

Program EVALPLOT
(Version 2015-2)

by

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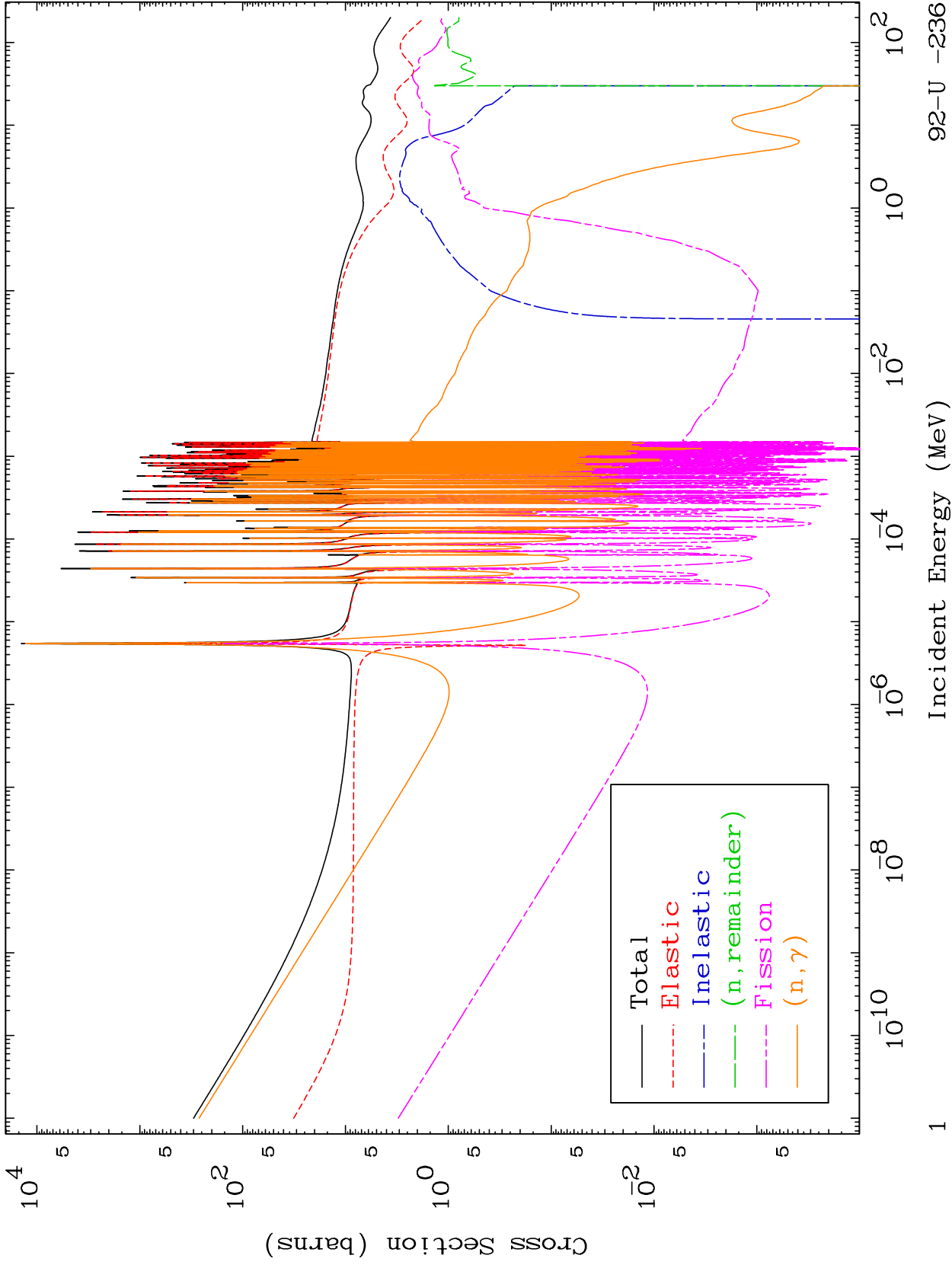
Press Mouse Button to Start

