

Program EVALPLOT  
(Version 2017-1)

by

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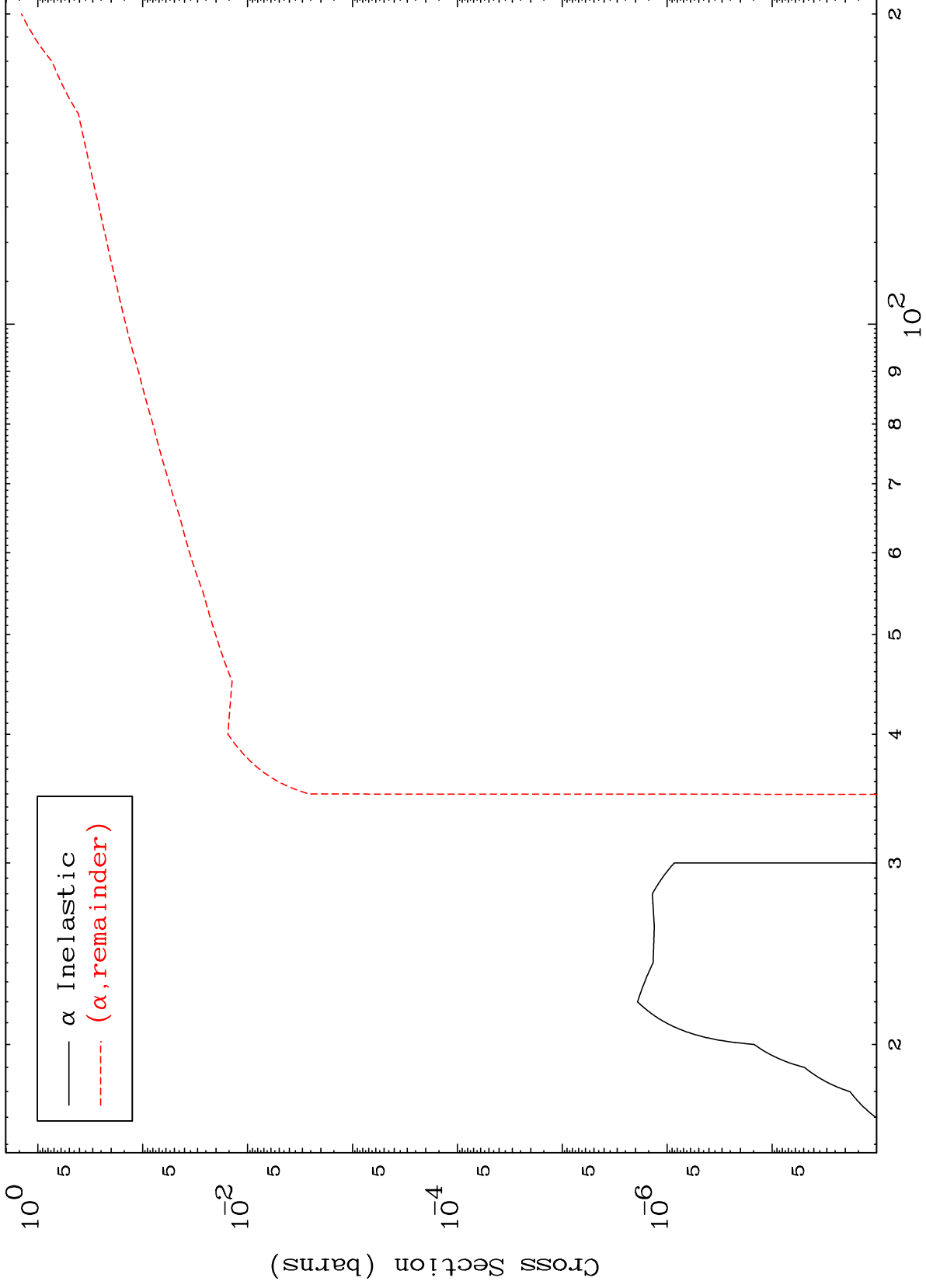
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 979

