

Program EVALPLOT
(Version 2017-1)

by

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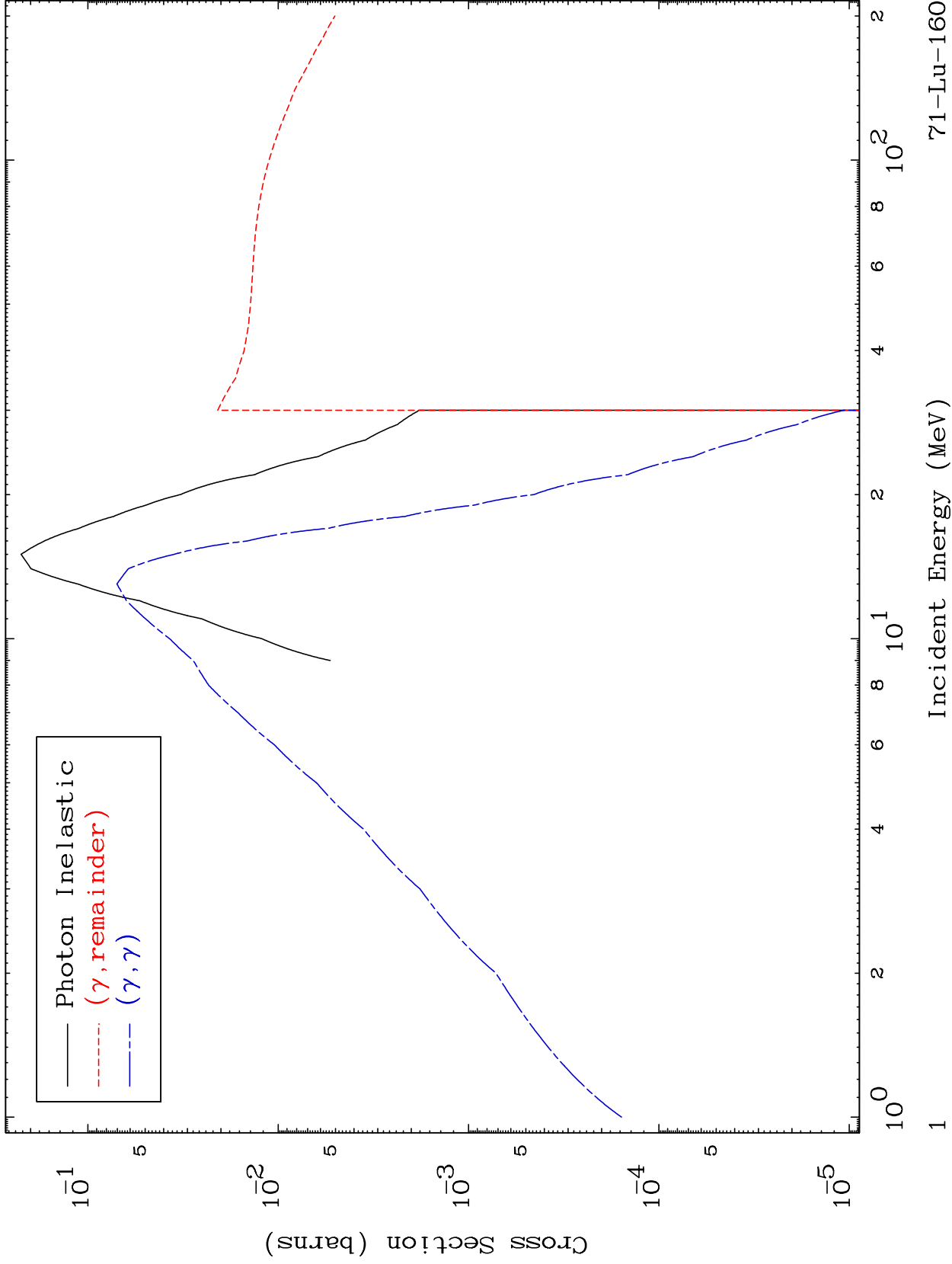
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7081

Photon Major
0 Kelvin Cross Sections

71-Lu-160



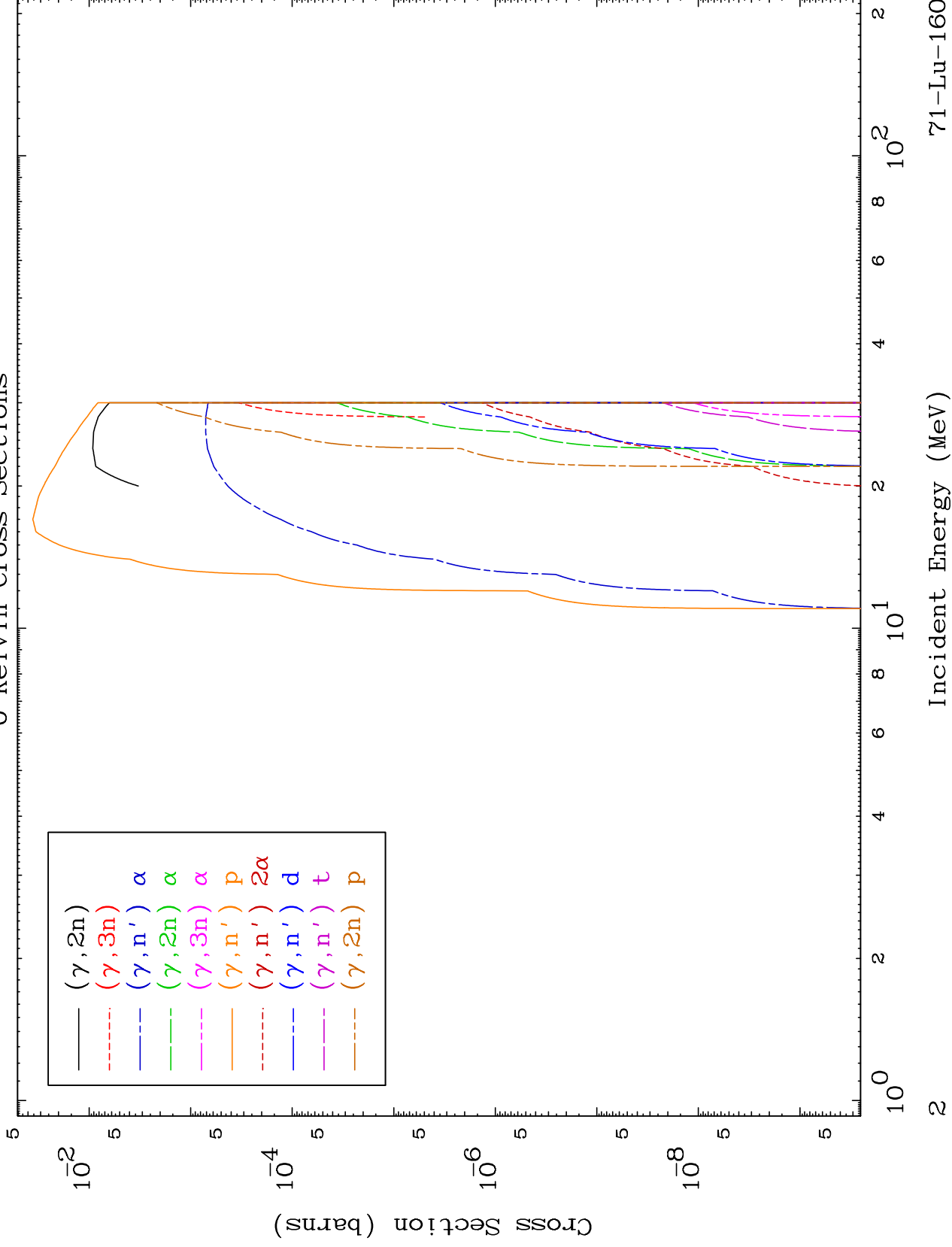
Incident Energy (MeV)

