

Program EVALPLOT  
(Version 2017-1)

by

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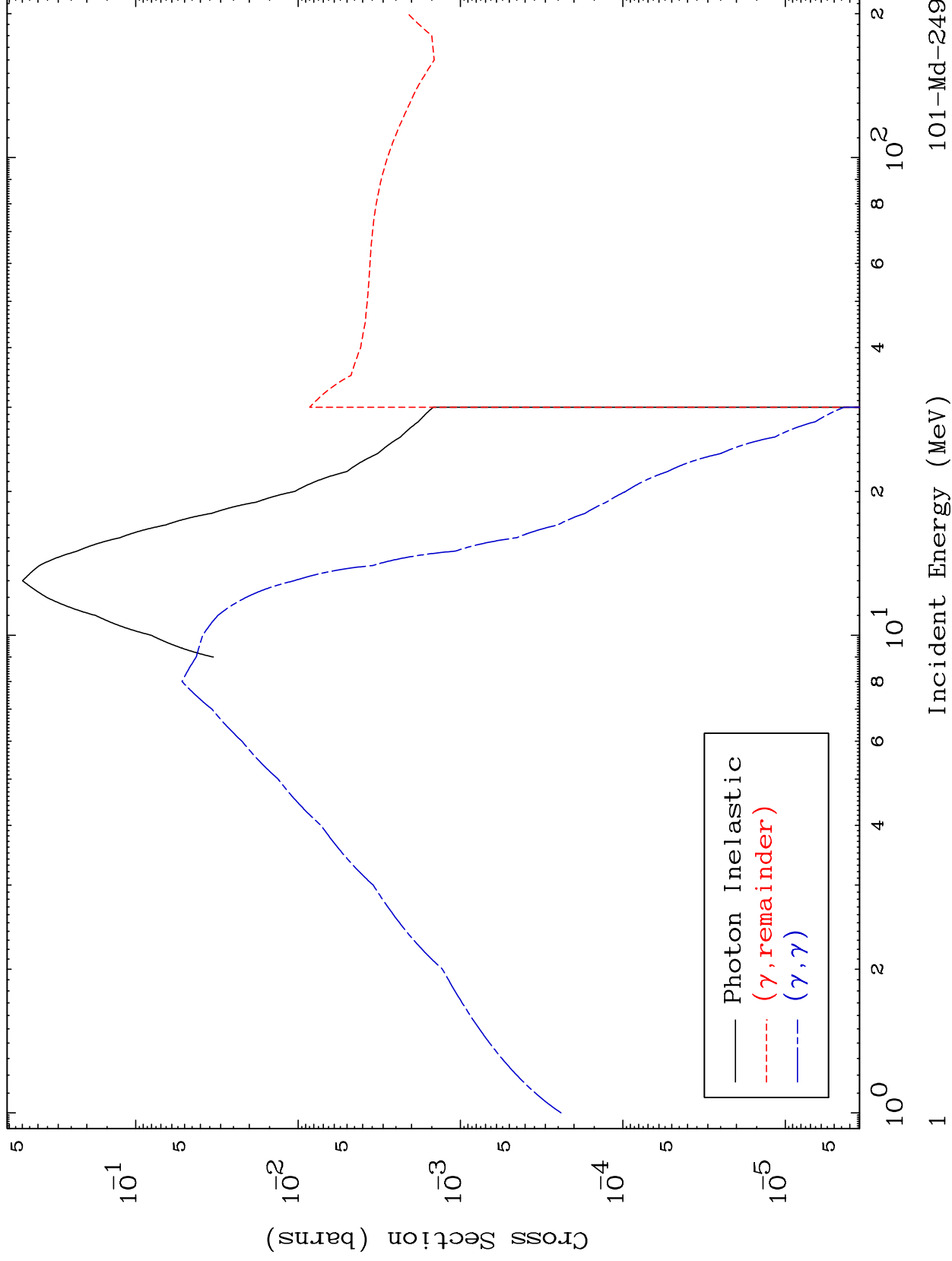
Press Mouse Button to Start