$\alpha$  Major  
0 Kelvin Cross Sections

109-Mt-279



1

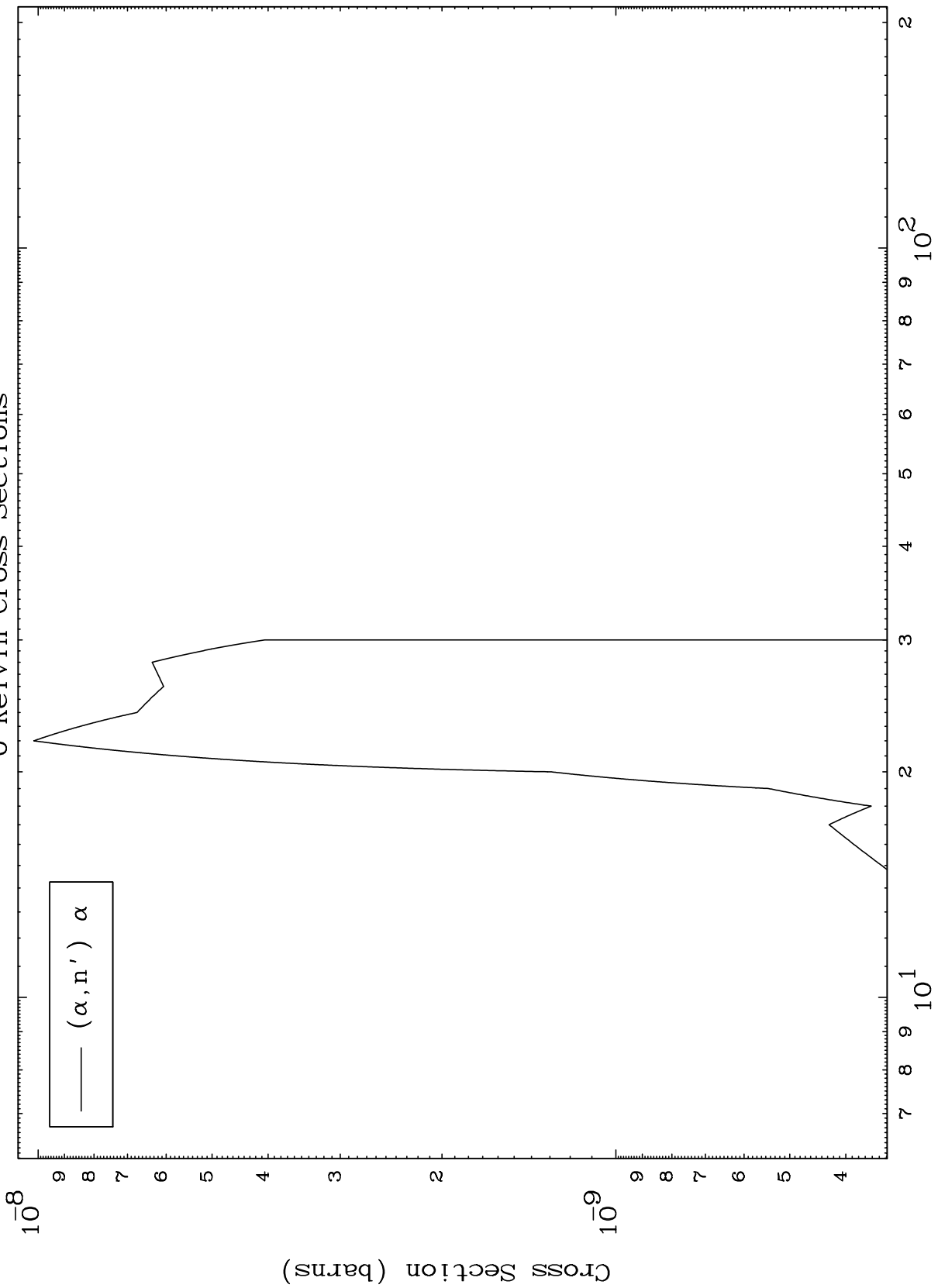
Incident Energy (MeV)

