

Program EVALPLOT  
(Version 2017-1)

by

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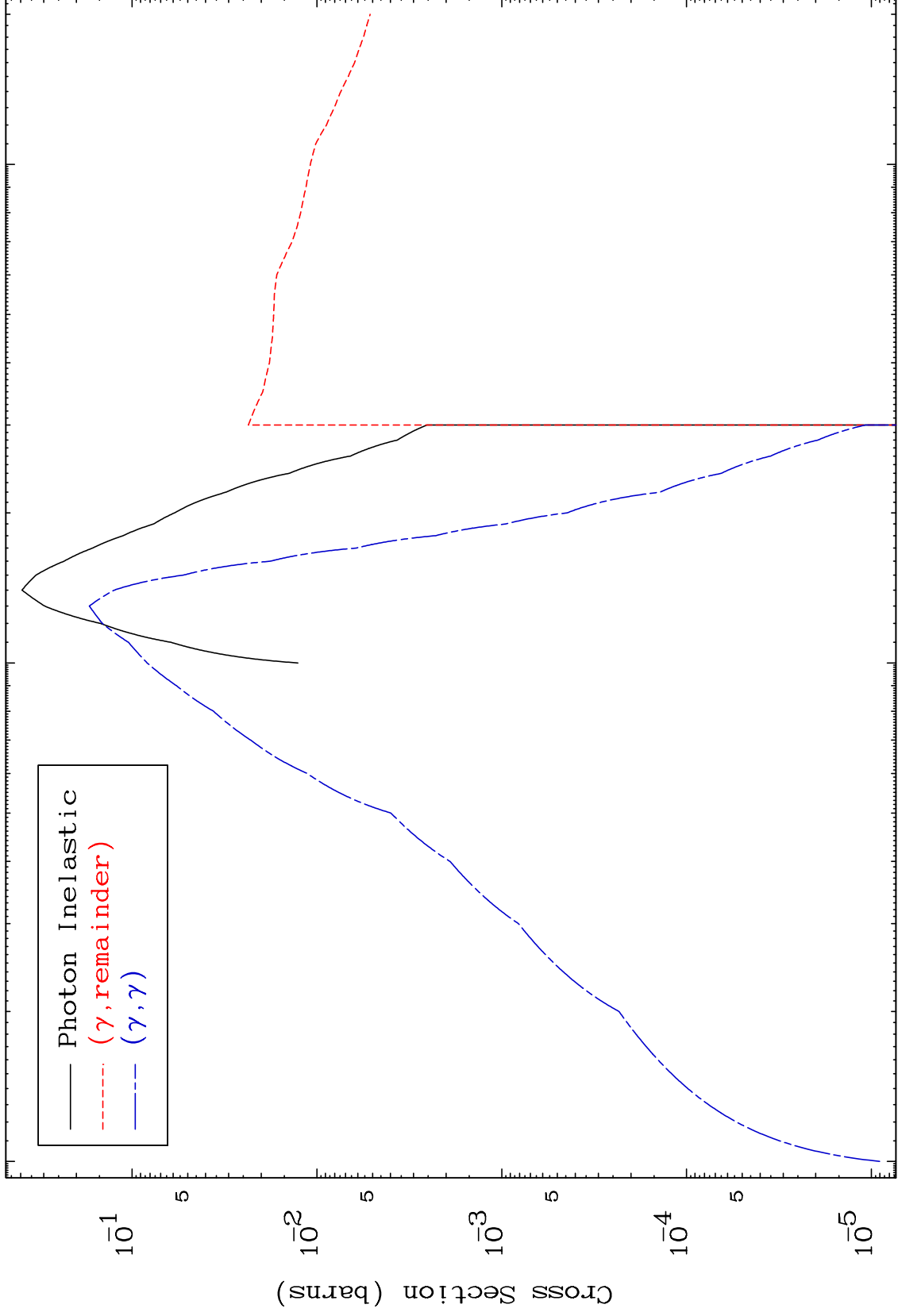
Press Mouse Button to Start

MAT 8610

Photon Major

86-Rn-206

0 Kelvin Cross Sections



— Photon Inelastic  
- - -  $(\gamma, \text{remainder})$   
- . -  $(\gamma, \gamma)$

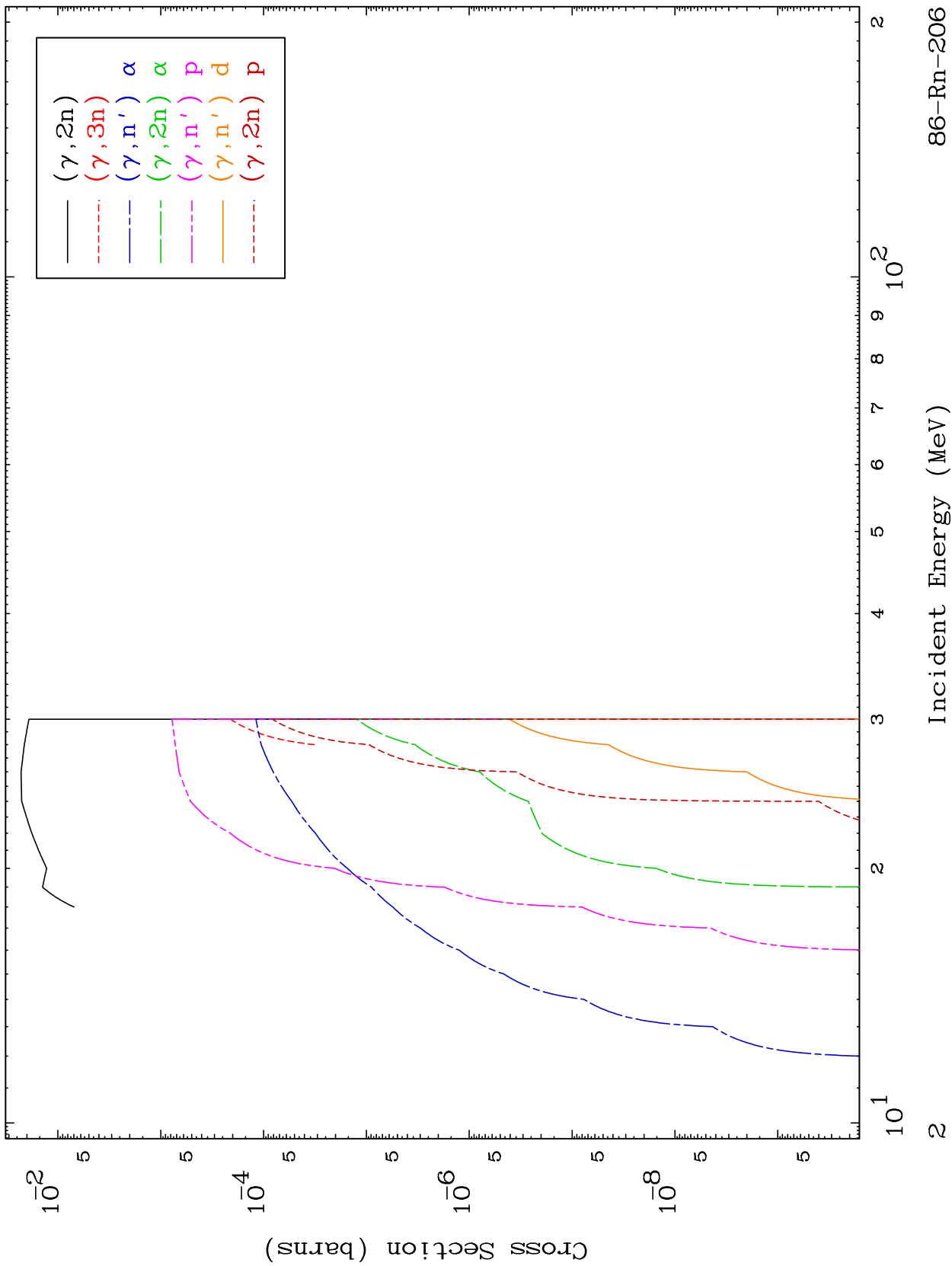
Incident Energy (MeV)

