

Program EVALPLOT
(Version 2017-1)

by

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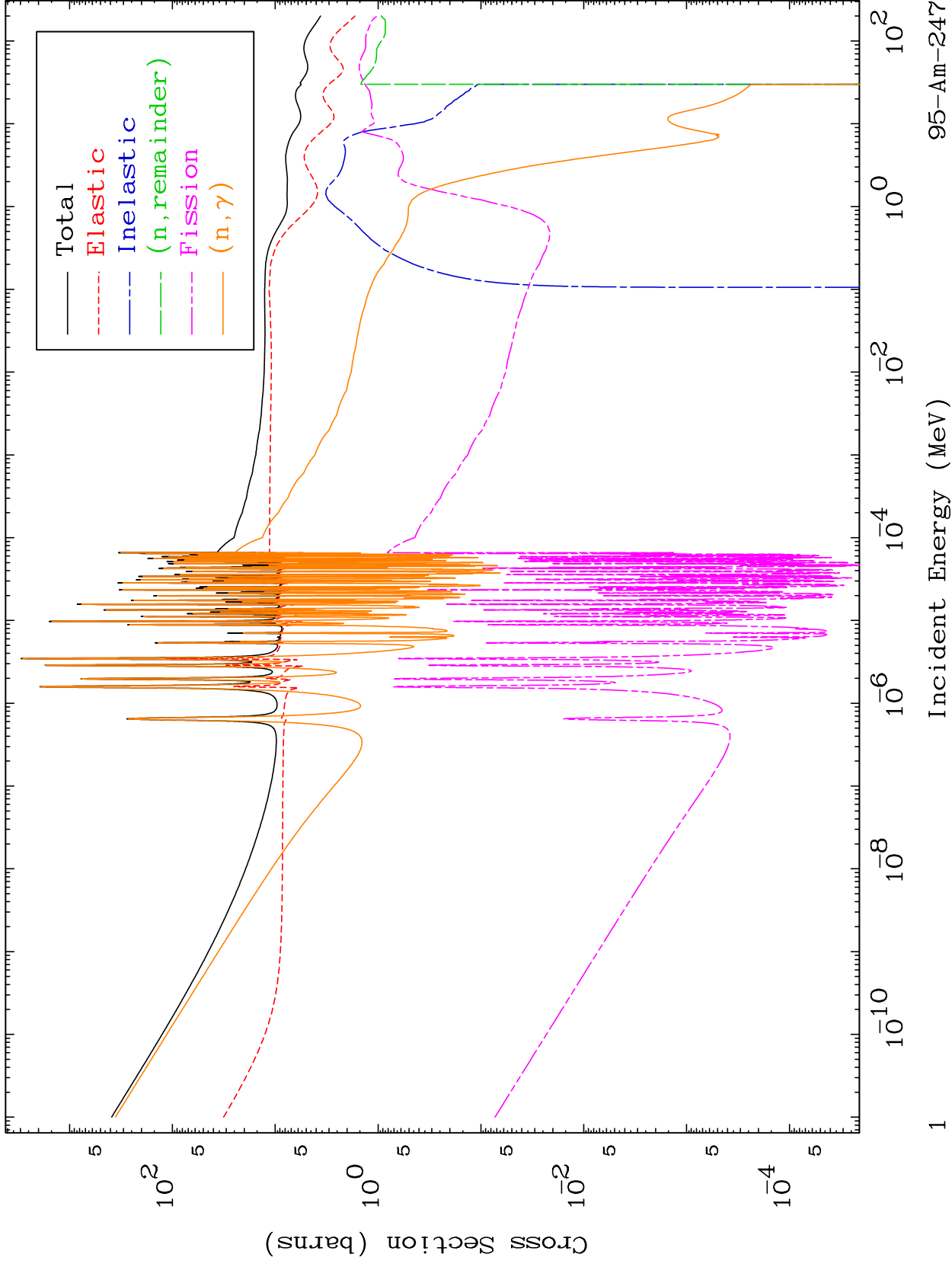
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Press Mouse Button to Start