109-Mt-279

MAT 979

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

109-Mt-279



2

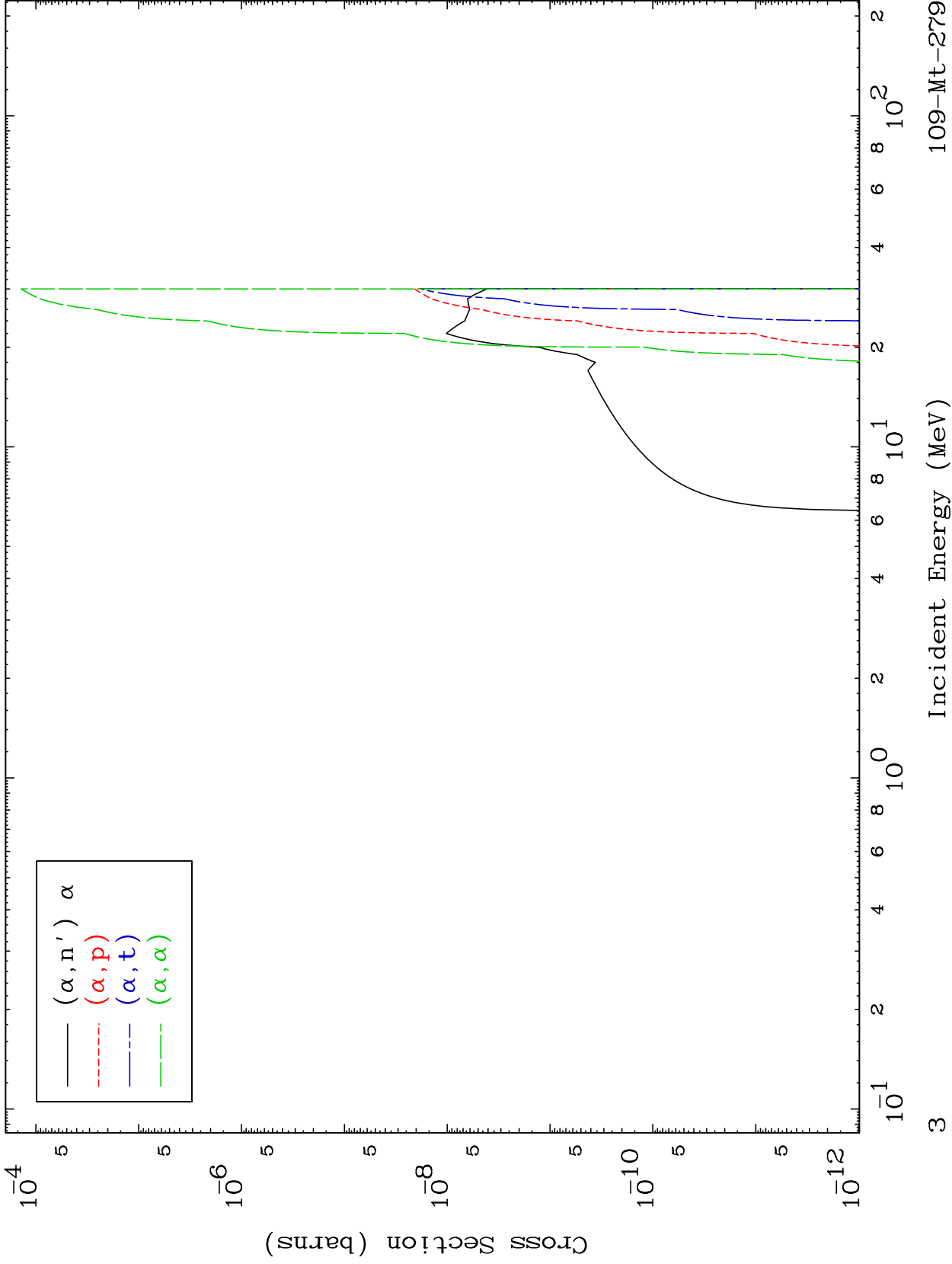
Incident Energy (MeV)