86-Rn-206

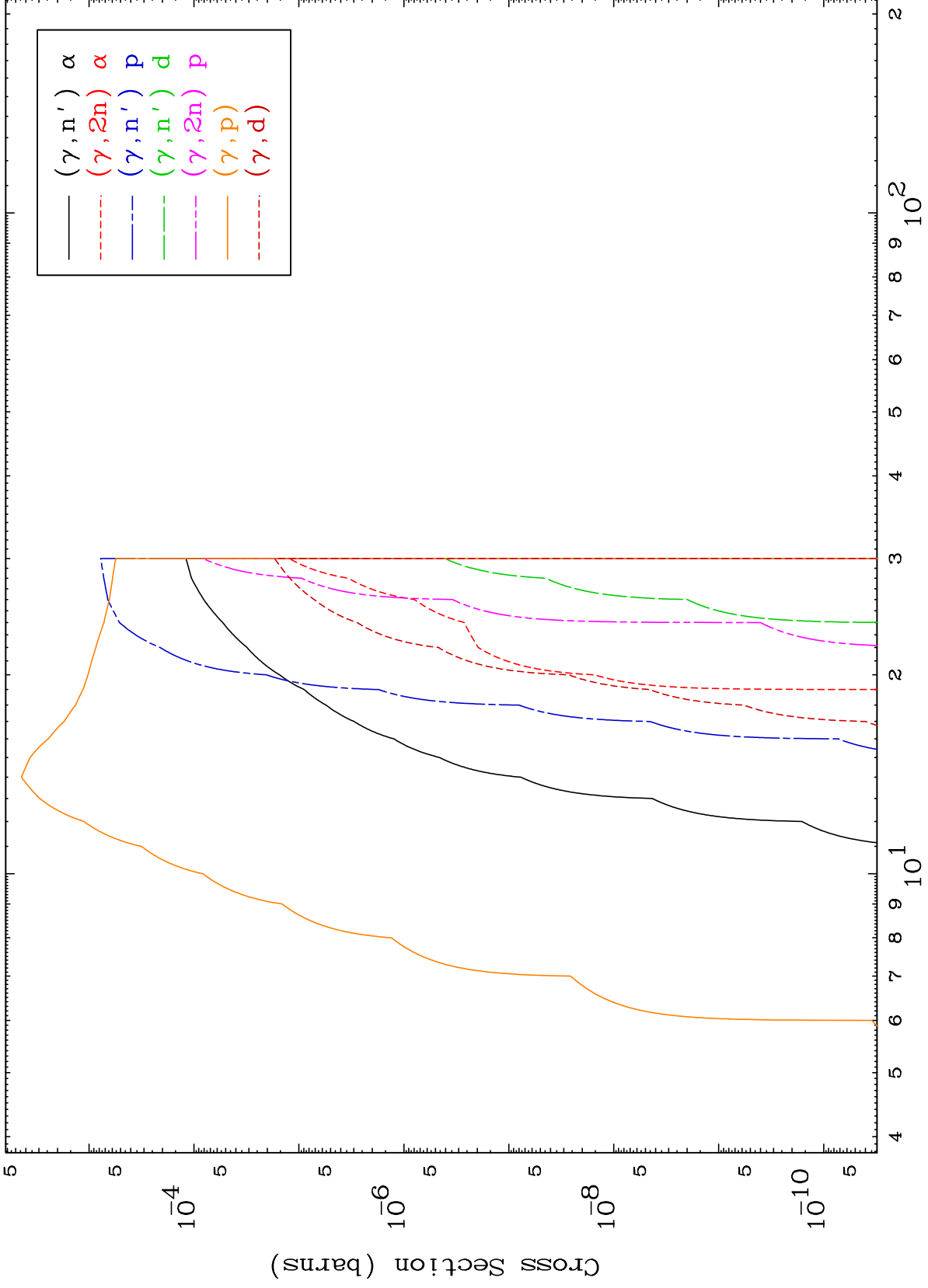
MAT 8610

Photon Neutron Production  
0 Kelvin Cross Sections

86-Rn-206



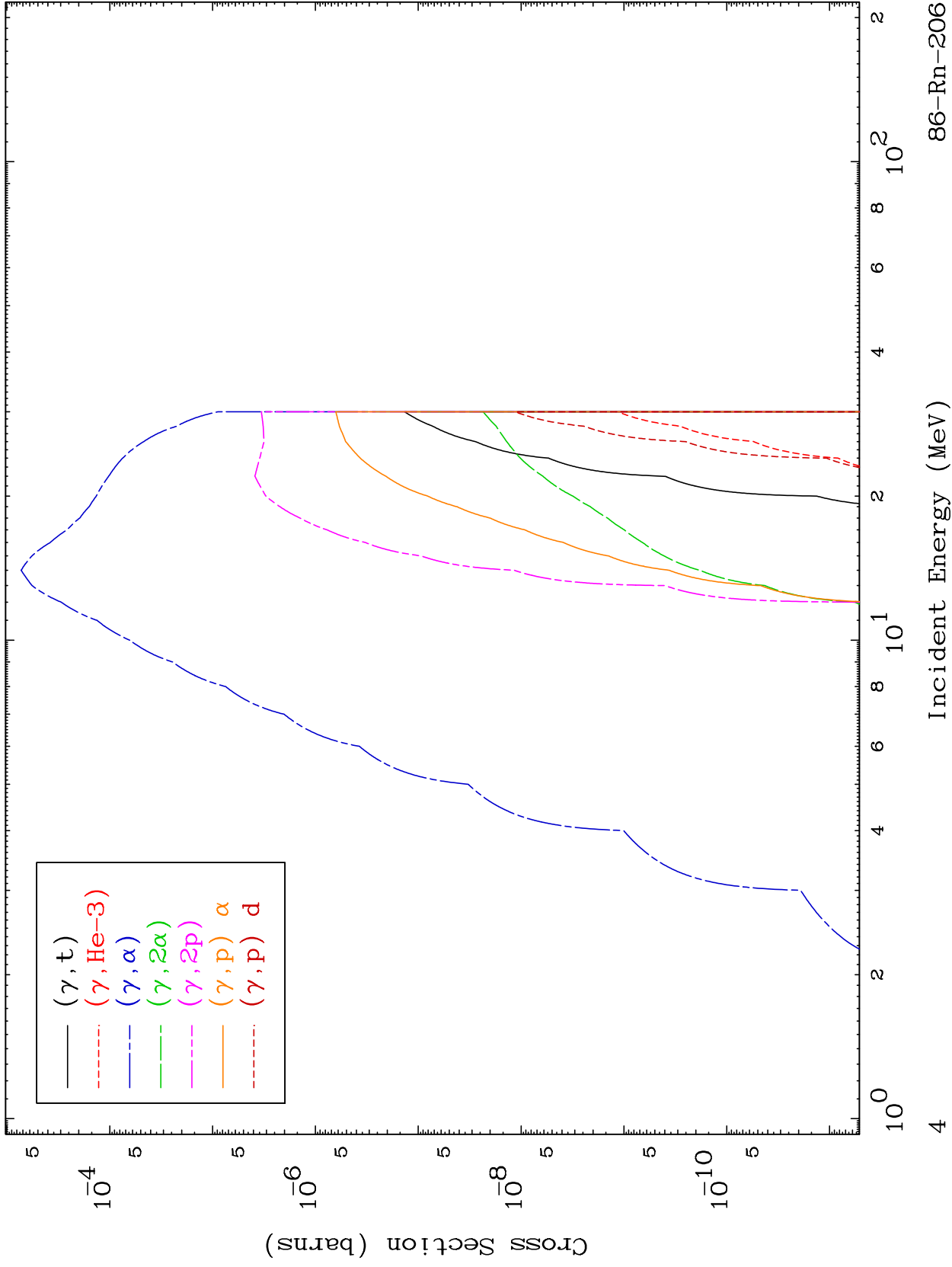
86-Rn-206



MAT 8610

Photon Charged Particle  
0 Kelvin Cross Sections

86-Rn-206



86-Rn-206

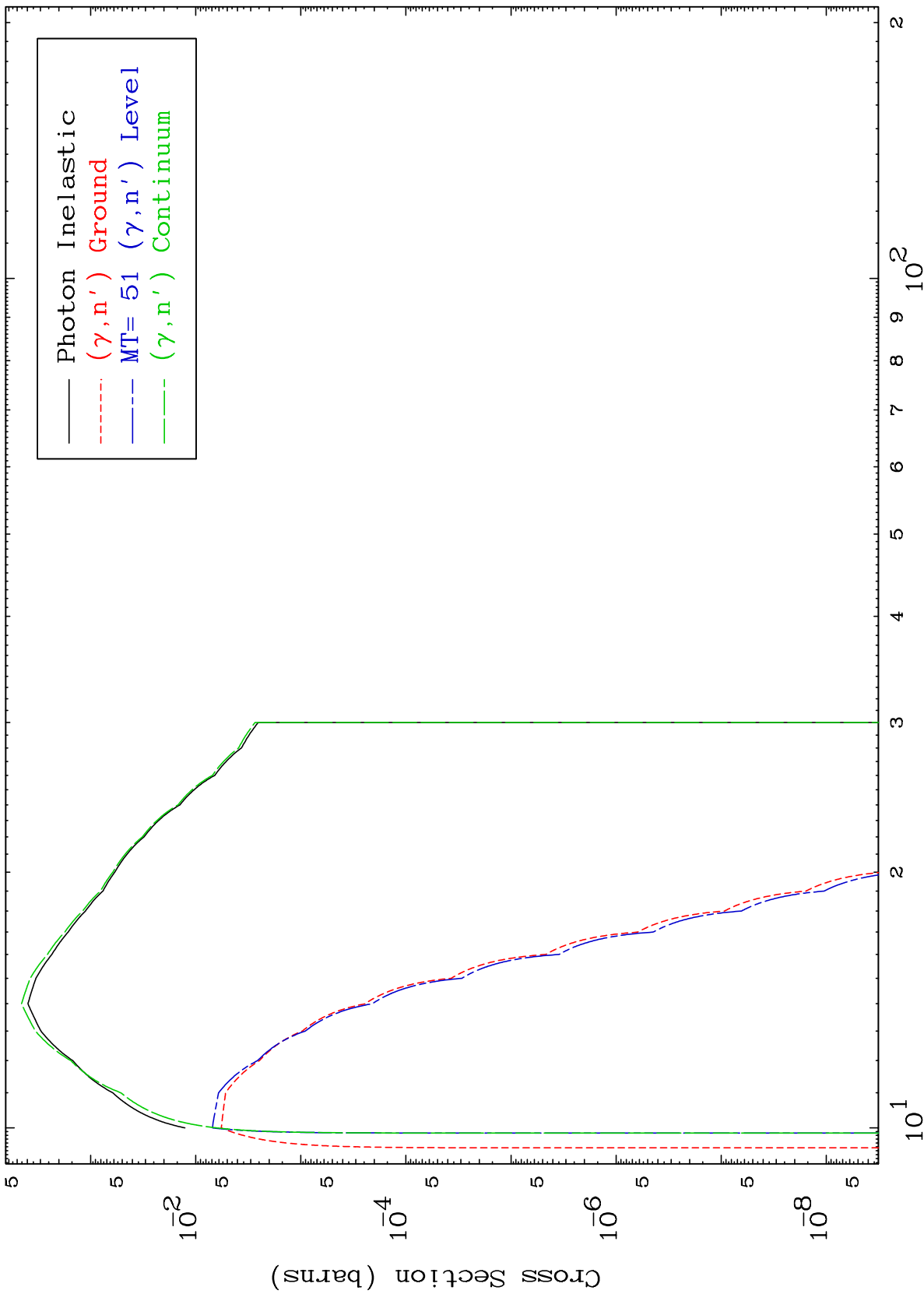
Incident Energy (MeV)

