

Program EVALPLOT  
(Version 2017-1)

by

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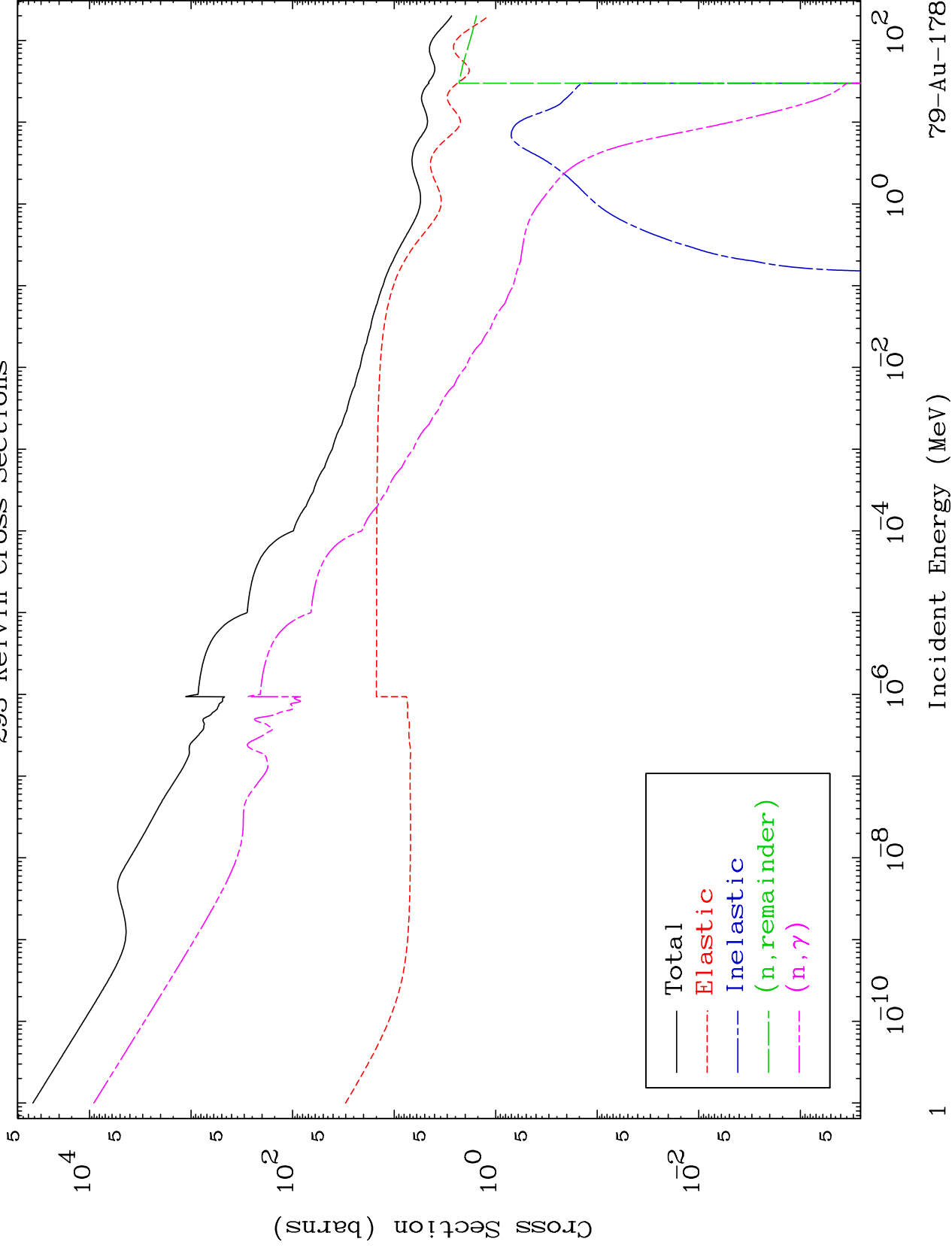
Press Mouse Button to Start