71-Lu-160

MAT 7081

Photon Neutron Production
0 Kelvin Cross Sections

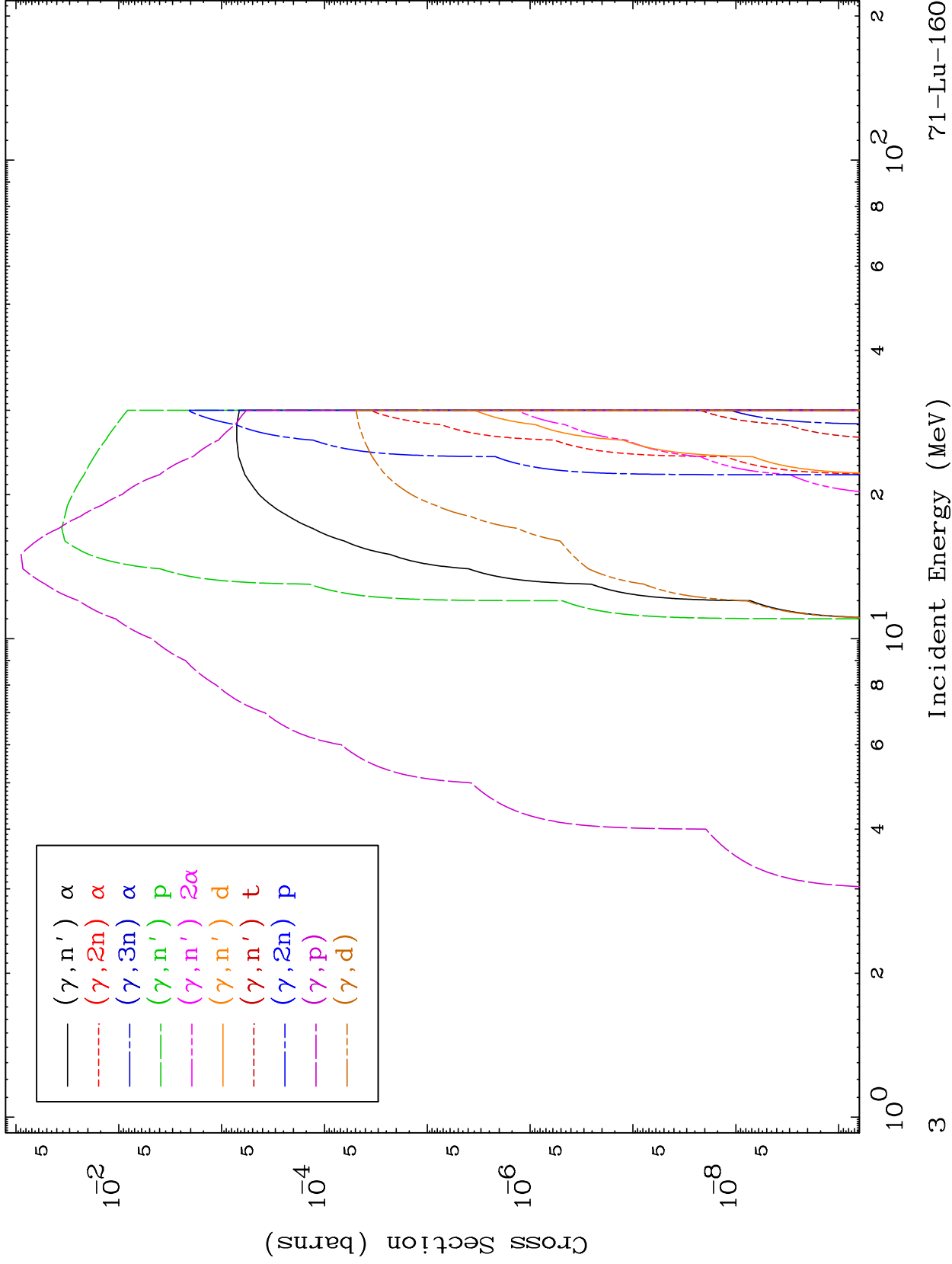
⁷¹Lu-160



MAT 7081

Photon Charged Particle
0 Kelvin Cross Sections

71-Lu-160



Incident Energy (MeV)

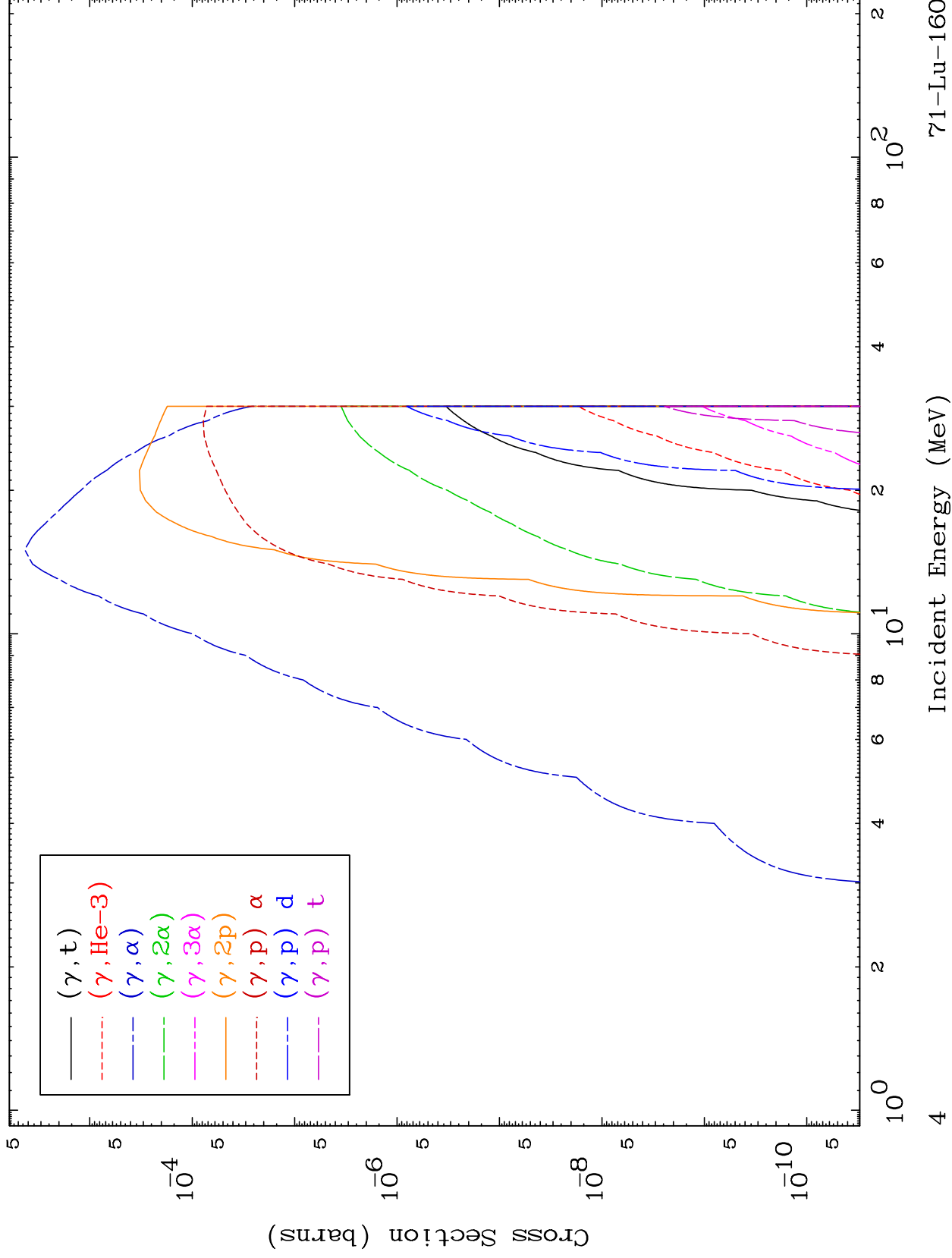
71-Lu-160

3

MAT 7081

Photon Charged Particle
0 Kelvin Cross Sections

71-Lu-160

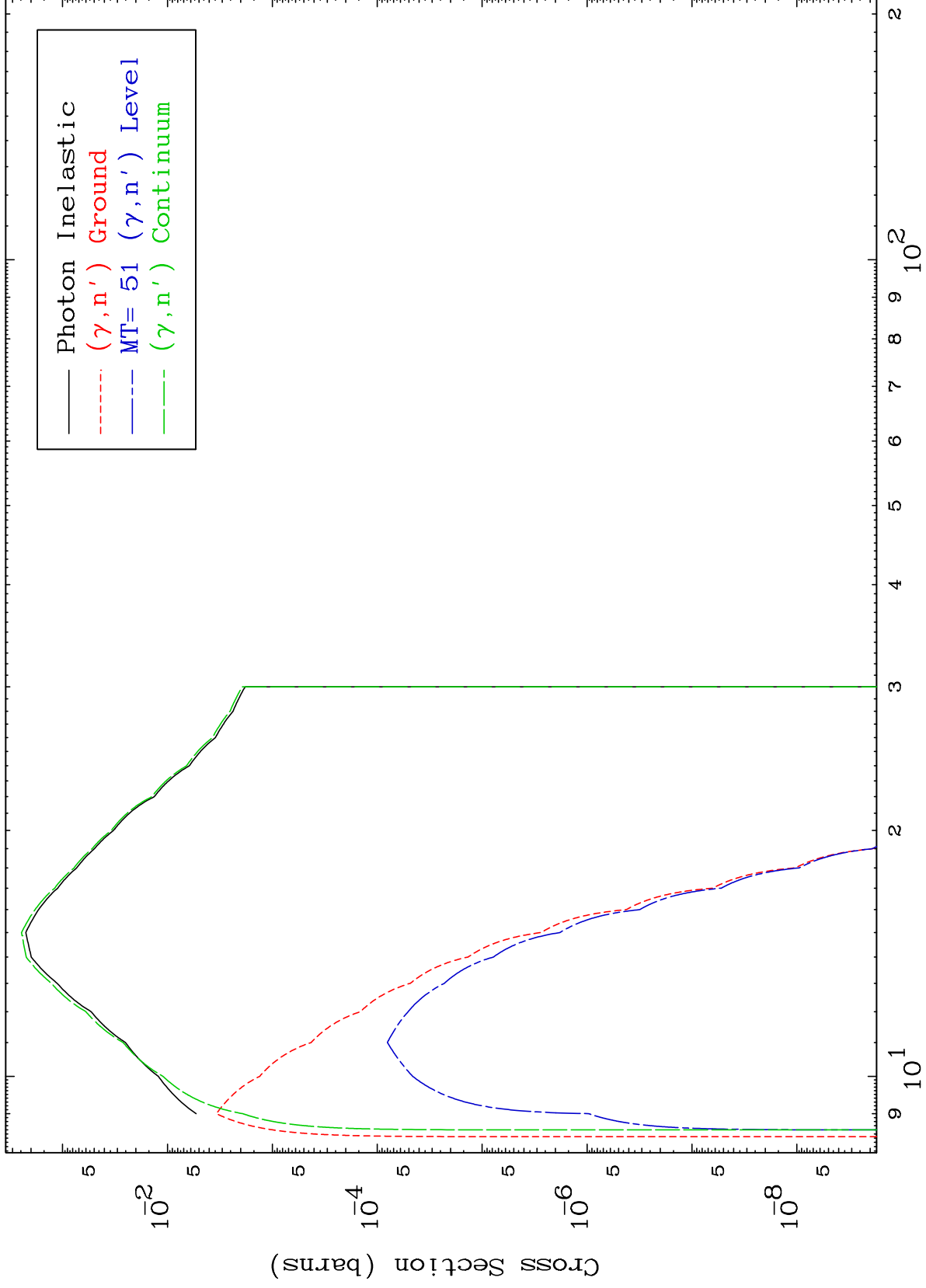


MAT 7081

(γ, n') Level

$^{71}\text{Lu-160}$

0 Kelvin Cross Sections



Incident Energy (MeV)