MAT 8610

( $\gamma, n'$ ) Level

86-Rn-206

0 Kelvin Cross Sections



Incident Energy (MeV)

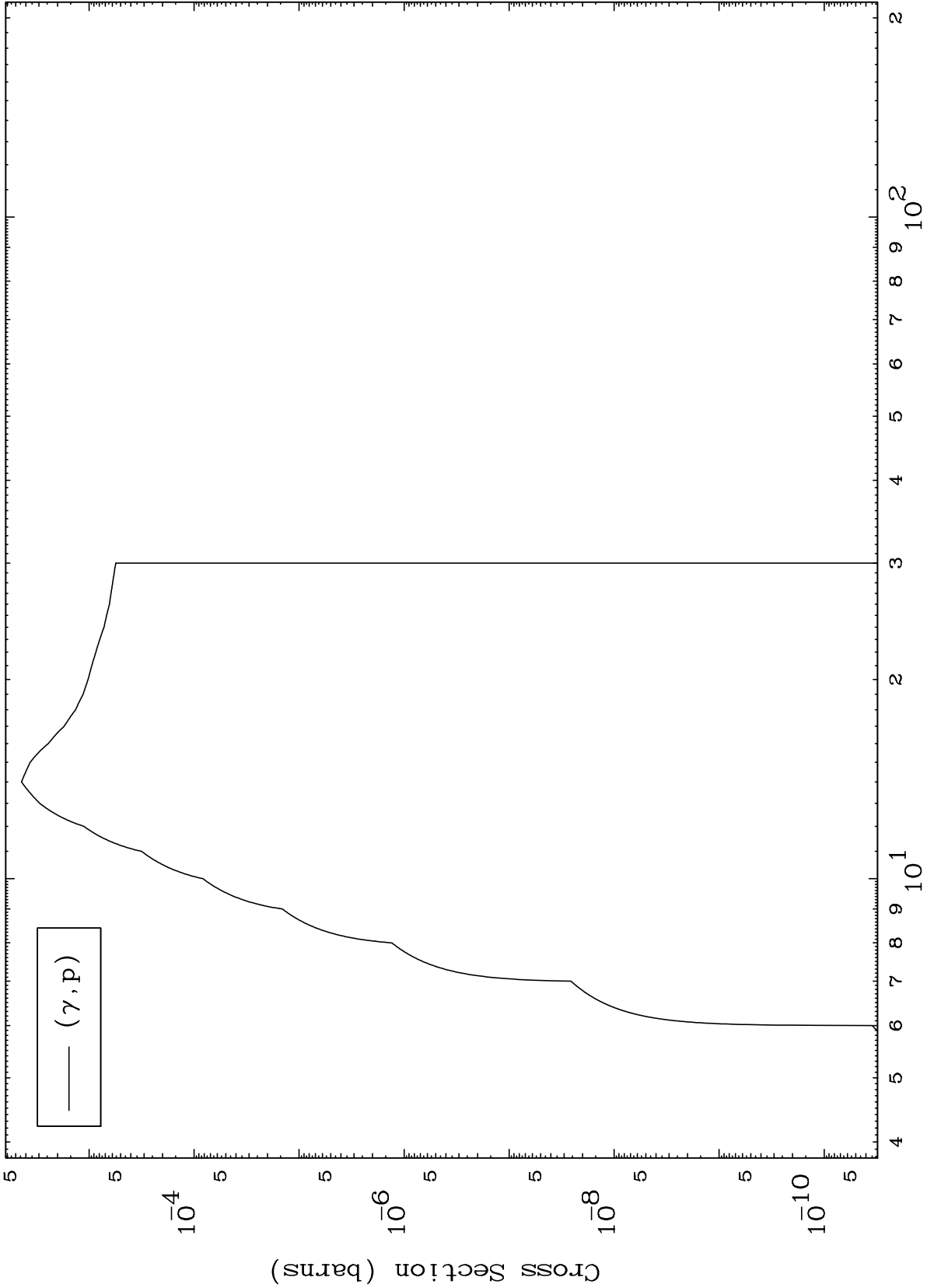
86-Rn-206

MAT 8610

( $\gamma, p$ ) Levels