MAT 7868

Major

293 Kelvin Cross Sections

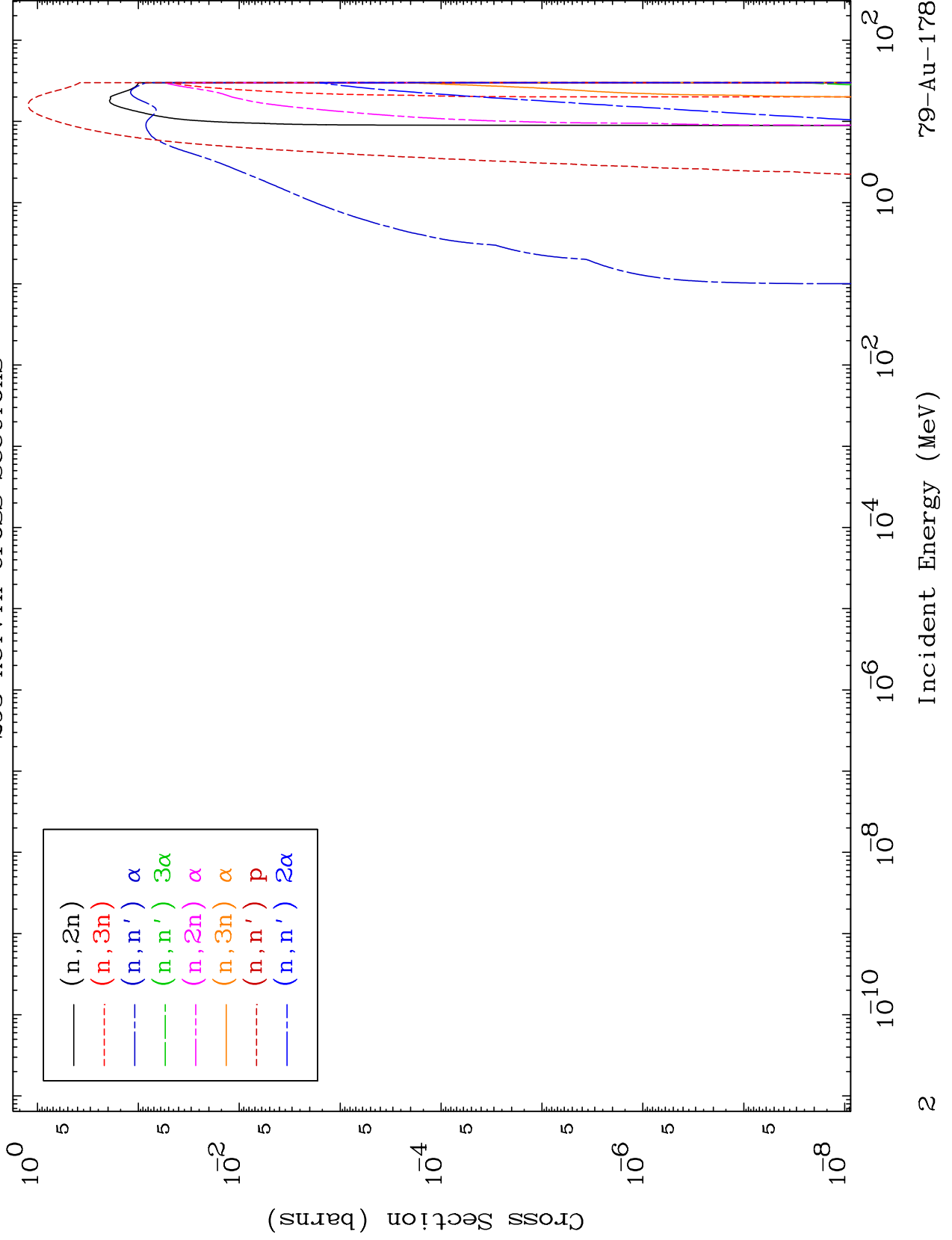
79-Au-178



MAT 7868

Neutron Production  
293 Kelvin Cross Sections

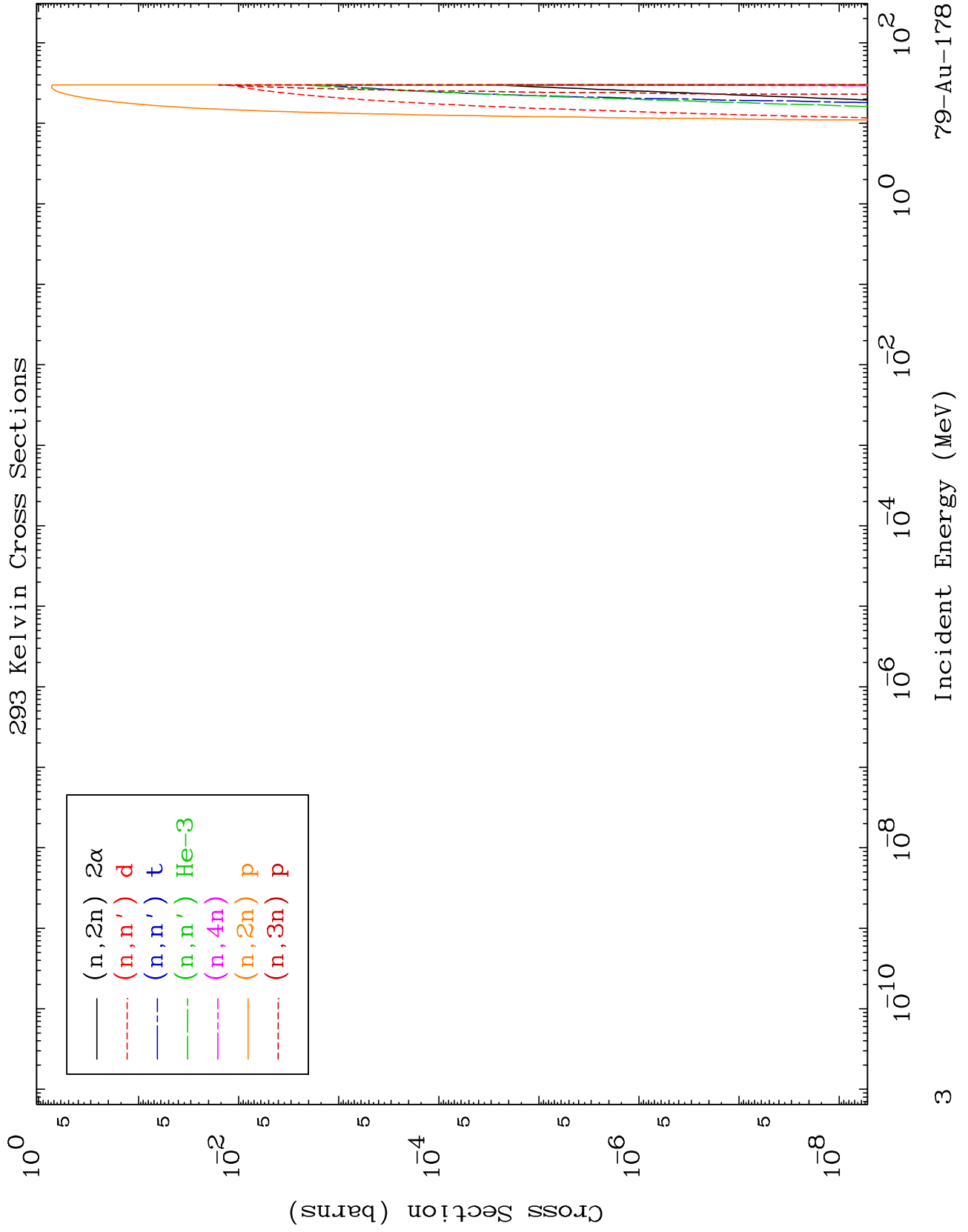
79-Au-178



MAT 7868

Neutron Production  
293 Kelvin Cross Sections

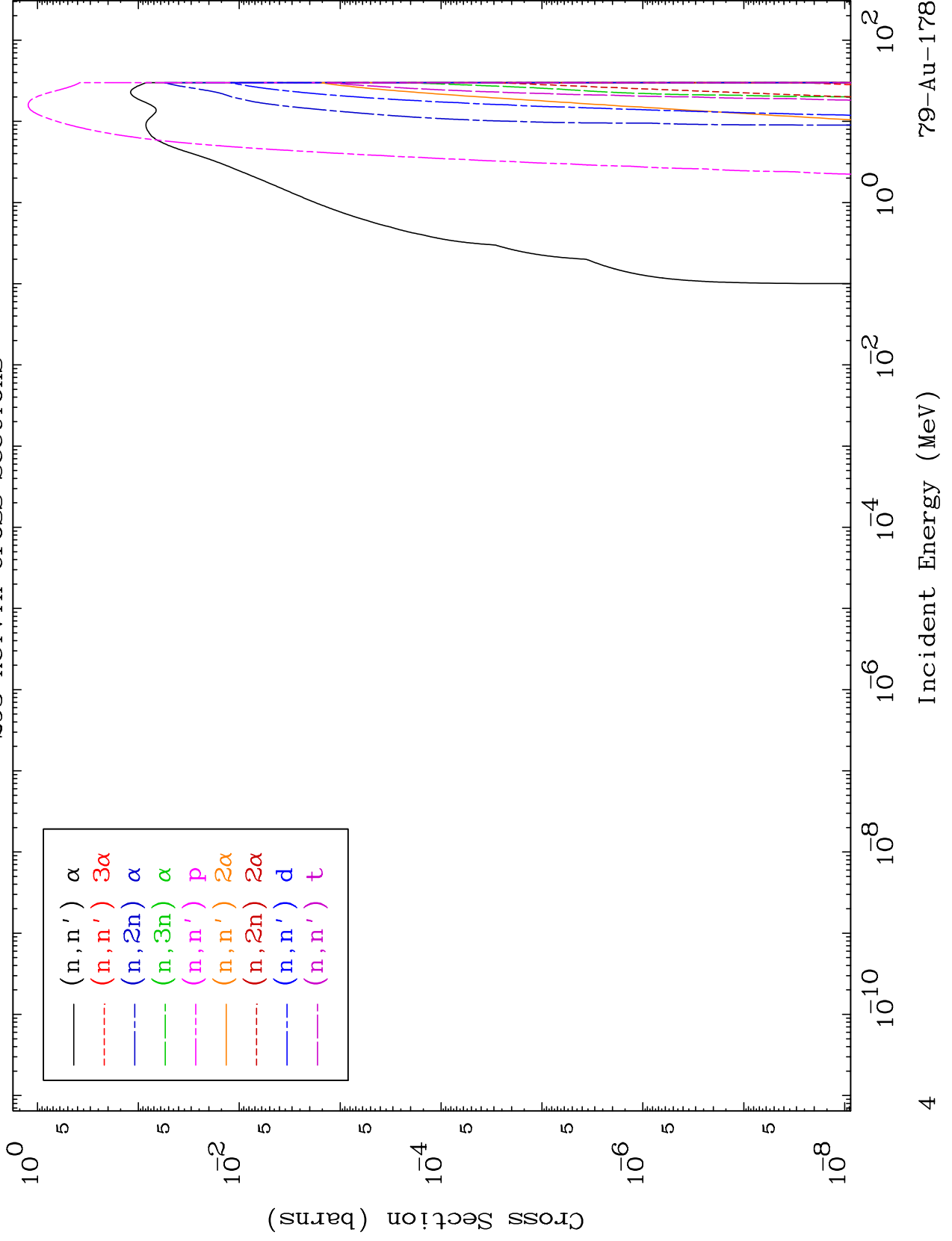
79-Au-178



MAT 7868

Charged Particle  
293 Kelvin Cross Sections

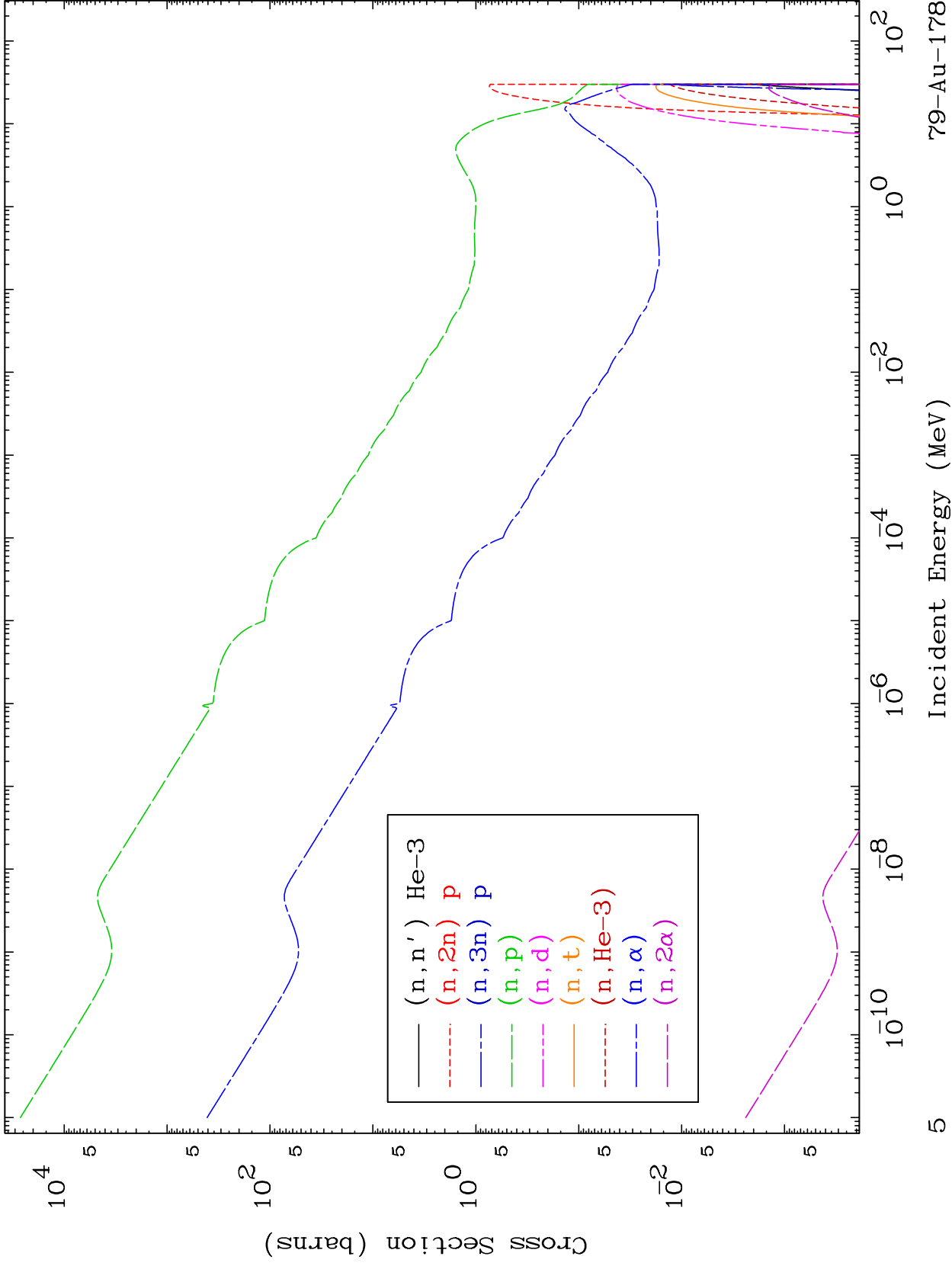