MAT 9561

Major
293 Kelvin Cross Sections

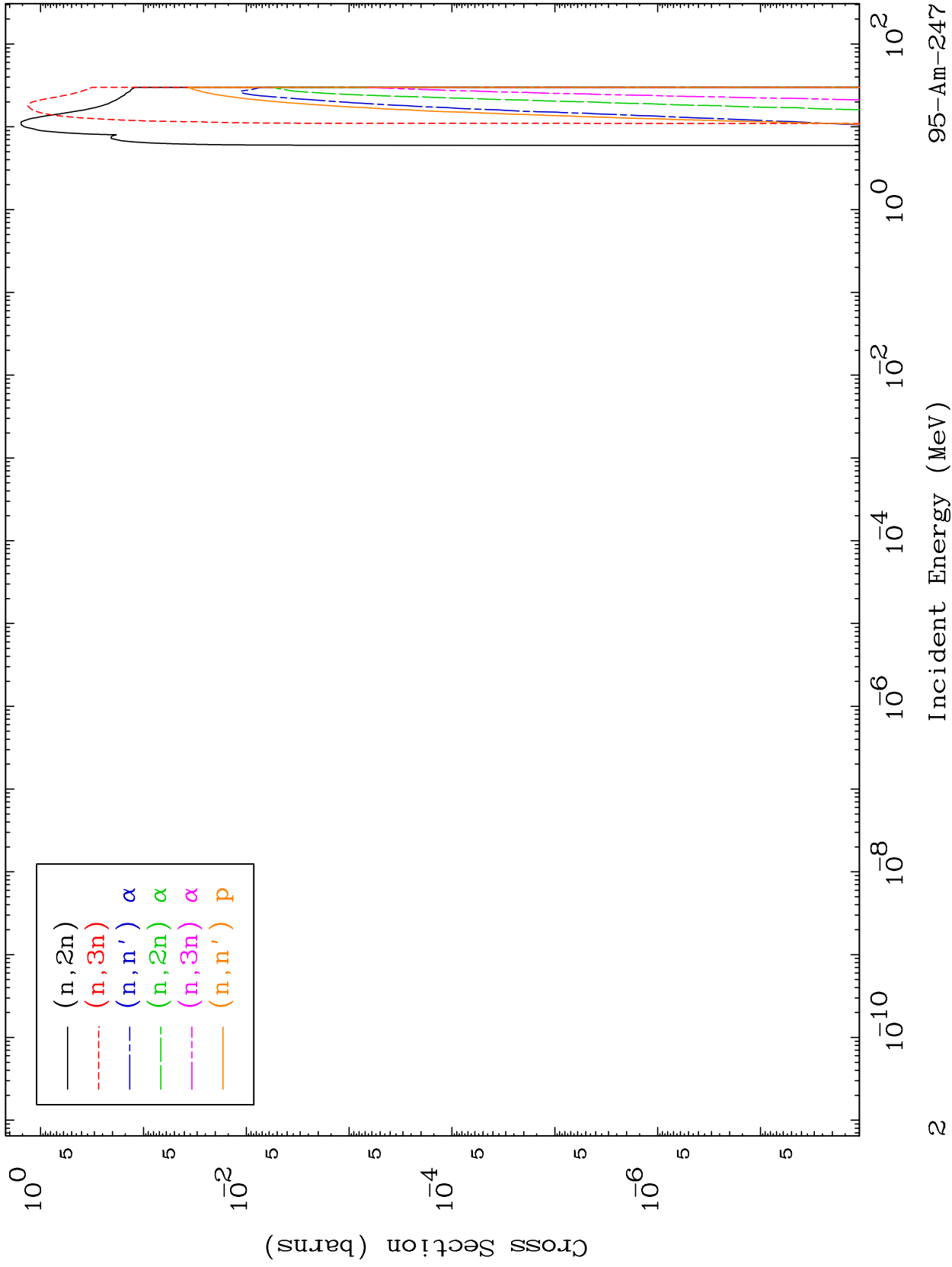
95-Am-247



MAT 9561

Neutron Production
293 Kelvin Cross Sections

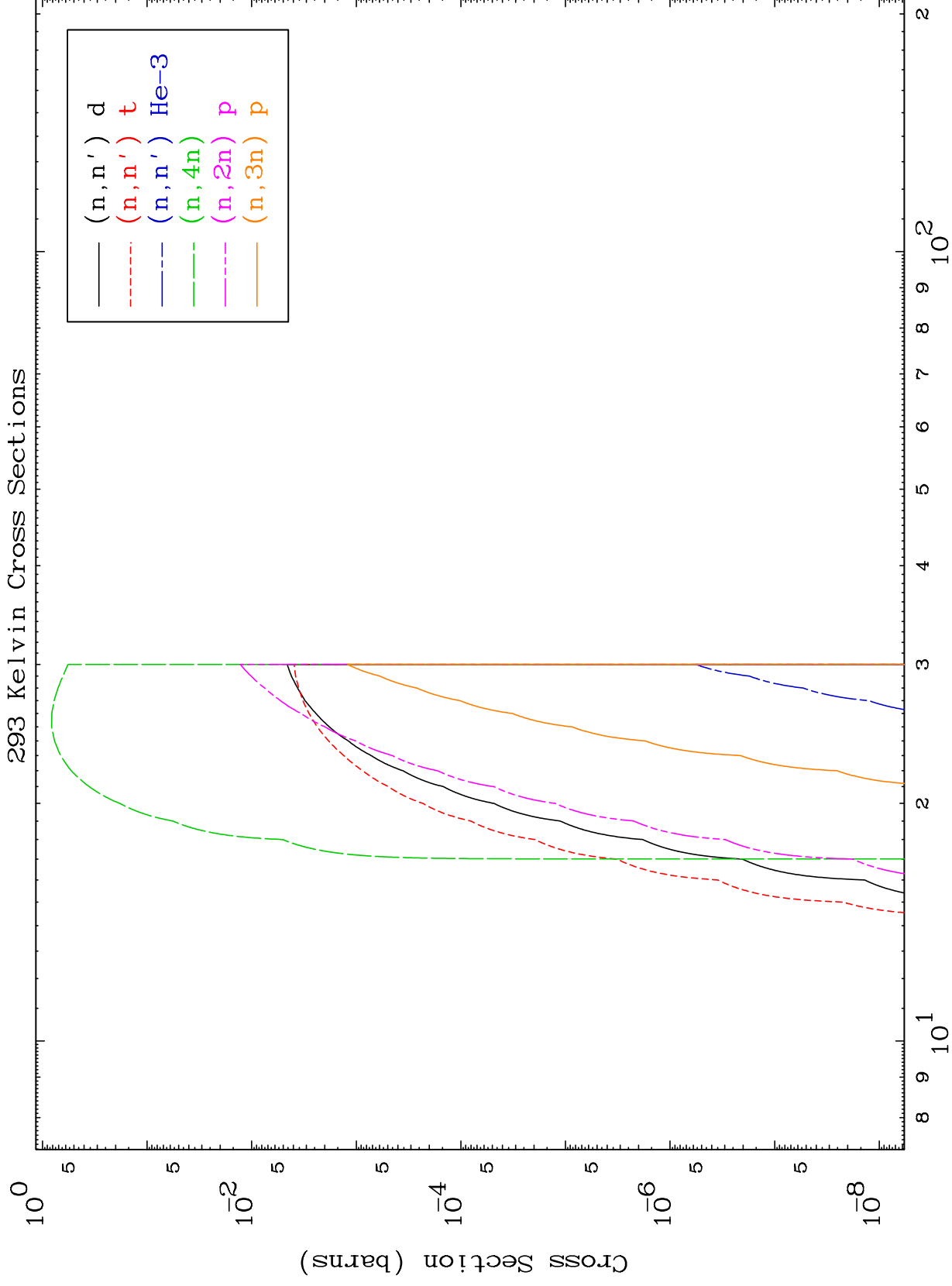
95-Am-247



MAT 9561

Neutron Production
293 Kelvin Cross Sections

95-Am-247



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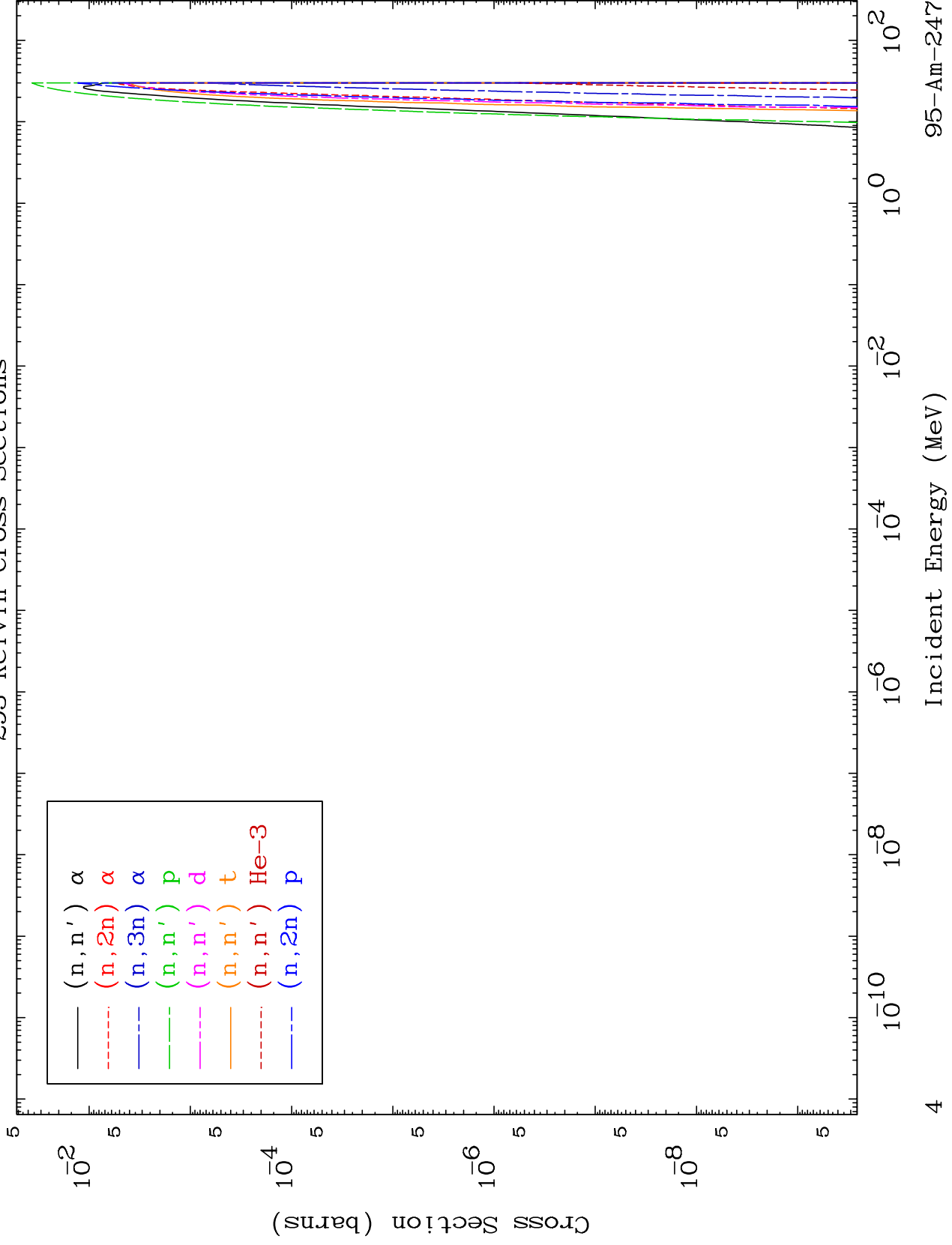
Incident Energy (MeV)

95-Am-247

MAT 9561

Charged Particle
293 Kelvin Cross Sections

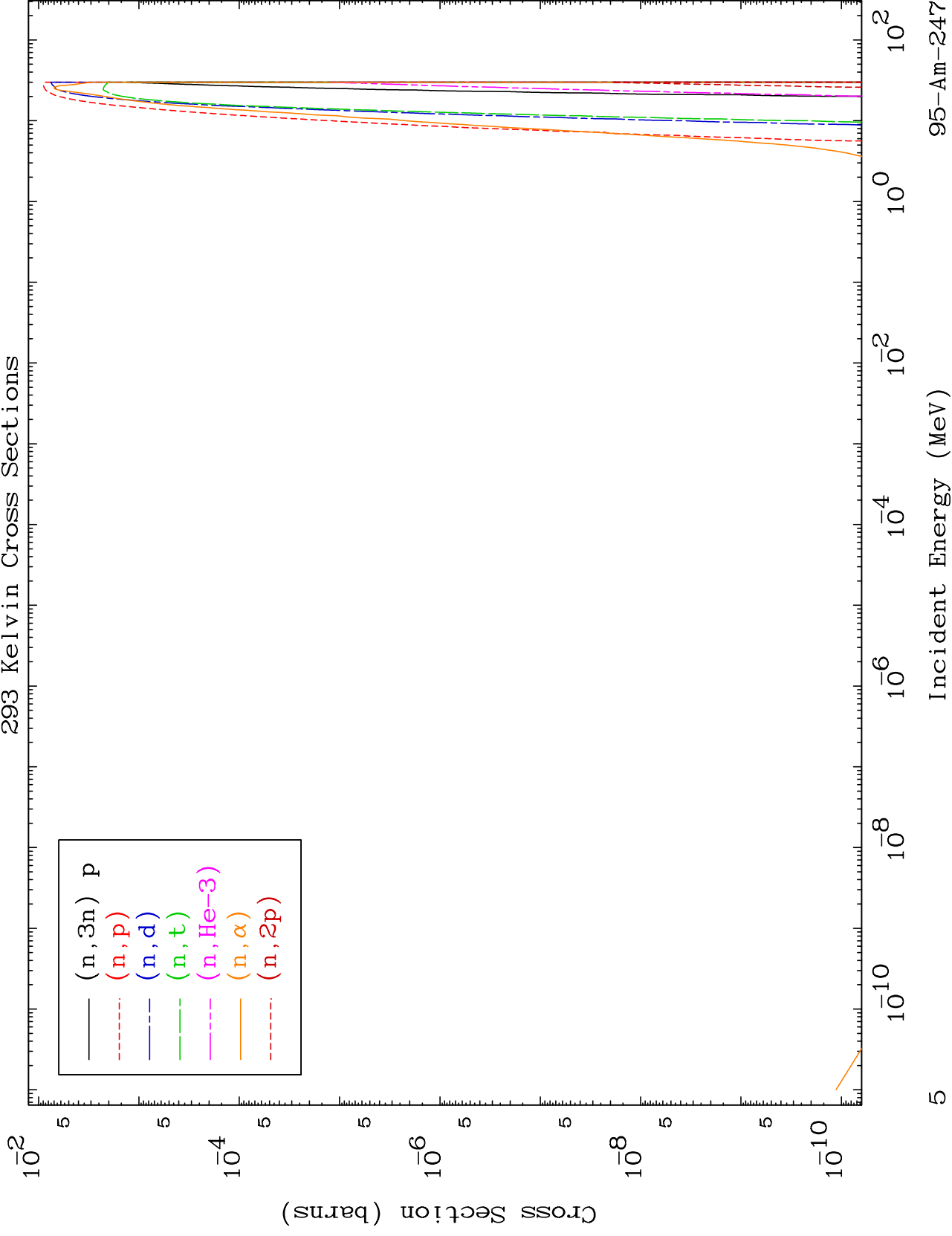
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Charged Particle
293 Kelvin Cross Sections

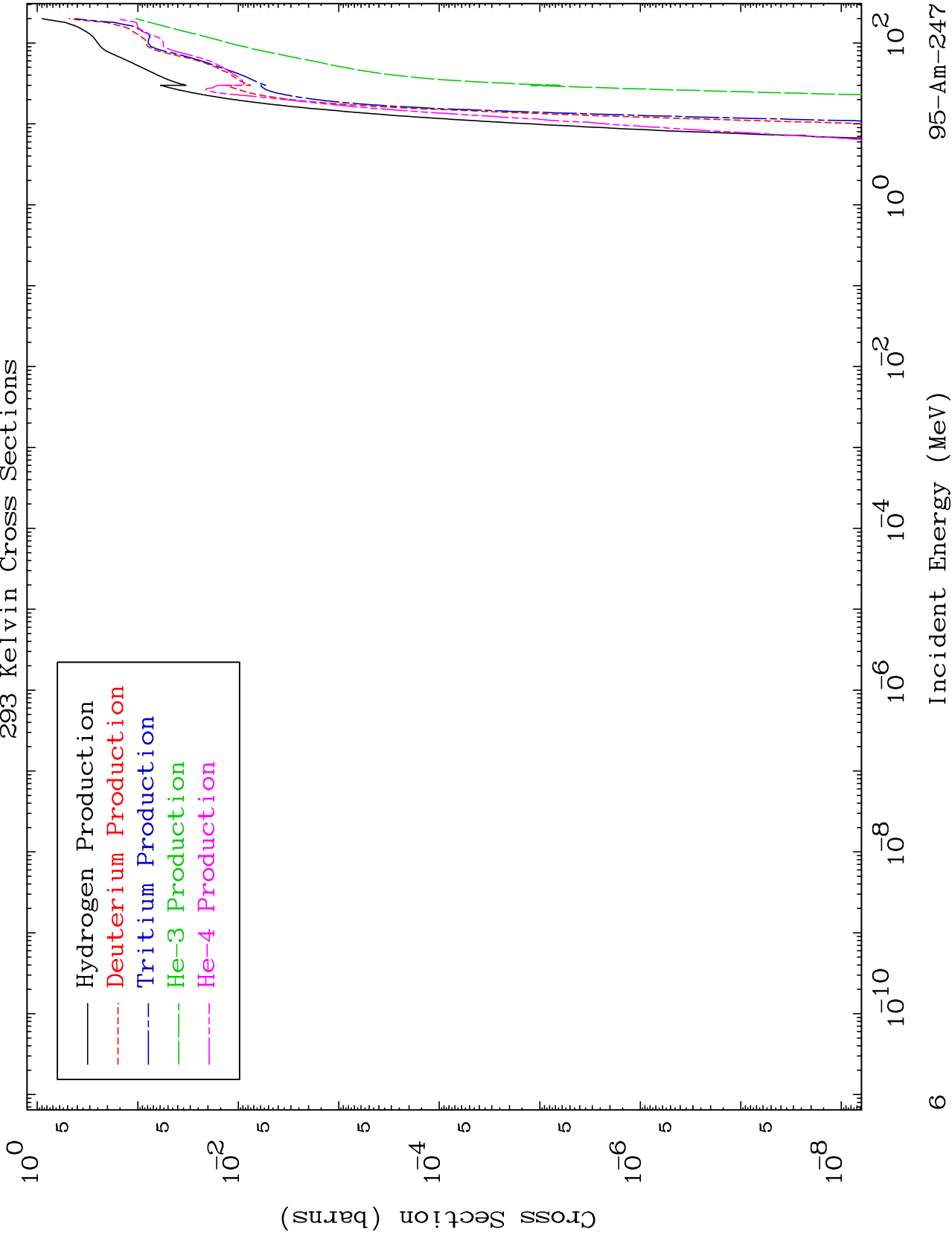
95-Am-247



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Particle Production
293 Kelvin Cross Sections

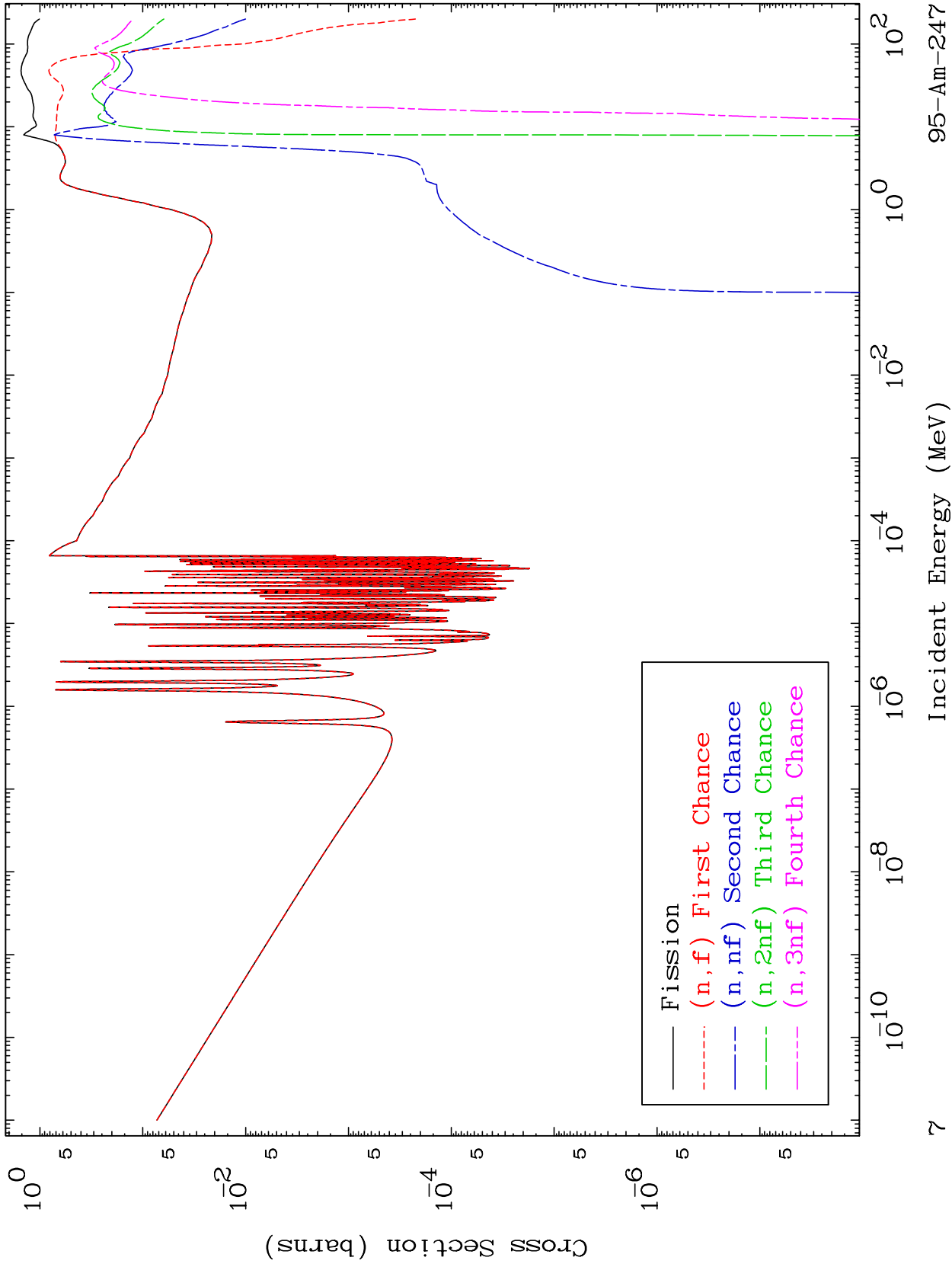
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Fission
293 Kelvin Cross Sections