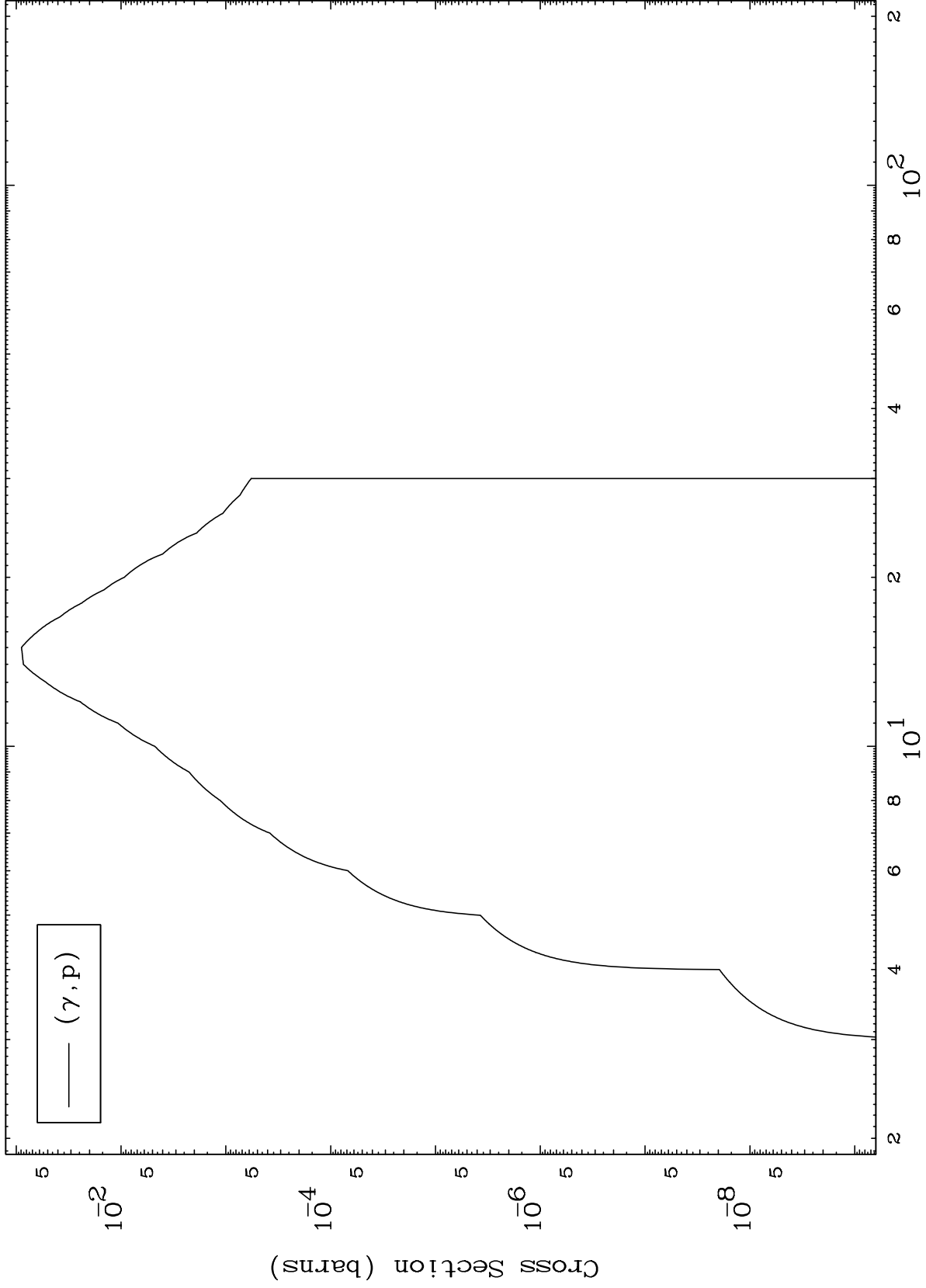
$^{71}\text{Lu-160}$

5

MAT 7081

(γ, p) Levels
0 Kelvin Cross Sections

71-Lu-160



6

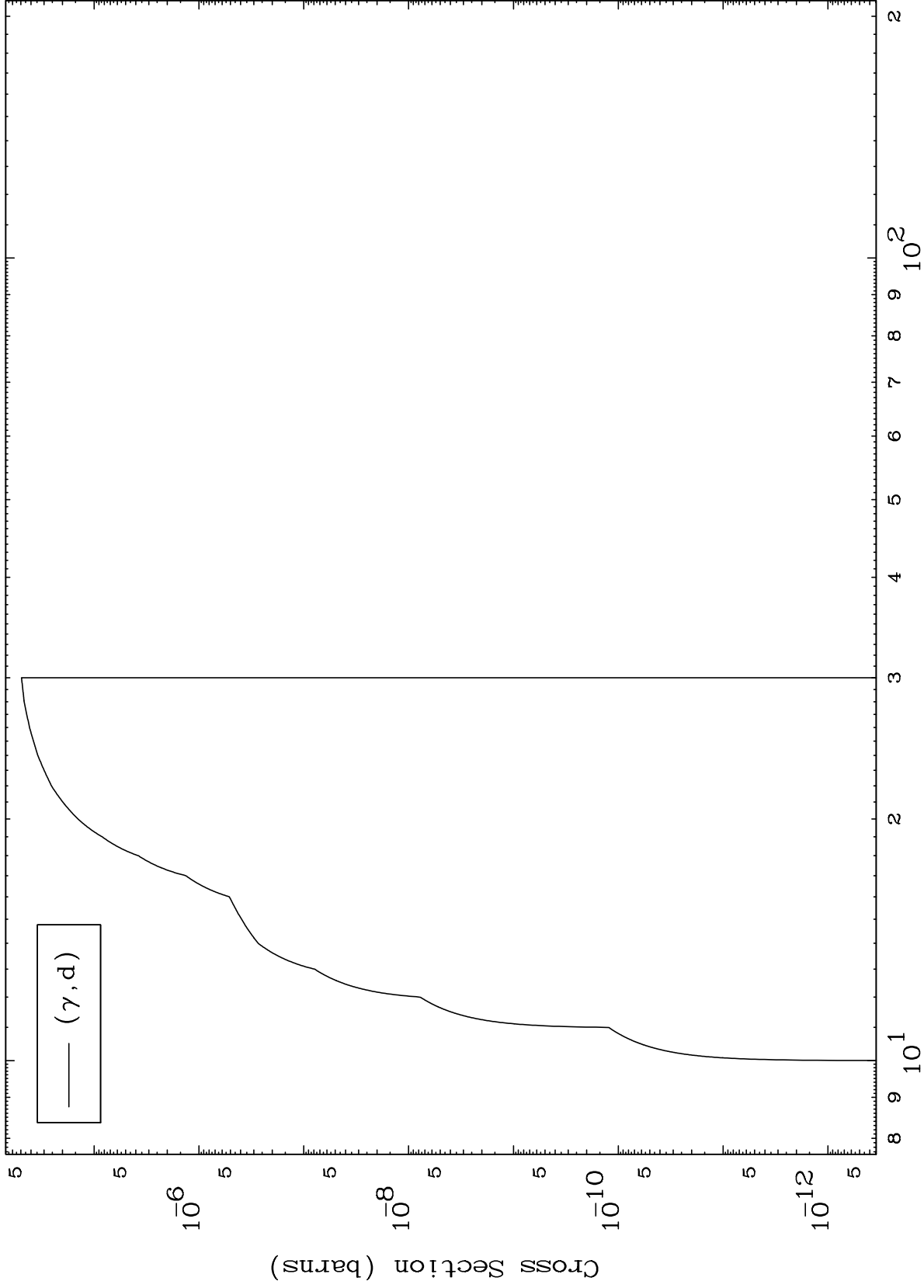
Incident Energy (MeV)

71-Lu-160

MAT 7081

(γ, d) Levels
0 Kelvin Cross Sections

⁷¹Lu-160



Incident Energy (MeV)

