

Program EVALPLOT
(Version 2015-2)

by

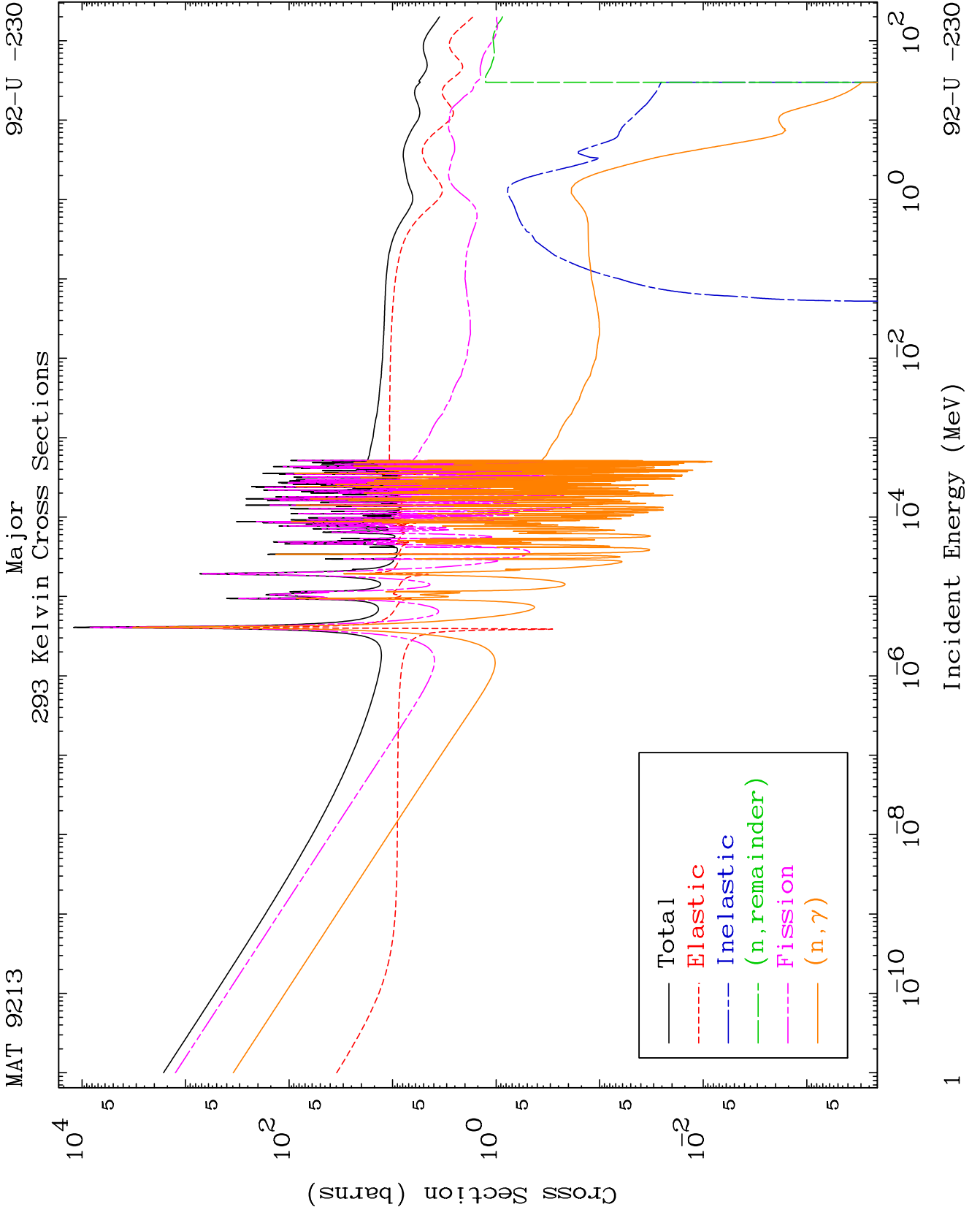
Dermott E. Cullen
(Present Contact Information)

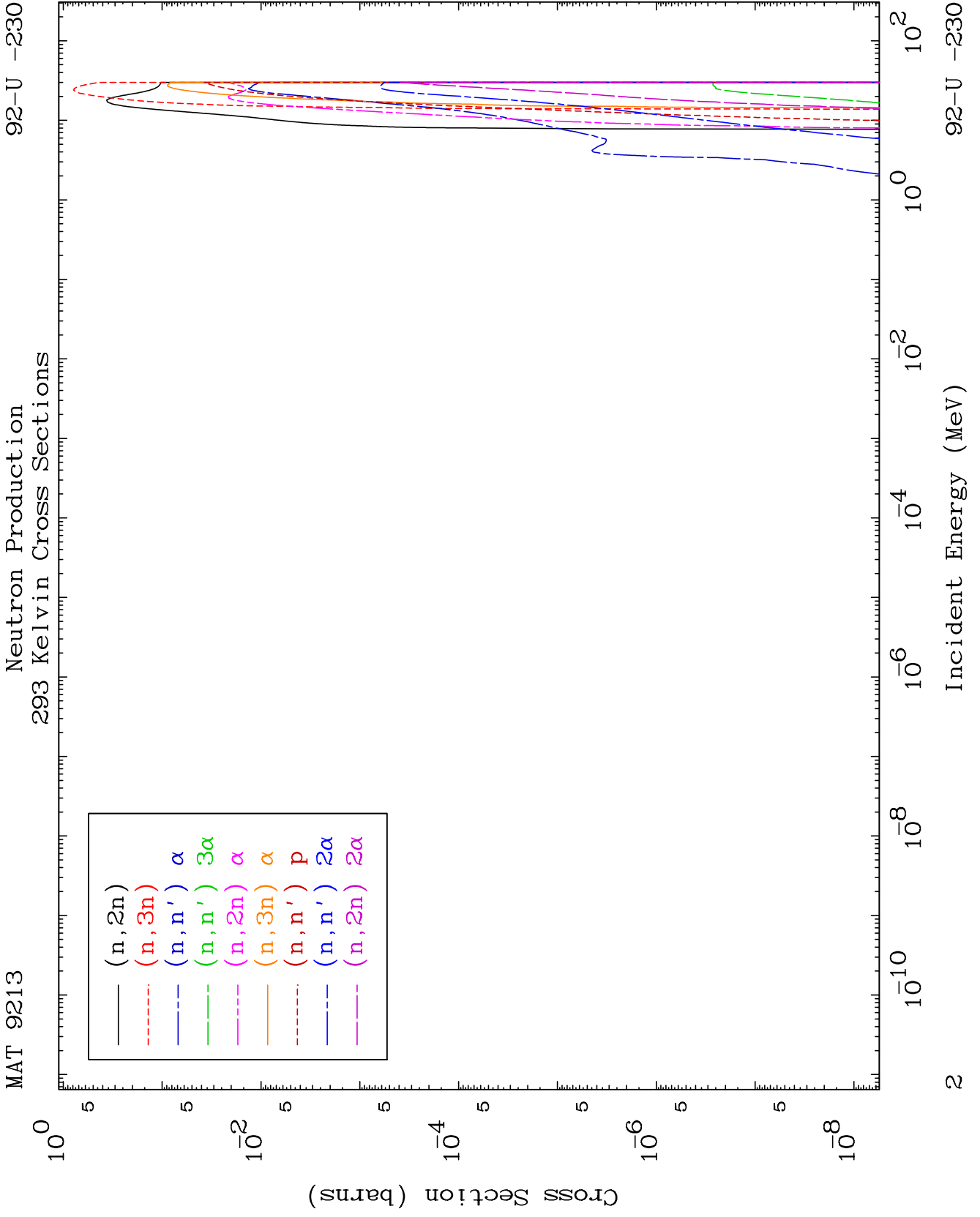
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Press Mouse Button to Start

