

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

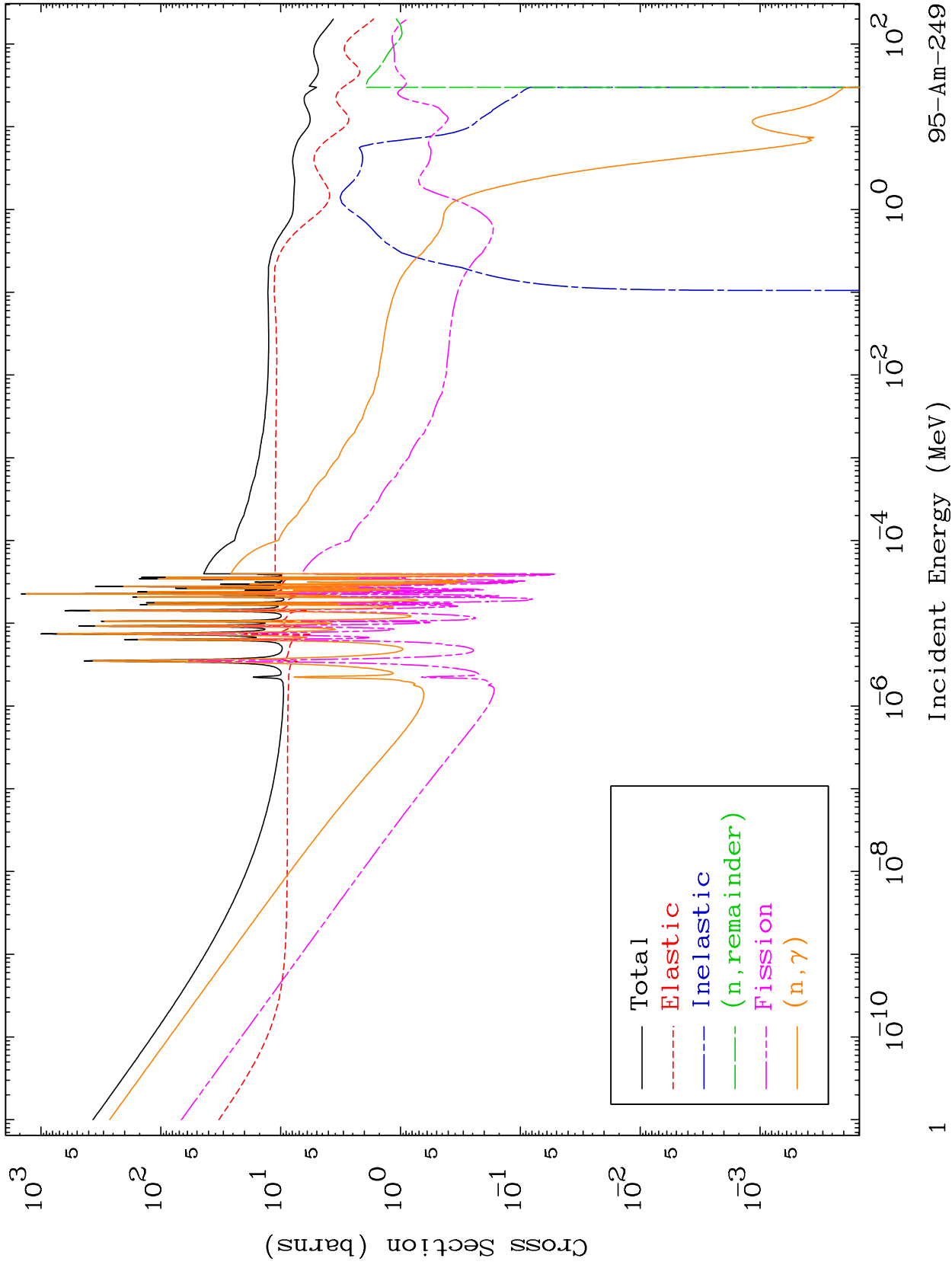
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Press Mouse Button to Start