MAT 9950

Photon Major

101-Md-249

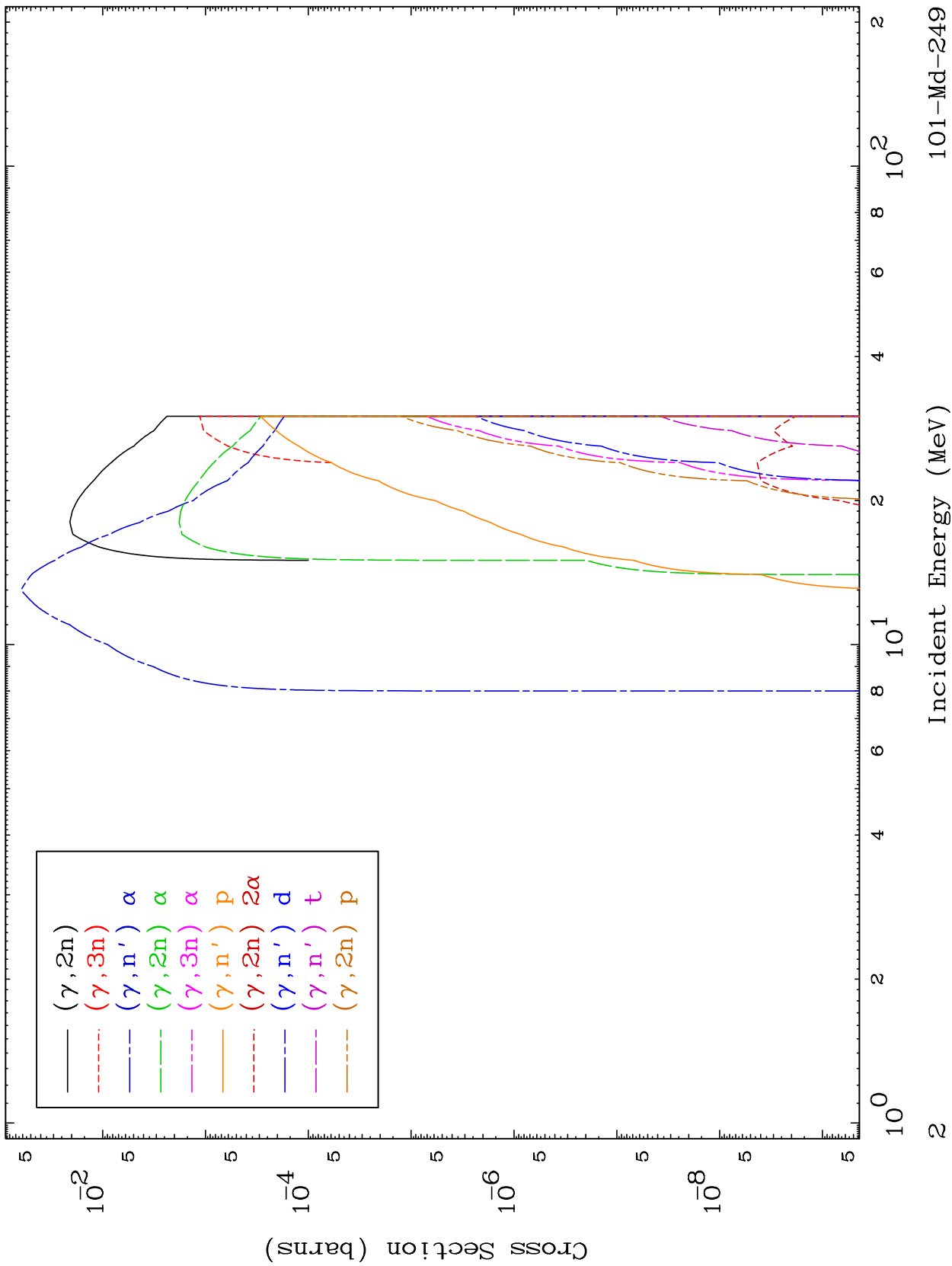
0 Kelvin Cross Sections



MAT 9950

Photon Neutron Production  
0 Kelvin Cross Sections

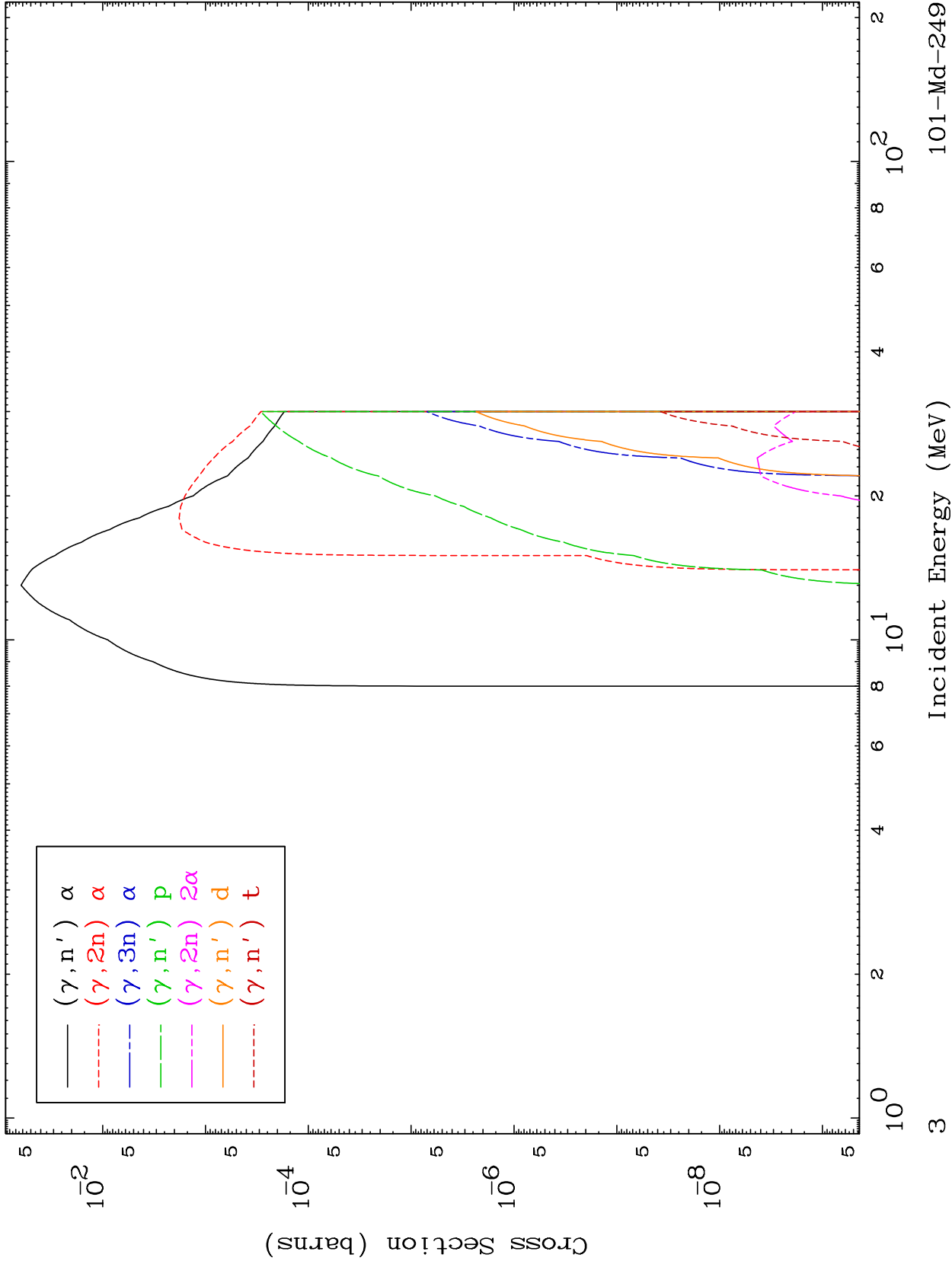
101-Md-249



MAT 9950

Photon Charged Particle  
0 Kelvin Cross Sections

101-Md-249



101-Md-249