79-Au-178

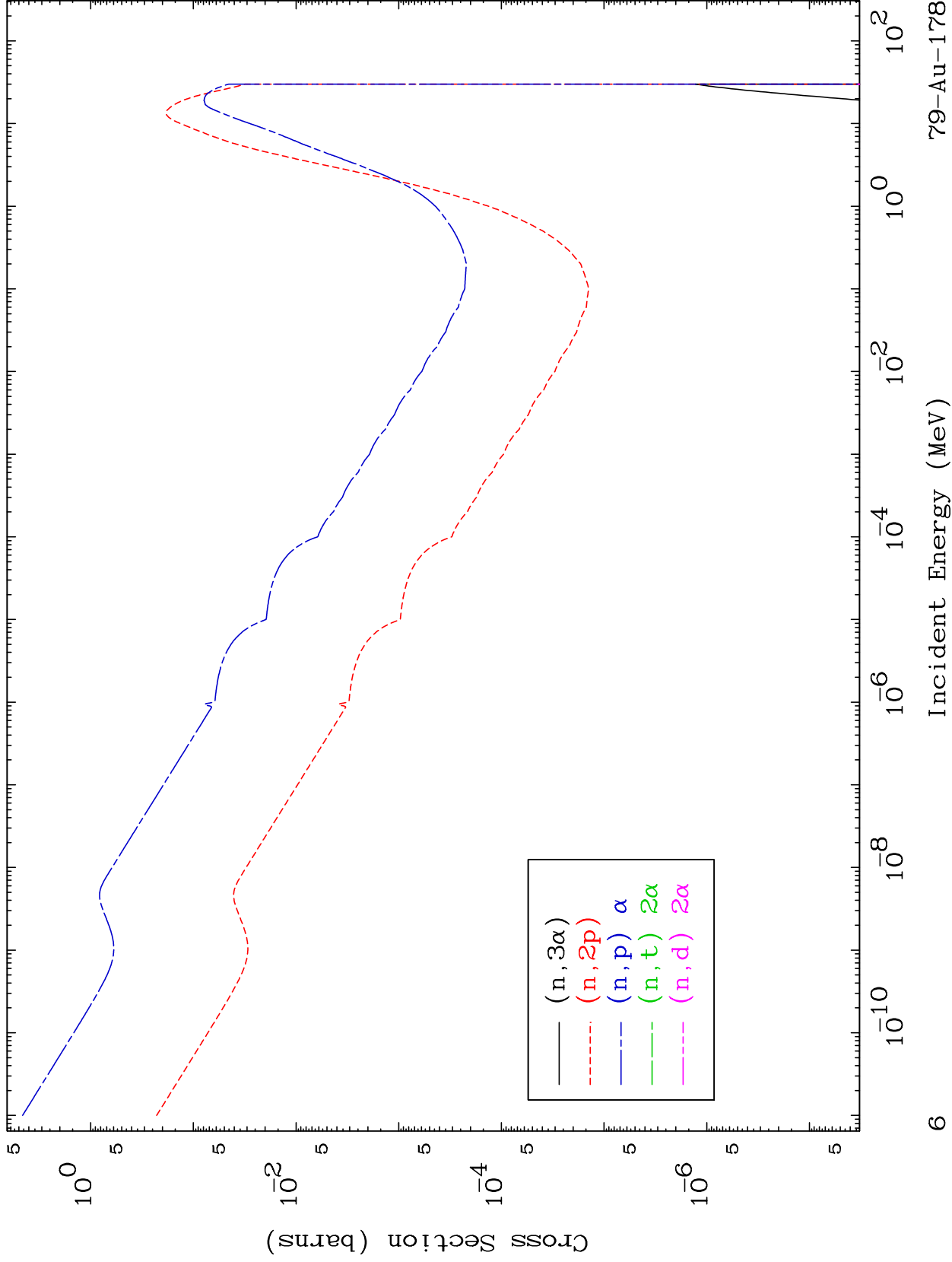


MAT 7868

Charged Particle  
293 Kelvin Cross Sections

79-Au-178

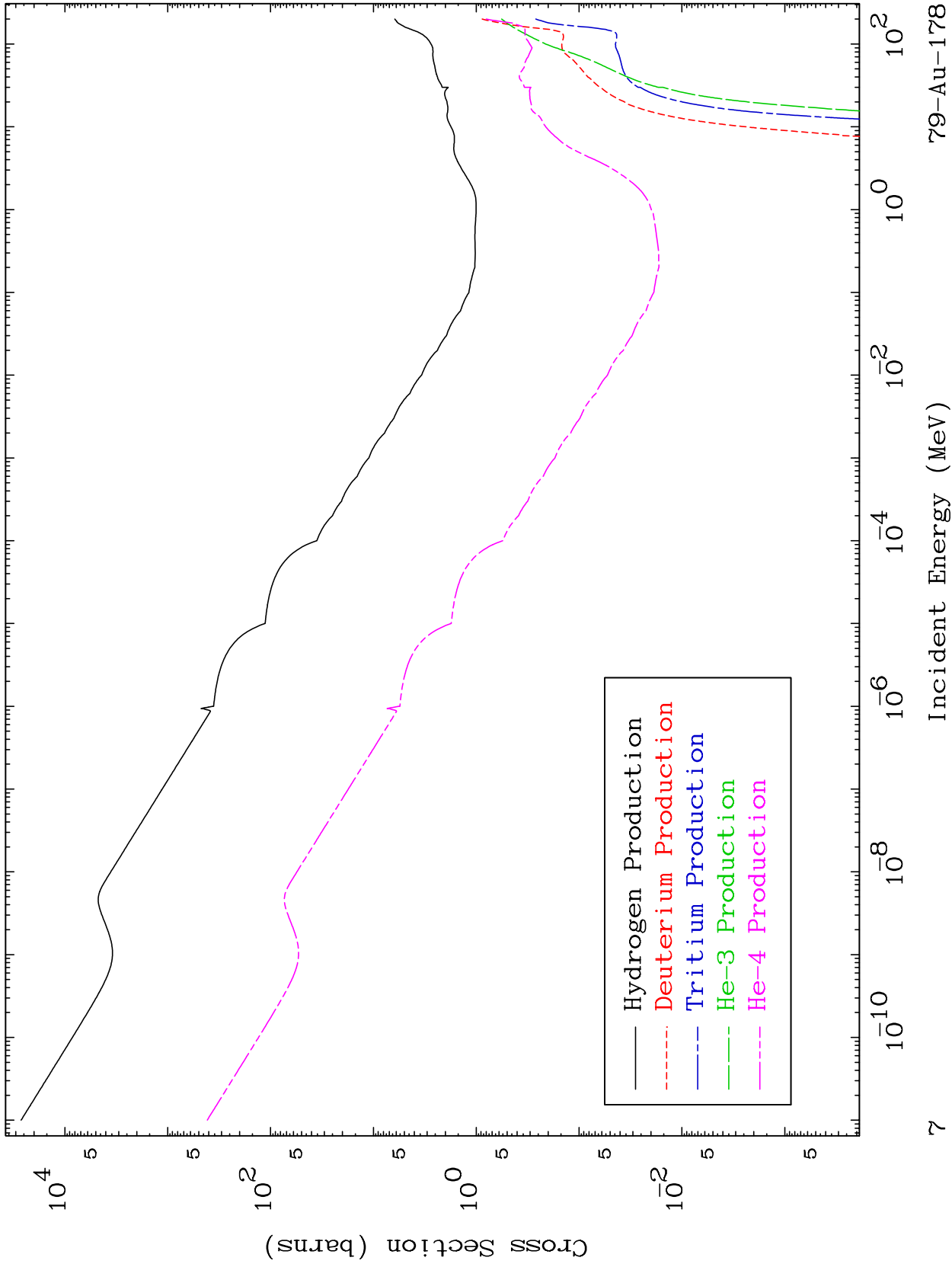




MAT 7868

Particle Production  
293 Kelvin Cross Sections

79-Au-178

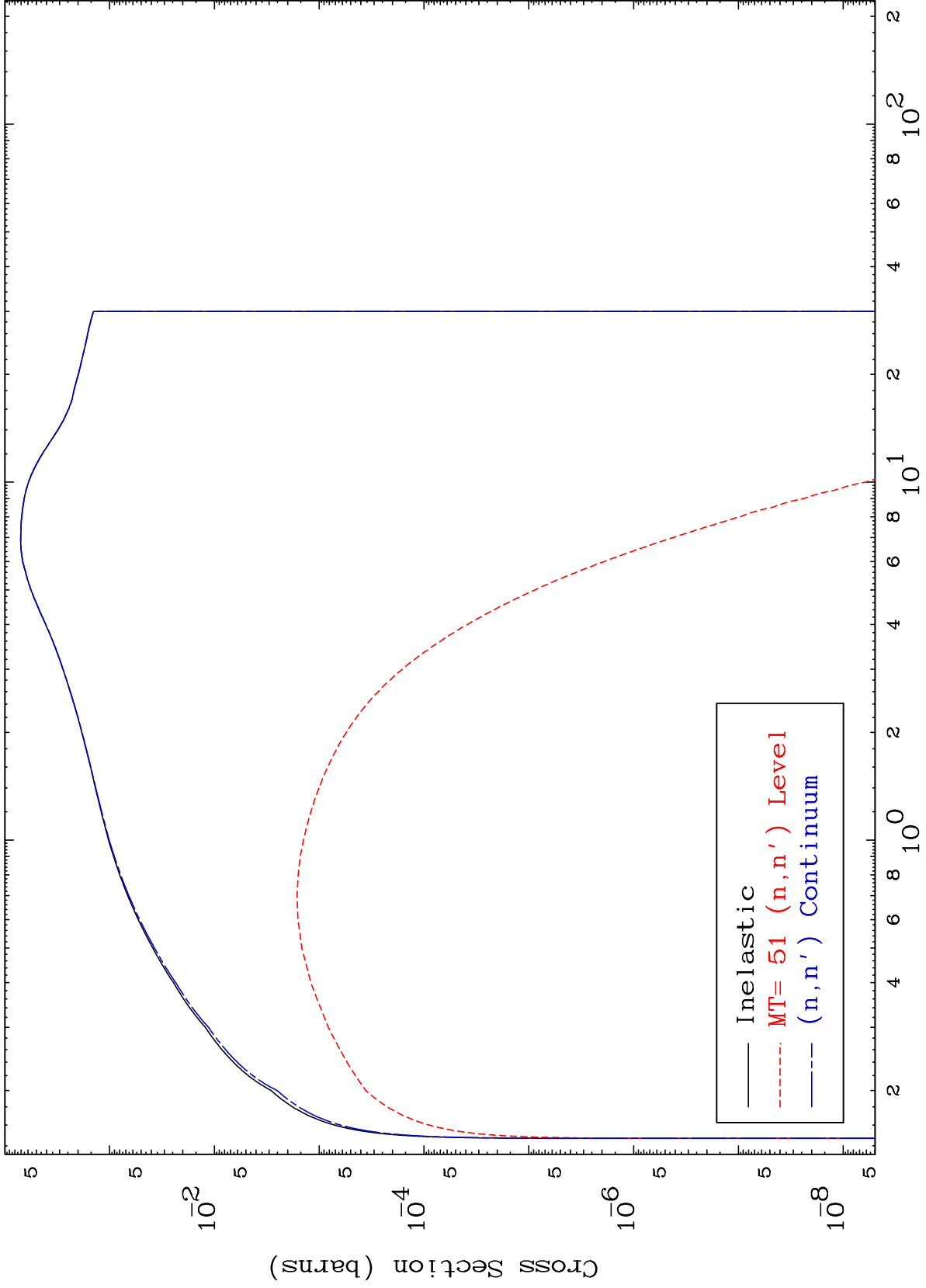




MAT 7868

(n,n') Level  
293 Kelvin Cross Sections

79-Au-178



8

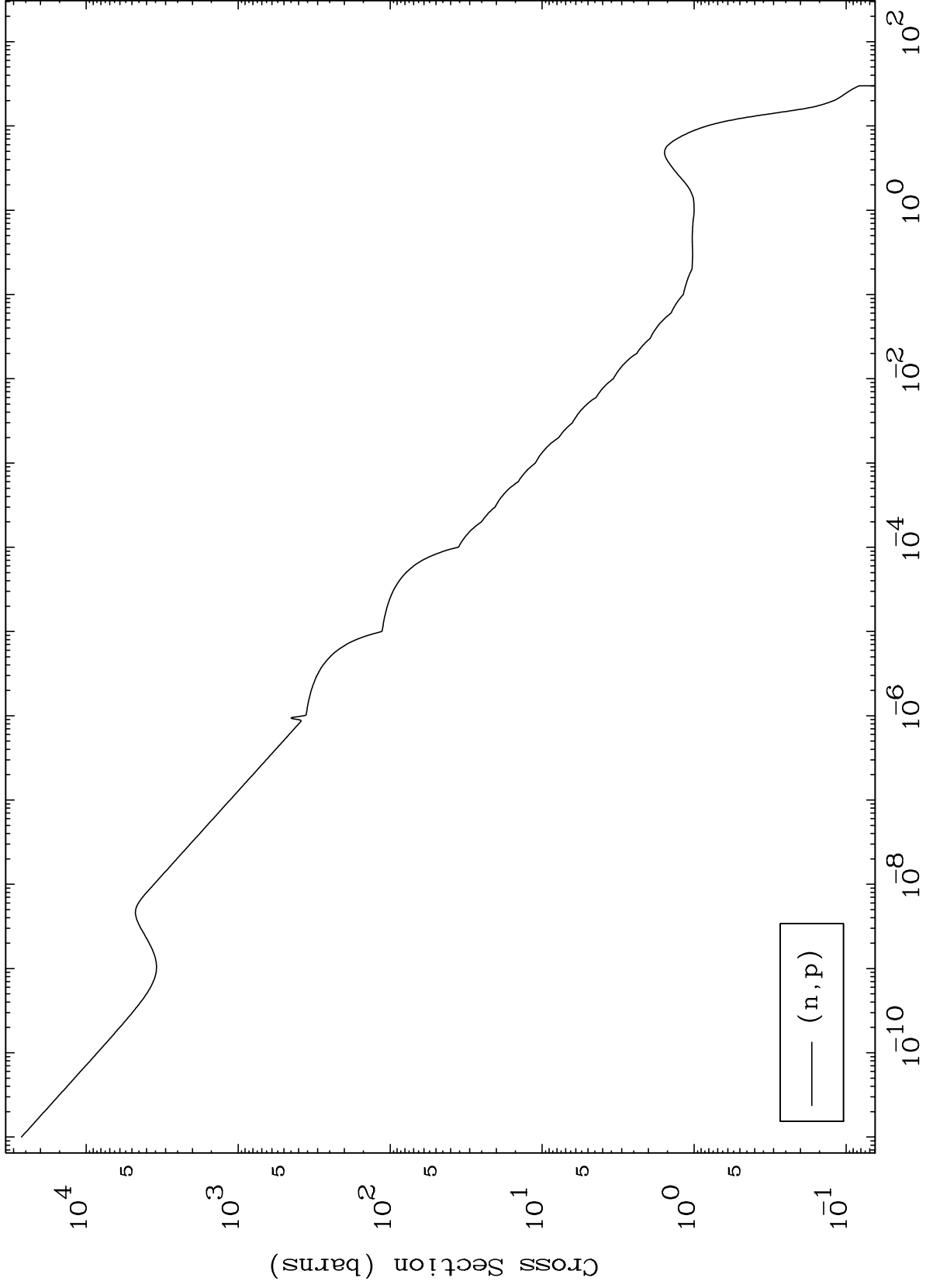
Incident Energy (MeV)