MAT 9231

Major

293 Kelvin Cross Sections

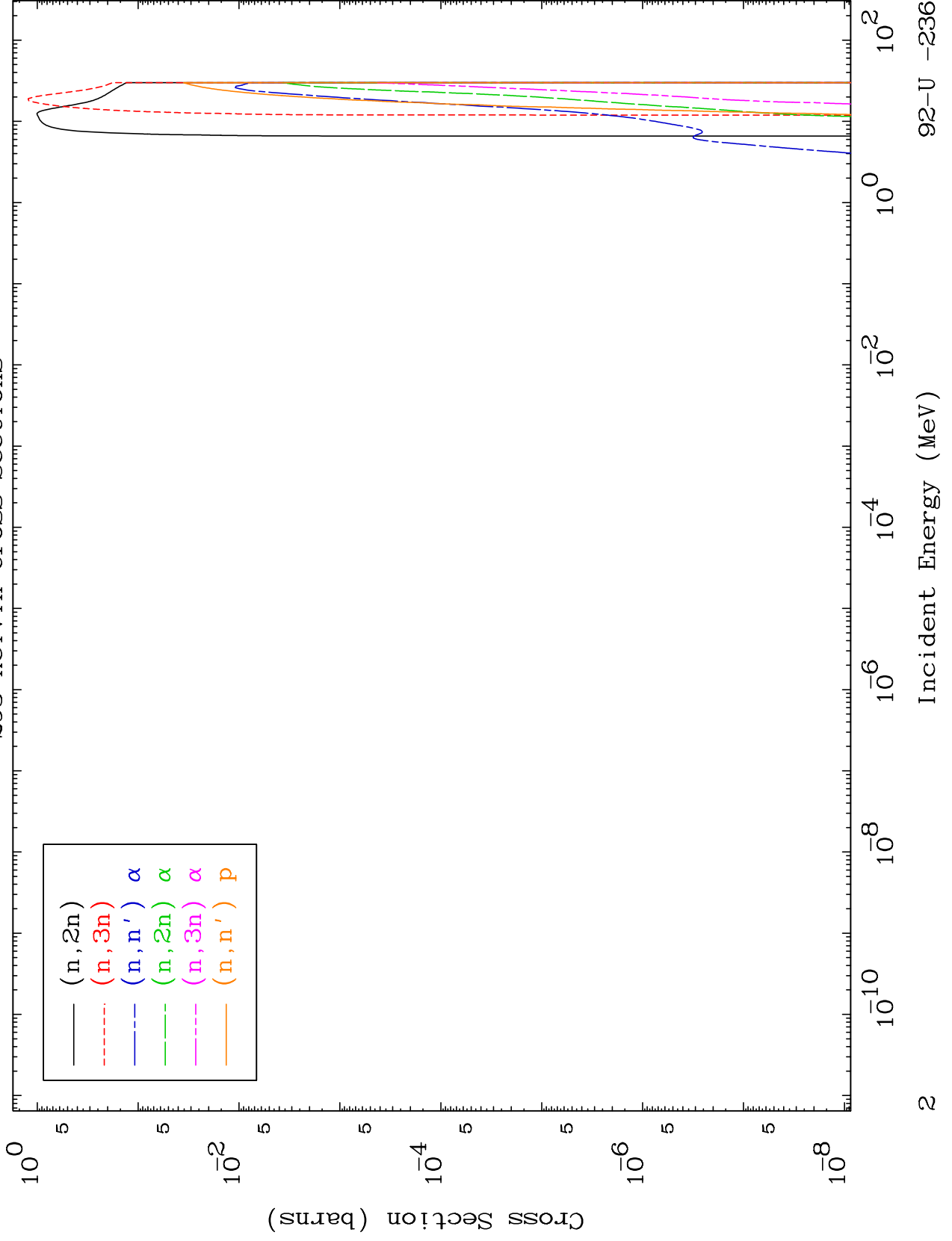
92-U -236

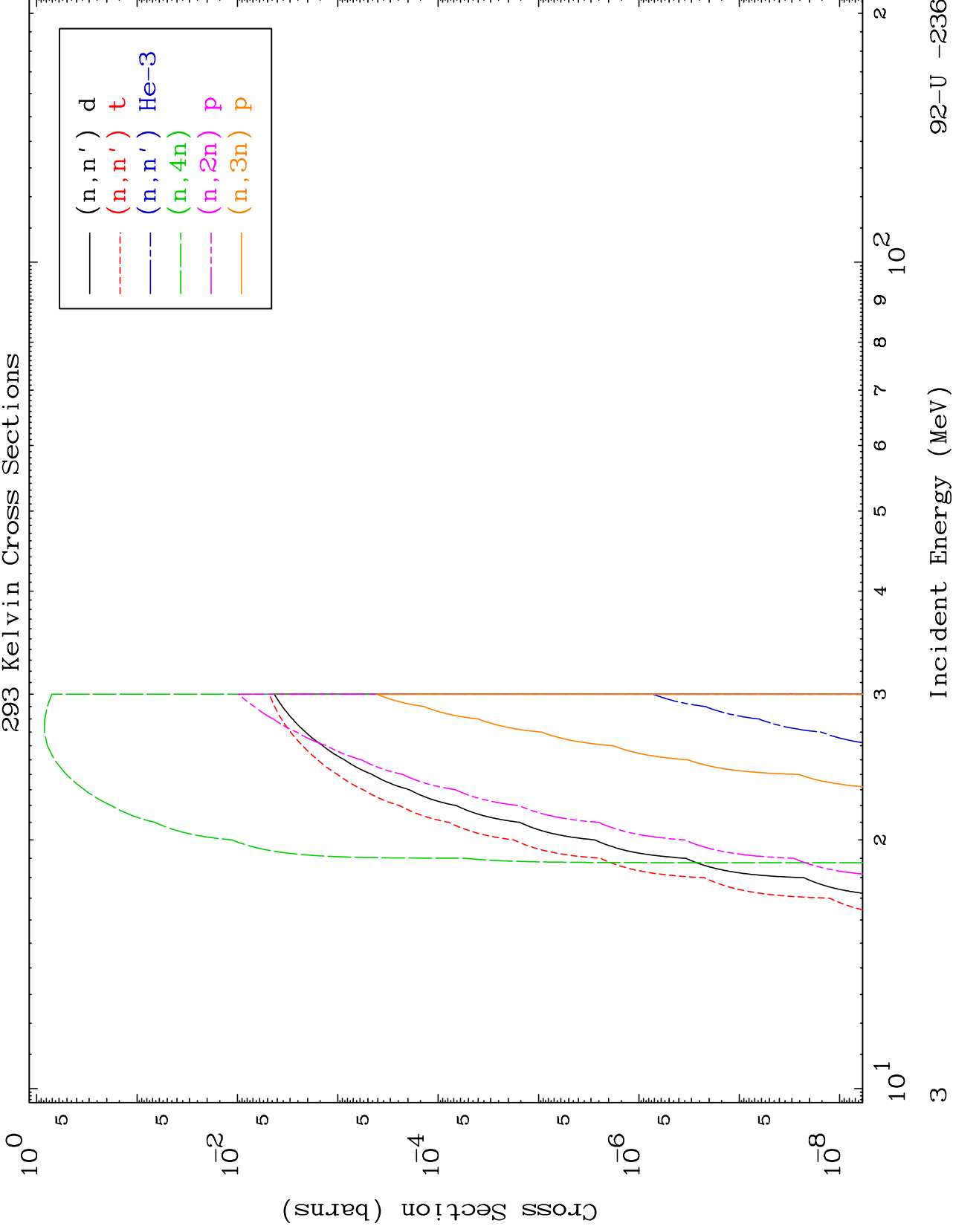


MAT 9231

Neutron Production
293 Kelvin Cross Sections

92-U -236

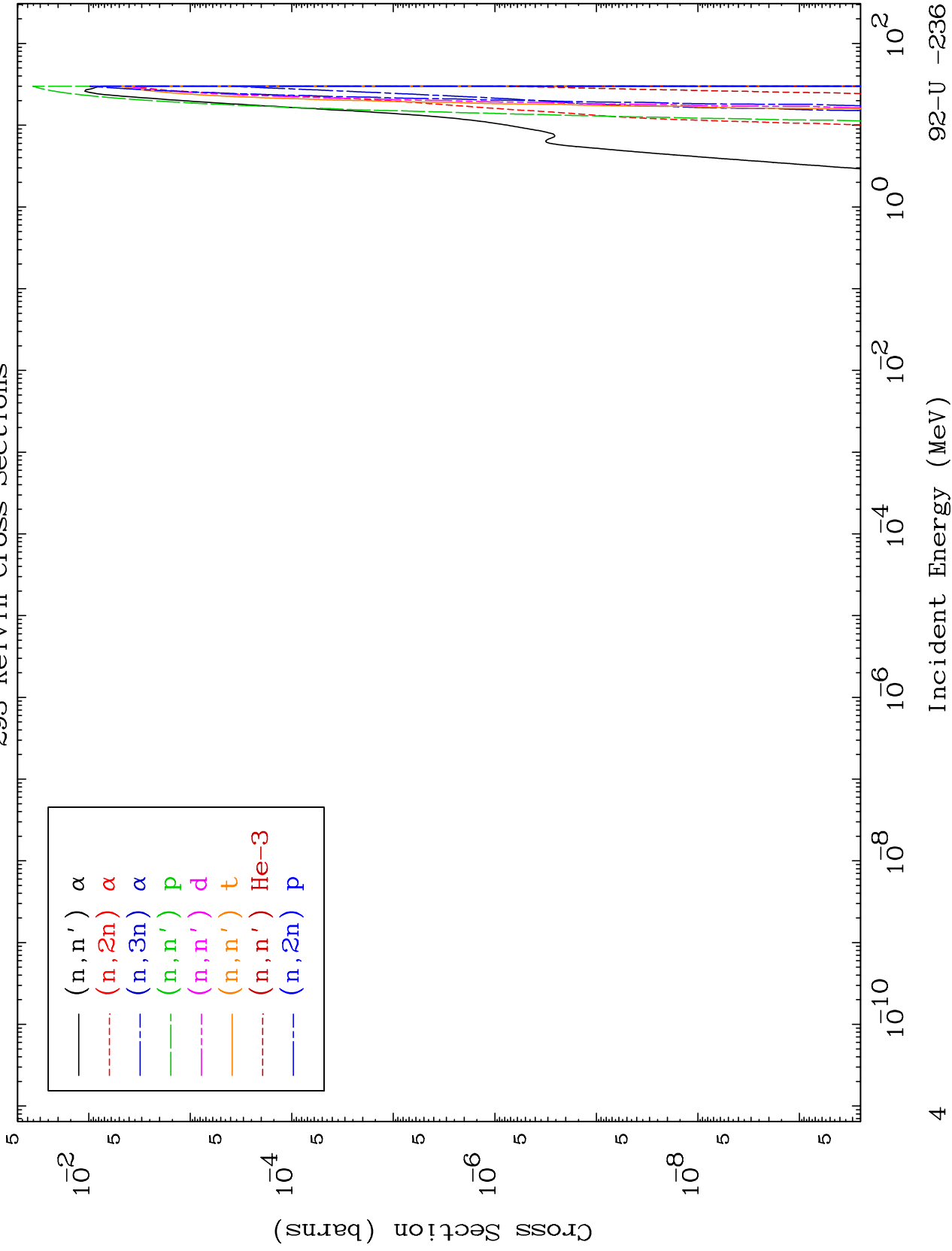




MAT 9231

Charged Particle
293 Kelvin Cross Sections

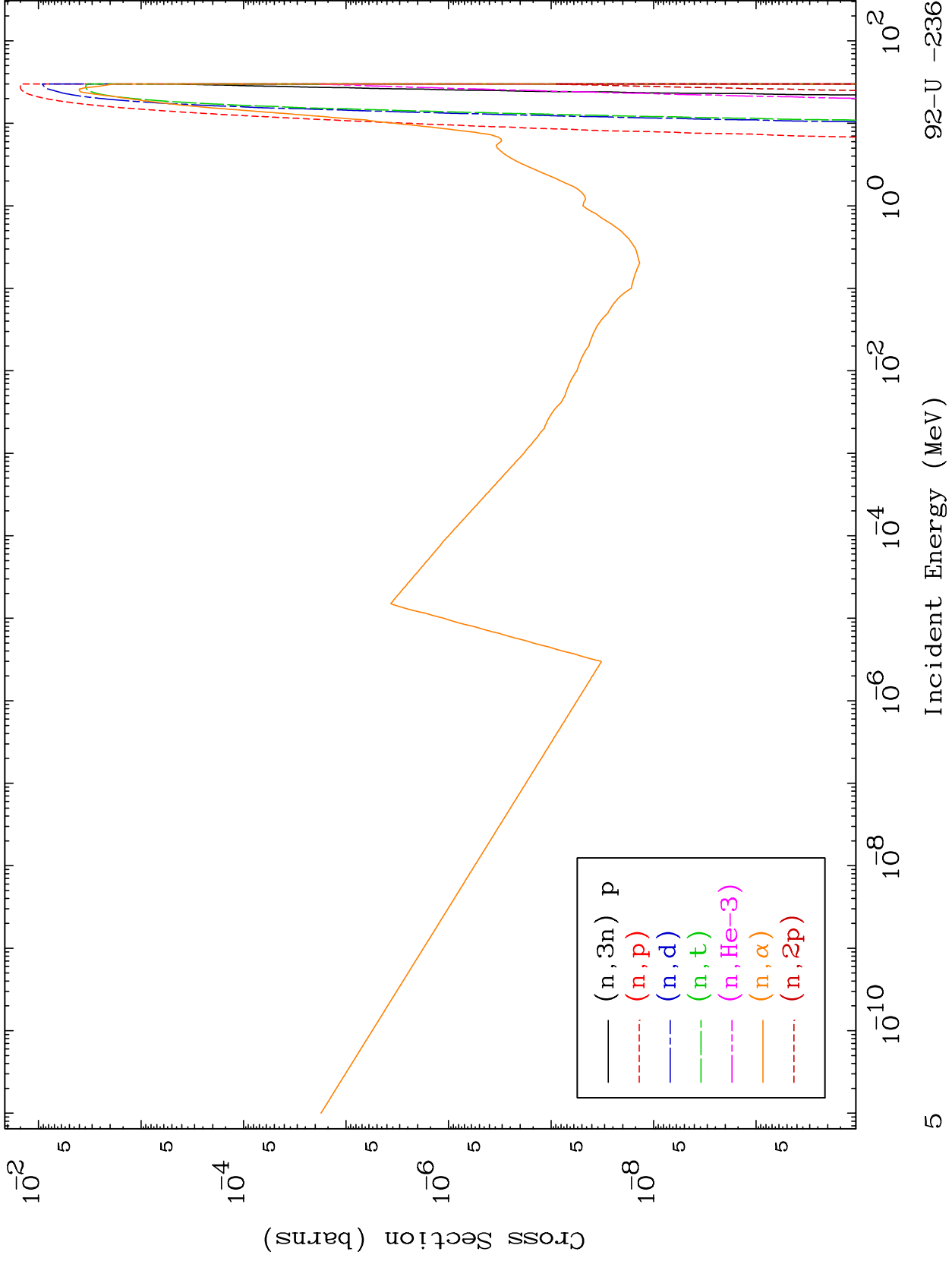
92-U -236



MAT 9231

Charged Particle
293 Kelvin Cross Sections

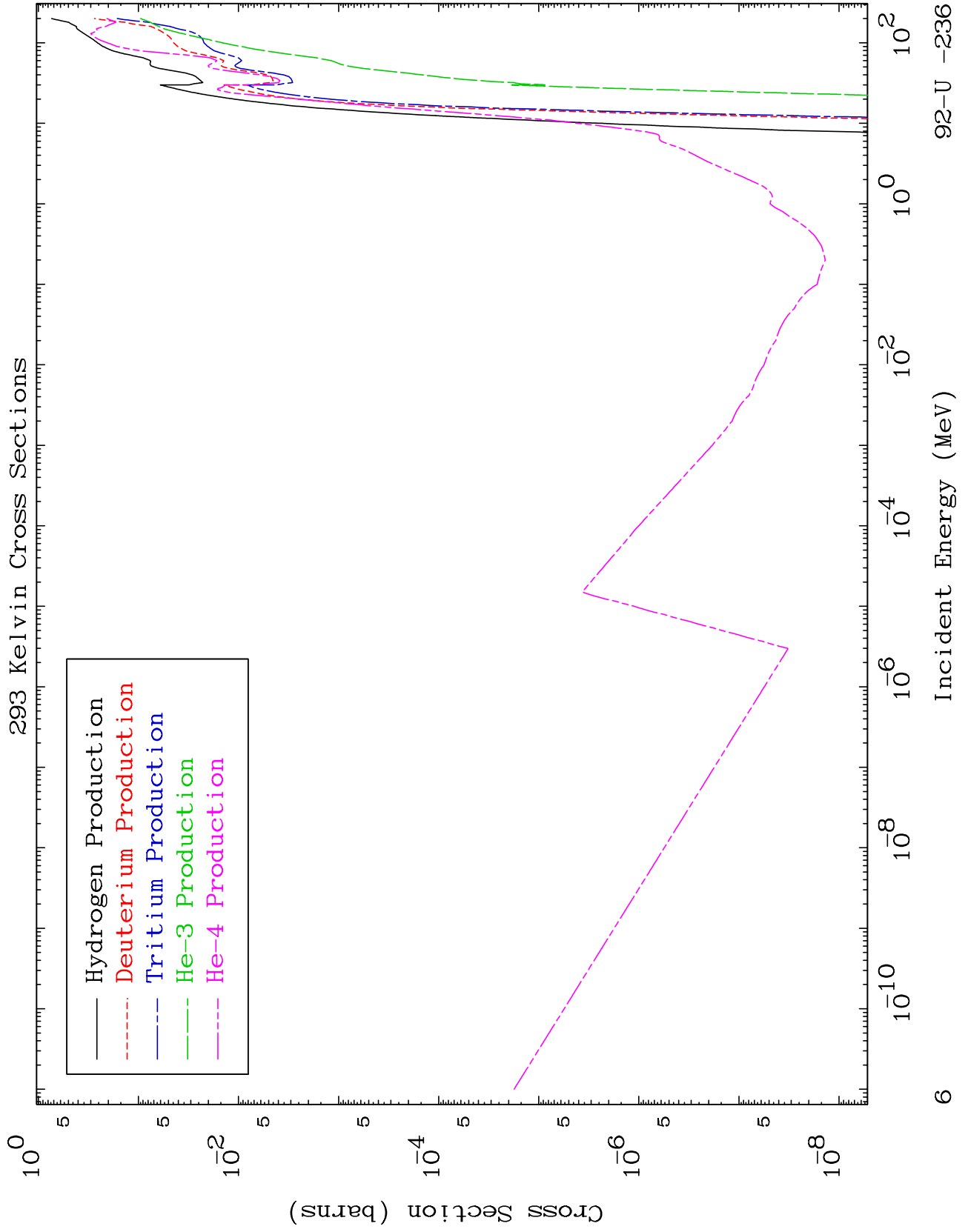
92-U -236



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Particle Production
293 Kelvin Cross Sections

