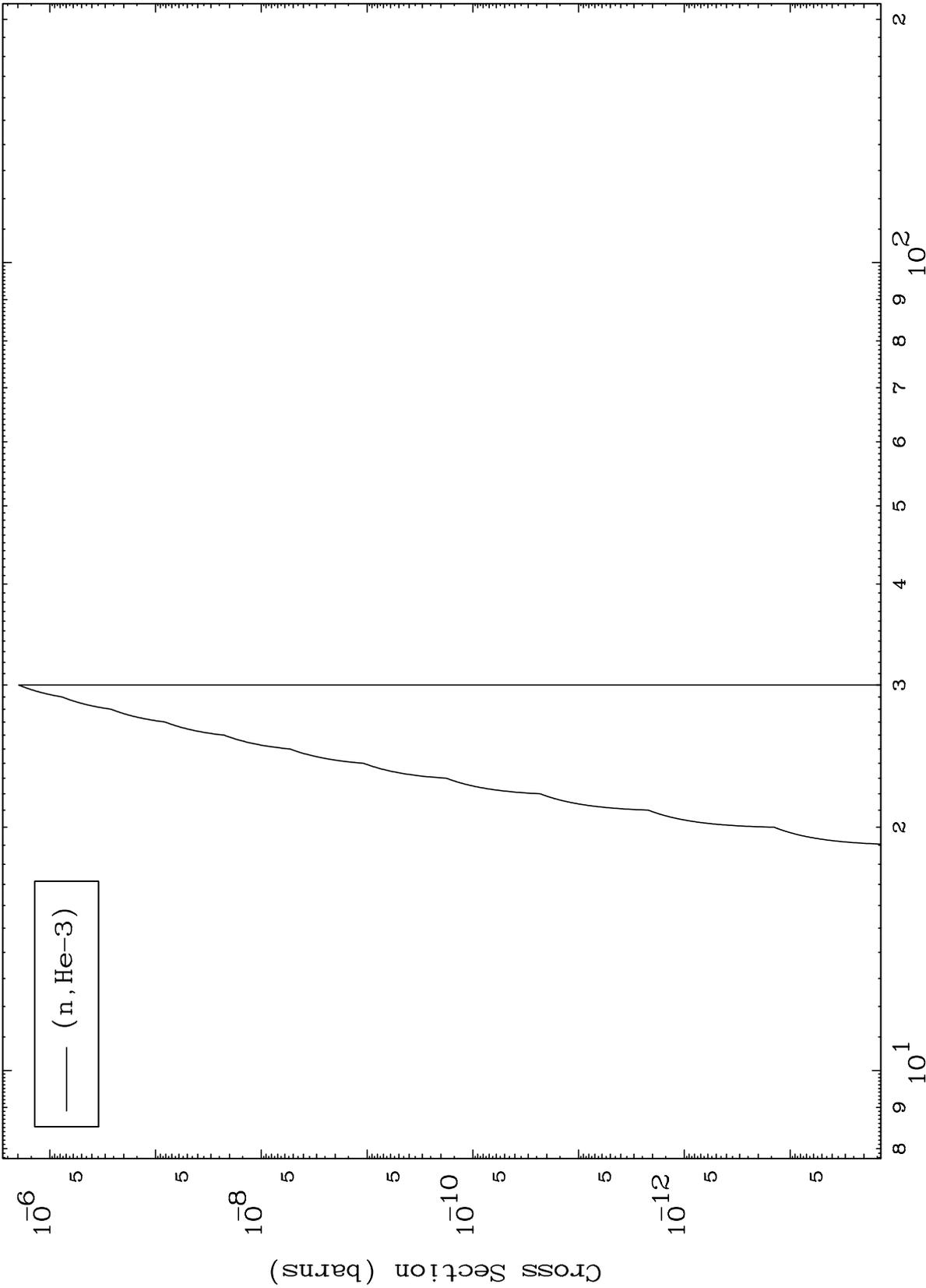
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(n,He3) Levels  
293 Kelvin Cross Sections

