

Program EVALPLOT  
(Version 2017-1)

by

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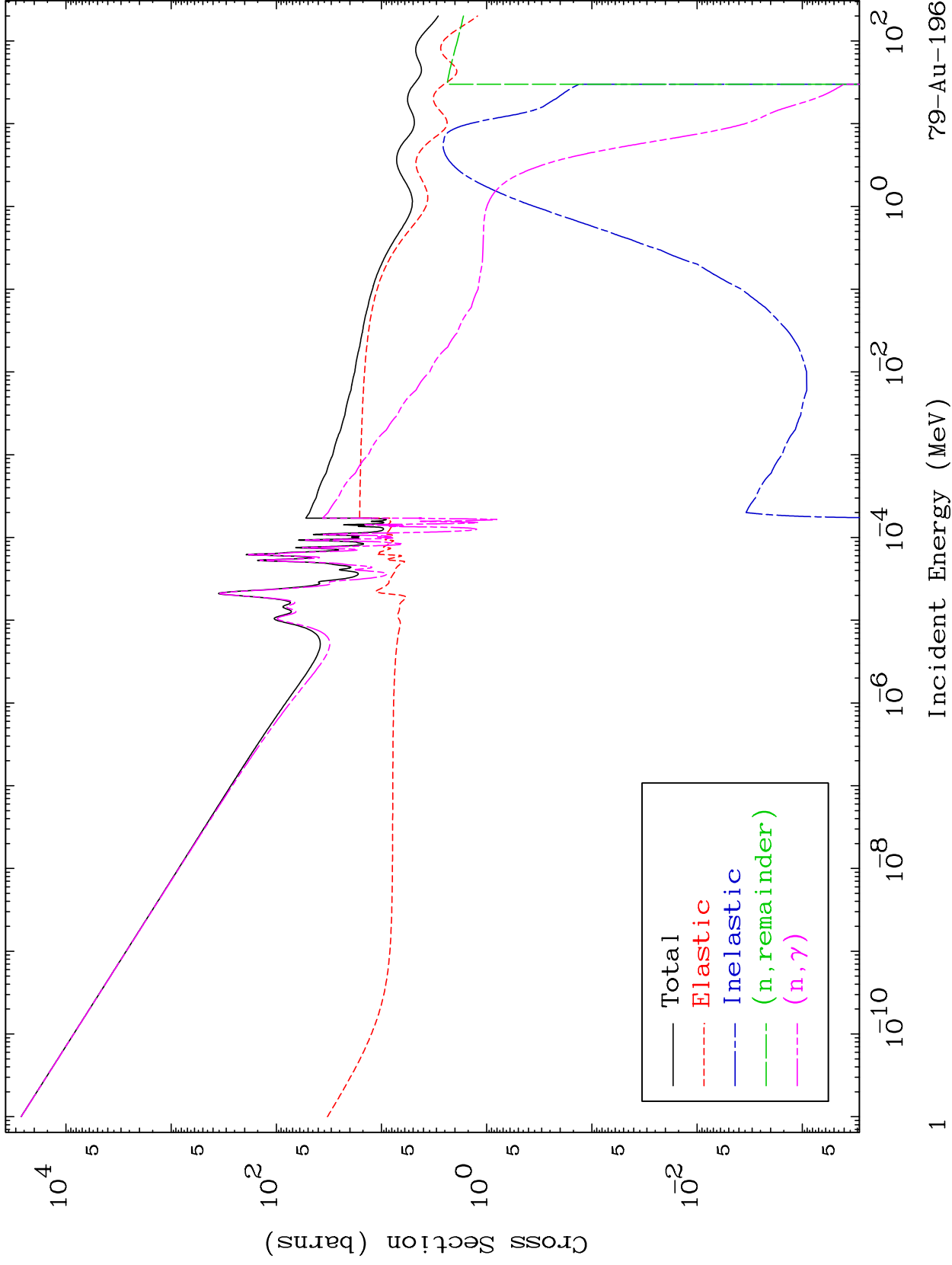
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7924

Major  
293 Kelvin Cross Sections

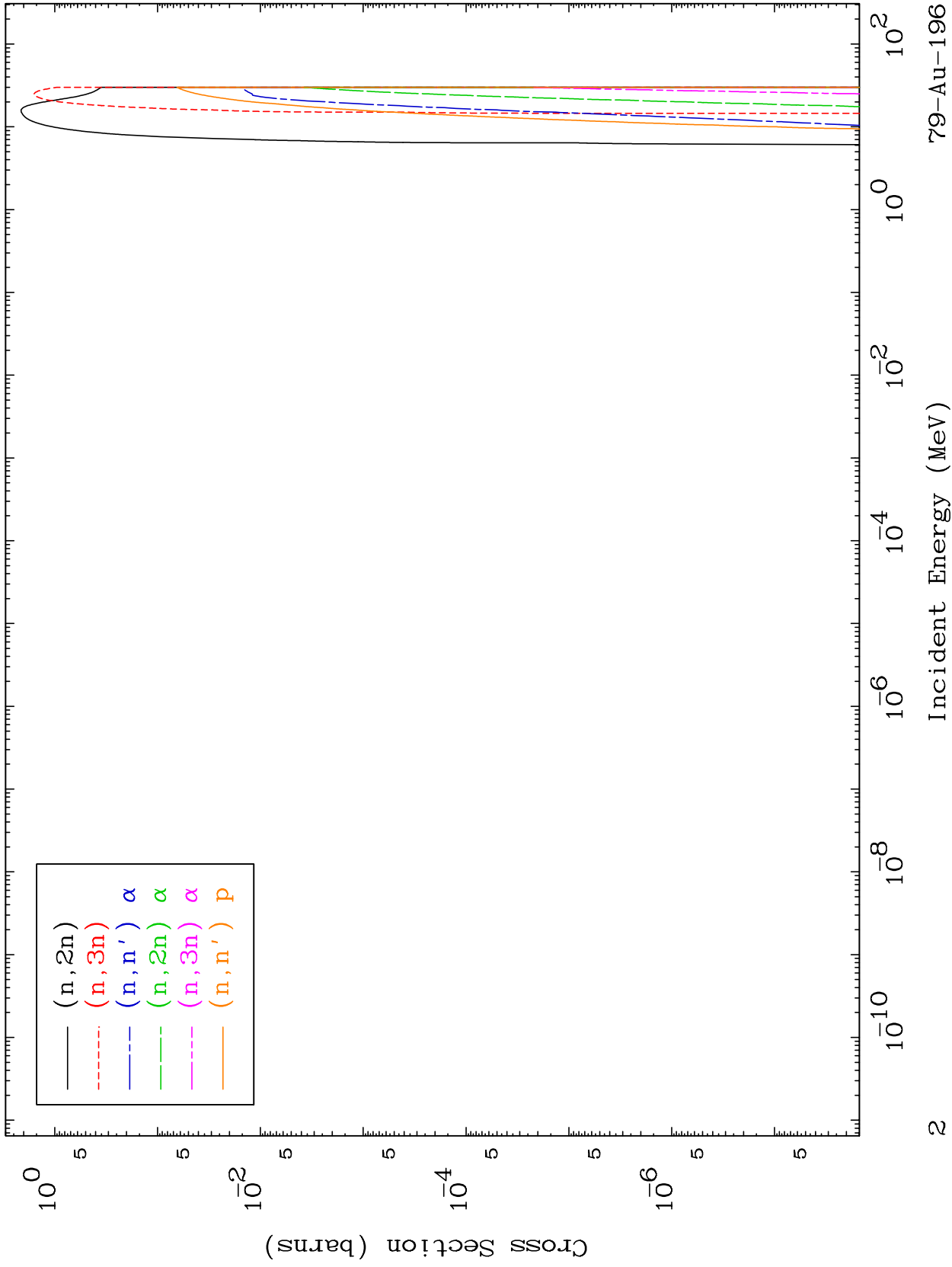
79-Au-196



MAT 7924

Neutron Production  
293 Kelvin Cross Sections

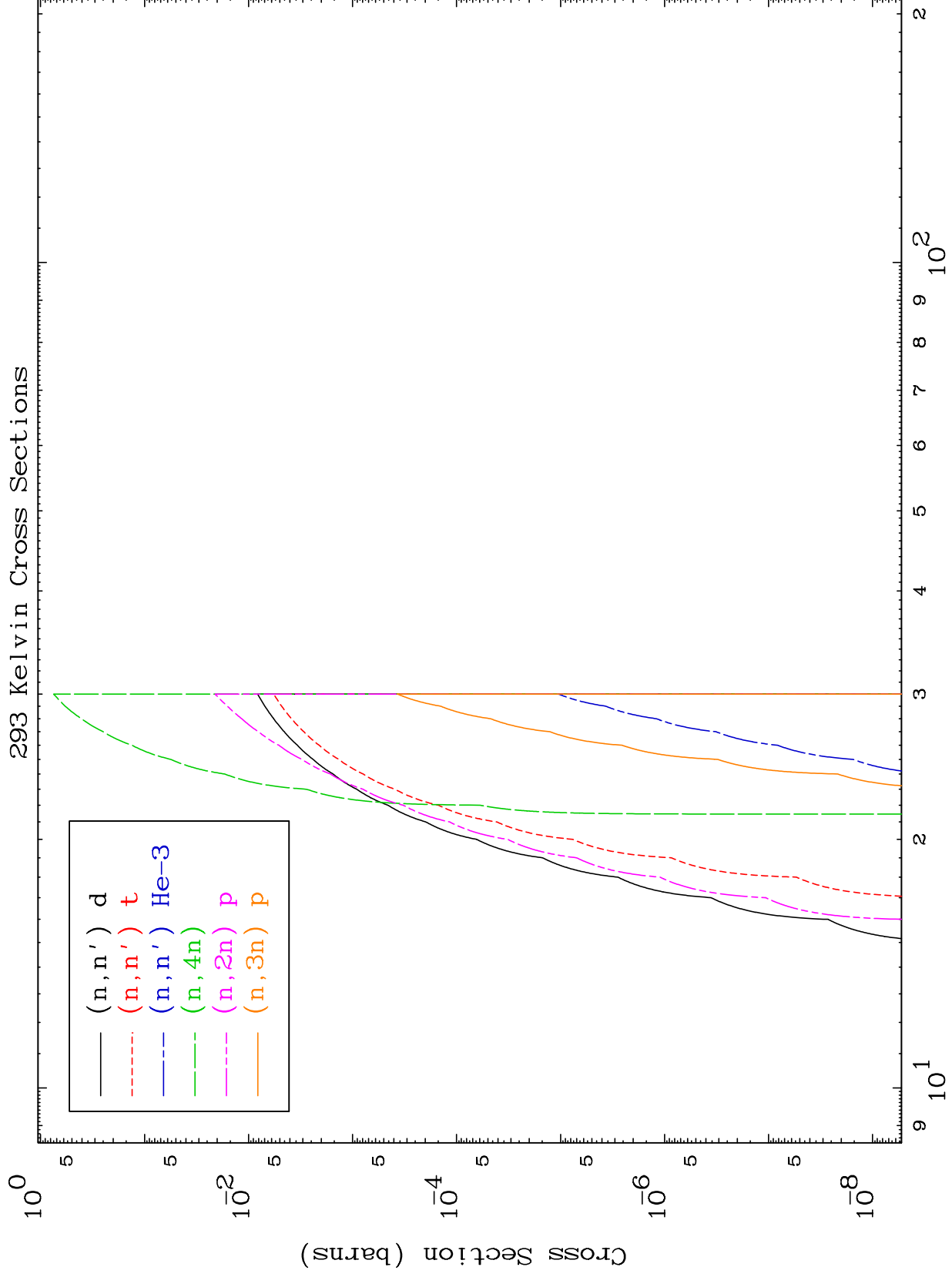
79-Au-196



MAT 7924

Neutron Production  
293 Kelvin Cross Sections

79-Au-196



Incident Energy (MeV)