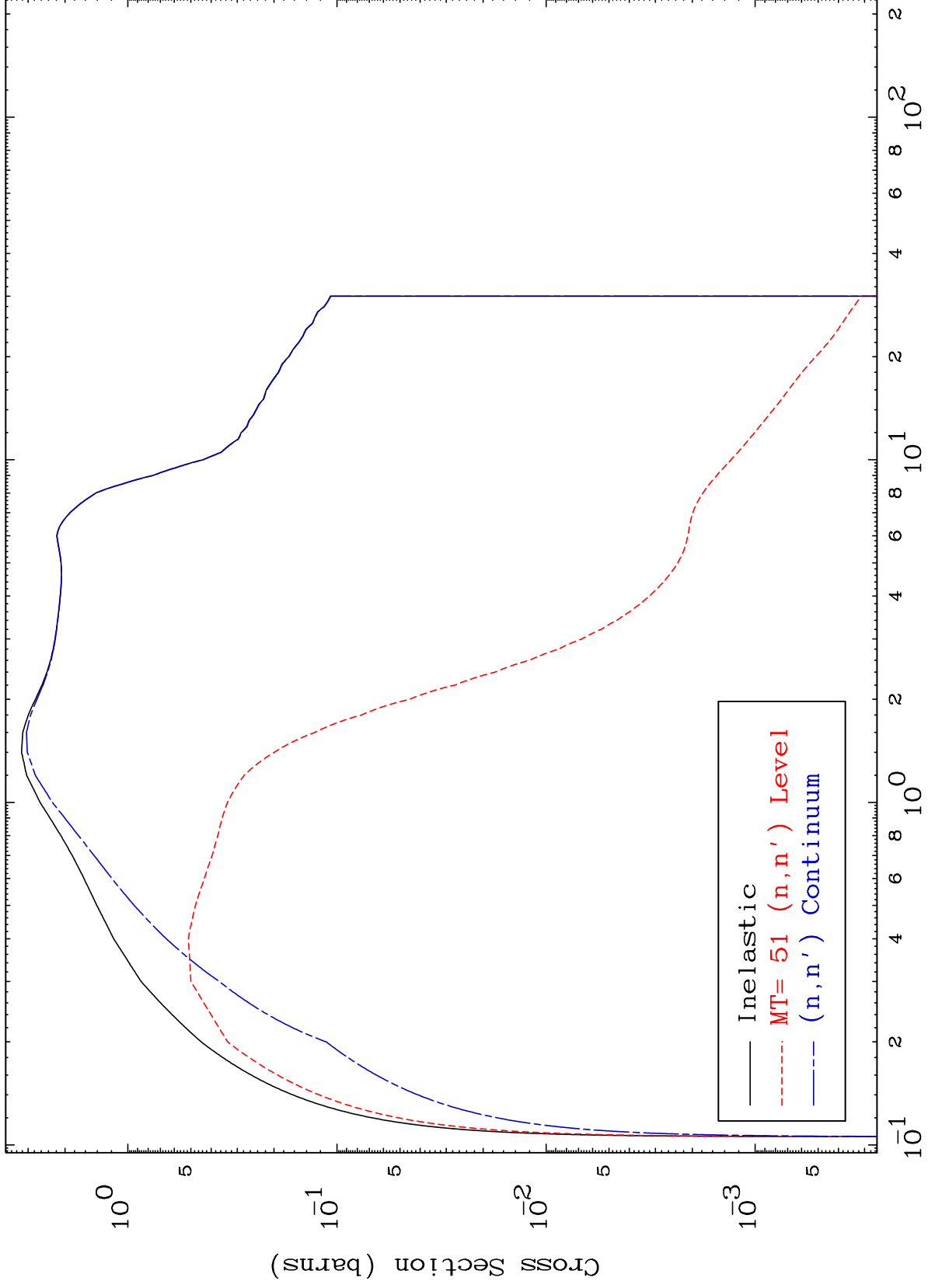
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(n,n') Level
293 Kelvin Cross Sections

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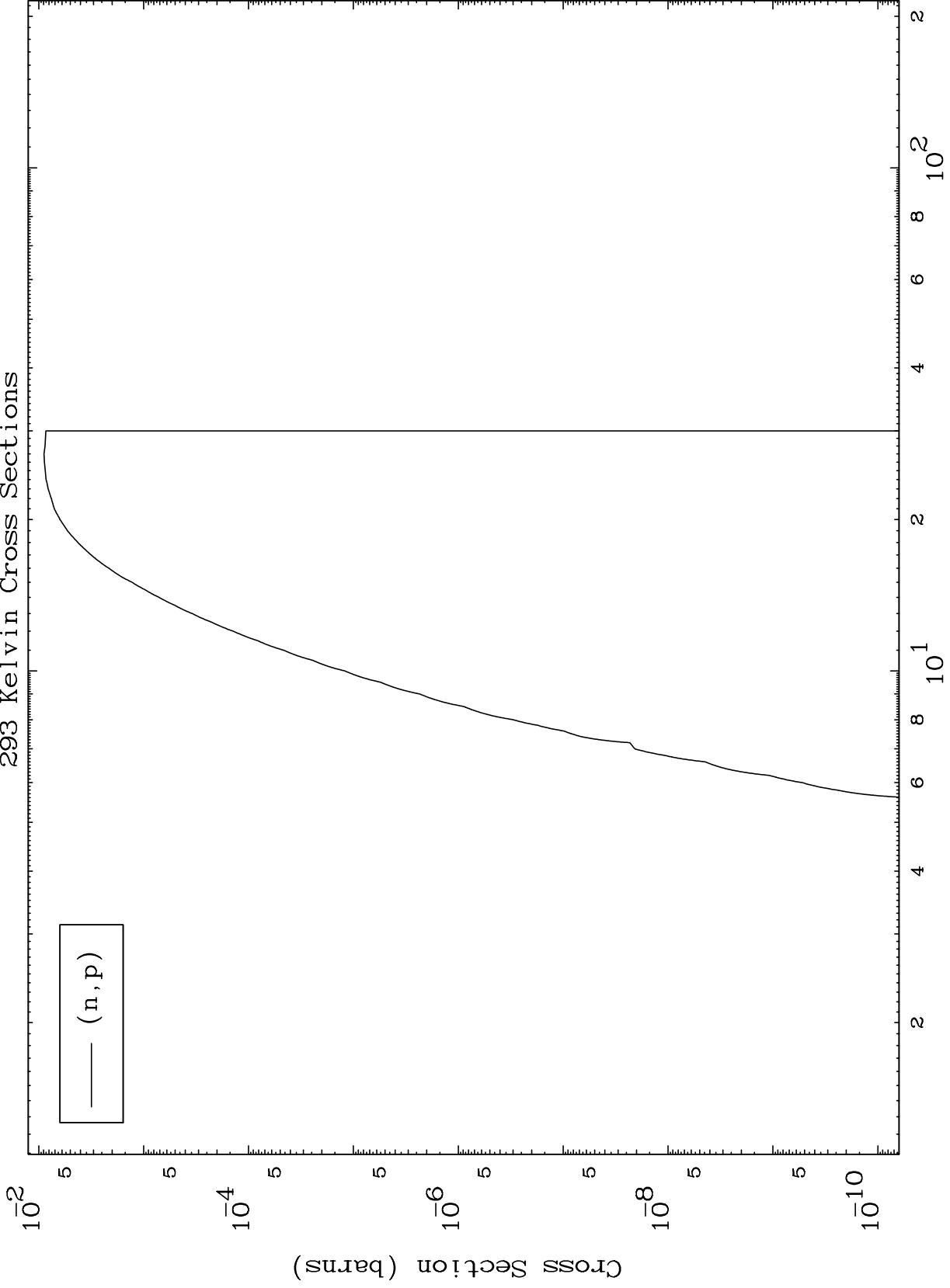
Incident Energy (MeV)

8

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(n,p) Levels
293 Kelvin Cross Sections

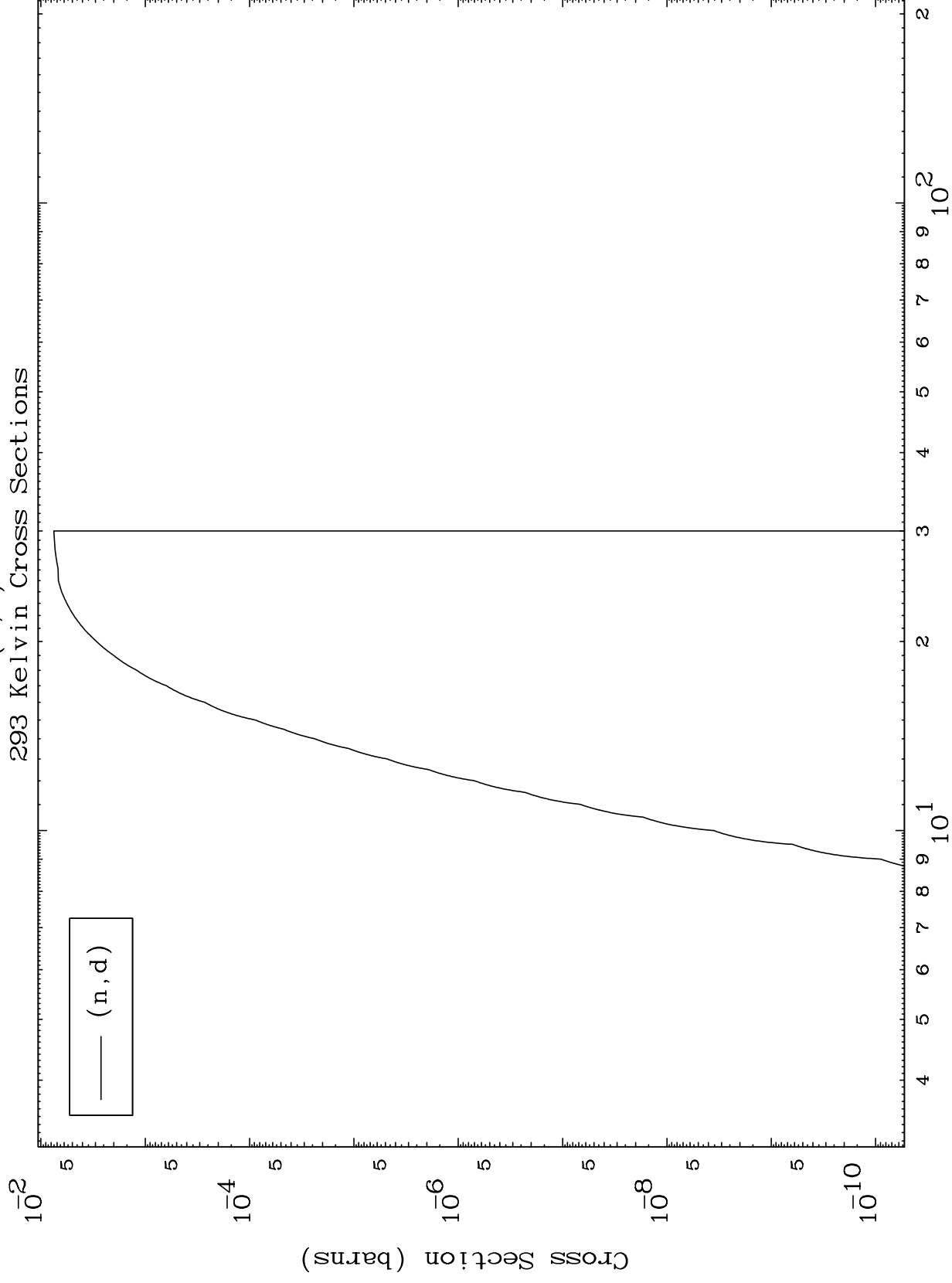
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(n,d) Levels
293 Kelvin Cross Sections