90-Th-237



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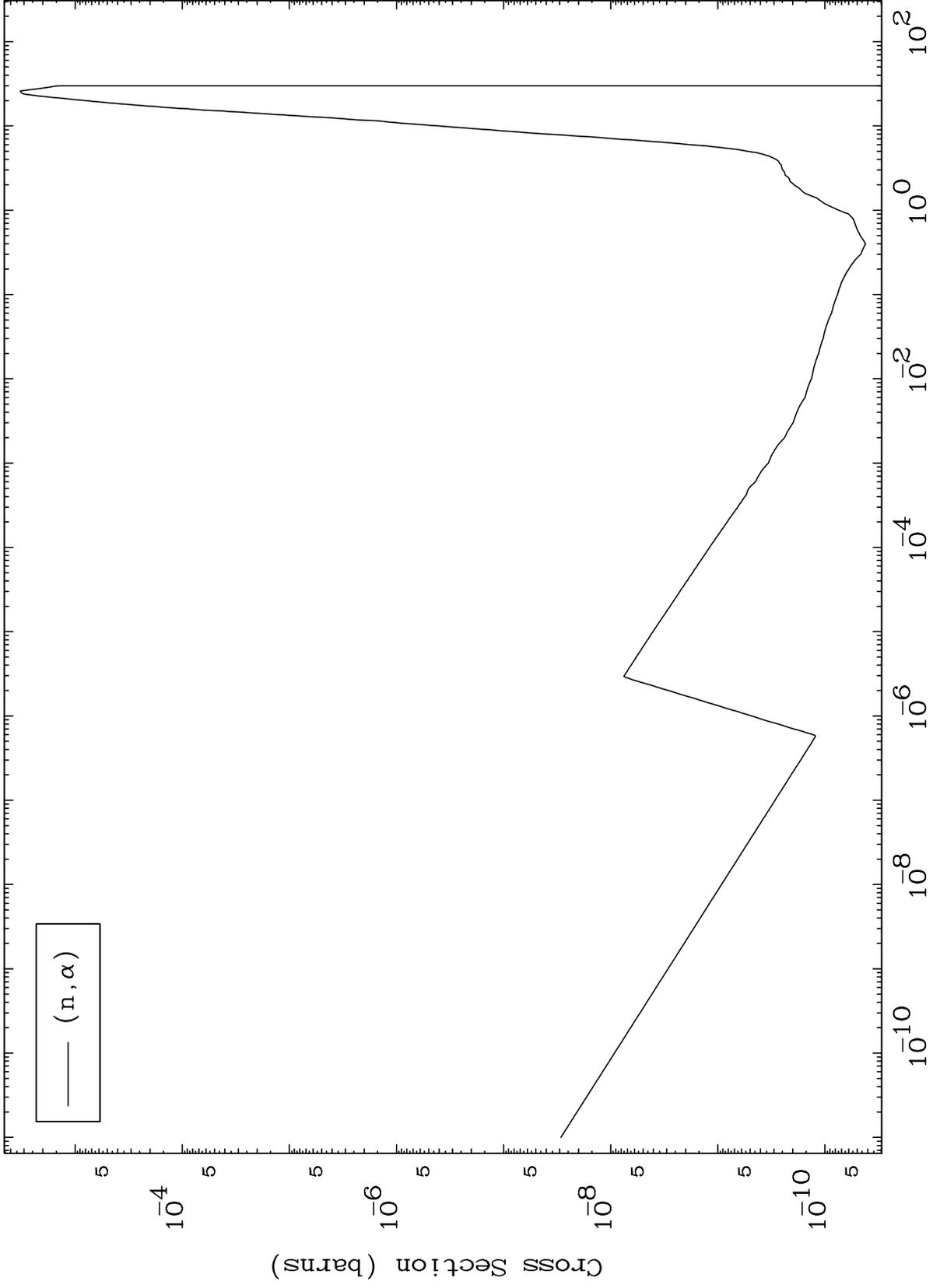
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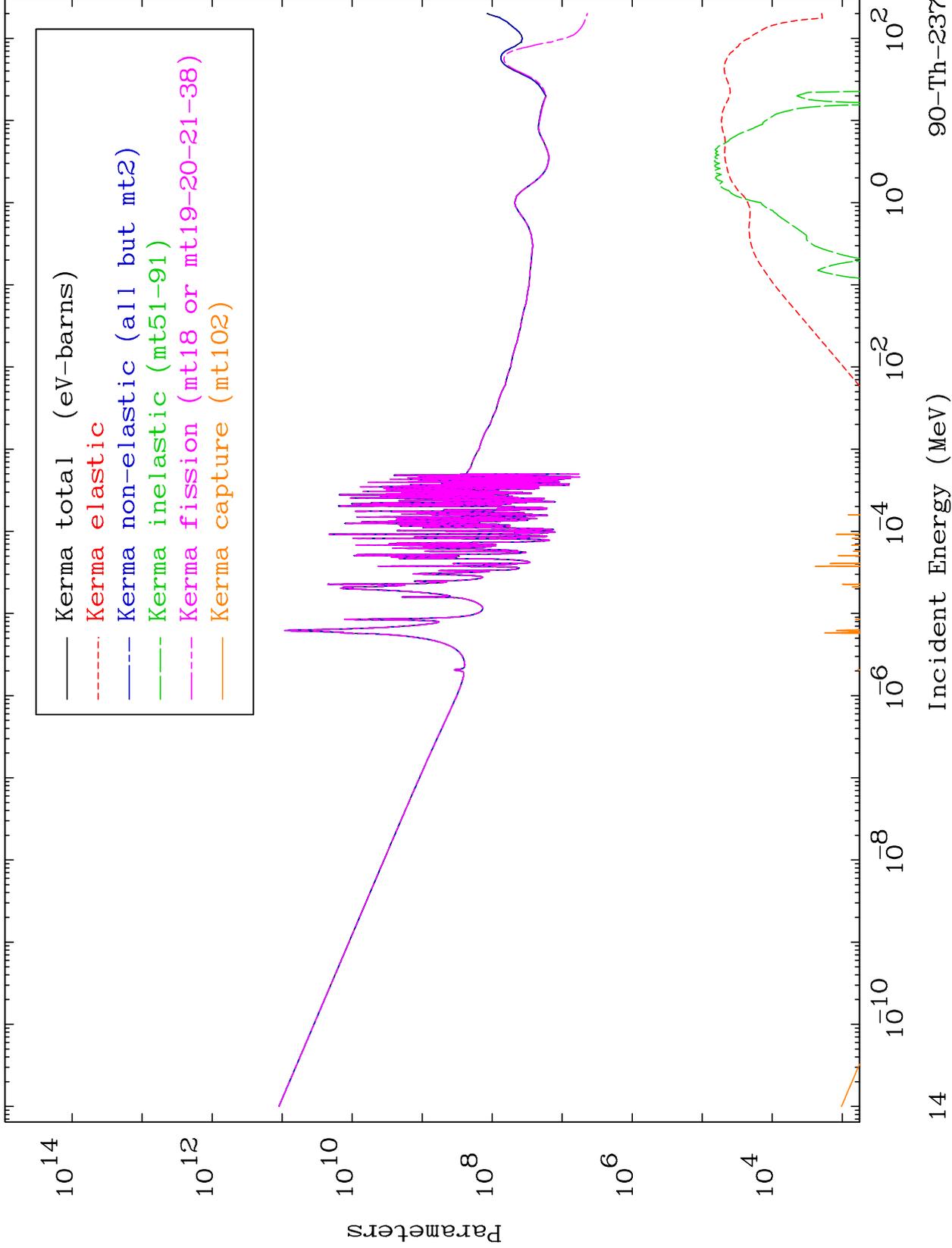
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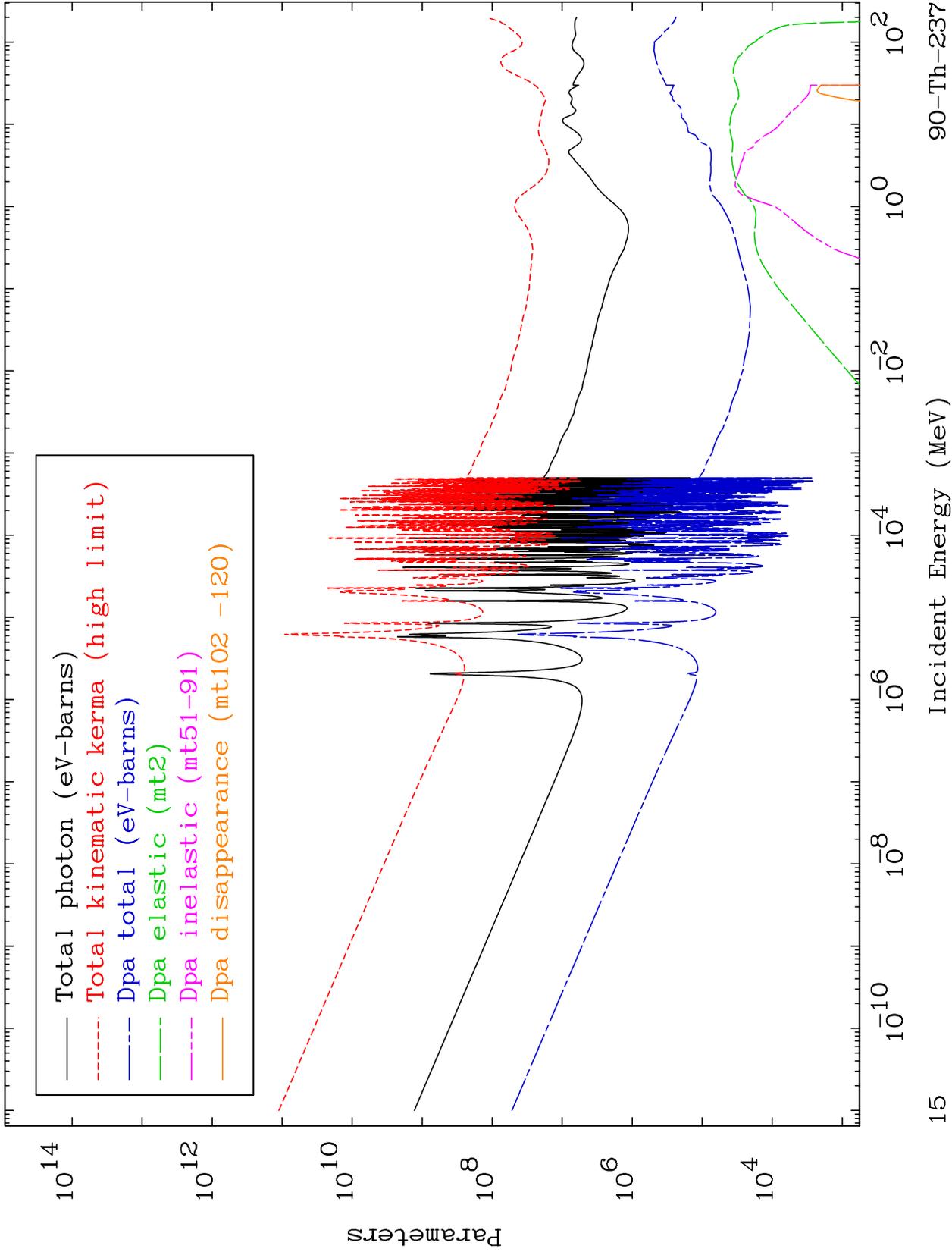
MAT 9055