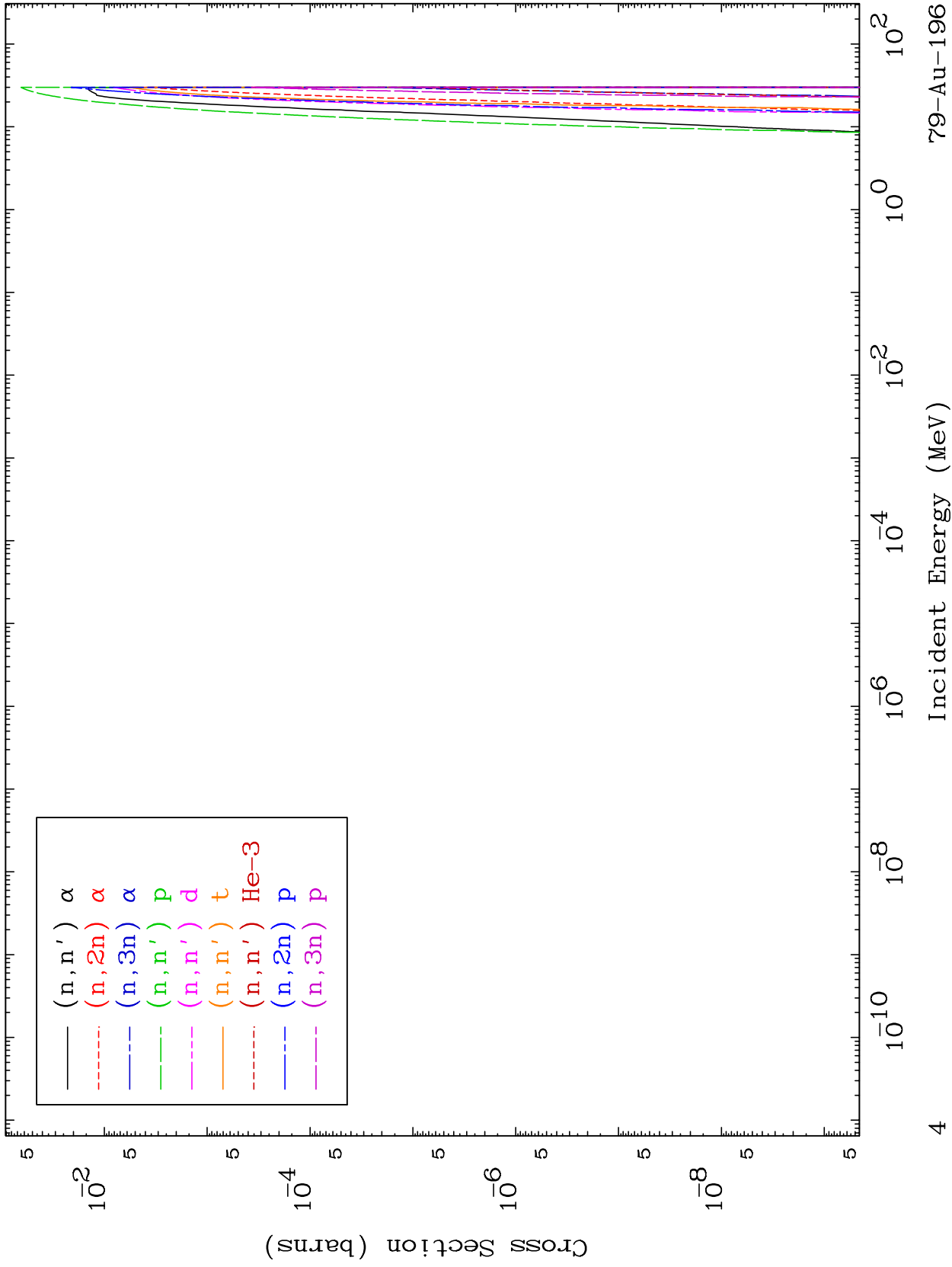
79-Au-196

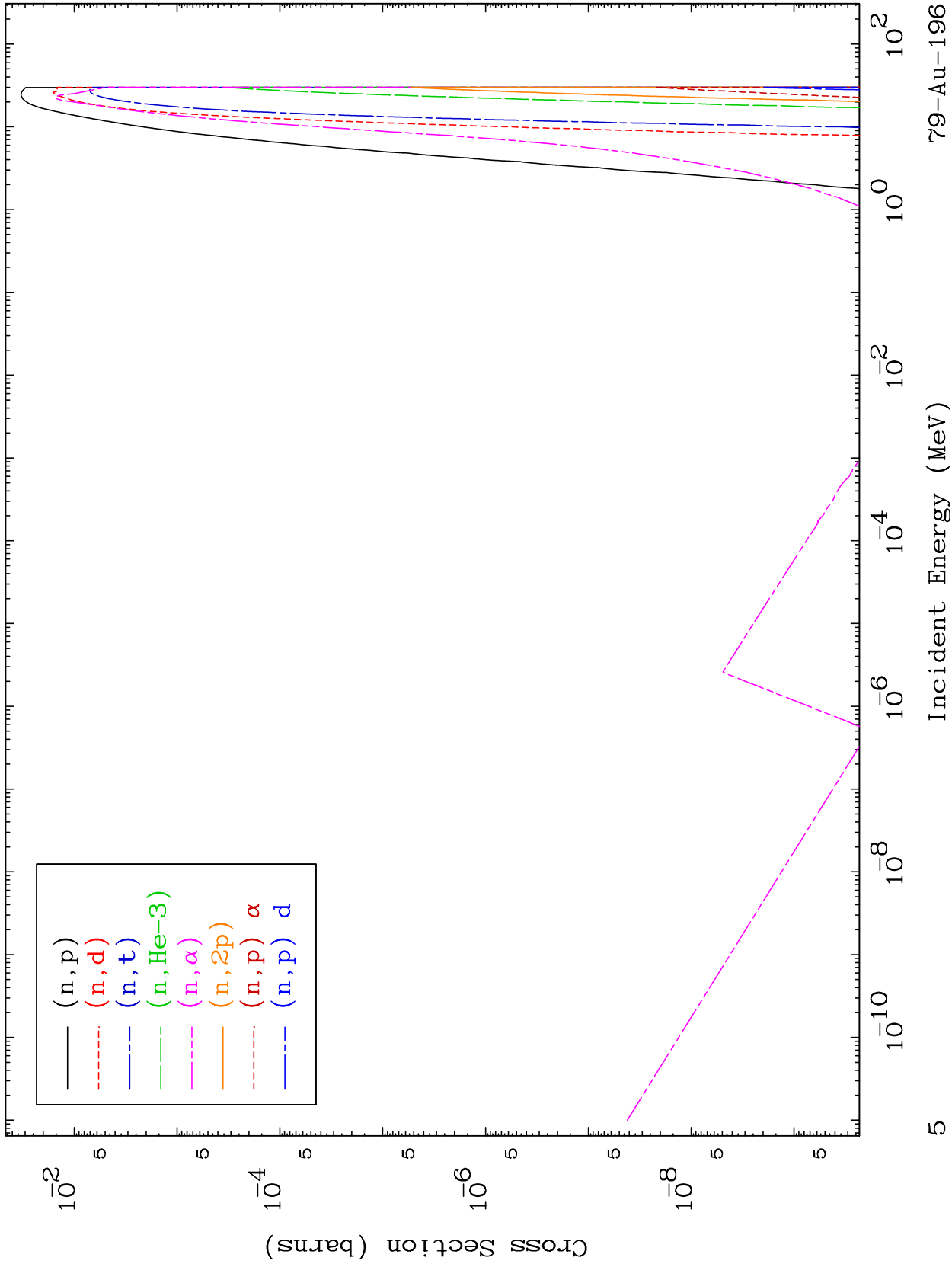
3

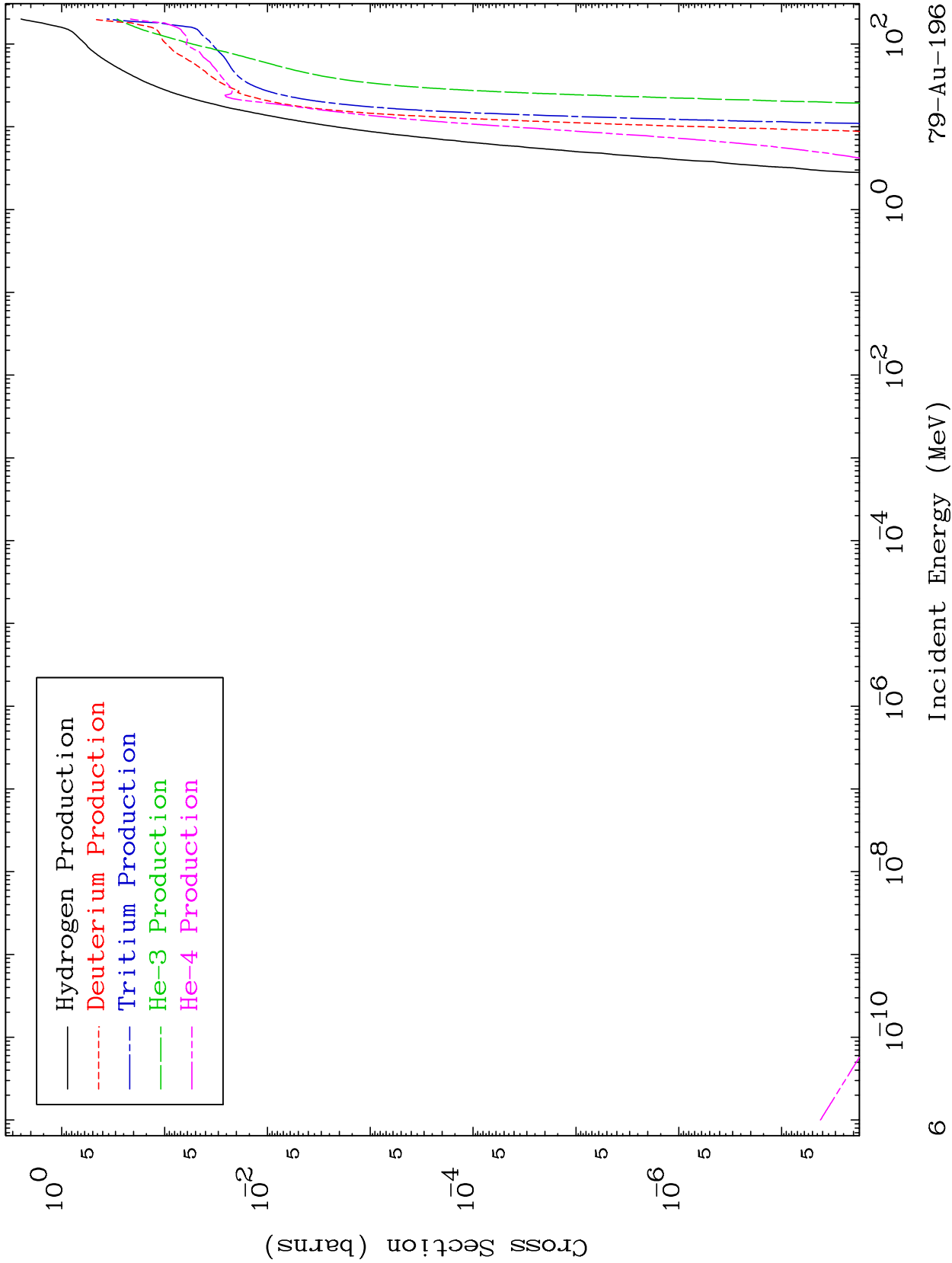
MAT 7924

Charged Particle  
293 Kelvin Cross Sections

79-Au-196



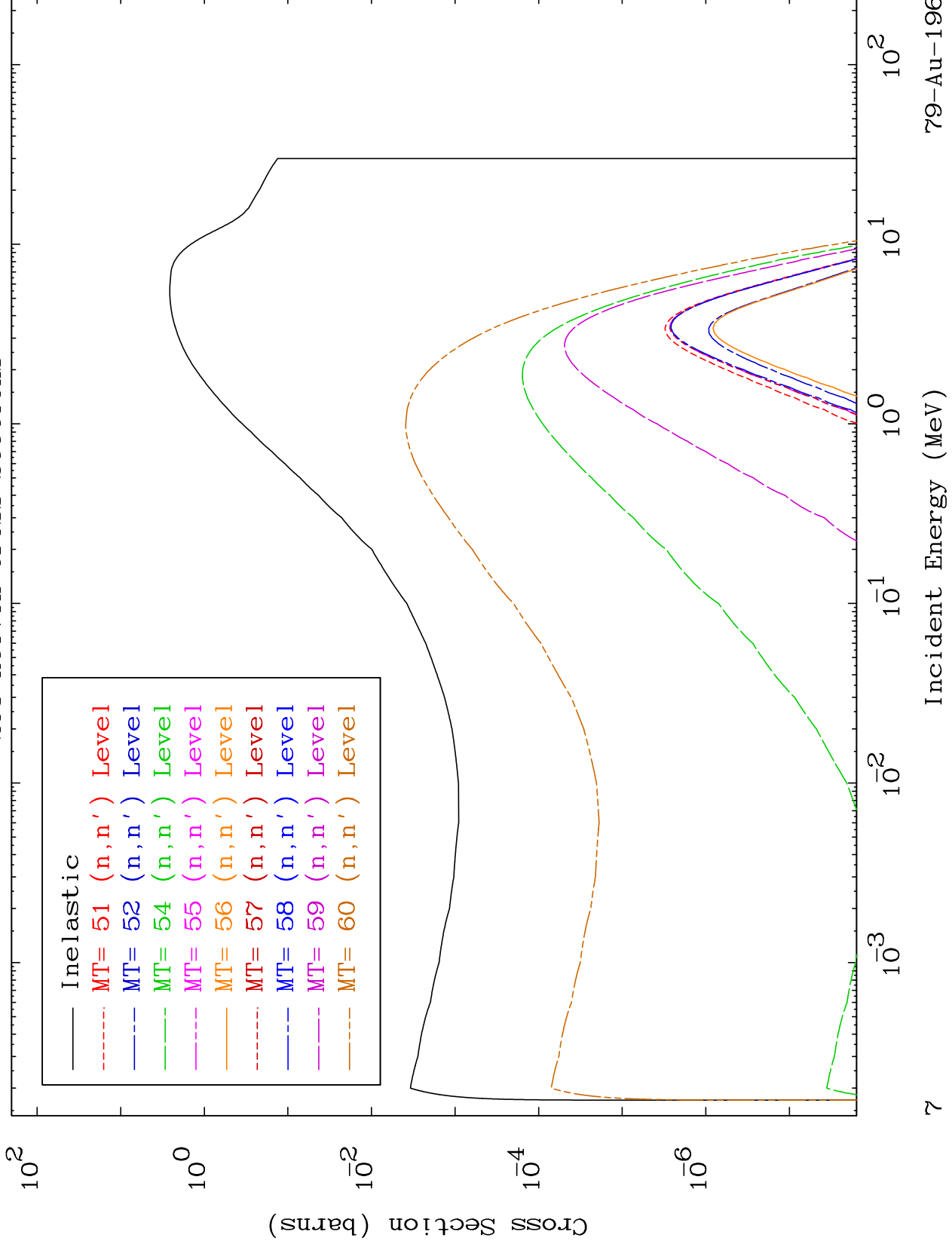




MAT 7924

(n,n') Level  
293 Kelvin Cross Sections

79-Au-196