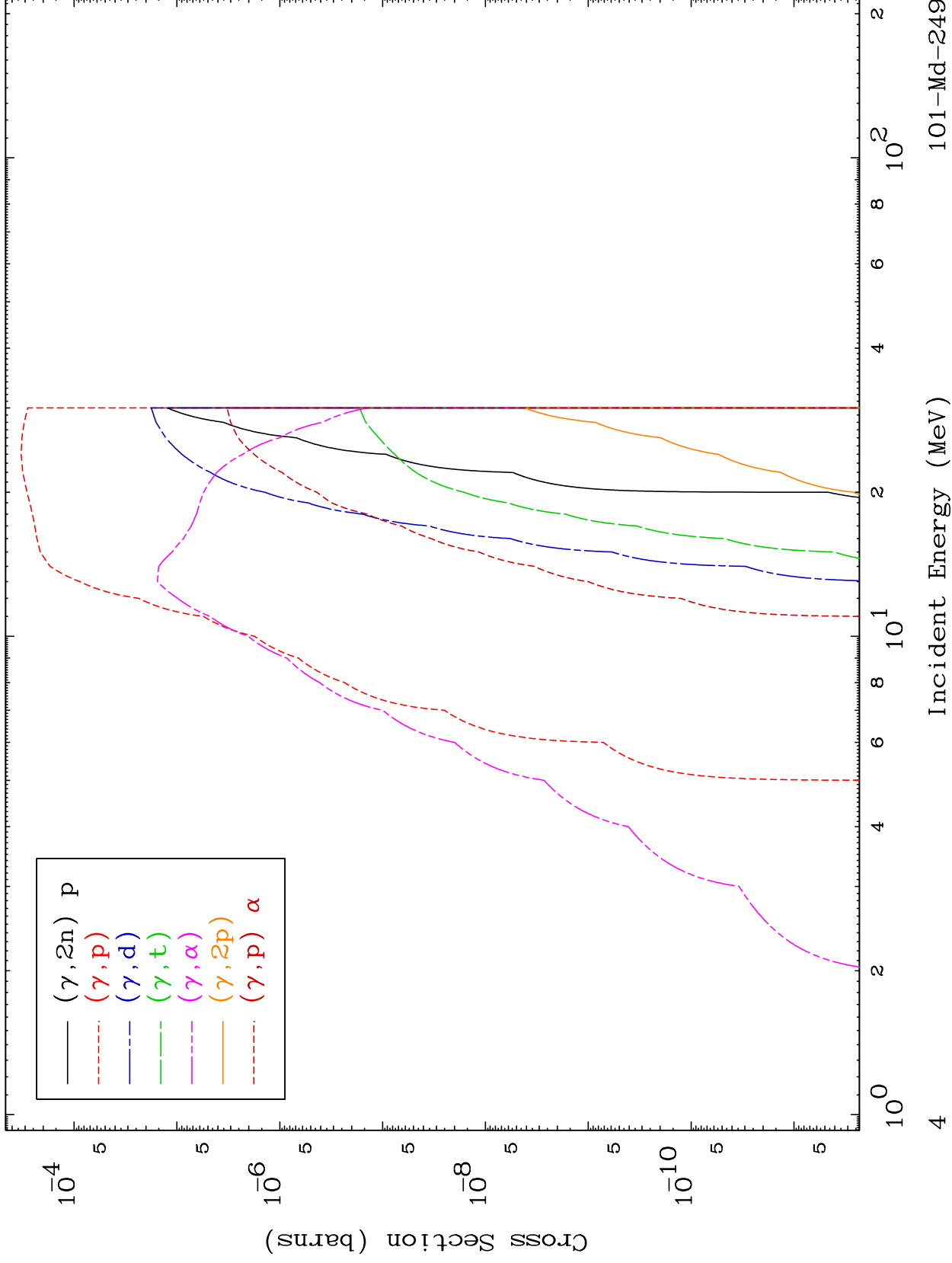
Incident Energy (MeV)

3

MAT 9950

Photon Charged Particle  
0 Kelvin Cross Sections

101-Md-249



101-Md-249

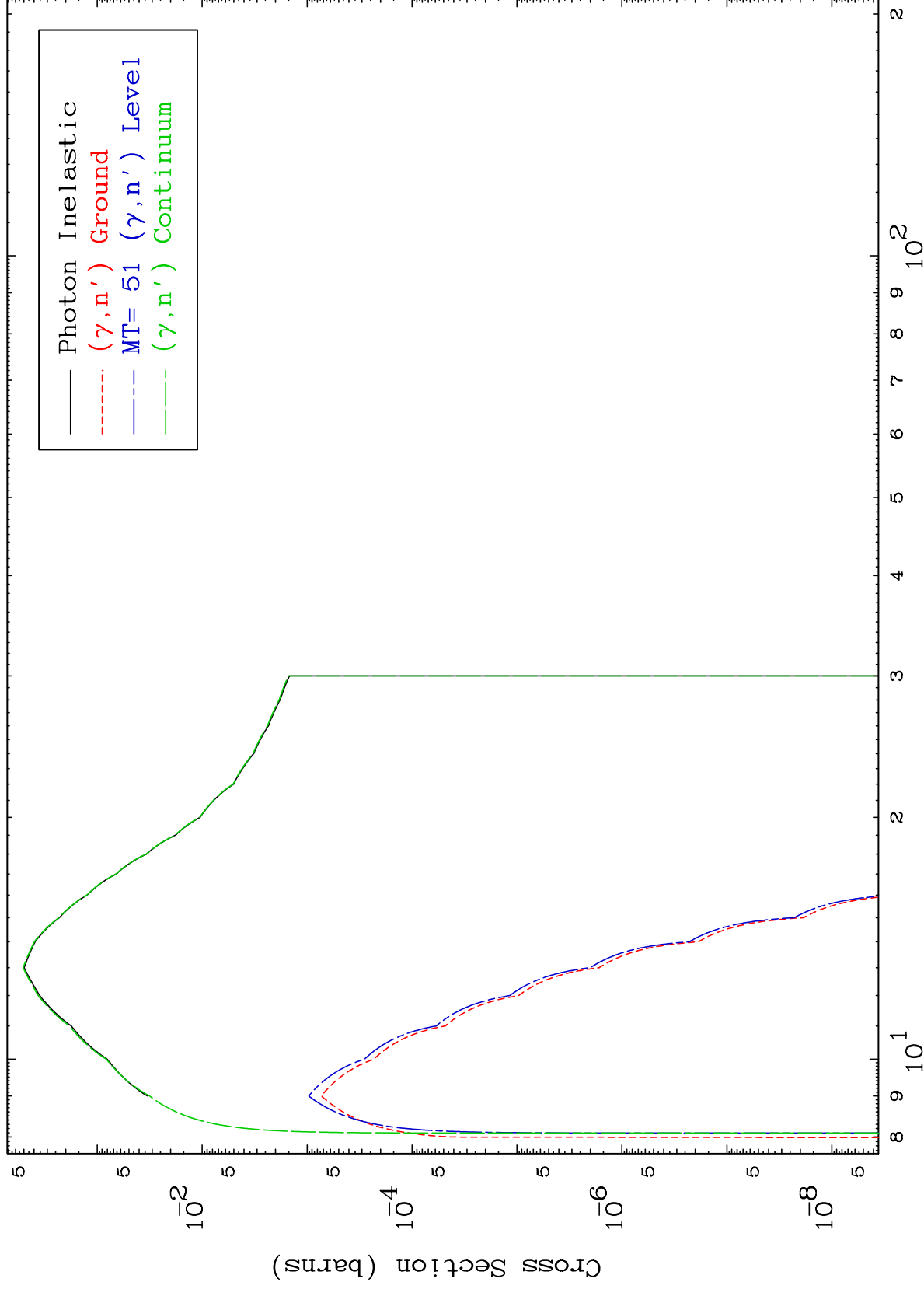
Incident Energy (MeV)

MAT 9950

$(\gamma, n')$  Level

101-Md-249

0 Kelvin Cross Sections



Incident Energy (MeV)