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

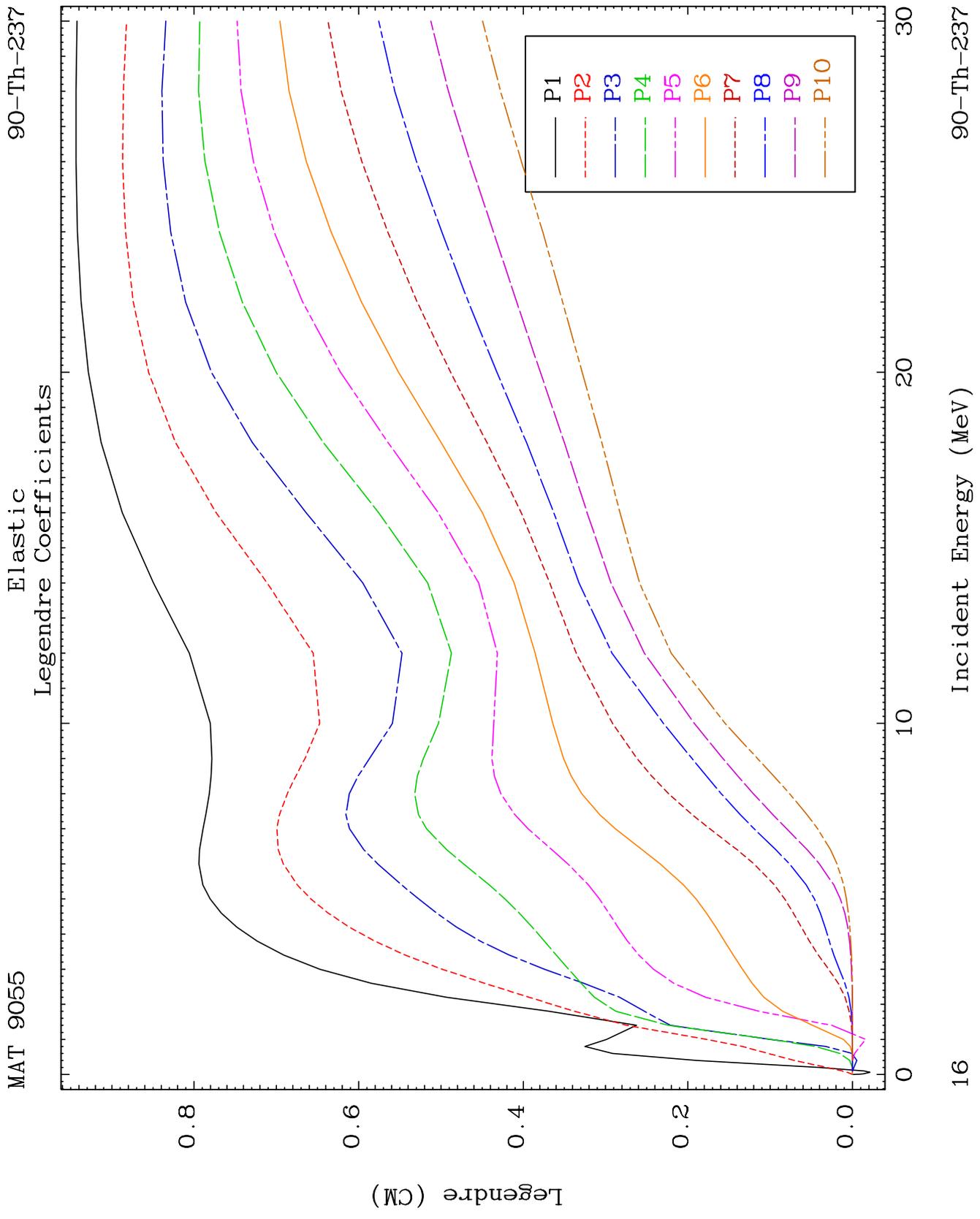
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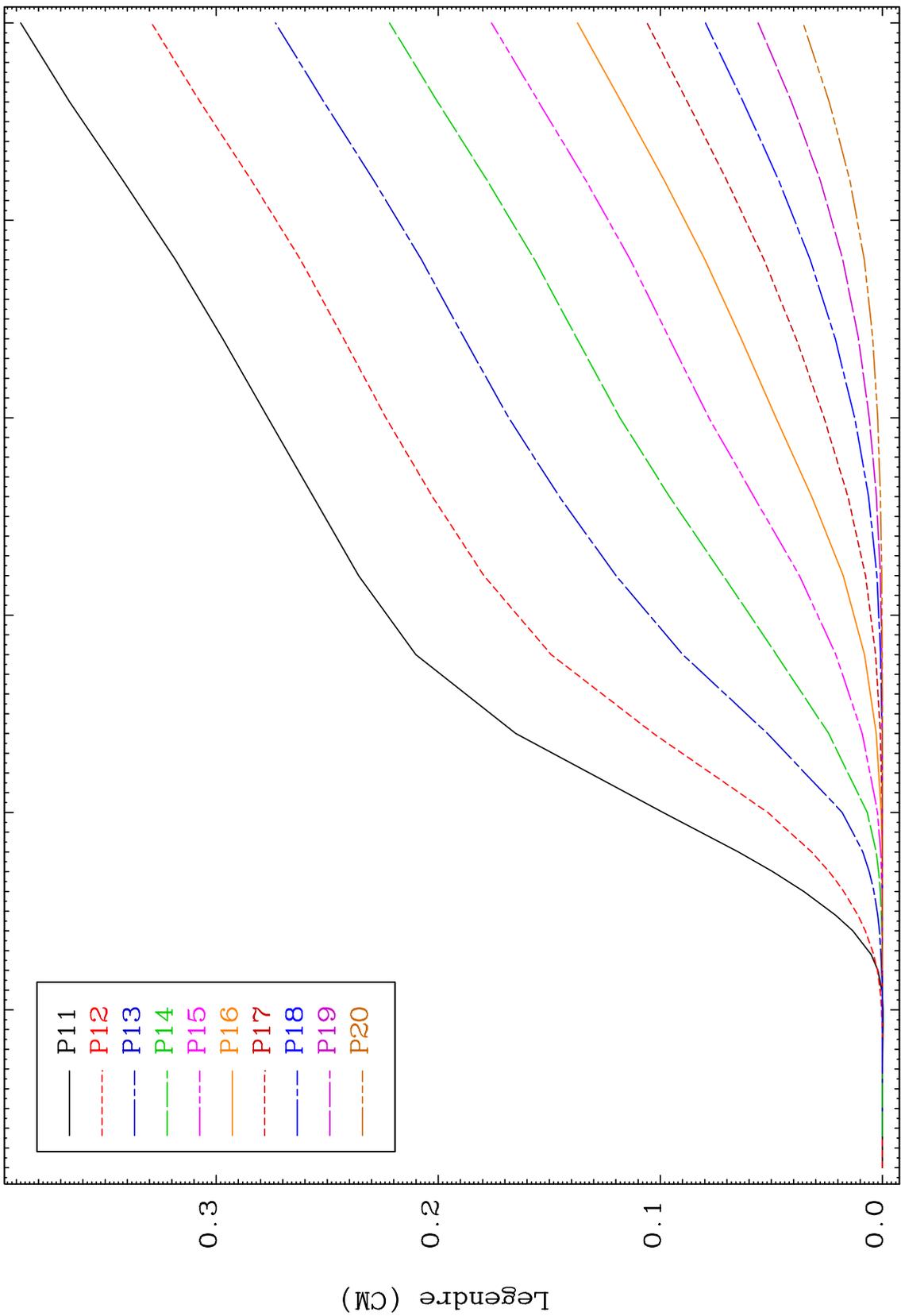