95-Am-247



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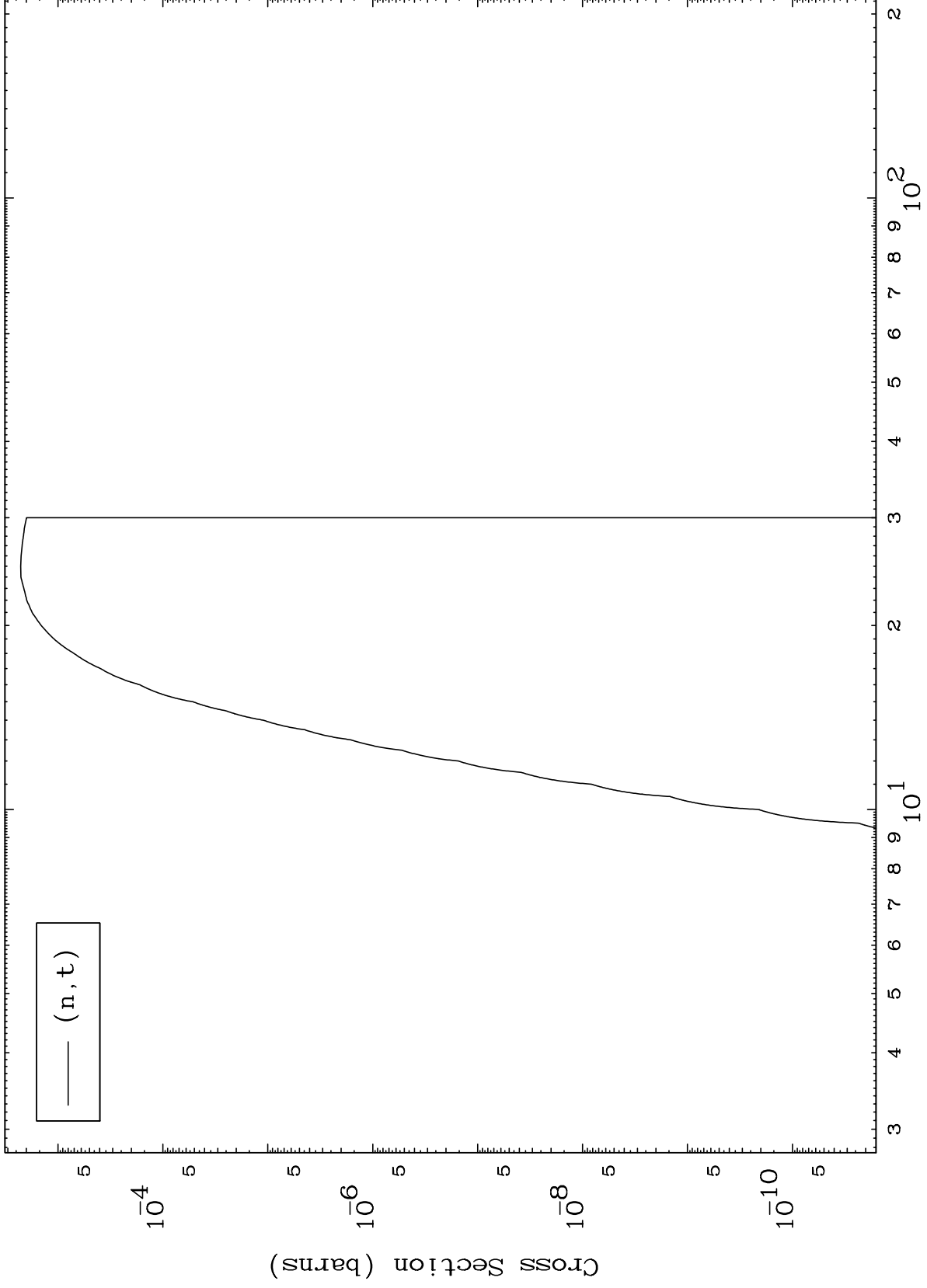
Incident Energy (MeV)

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MAT 9561

(n,t) Levels
293 Kelvin Cross Sections

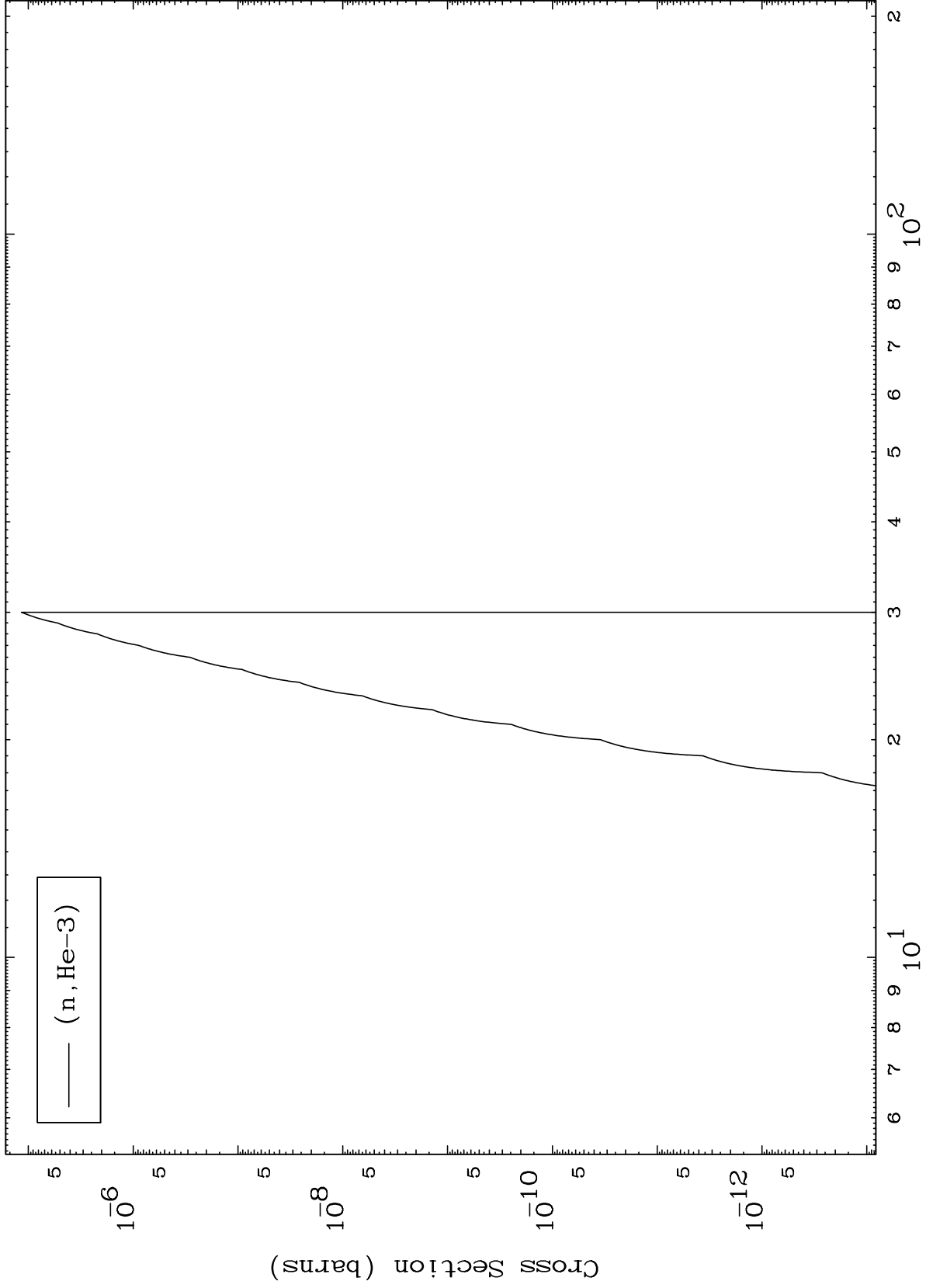
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(n,He3) Levels
293 Kelvin Cross Sections