79-Au-178

MAT 7868

(n,p) Levels  
293 Kelvin Cross Sections

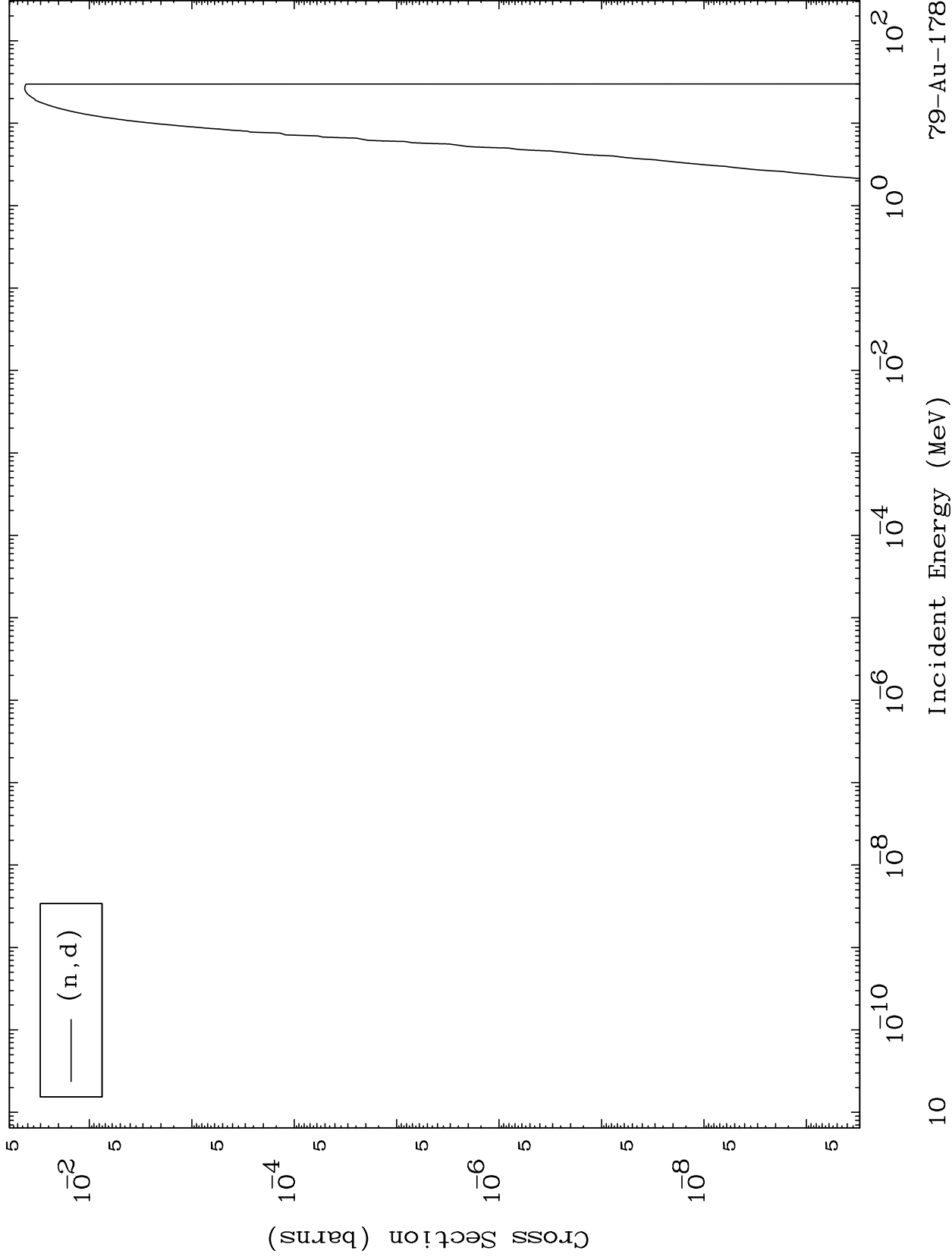
79-Au-178



MAT 7868

(n,d) Levels  
293 Kelvin Cross Sections

79-Au-178



10

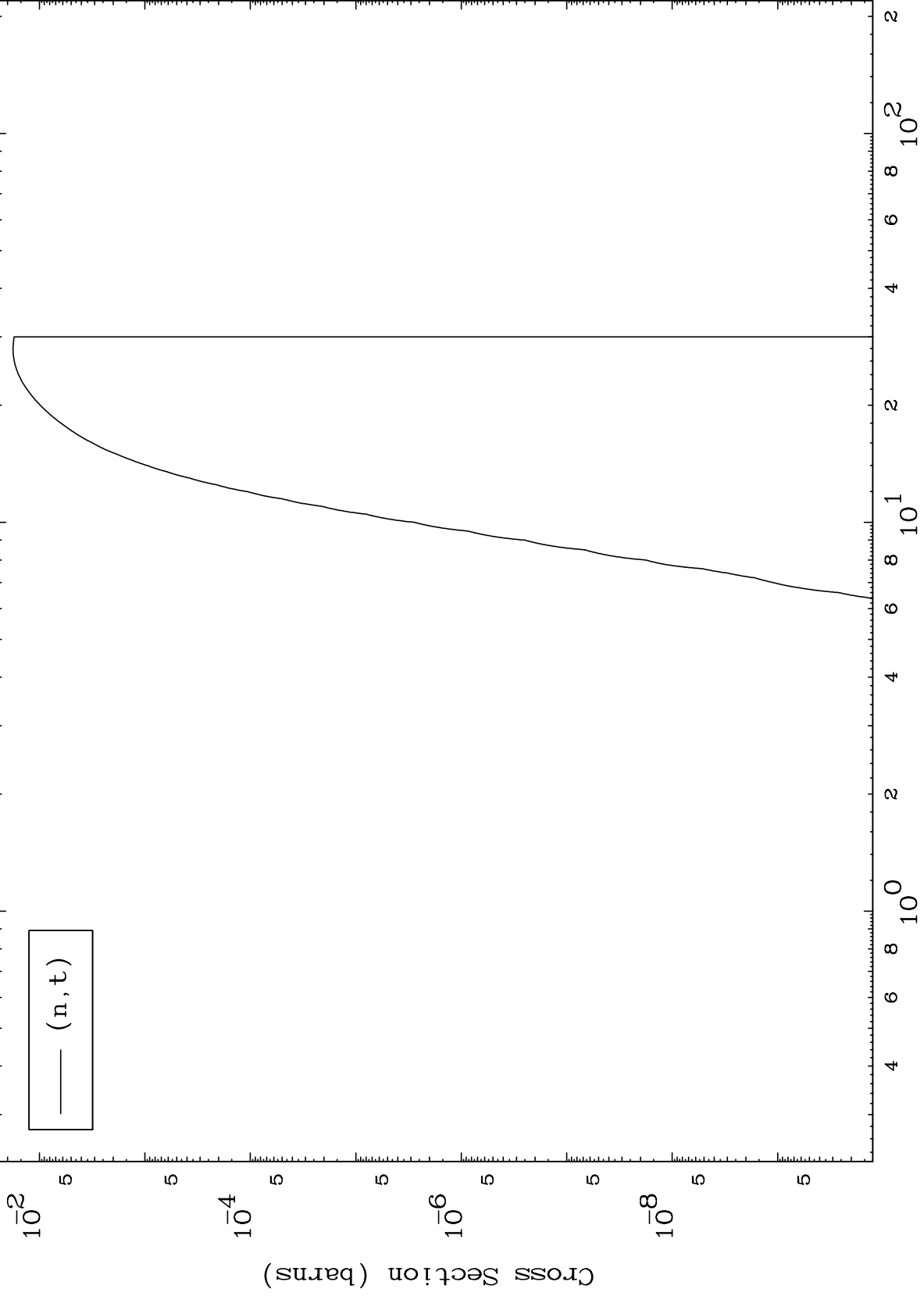
Incident Energy (MeV)

79-Au-178

MAT 7868

(n,t) Levels  
293 Kelvin Cross Sections

79-Au-178

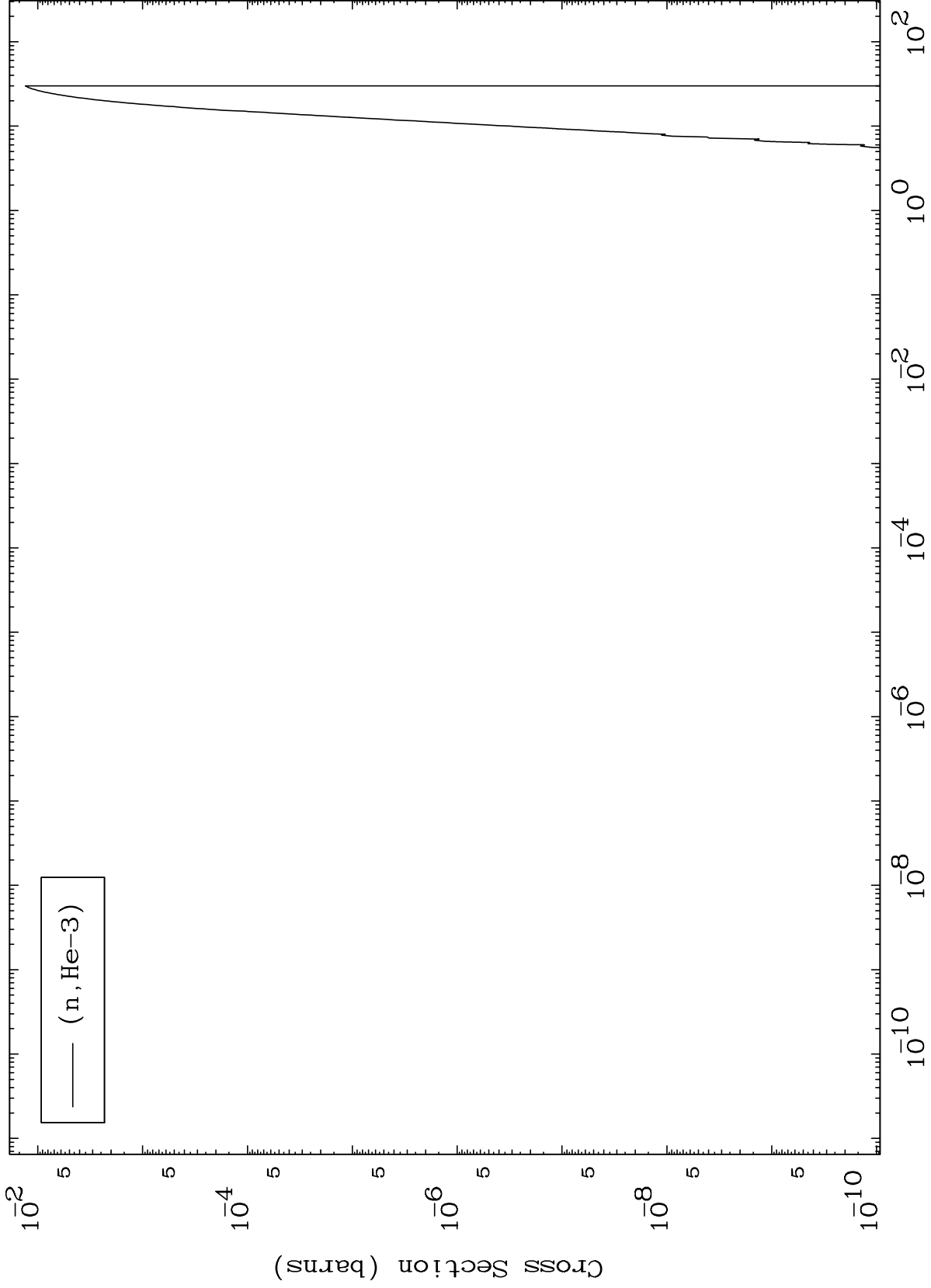


(n,t)

MAT 7868

(n,He3) Levels  
293 Kelvin Cross Sections

79-Au-178



12

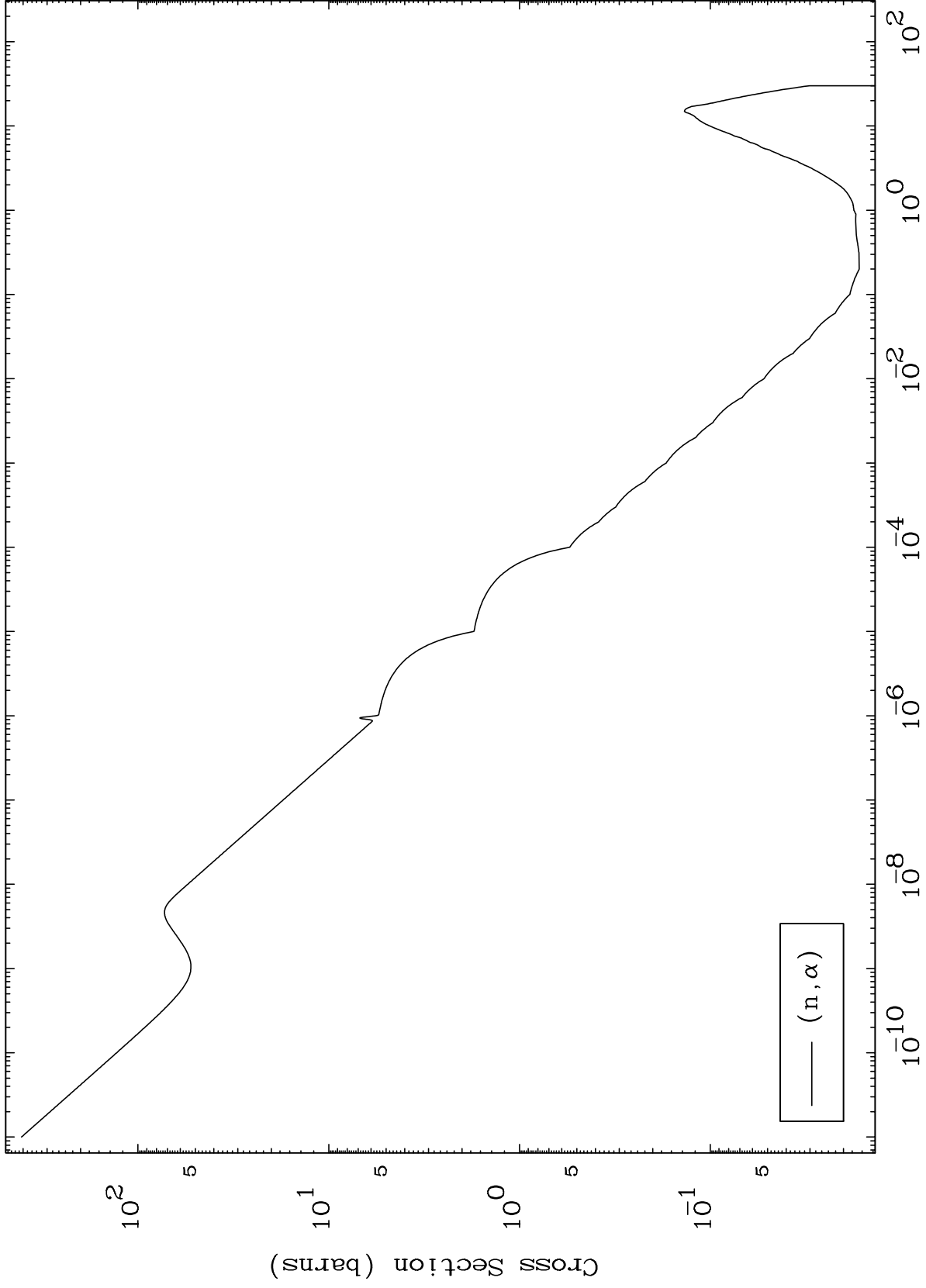
Incident Energy (MeV)