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Elastic Legendre Coefficients

90-Th-237



17

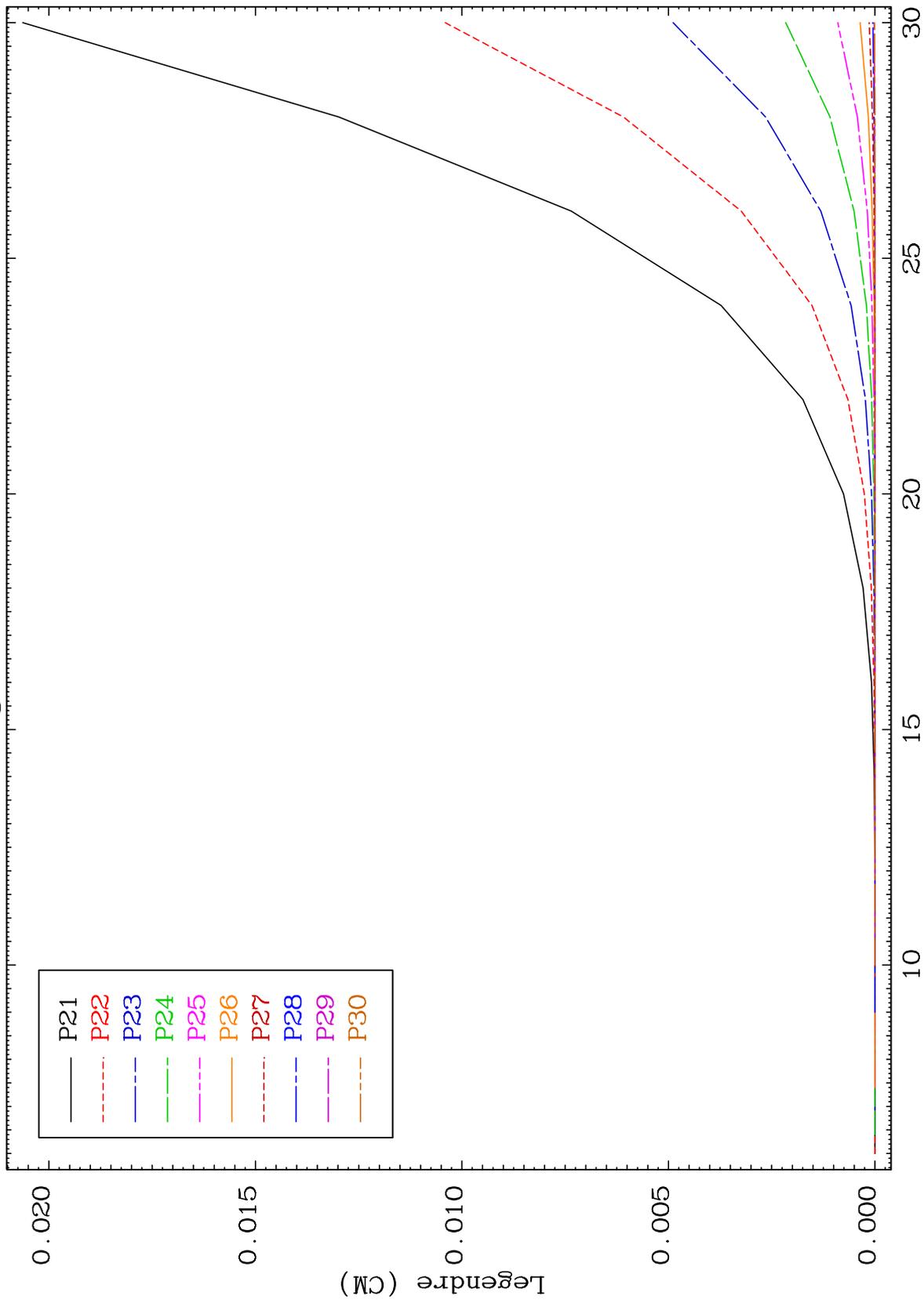
Incident Energy (MeV)