86-Rn-206

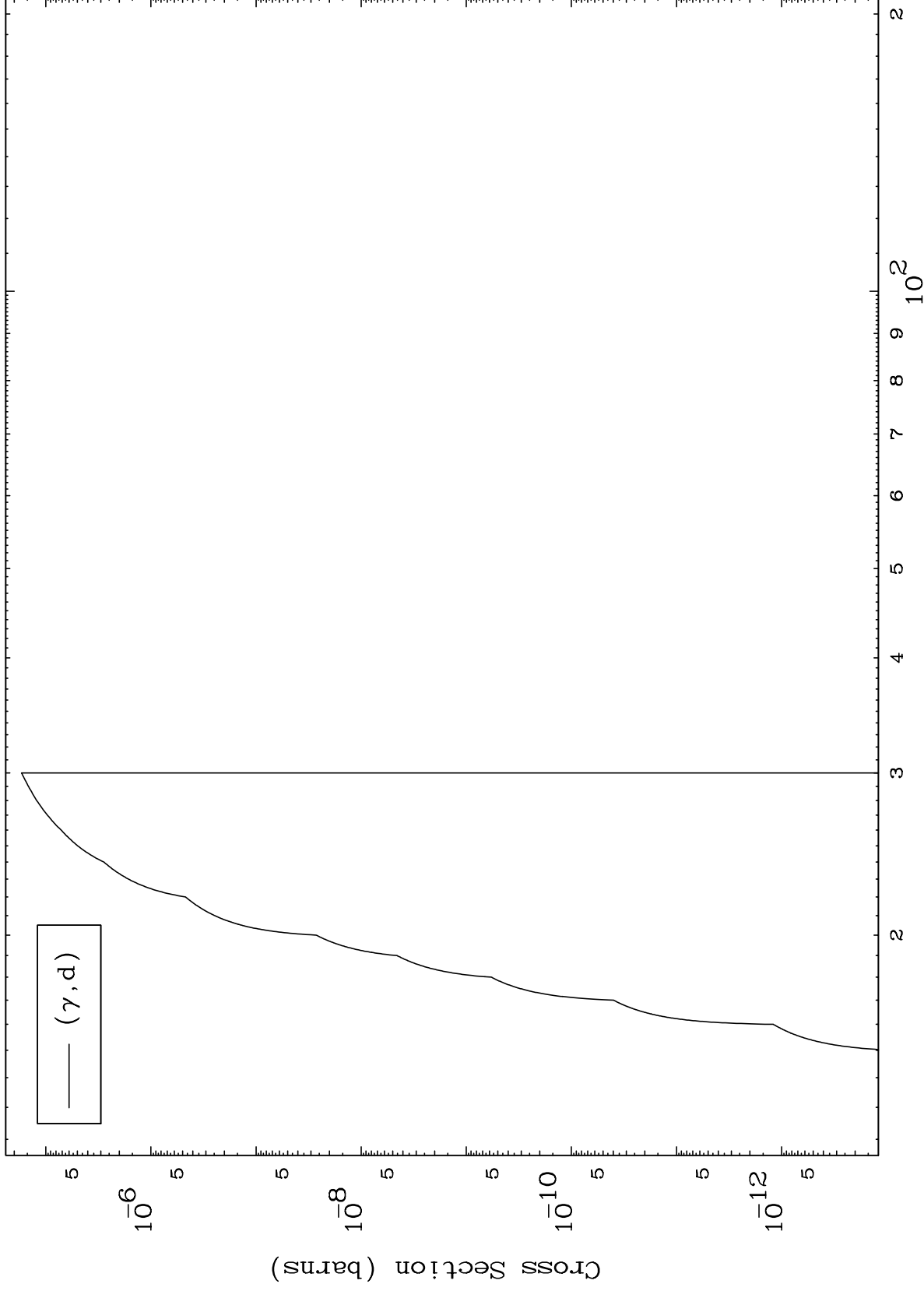
0 Kelvin Cross Sections



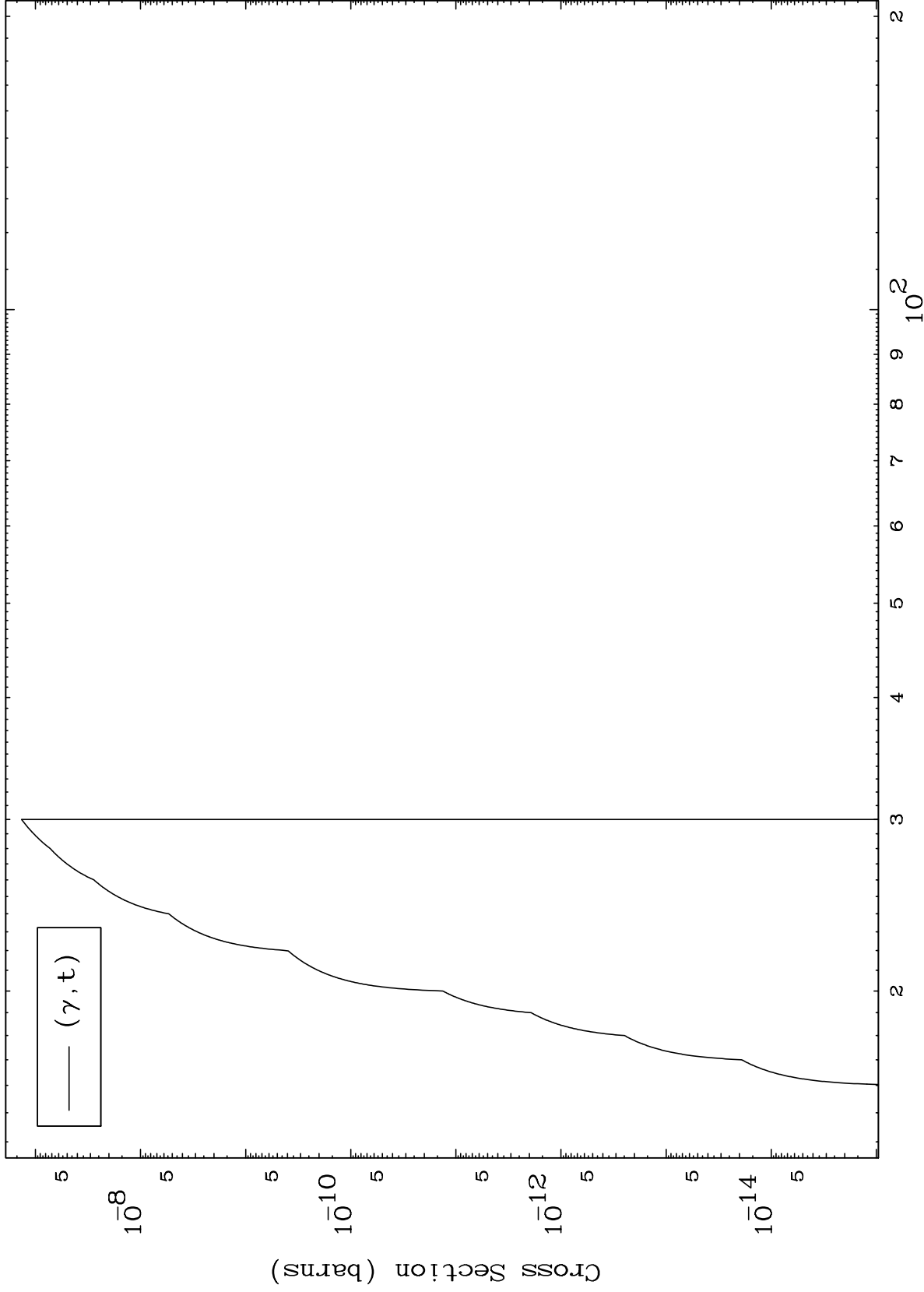
6

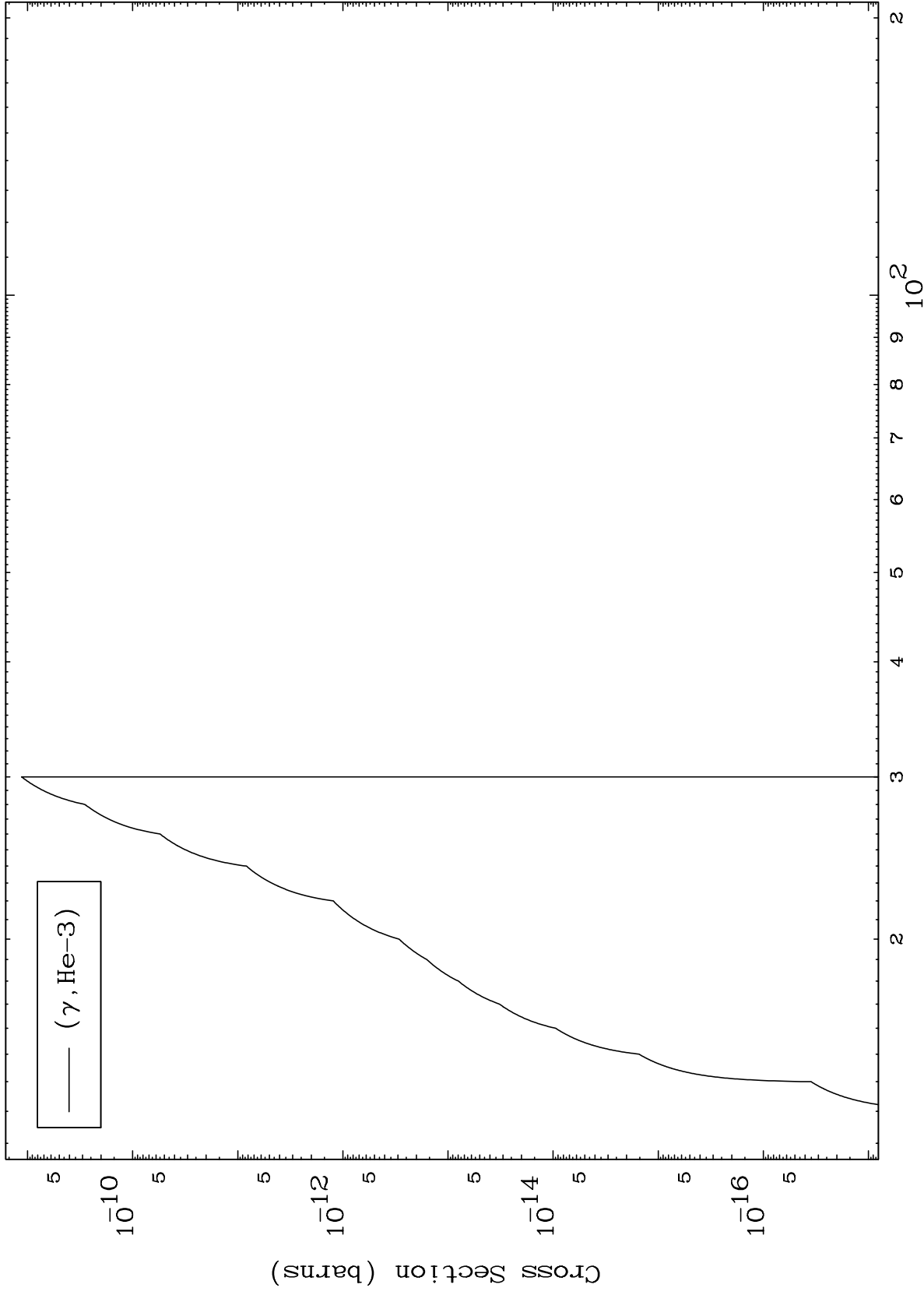
Incident Energy (MeV)