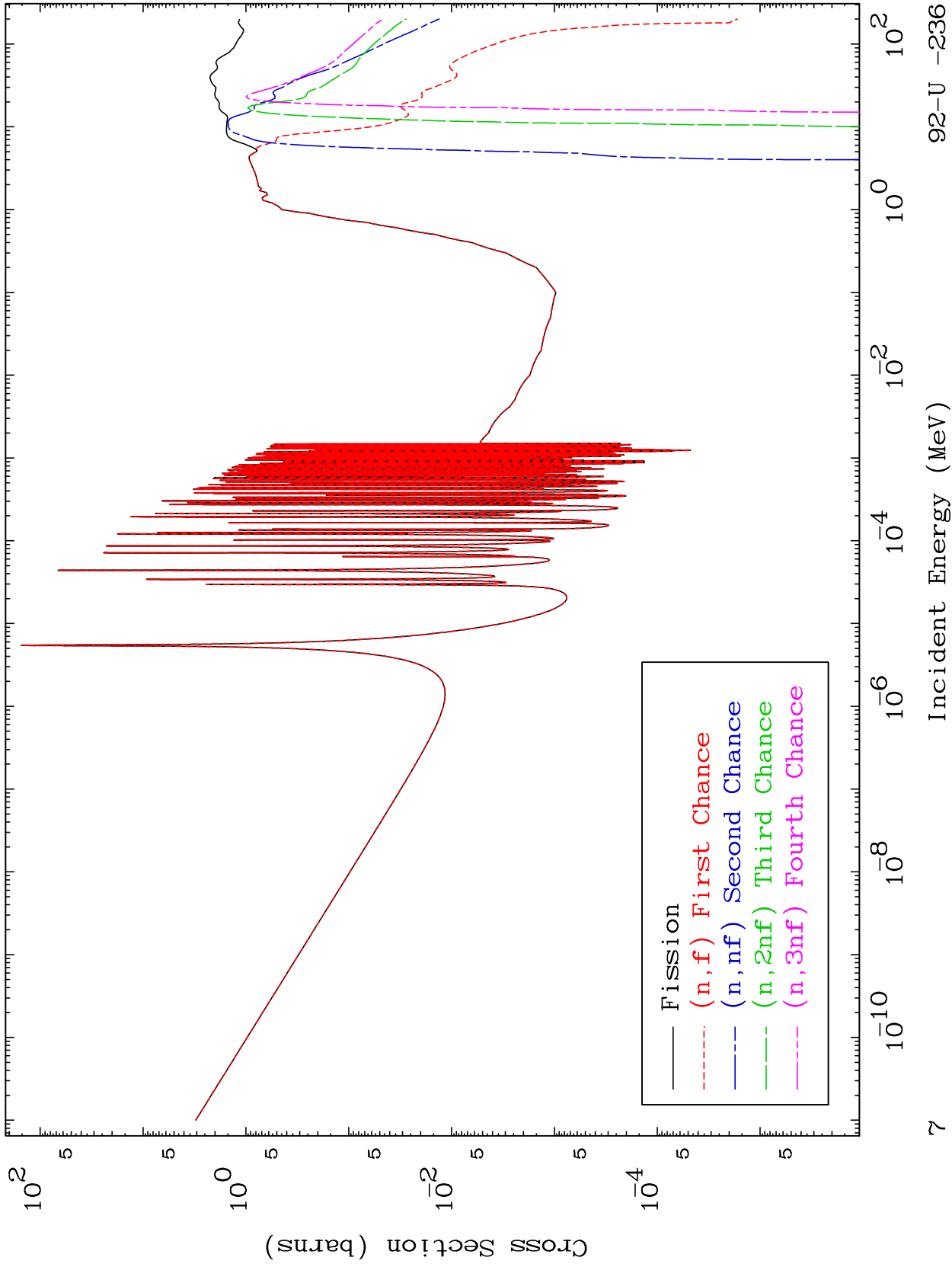
92-U -236



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Fission
293 Kelvin Cross Sections

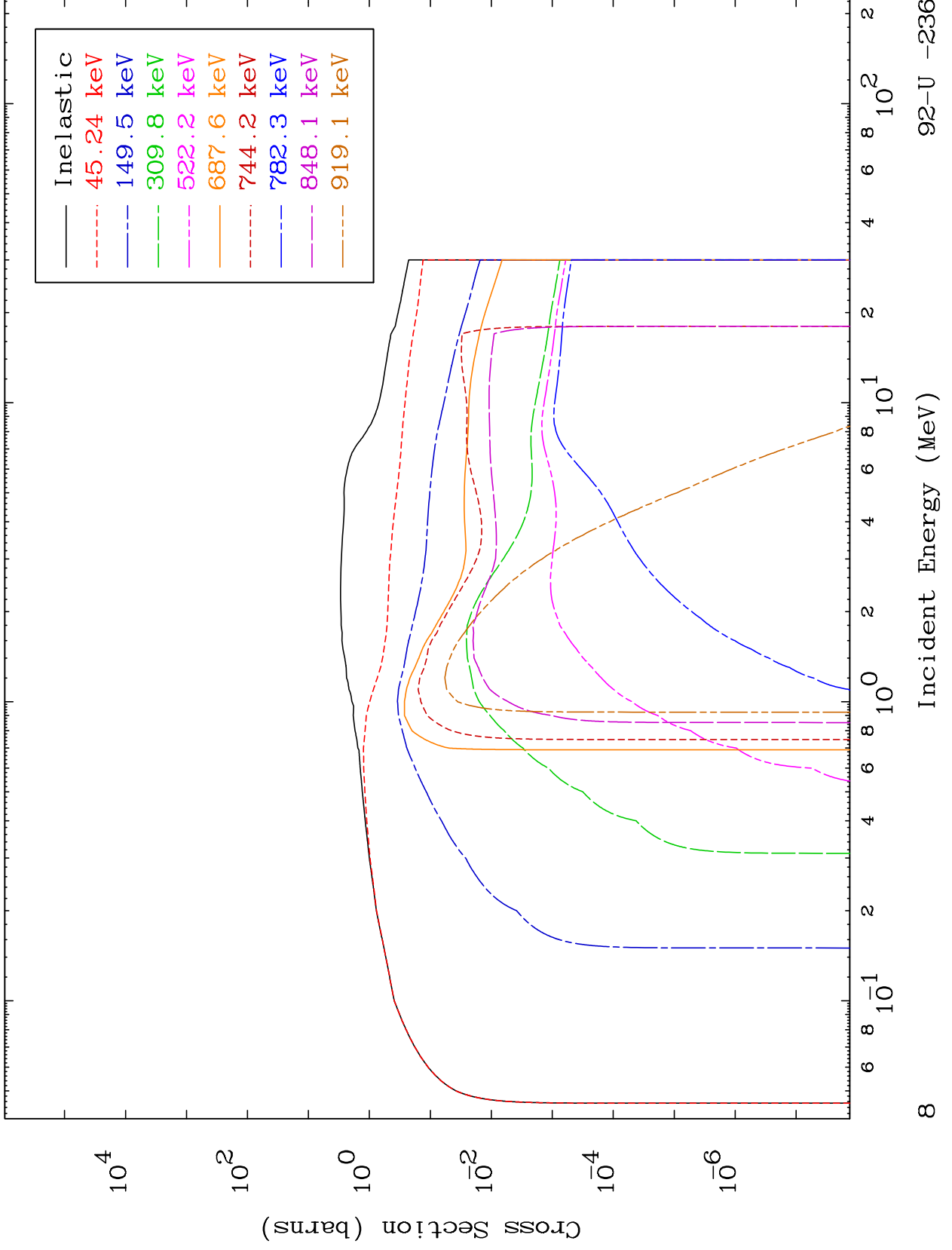
92-U -236



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(n,n') Level
293 Kelvin Cross Sections

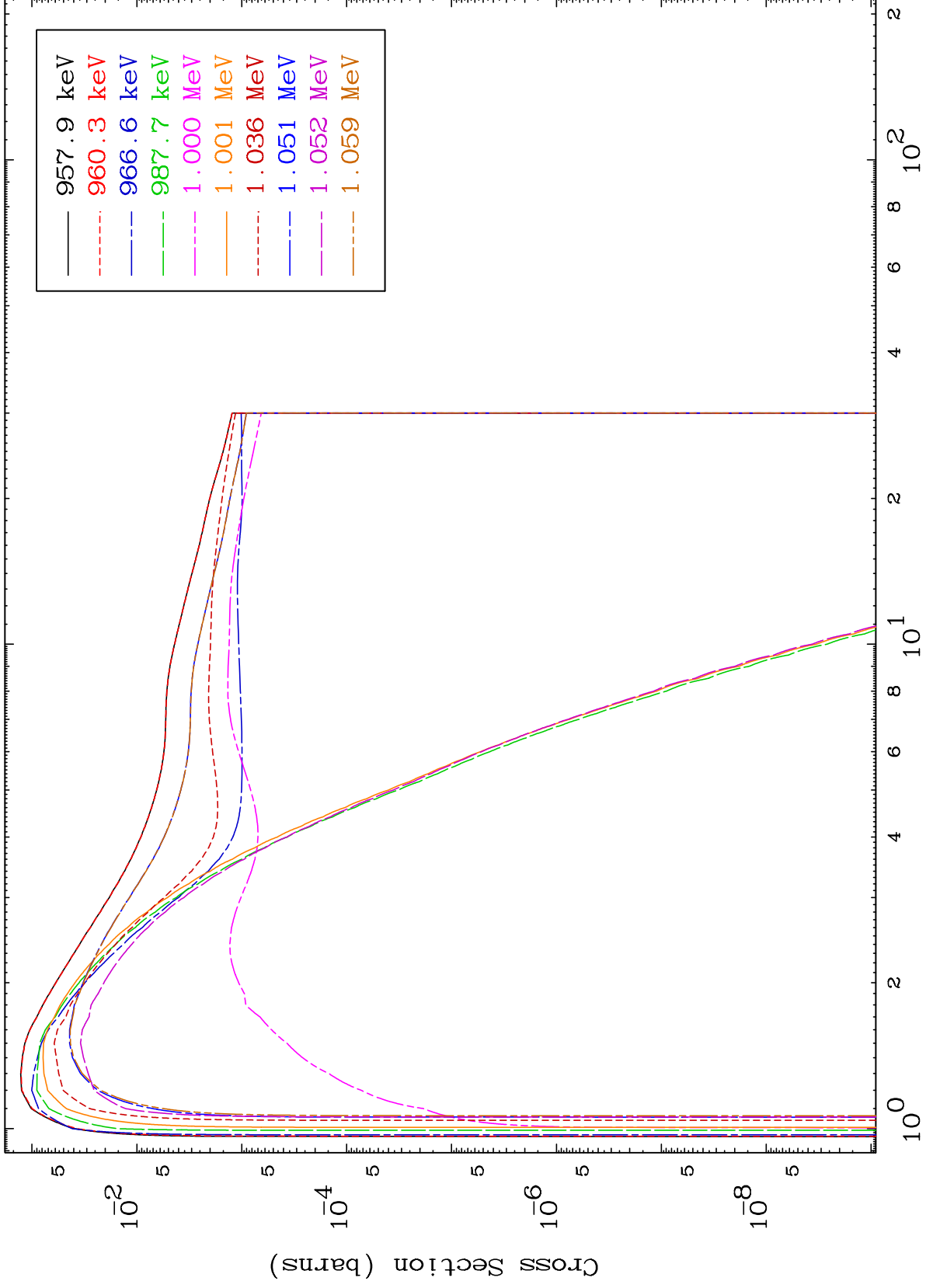
92-U -236



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(n,n') Level
293 Kelvin Cross Sections

92-U -236



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Incident Energy (MeV)