109-Mt-279

MAT 979

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

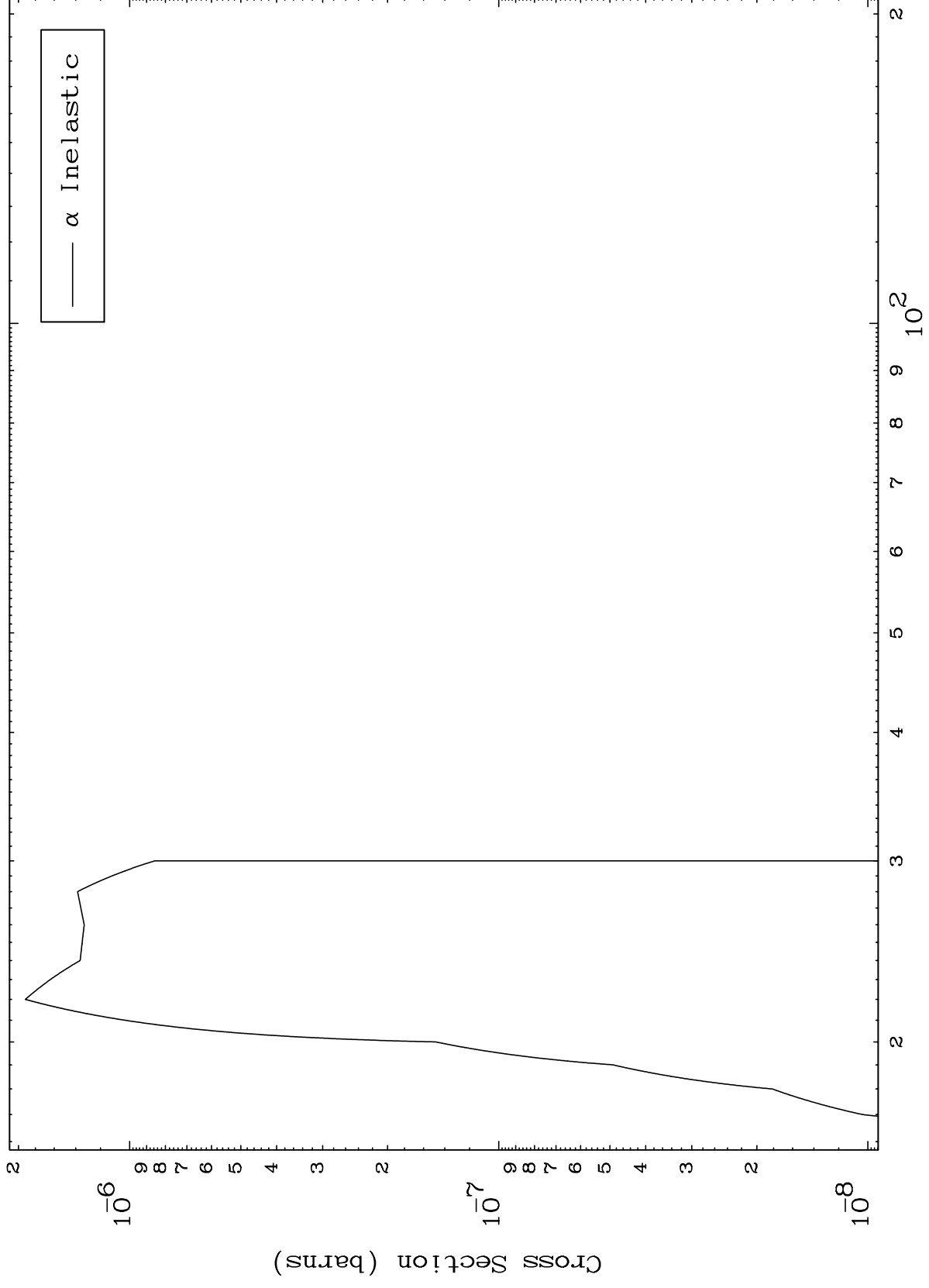
109-Mt-279



MAT 979