MAT 9567

Major
293 Kelvin Cross Sections

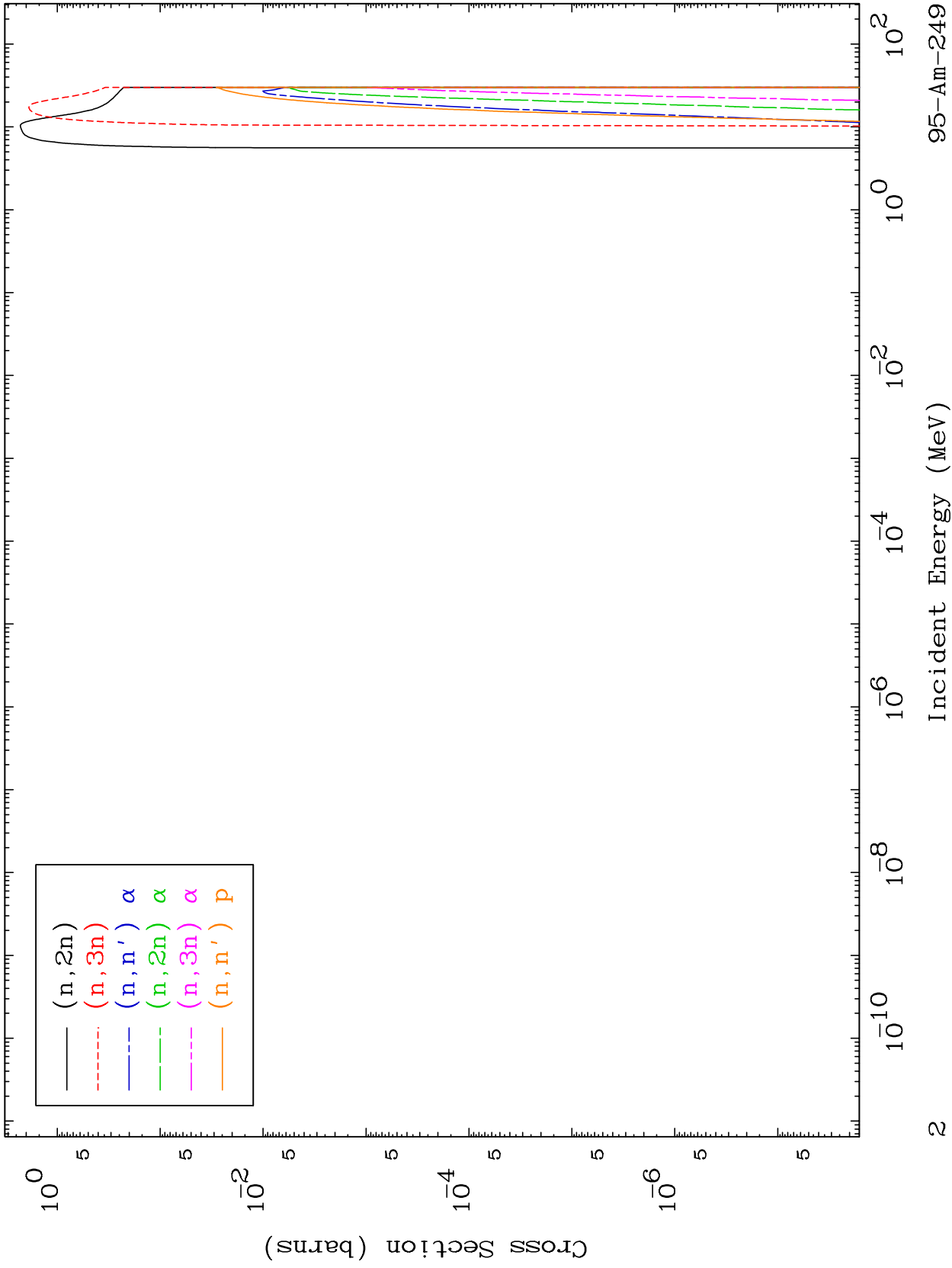
95-Am-249



MAT 9567

Neutron Production
293 Kelvin Cross Sections

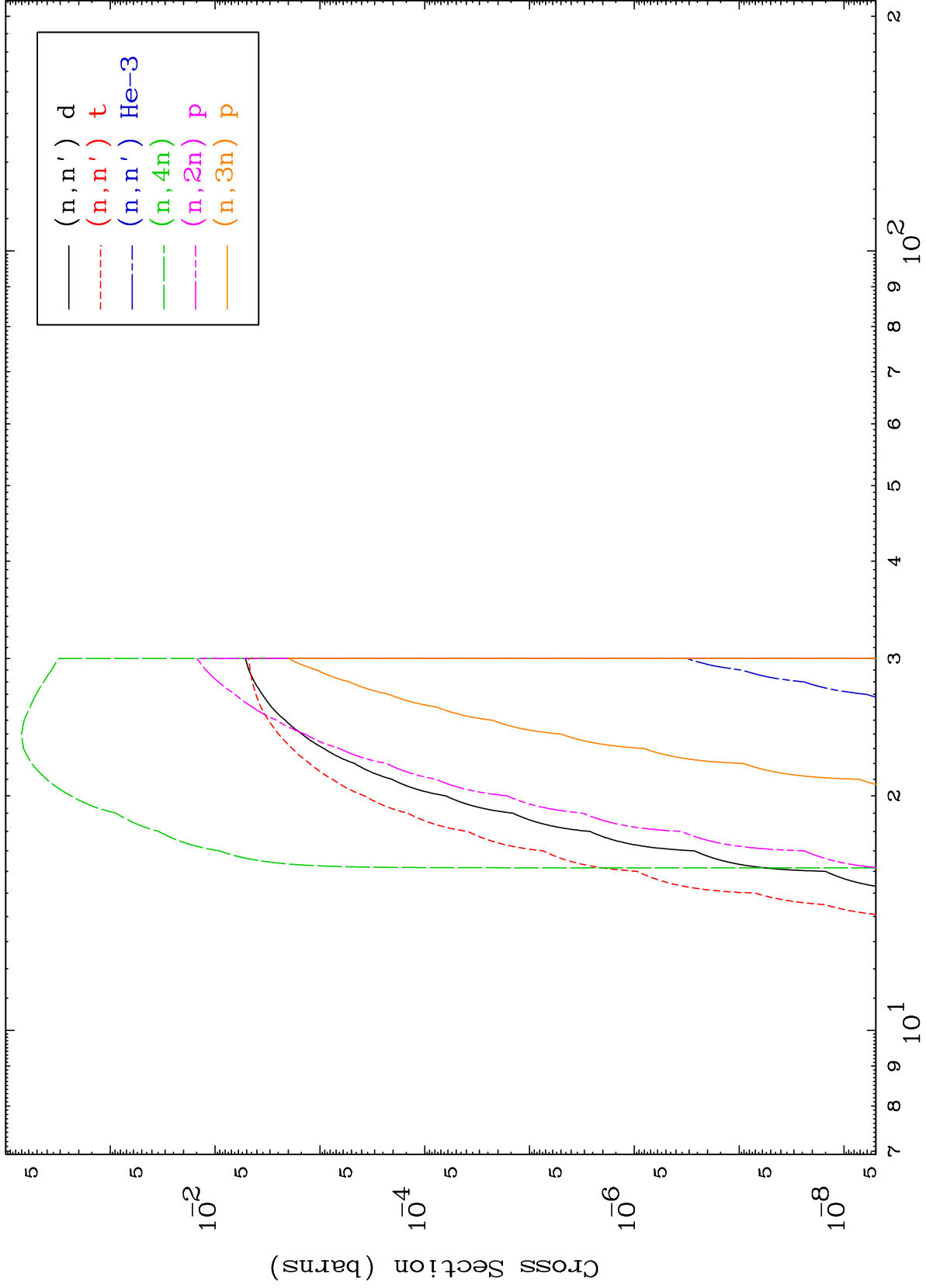
95-Am-249



MAT 9567

Neutron Production
293 Kelvin Cross Sections

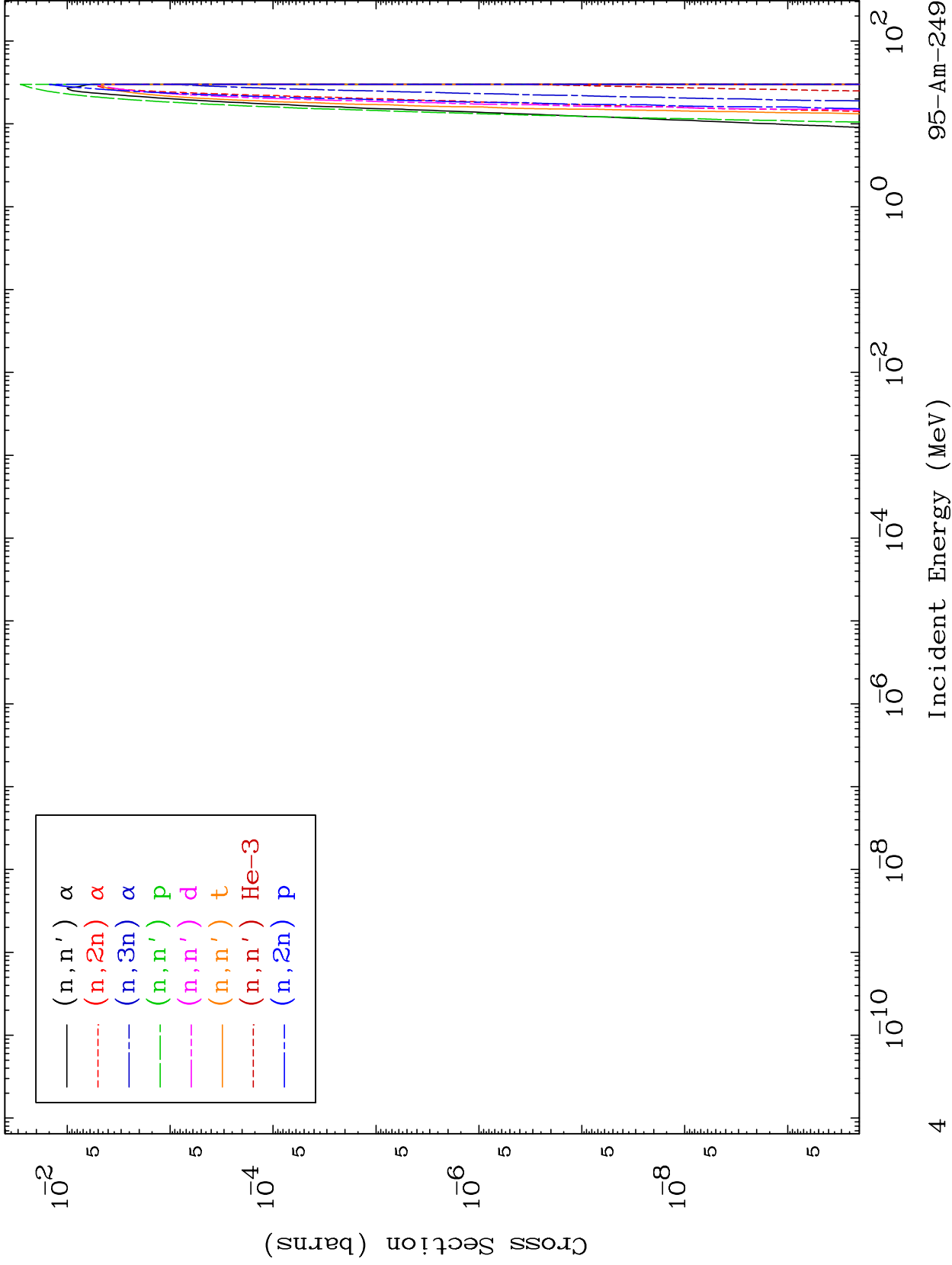
95-Am-249



MAT 9567

Charged Particle
293 Kelvin Cross Sections

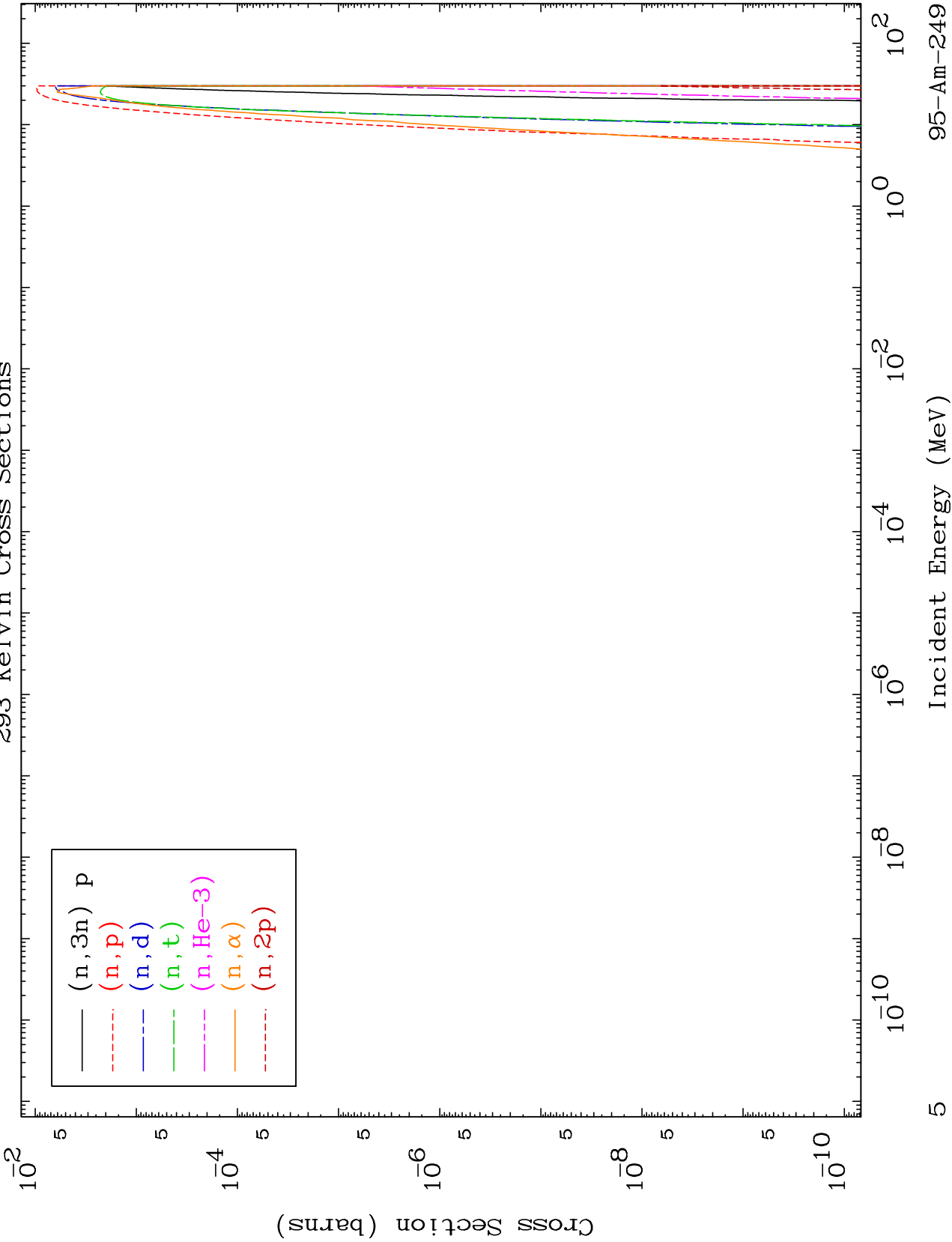
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Charged Particle
293 Kelvin Cross Sections

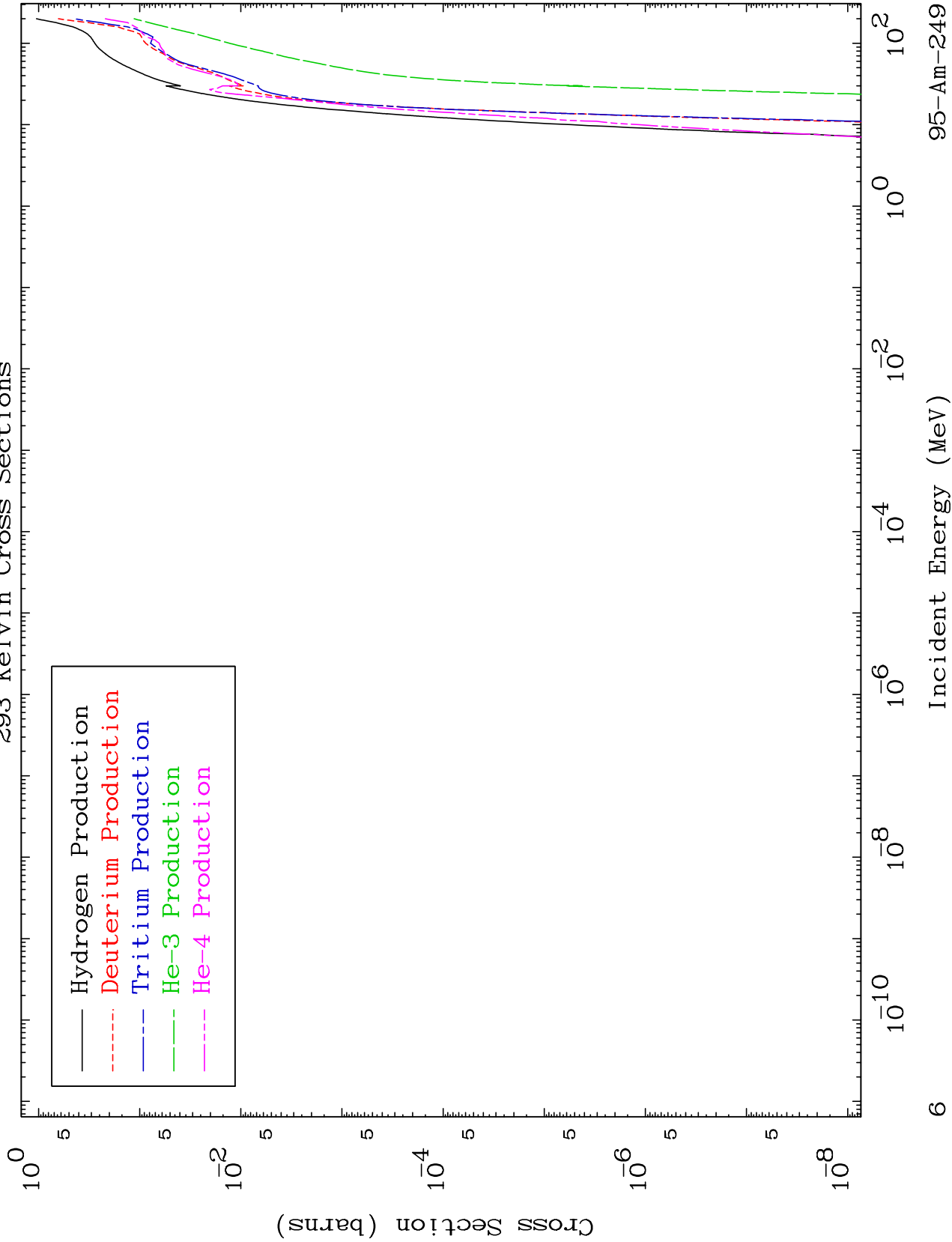
95-Am-249



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Particle Production
293 Kelvin Cross Sections