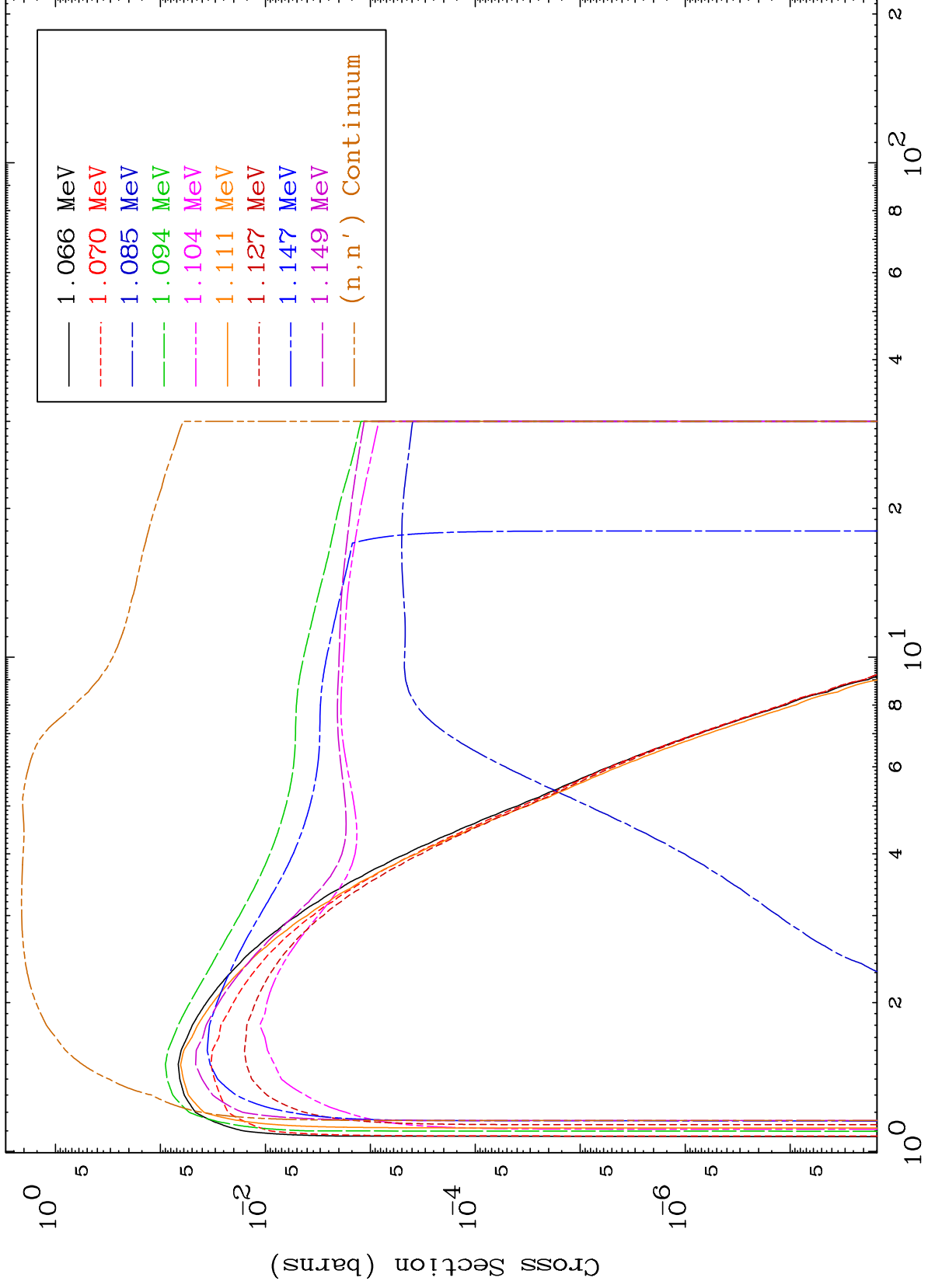
92-U -236

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(n,n') Level

92-U -236

293 Kelvin Cross Sections



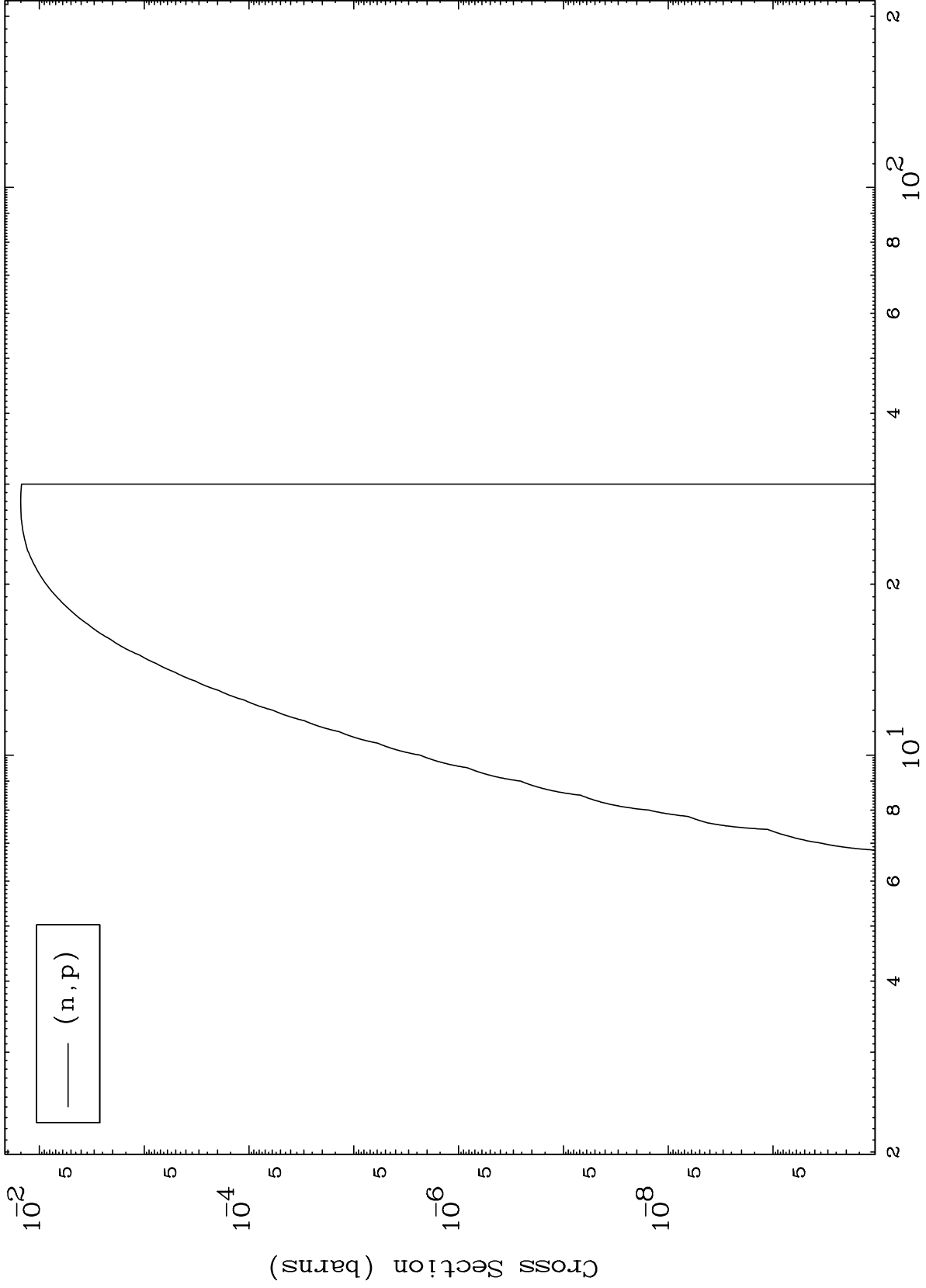
Incident Energy (MeV)