79-Au-178

MAT 7868

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

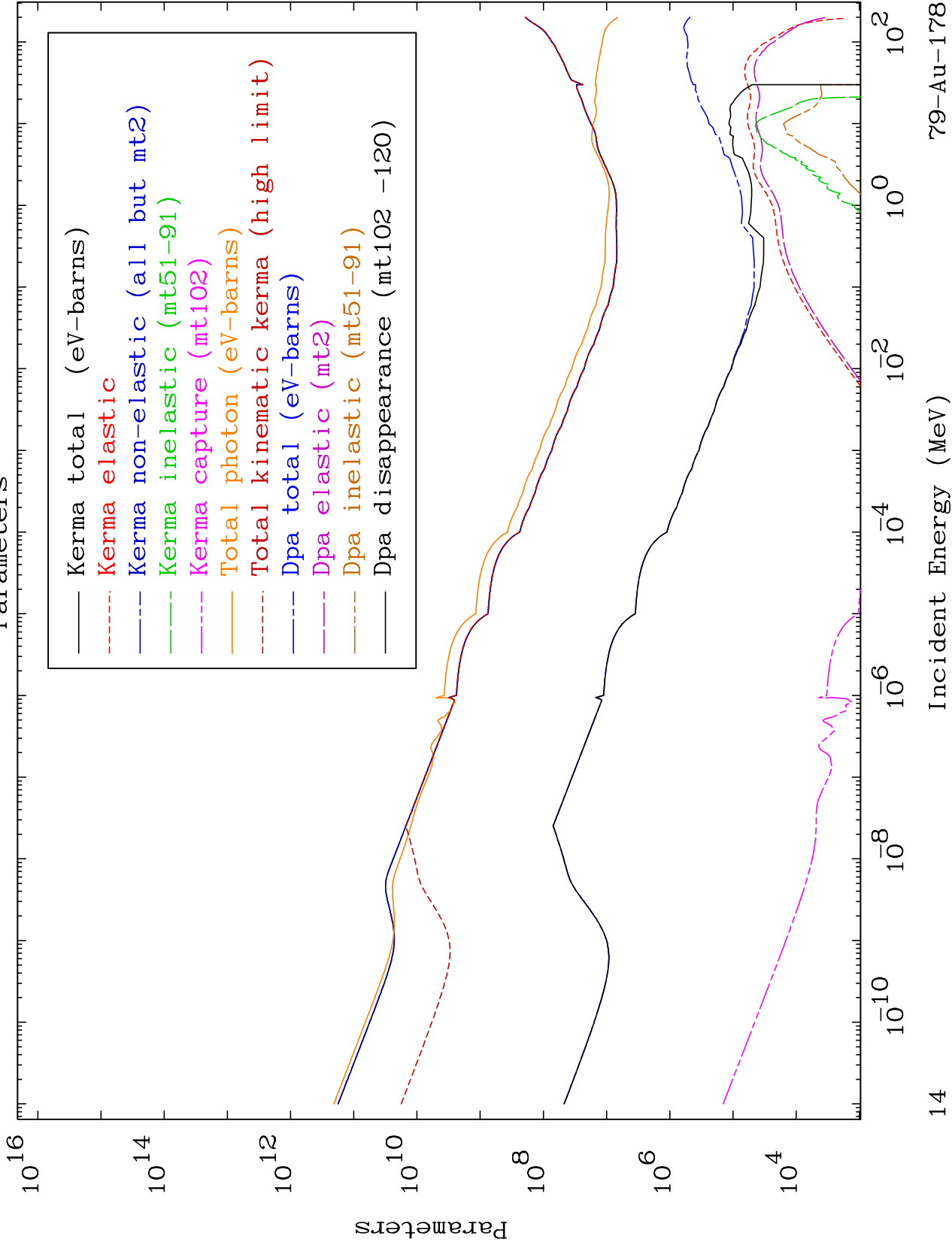
79-Au-178



13

Incident Energy (MeV)

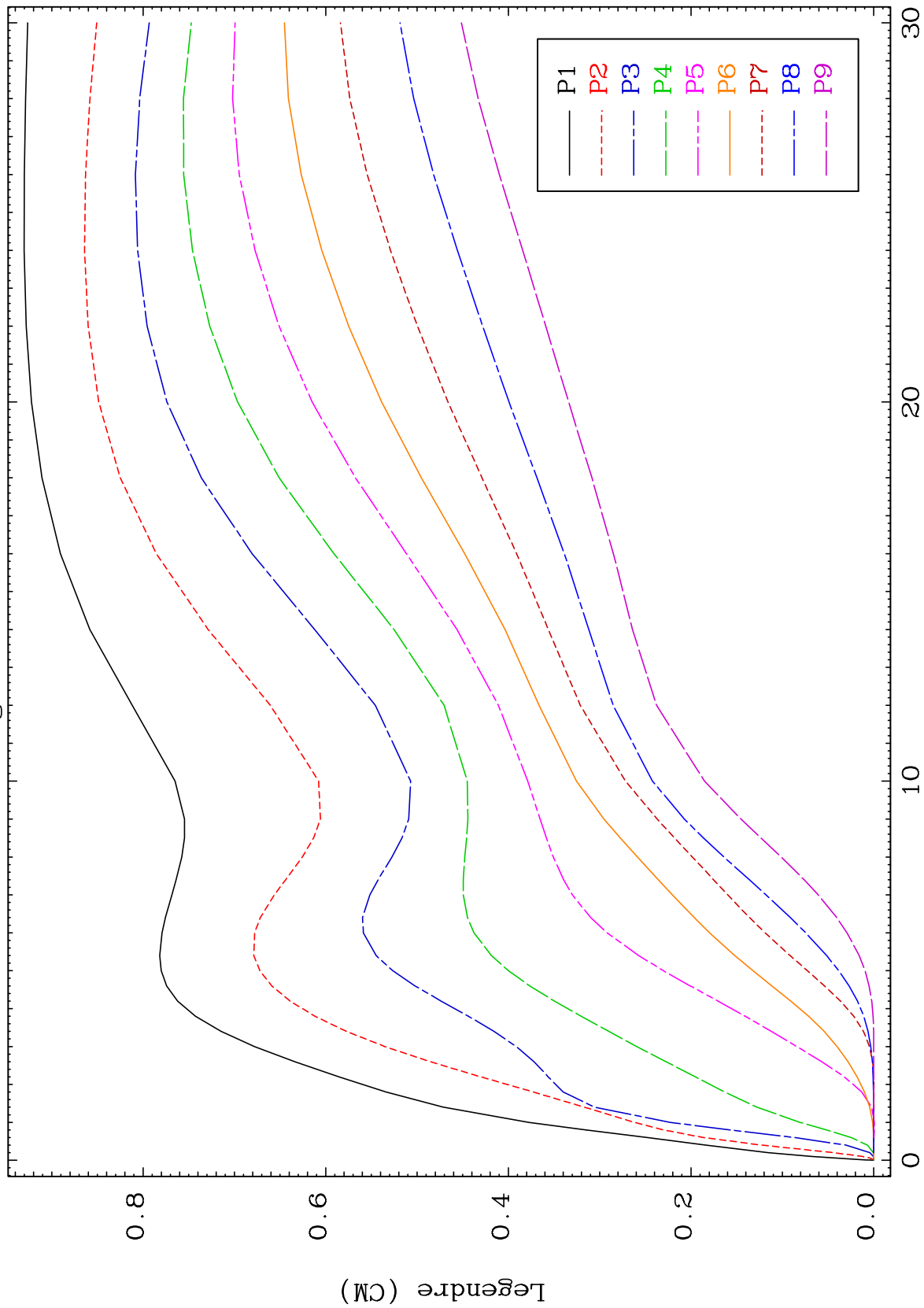
79-Au-178



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Elastic Legendre Coefficients

79-Au-178



15

Incident Energy (MeV)