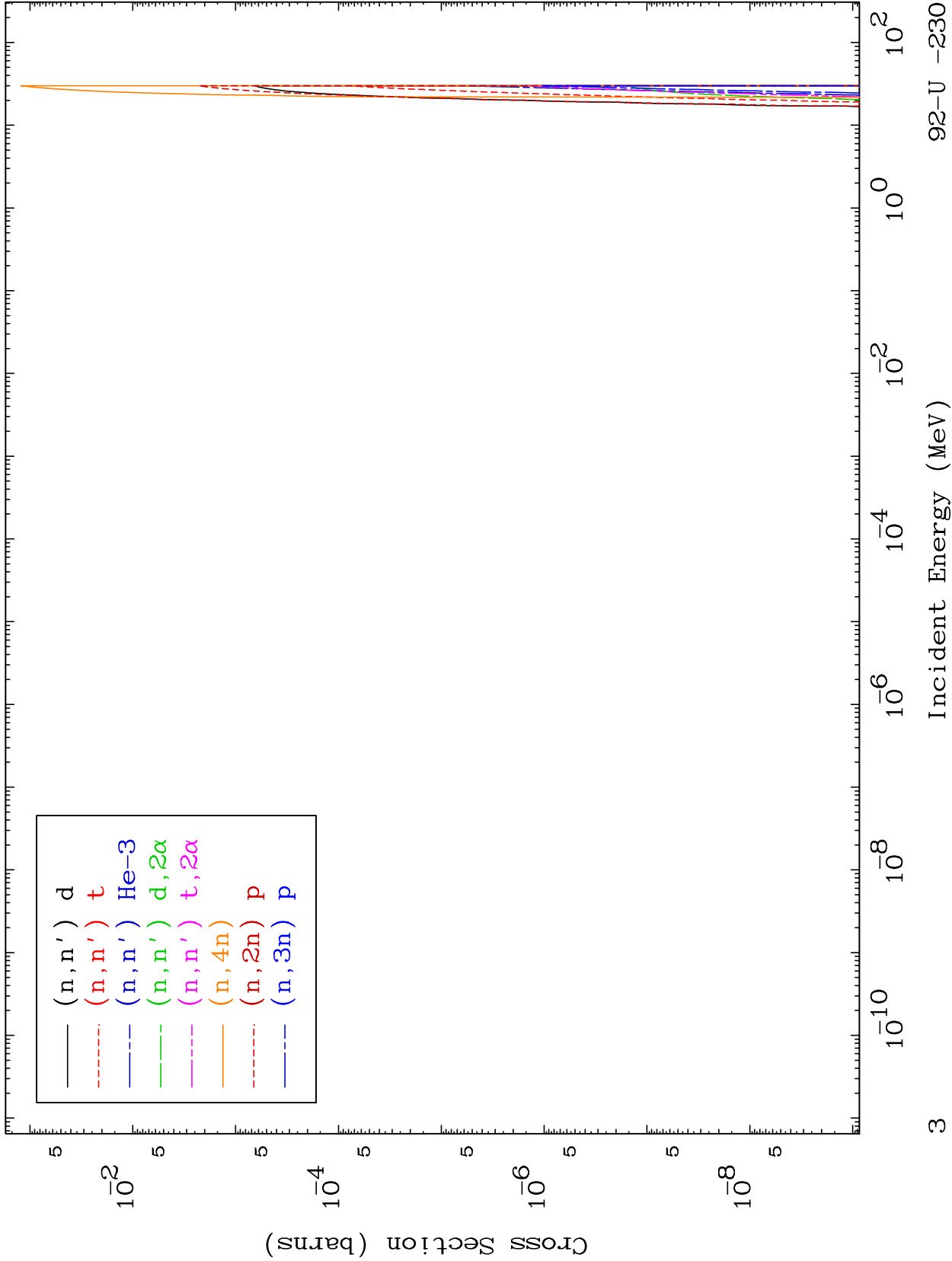




MAT 9213

Neutron Production
293 Kelvin Cross Sections

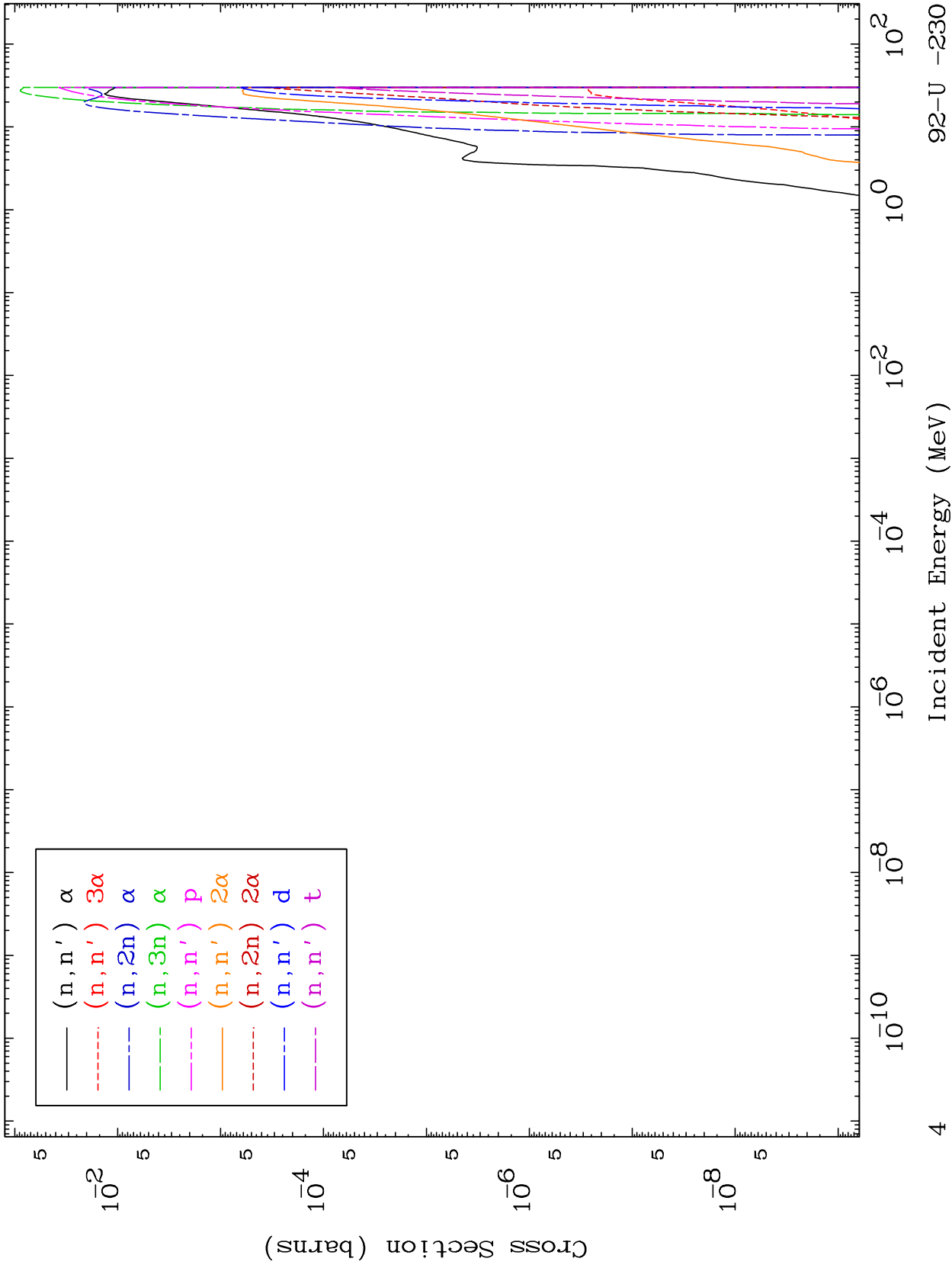
92-U -230



MAT 9213

Charged Particle
293 Kelvin Cross Sections

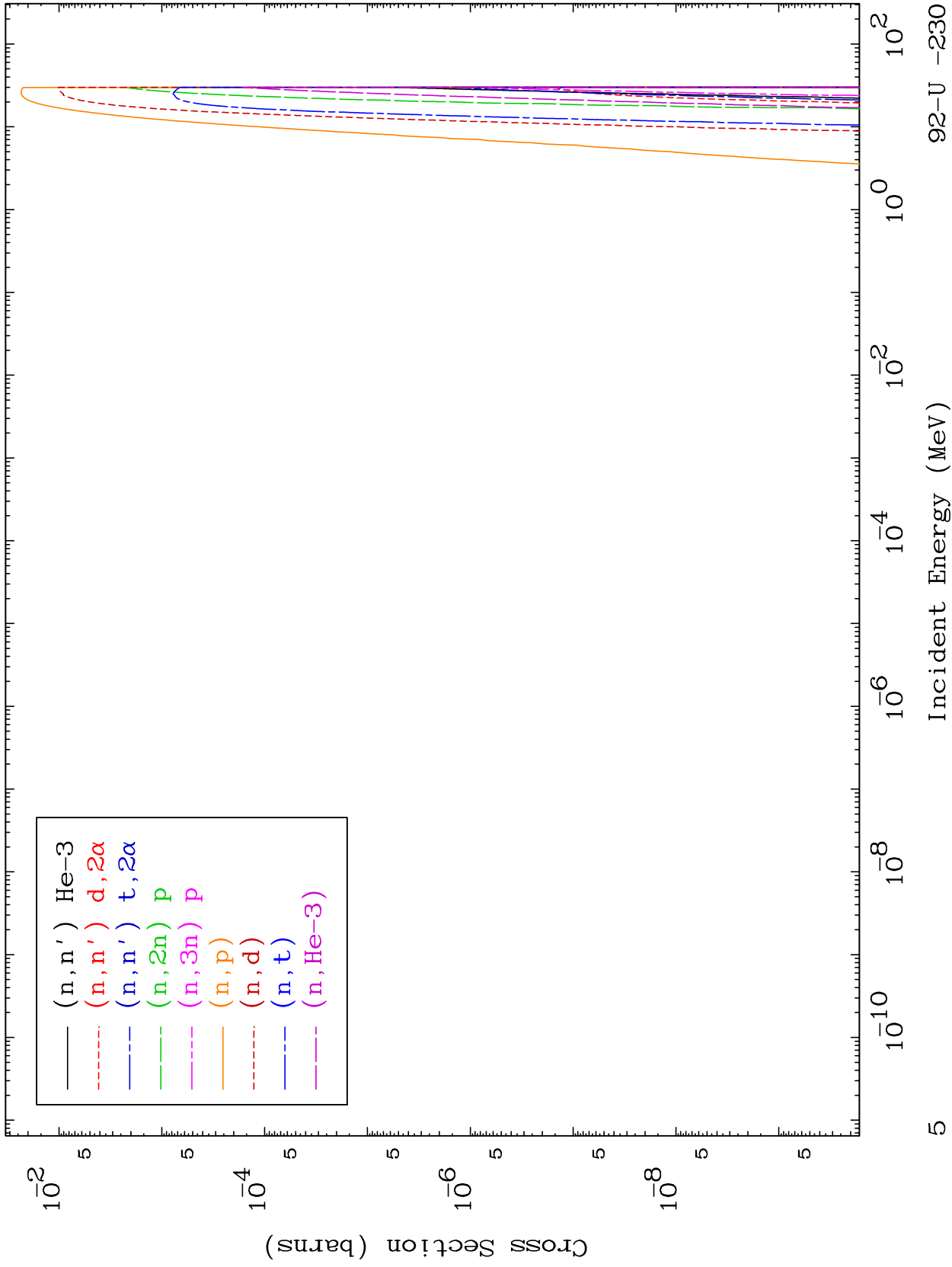
92-U -230



MAT 9213

Charged Particle
293 Kelvin Cross Sections

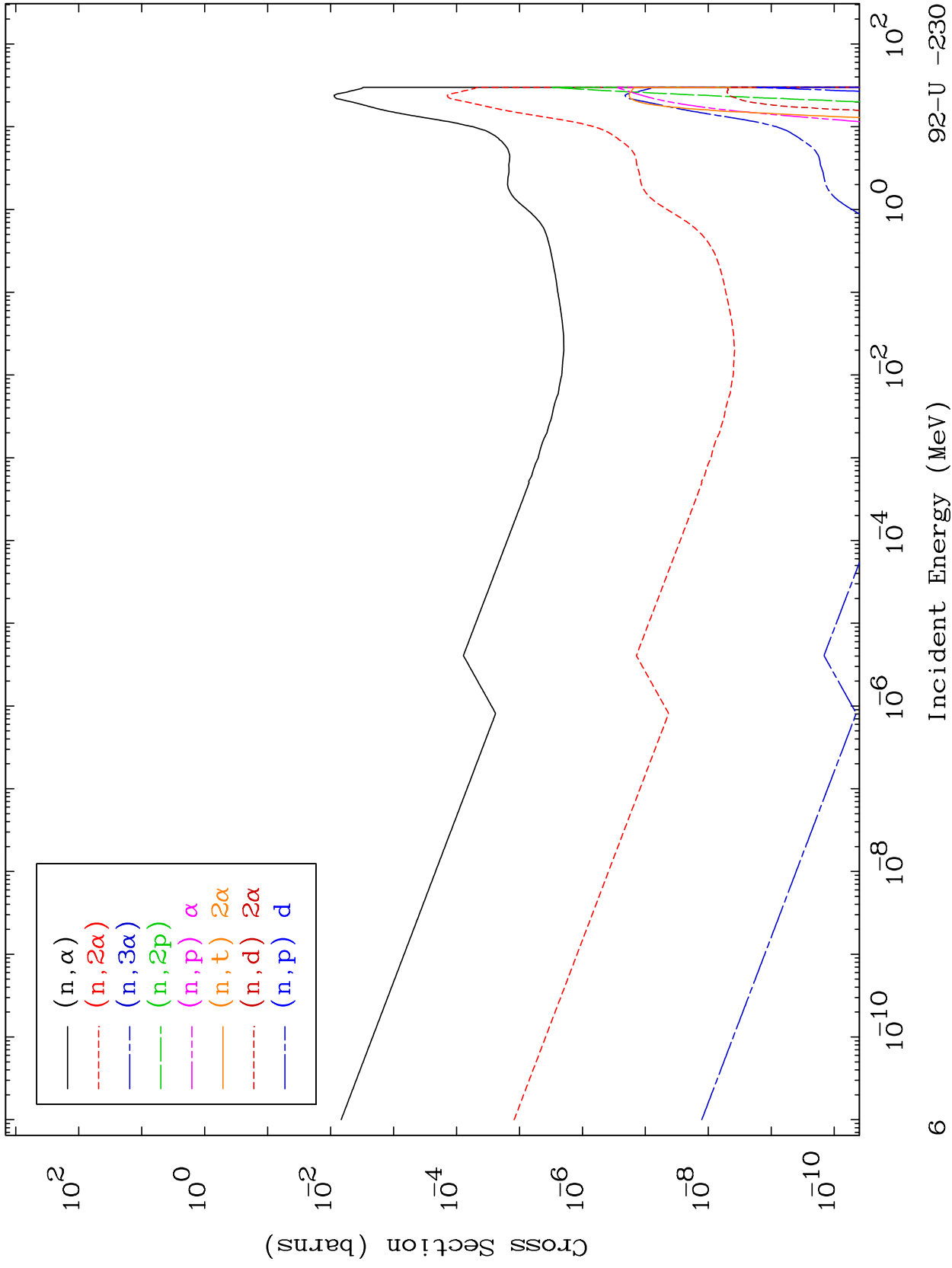
92-U -230



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Charged Particle
293 Kelvin Cross Sections