90-Th-237

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### Elastic Legendre Coefficients

90-Th-237



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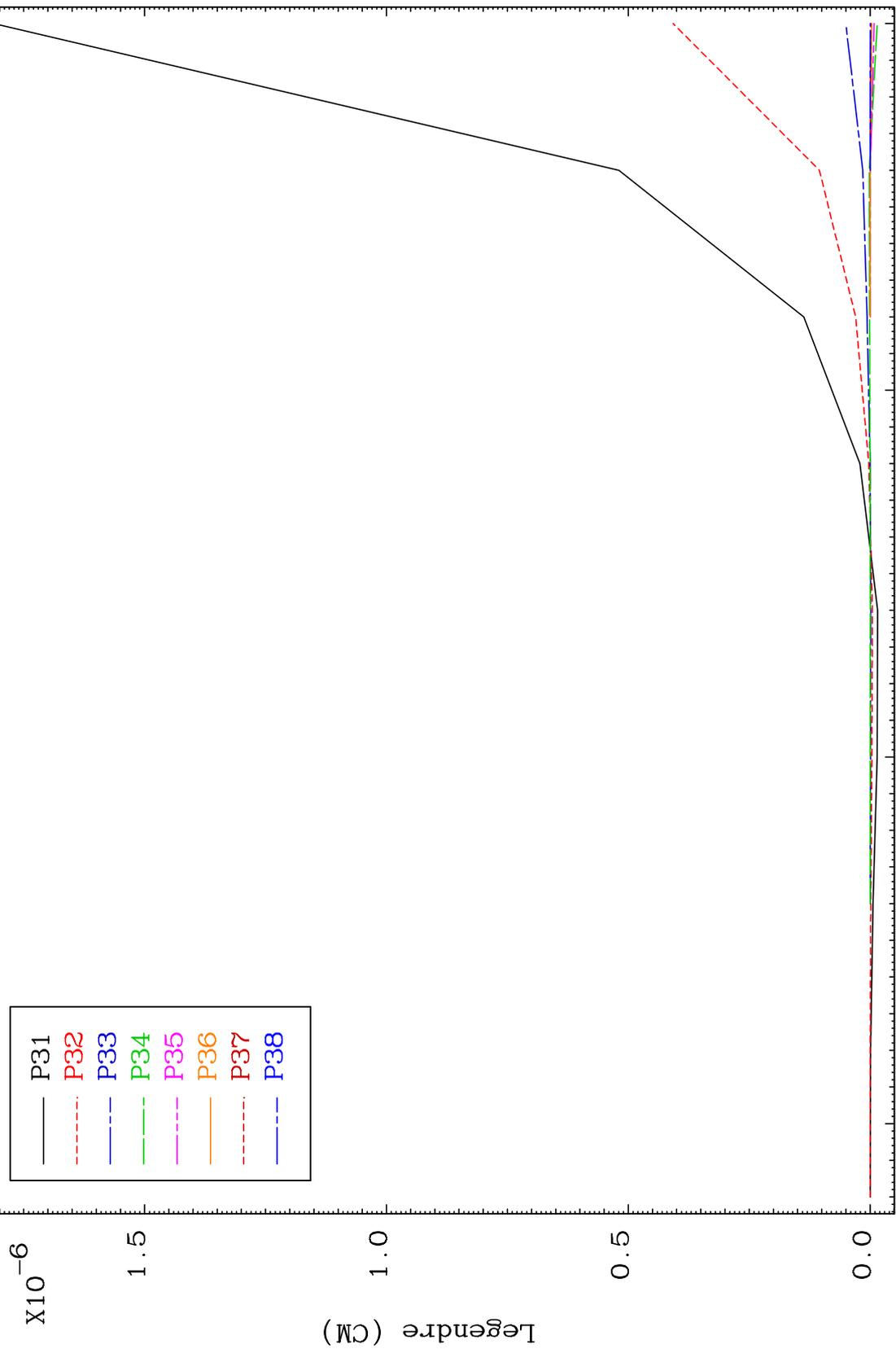
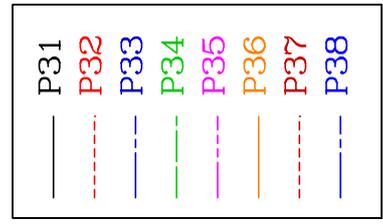
Incident Energy (MeV)