92-U -236

MAT 9231

(n,p) Levels
293 Kelvin Cross Sections

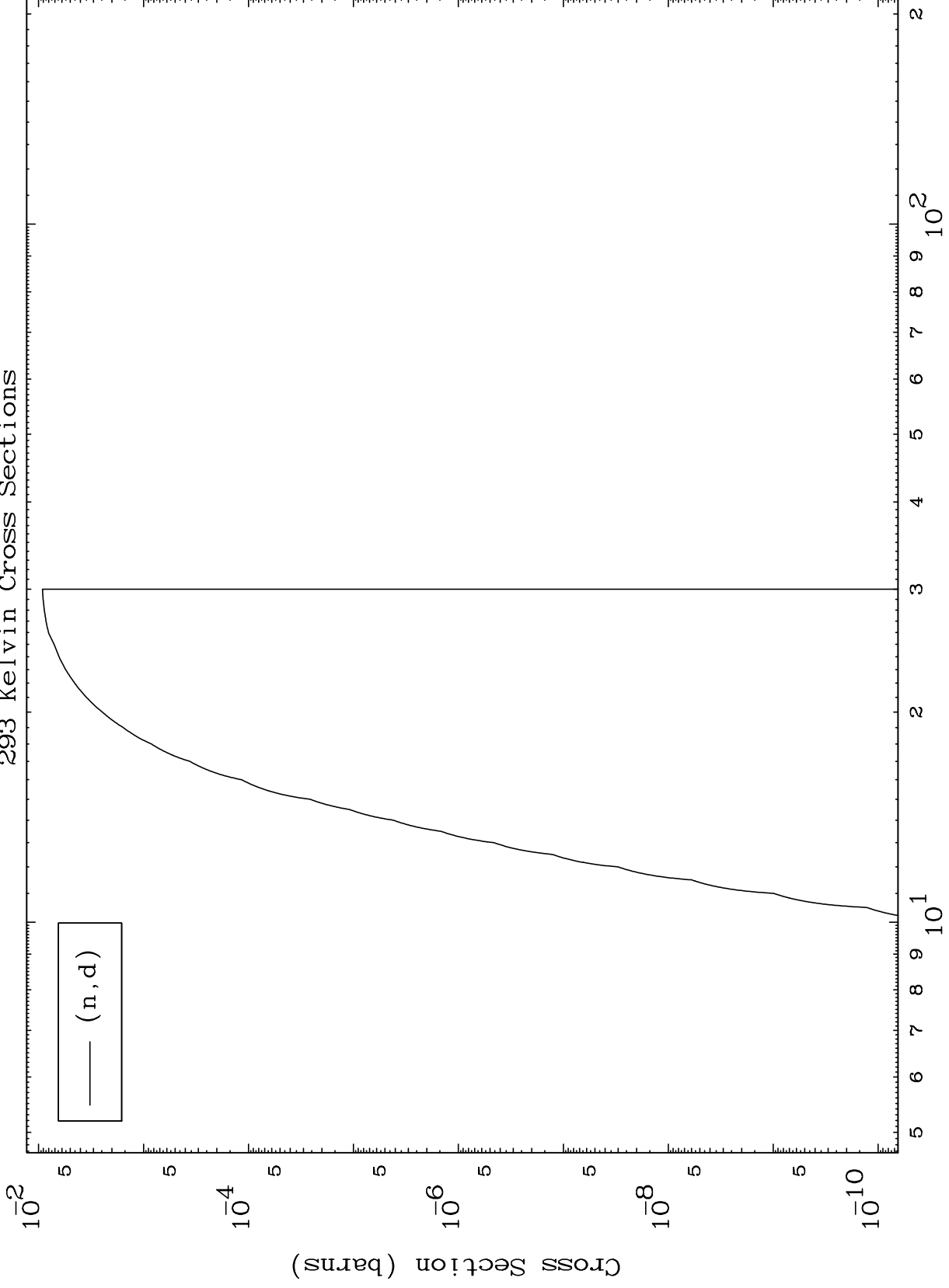
92-U -236



MAT 9231

(n,d) Levels
293 Kelvin Cross Sections

92-U -236



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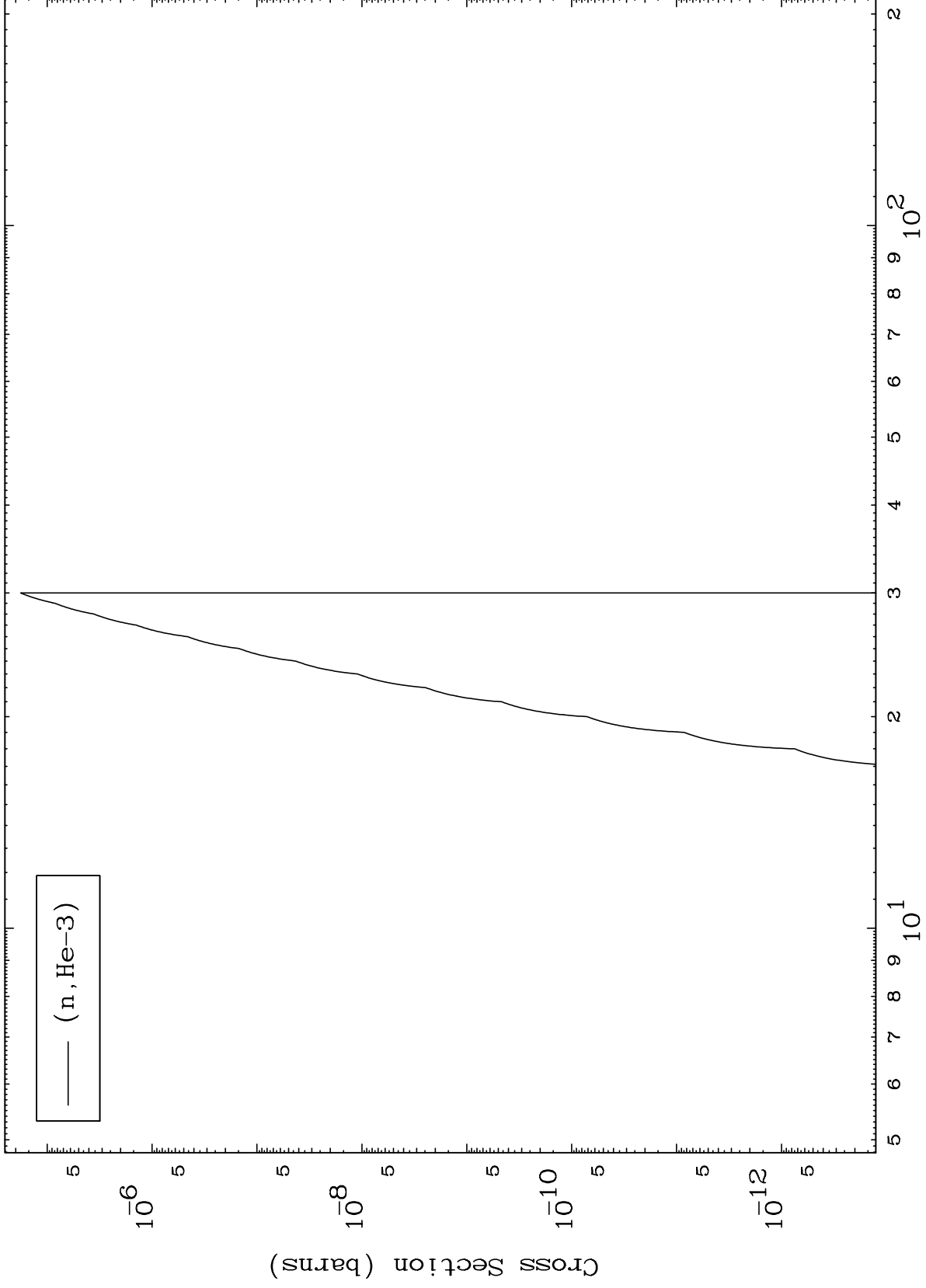
Incident Energy (MeV)

92-U -236

MAT 9231

(n,He3) Levels
293 Kelvin Cross Sections

92-U -236



14

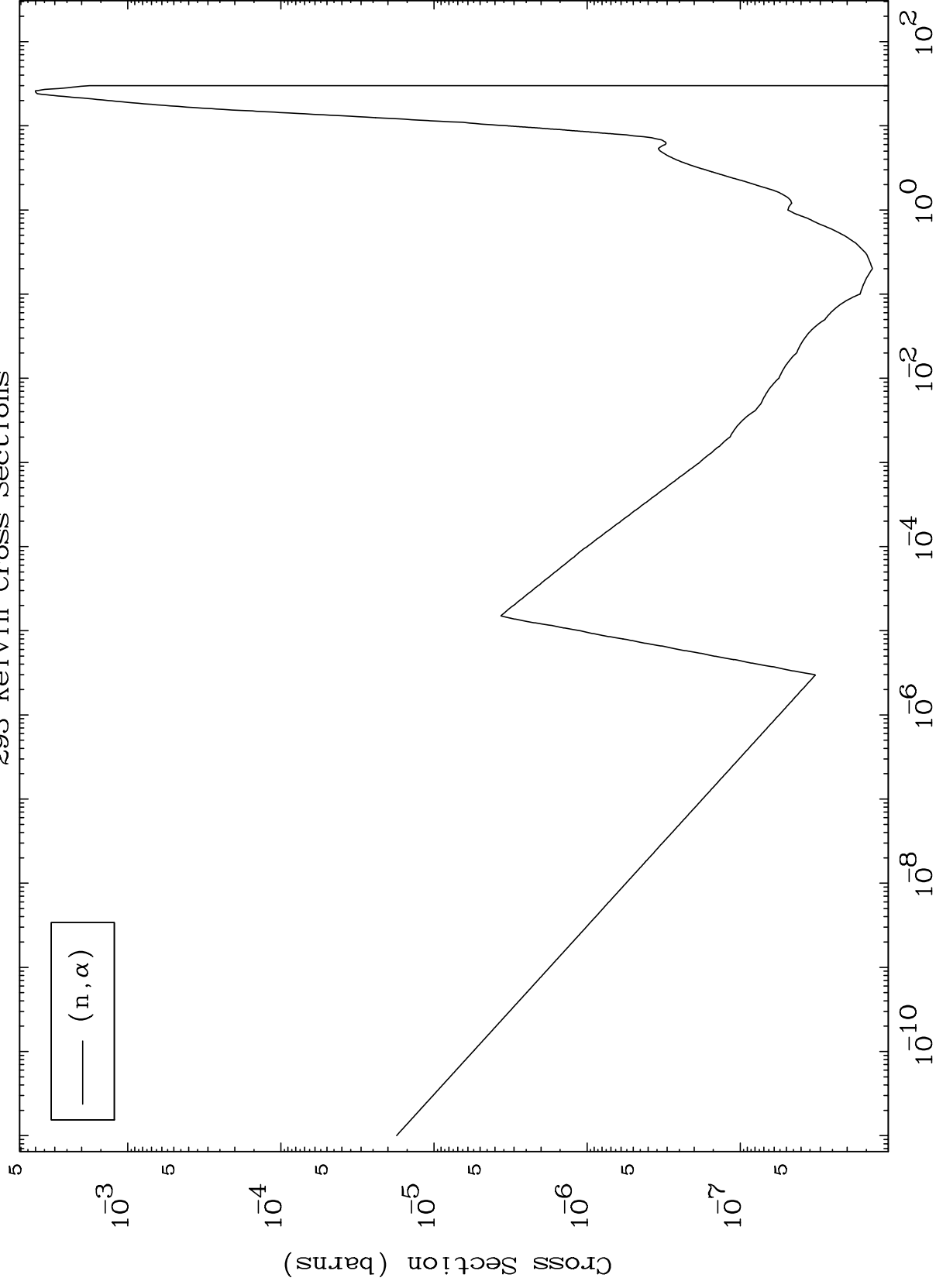
Incident Energy (MeV)

92-U -236

MAT 9231

(n, α) Levels
293 Kelvin Cross Sections

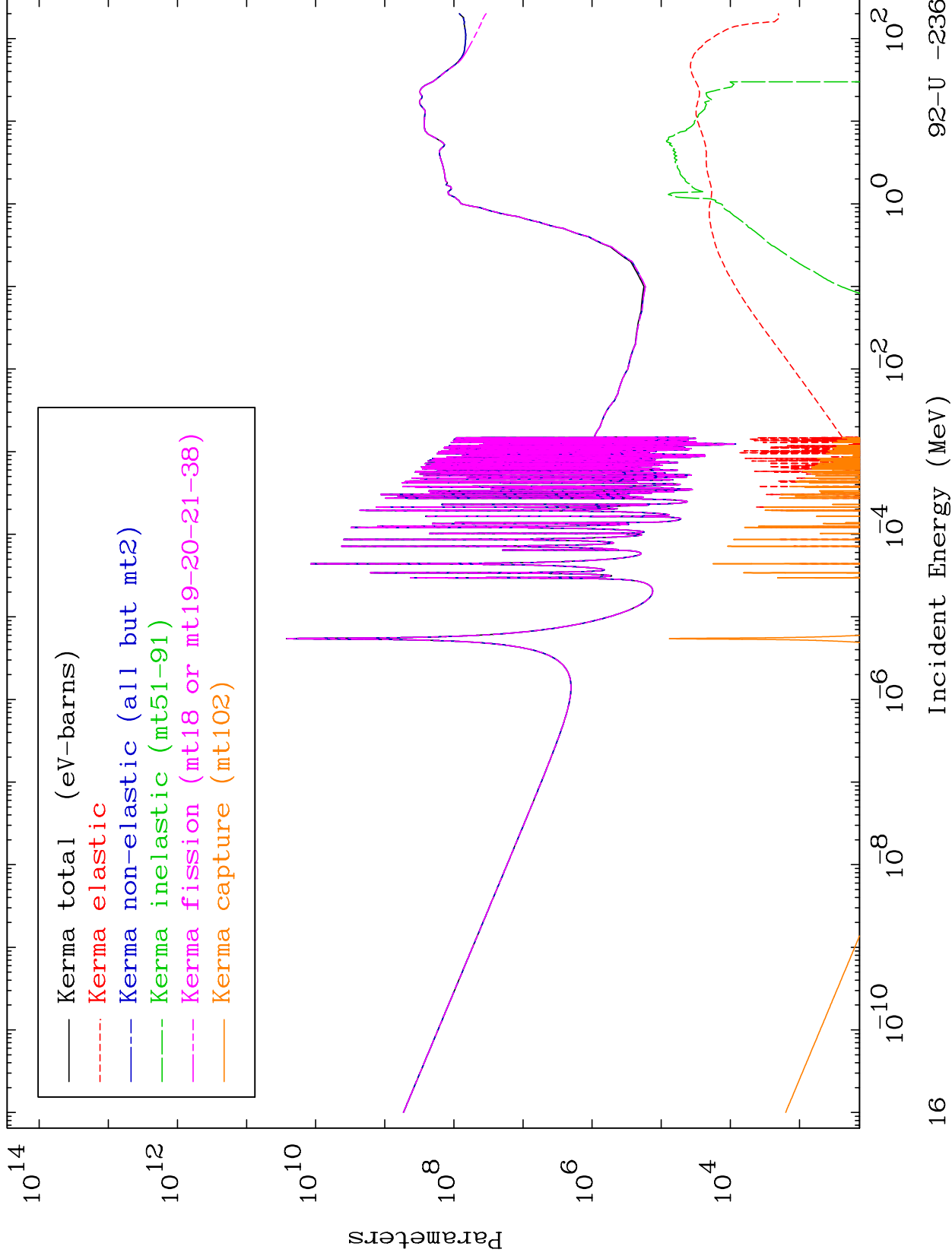
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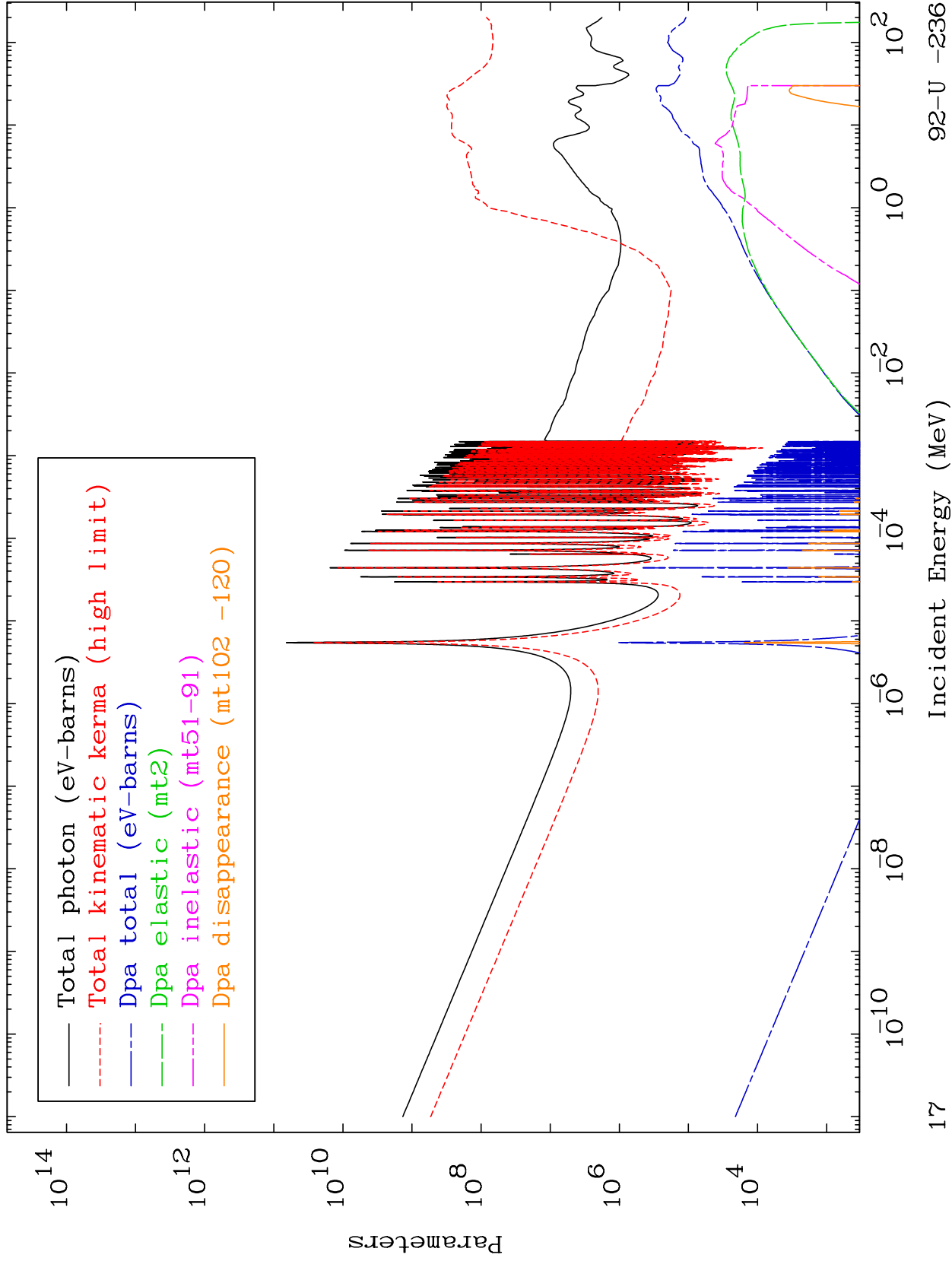