86-Rn-206







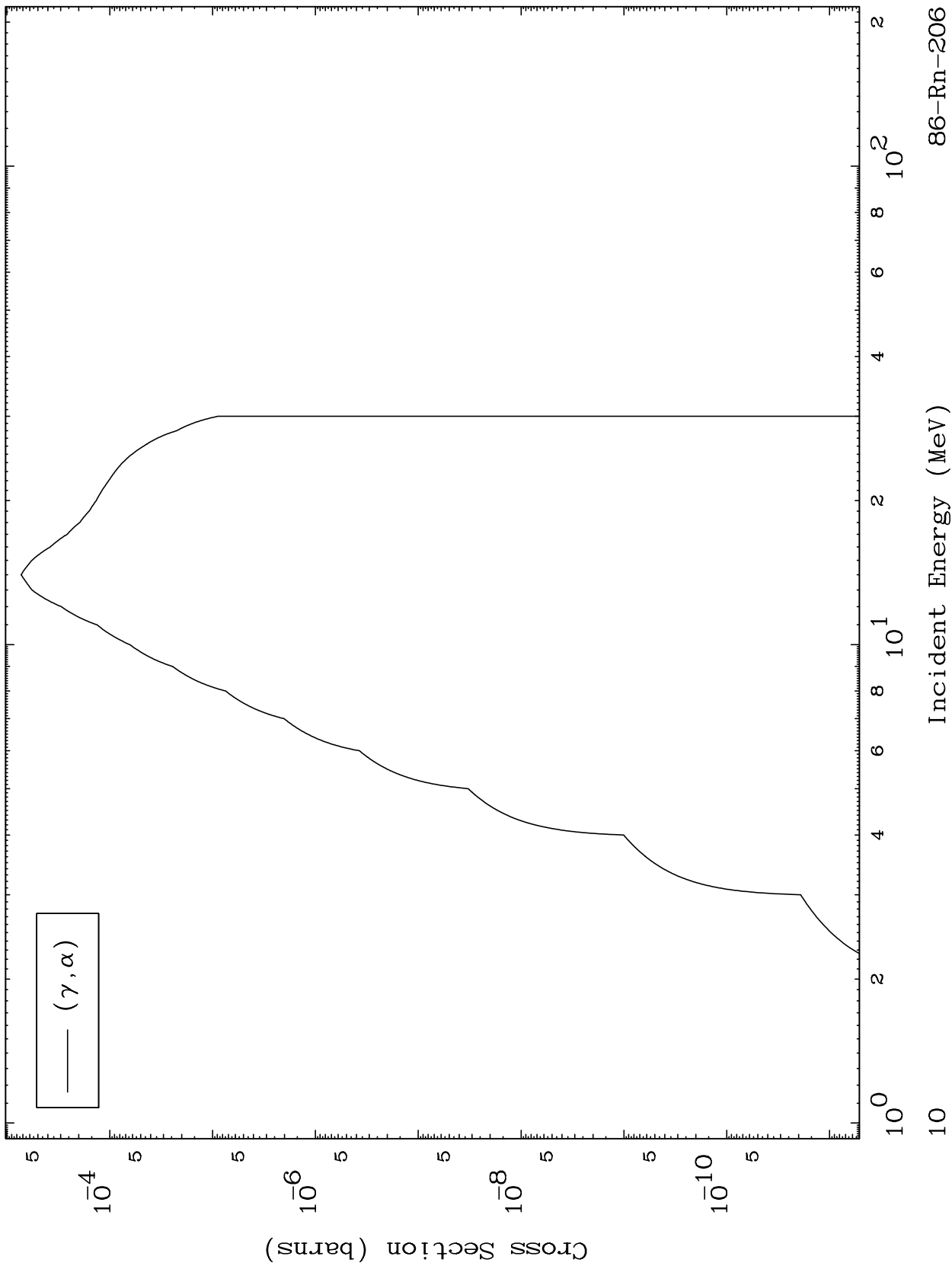


MAT 8610

( $\gamma, \alpha$ ) Levels

86-Rn-206

0 Kelvin Cross Sections



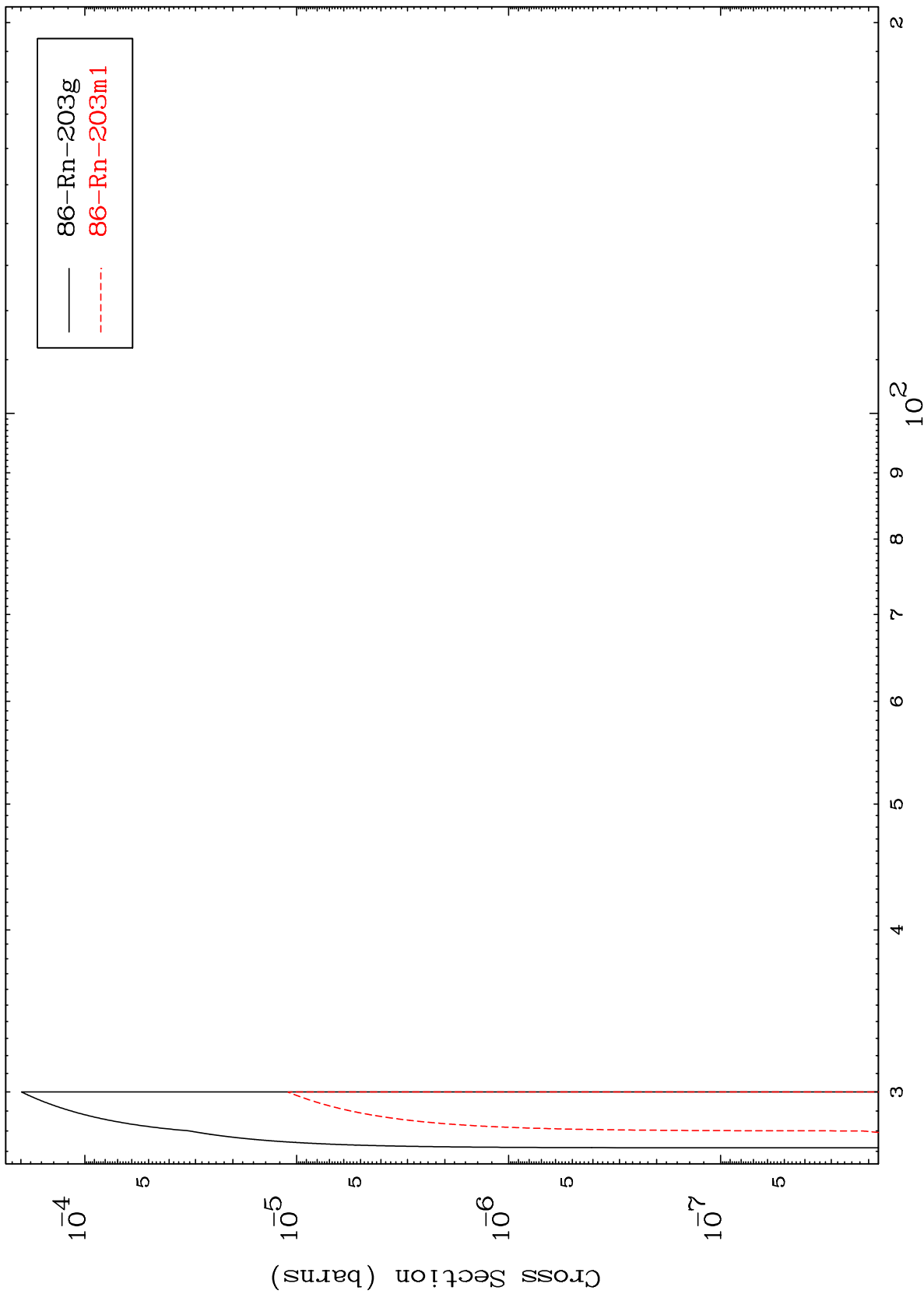
Incident Energy (MeV)

86-Rn-206

MAT 8610

86-Rn-206

( $\gamma, 3n$ )  
Radionuclide Production Cross Section



86-Rn-203g  
86-Rn-203m1

86-Rn-206

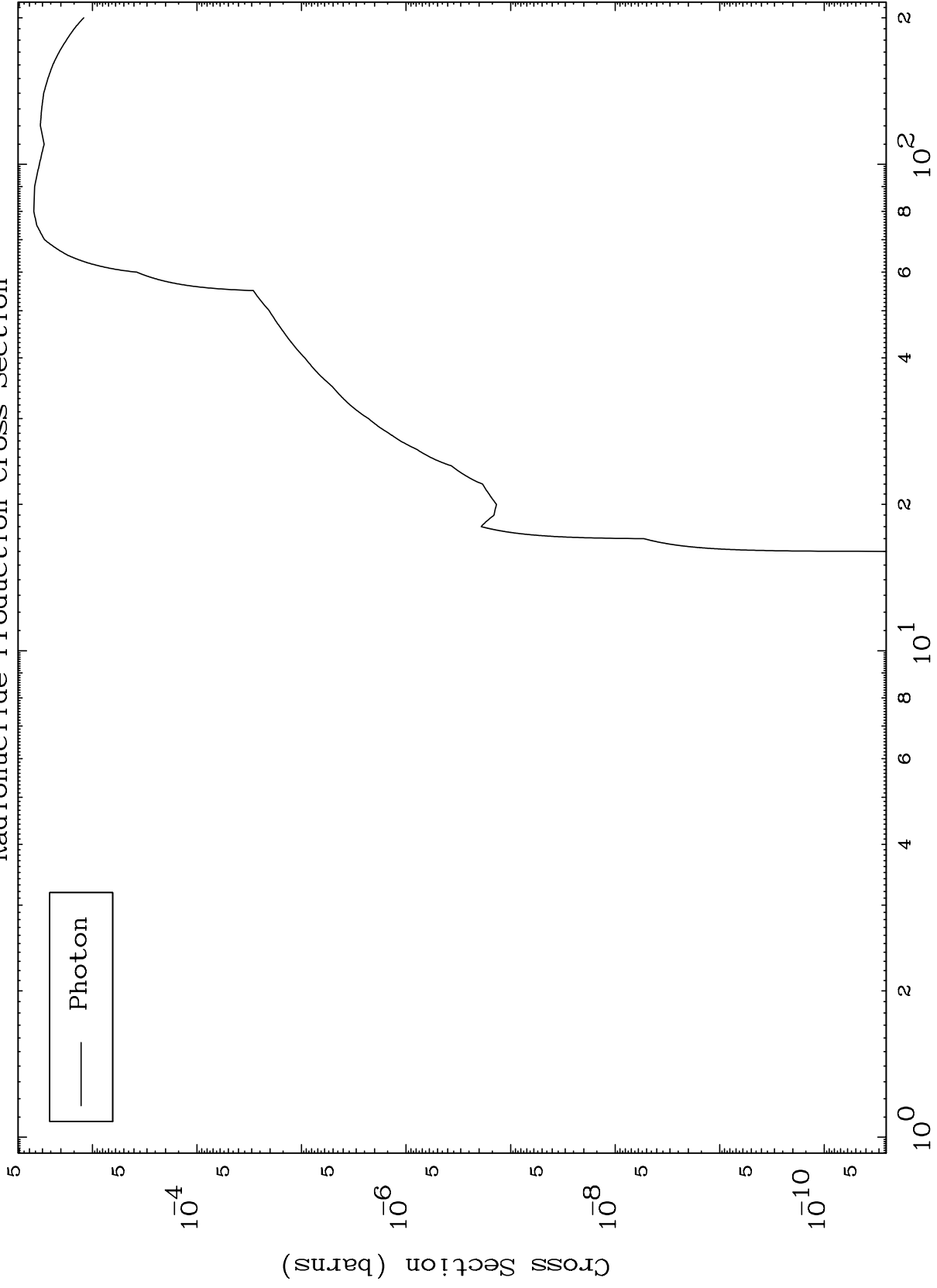
Incident Energy (MeV)

11

MAT 8610

Photon Fission  
Radionuclide Production Cross Section

86-Rn-206



Incident Energy (MeV)