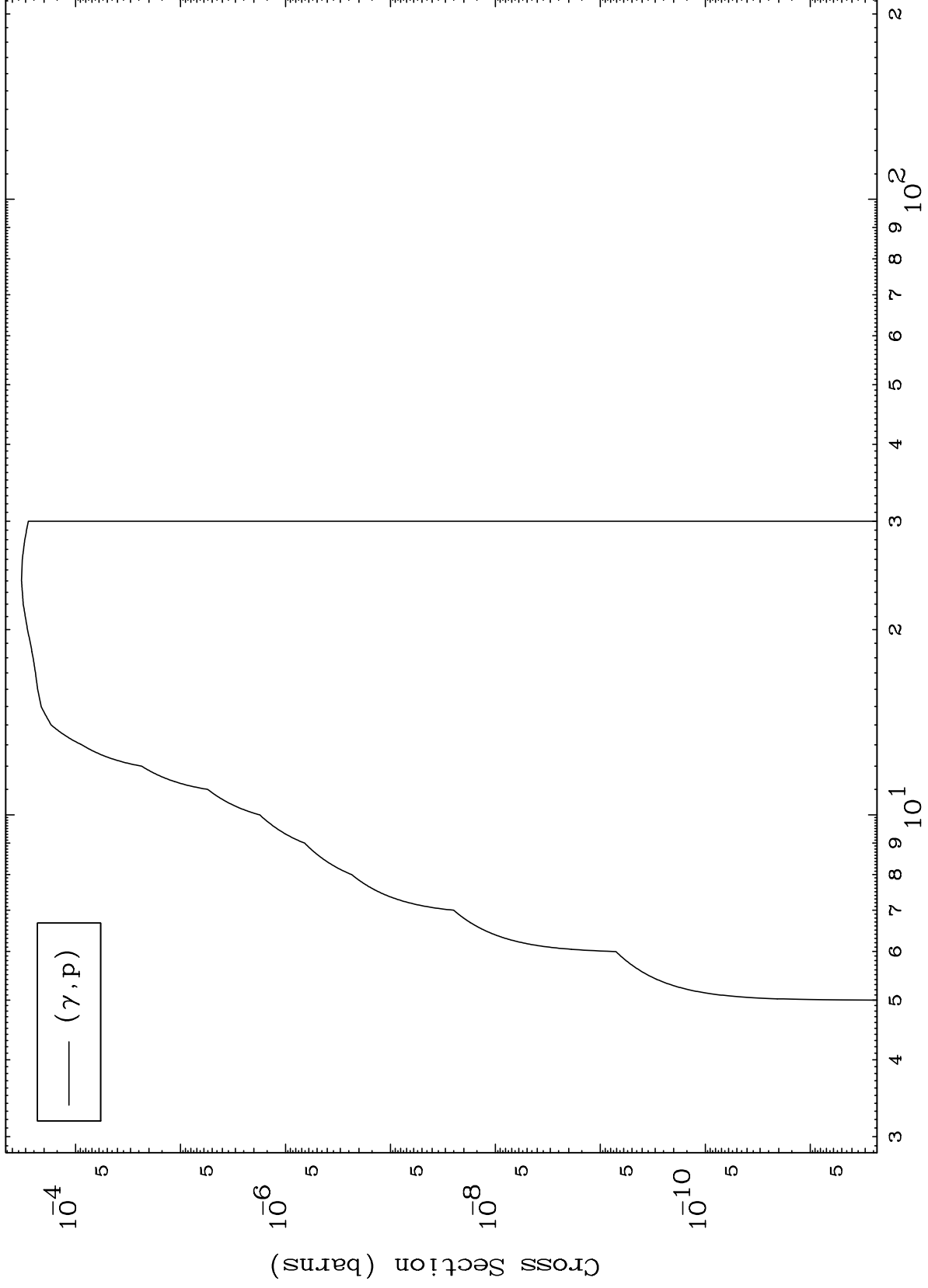
101-Md-249

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( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



( $\gamma, p$ )

Incident Energy (MeV)