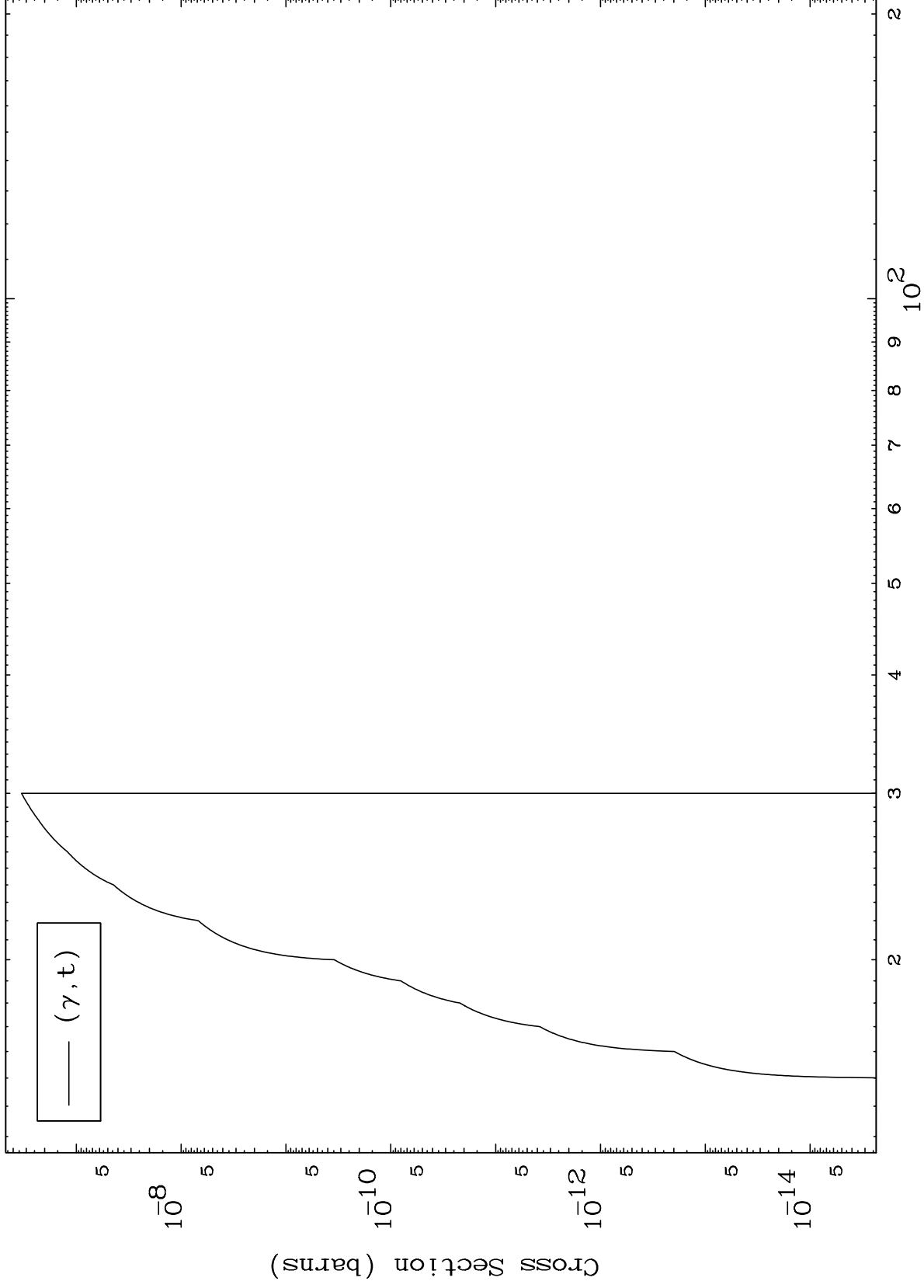
⁷¹Lu-160

7

MAT 7081

(γ, t) Levels
0 Kelvin Cross Sections

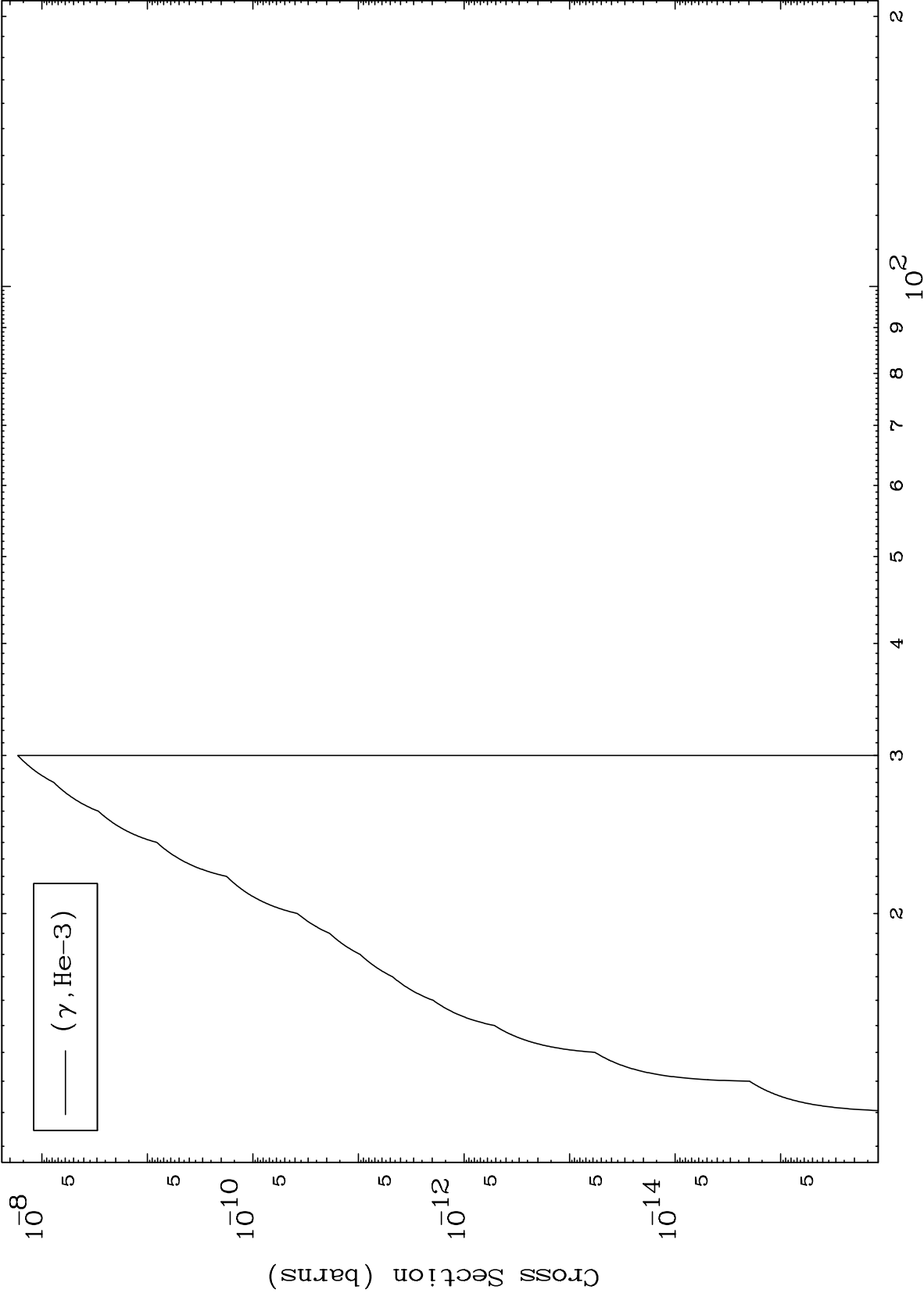
71-Lu-160



8

Incident Energy (MeV)