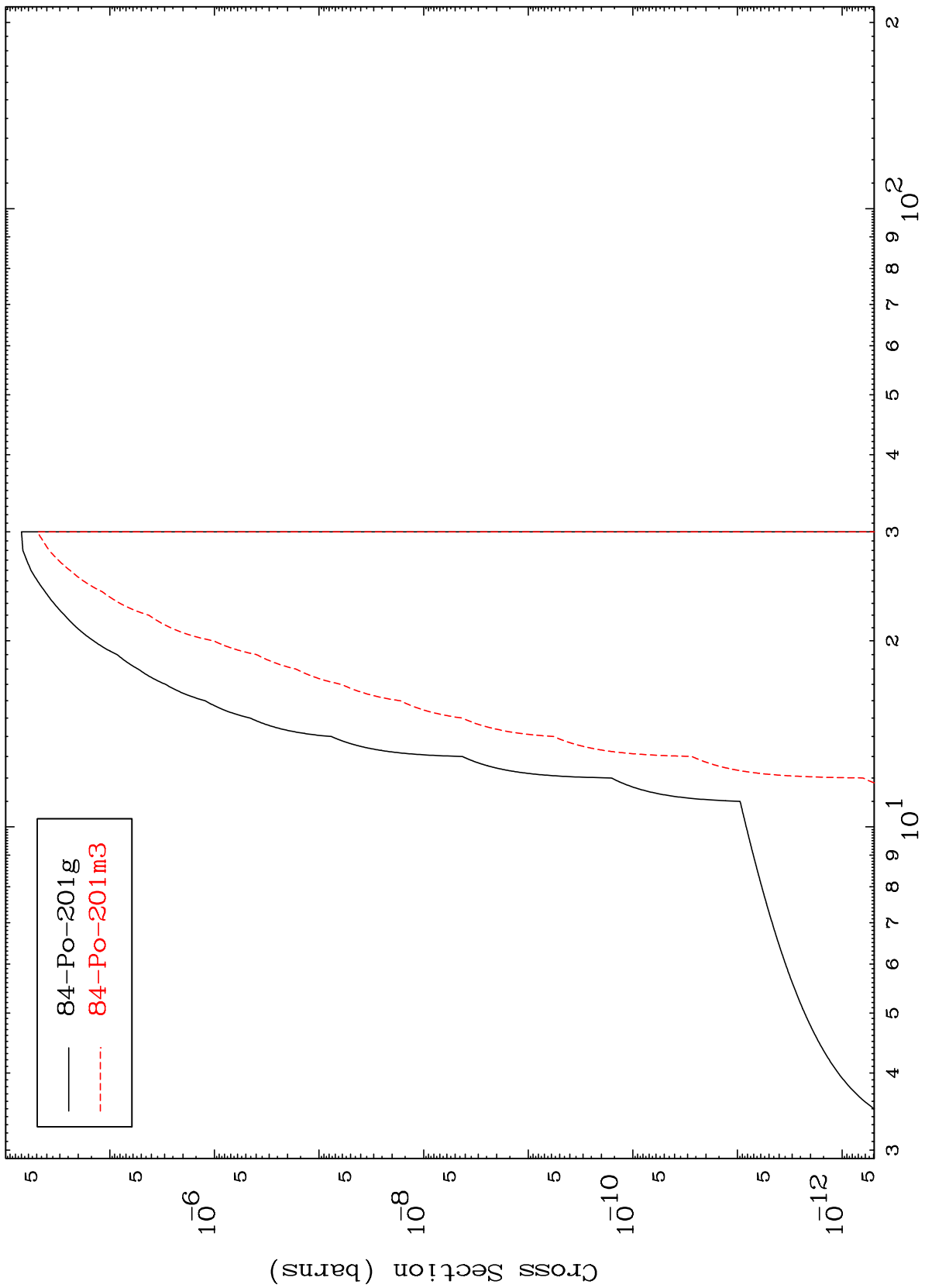
86-Rn-206

MAT 8610

$(\gamma, n')$   $\alpha$

86-Rn-206

Radionuclide Production Cross Section



13

Incident Energy (MeV)

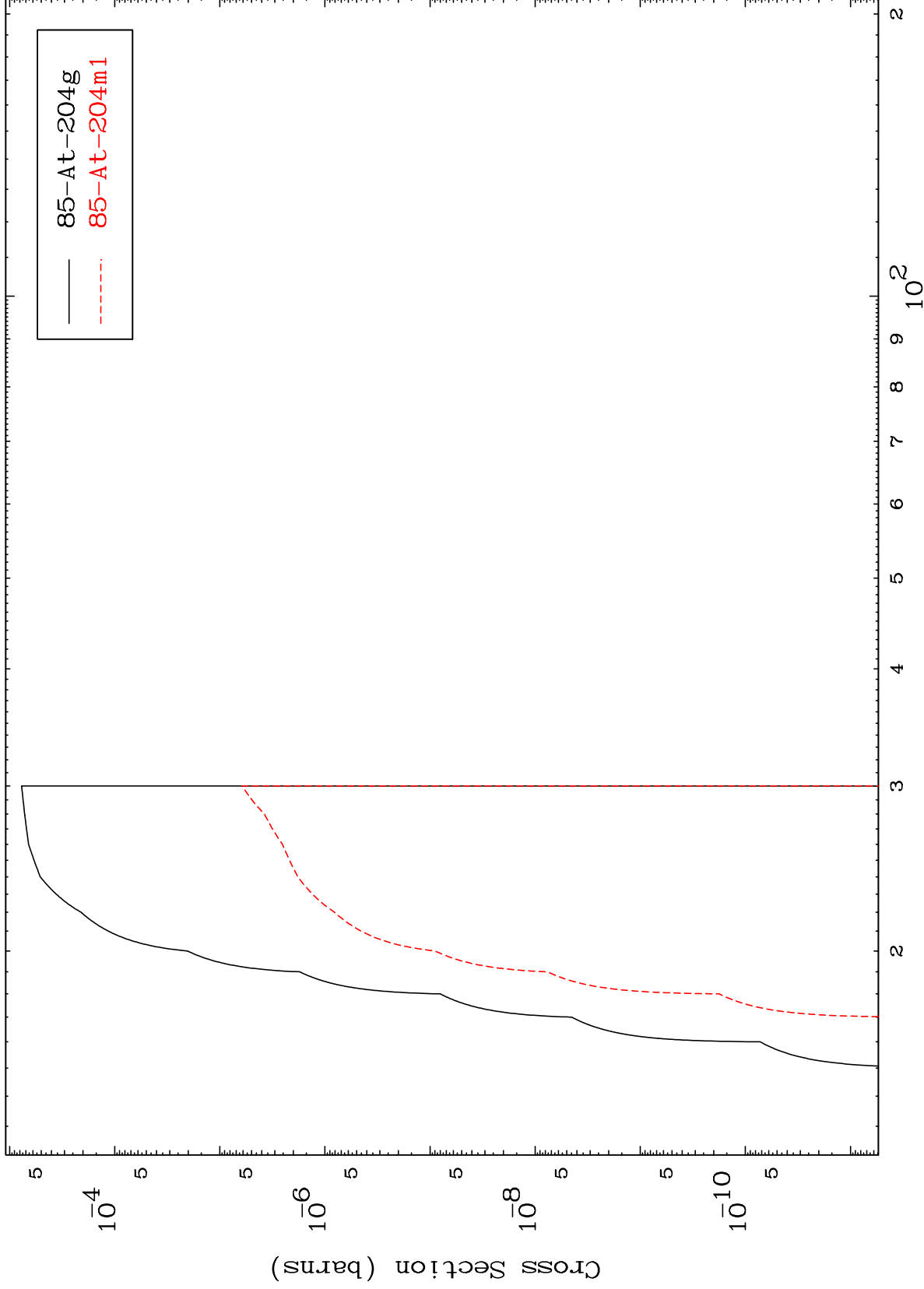
86-Rn-206

MAT 8610

$(\gamma, n')$  p

86-Rn-206

Radionuclide Production Cross Section



14

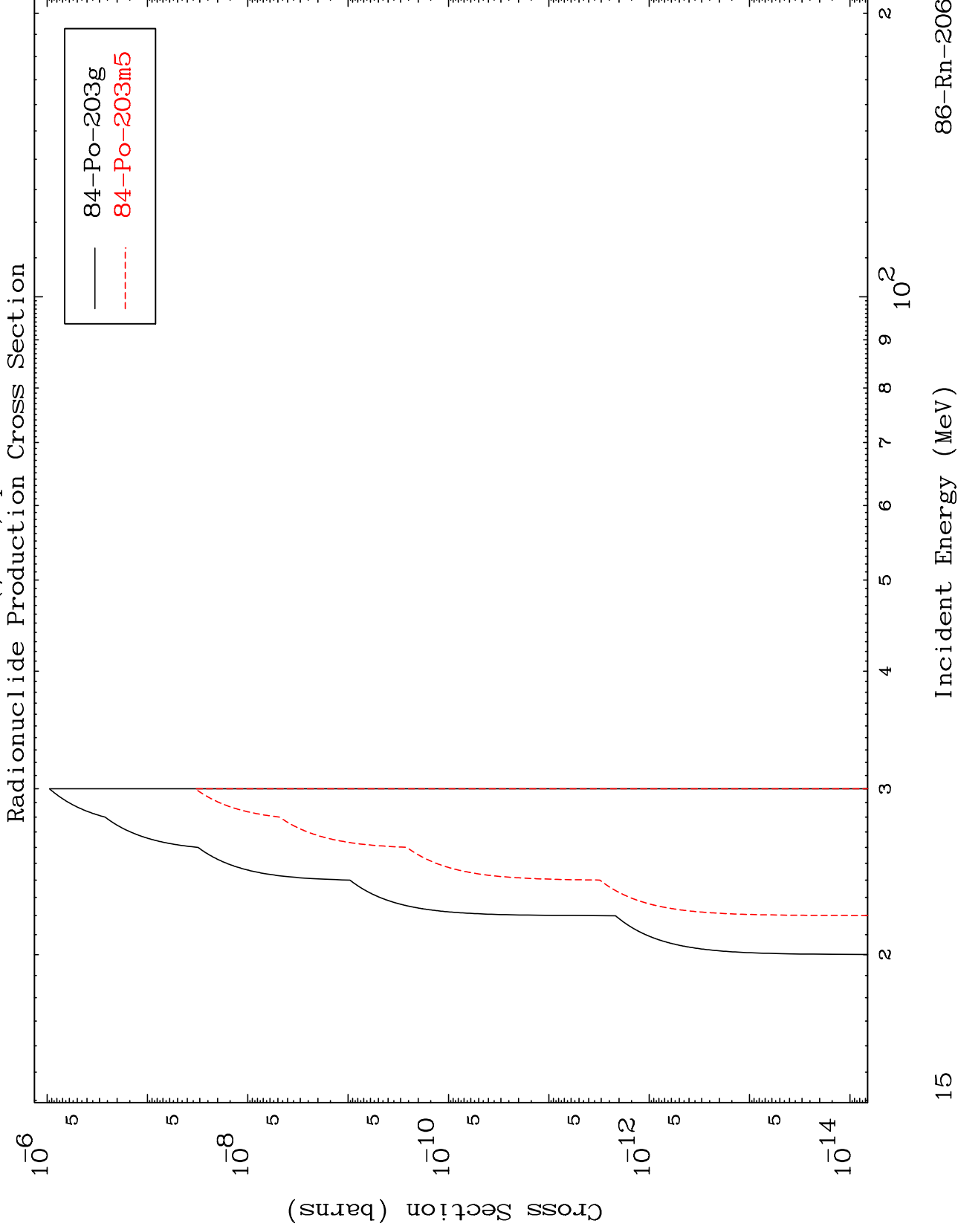
Incident Energy (MeV)