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

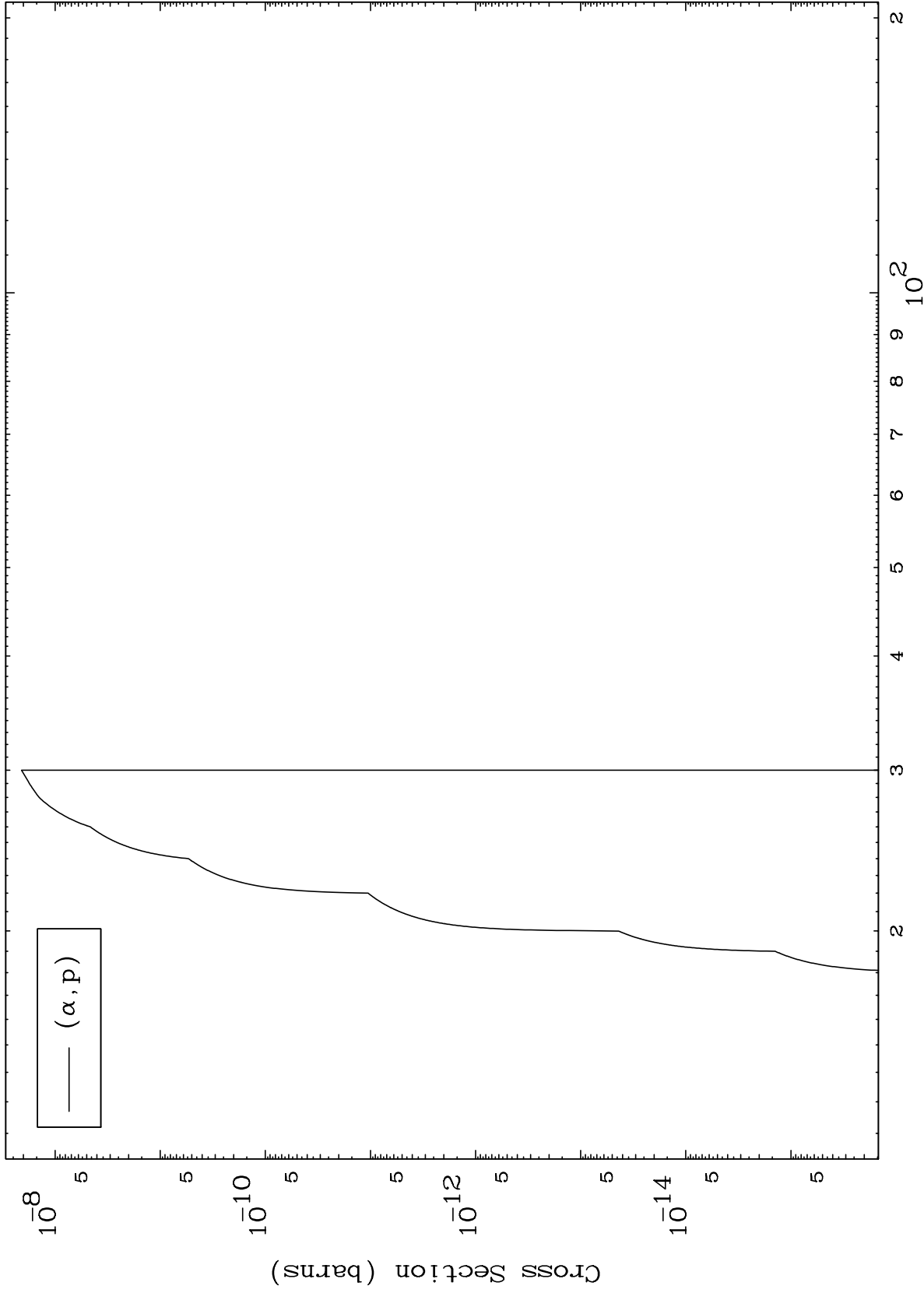
109-Mt-279



4

Incident Energy (MeV)