71-Lu-160

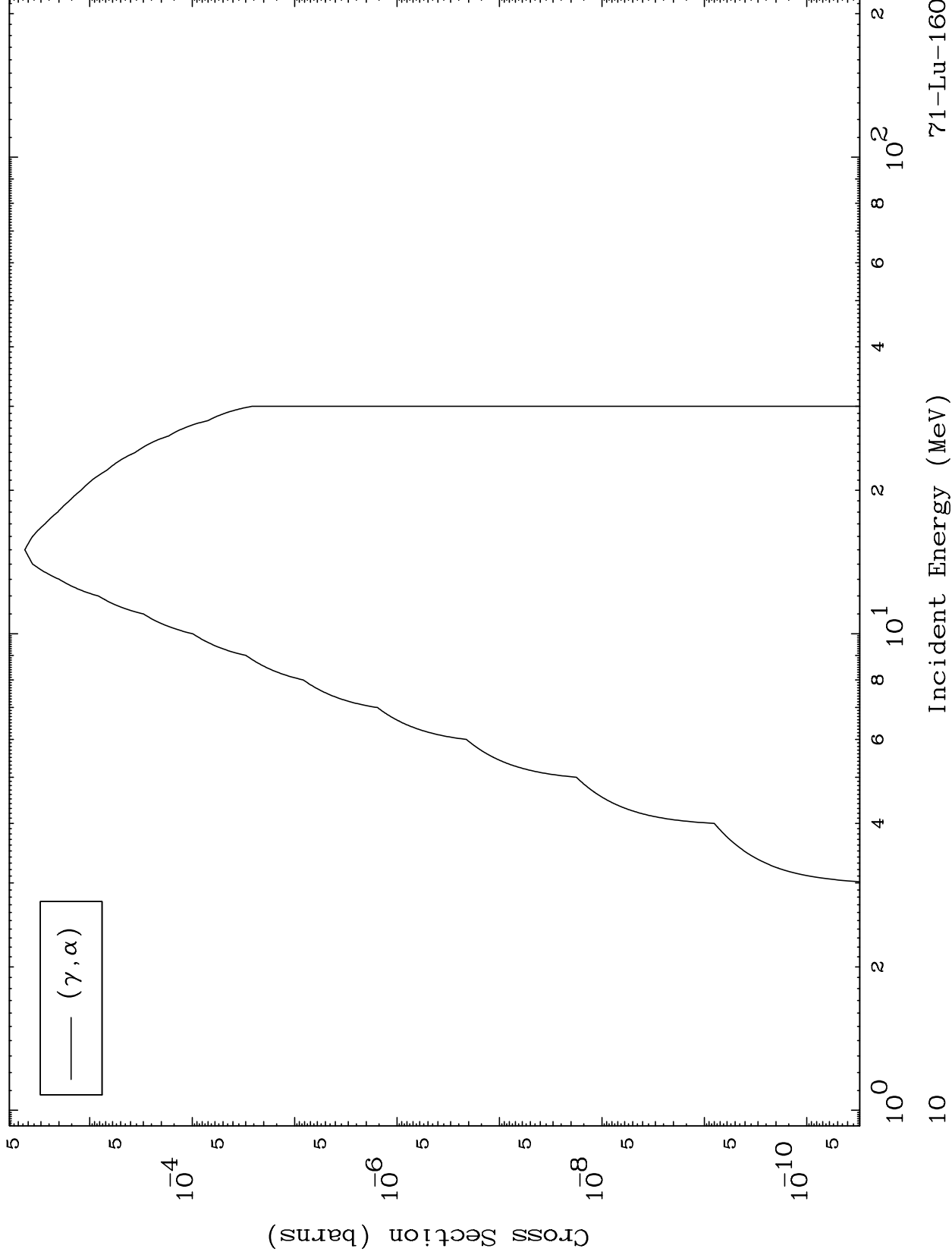


MAT 7081

(γ, α) Levels

$^{71}\text{Lu-160}$

0 Kelvin Cross Sections



(γ, α)

Incident Energy (MeV)

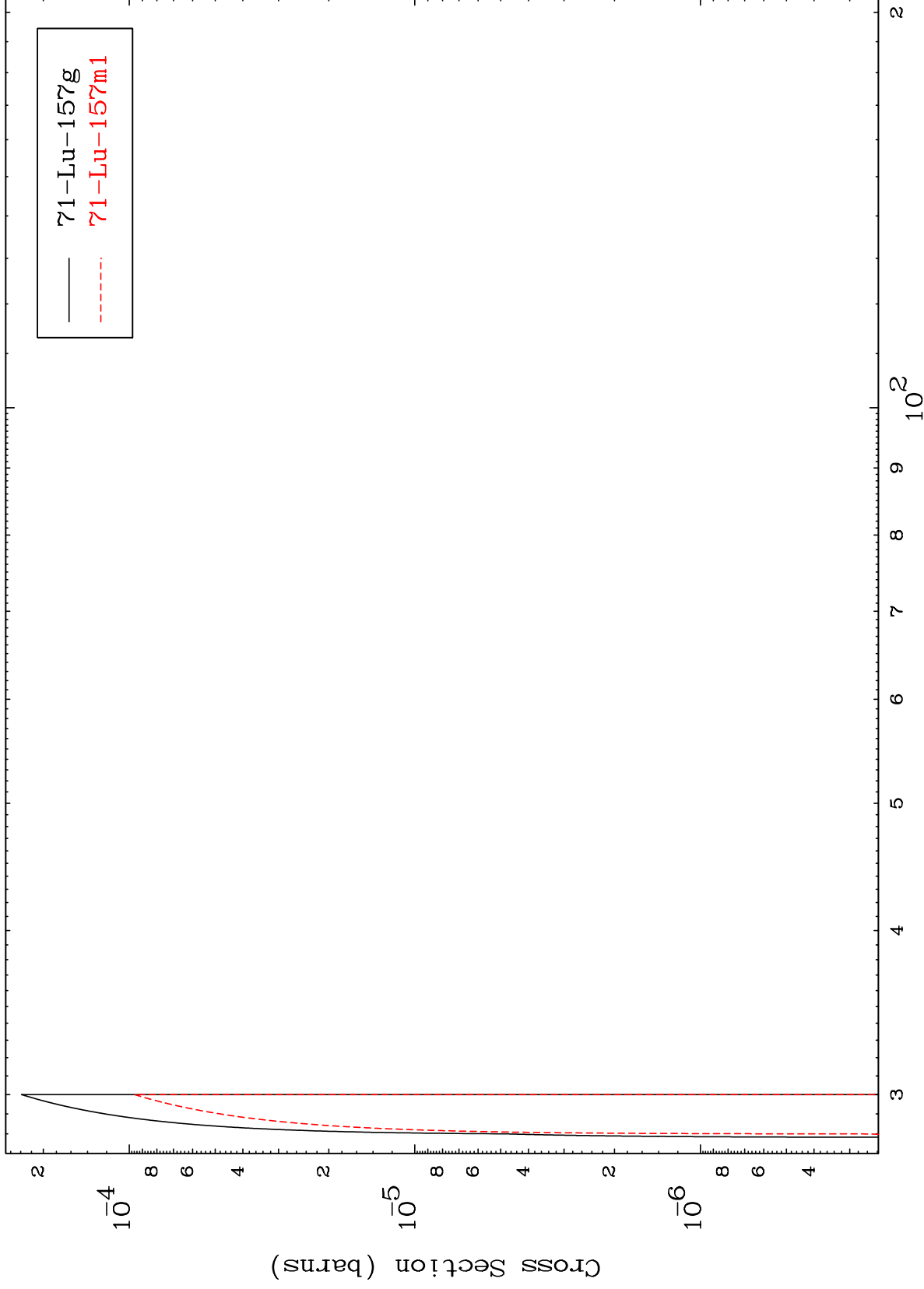
$^{71}\text{Lu-160}$

MAT 7081

($\gamma, 3n$)

71-Lu-160

Radionuclide Production Cross Section



11

Incident Energy (MeV)