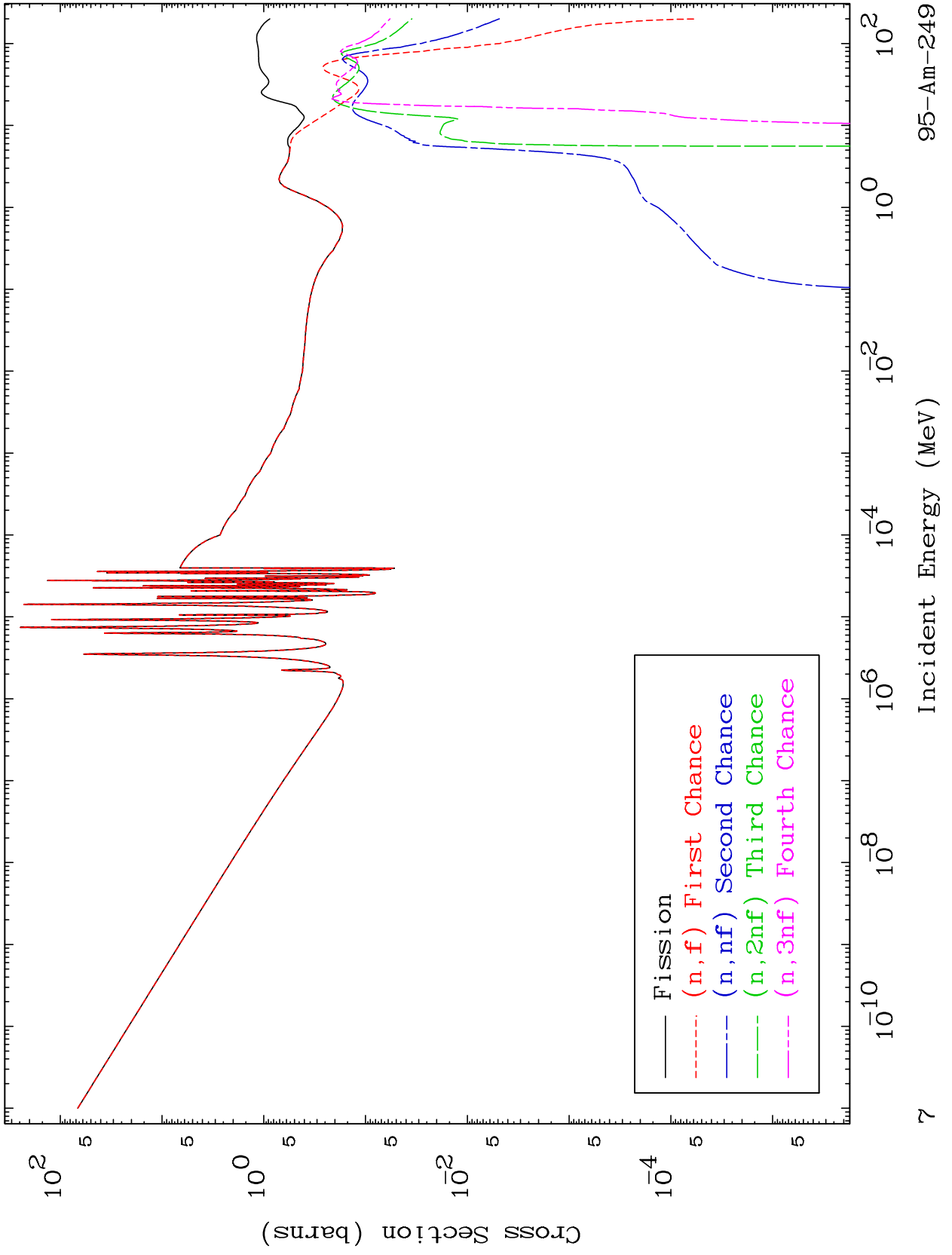
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Fission
293 Kelvin Cross Sections

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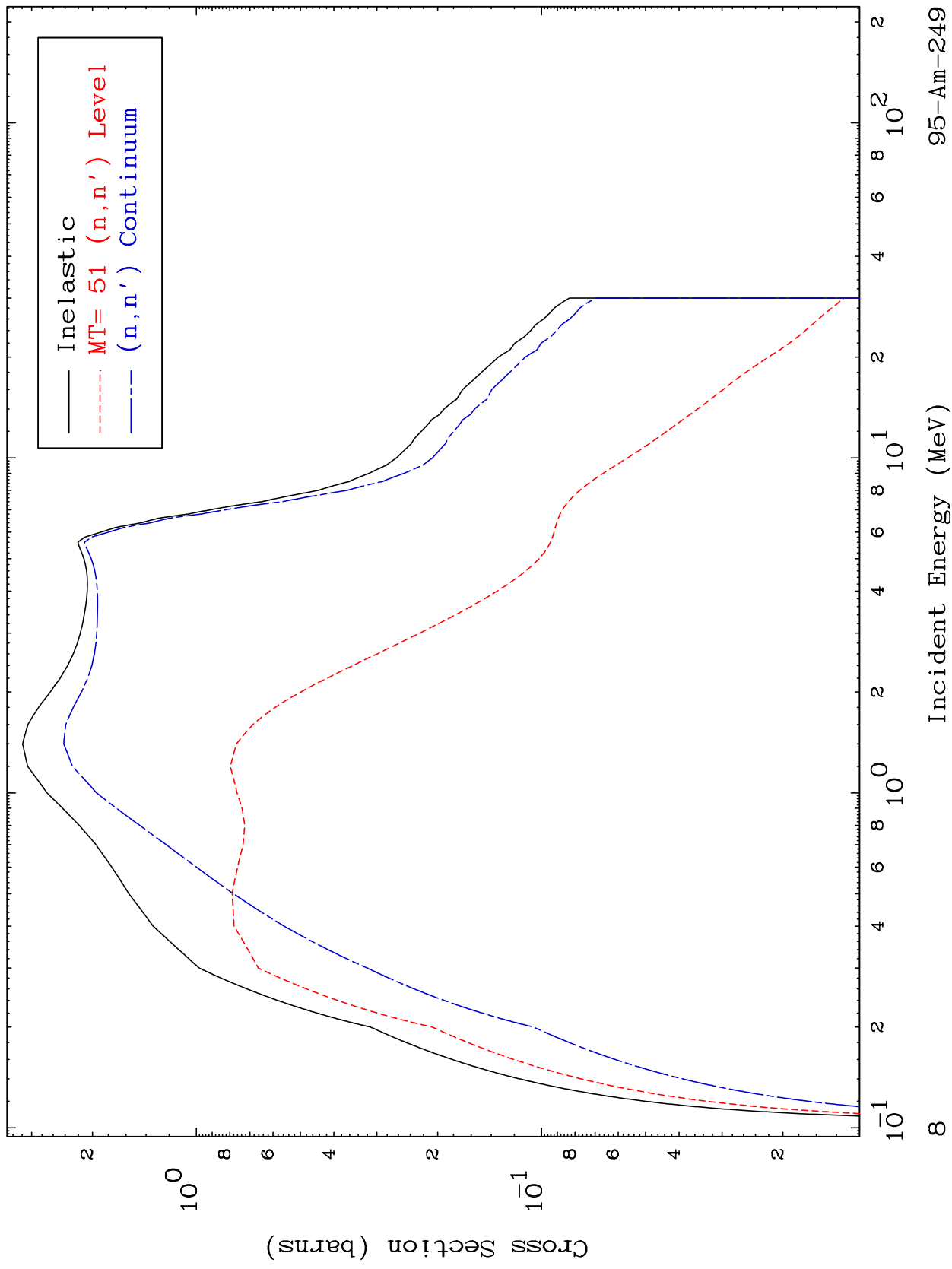


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(n,n') Level

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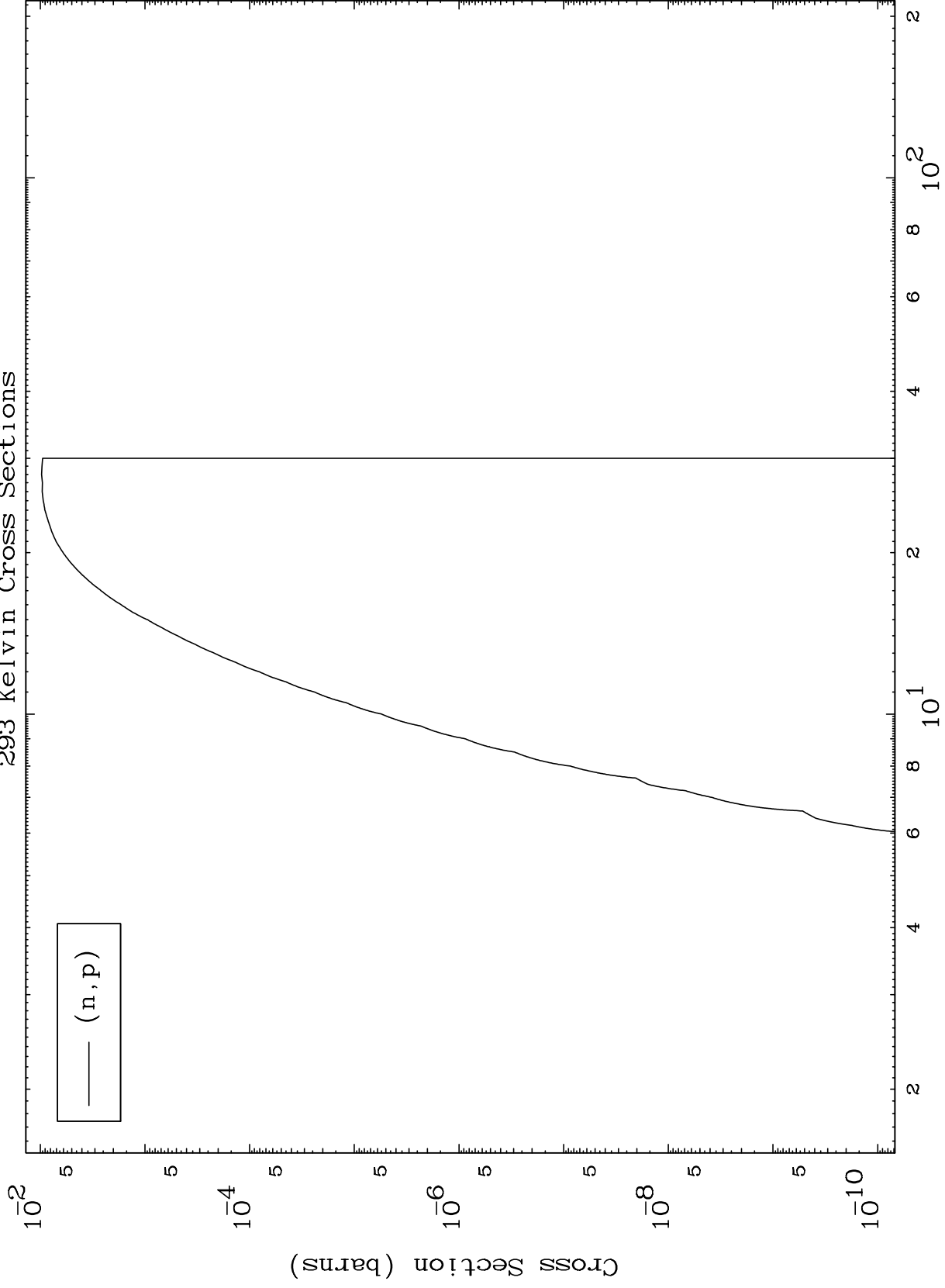
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