95-Am-247



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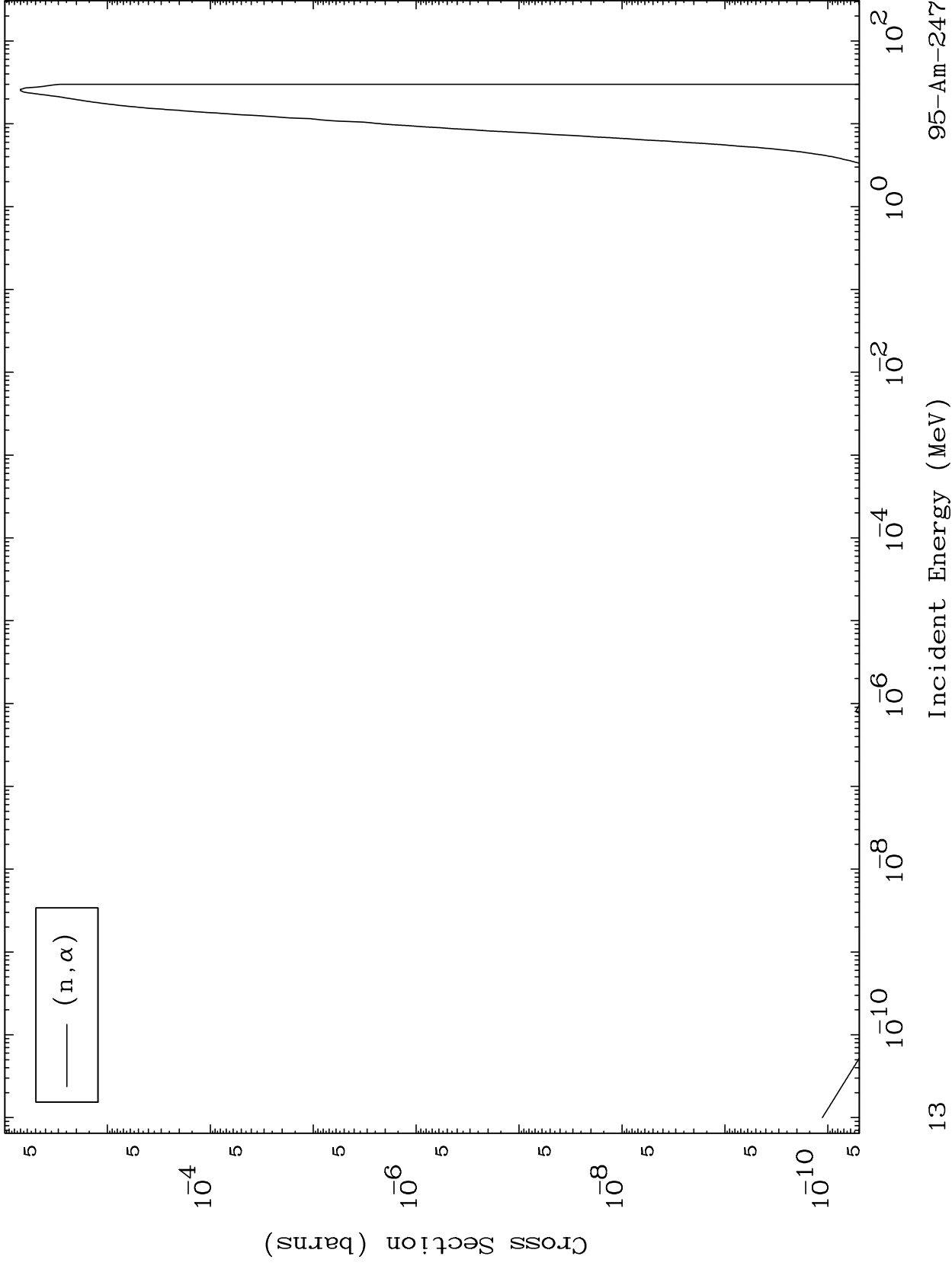
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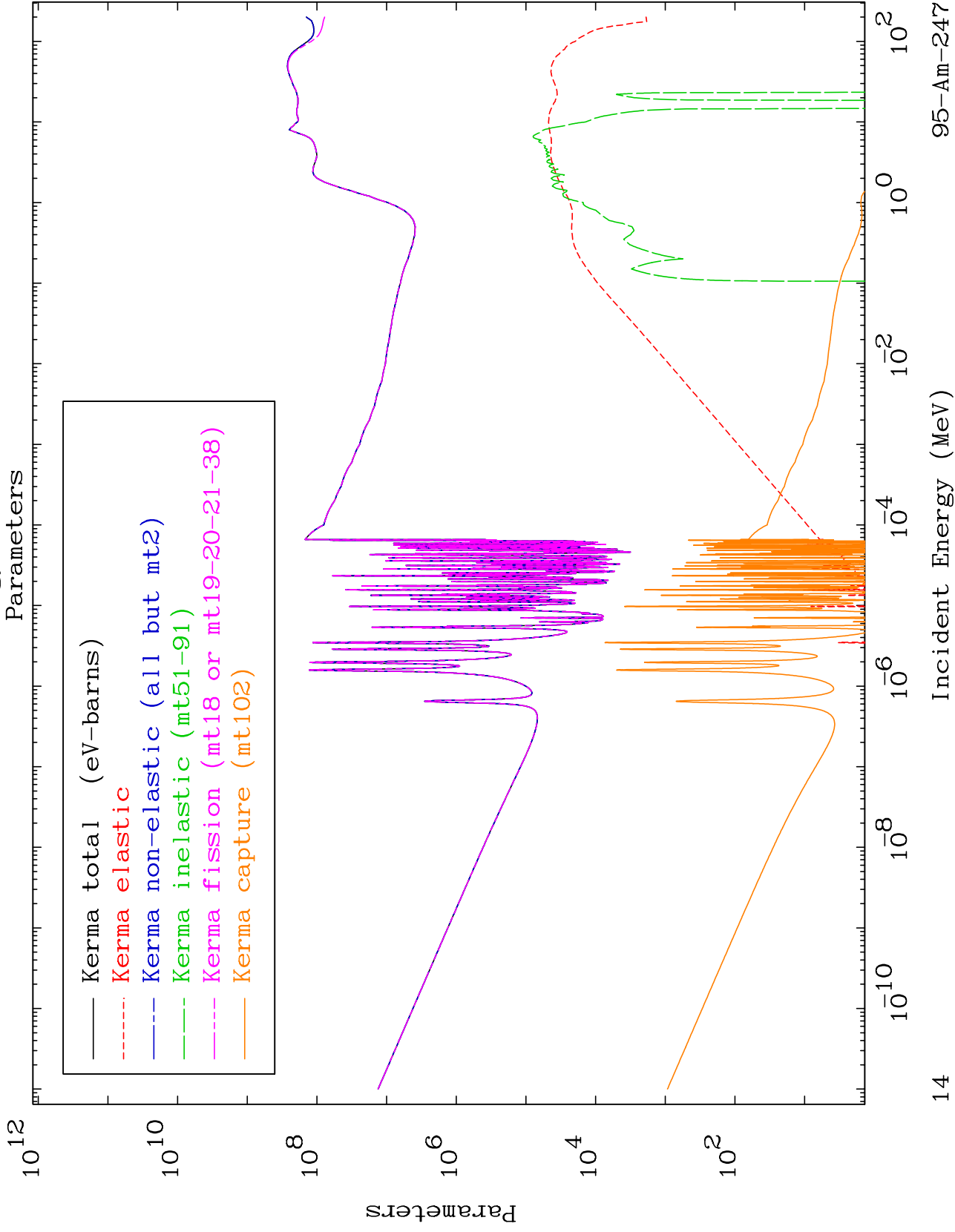
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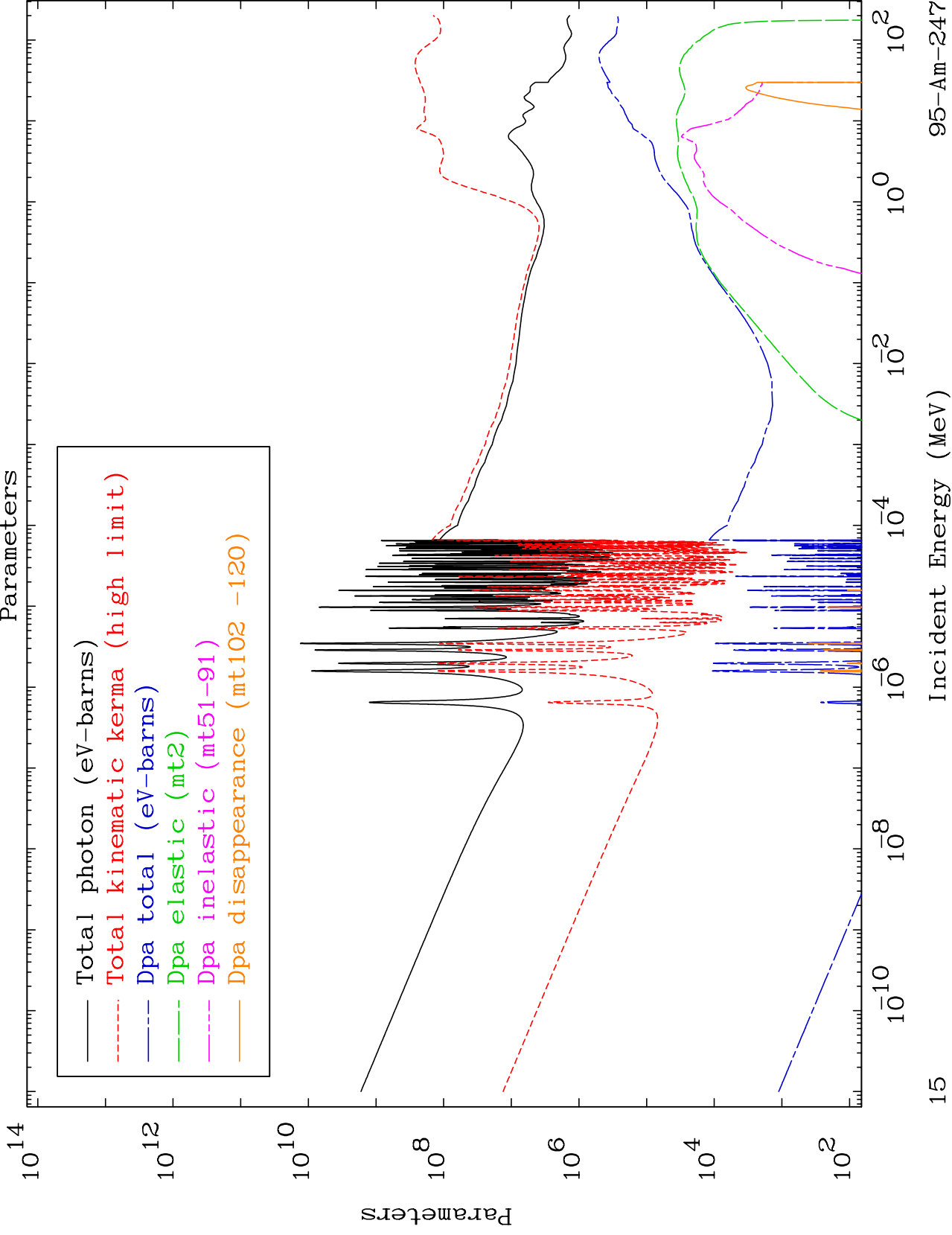
MAT 9561