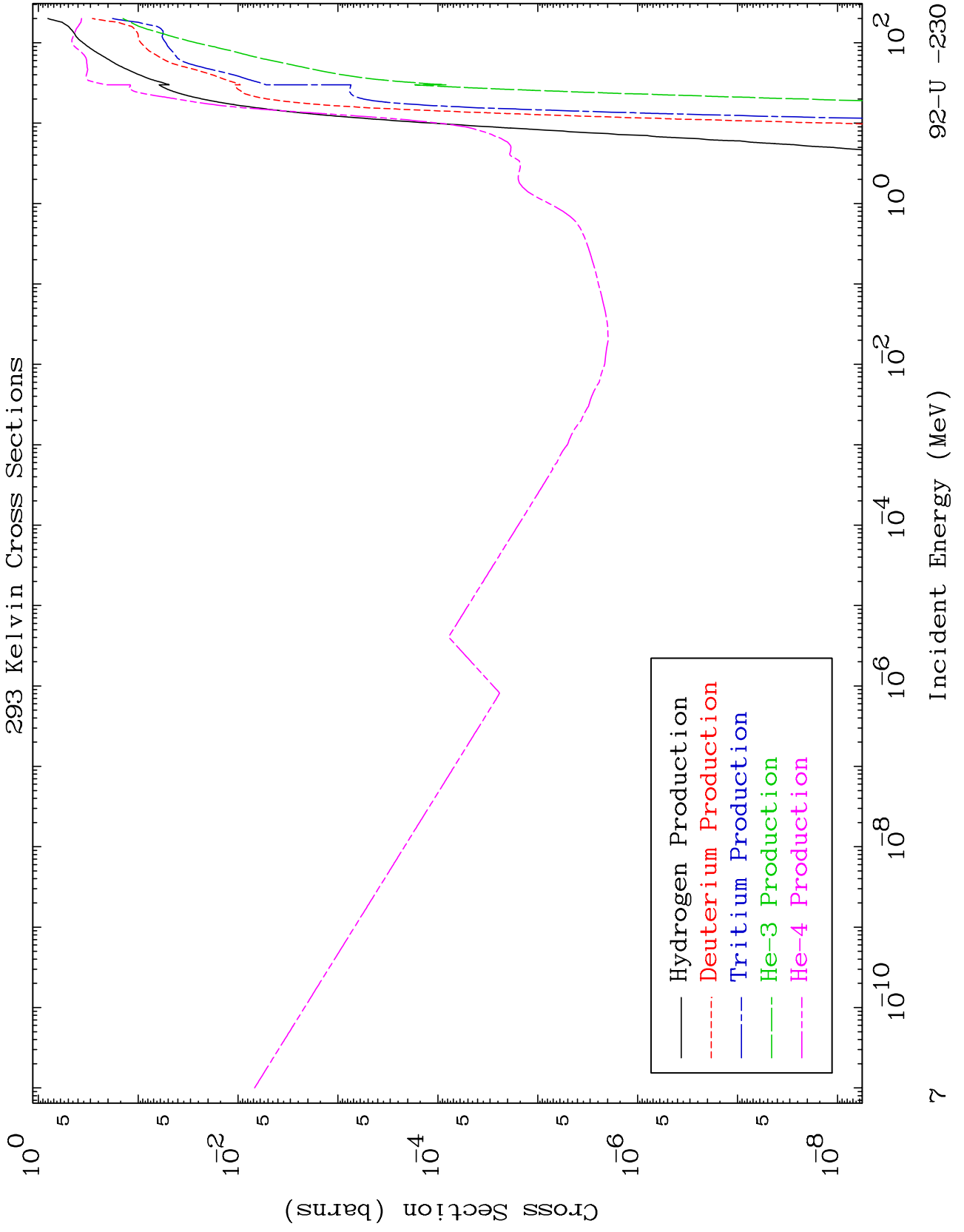
92-U -230



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Particle Production
293 Kelvin Cross Sections

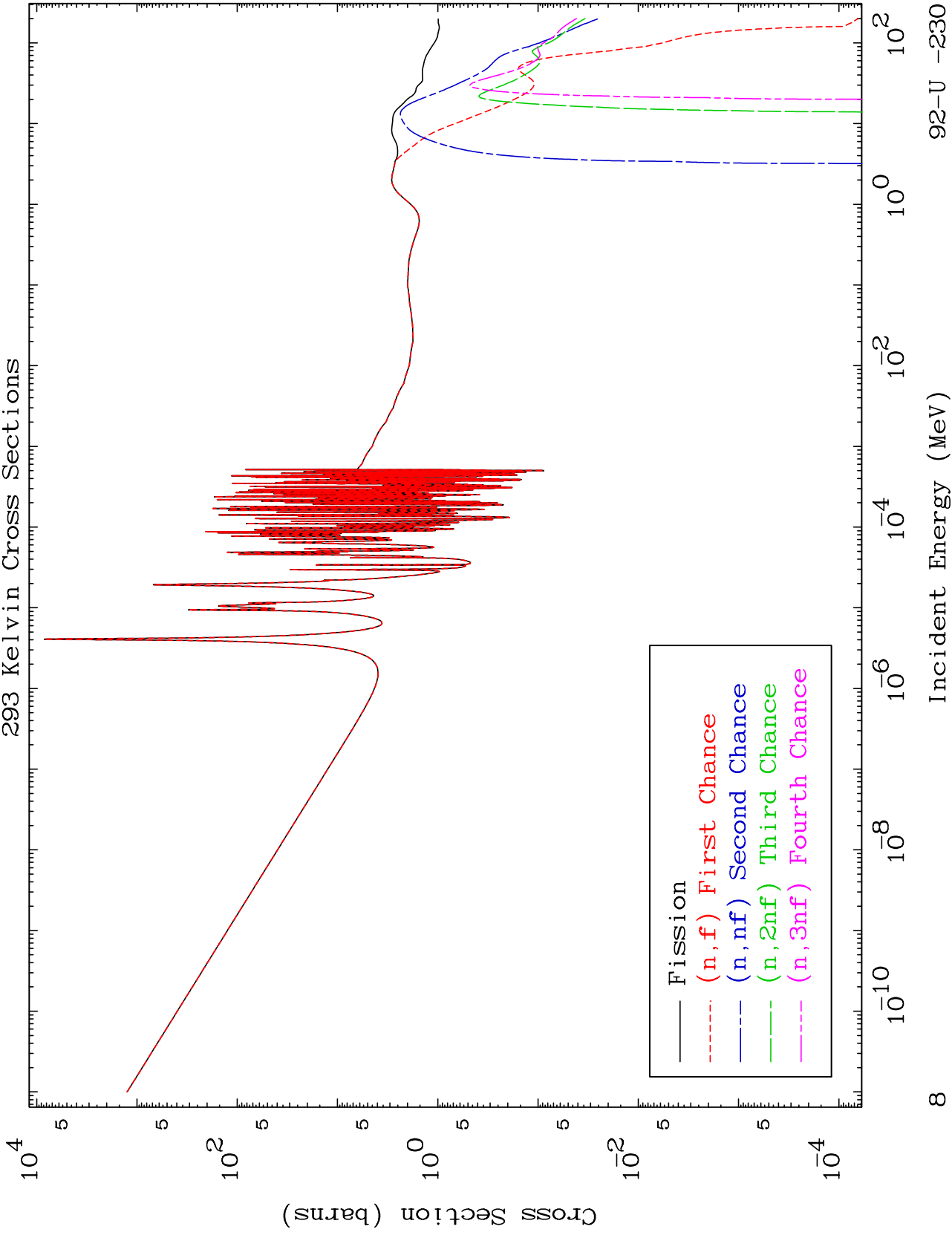
92-U -230



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Fission
293 Kelvin Cross Sections

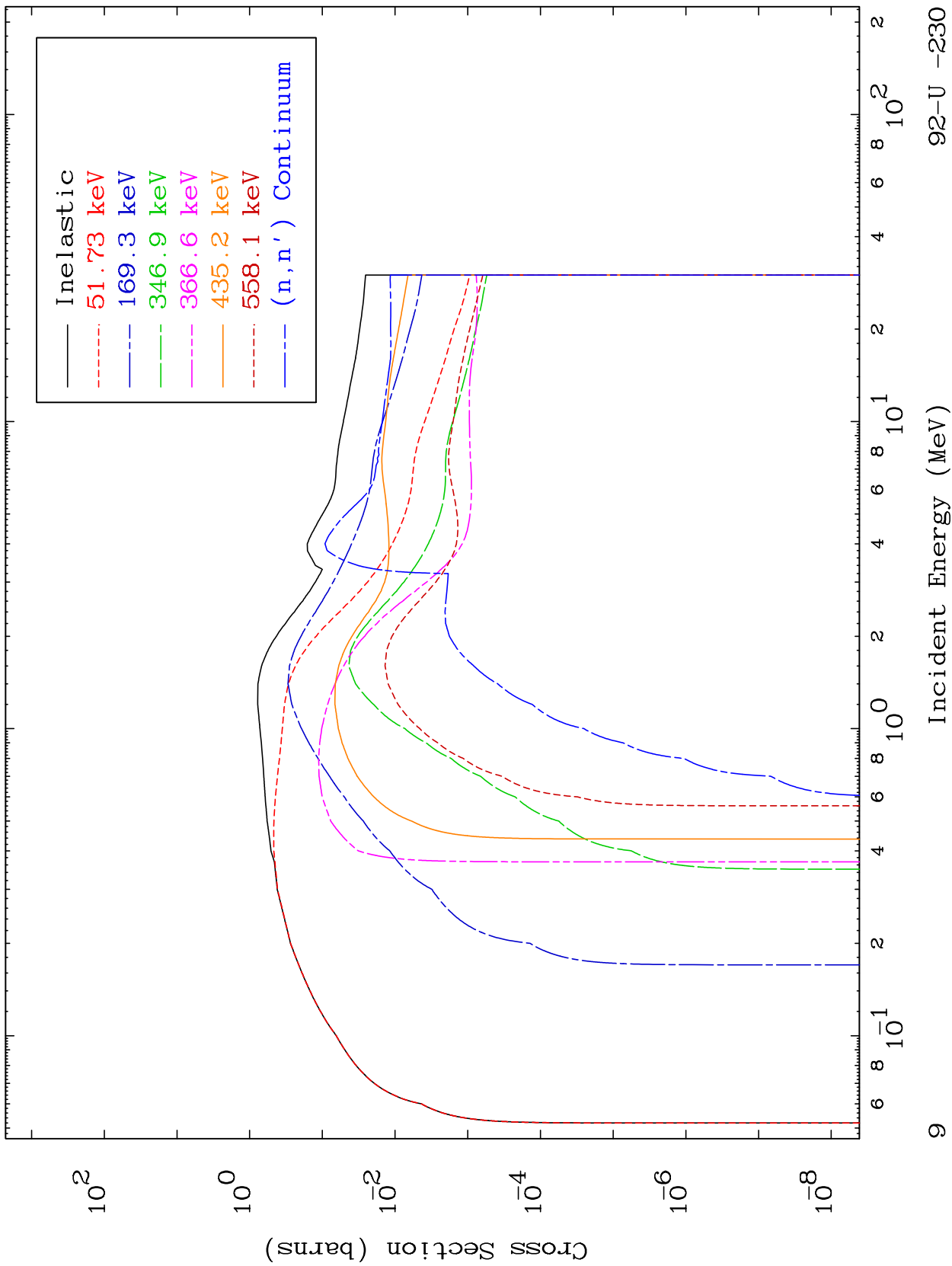
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(n,n') Level
293 Kelvin Cross Sections