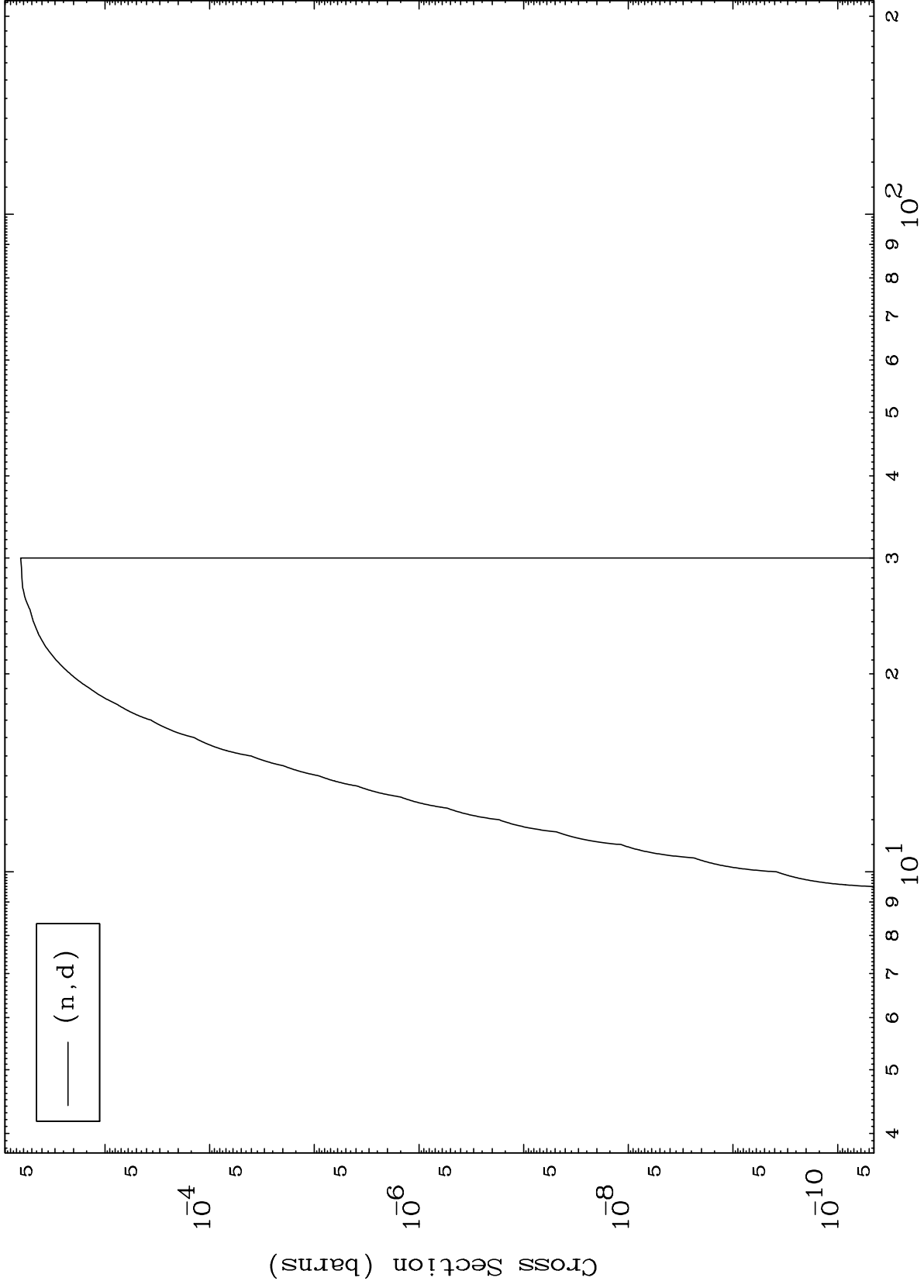
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(n,d) Levels
293 Kelvin Cross Sections

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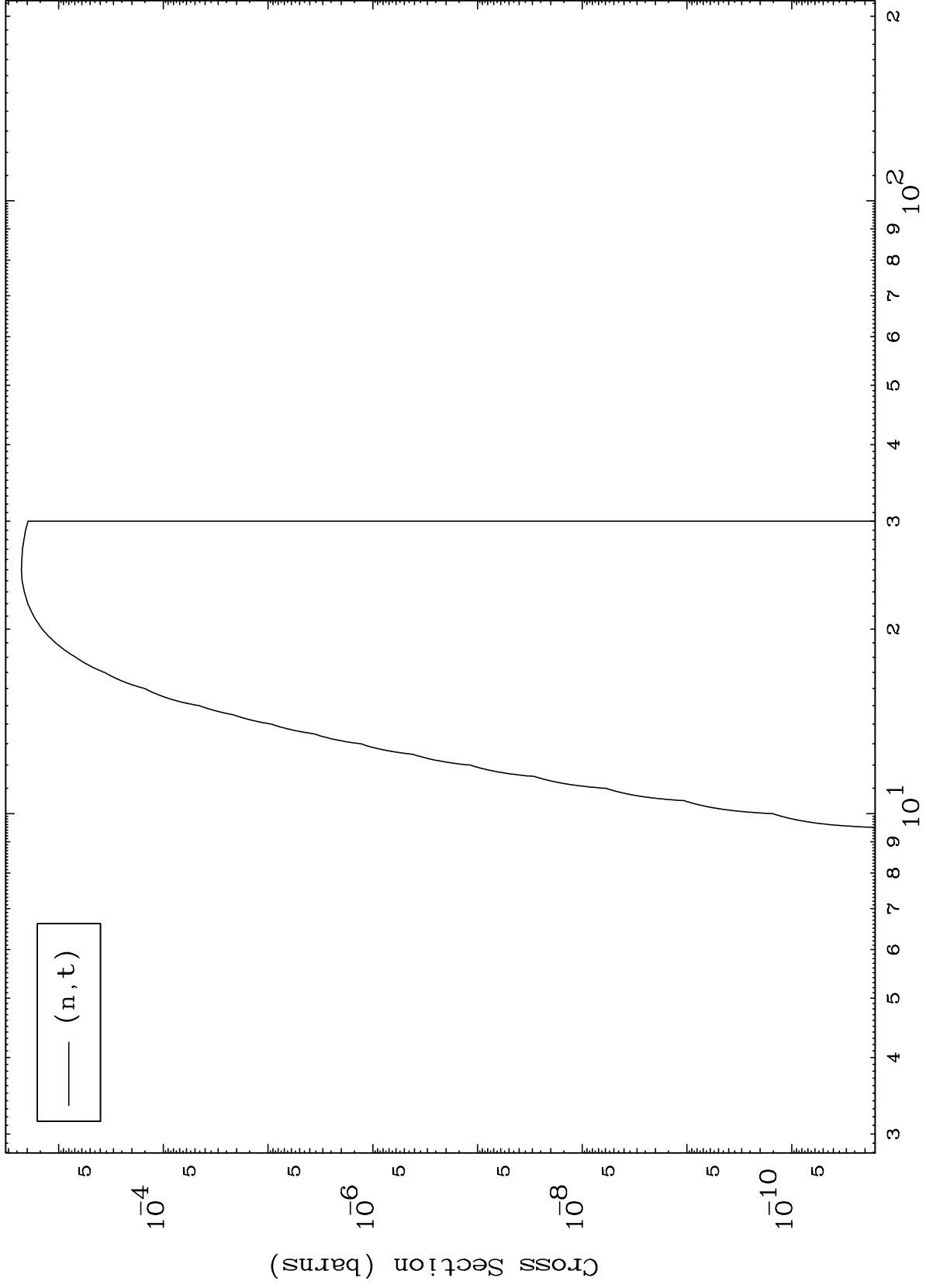
Incident Energy (MeV)

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MAT 9567

(n,t) Levels
293 Kelvin Cross Sections

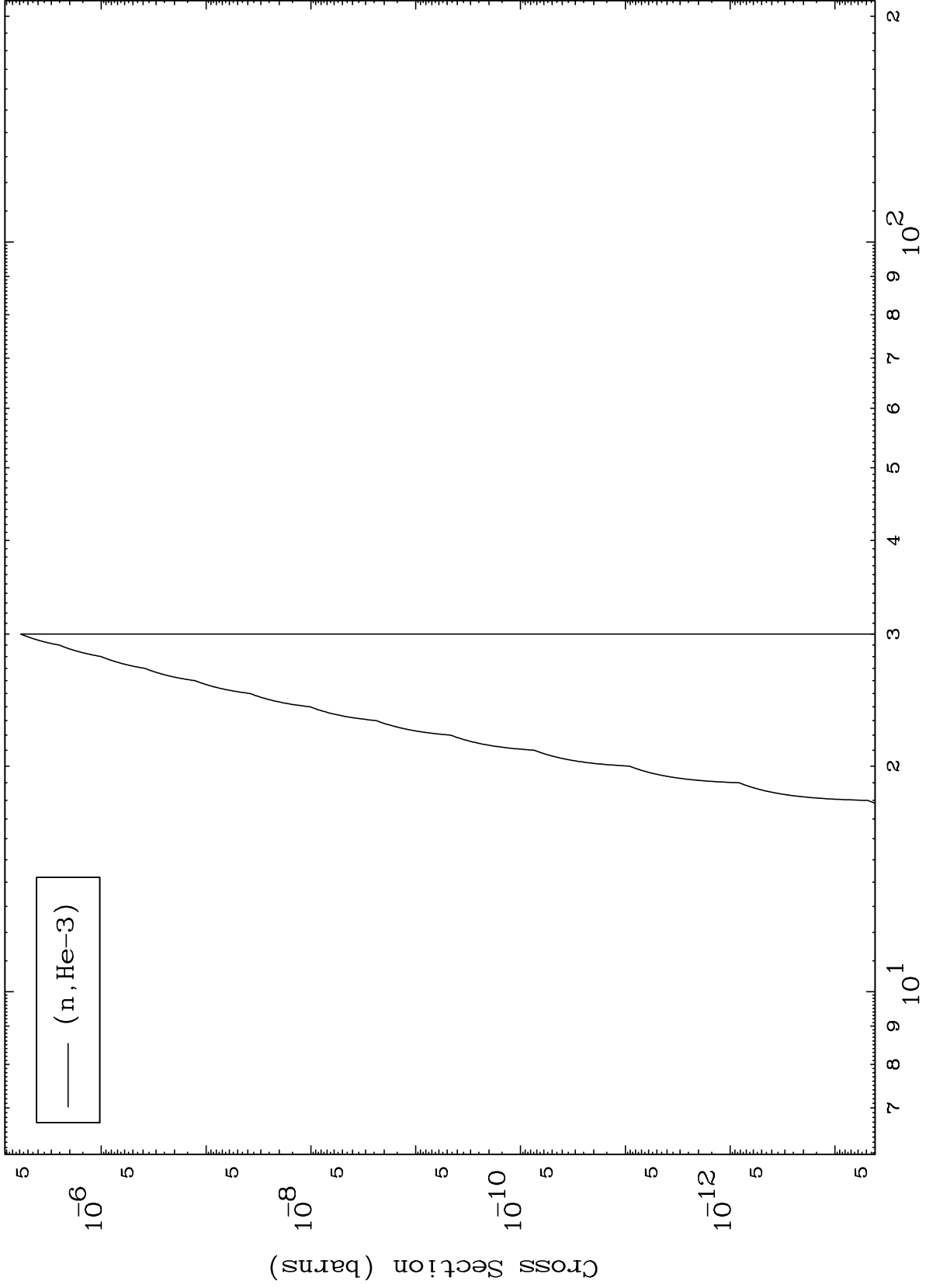
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(n,He3) Levels
293 Kelvin Cross Sections