90-Th-237

MAT 90555

Elastic Legendre Coefficients

90-Th-237

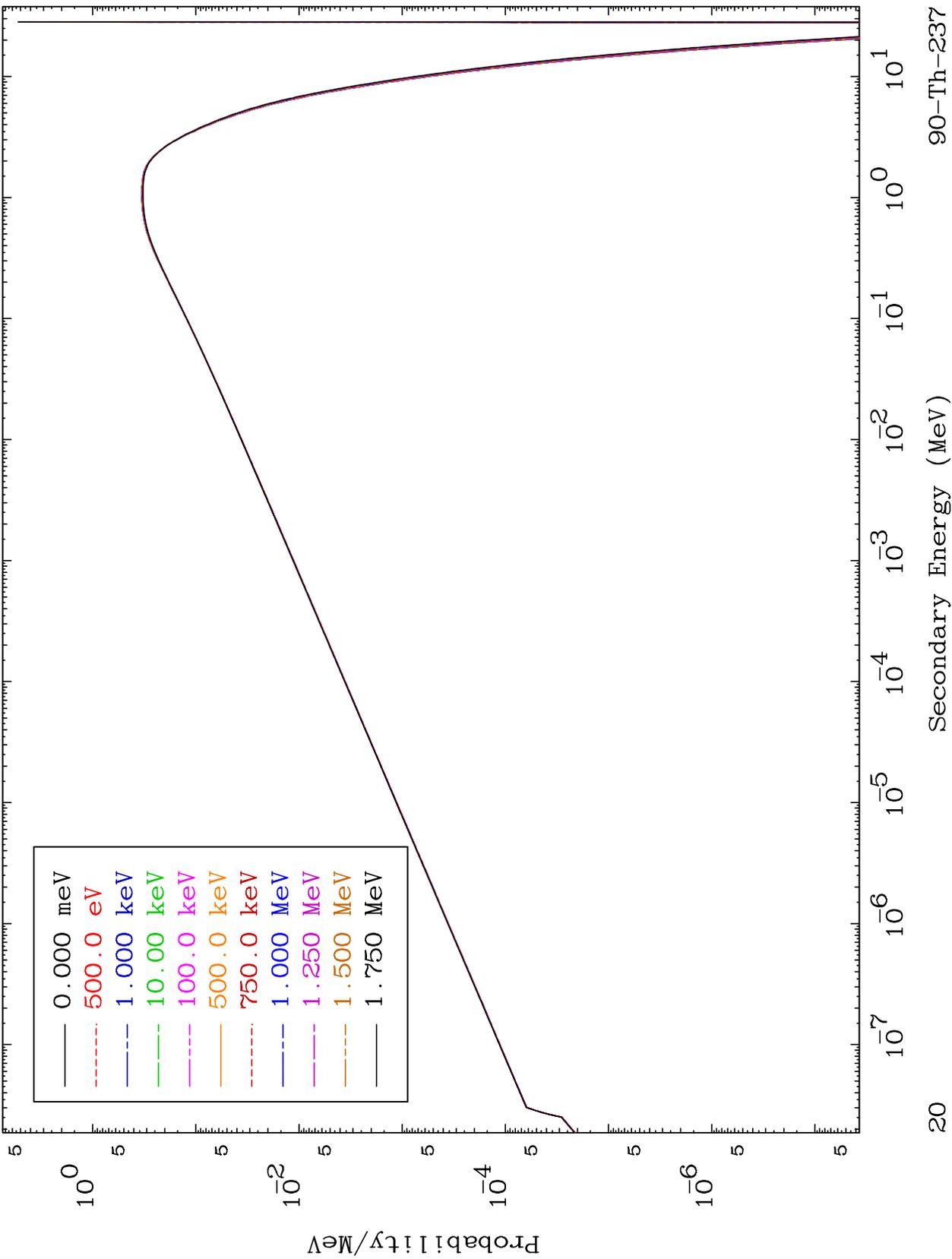


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Incident Energy (MeV)

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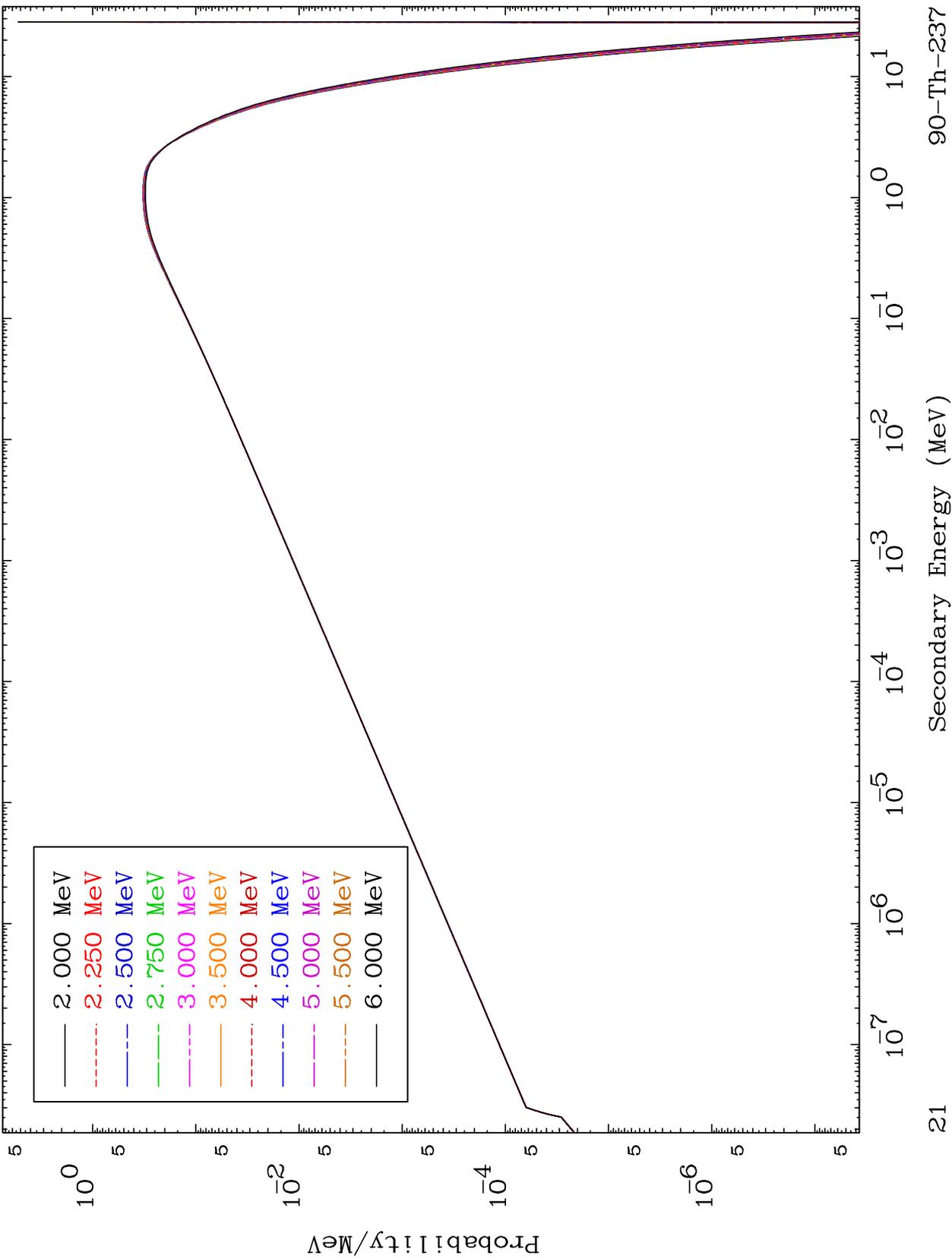
Fission Energy Distribution