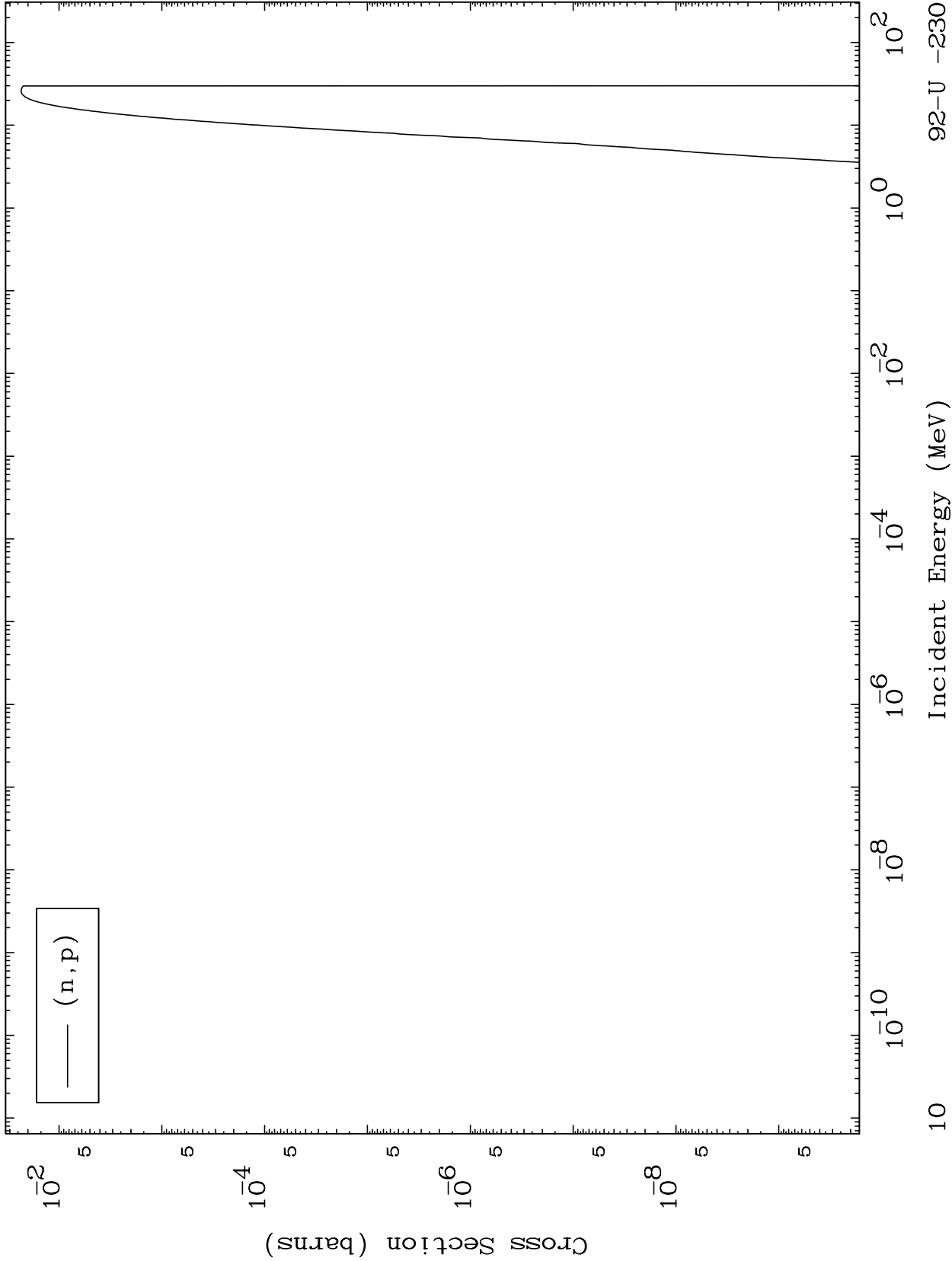
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(n,p) Levels
293 Kelvin Cross Sections

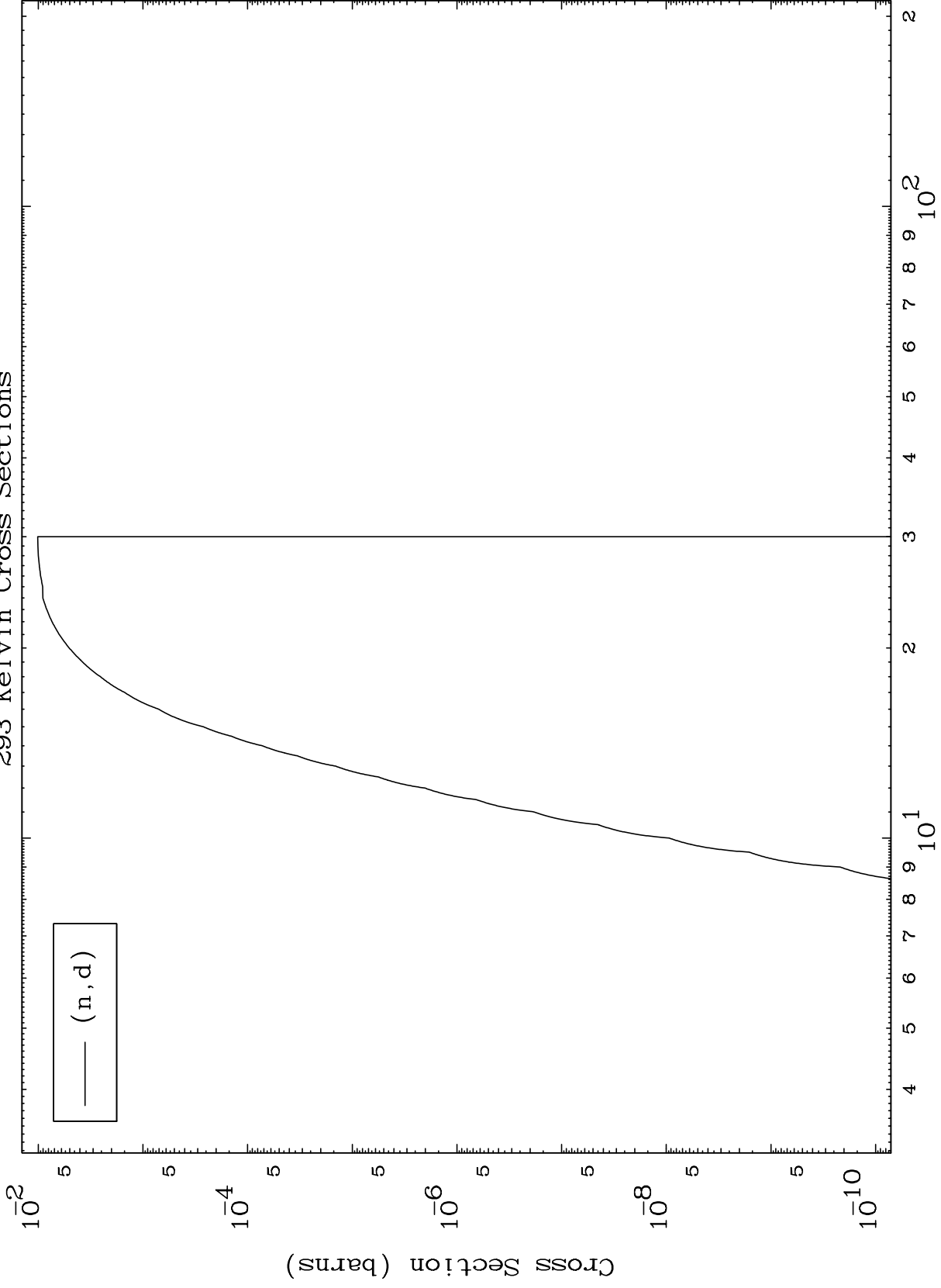
92-U -230



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(n,d) Levels
293 Kelvin Cross Sections

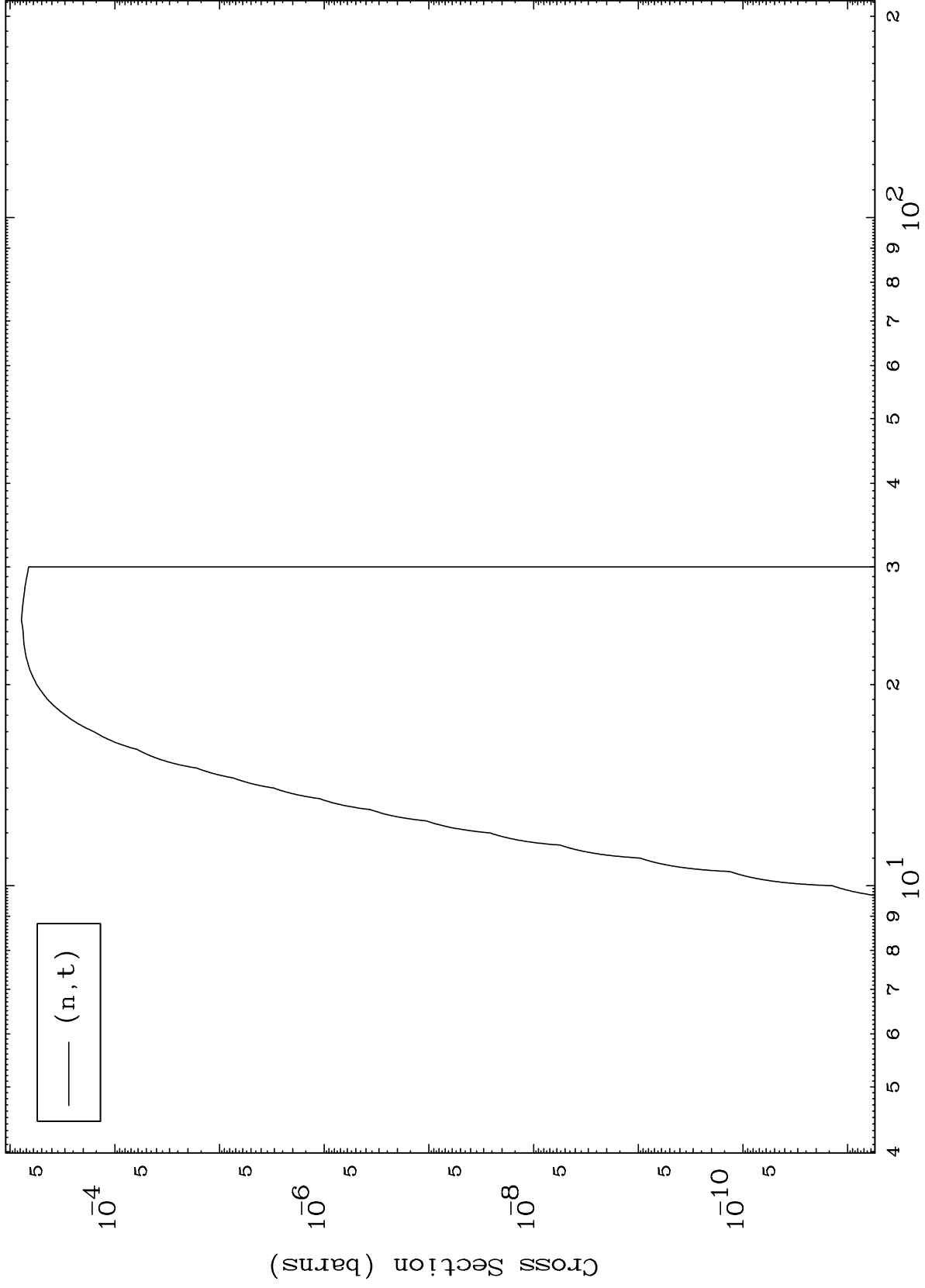
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(n,t) Levels
293 Kelvin Cross Sections