101-Md-249

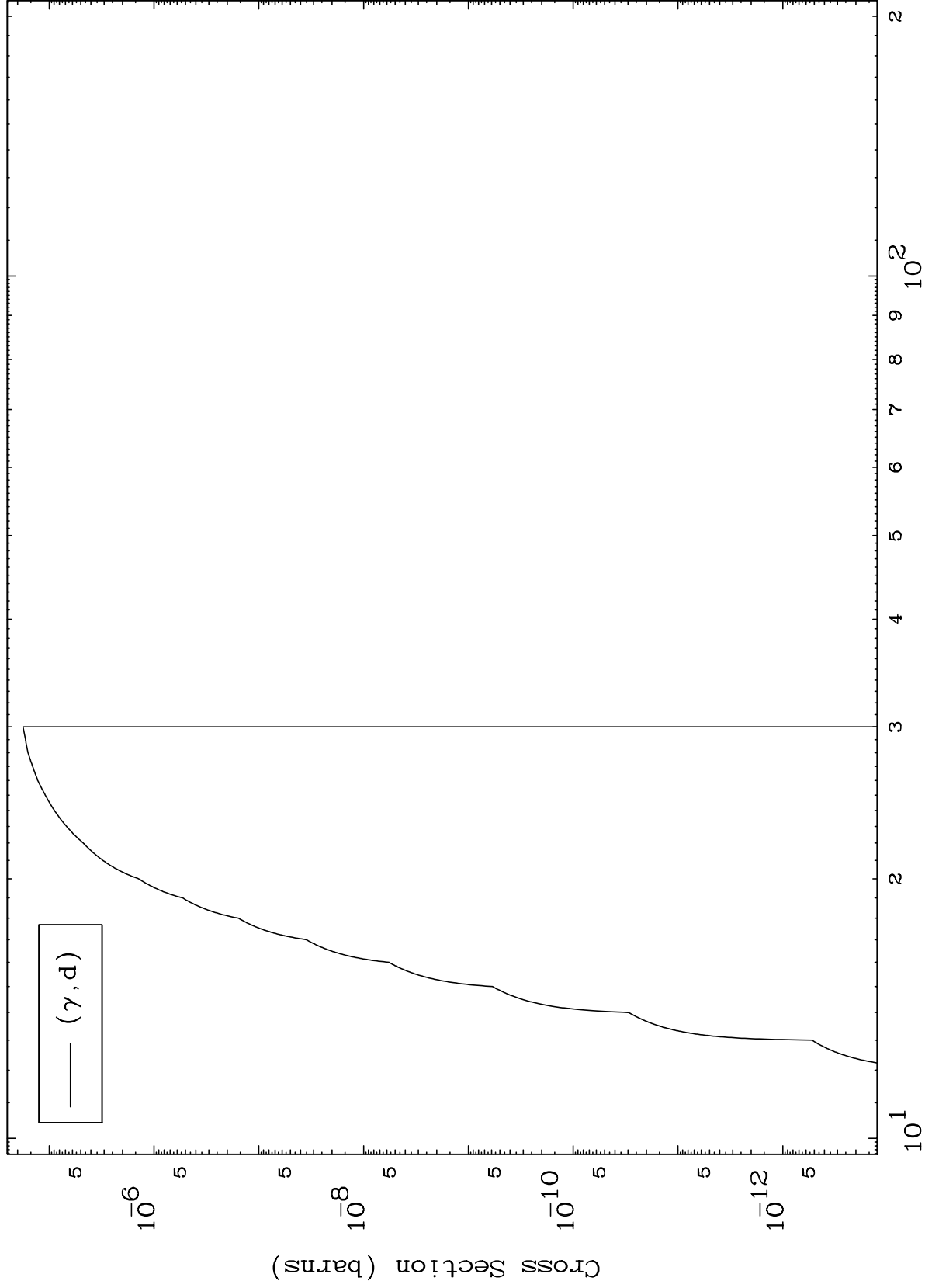
6

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( $\gamma, d$ ) Levels

101-Md-249

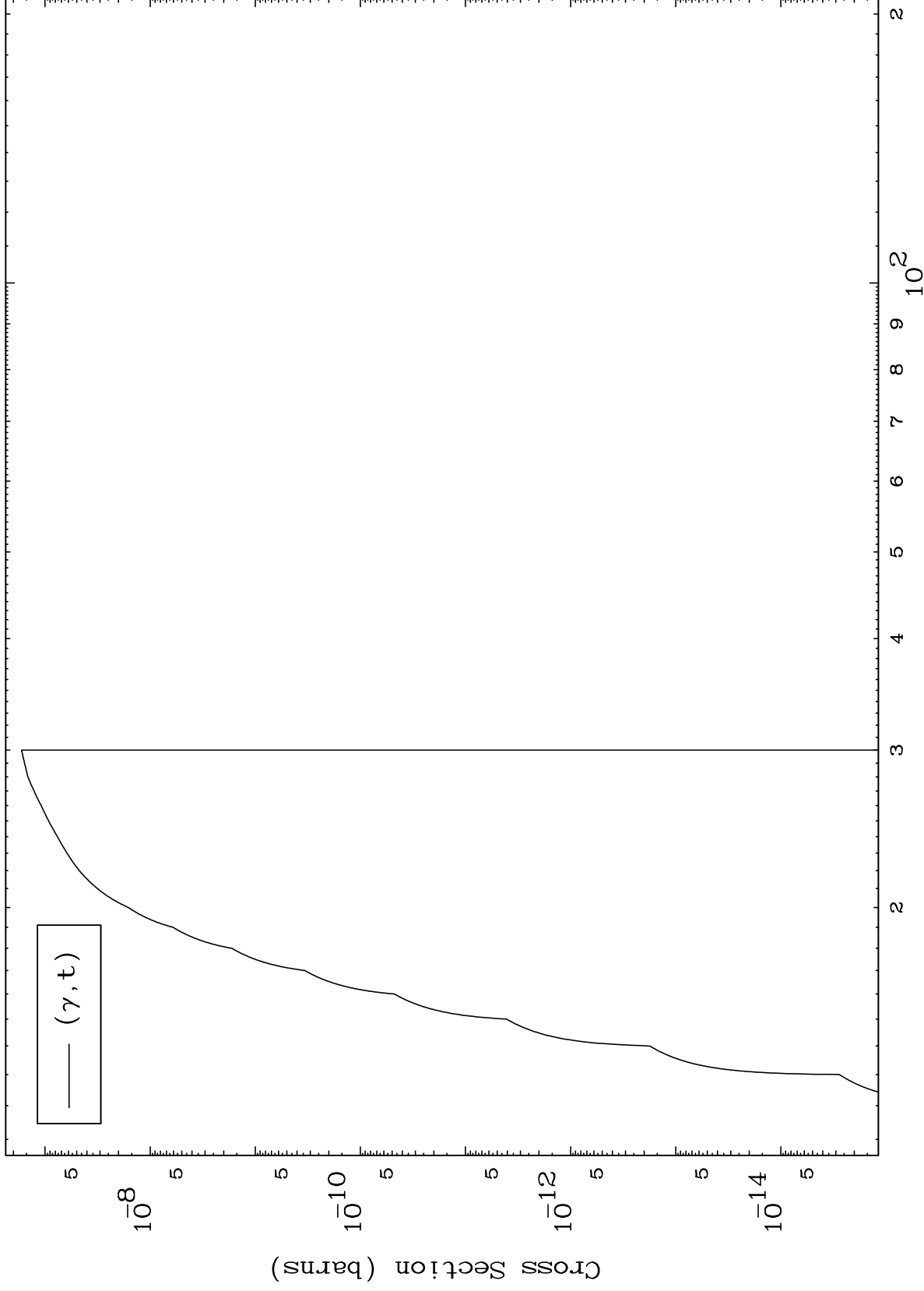
0 Kelvin Cross Sections



Incident Energy (MeV)