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### Fission Energy Distribution

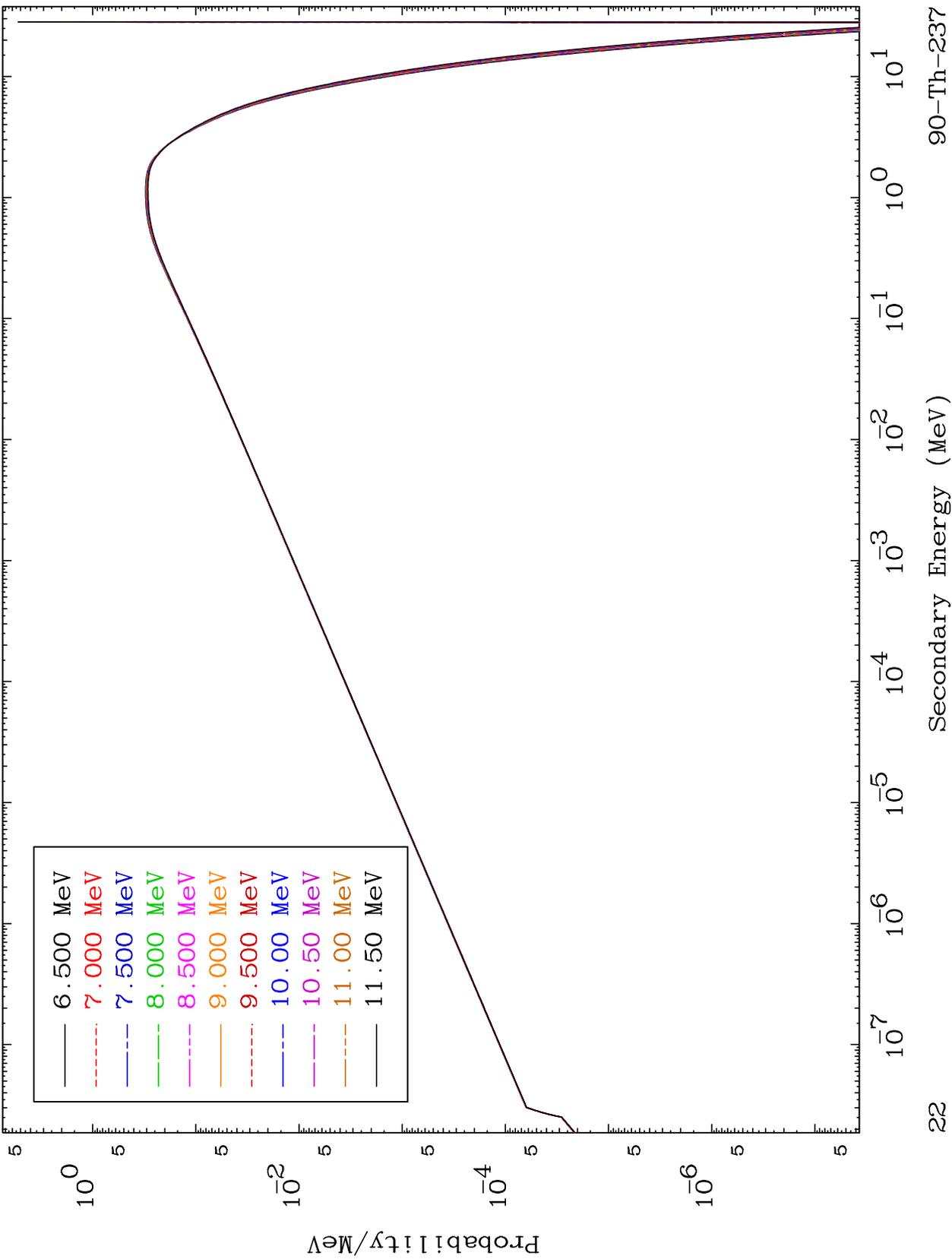
90-Th-237



90-Th-237

Secondary Energy (MeV)