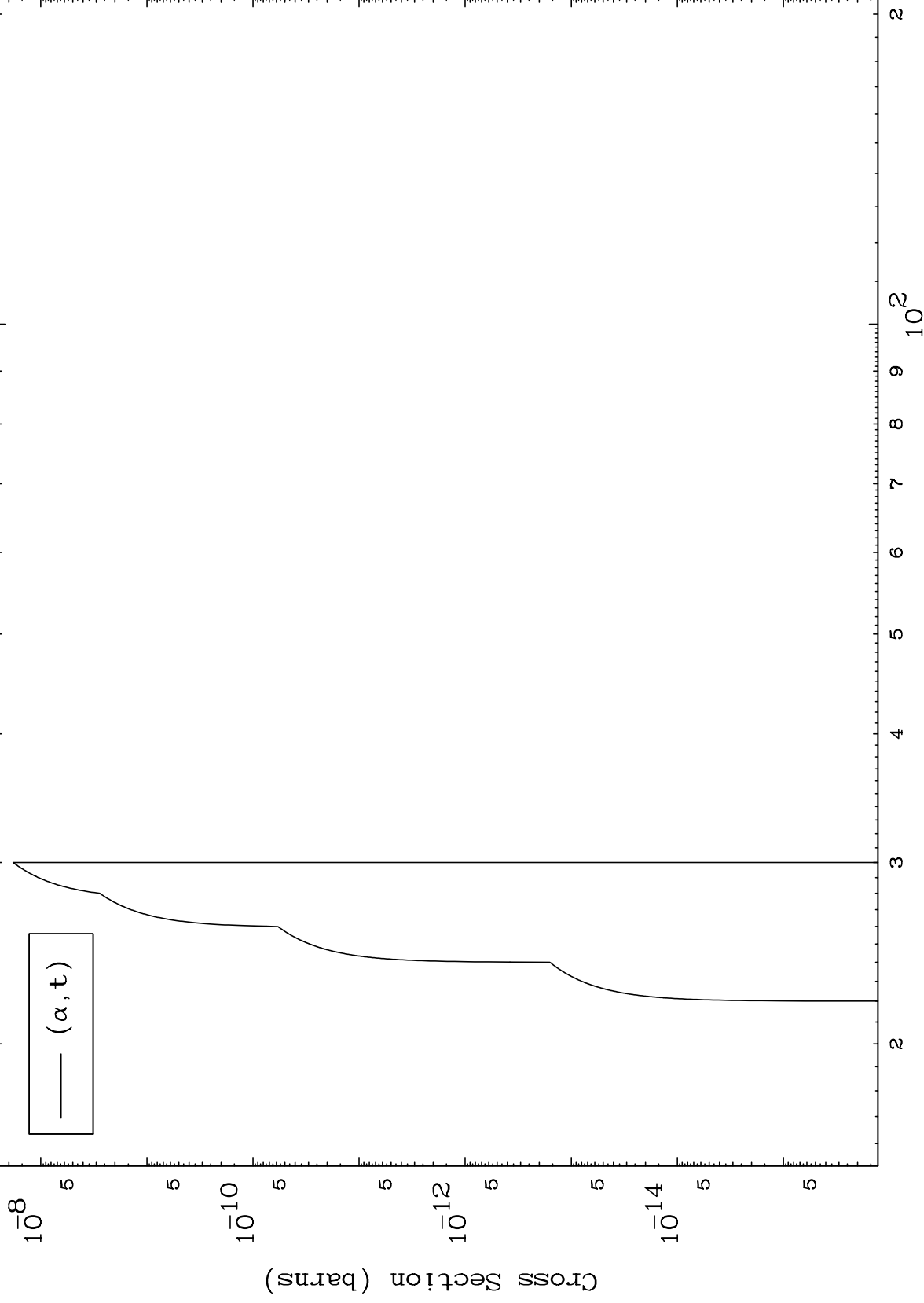
109-Mt-279



MAT 979

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

109-Mt-279



6

Incident Energy (MeV)