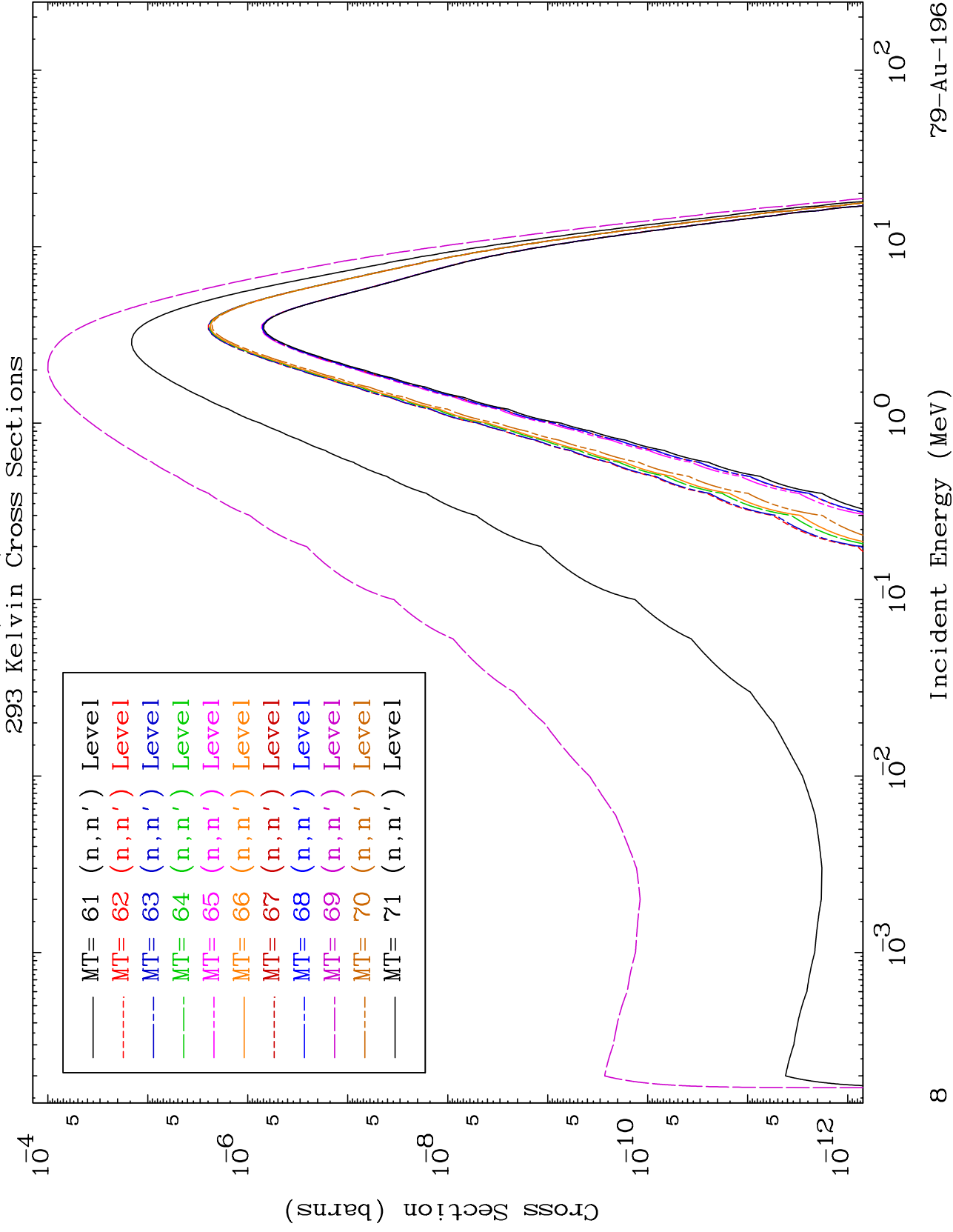




MAT 7924

(n,n') Level

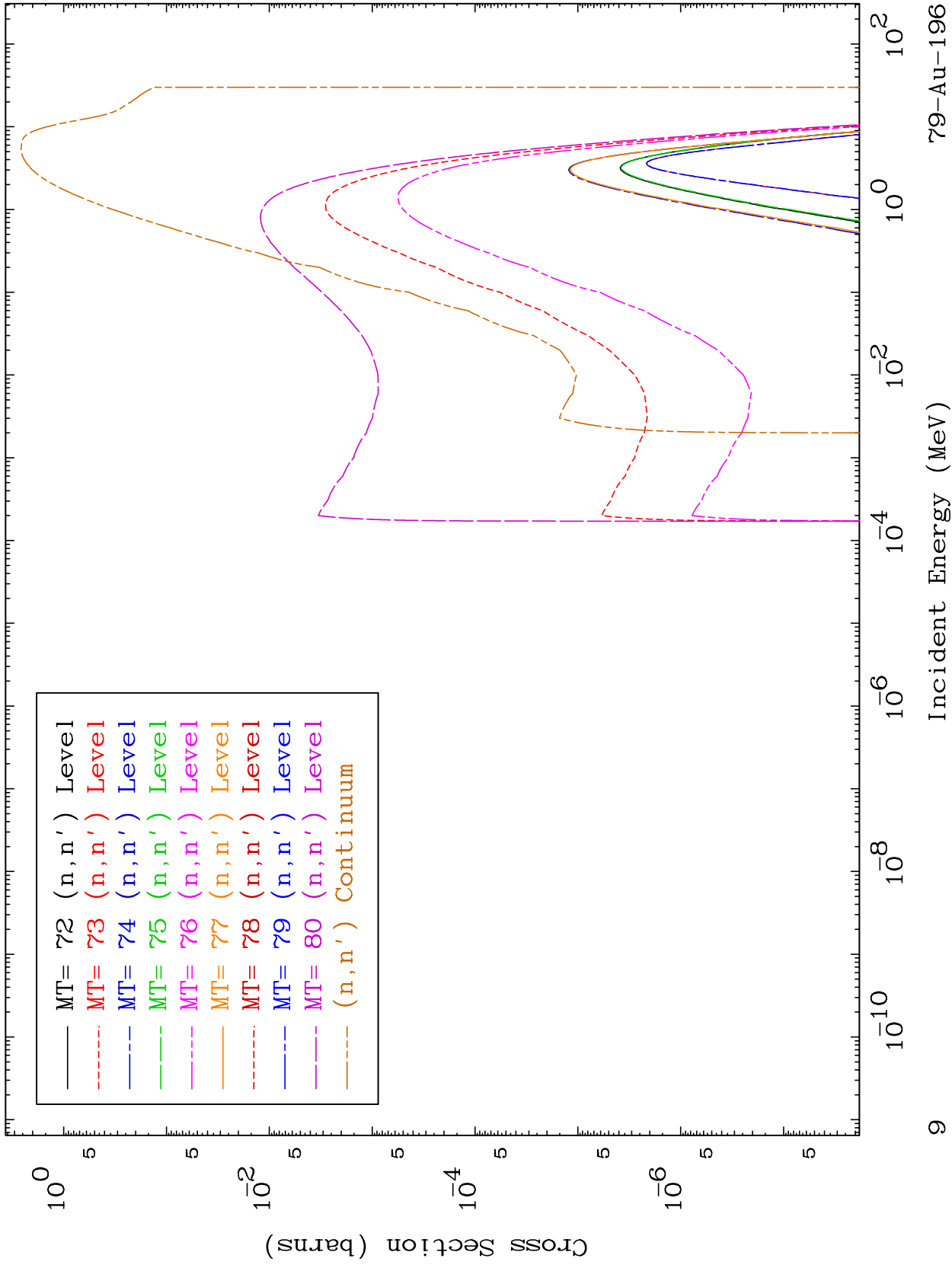
79-Au-196



MAT 7924

(n,n') Level  
293 Kelvin Cross Sections

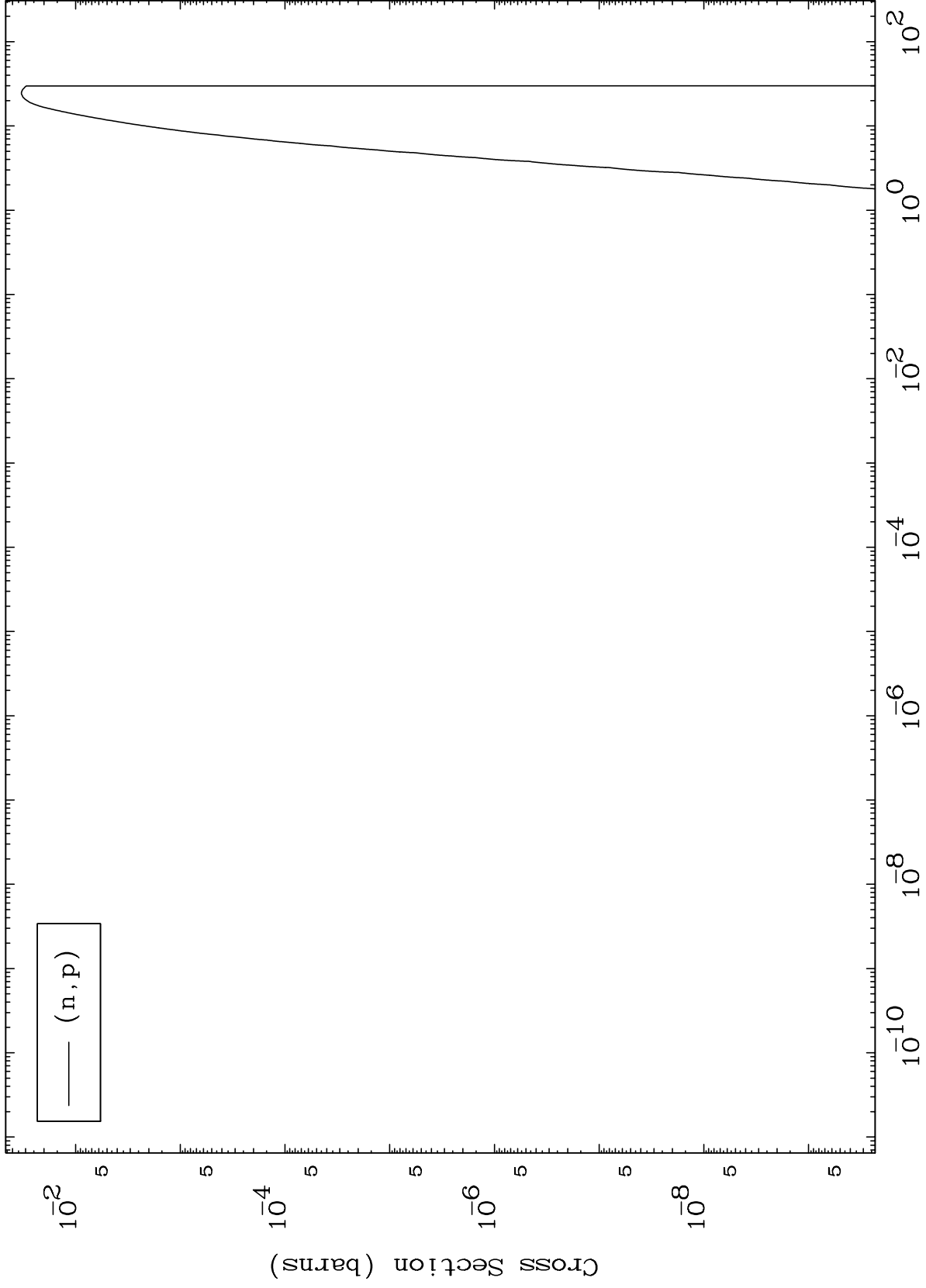
79-Au-196



MAT 7924

(n,p) Levels  
293 Kelvin Cross Sections

79-Au-196



10

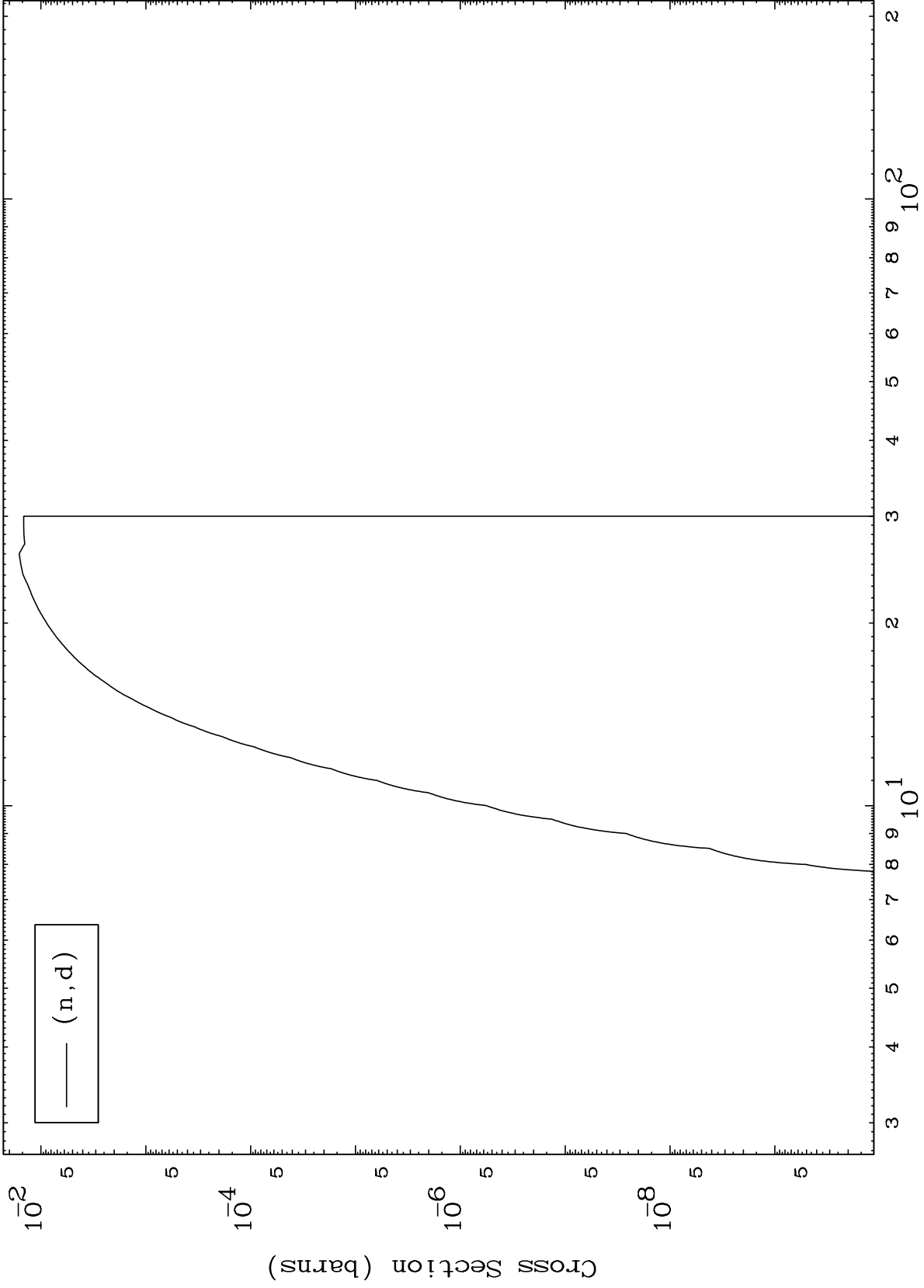