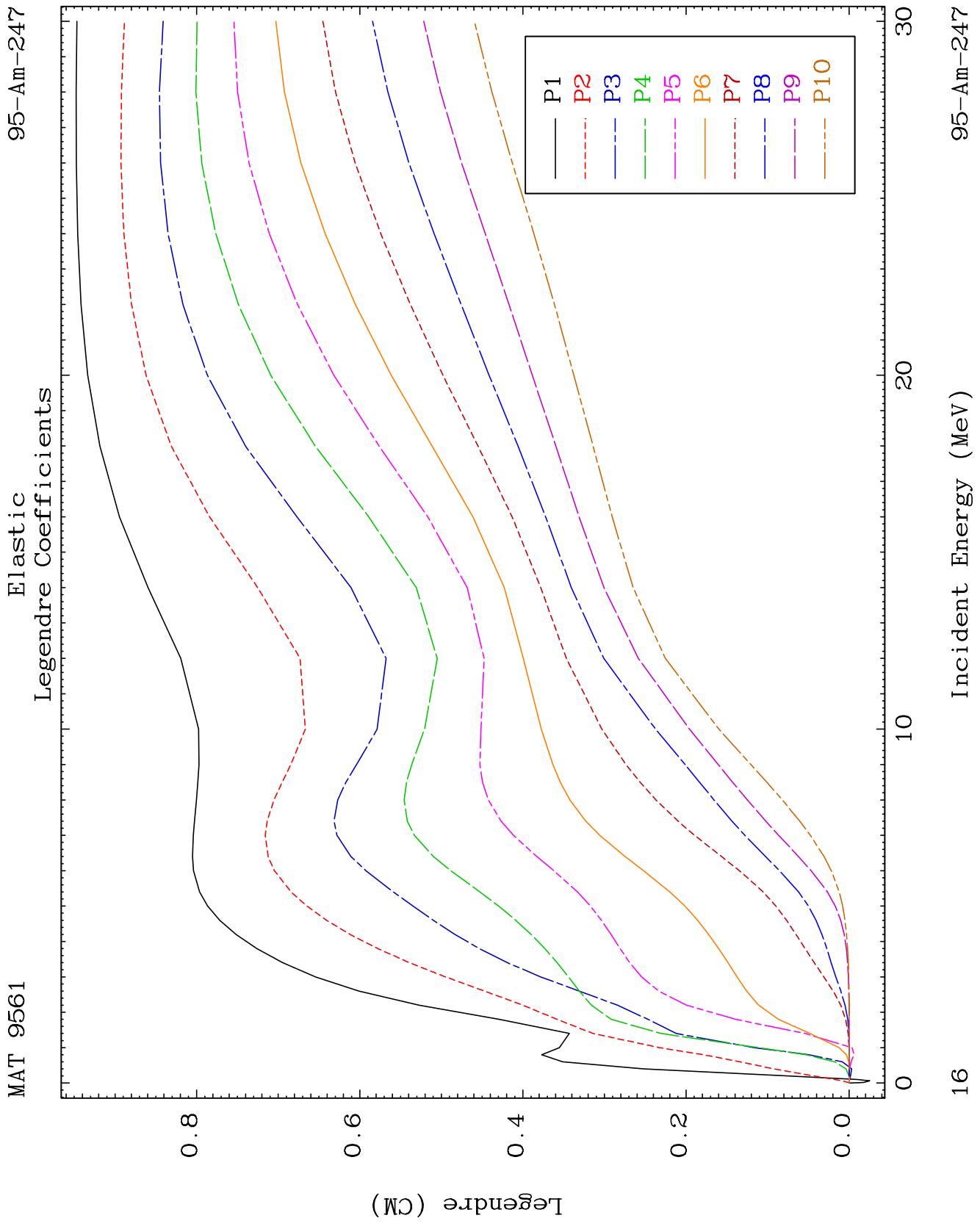
(n, α) Levels
293 Kelvin Cross Sections

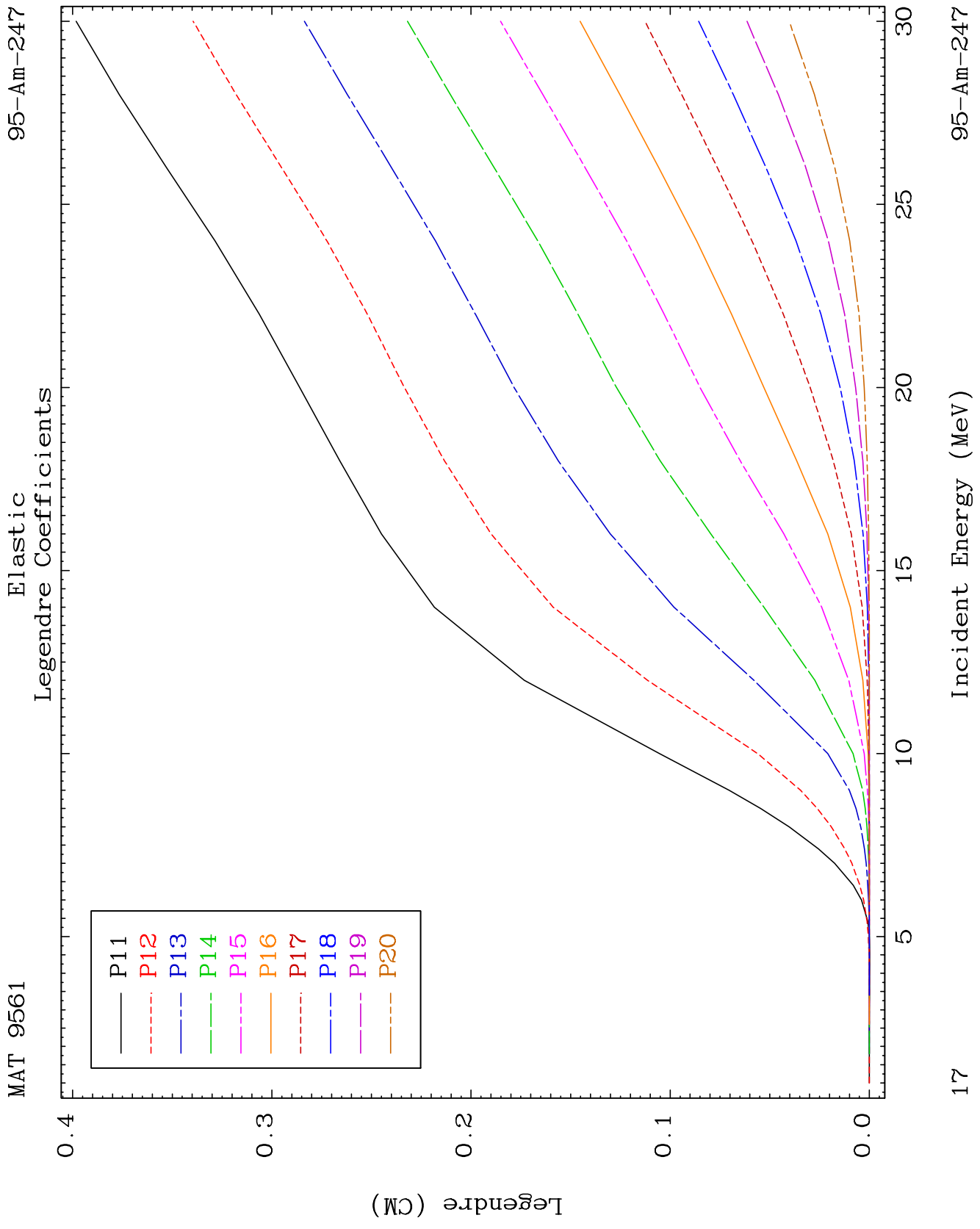
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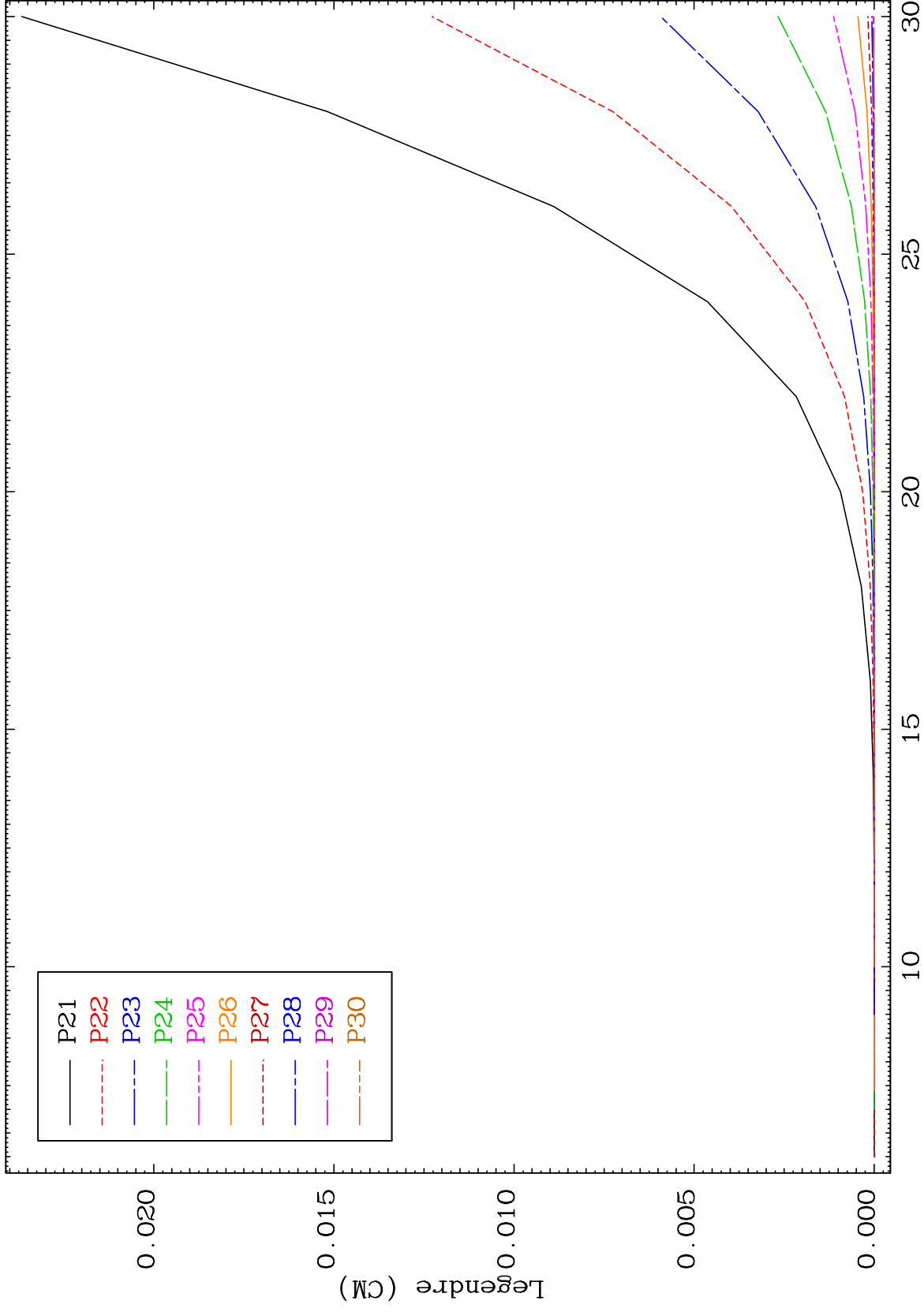




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Elastic Legendre Coefficients

95-Am-247



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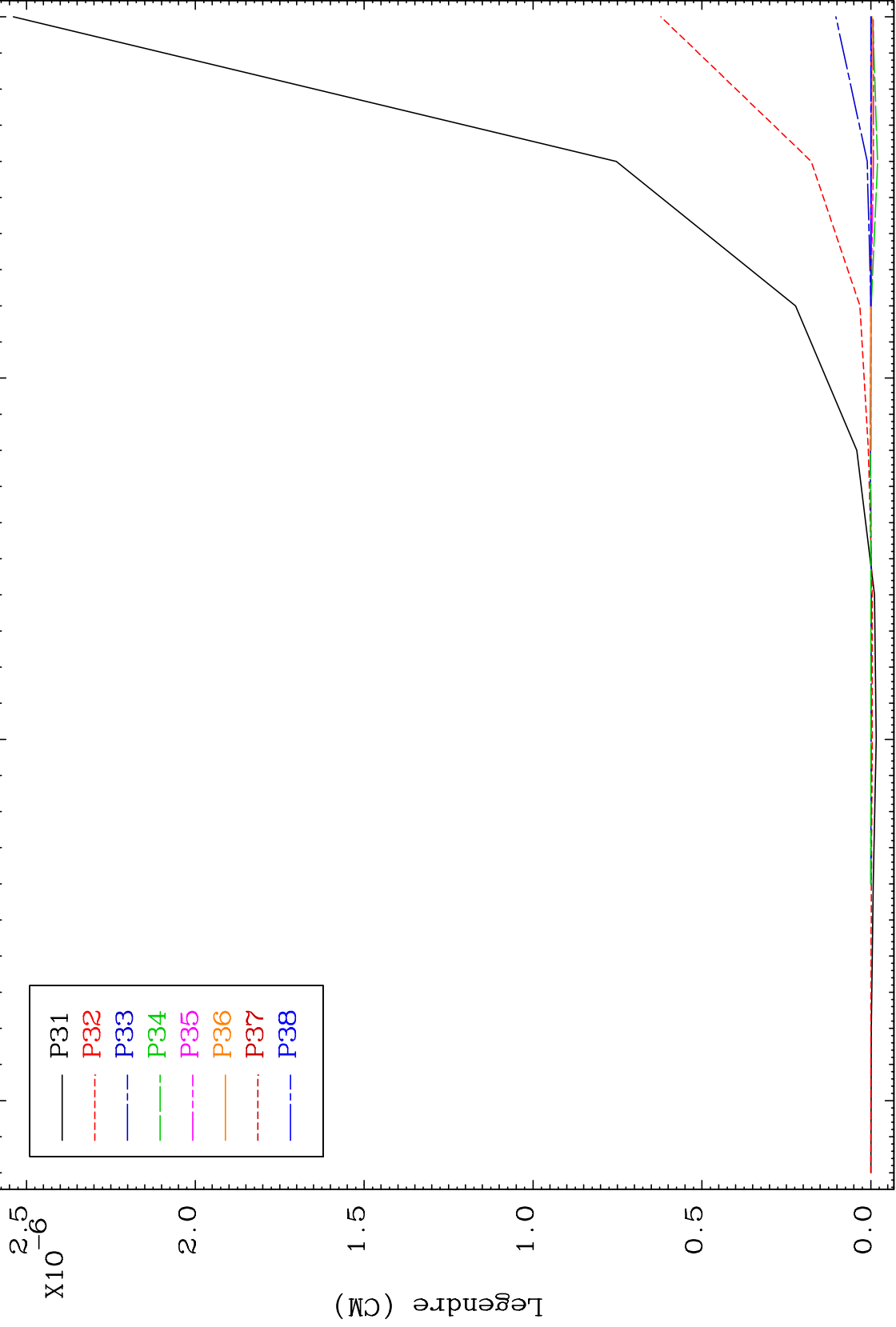
Incident Energy (MeV)

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Elastic
Legendre Coefficients

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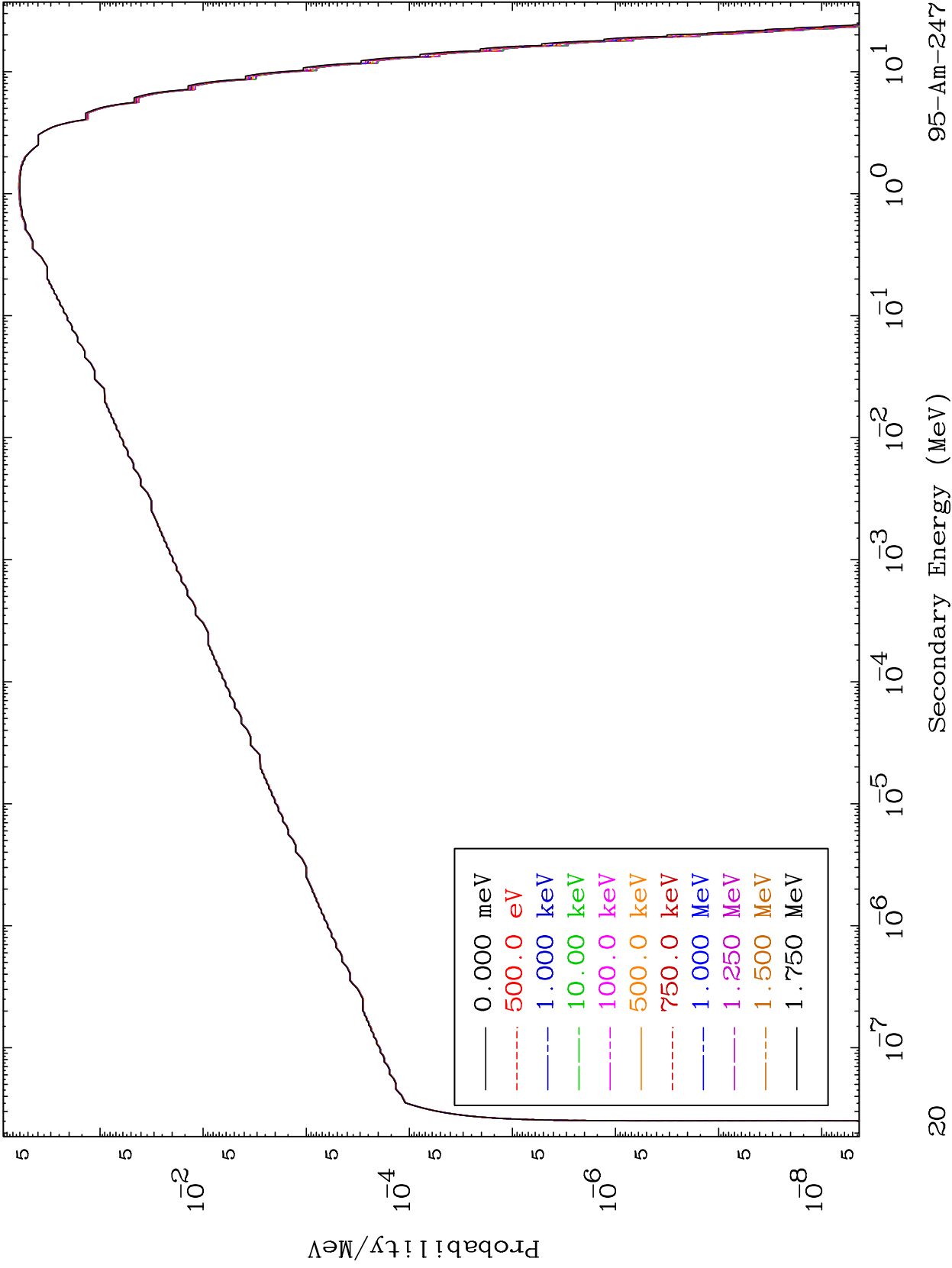
Incident Energy (MeV)