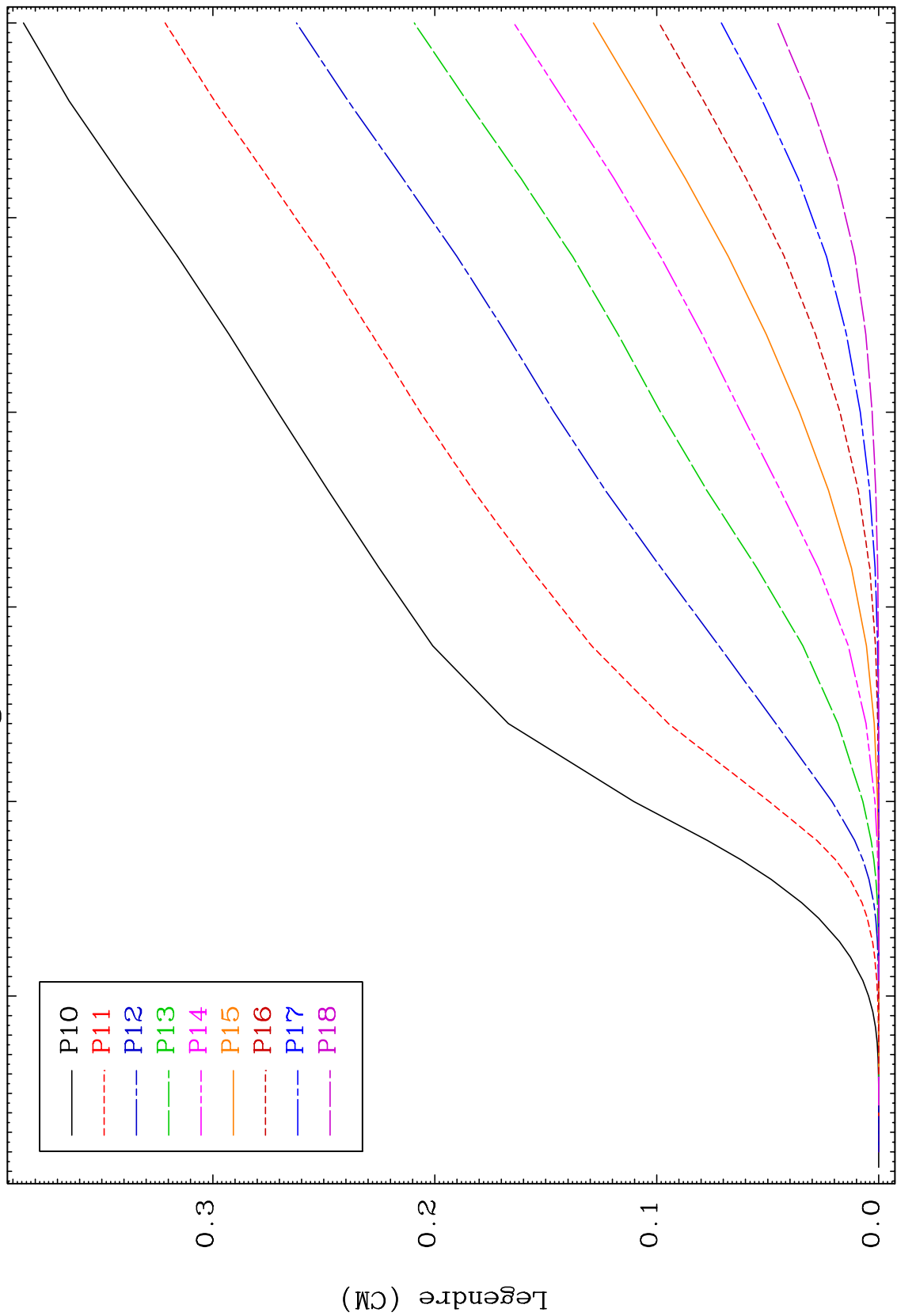
79-Au-178



MAT 7868

Elastic Legendre Coefficients

<sup>79</sup>Au-178



16

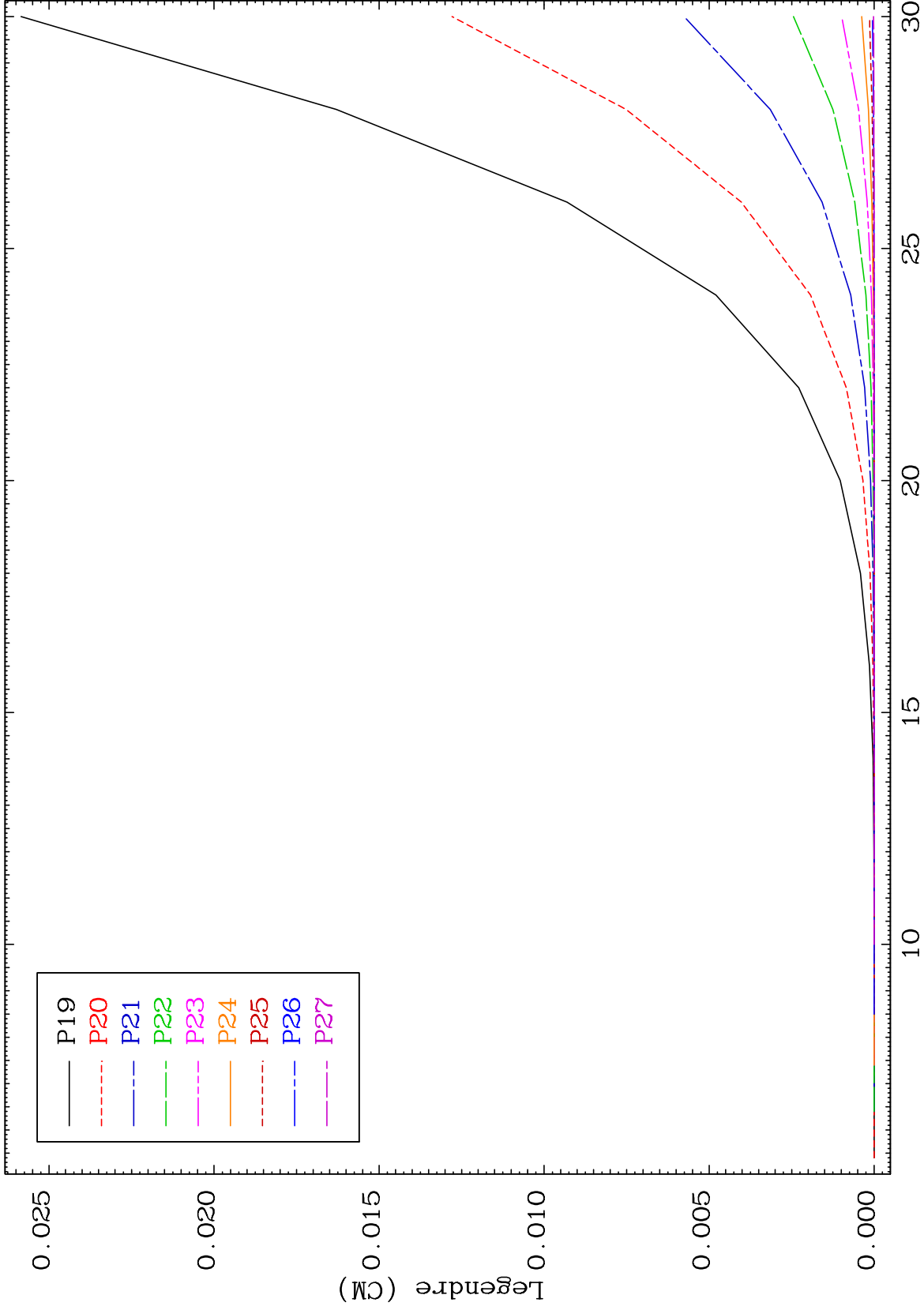
Incident Energy (MeV)

<sup>79</sup>Au-178

MAT 7868

### Elastic Legendre Coefficients

79-Au-178



17

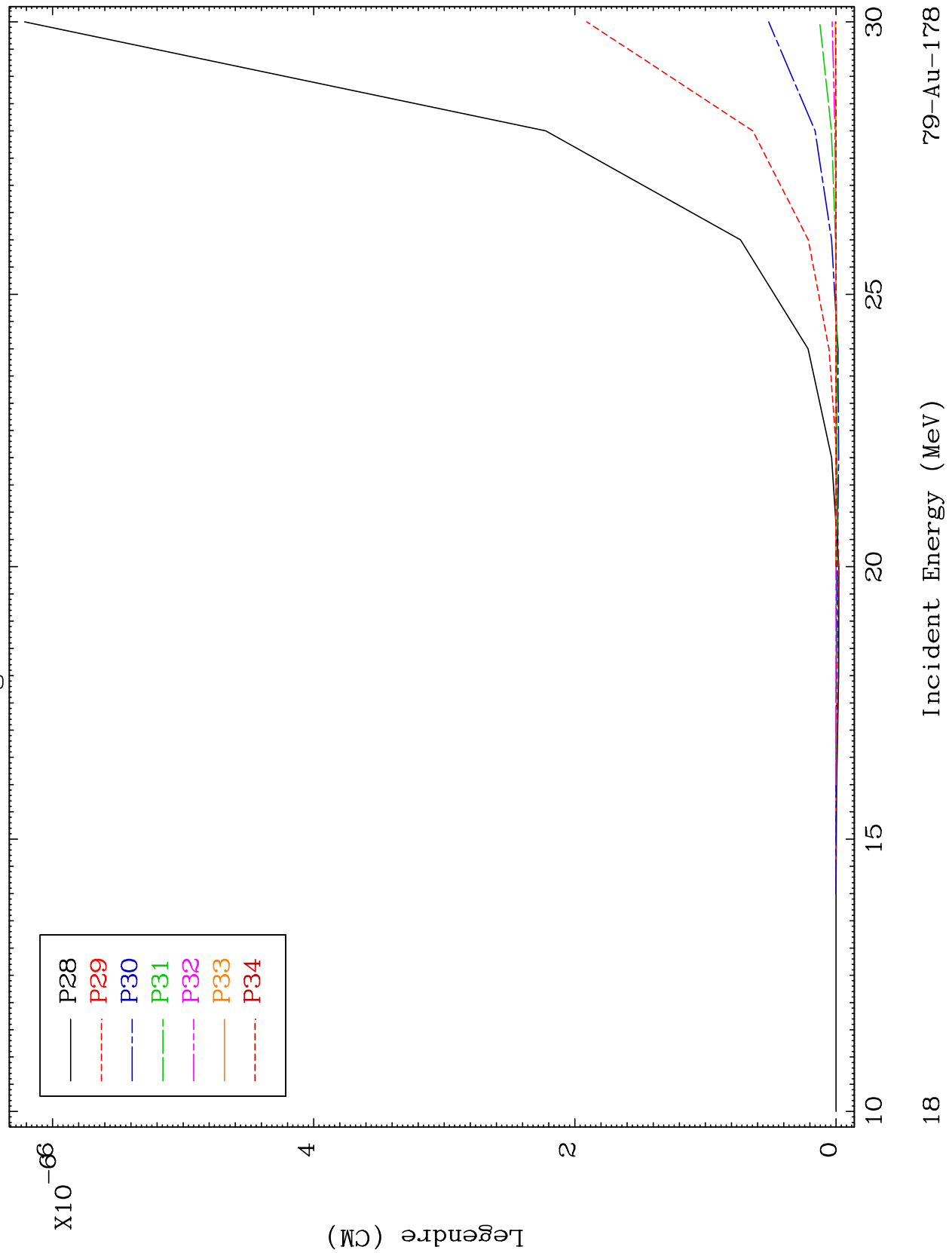
Incident Energy (MeV)

79-Au-178

MAT 7868

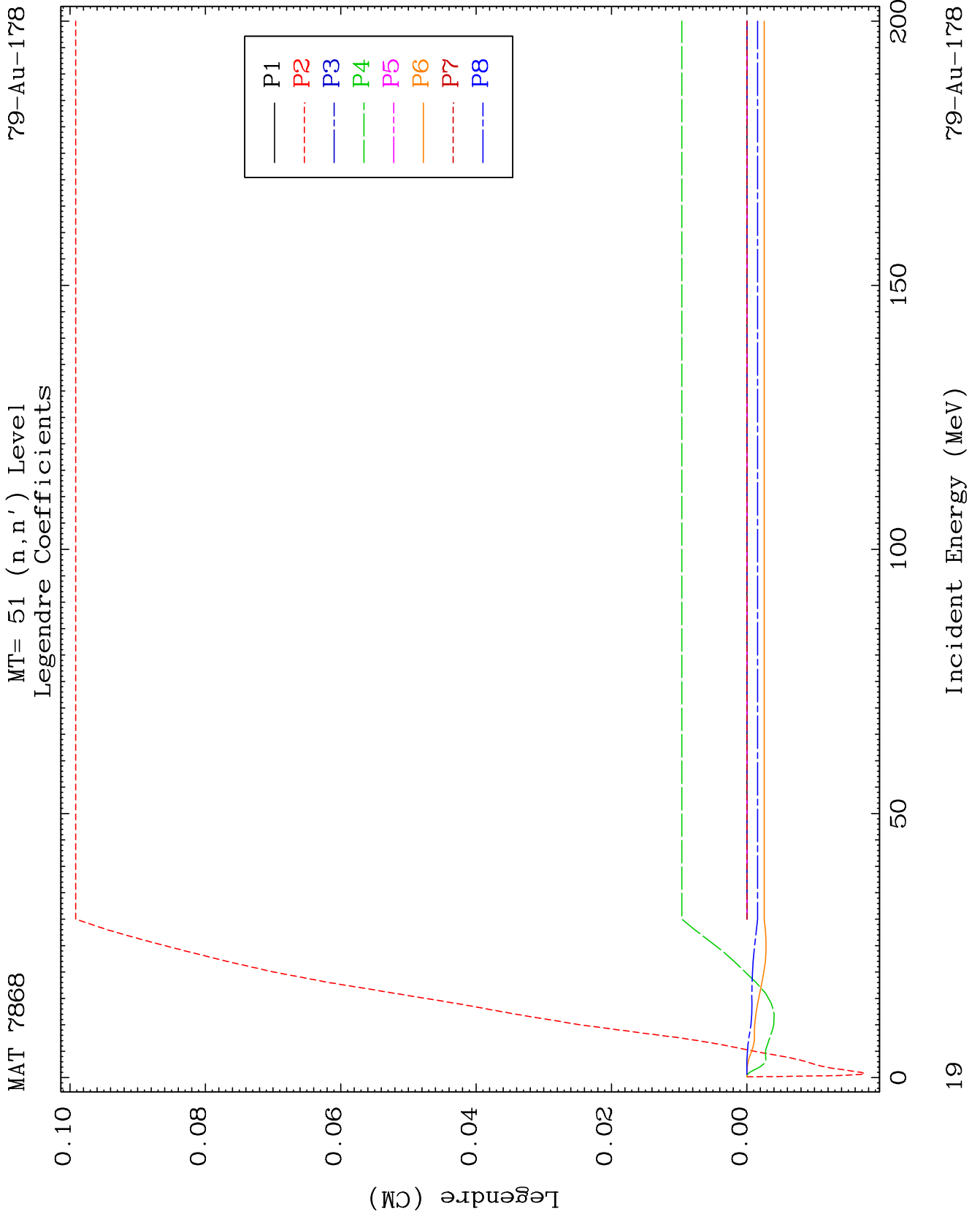
Elastic Legendre Coefficients

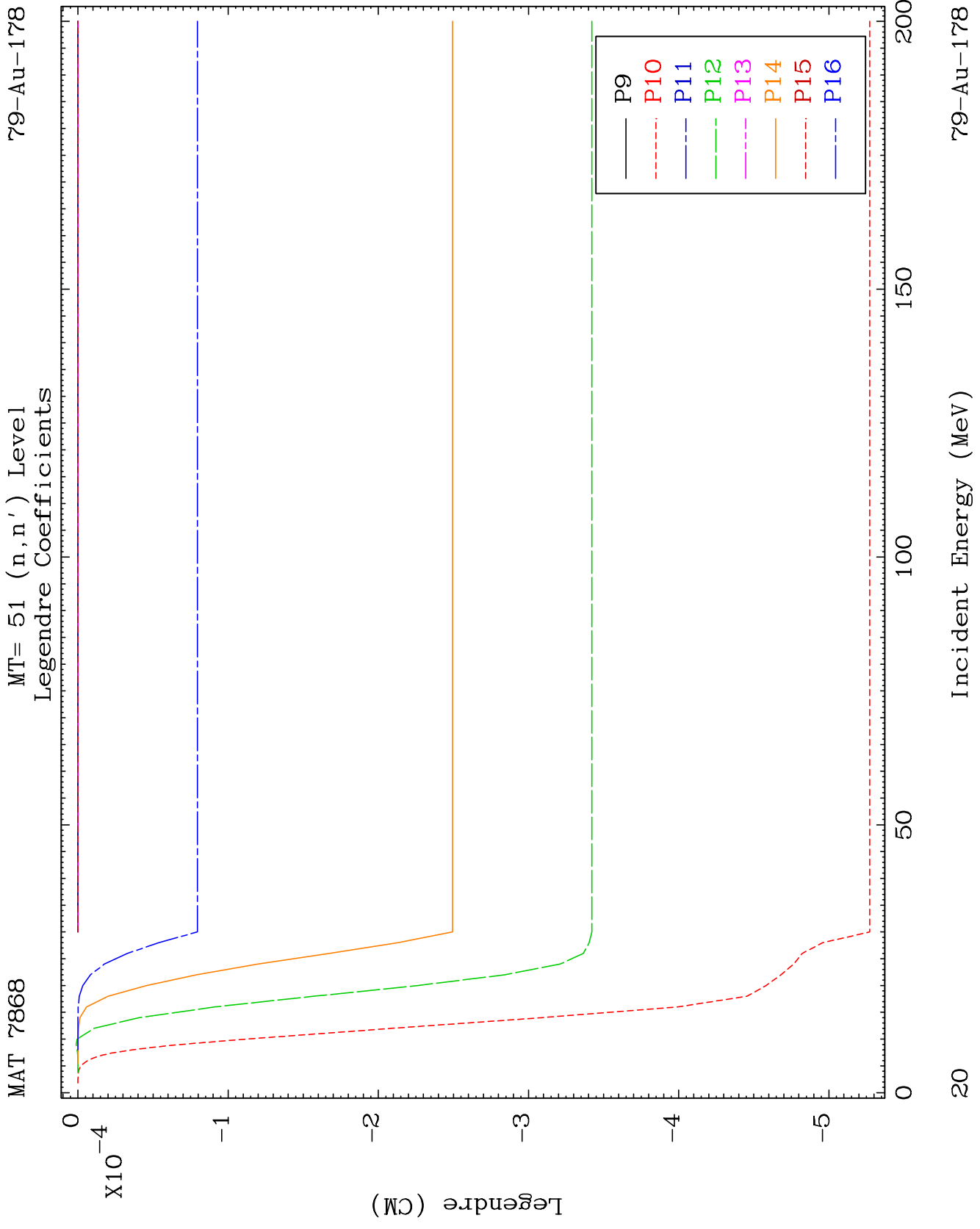
79-Au-178



79-Au-178

Incident Energy (MeV)