86-Rn-206

MAT 8610

$(\gamma, 2n)$  p

86-Rn-206



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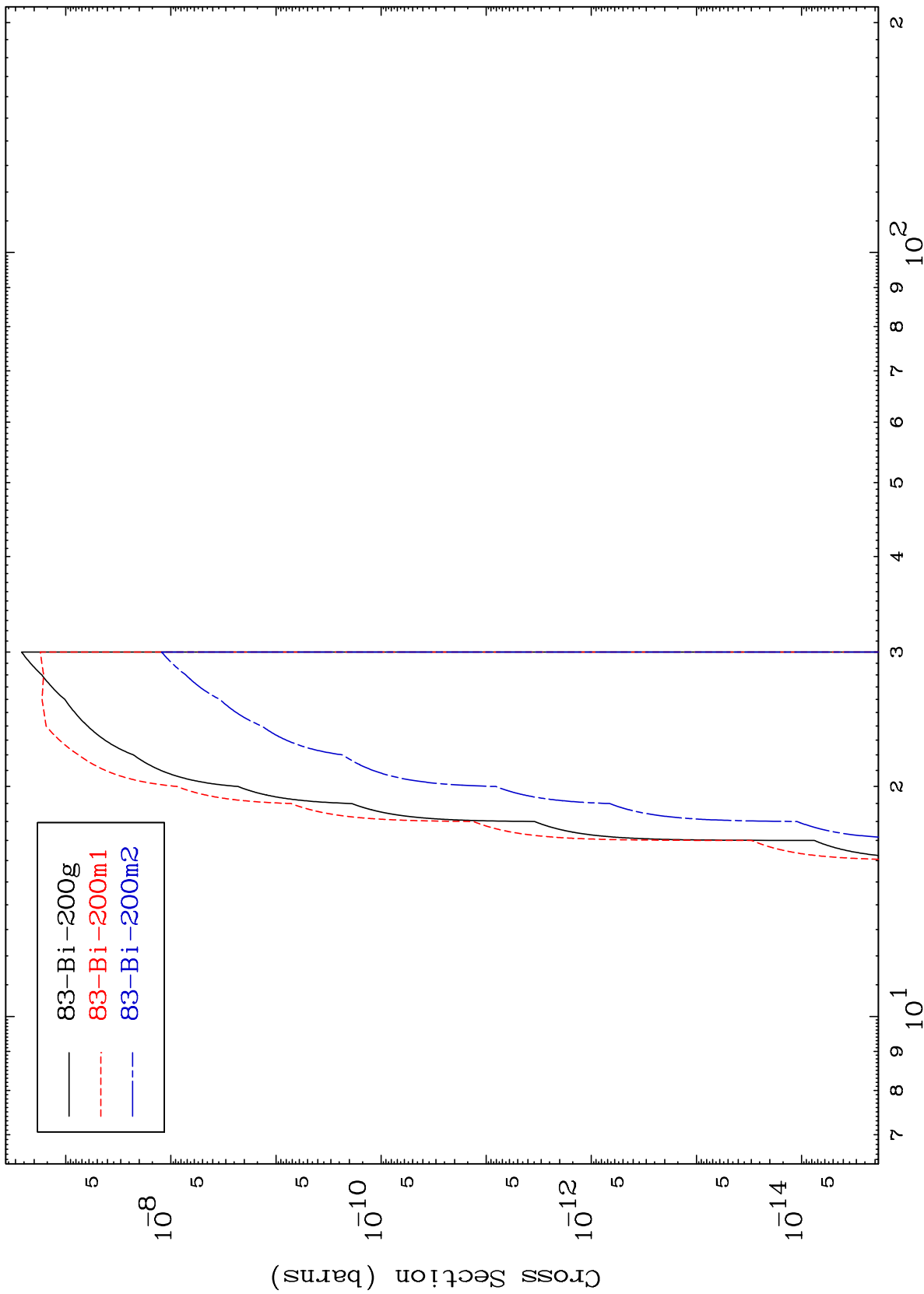
86-Rn-206



MAT 8610

86-Rn-206

( $\gamma, n'$ ) p  $\alpha$   
Radionuclide Production Cross Section



16

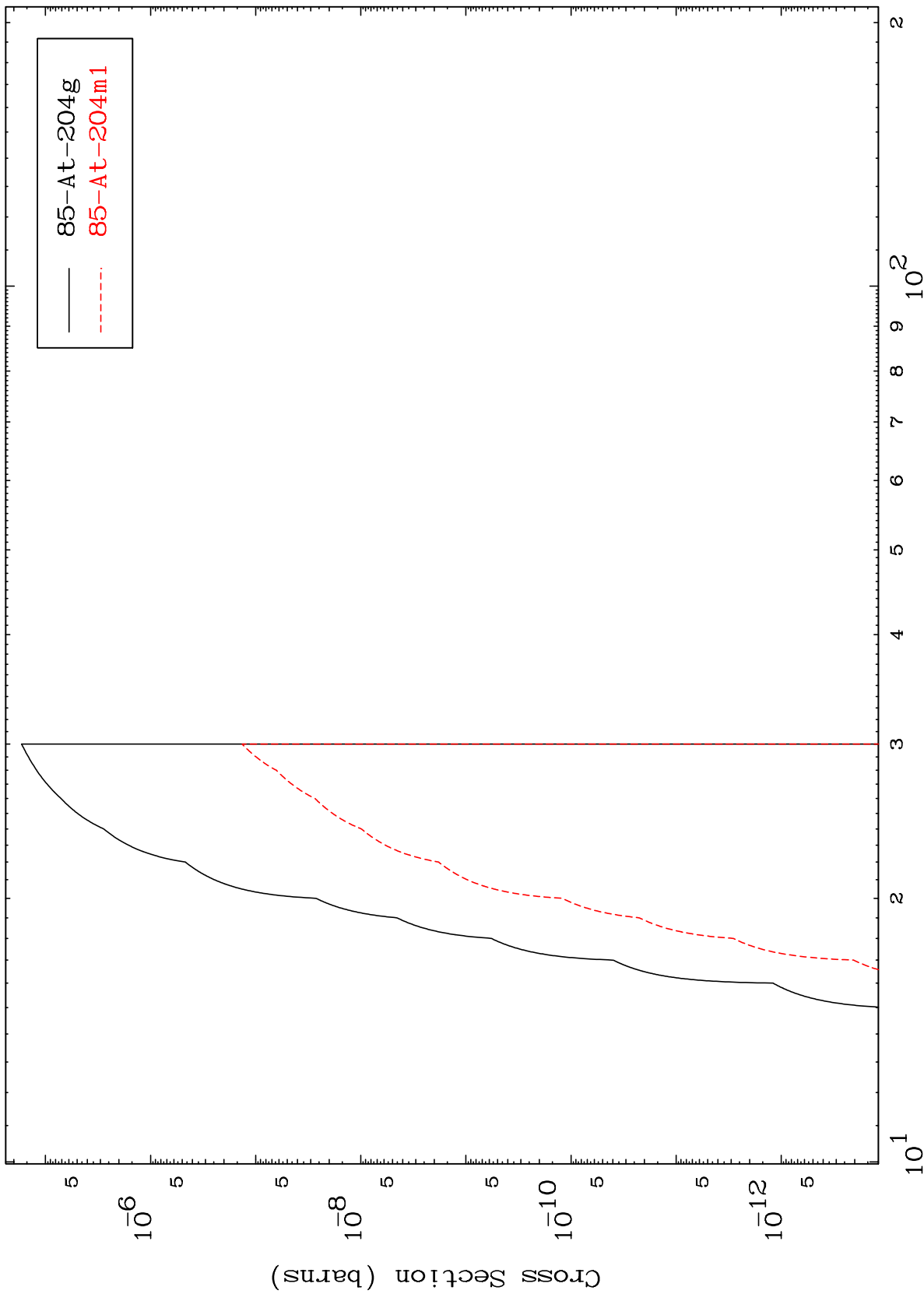
Incident Energy (MeV)

86-Rn-206

MAT 8610

86-Rn-206

( $\gamma, d$ )  
Radionuclide Production Cross Section



86-Rn-206

Incident Energy (MeV)