92-U -230



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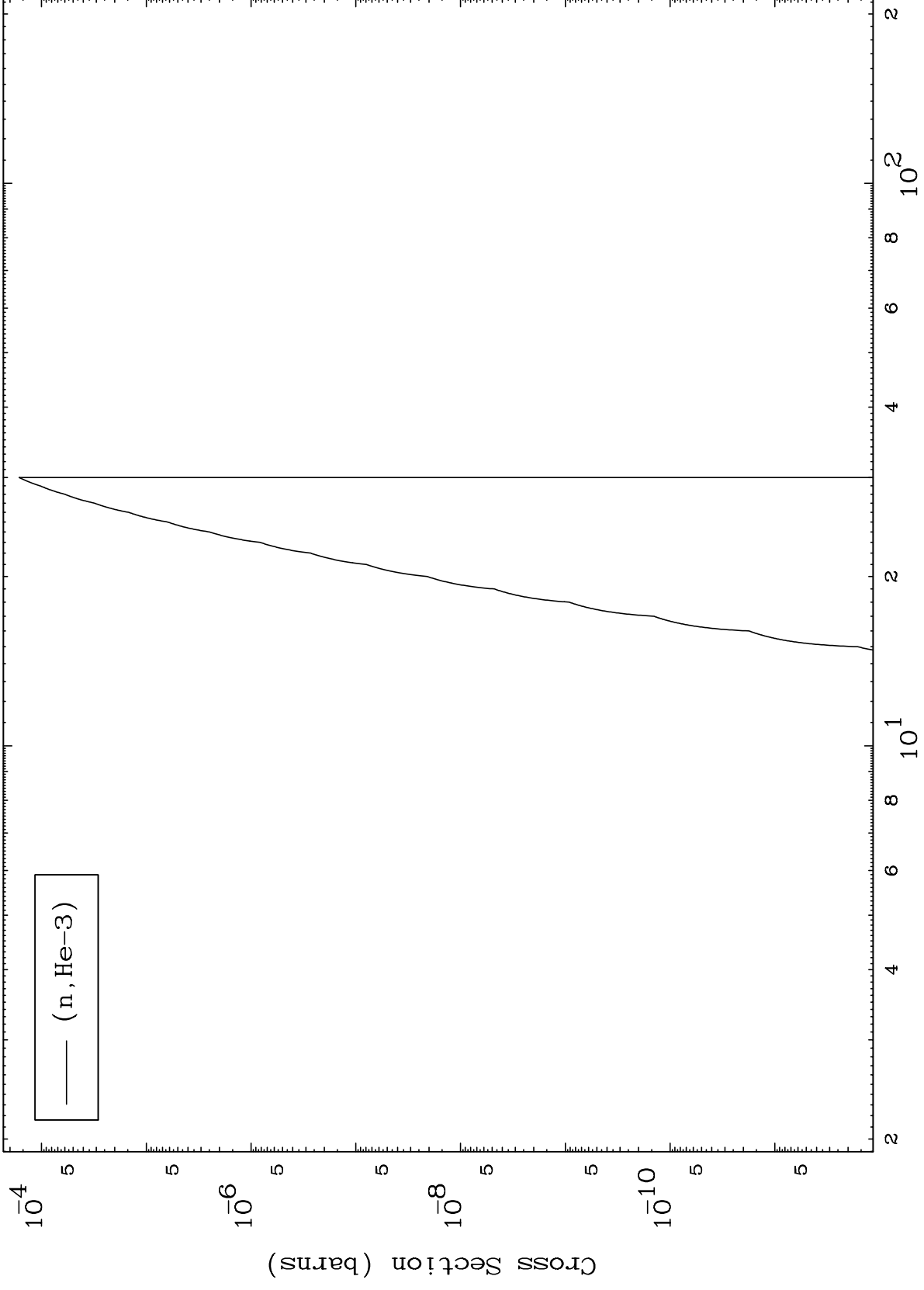
Incident Energy (MeV)