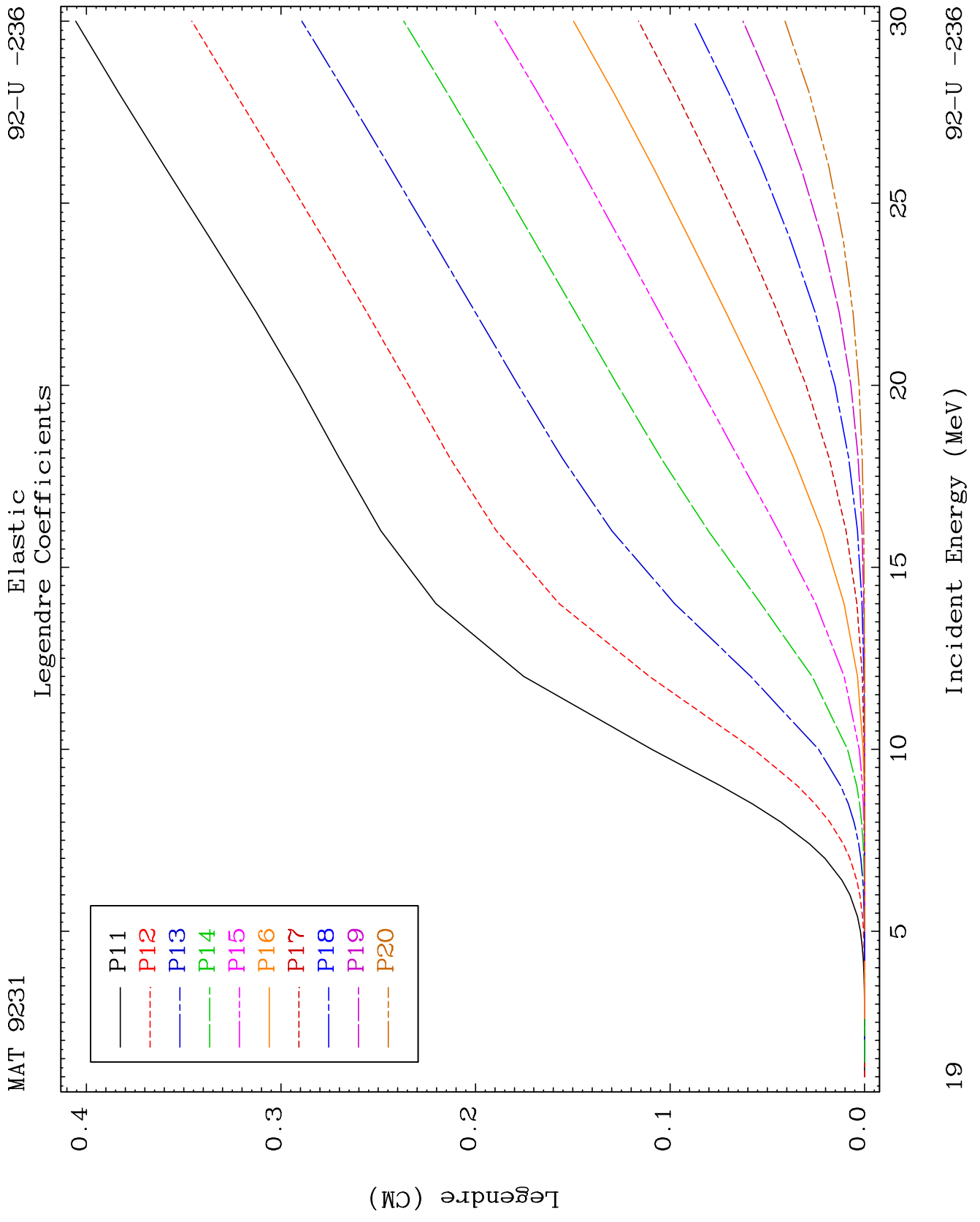
15

Incident Energy (MeV)

92-U -236



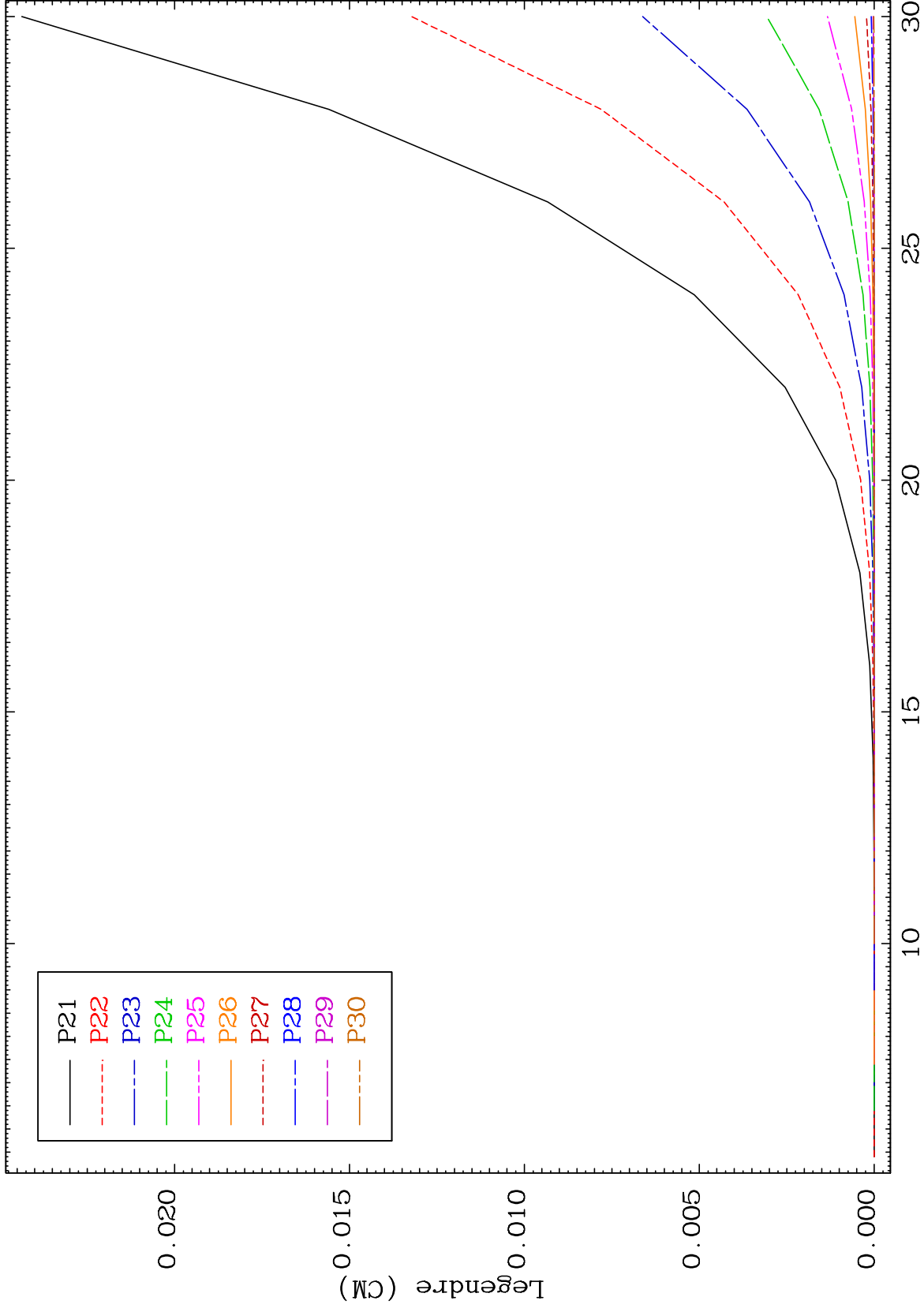




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Elastic Legendre Coefficients

92-U -236



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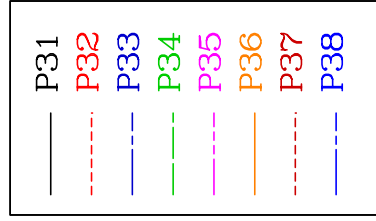
Incident Energy (MeV)