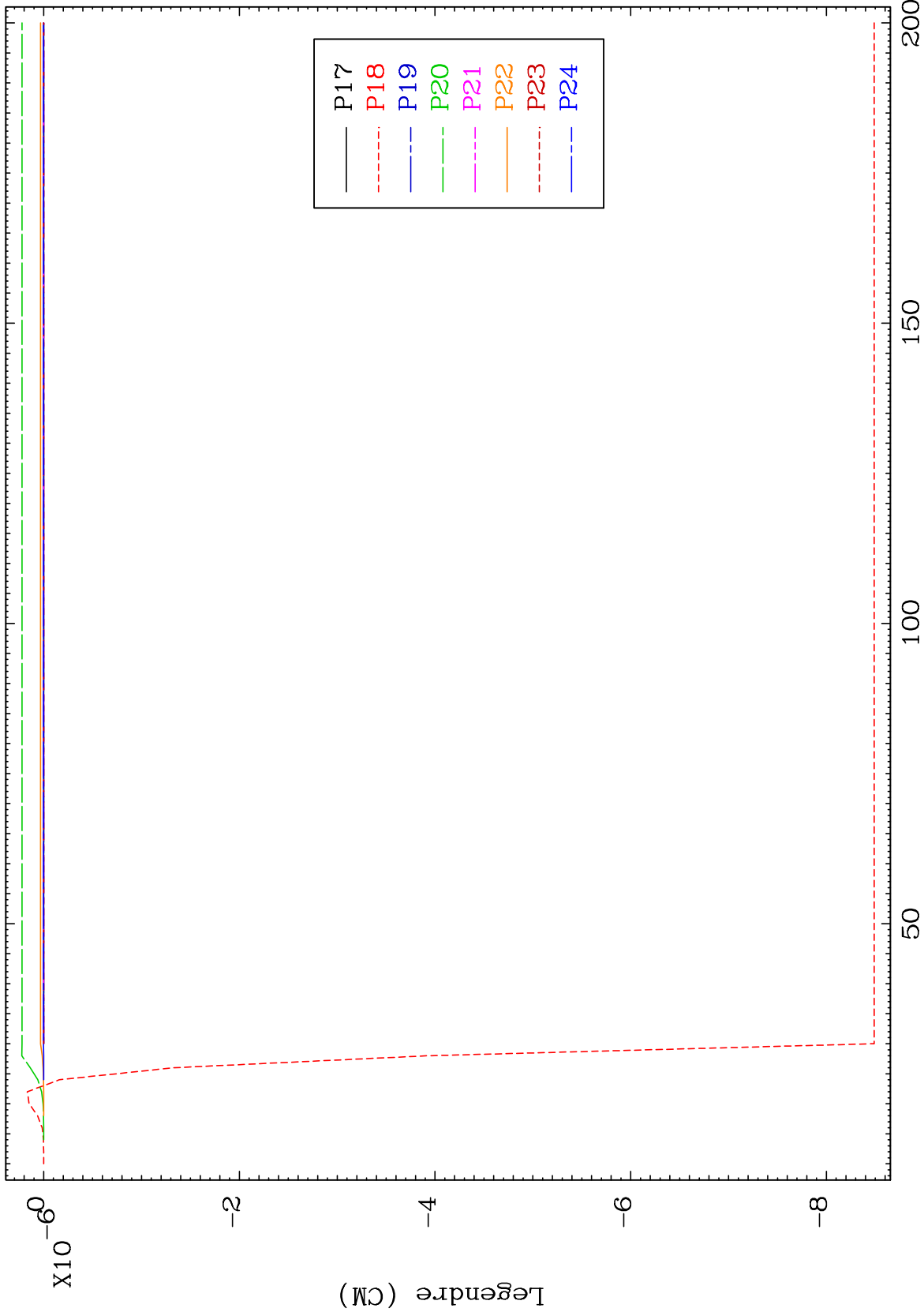




MAT 7868

MT= 51 (n,n') Level  
Legendre Coefficients

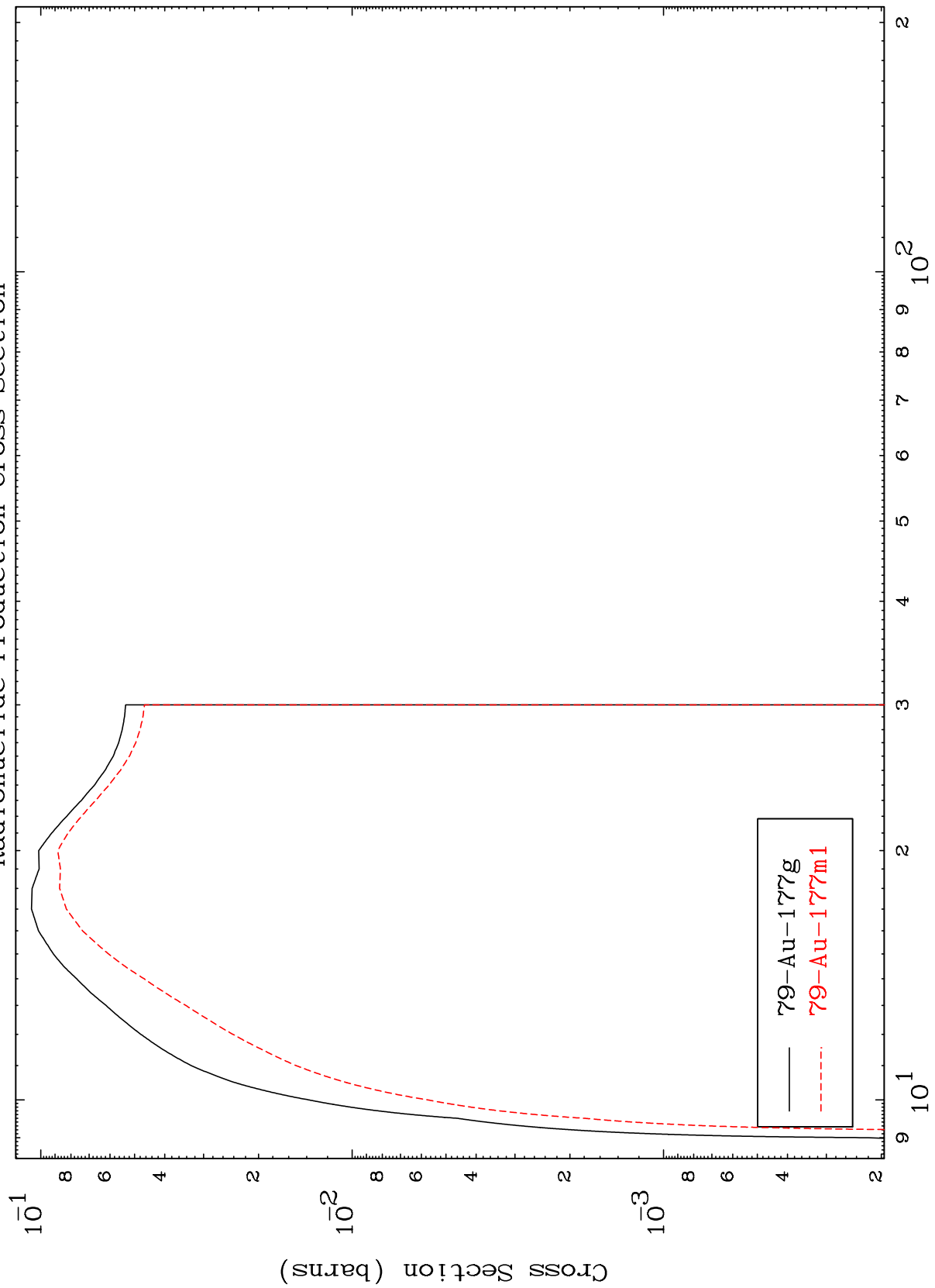
79-Au-178



MAT 7868

79-Au-178

(n,2n)  
Radionuclide Production Cross Section



79-Au-178

Incident Energy (MeV)

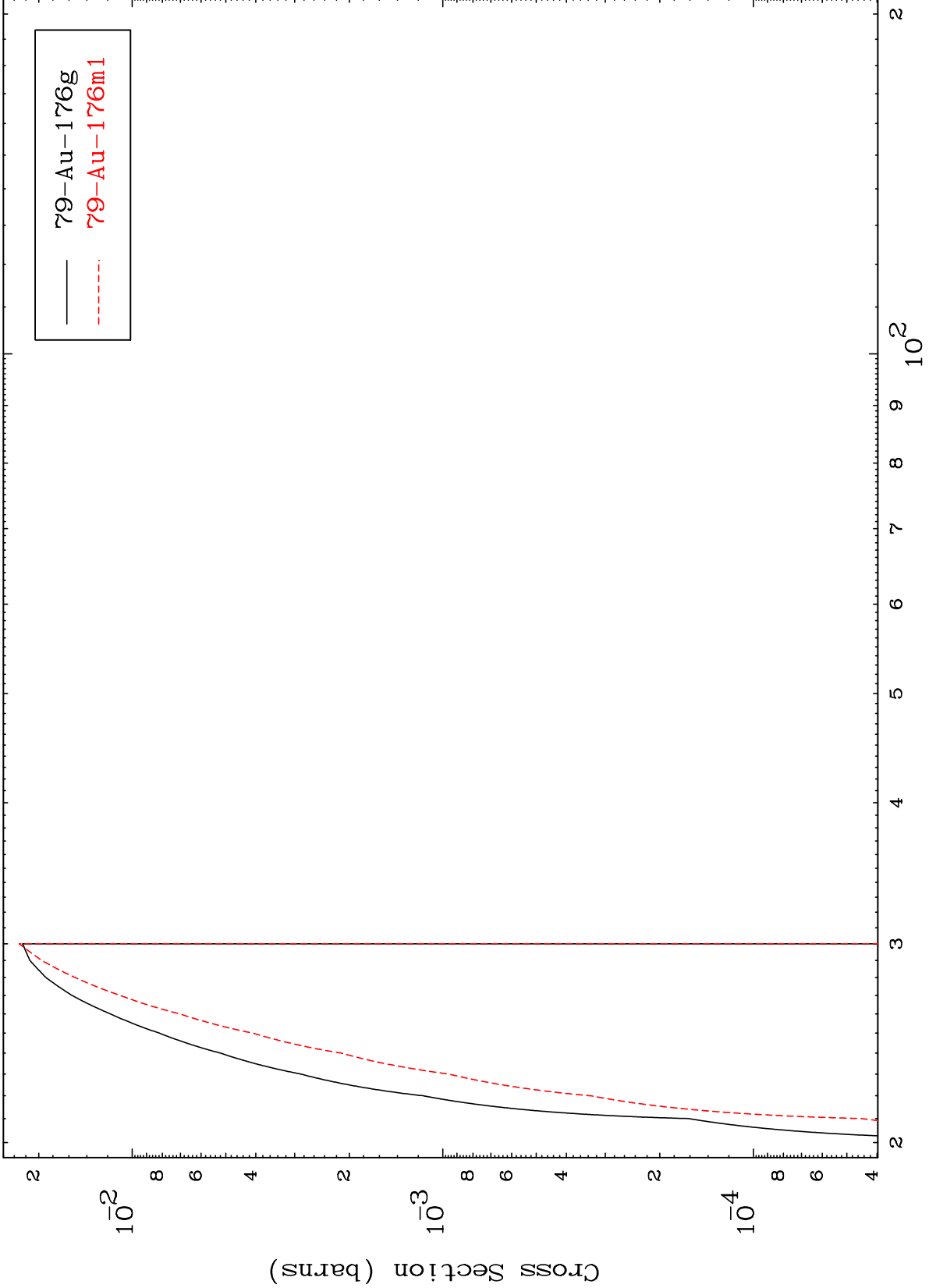
22

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(n,3n)