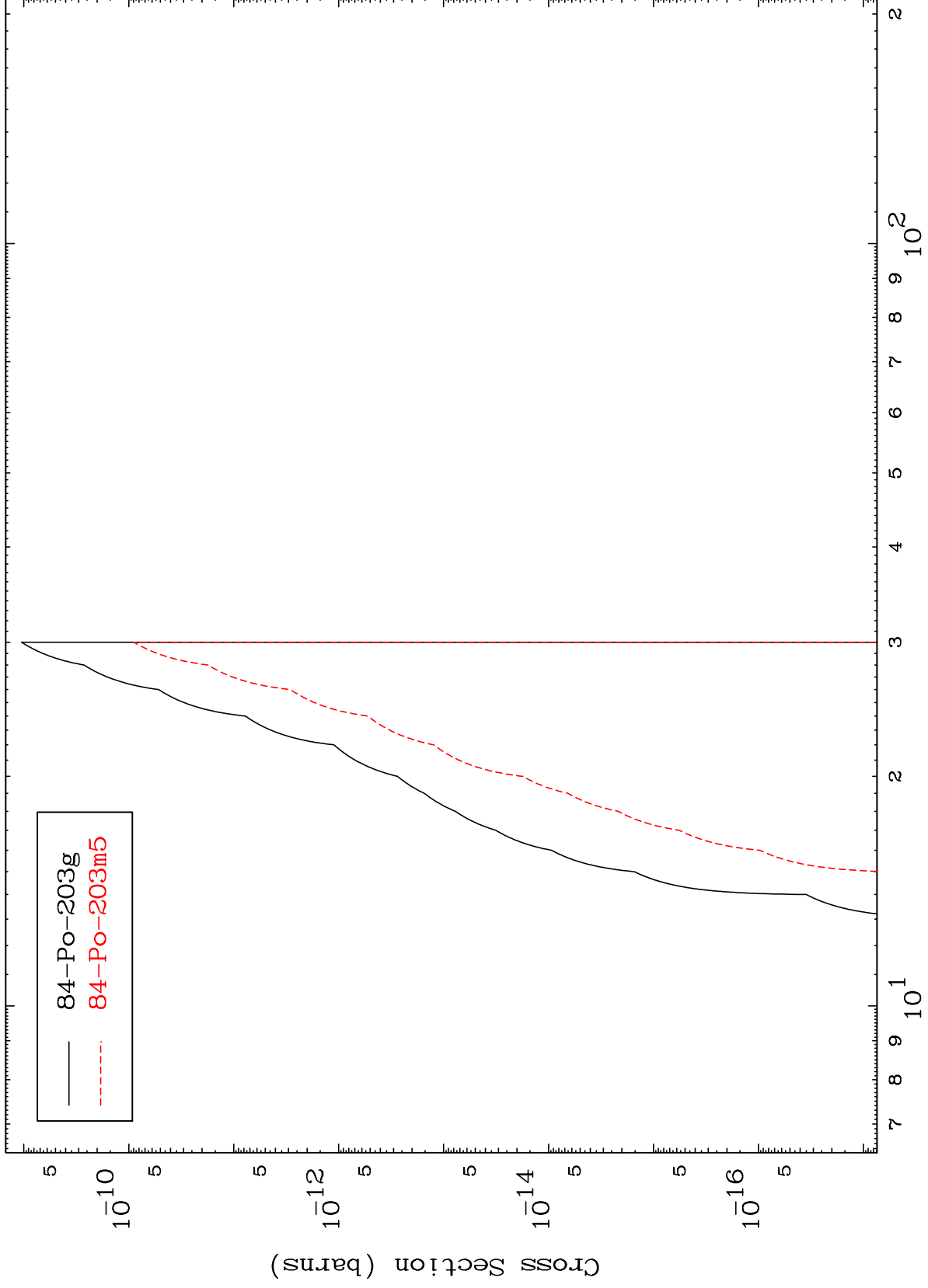
17

MAT 8610

( $\gamma, \text{He-3}$ )

86-Rn-206

Radionuclide Production Cross Section



18

Incident Energy (MeV)

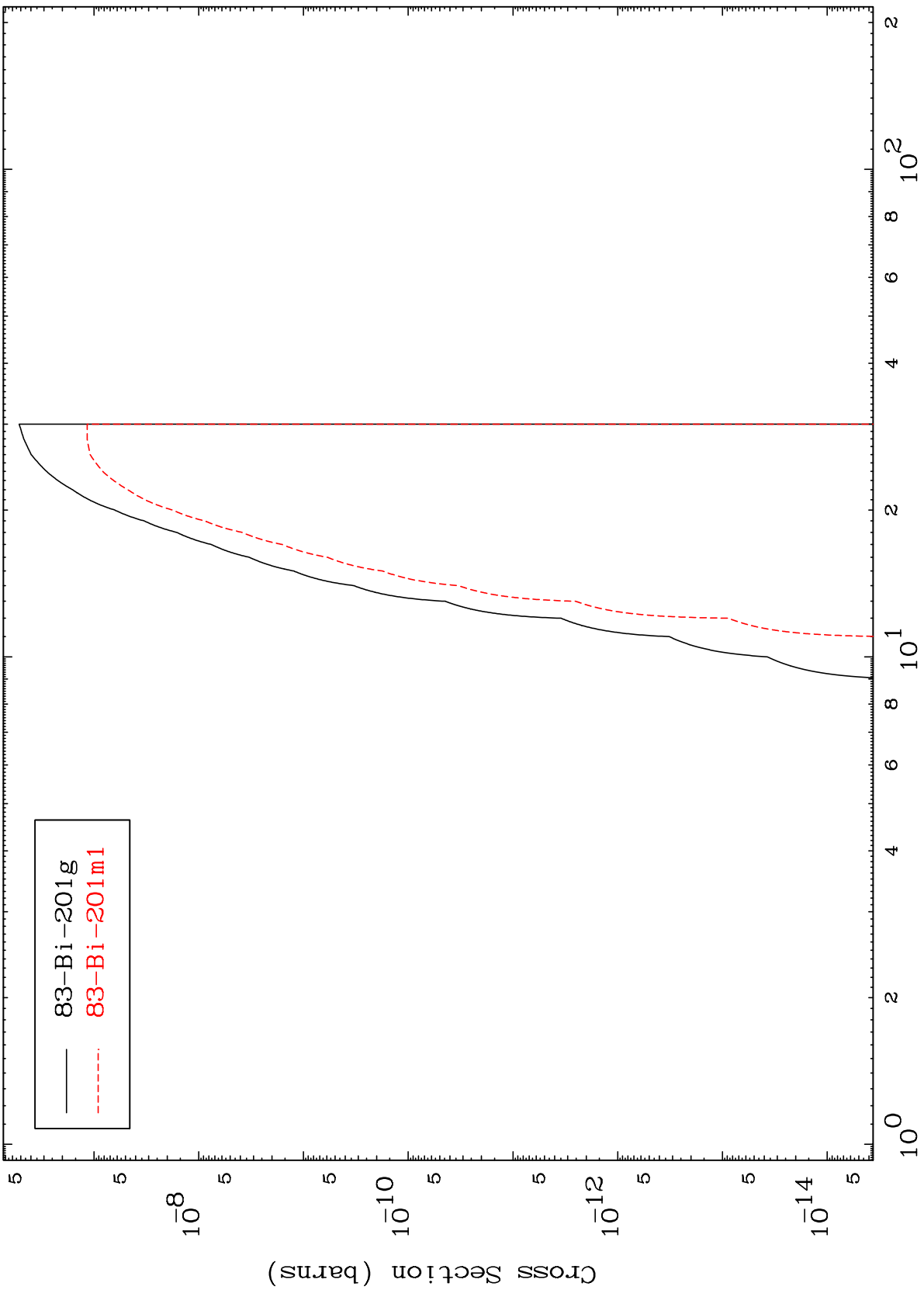
86-Rn-206

MAT 8610

( $\gamma, p$ )  $\alpha$

86-Rn-206

Radionuclide Production Cross Section



Incident Energy (MeV)

86-Rn-206

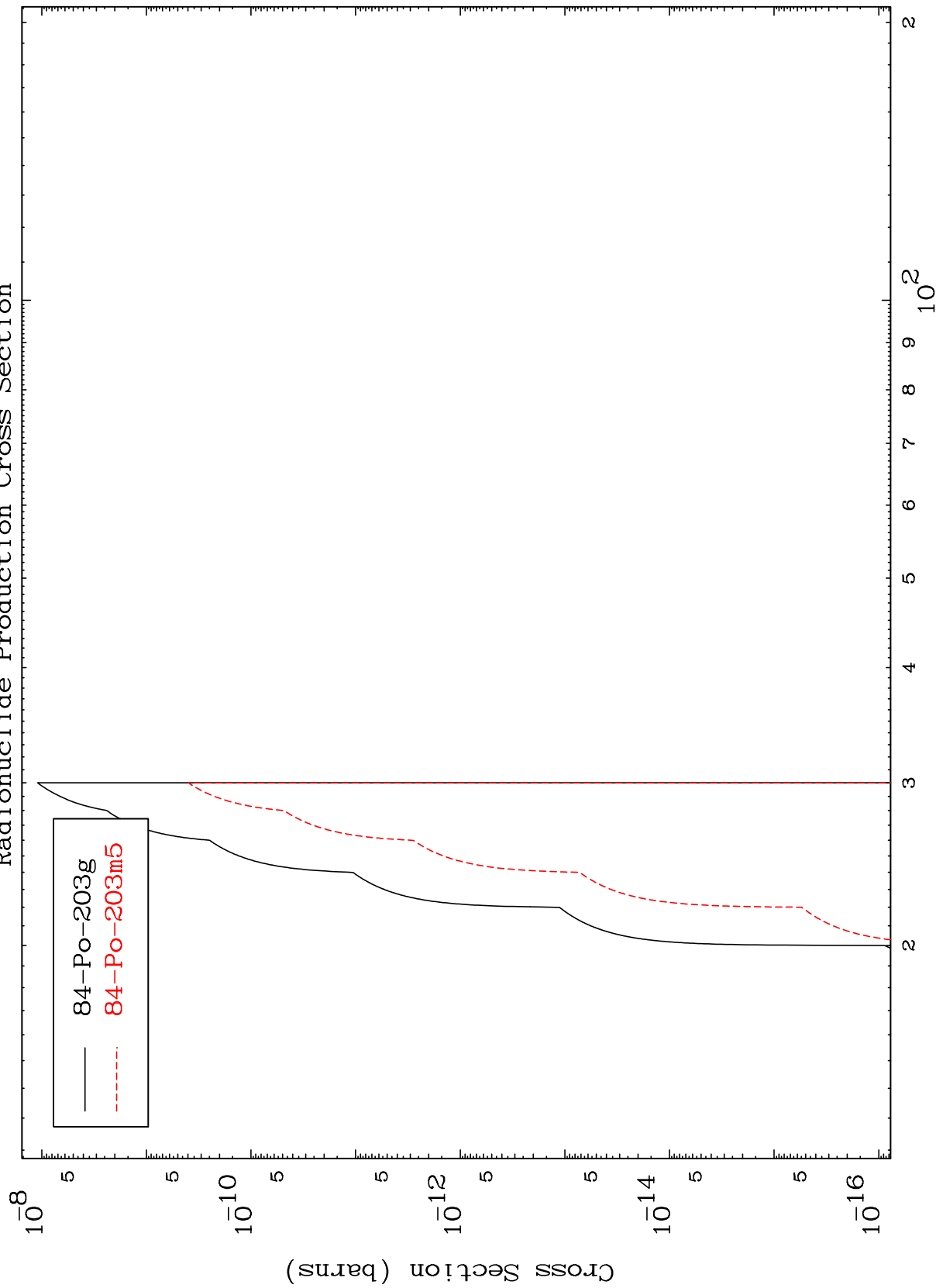
19

MAT 8610

( $\gamma, p$ ) d

86-Rn-206

Radionuclide Production Cross Section



20

Incident Energy (MeV)

86-Rn-206

MAT 8610

( $\gamma, d$ )  $\alpha$

86-Rn-206

Radionuclide Production Cross Section