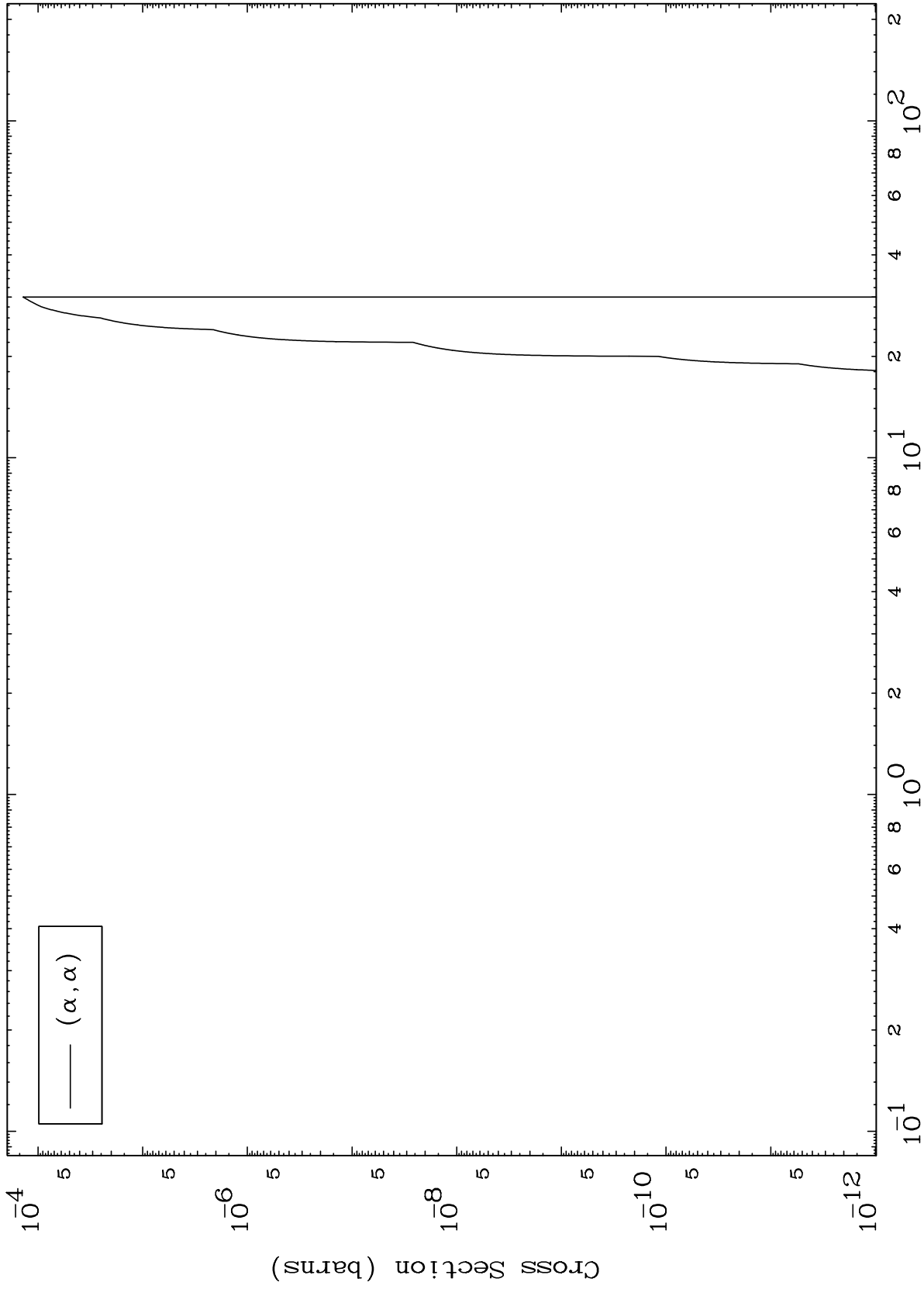
109-Mt-279

MAT 979

( $\alpha, \alpha$ ) Levels

109-Mt-279

0 Kelvin Cross Sections



Incident Energy (MeV)