92-U -230

MAT 9213

(n,He3) Levels
293 Kelvin Cross Sections

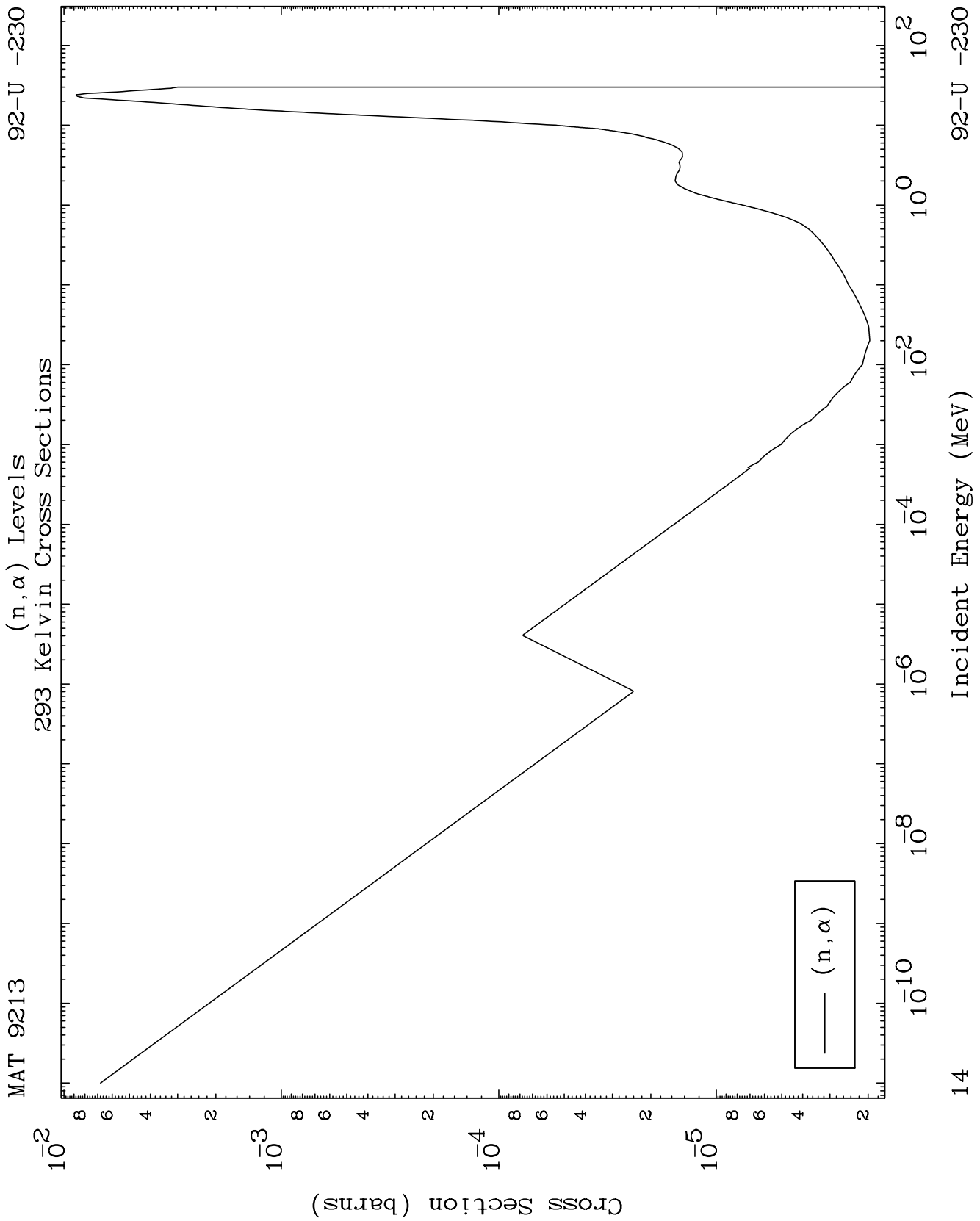
92-U -230

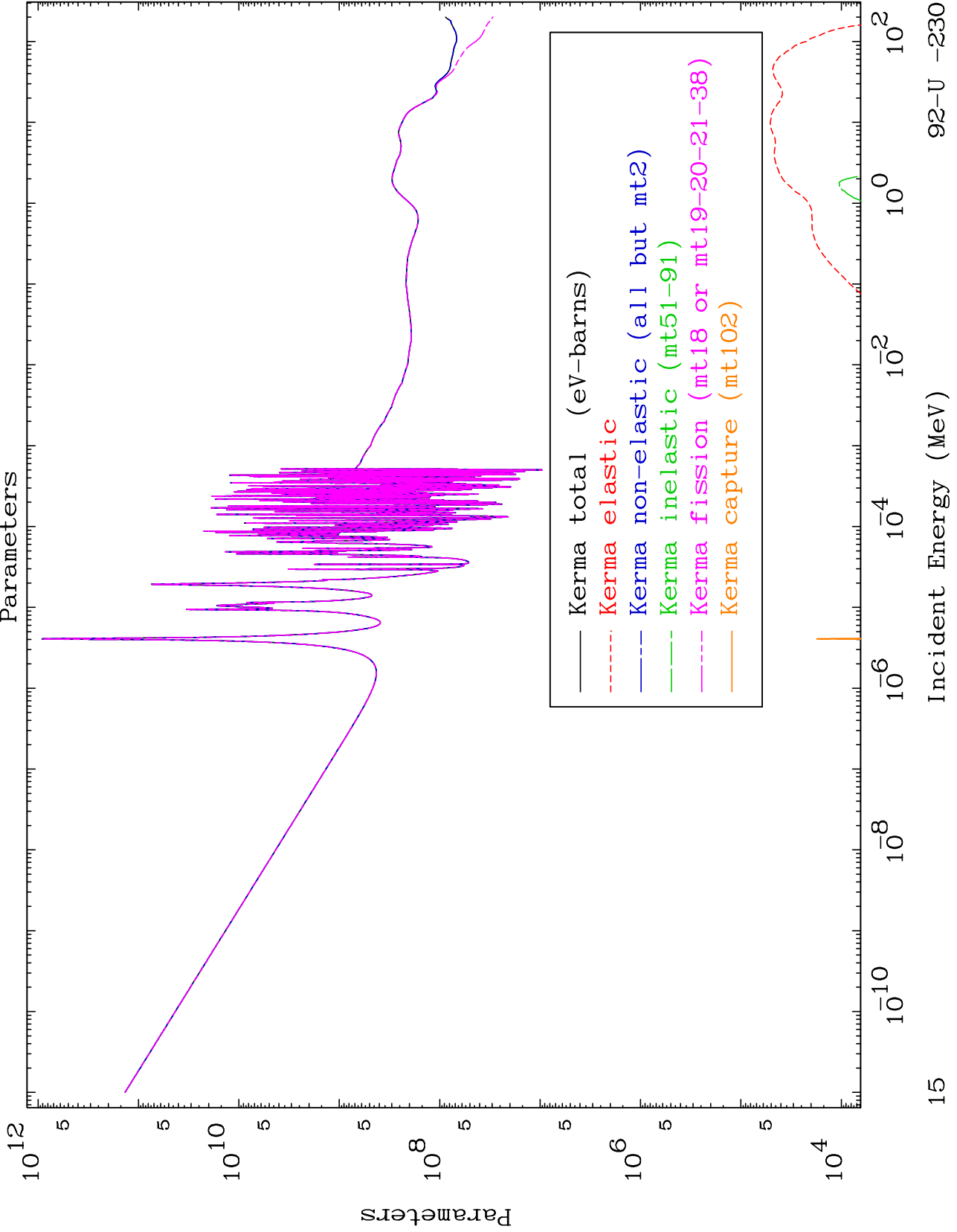


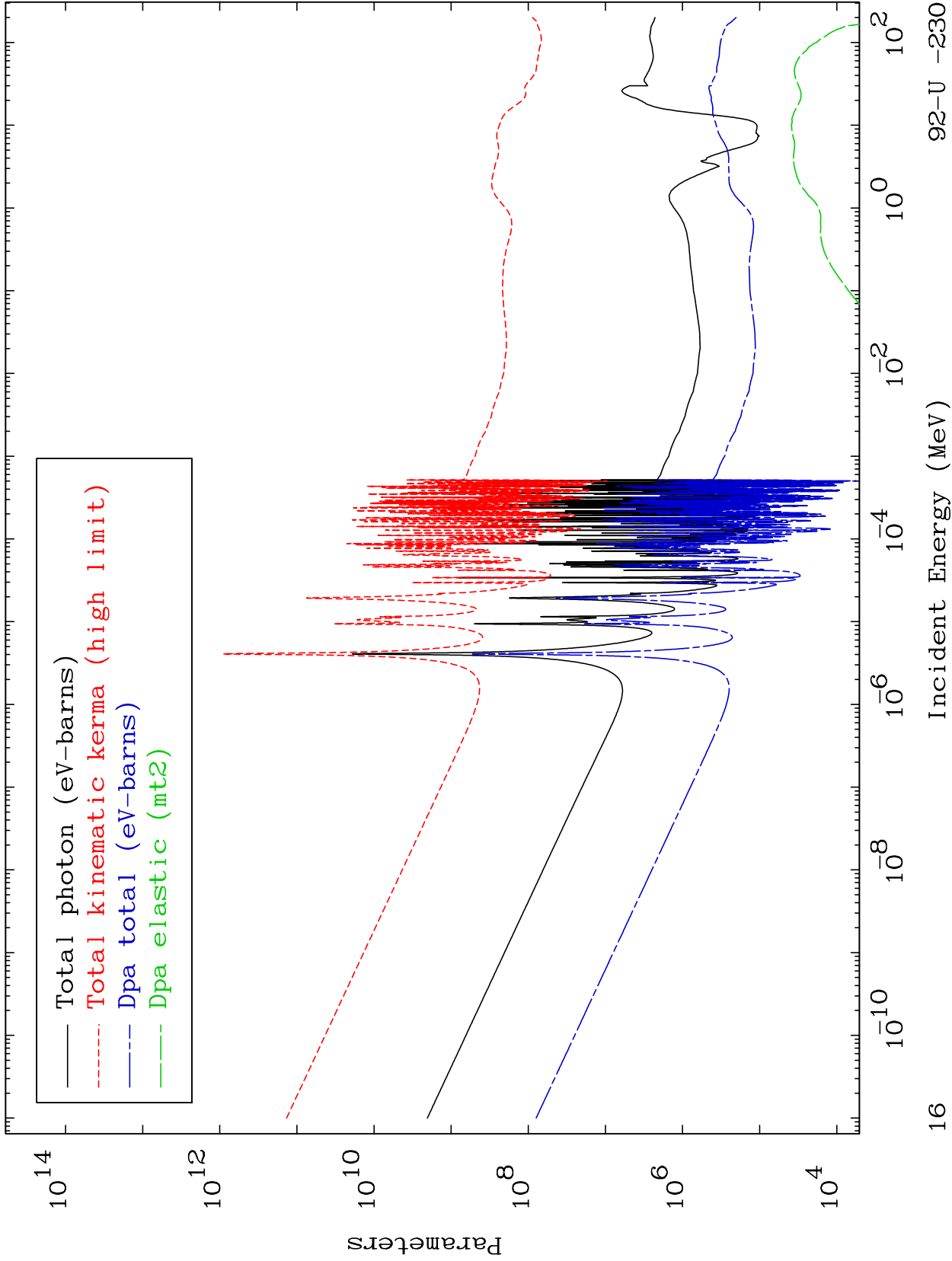
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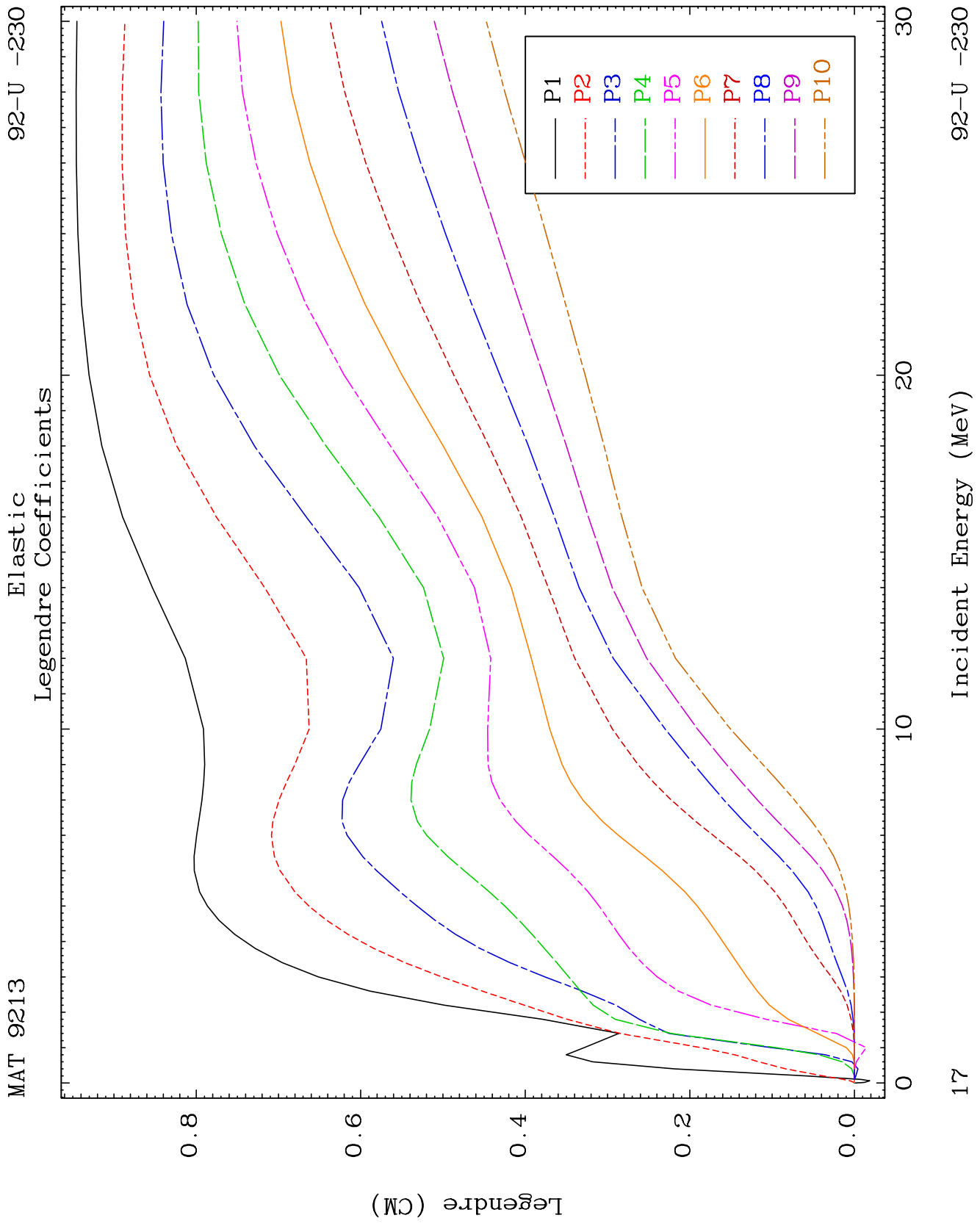
Incident Energy (MeV)