95-Am-249



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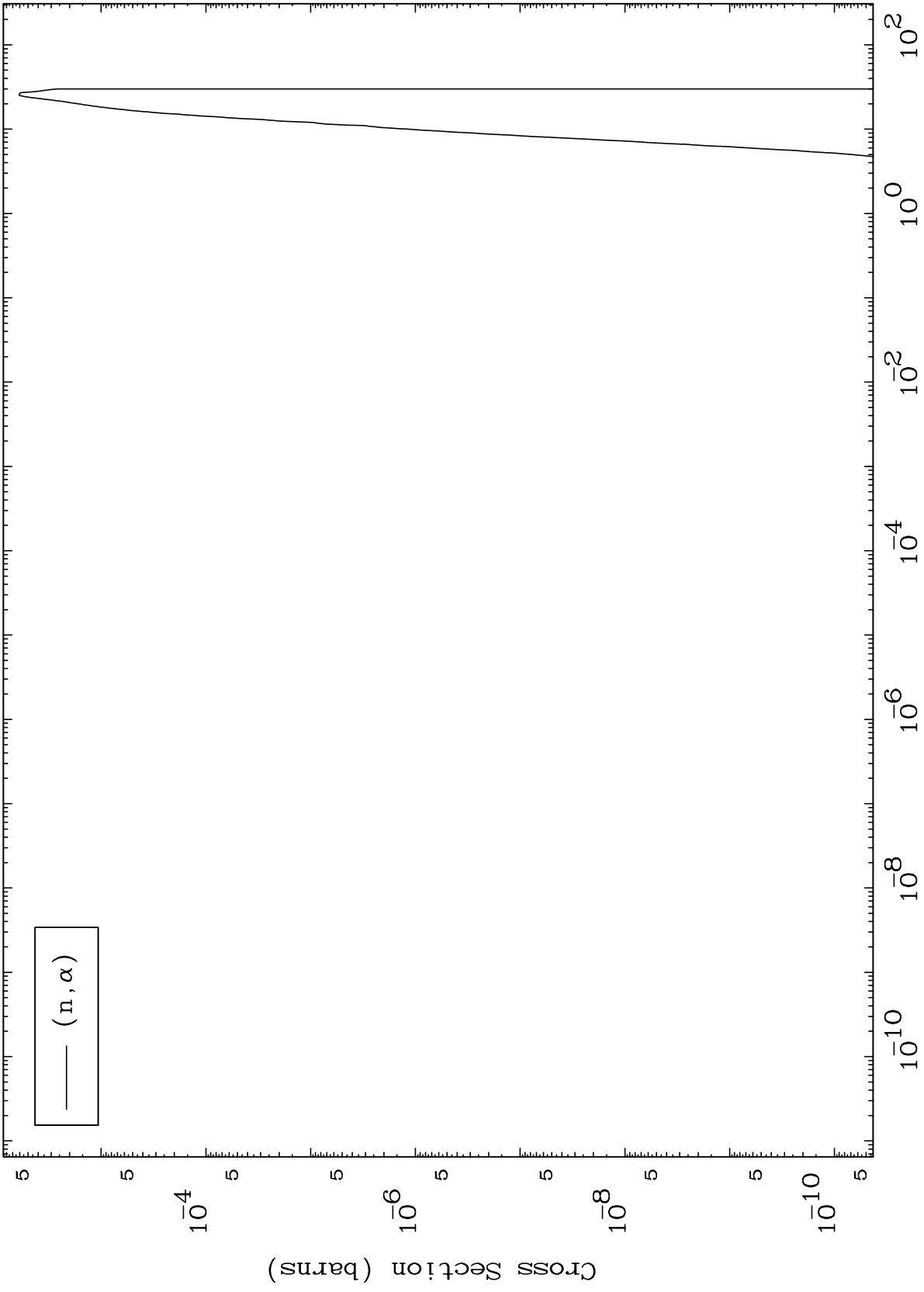
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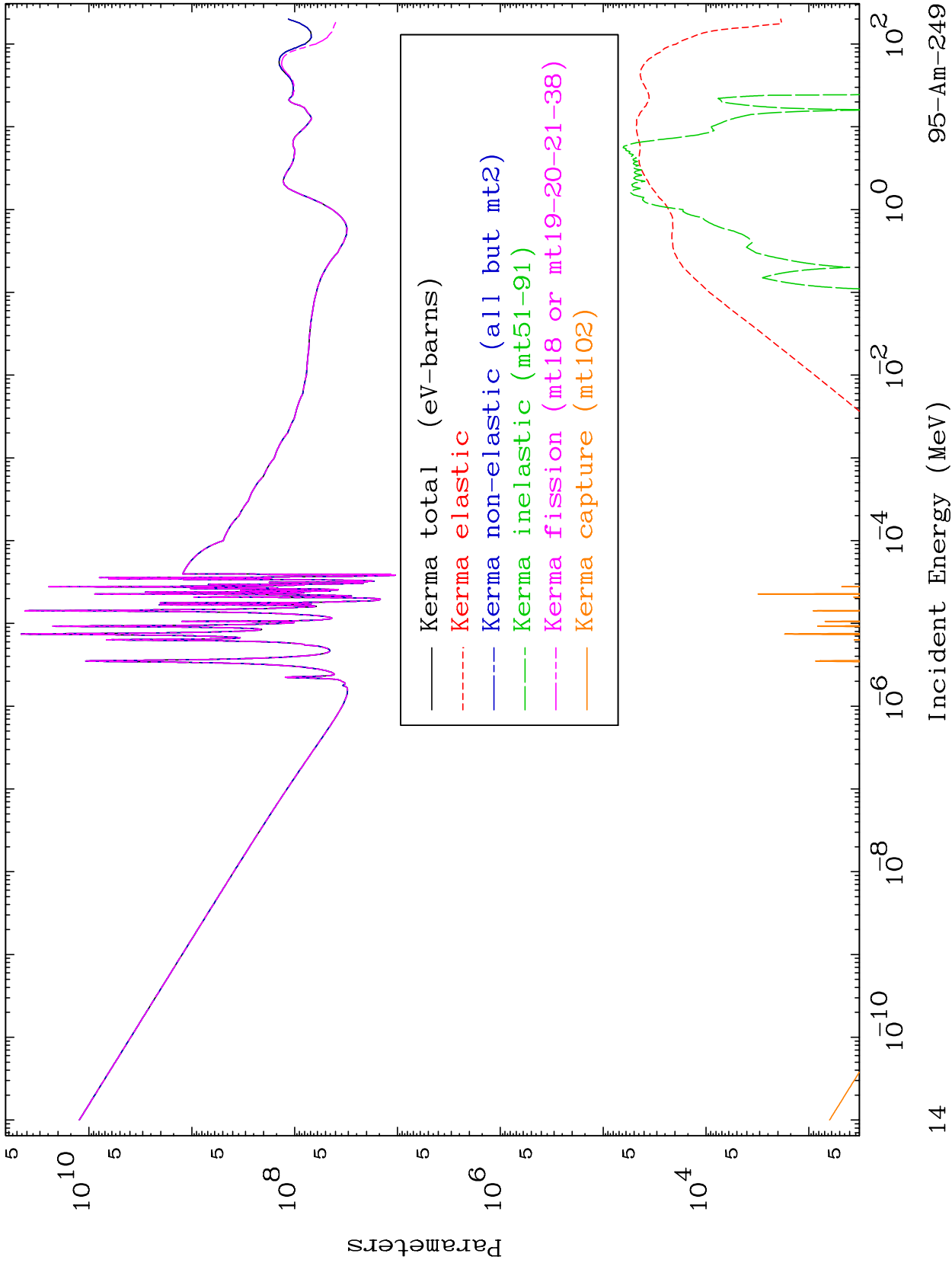
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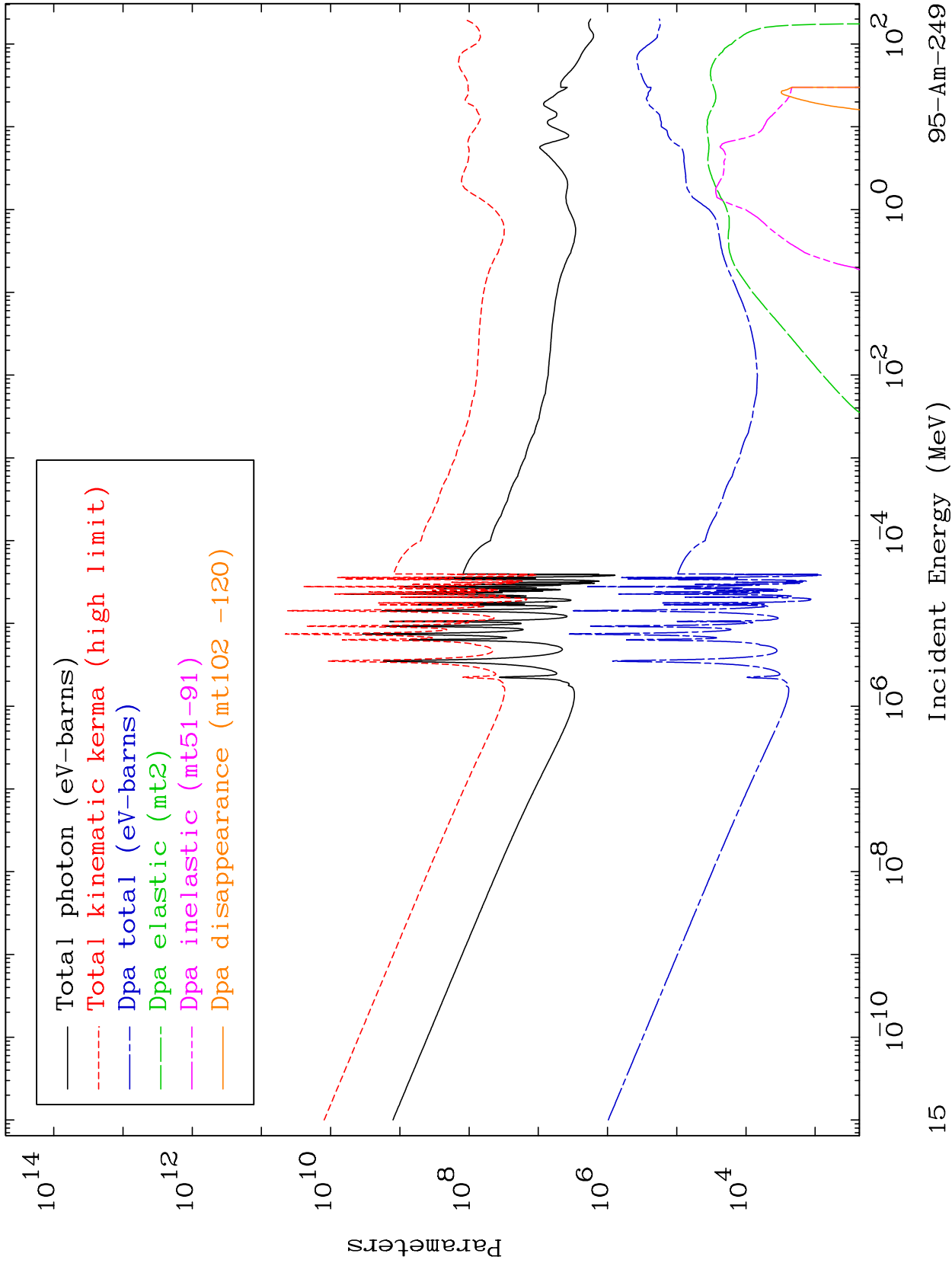
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(n, α) Levels
293 Kelvin Cross Sections