Incident Energy (MeV)

79-Au-196

MAT 7924

(n,d) Levels  
293 Kelvin Cross Sections

79-Au-196



11

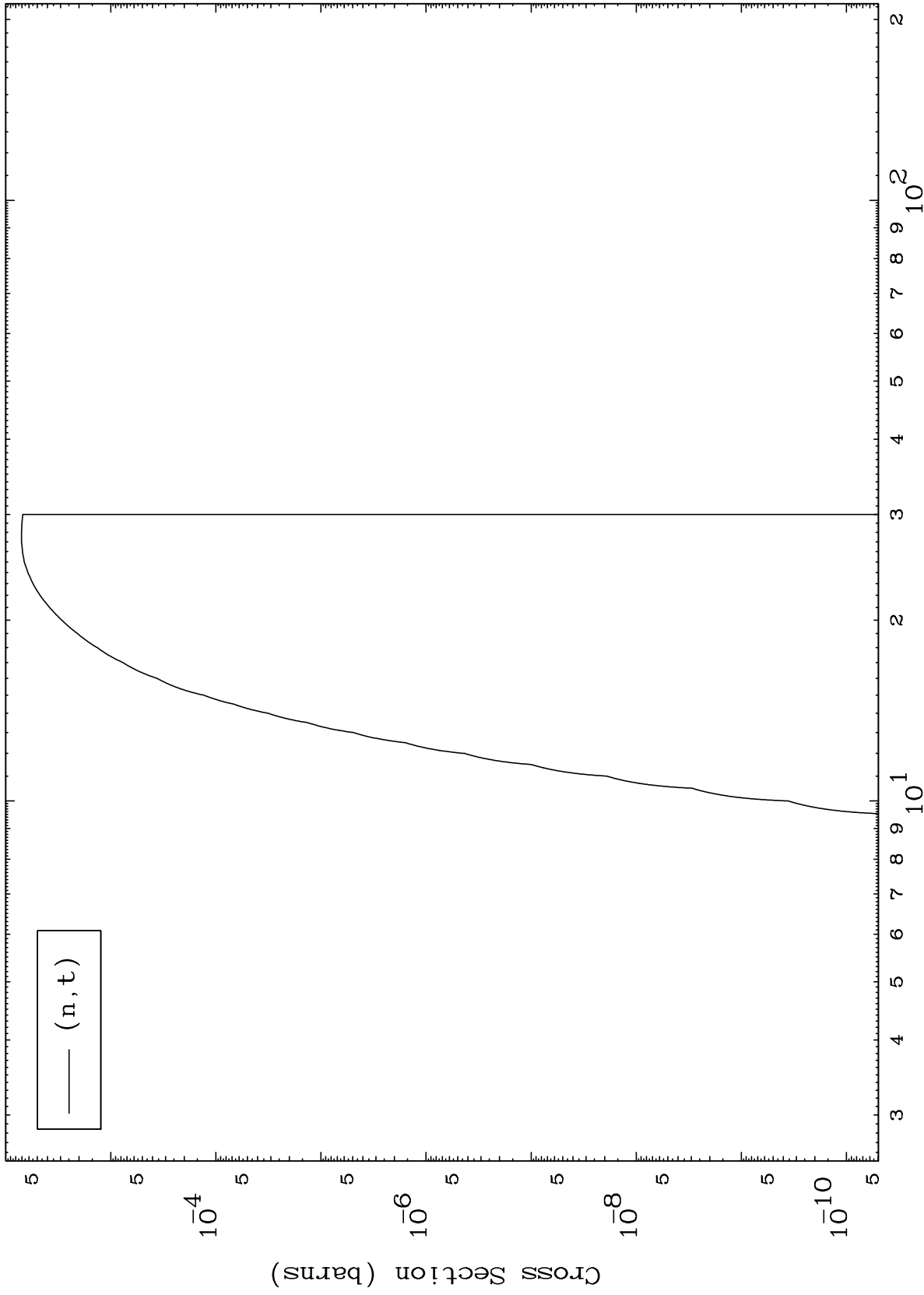
Incident Energy (MeV)

79-Au-196

MAT 7924

(n,t) Levels  
293 Kelvin Cross Sections

79-Au-196



12

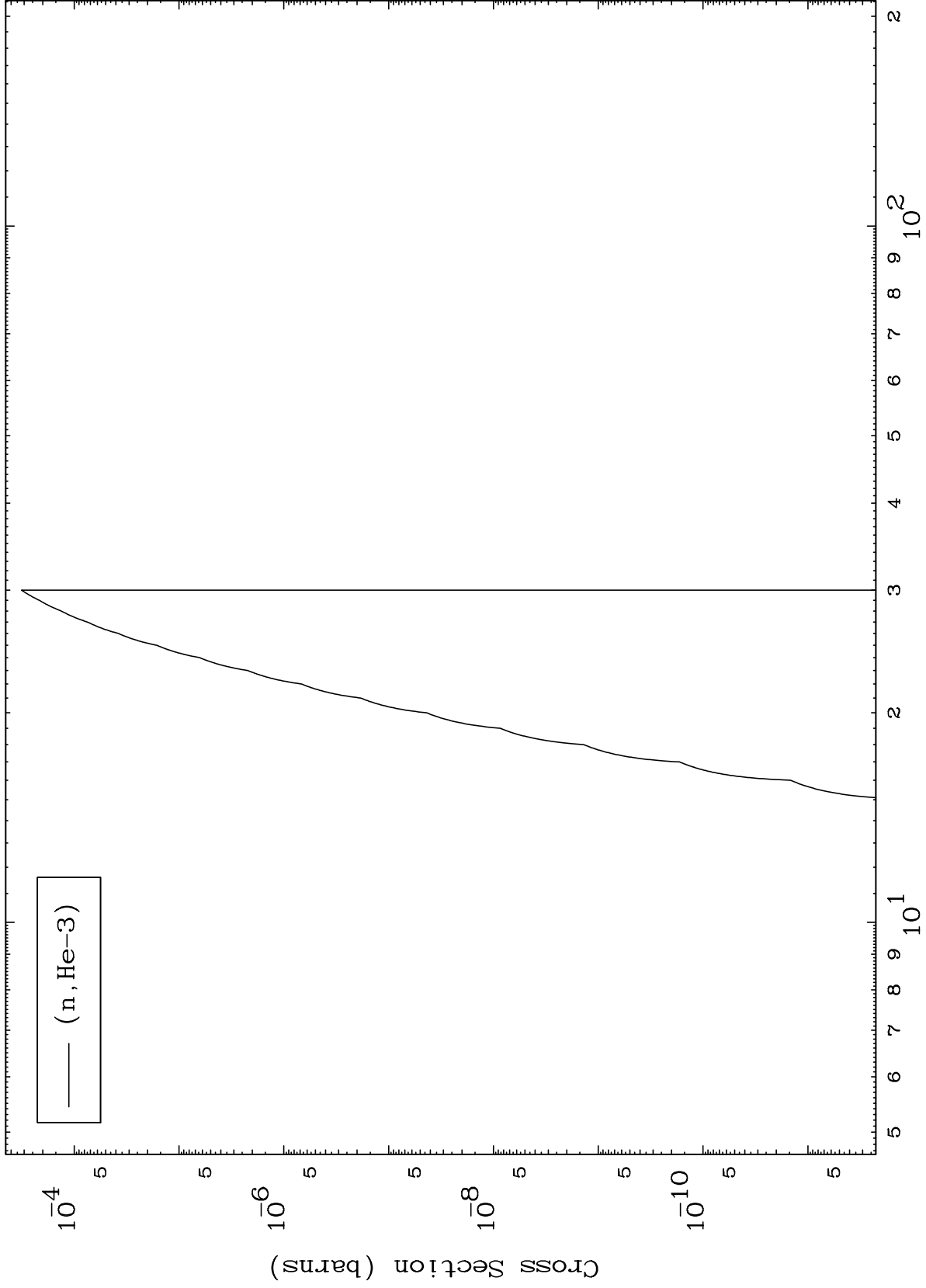
Incident Energy (MeV)