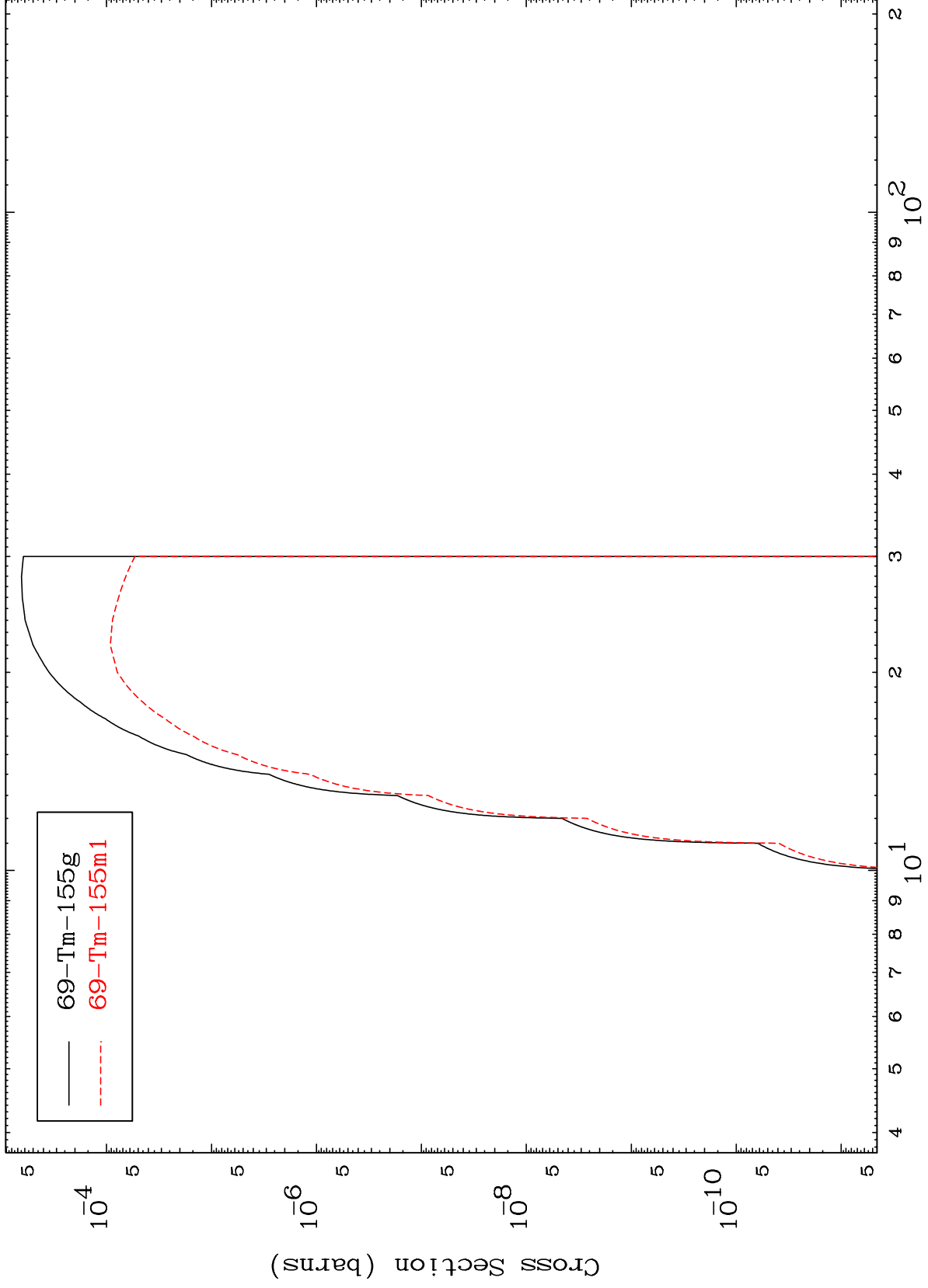
71-Lu-160

MAT 7081

(γ, n') α

$^{71}\text{Lu-160}$

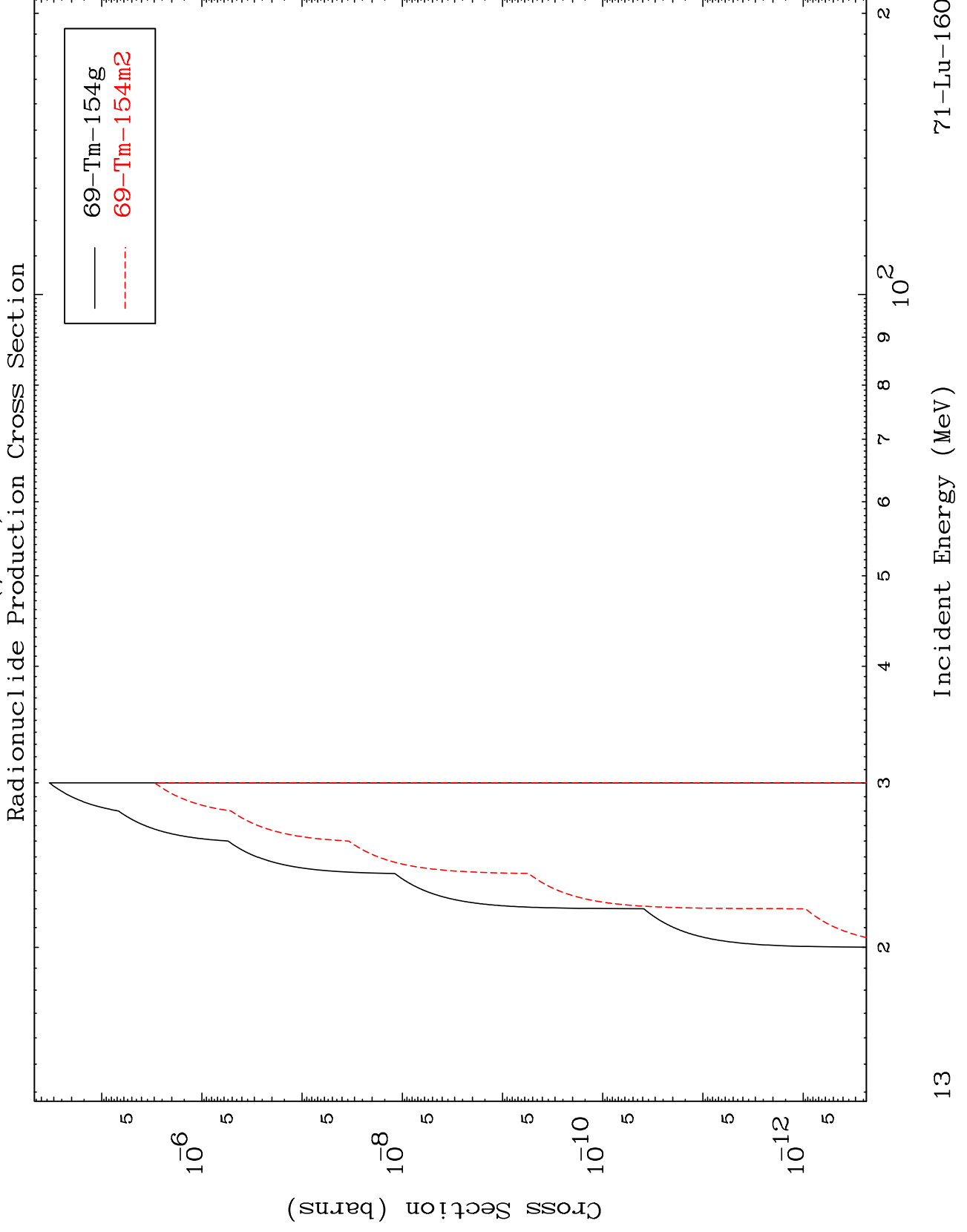
Radionuclide Production Cross Section



12

Incident Energy (MeV)

$^{71}\text{Lu-160}$

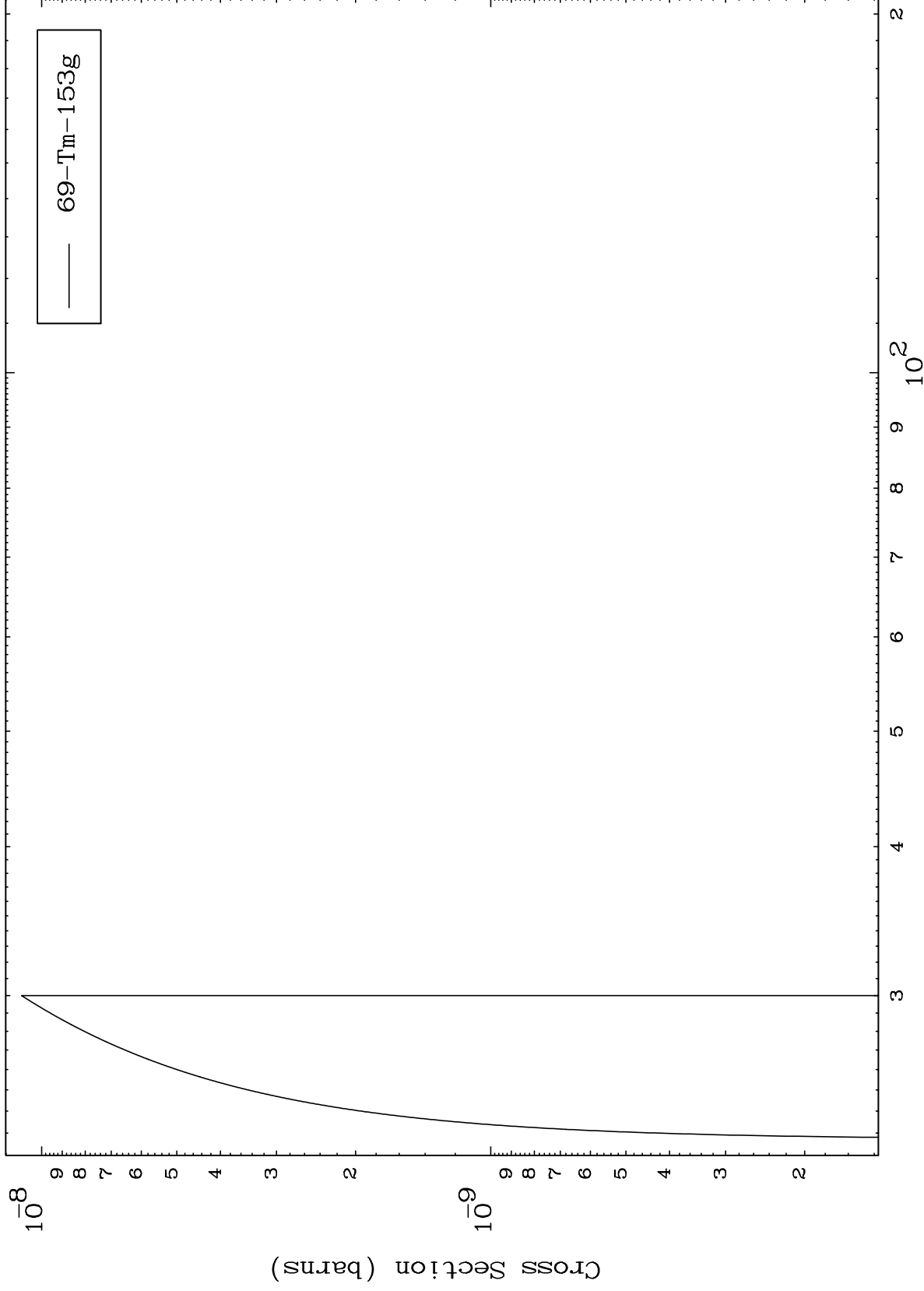


MAT 7081

$(\gamma, 3n) \alpha$

⁷¹Lu-160

Radionuclide Production Cross Section



14

Incident Energy (MeV)