92-U -236

MAT 9231

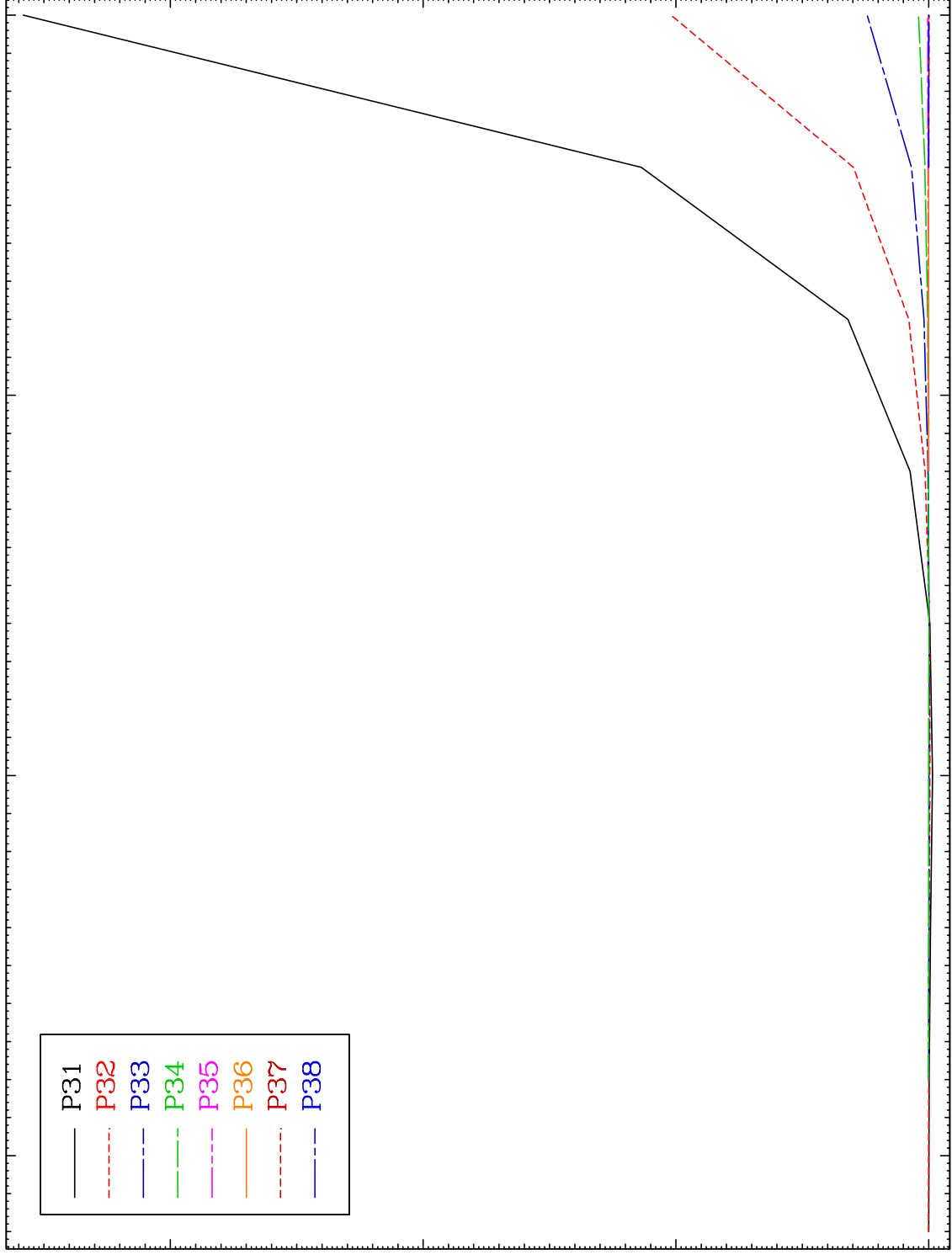
Elastic Legendre Coefficients

92-U -236



$\times 10^{-6}$

Legendre (CM)

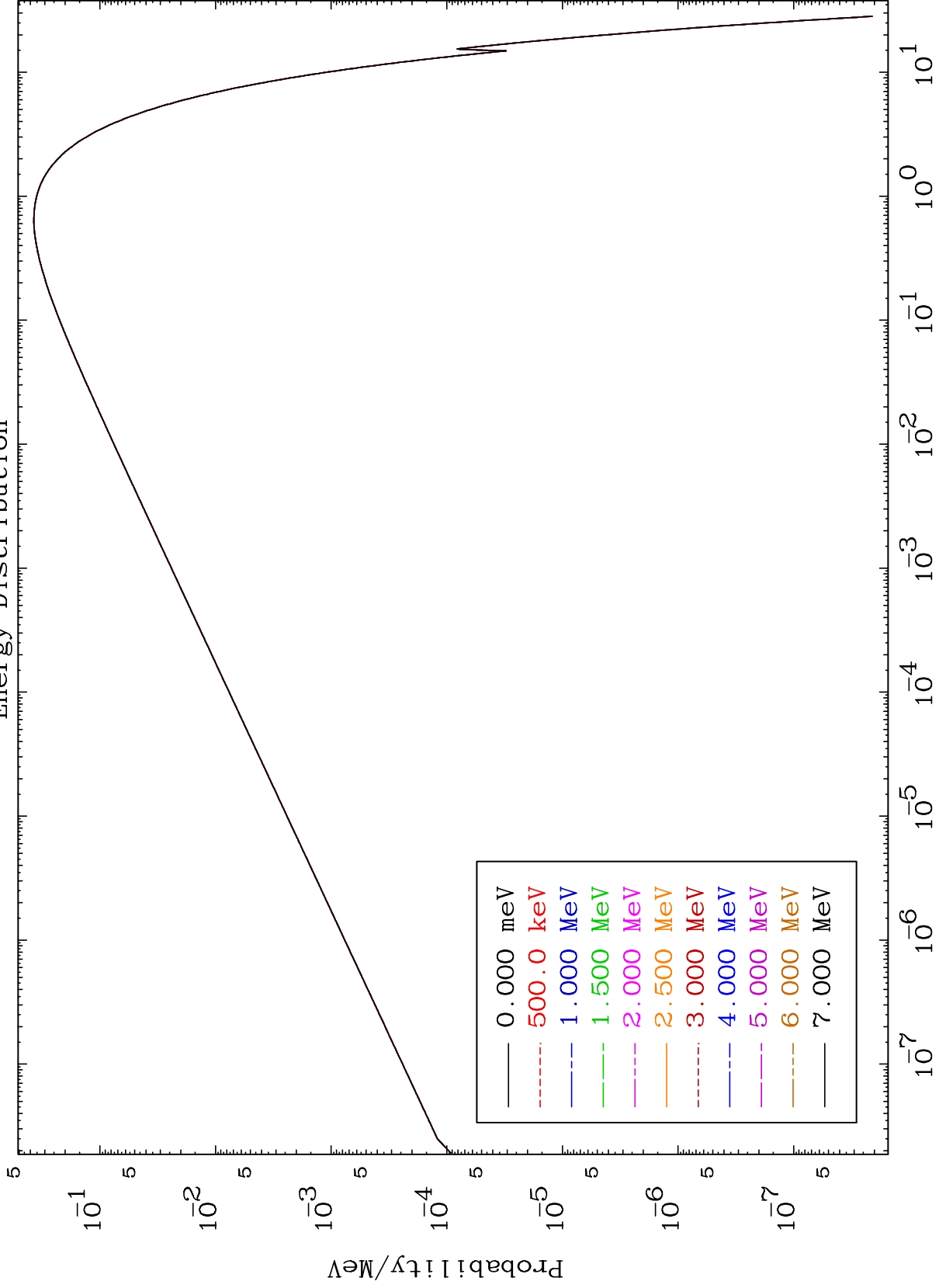


21

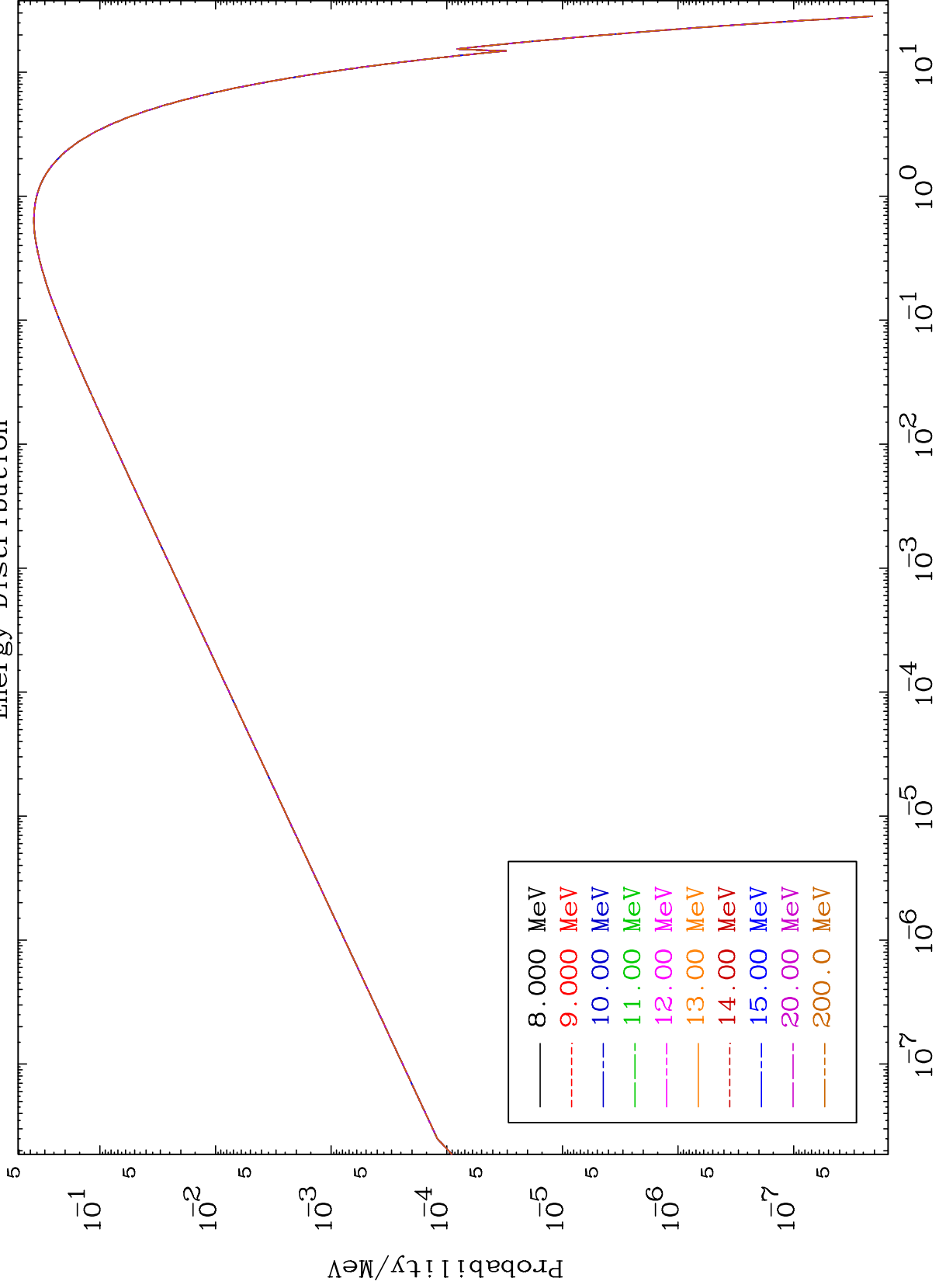
Incident Energy (MeV)