101-Md-249



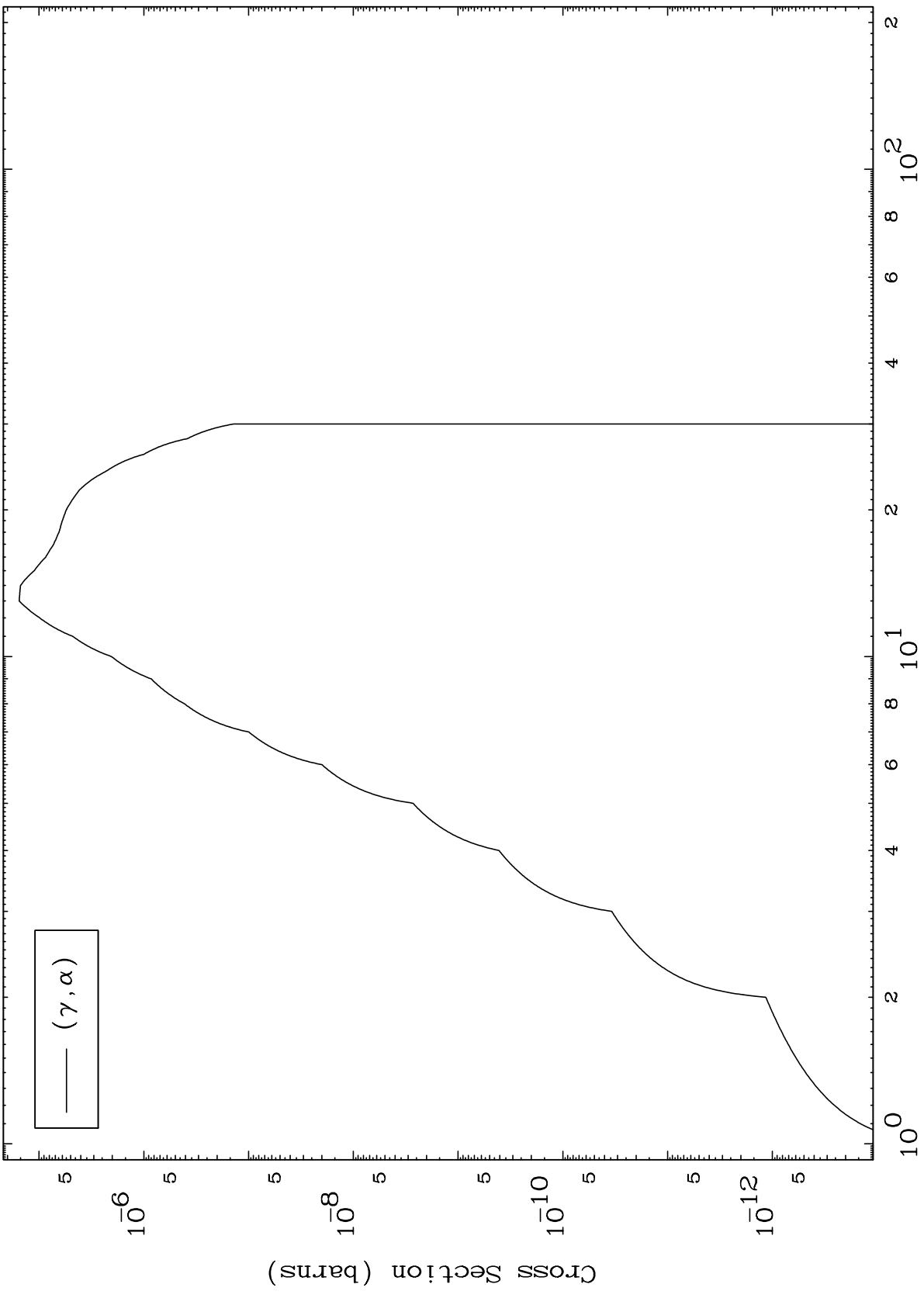


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( $\gamma, \alpha$ ) Levels

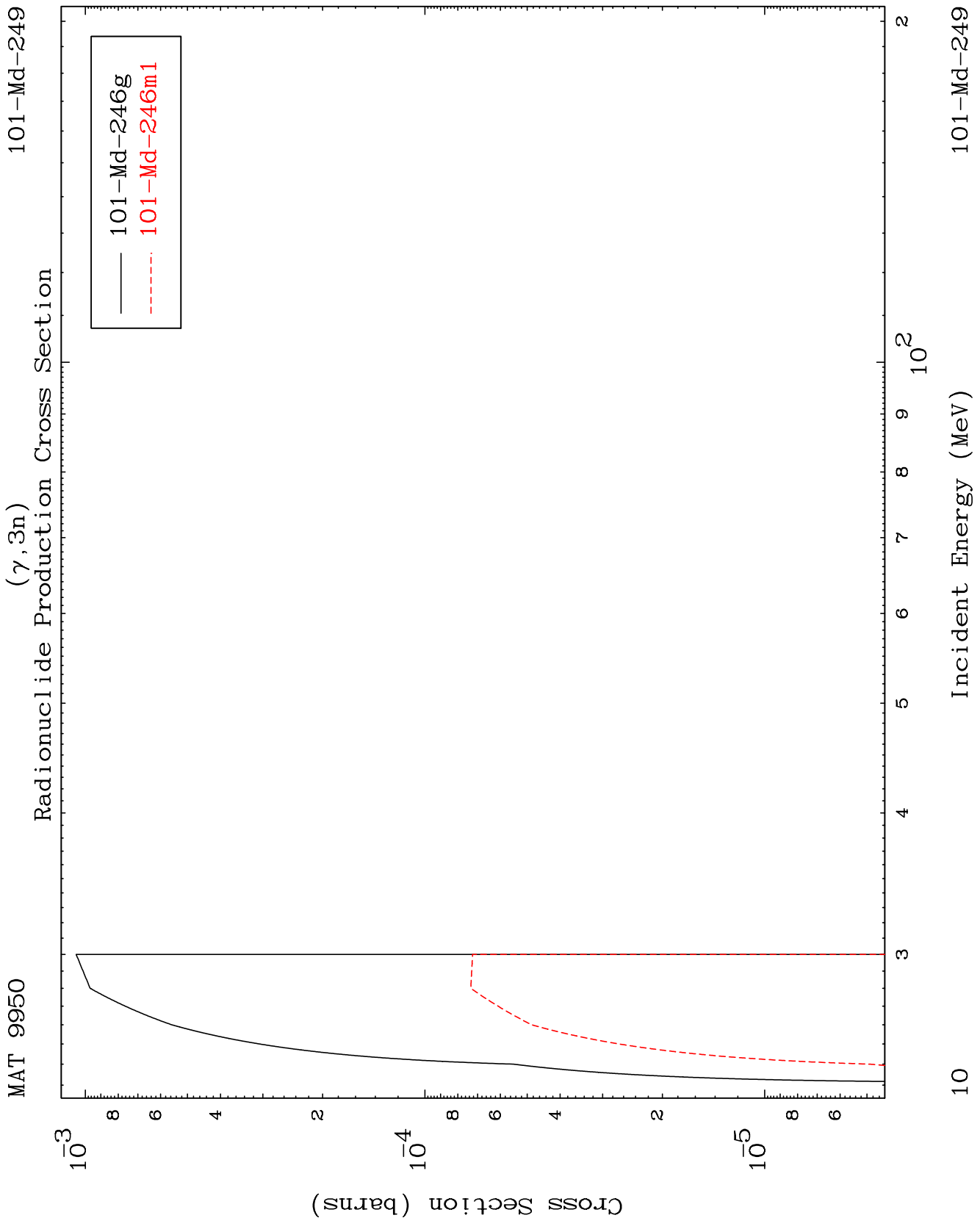
101-Md-249

0 Kelvin Cross Sections



Incident Energy (MeV)