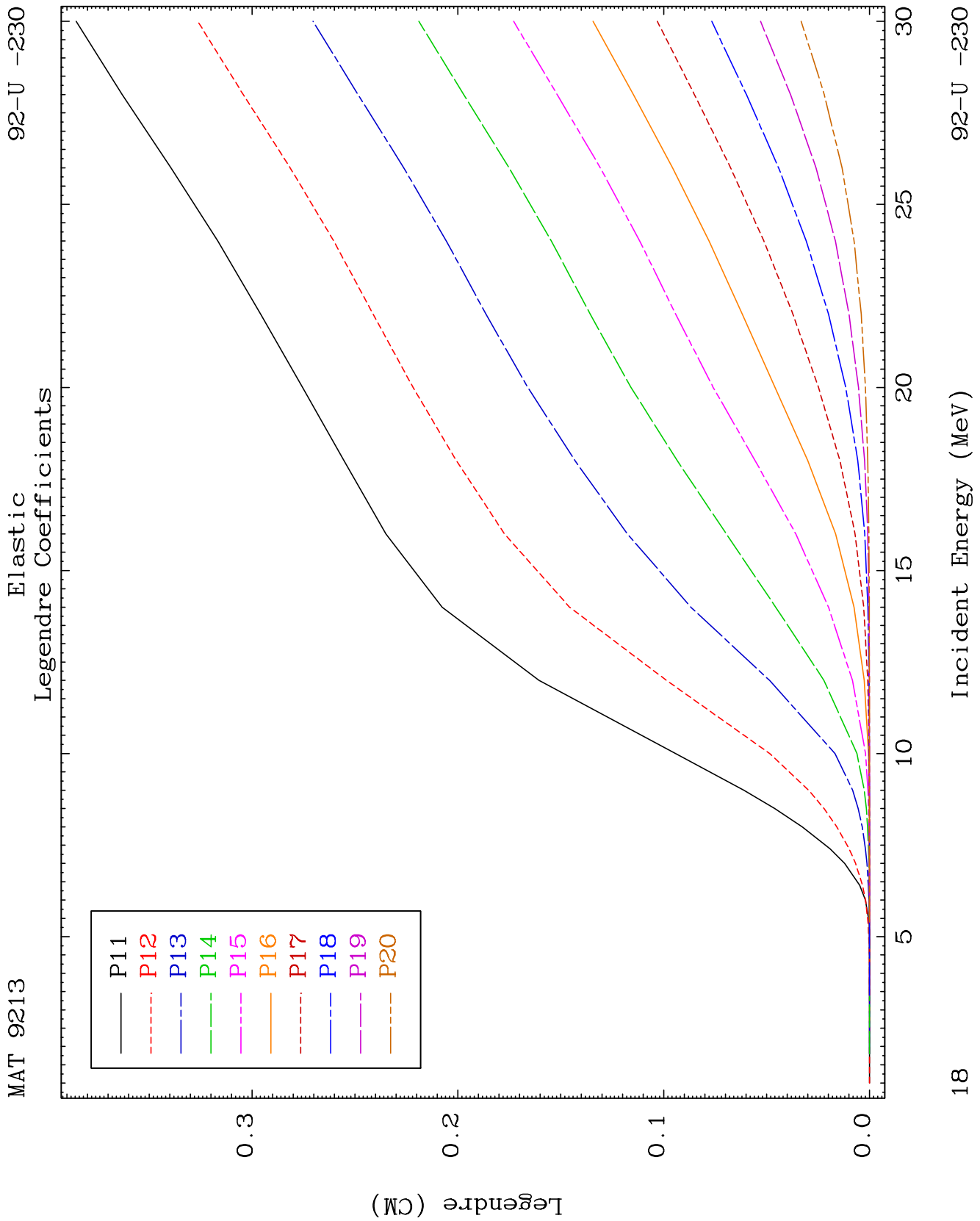
92-U -230







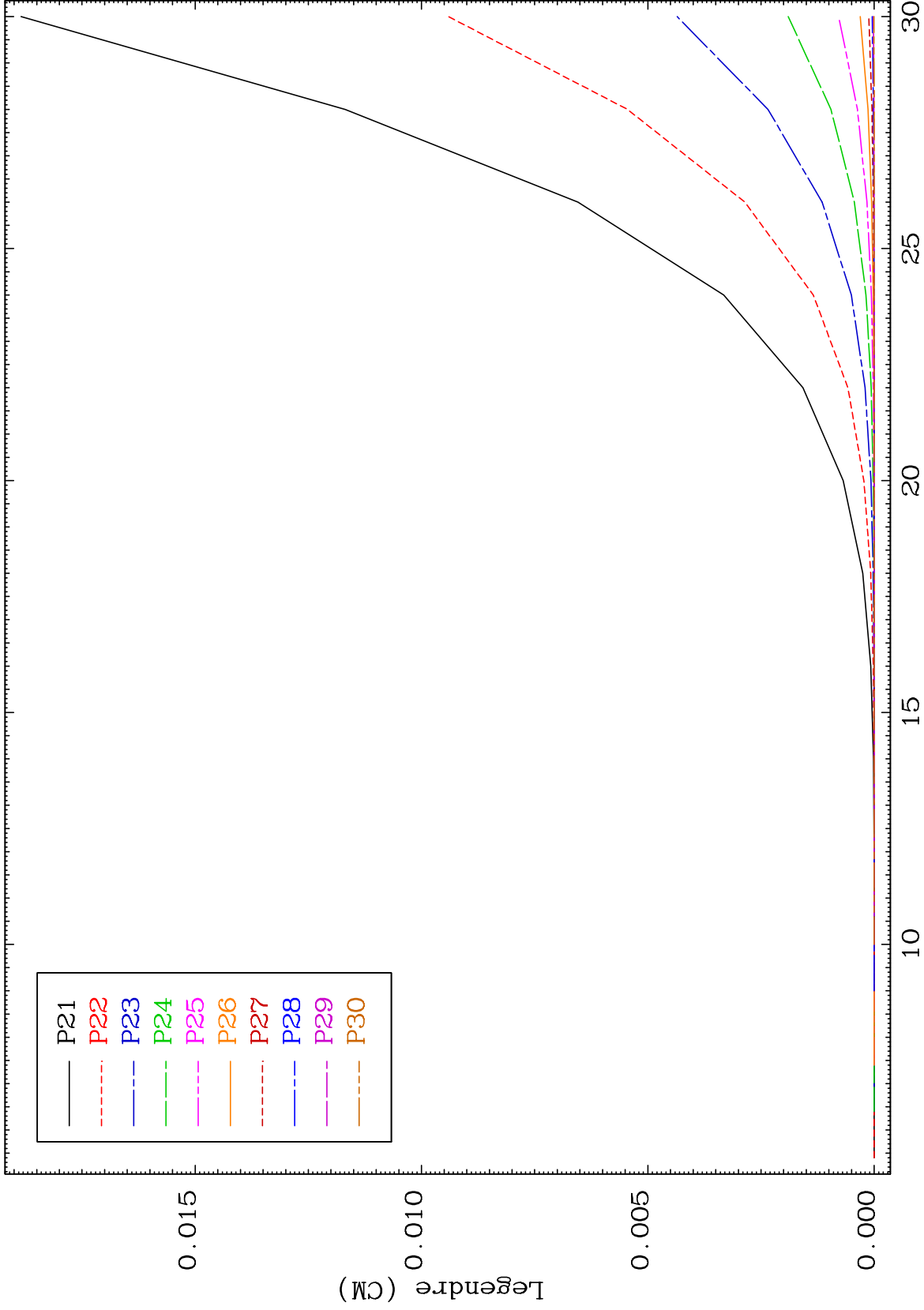




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Elastic Legendre Coefficients

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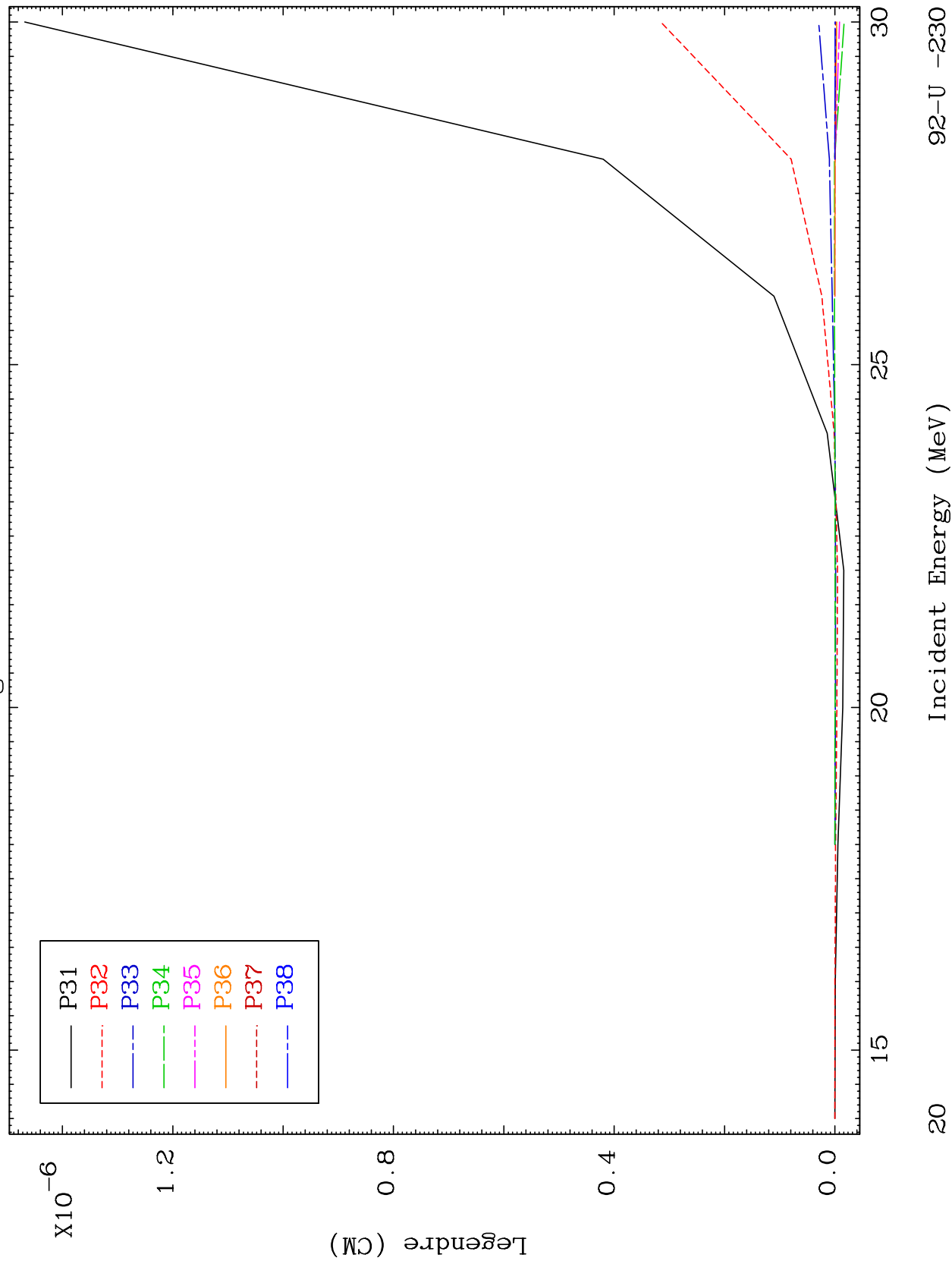
Incident Energy (MeV)

92-U -230

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Elastic Legendre Coefficients