<sup>79</sup>Au-178

Radionuclide Production Cross Section



— <sup>79</sup>Au-176g  
- - - <sup>79</sup>Au-176m1

23

Incident Energy (MeV)

<sup>79</sup>Au-178

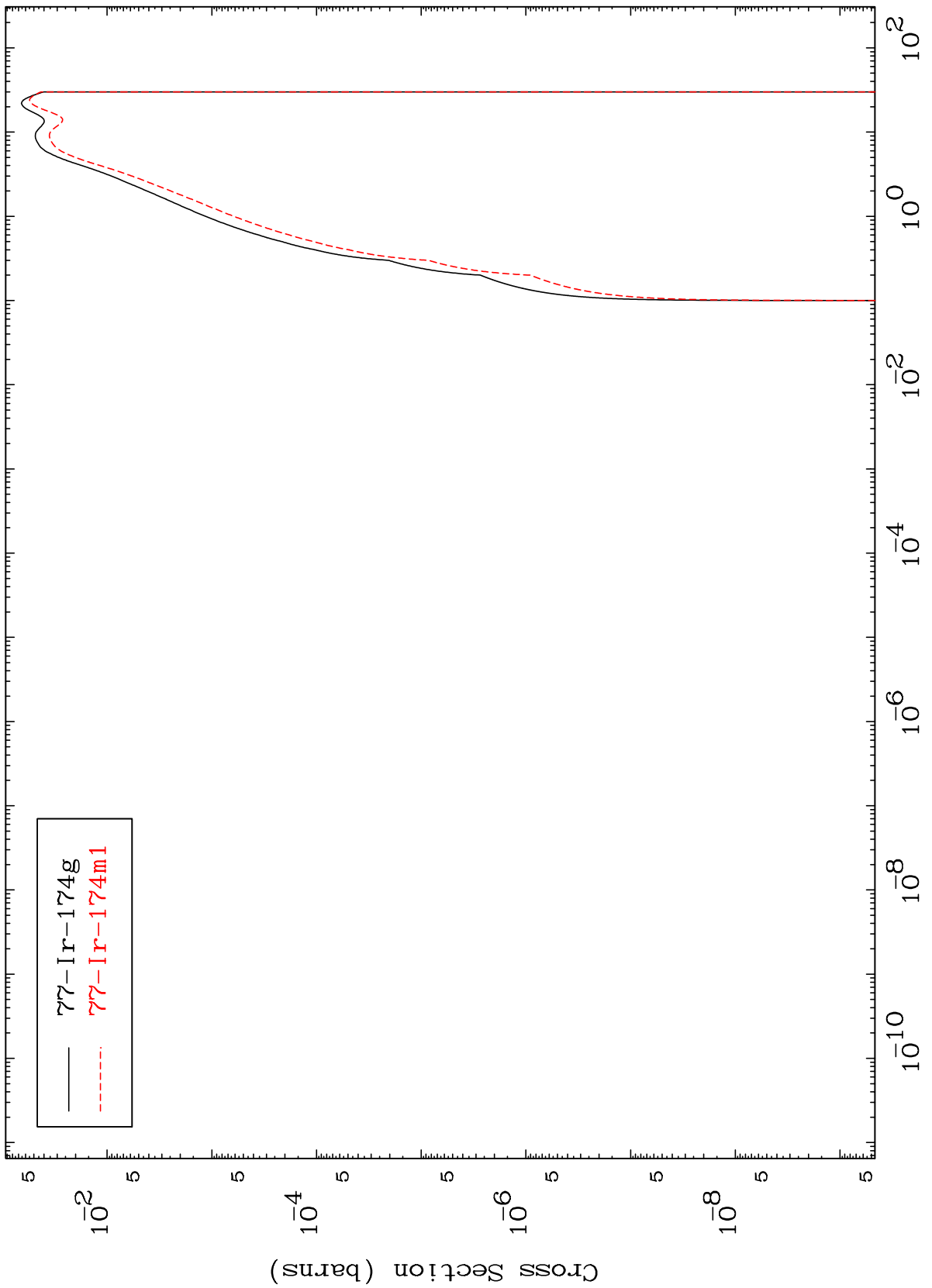


MAT 7868

$(n, n')$   $\alpha$

79-Au-178

Radionuclide Production Cross Section



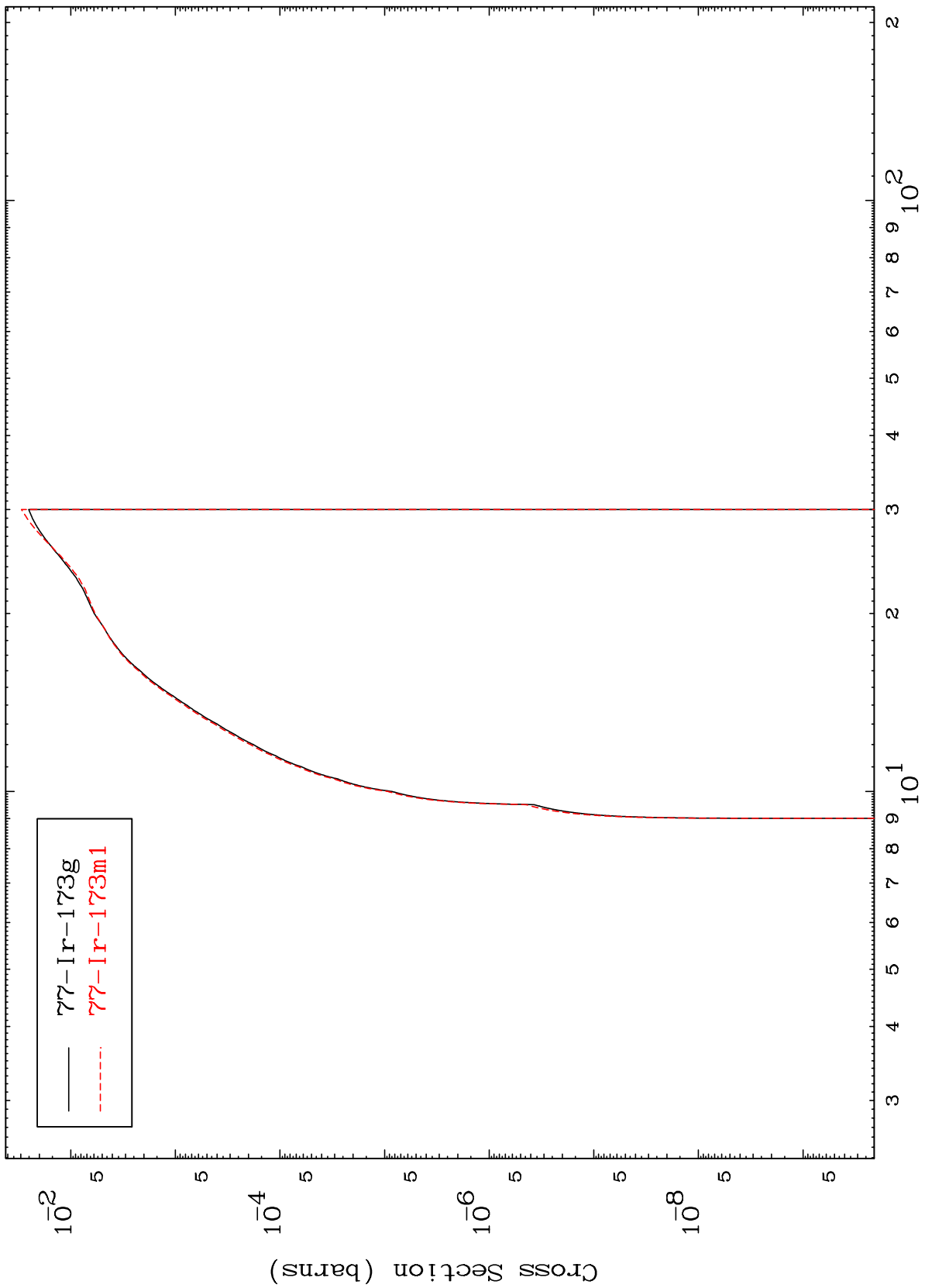
—  $^{77}\text{Ir-174g}$   
- - -  $^{77}\text{Ir-174m1}$

MAT 7868

(n,2n)  $\alpha$

79-Au-178

Radionuclide Production Cross Section

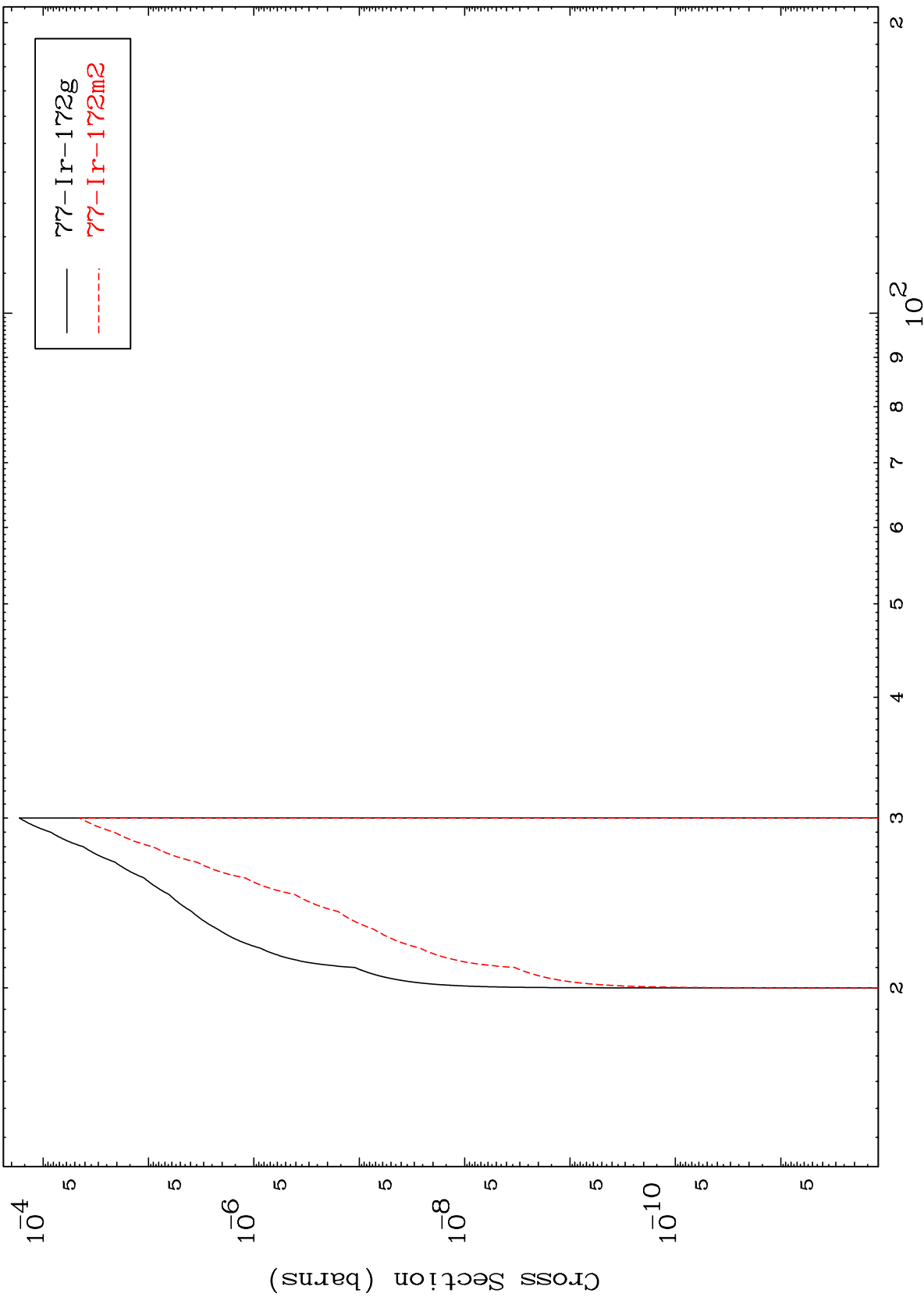


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Incident Energy (MeV)

79-Au-178

Radionuclide Production Cross Section



MAT 7868

$^{75}\text{Re-169g}$   $2\alpha$

$^{79}\text{Au-178}$

Radionuclide Production Cross Section