92-U -236

Fission Energy Distribution



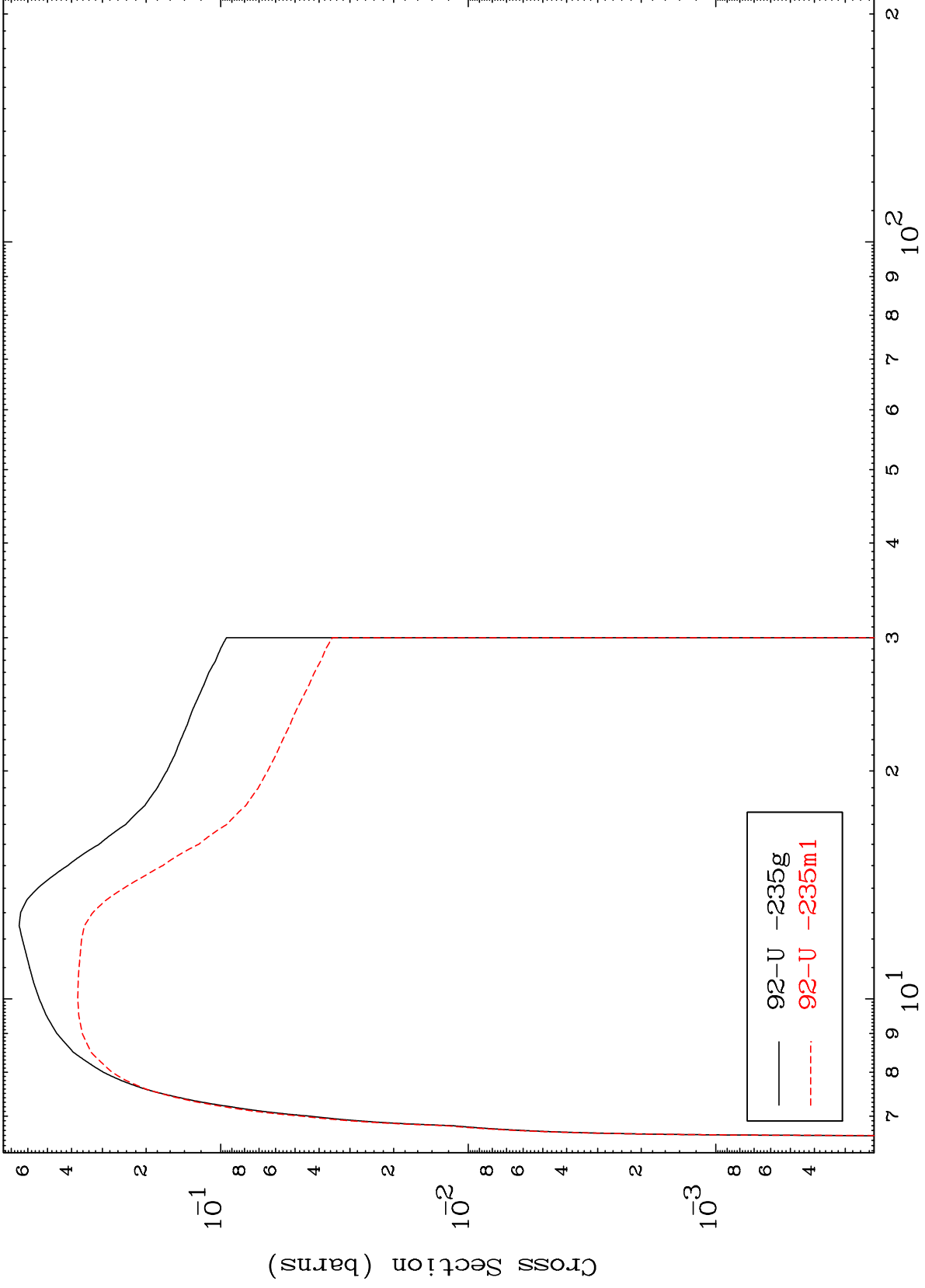
Fission
Energy Distribution



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(n,2n)
Radionuclide Production Cross Section



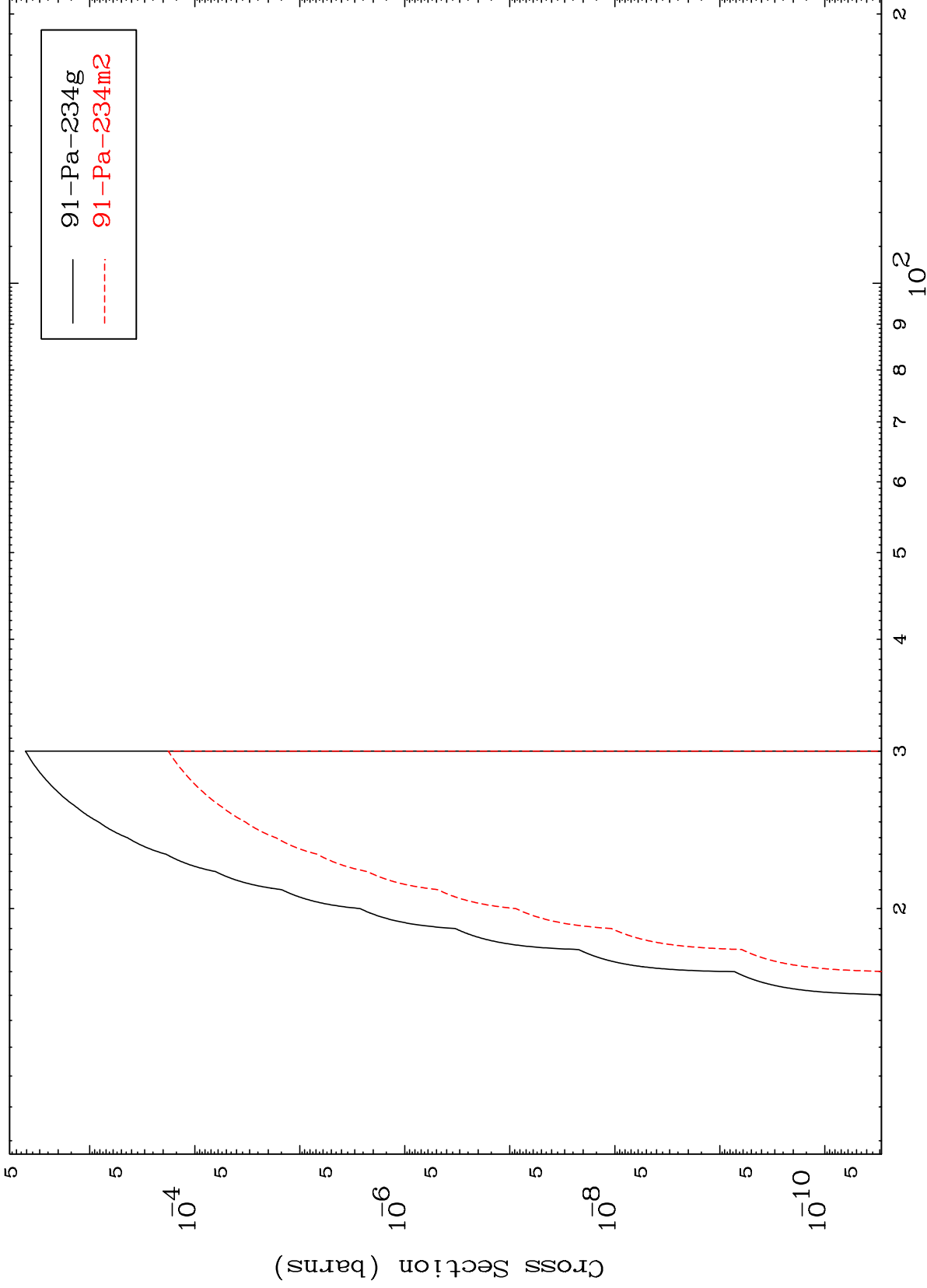
24

Incident Energy (MeV)

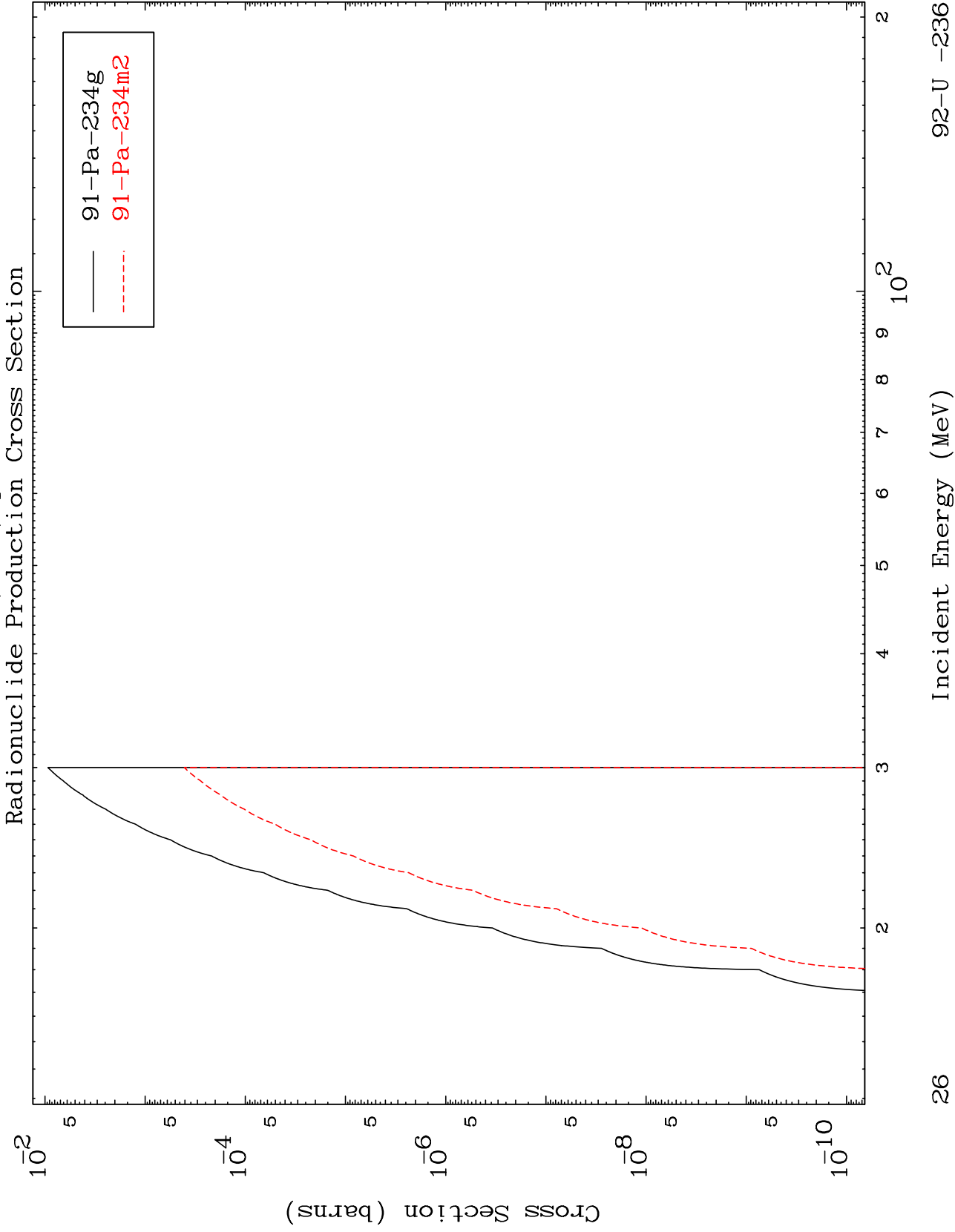
92-U -236

(n,n') d

Radionuclide Production Cross Section



91-Pa-234g
91-Pa-234m2



(n,t)
Radionuclide Production Cross Section