95-Am-249



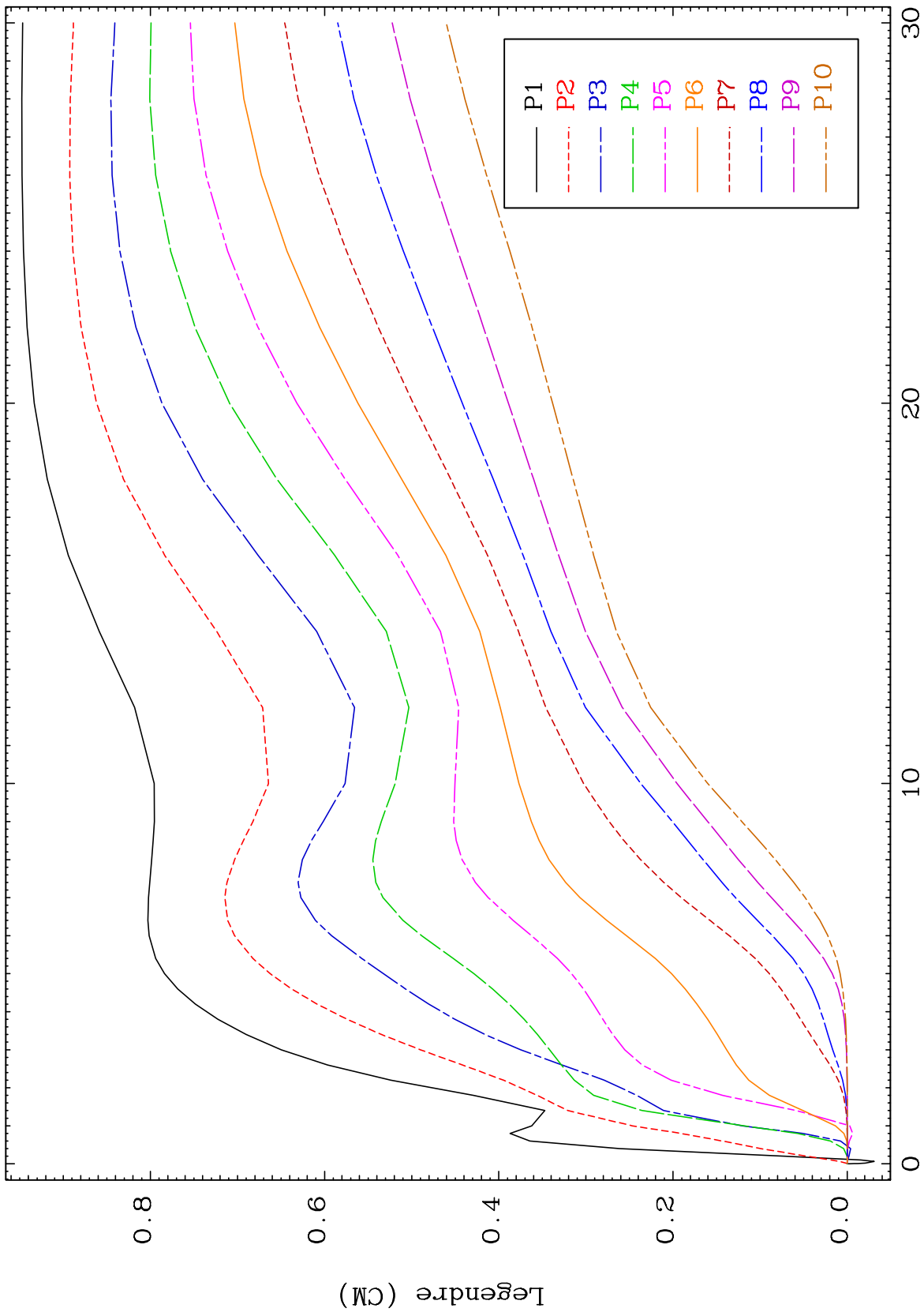




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95-Am-249

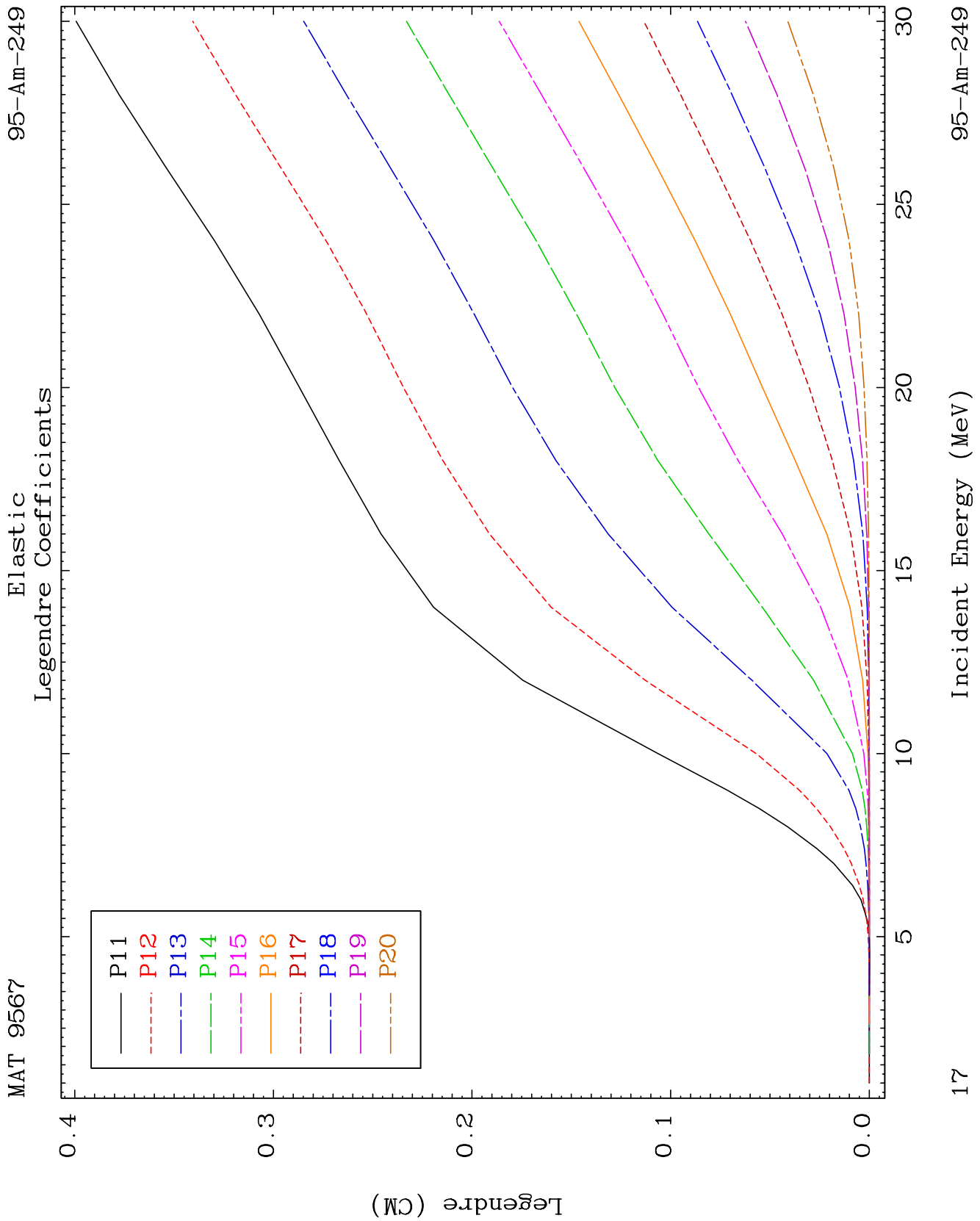
Elastic Legendre Coefficients



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Incident Energy (MeV)

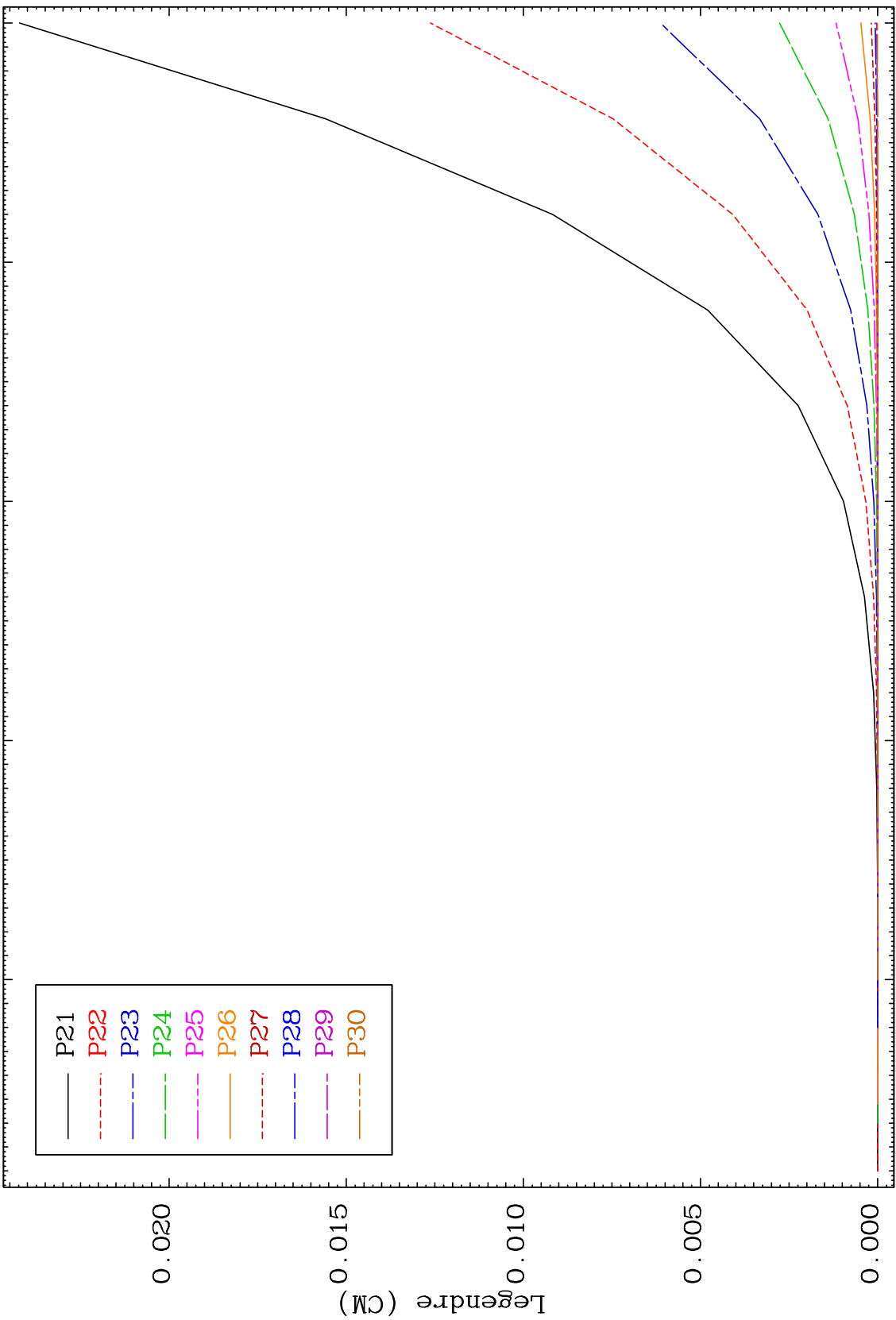
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Elastic Legendre Coefficients

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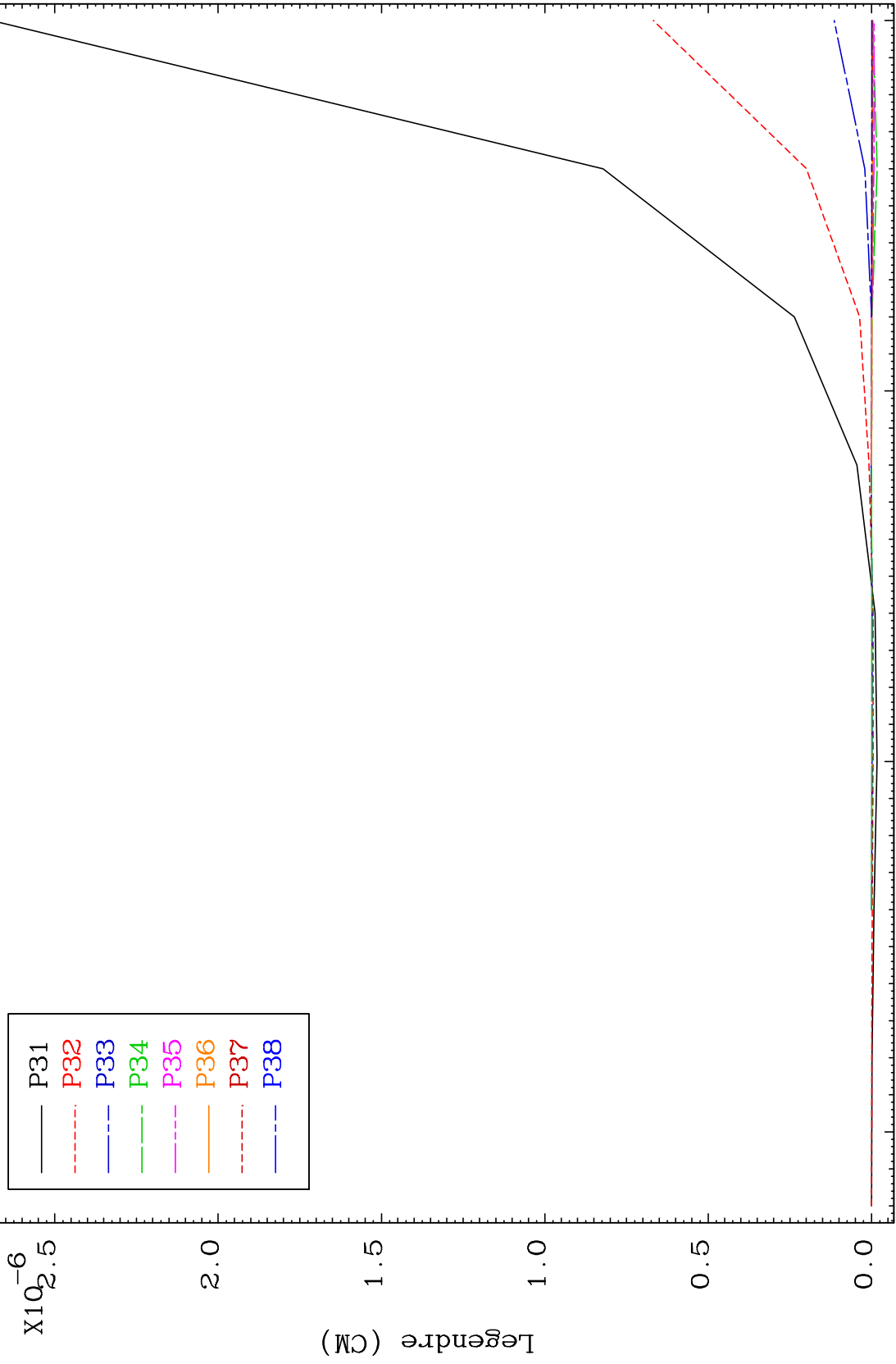
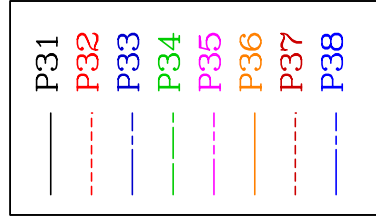
Incident Energy (MeV)