79-Au-196

MAT 7924

(n,He3) Levels  
293 Kelvin Cross Sections

79-Au-196



13

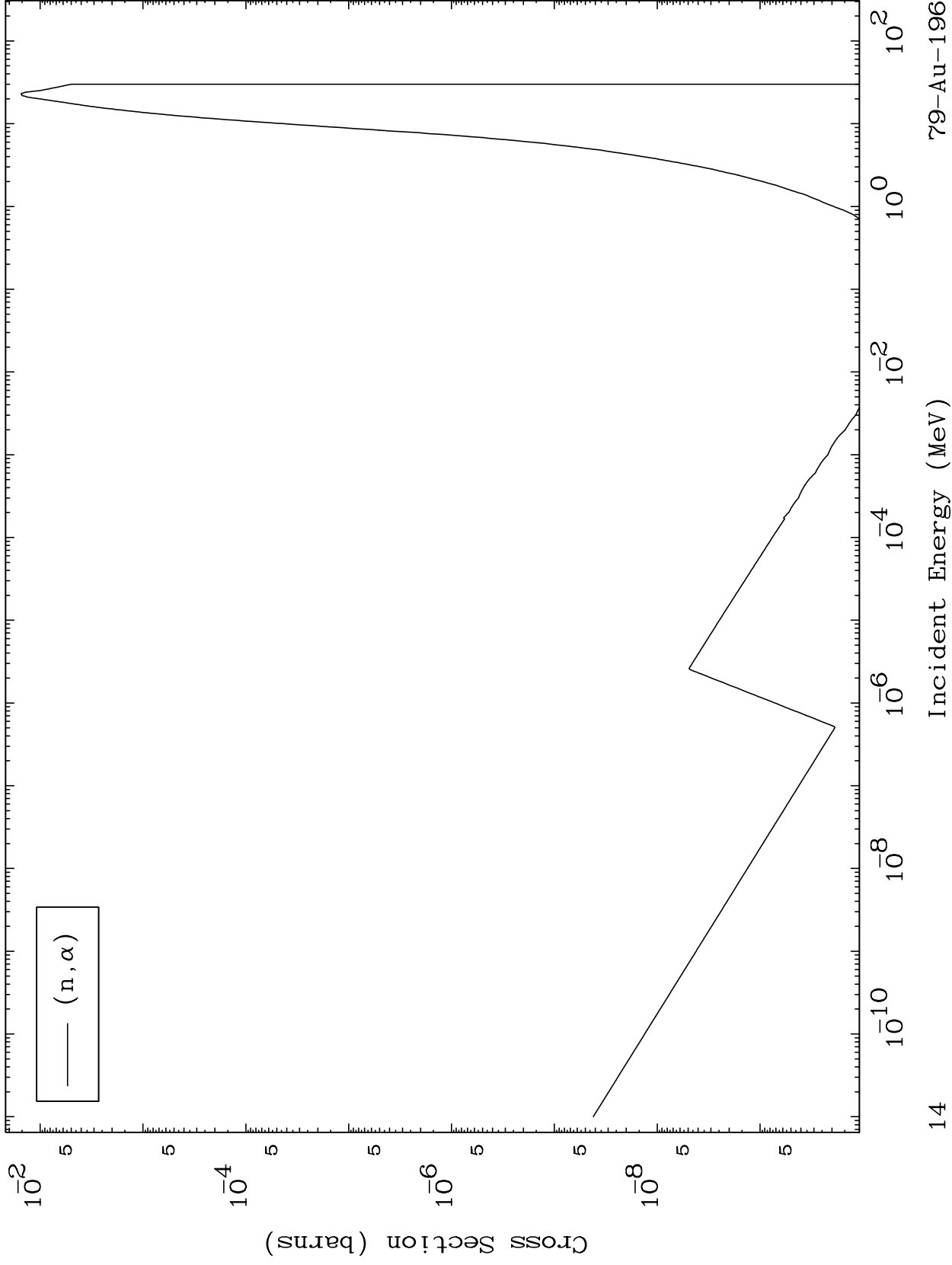
Incident Energy (MeV)

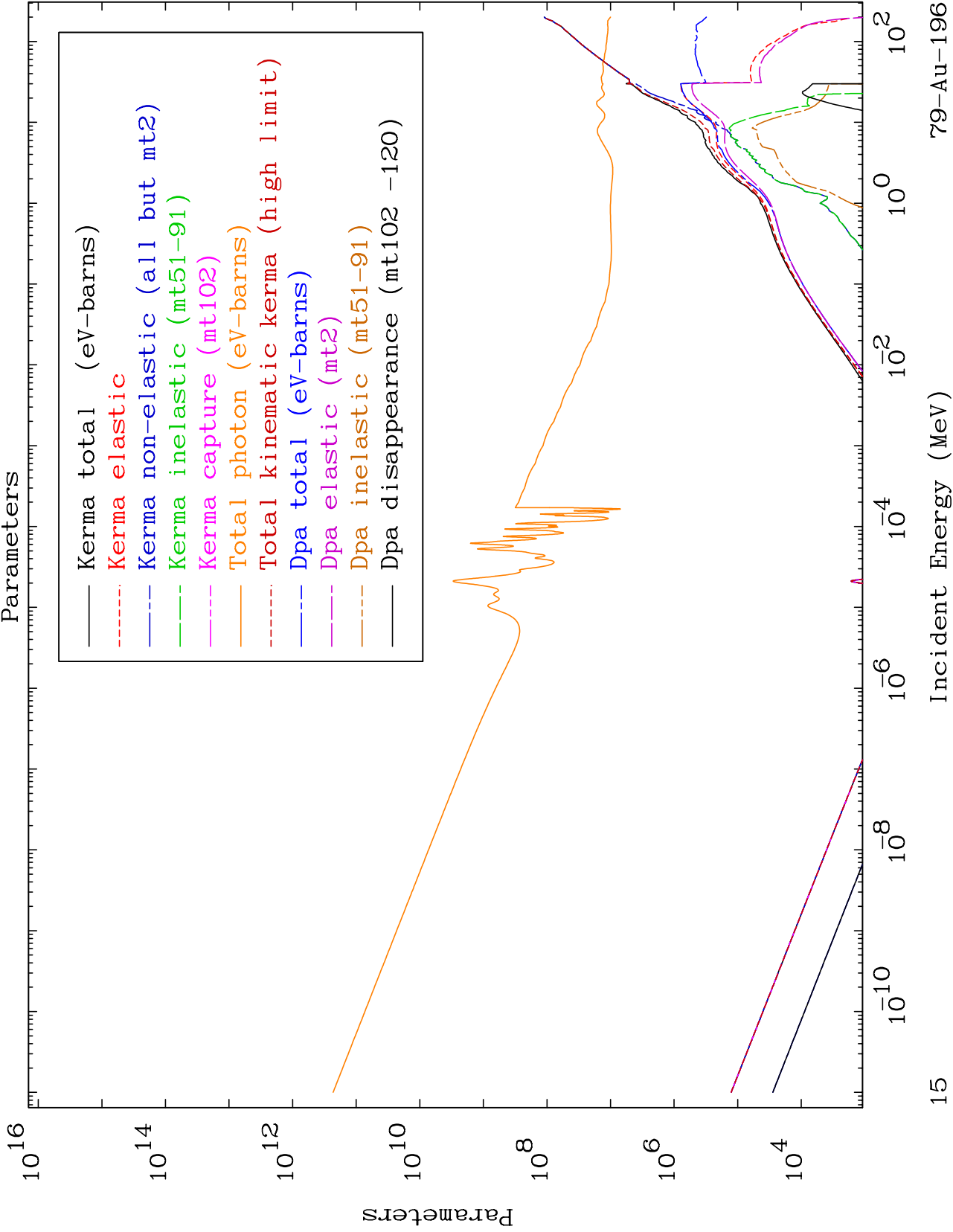
79-Au-196

MAT 7924

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

79-Au-196



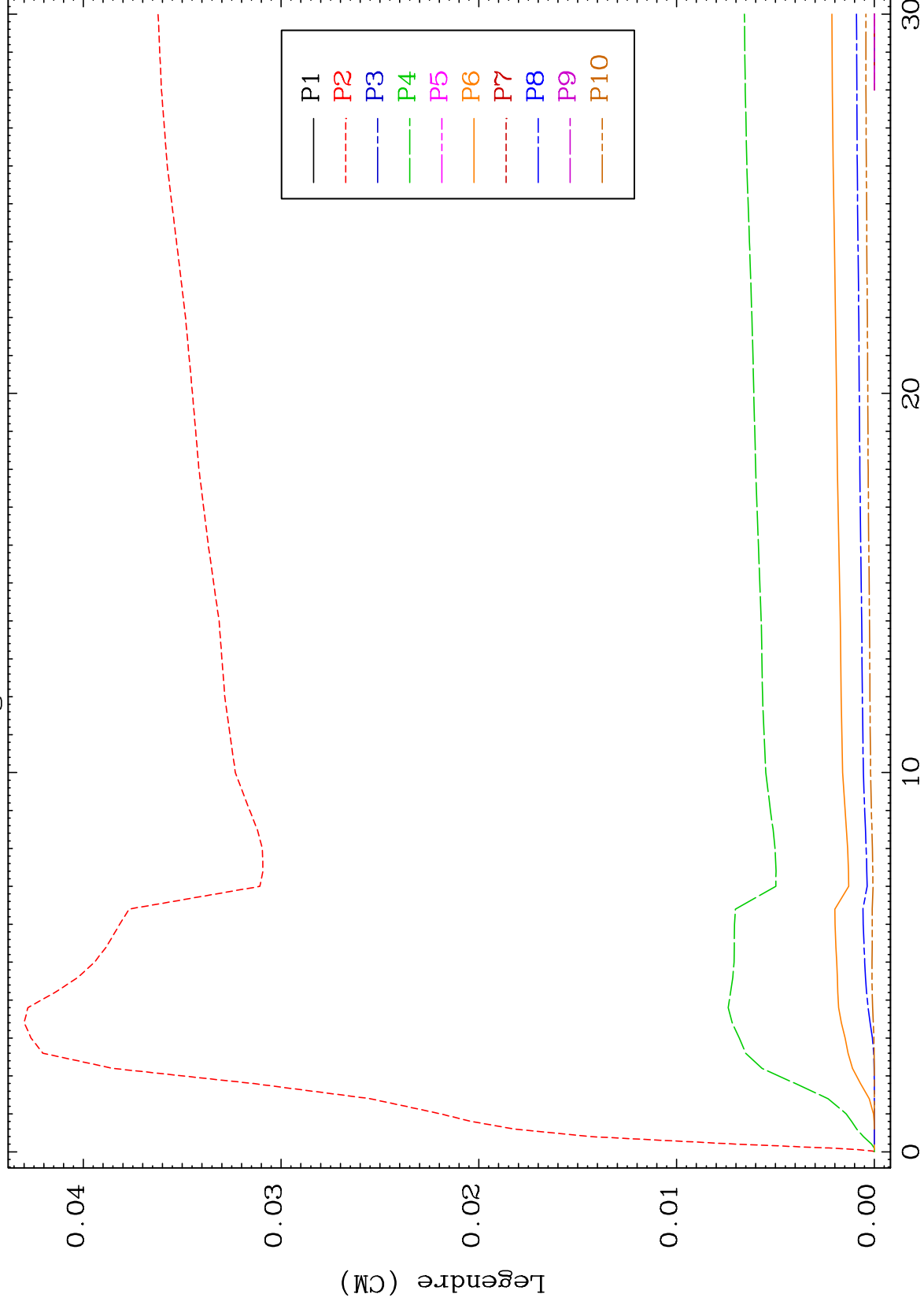




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Elastic Legendre Coefficients