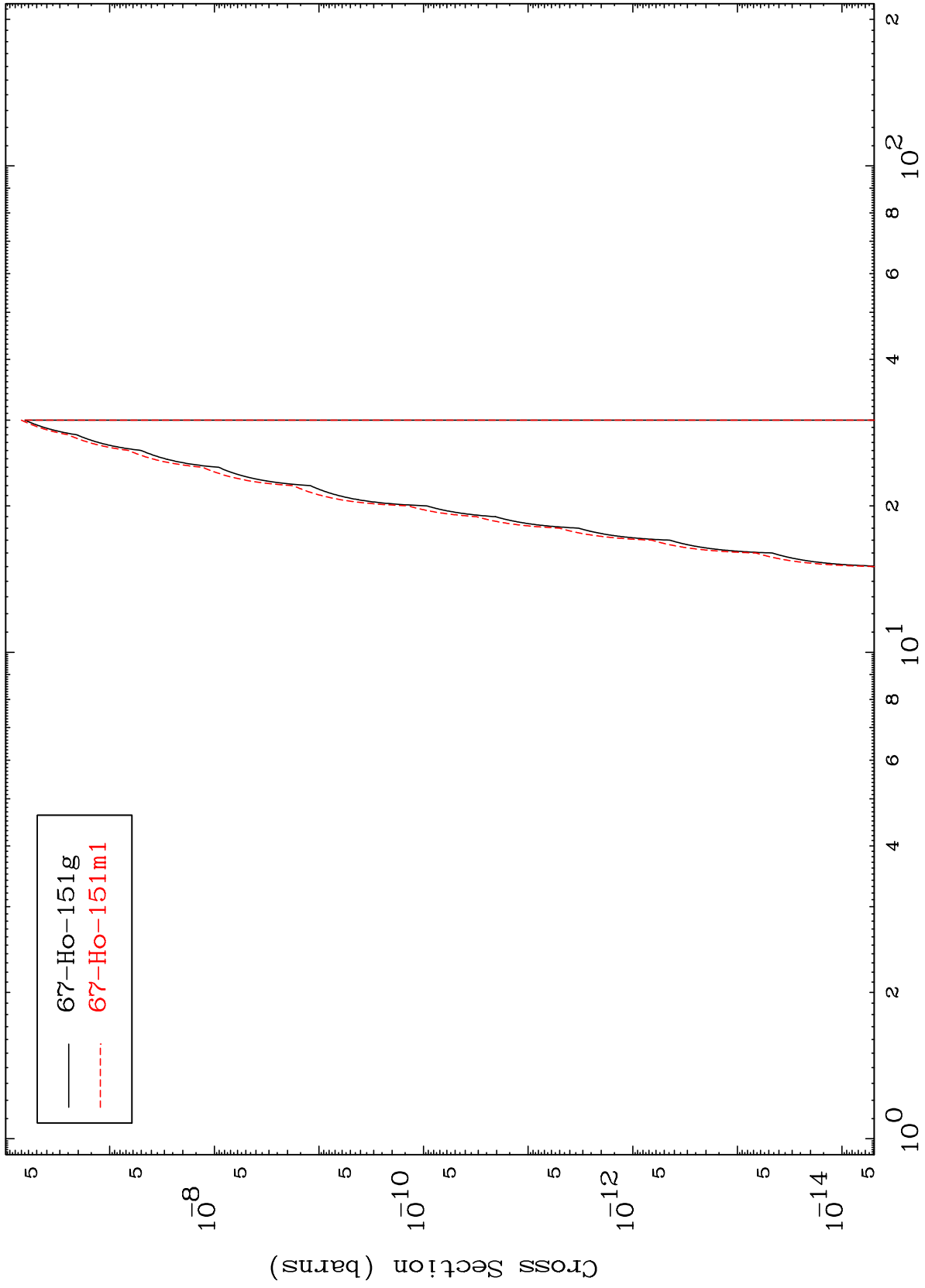
⁷¹Lu-160

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(γ, n') 2α

$^{71}\text{Lu-160}$

Radionuclide Production Cross Section



15

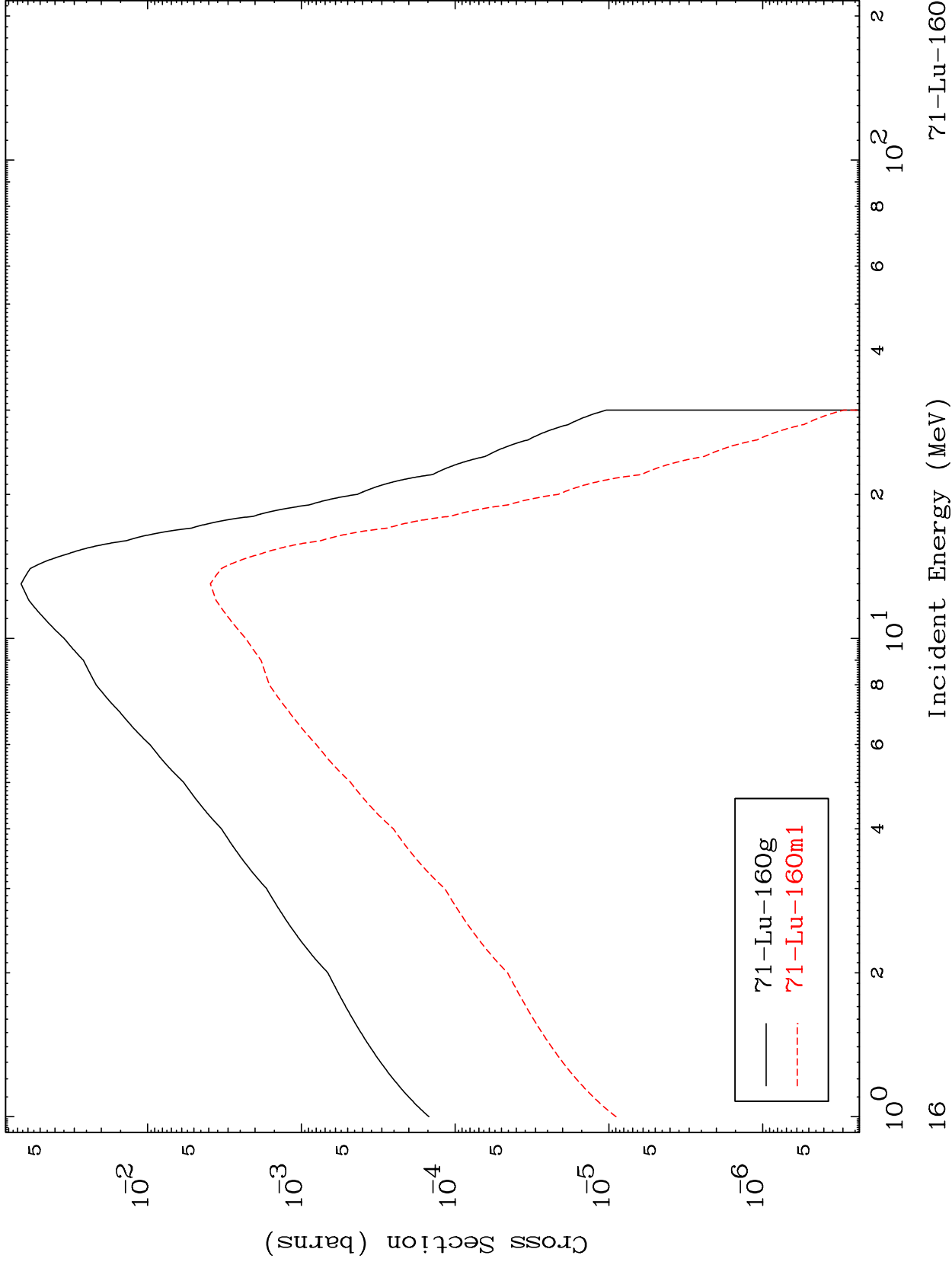
Incident Energy (MeV)

$^{71}\text{Lu-160}$

MAT 7081

⁷¹Lu-160

(γ, γ)
Radionuclide Production Cross Section

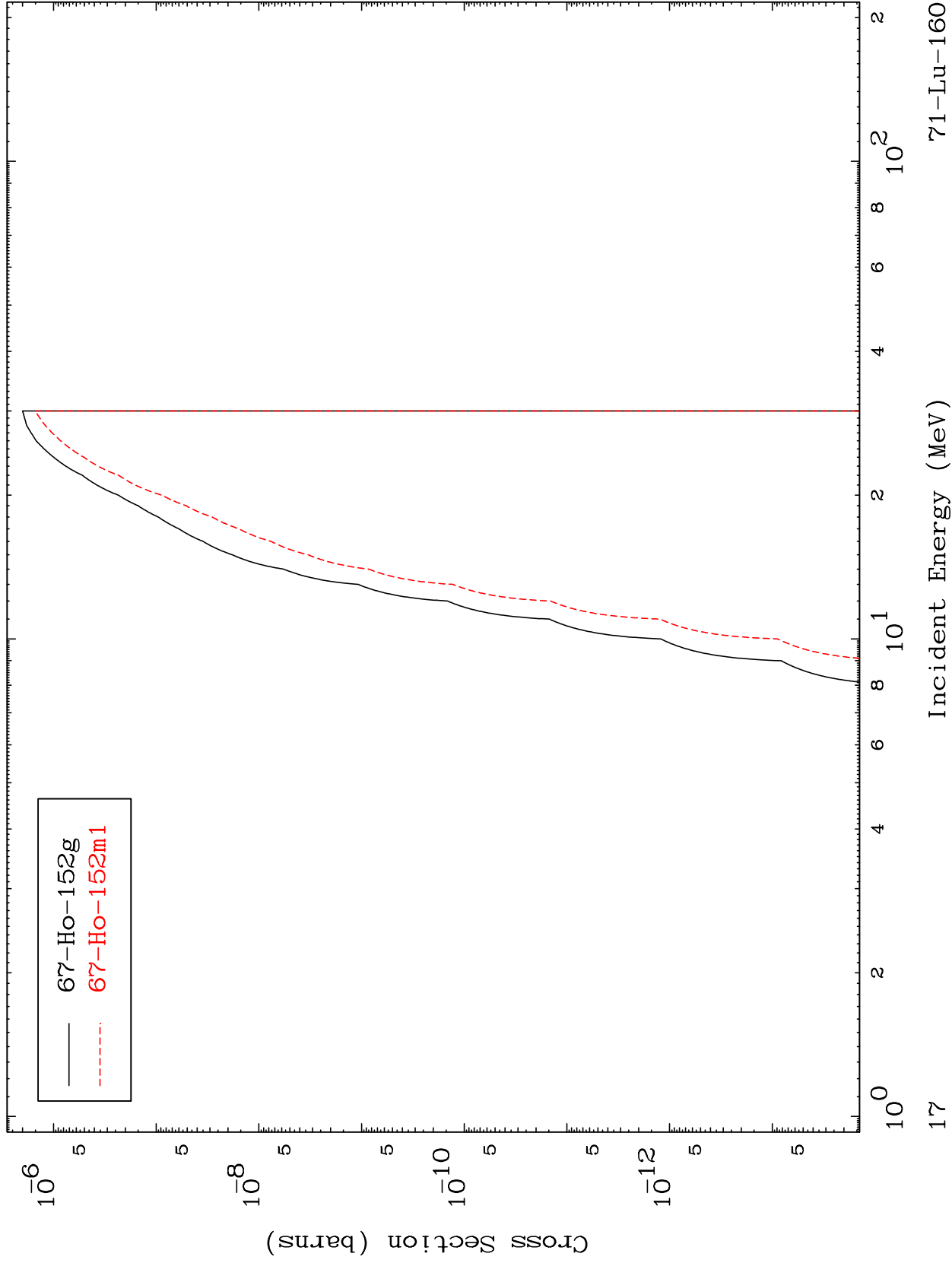


MAT 7081

($\gamma, 2\alpha$)