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Fission Energy Distribution

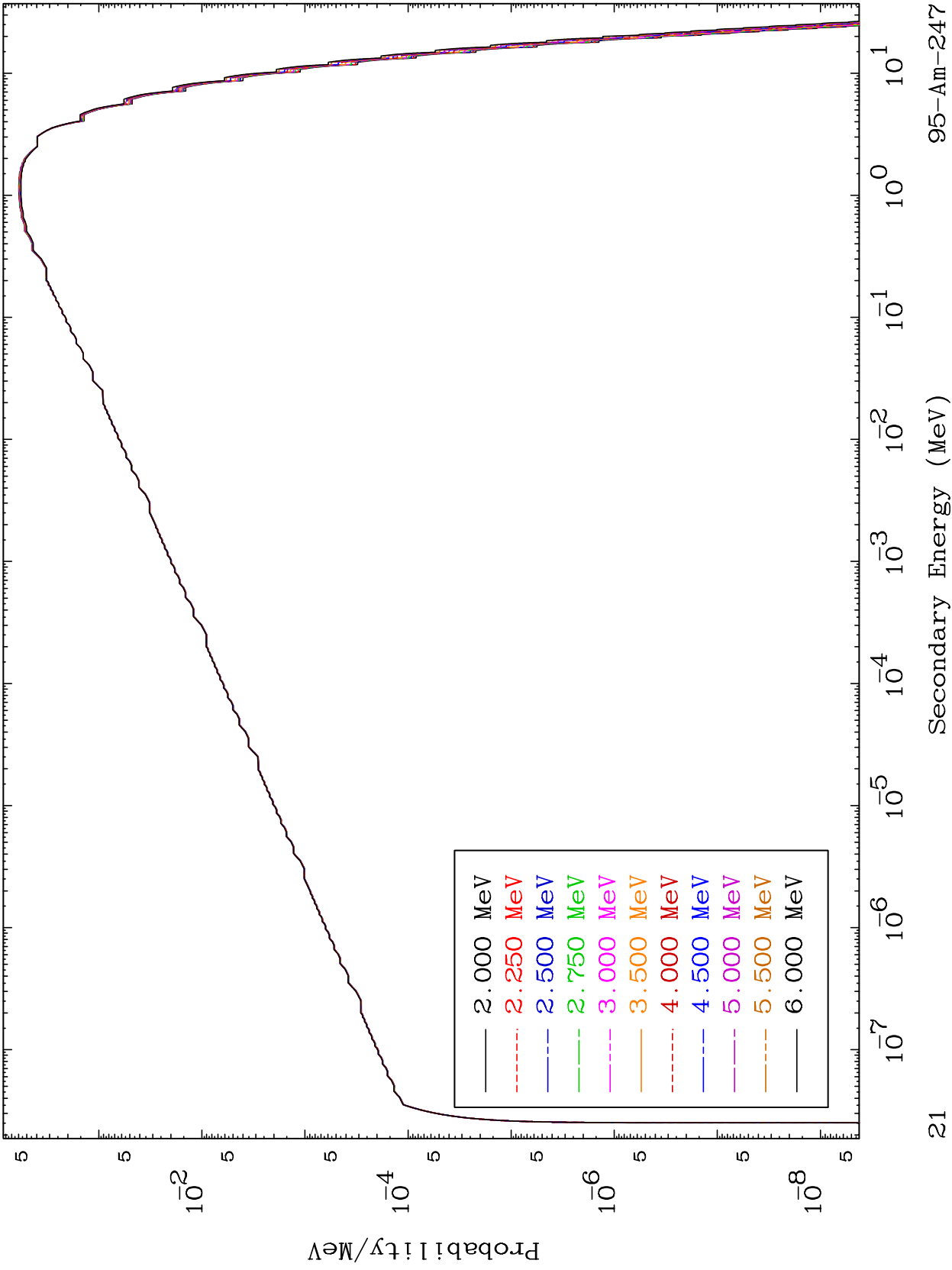
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Fission Energy Distribution

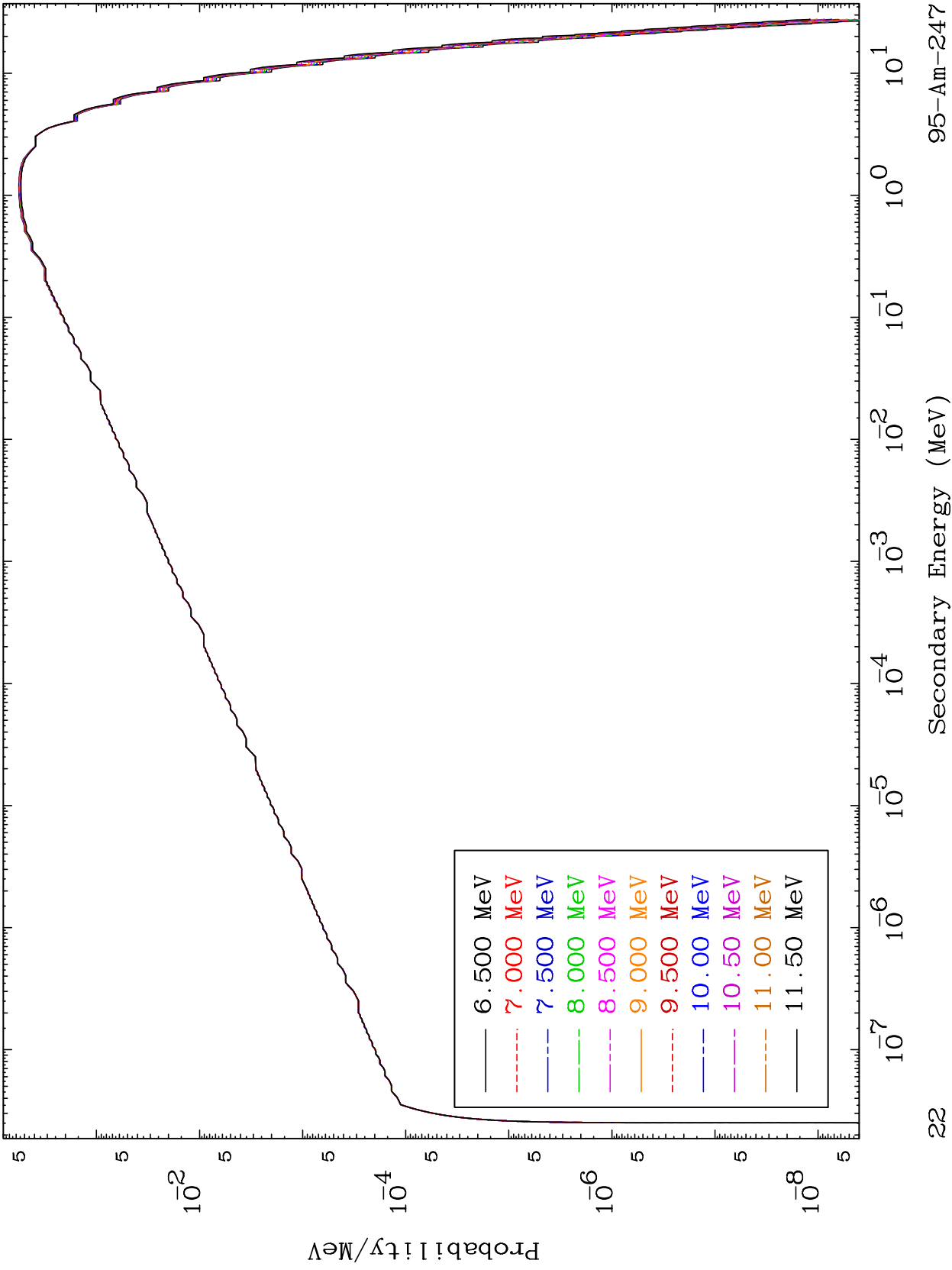
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Fission Energy Distribution

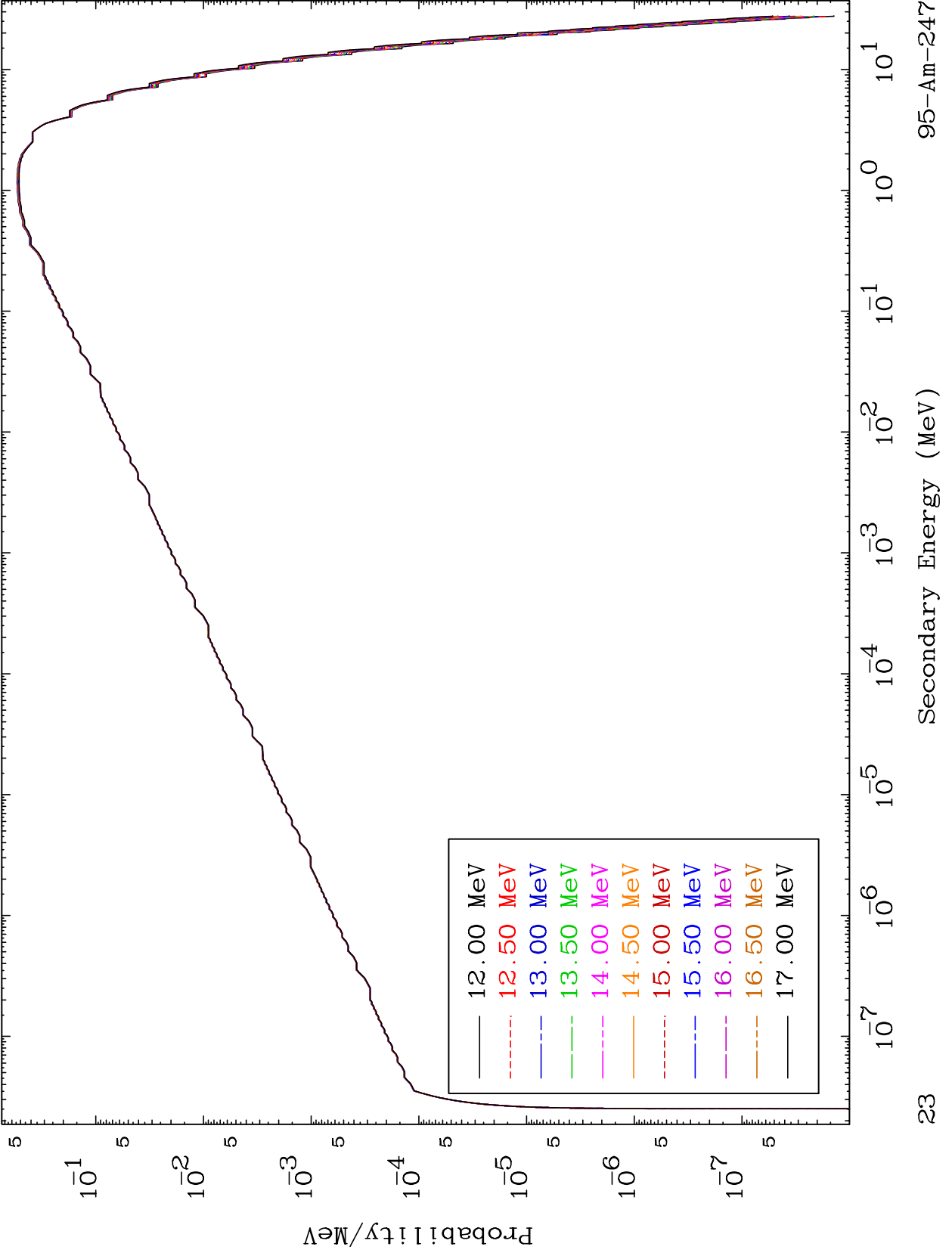
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Fission Energy Distribution

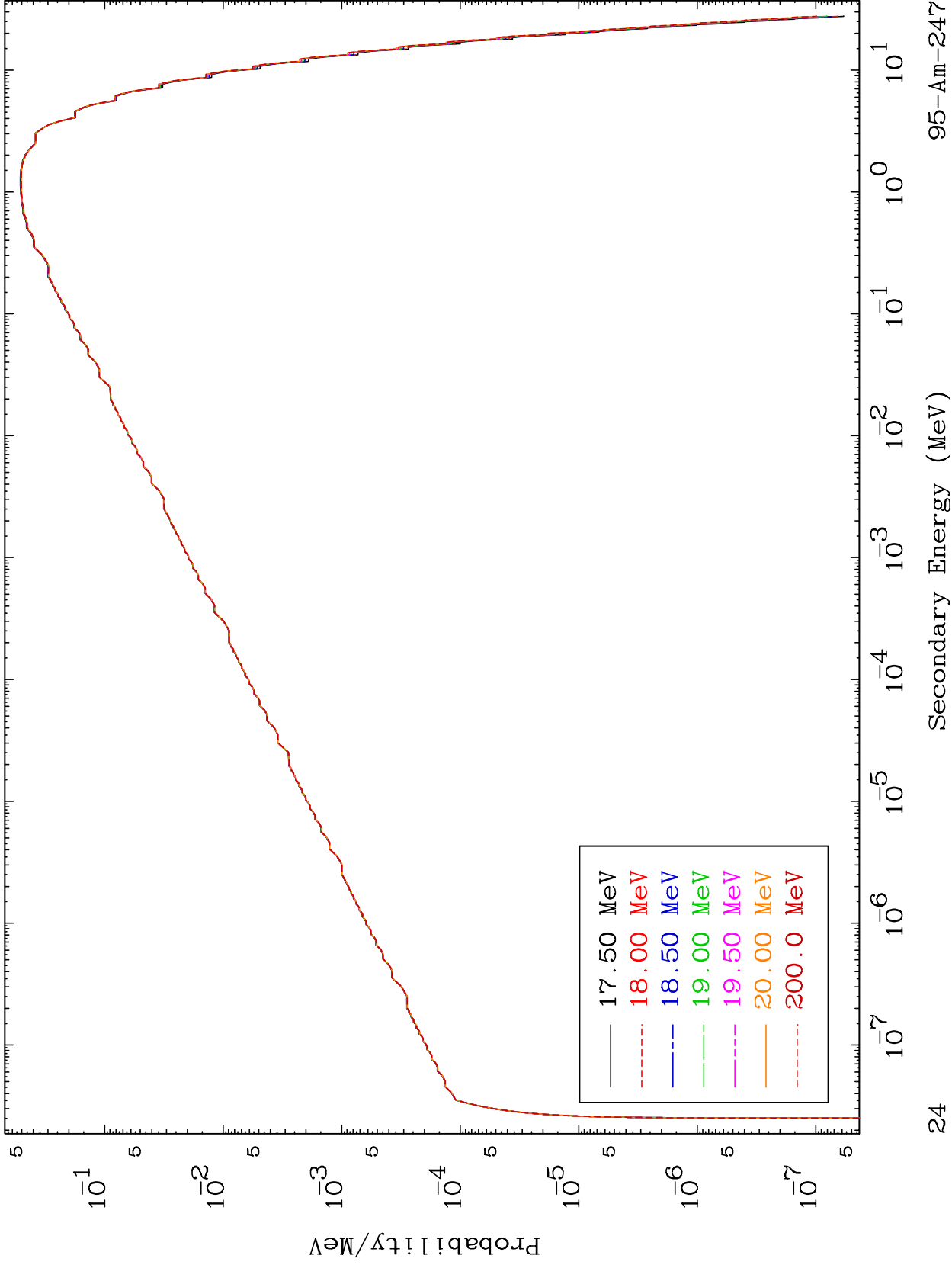
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Fission Energy Distribution