79-Au-196



16

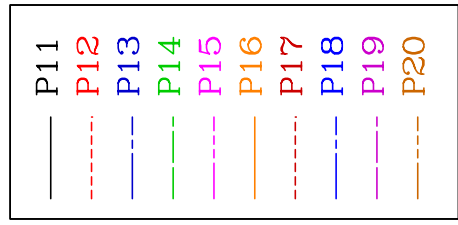
Incident Energy (MeV)

79-Au-196

MAT 7924

Elastic  
Legendre Coefficients

79-Au-196



$\times 10^{-4}$   
2.0  
1.5  
1.0  
0.5  
0.0

Legendre (CM)

5

10

15

20

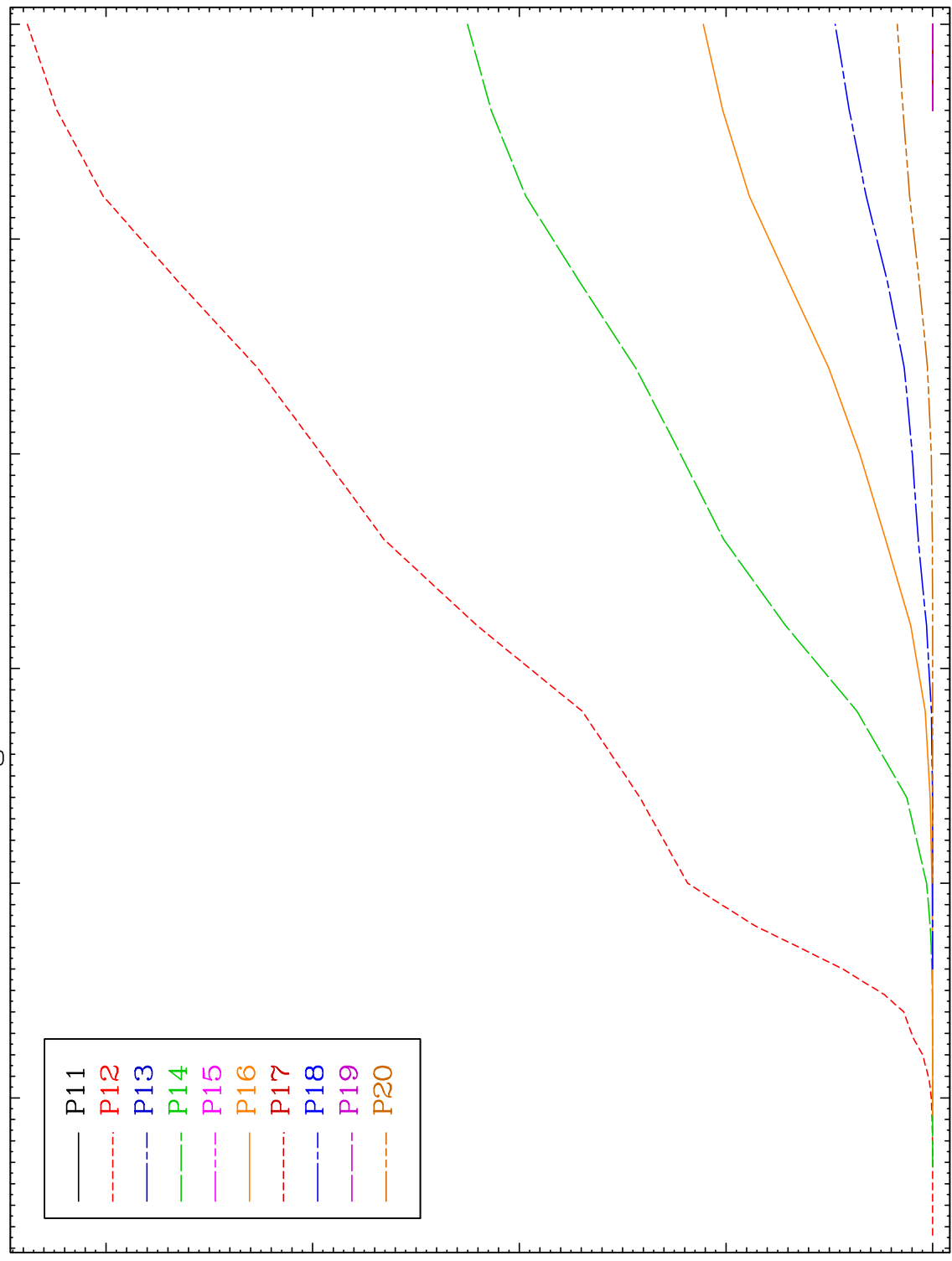
25

30

17

Incident Energy (MeV)

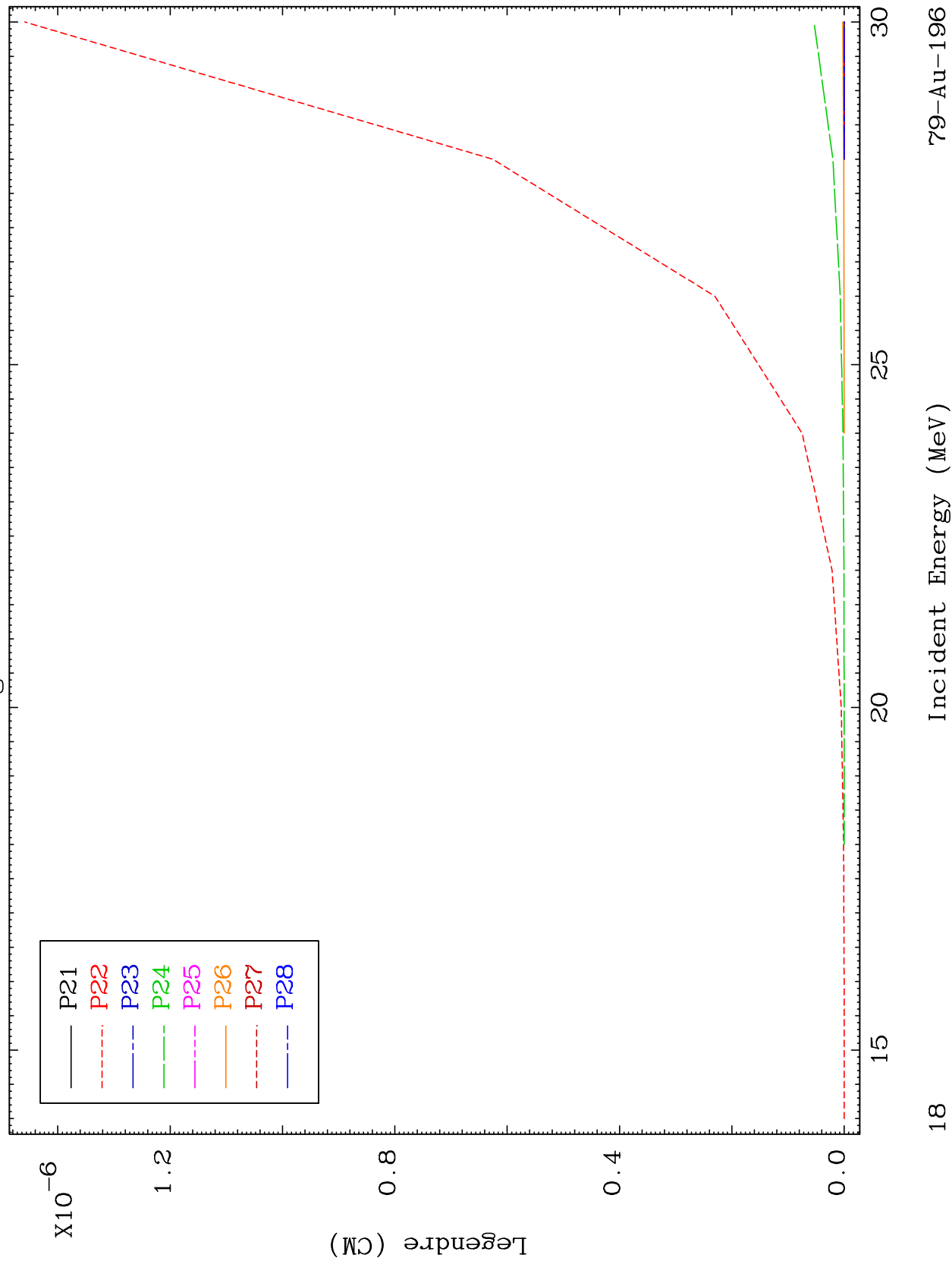
79-Au-196



MAT 7924

Elastic Legendre Coefficients

79-Au-196



18

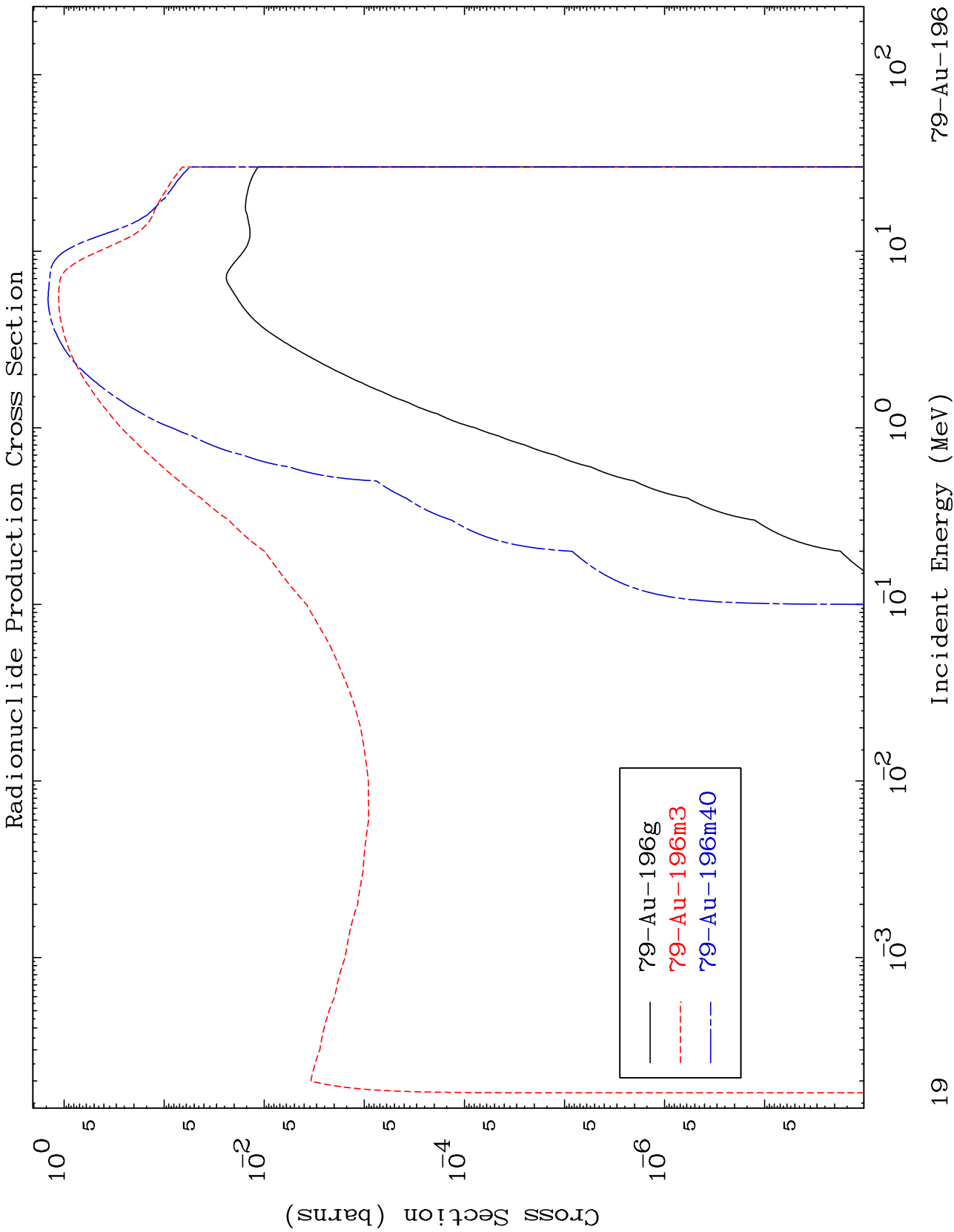
Incident Energy (MeV)