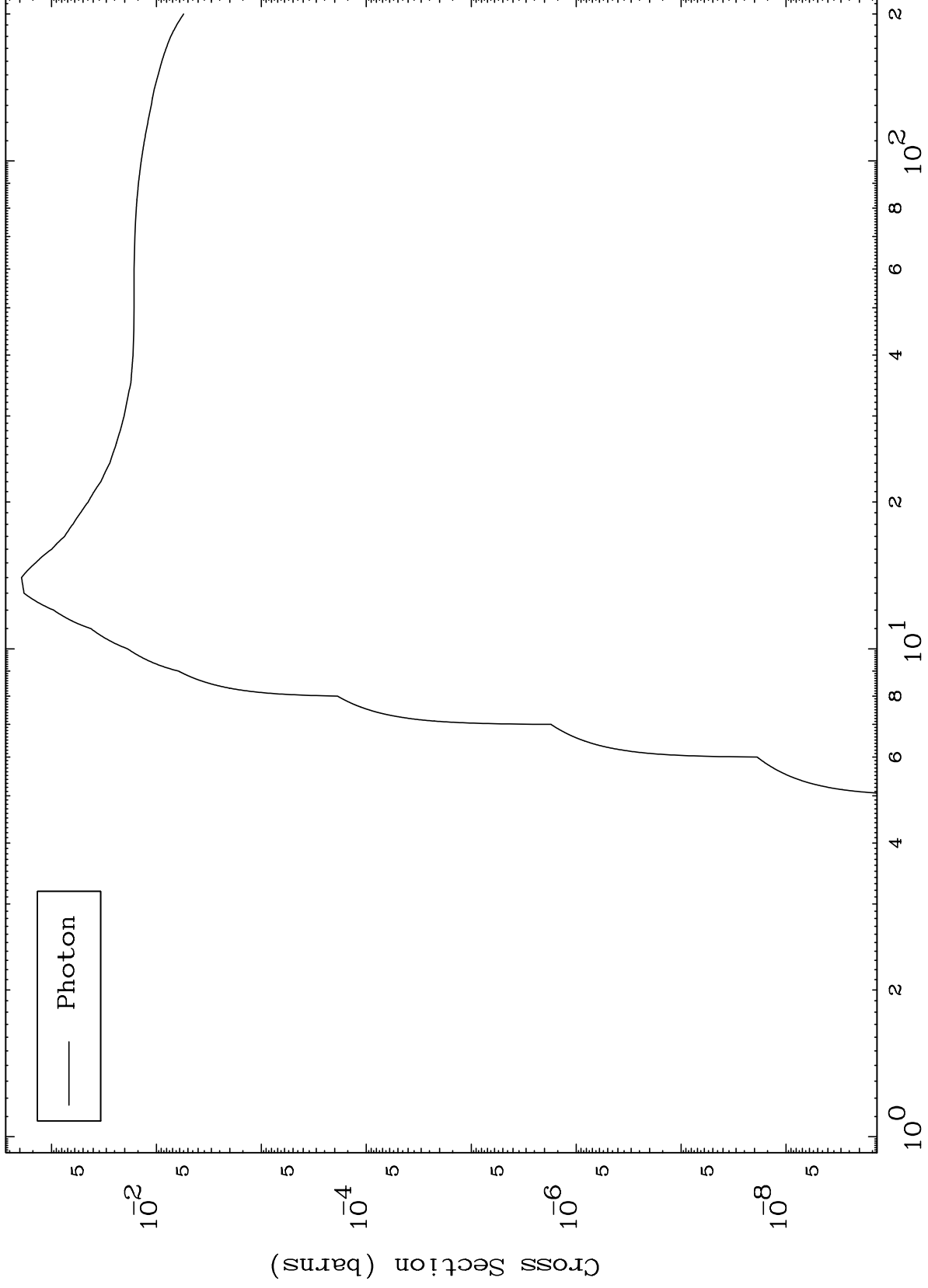
101-Md-249



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Photon Fission  
Radionuclide Production Cross Section

101-Md-249



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Incident Energy (MeV)

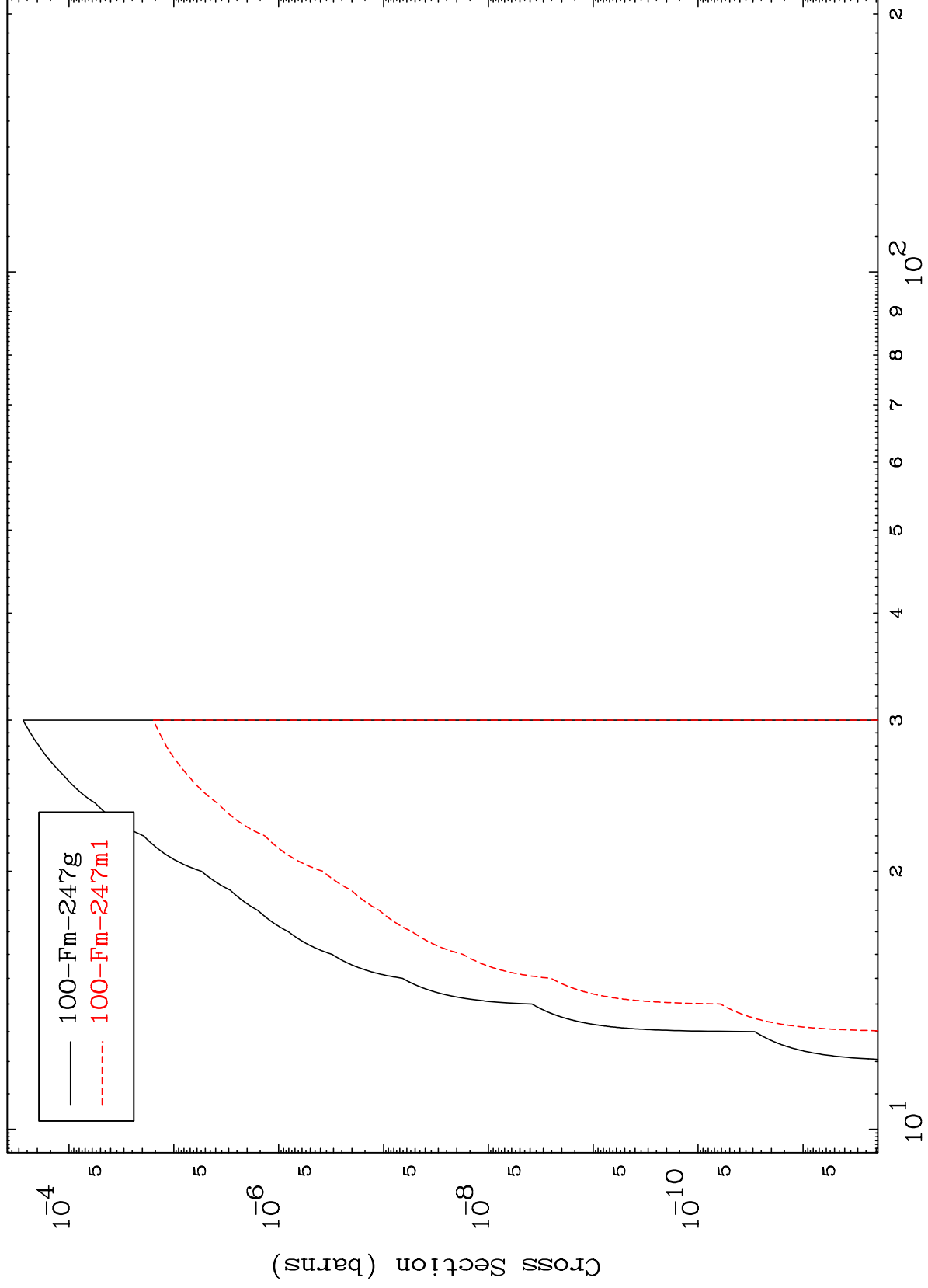
101-Md-249

MAT 9950

( $\gamma, n'$ ) p

101-Md-249

Radionuclide Production Cross Section



Incident Energy (MeV)