⁷¹Lu-160

Radionuclide Production Cross Section



Incident Energy (MeV)

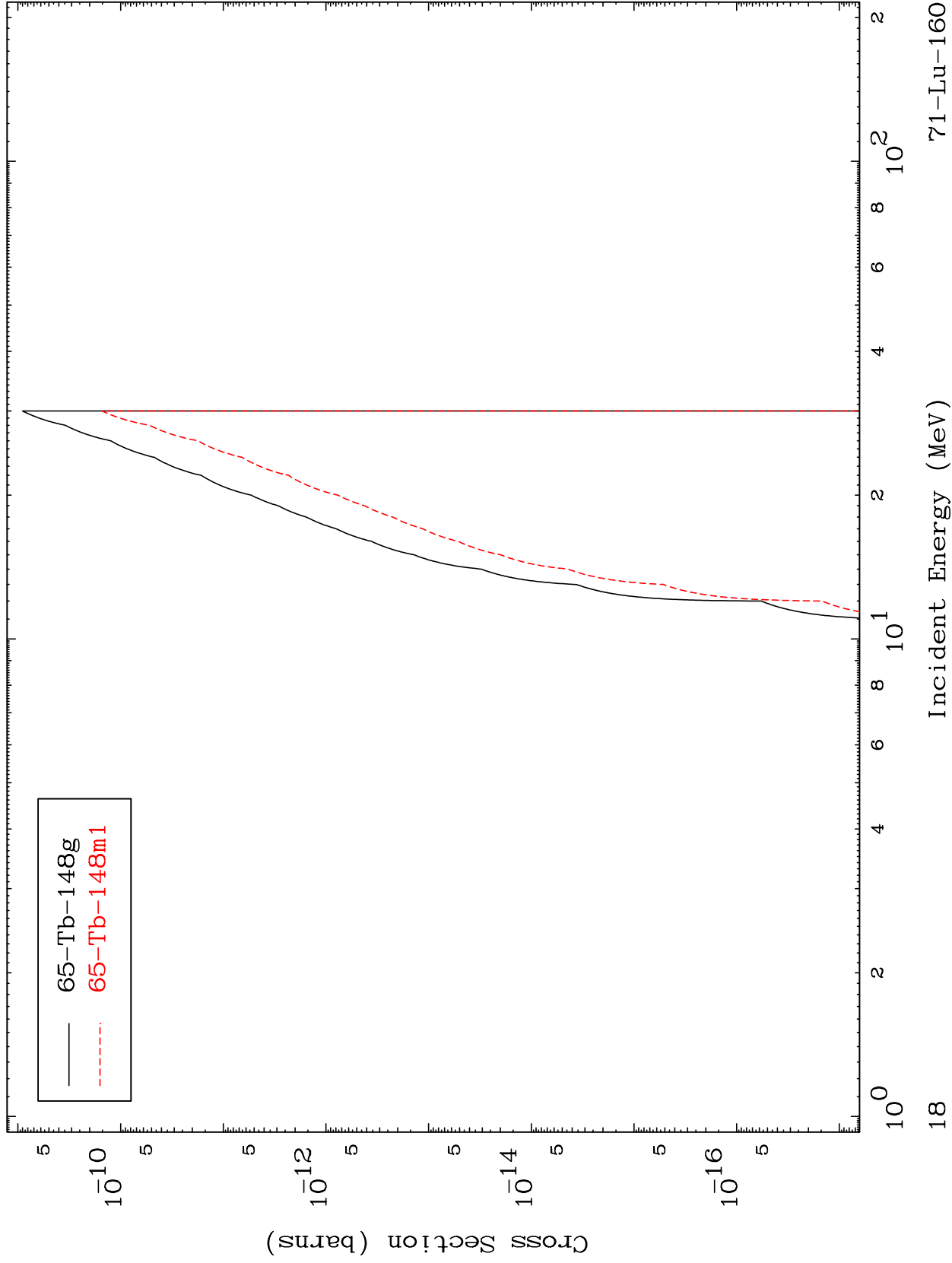
⁷¹Lu-160

MAT 7081

($\gamma, 3\alpha$)

71-Lu-160

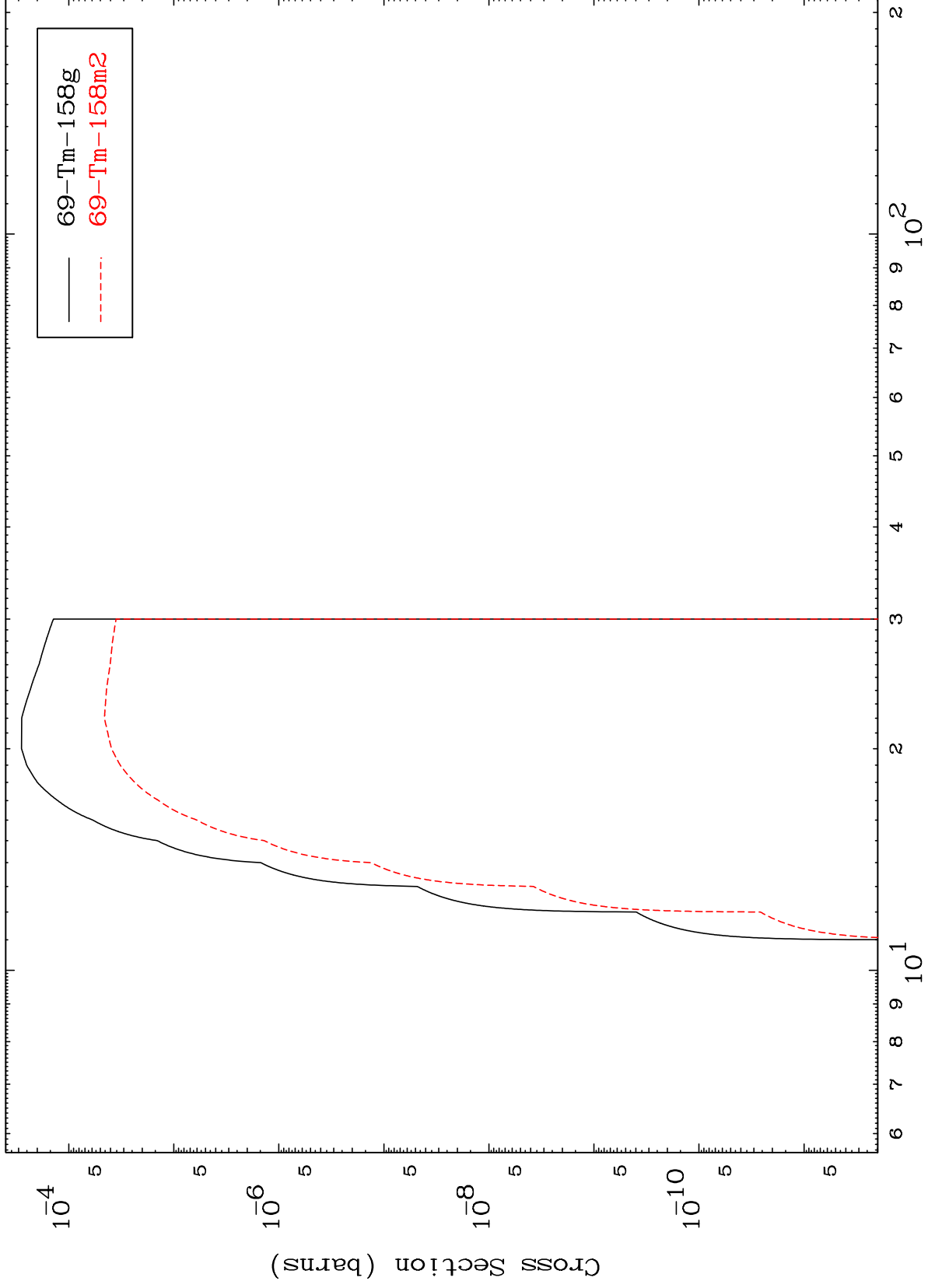
Radionuclide Production Cross Section



MAT 7081

71-Lu-160

($\gamma, 2p$)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

71-Lu-160