79-Au-196

MAT 7924

79-Au-196

Inelastic  
Radionuclide Production Cross Section

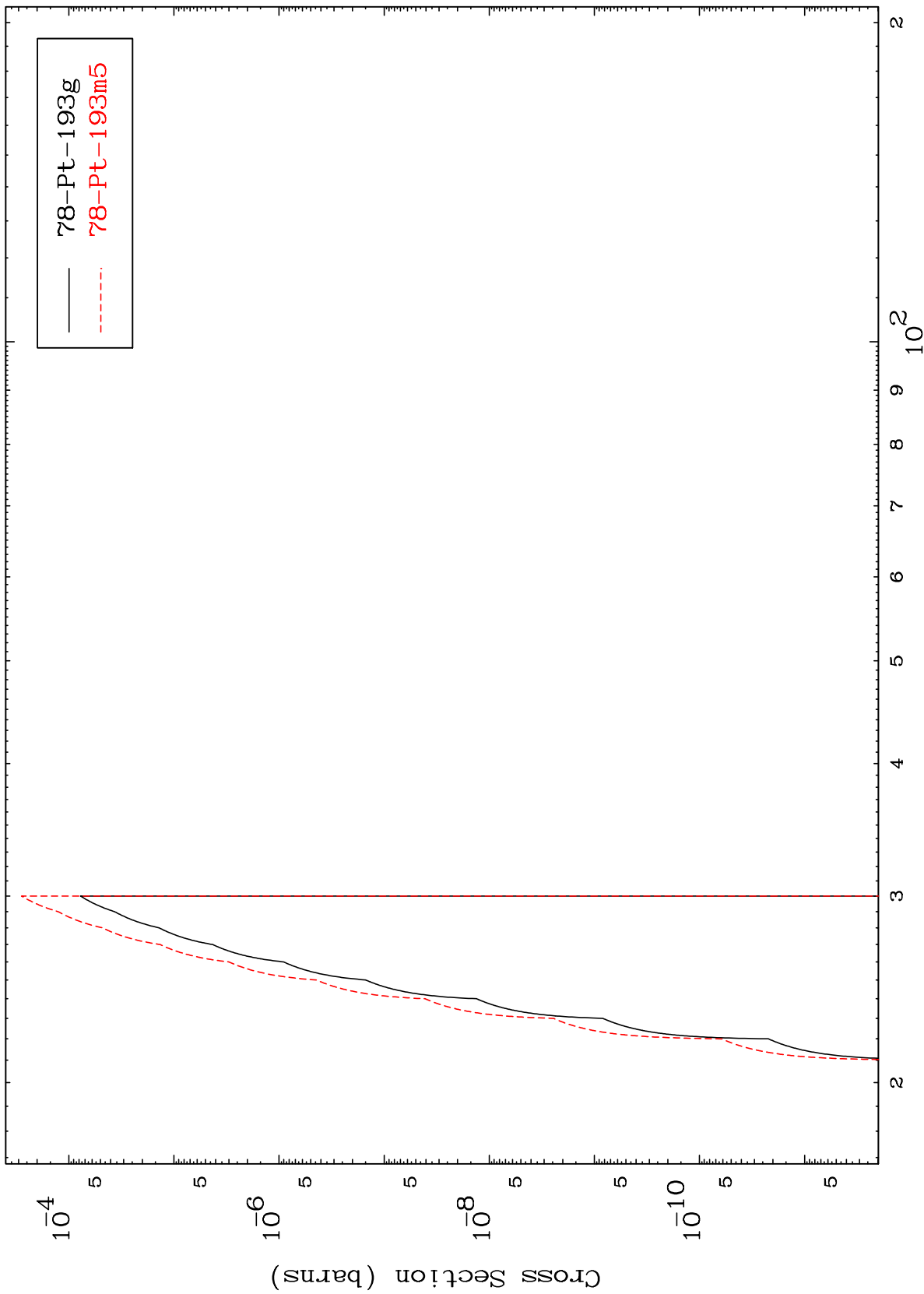


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(n,2n) d

79-Au-196

Radionuclide Production Cross Section



20

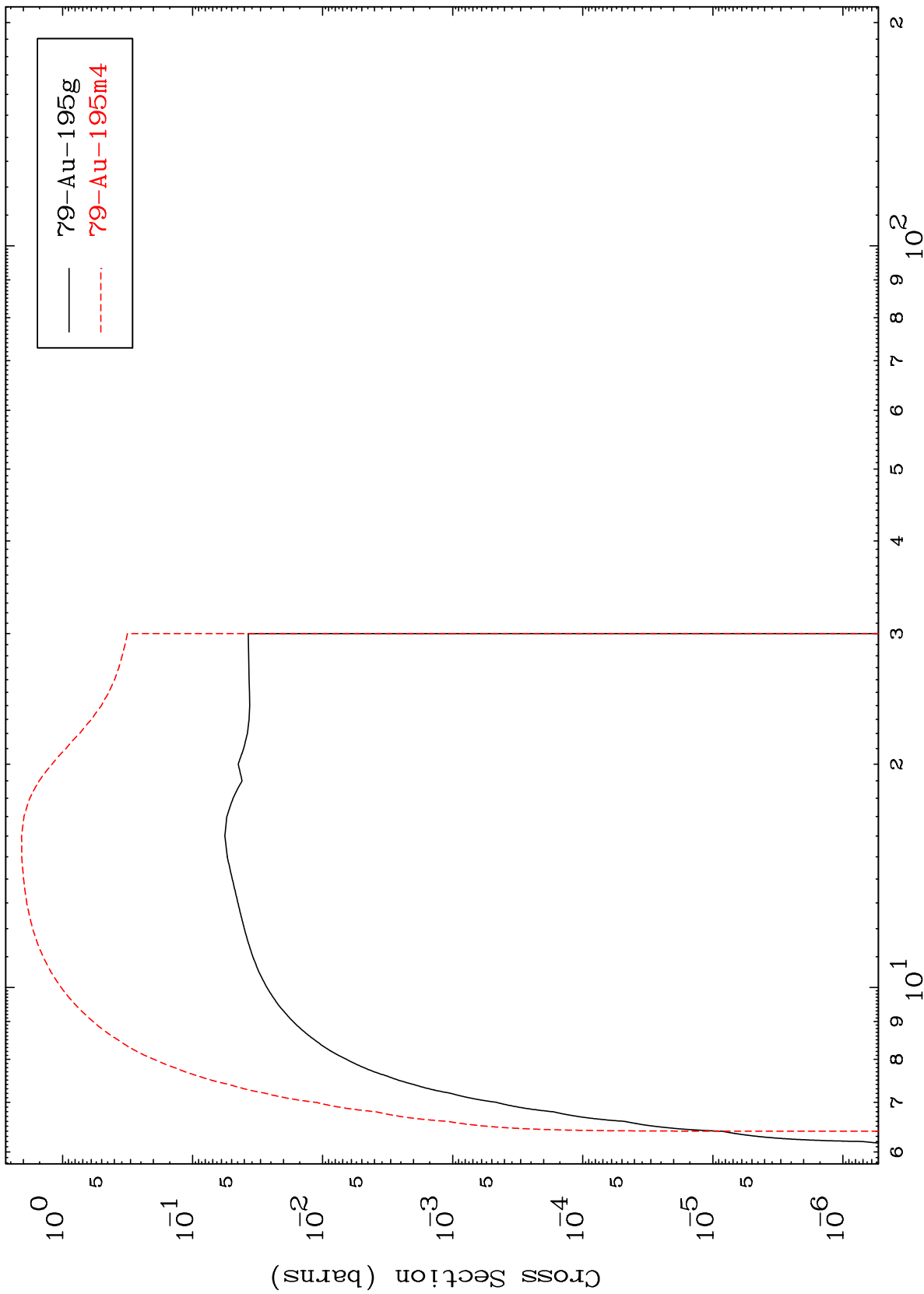
Incident Energy (MeV)

79-Au-196

MAT 7924

79-Au-196

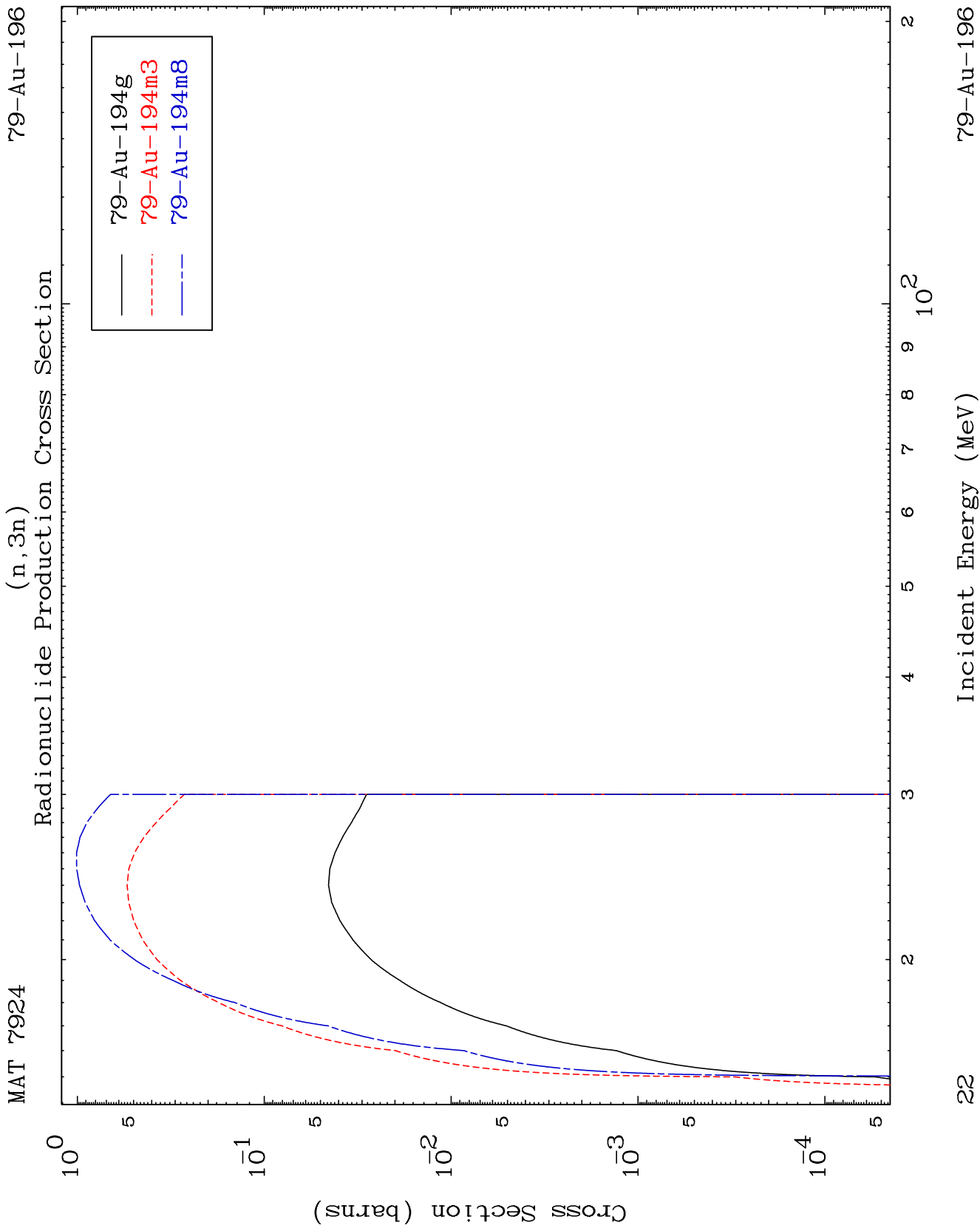
(n,2n)  
Radionuclide Production Cross Section