109-Mt-279

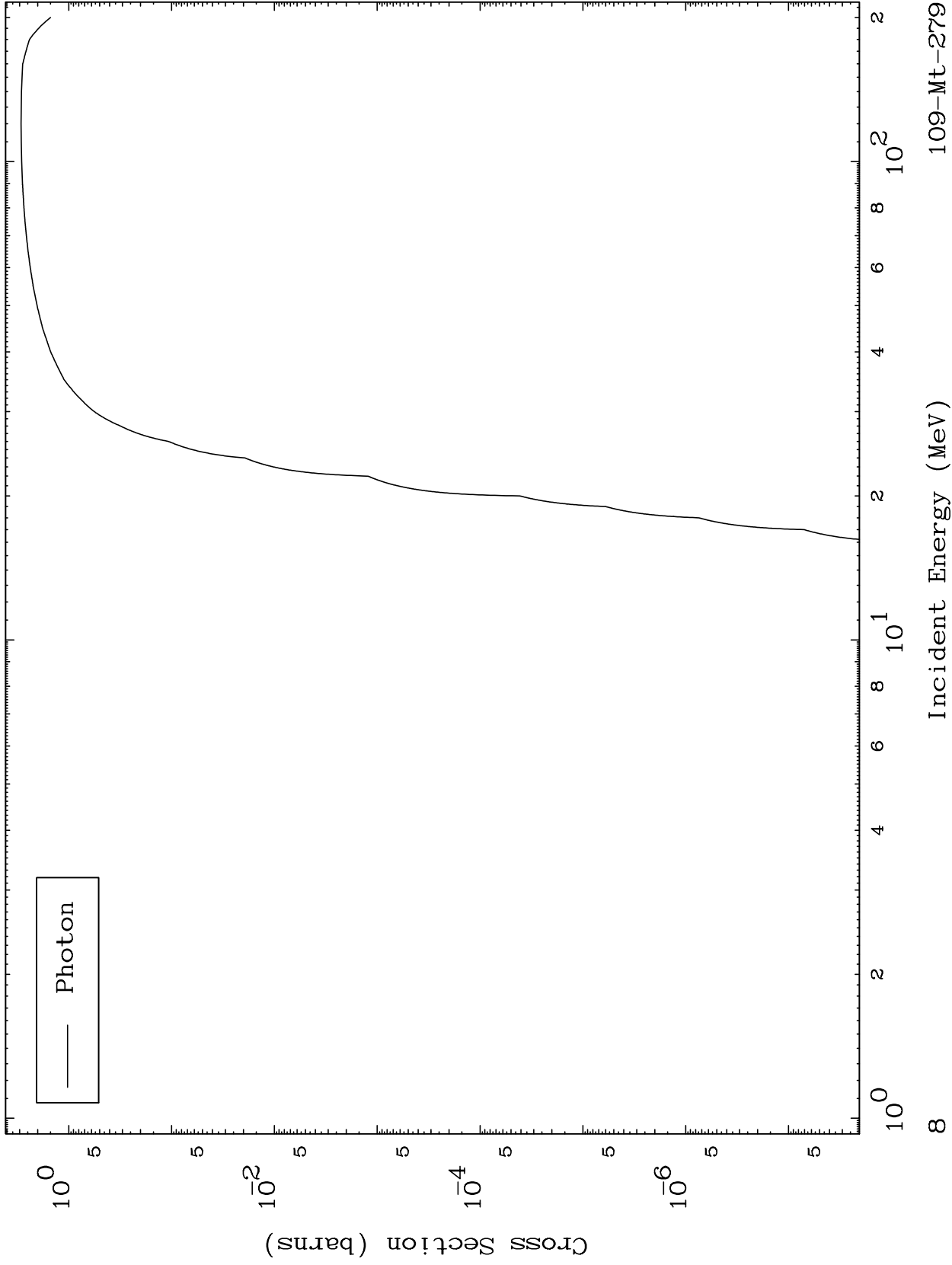
7



MAT 979

109-Mt-279

$\alpha$  Fission  
Radionuclide Production Cross Section



109-Mt-279

Incident Energy (MeV)