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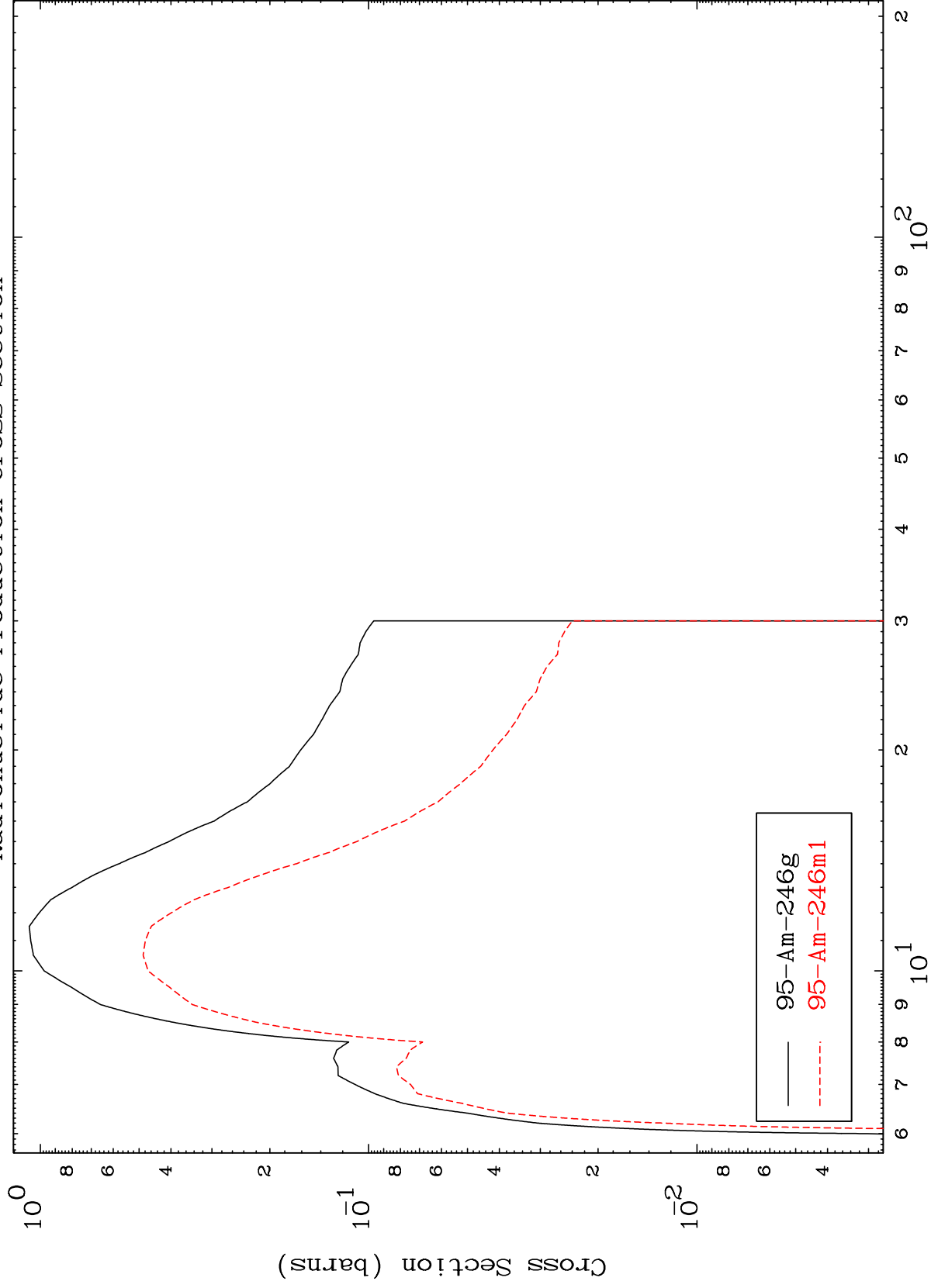


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Radionuclide Production Cross Section

(n,2n)



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Incident Energy (MeV)

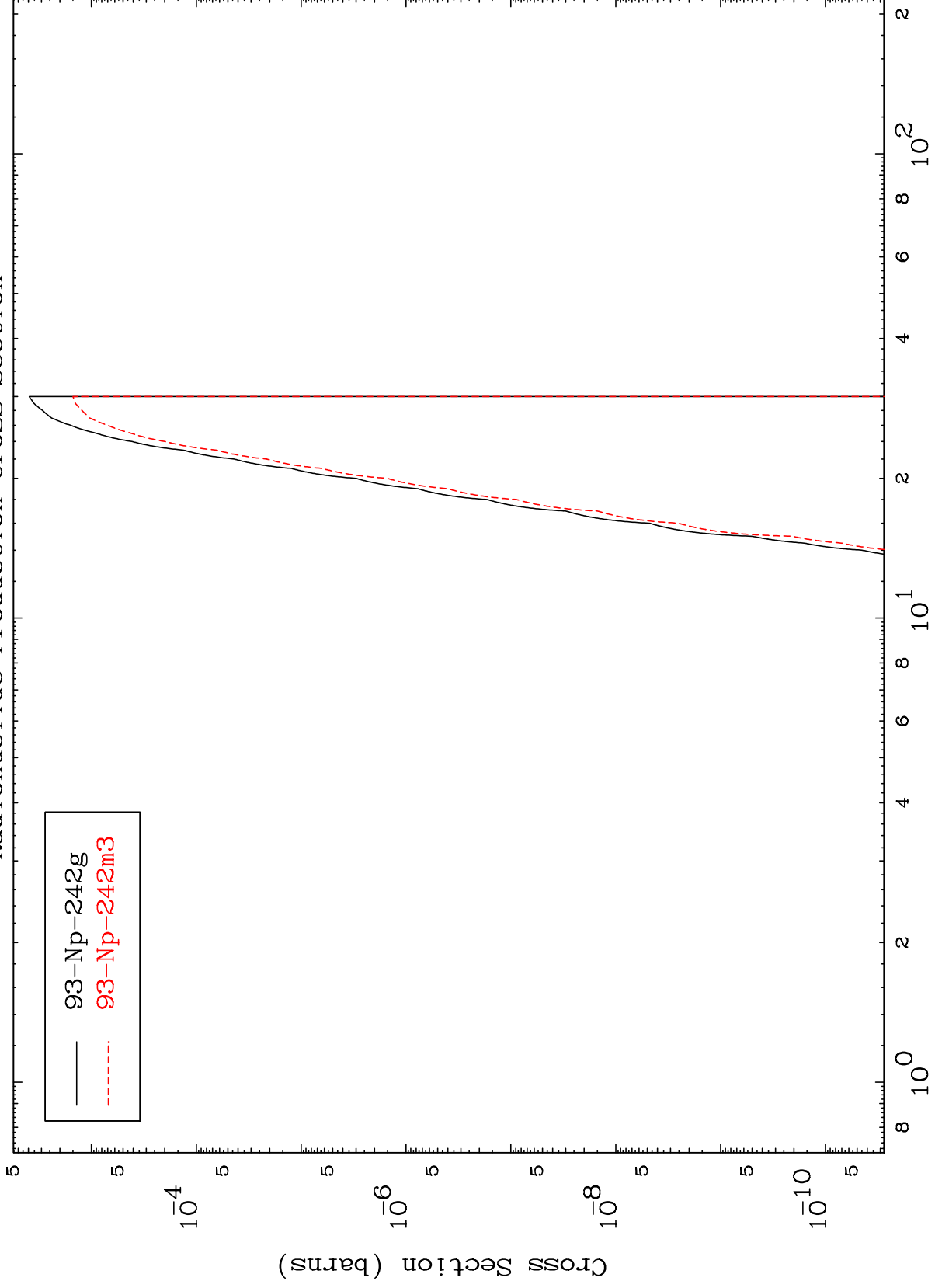
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(n,2n) α

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Radionuclide Production Cross Section



26

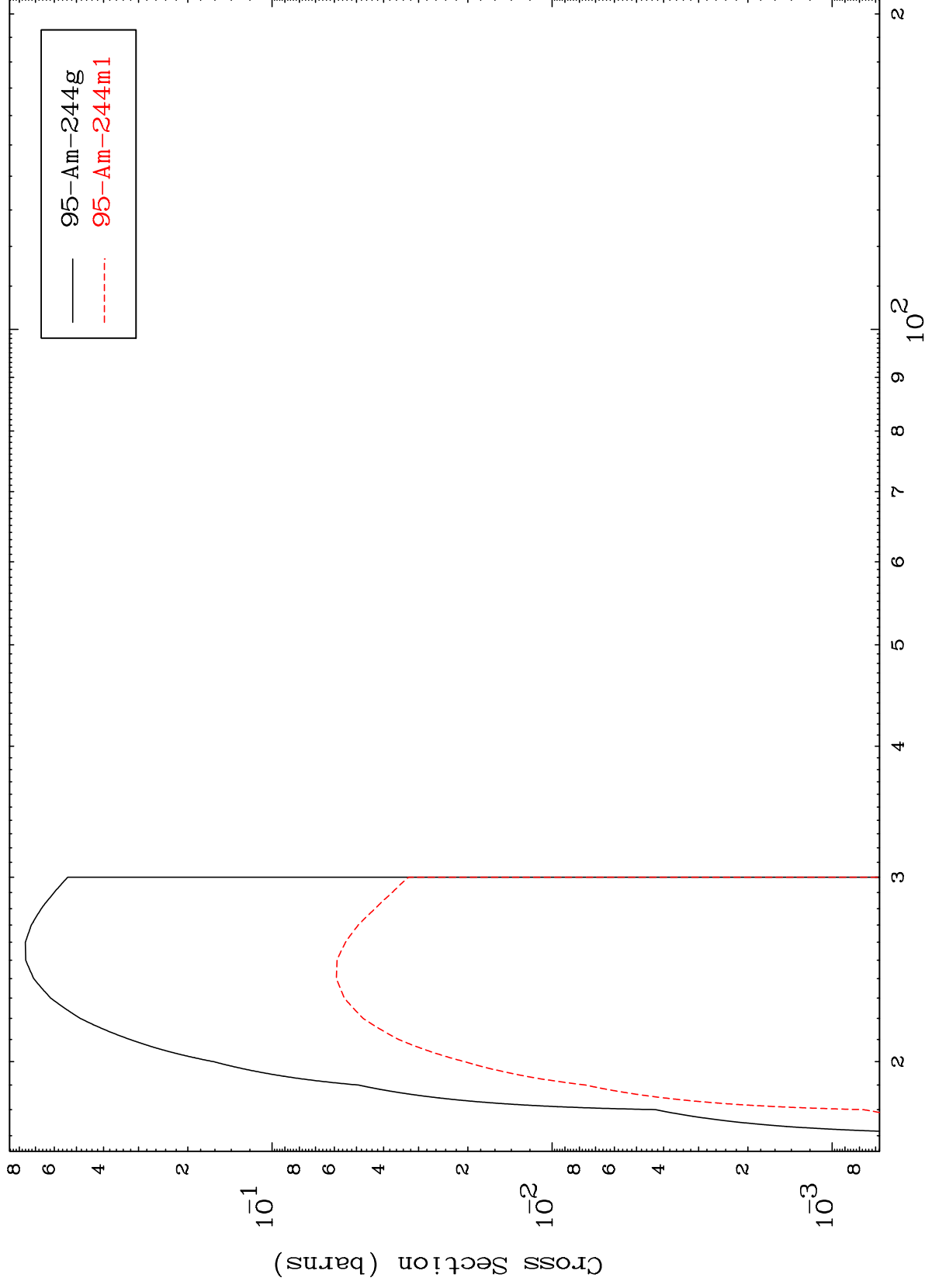
Incident Energy (MeV)

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(n,4n)
Radionuclide Production Cross Section



27

Incident Energy (MeV)

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