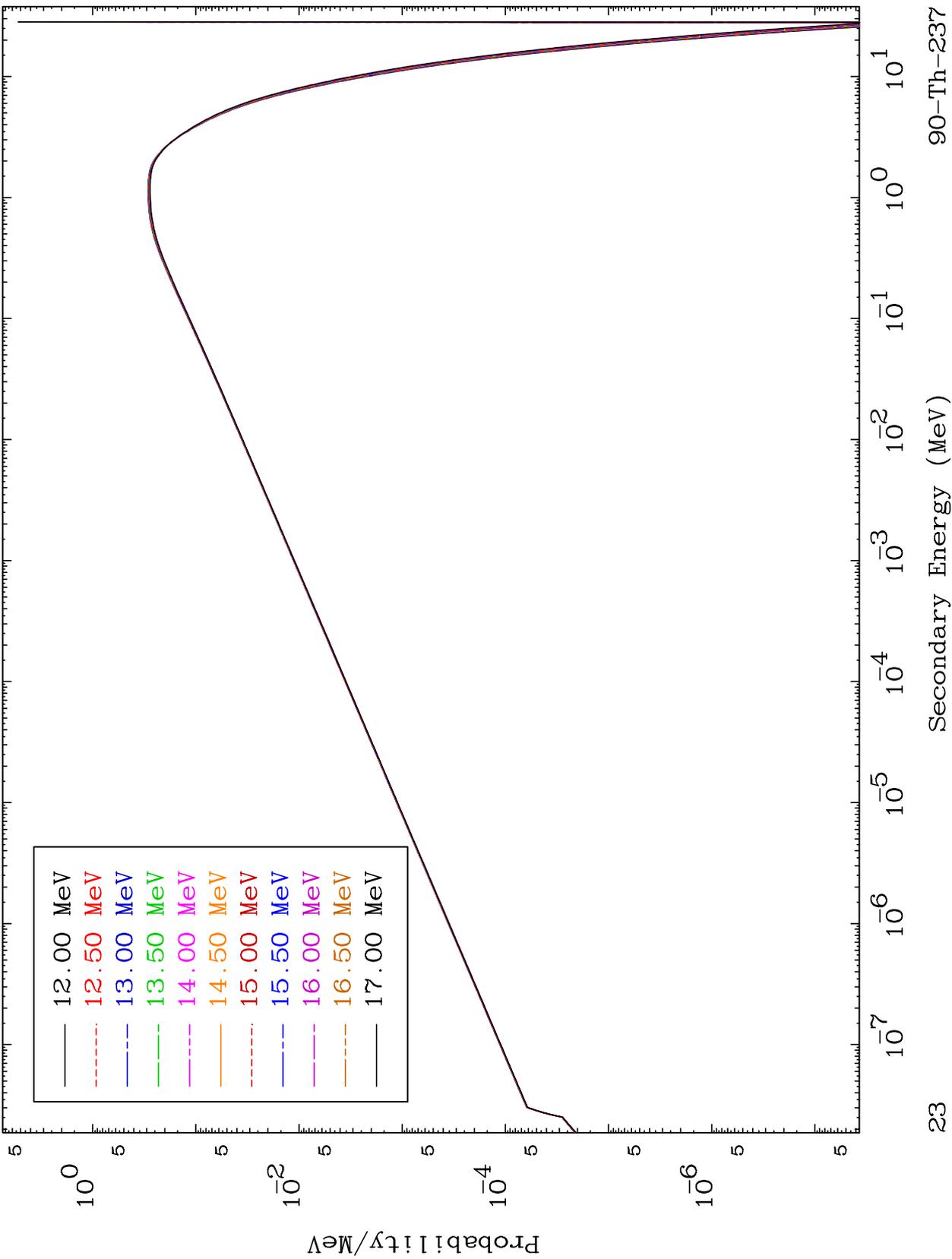
Fission Energy Distribution



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### Fission Energy Distribution

90-Th-237



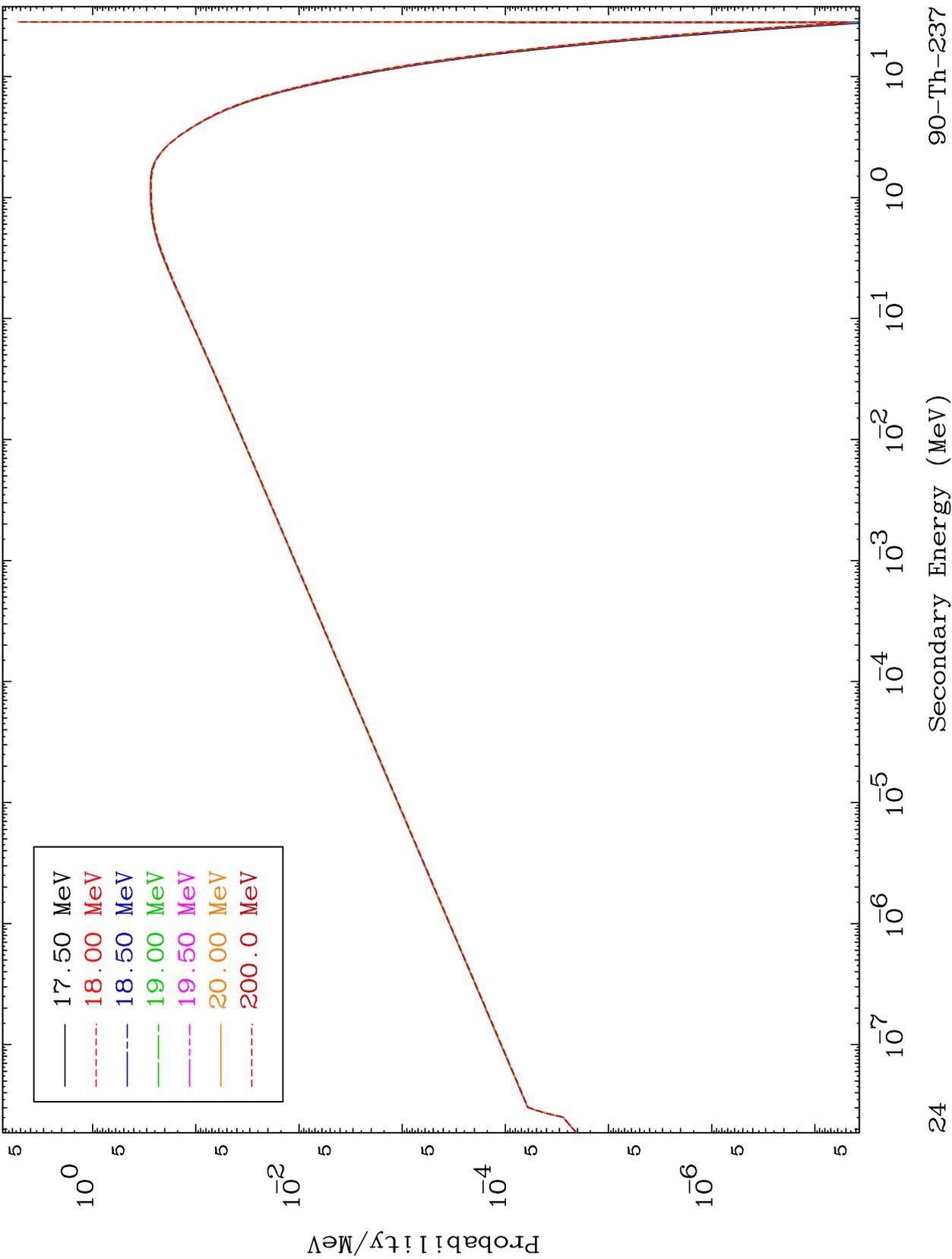
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### Fission Energy Distribution

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Secondary Energy (MeV)