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Elastic
Legendre Coefficients

95-Am-249



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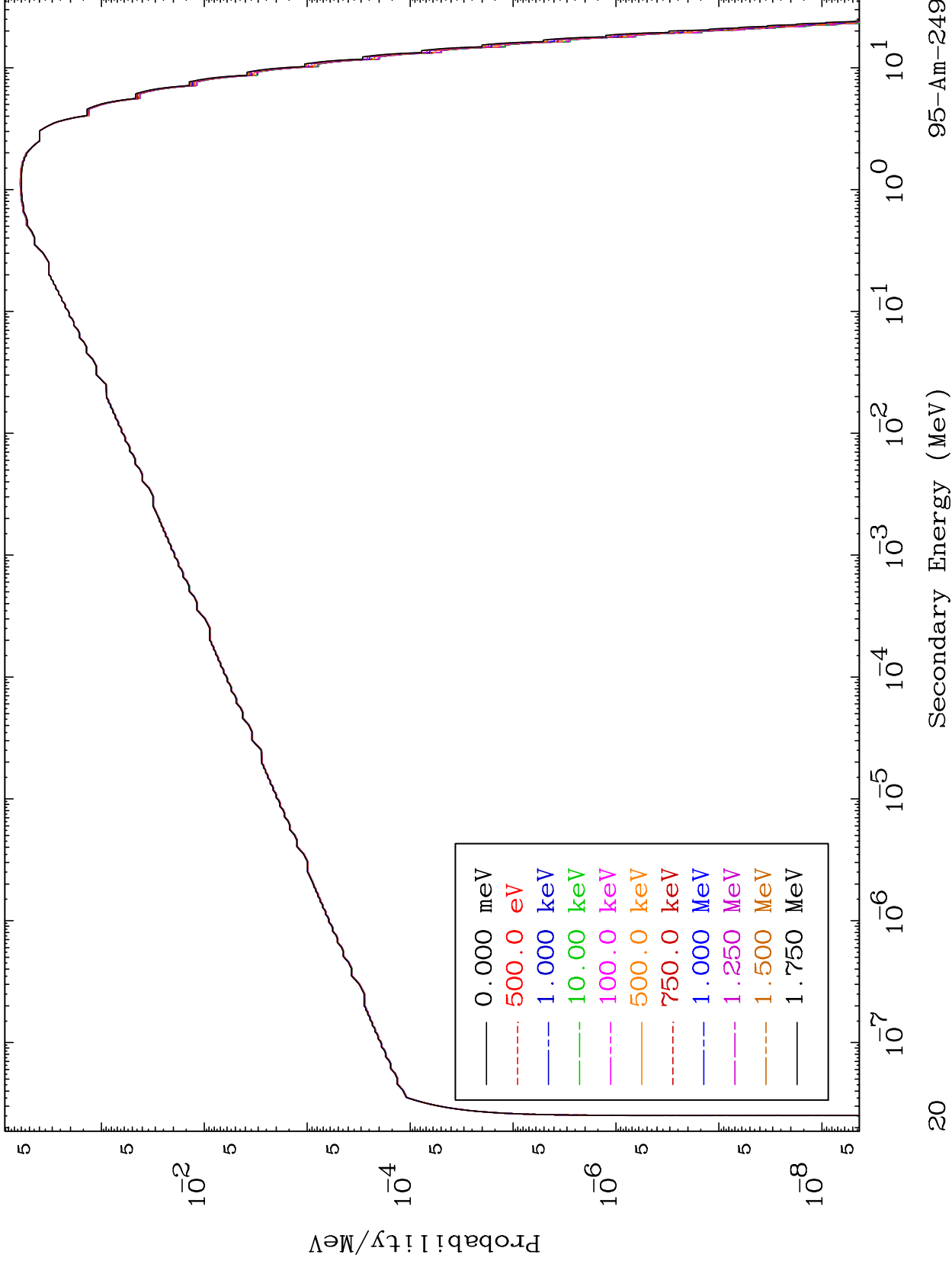
Incident Energy (MeV)

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Fission Energy Distribution

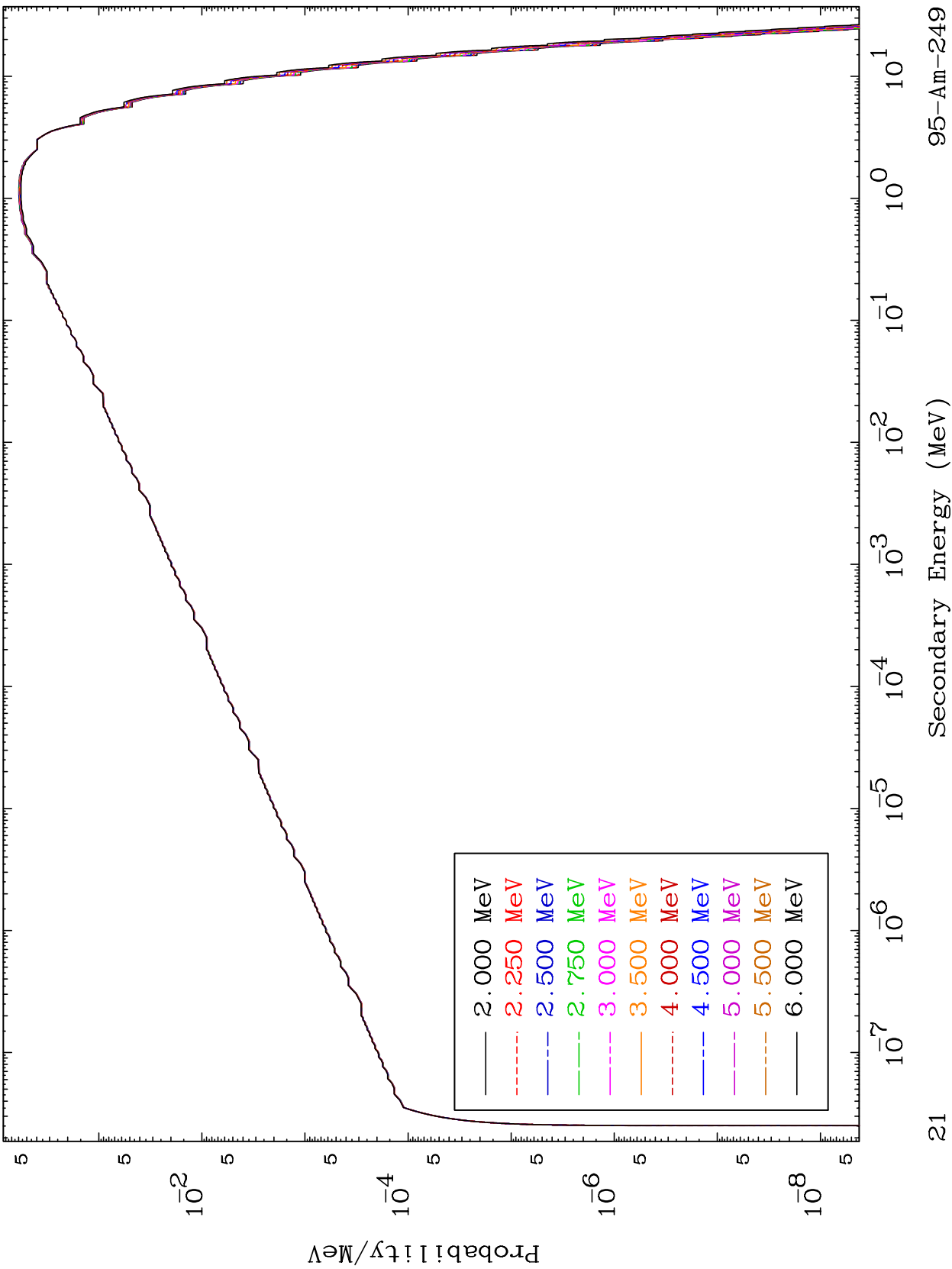
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Fission Energy Distribution

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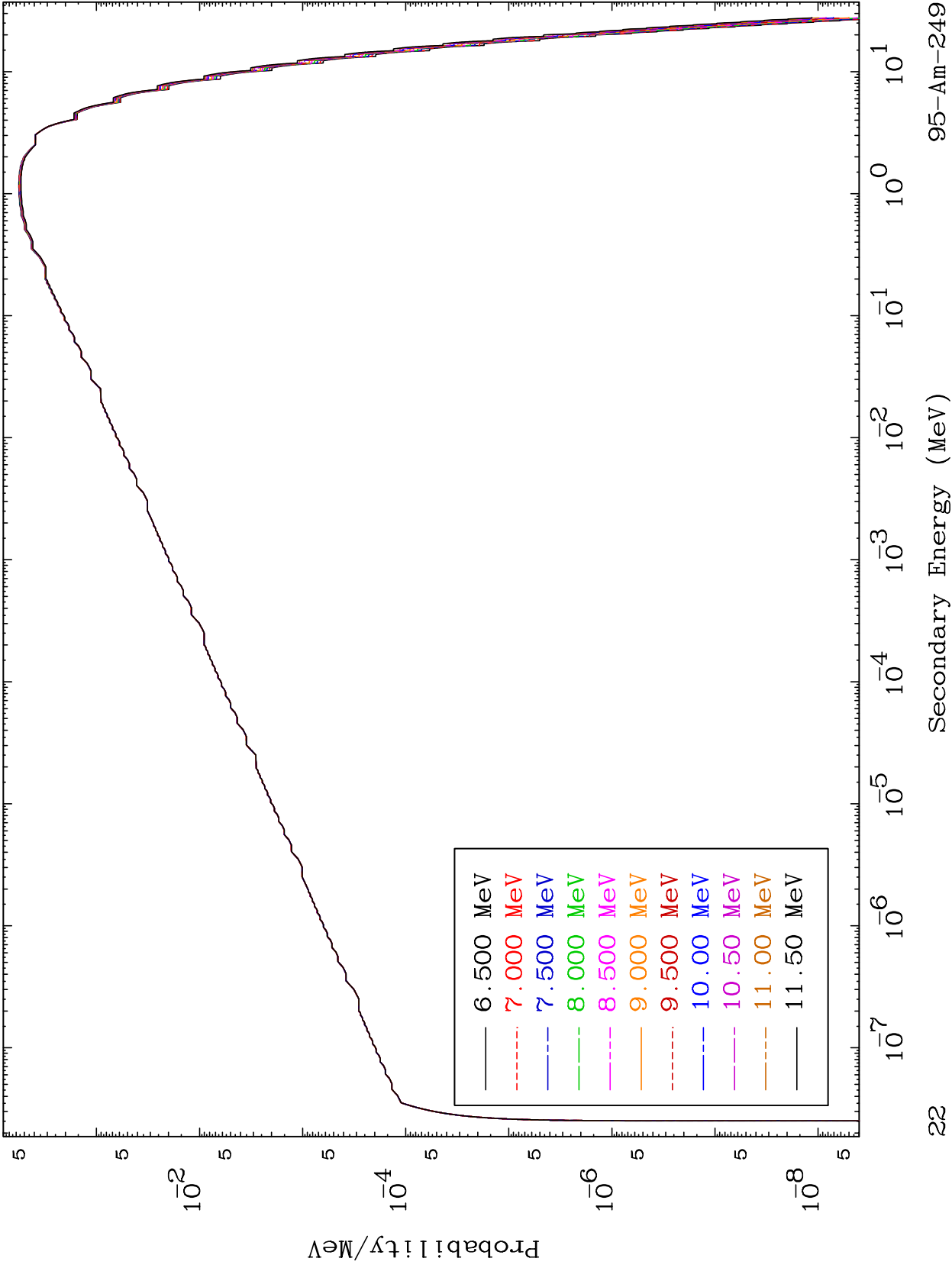