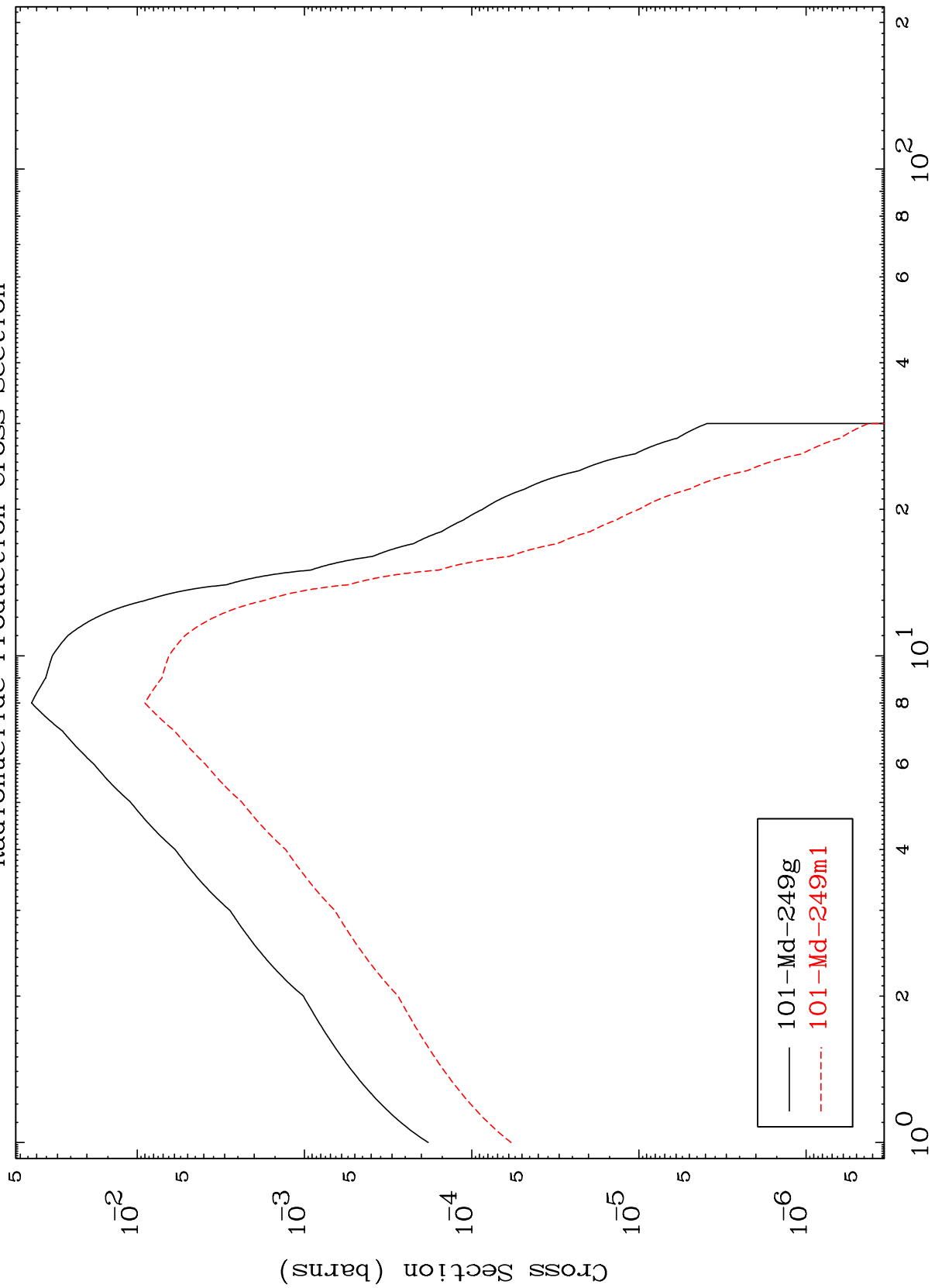
101-Md-249

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101-Md-249

$(\gamma, \gamma)$   
Radionuclide Production Cross Section



— 101-Md-249g  
- - - 101-Md-249m1

101-Md-249

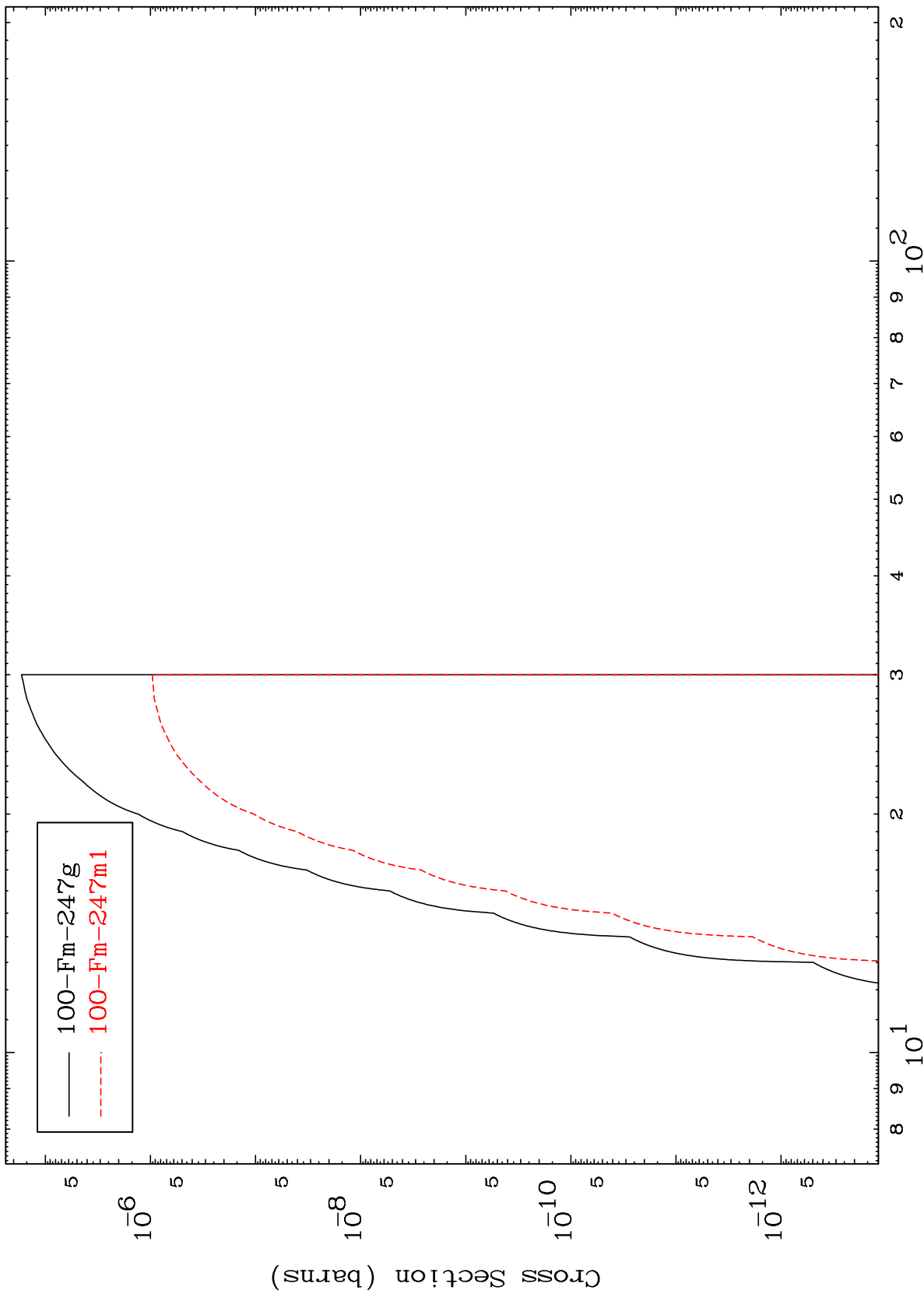
Incident Energy (MeV)

13

MAT 9950

101-Md-249

( $\gamma, d$ )  
Radionuclide Production Cross Section



101-Md-249

Incident Energy (MeV)

14