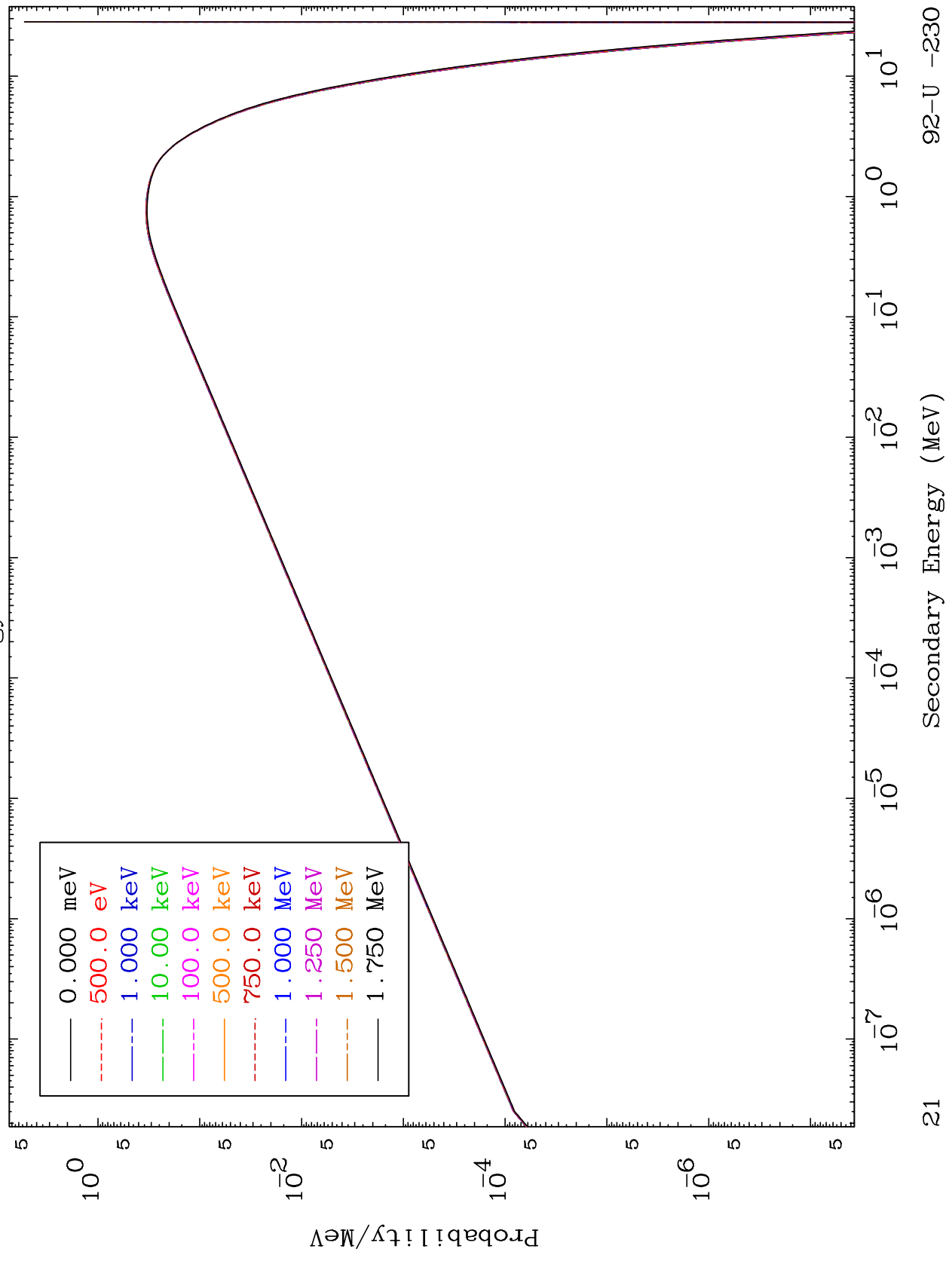
92-U -230



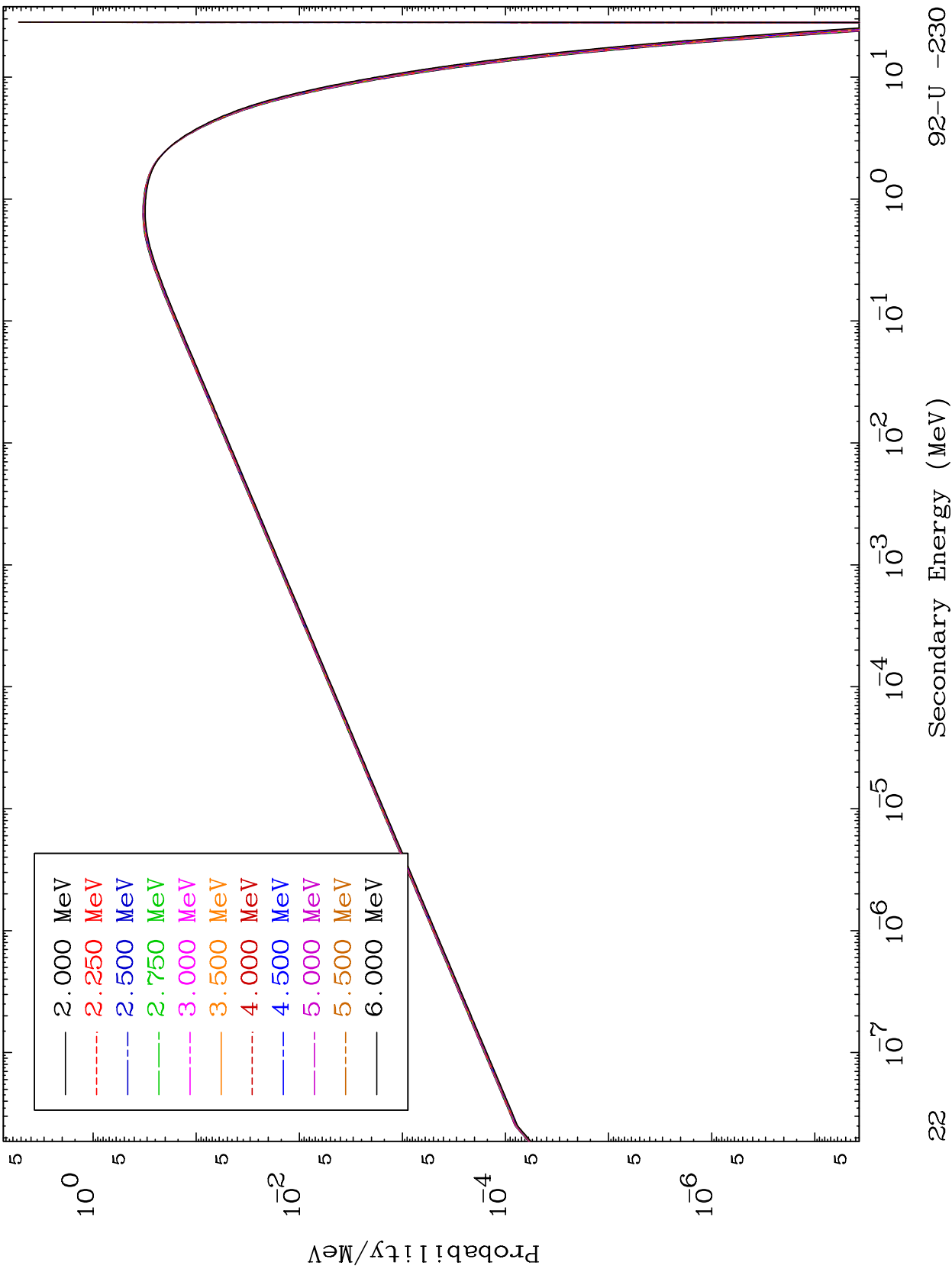
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Fission Energy Distribution

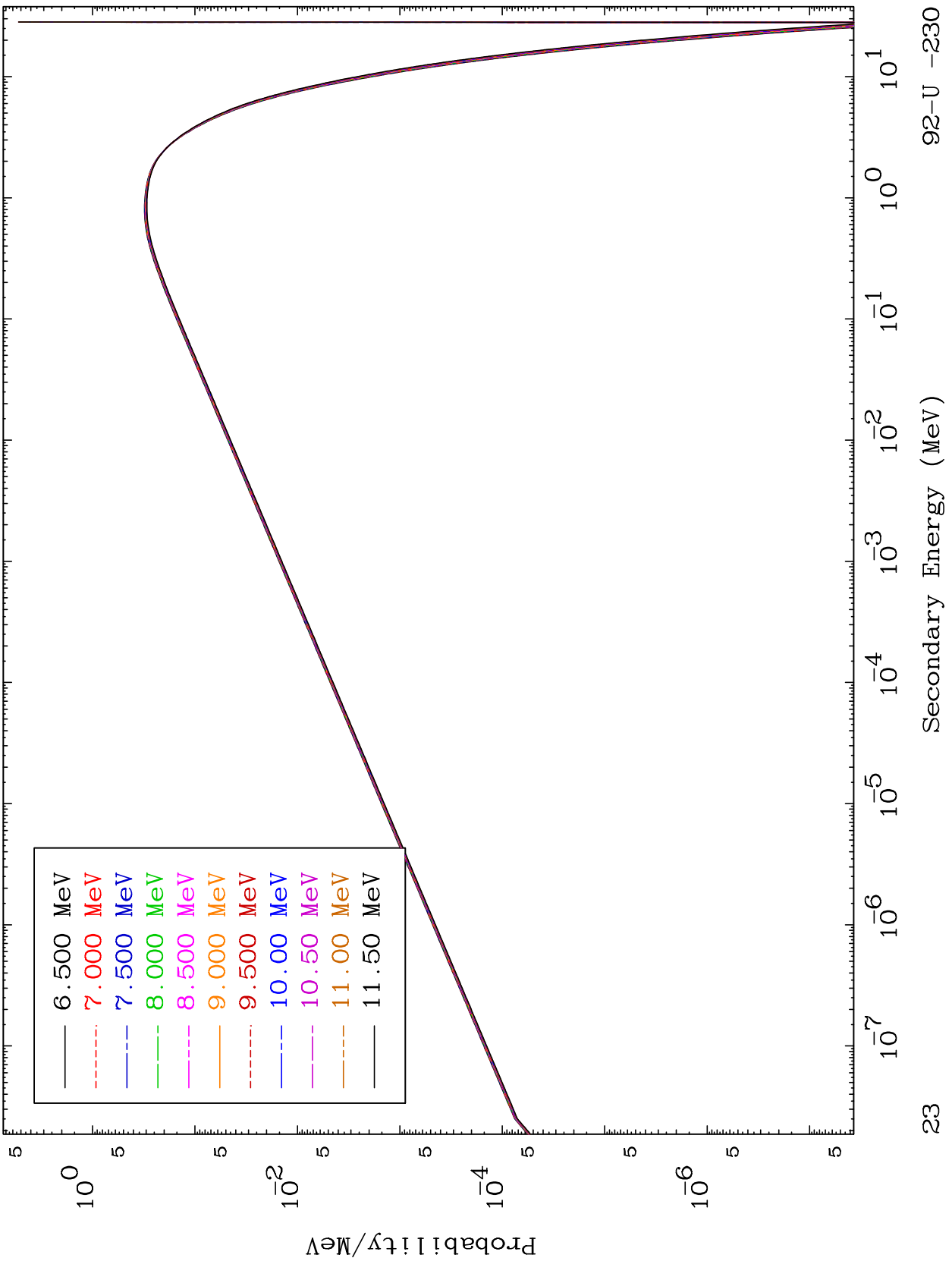
92-U -230



Fission Energy Distribution



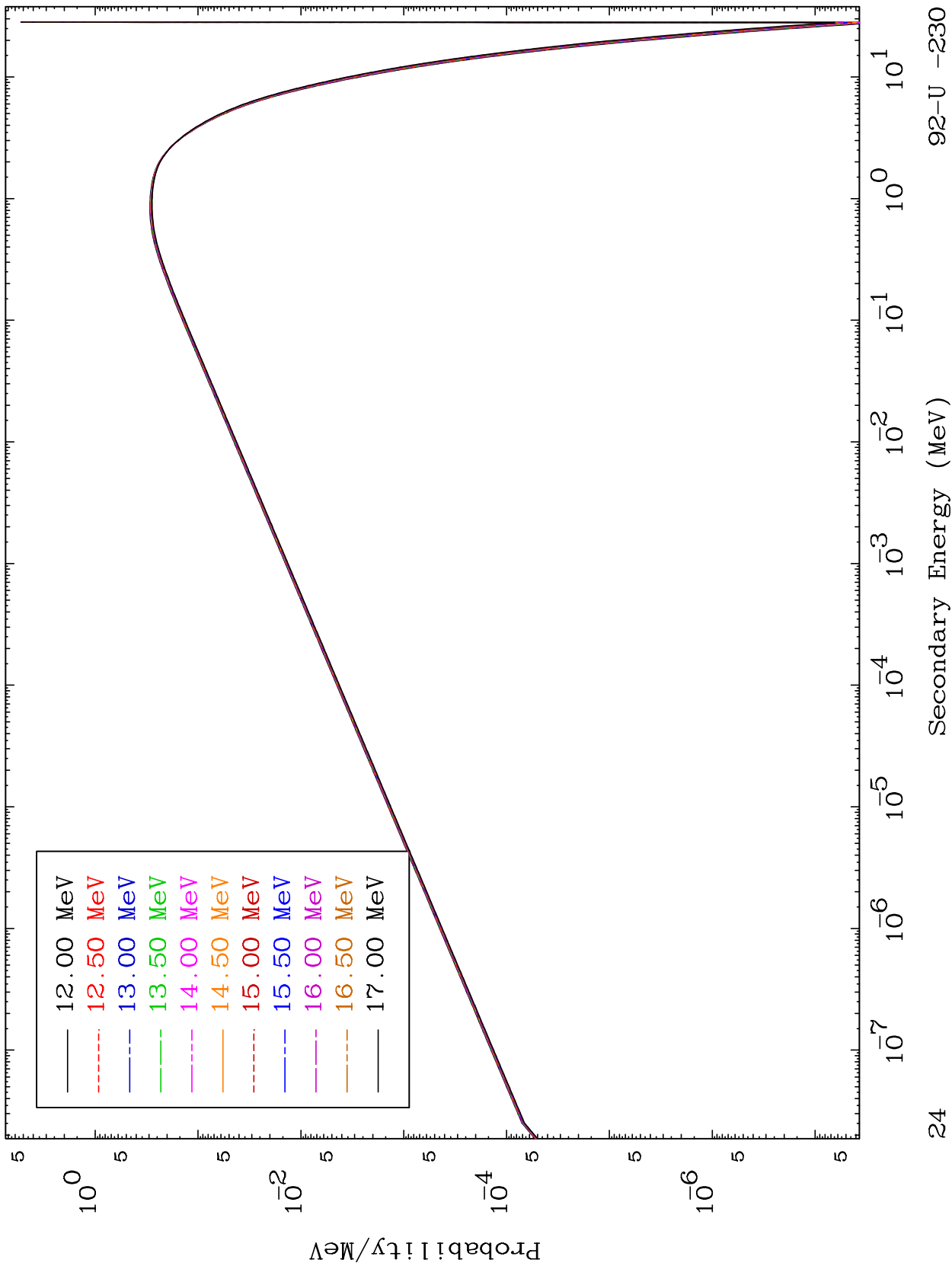
Fission Energy Distribution



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Fission
Energy Distribution

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Fission
Energy Distribution