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Fission Energy Distribution

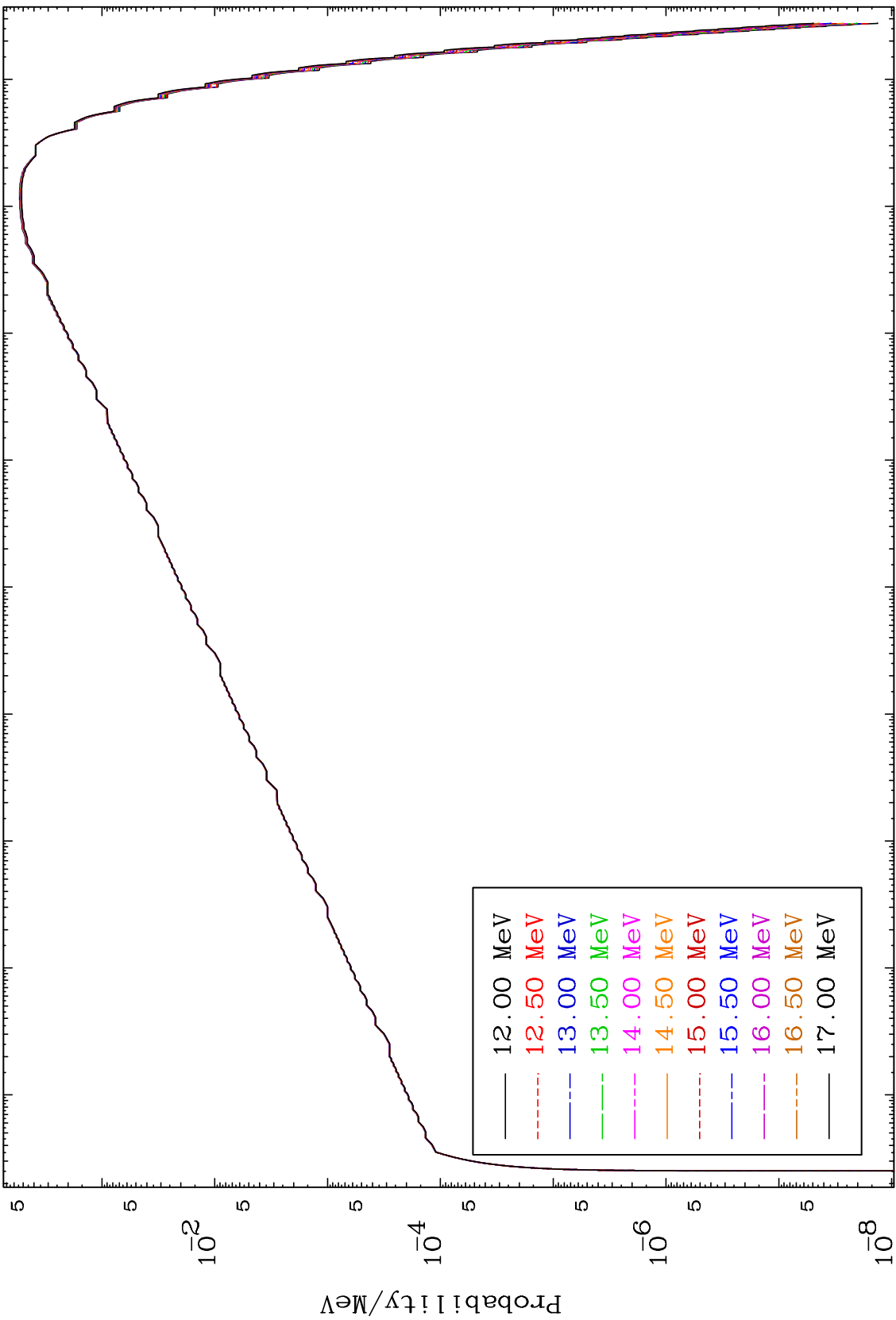
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Fission Energy Distribution

95-Am-249

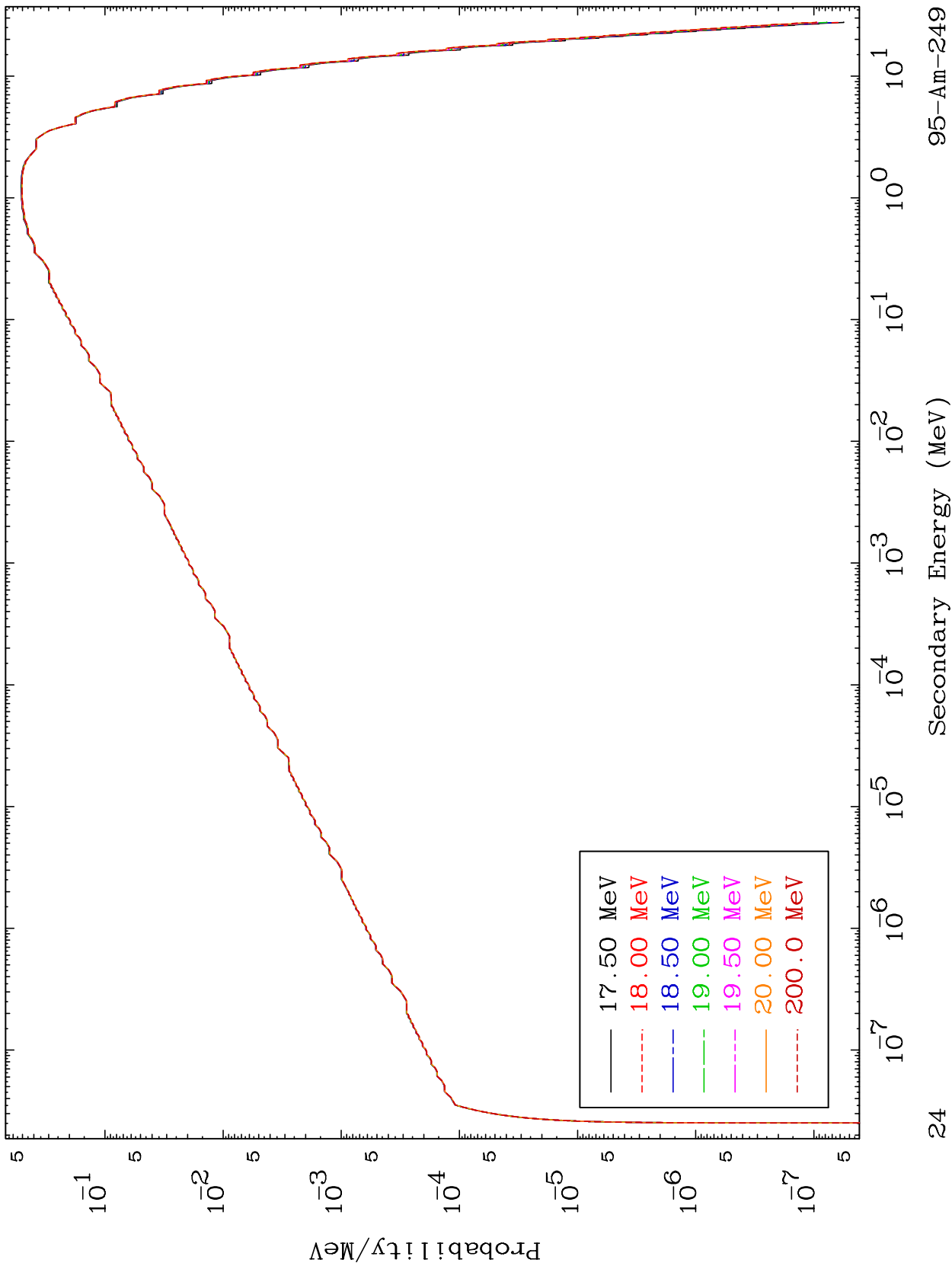


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Fission Energy Distribution

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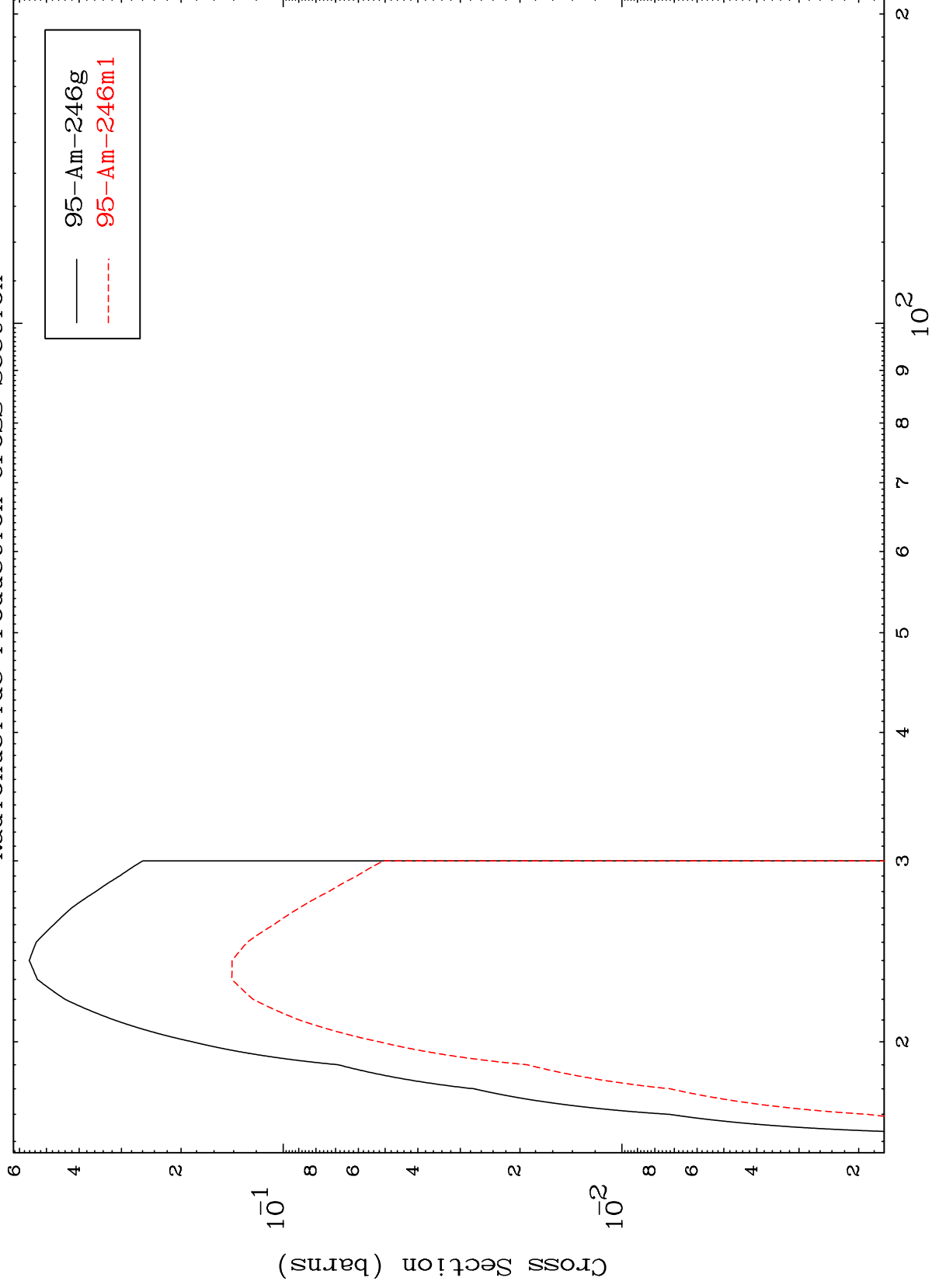


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(n,4n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

95-Am-249