21

Incident Energy (MeV)

79-Au-196

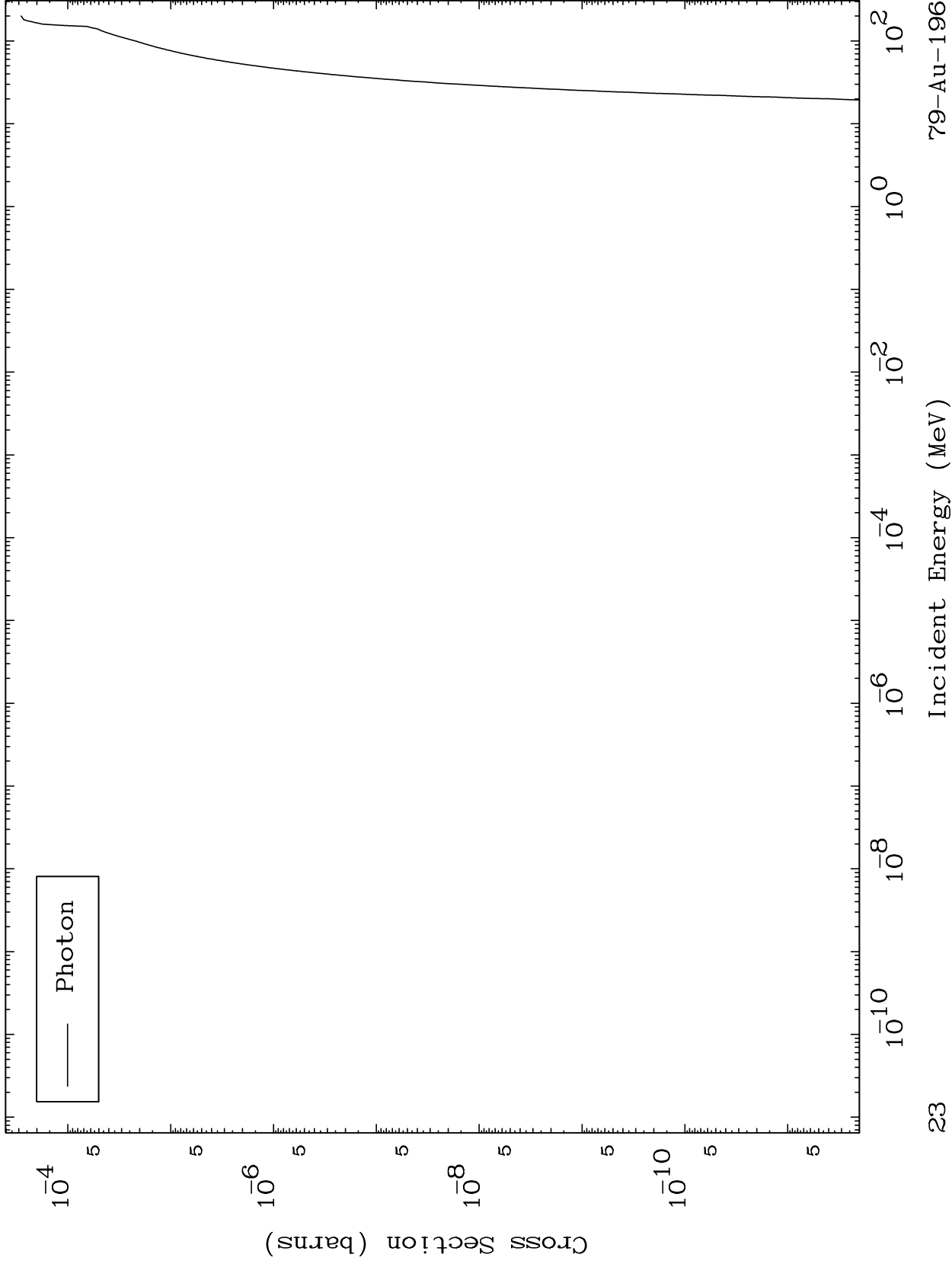


MAT 7924

Fission

79-Au-196

Radionuclide Production Cross Section

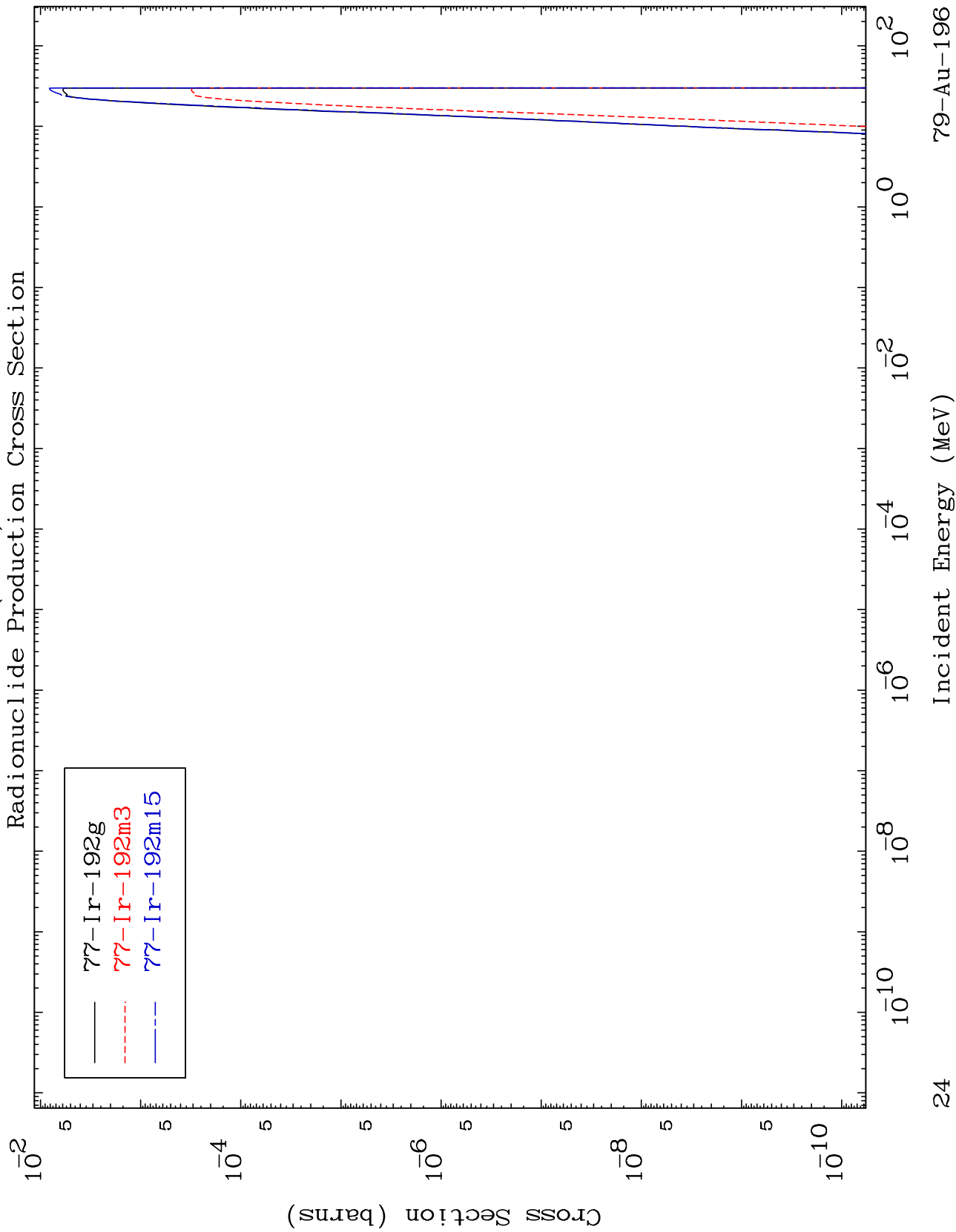




MAT 7924

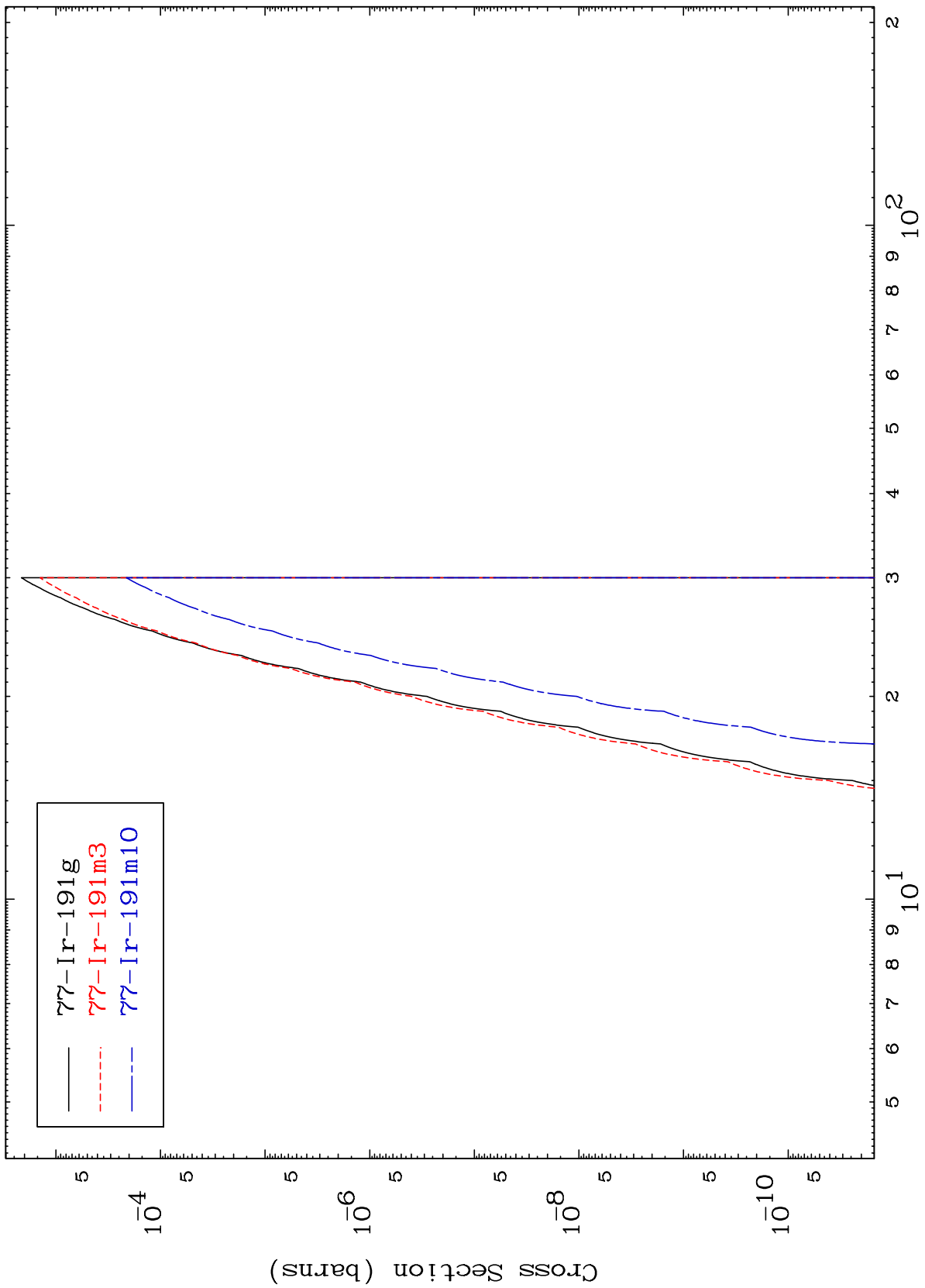
$(n, n') \alpha$

$^{79}\text{Au-196}$

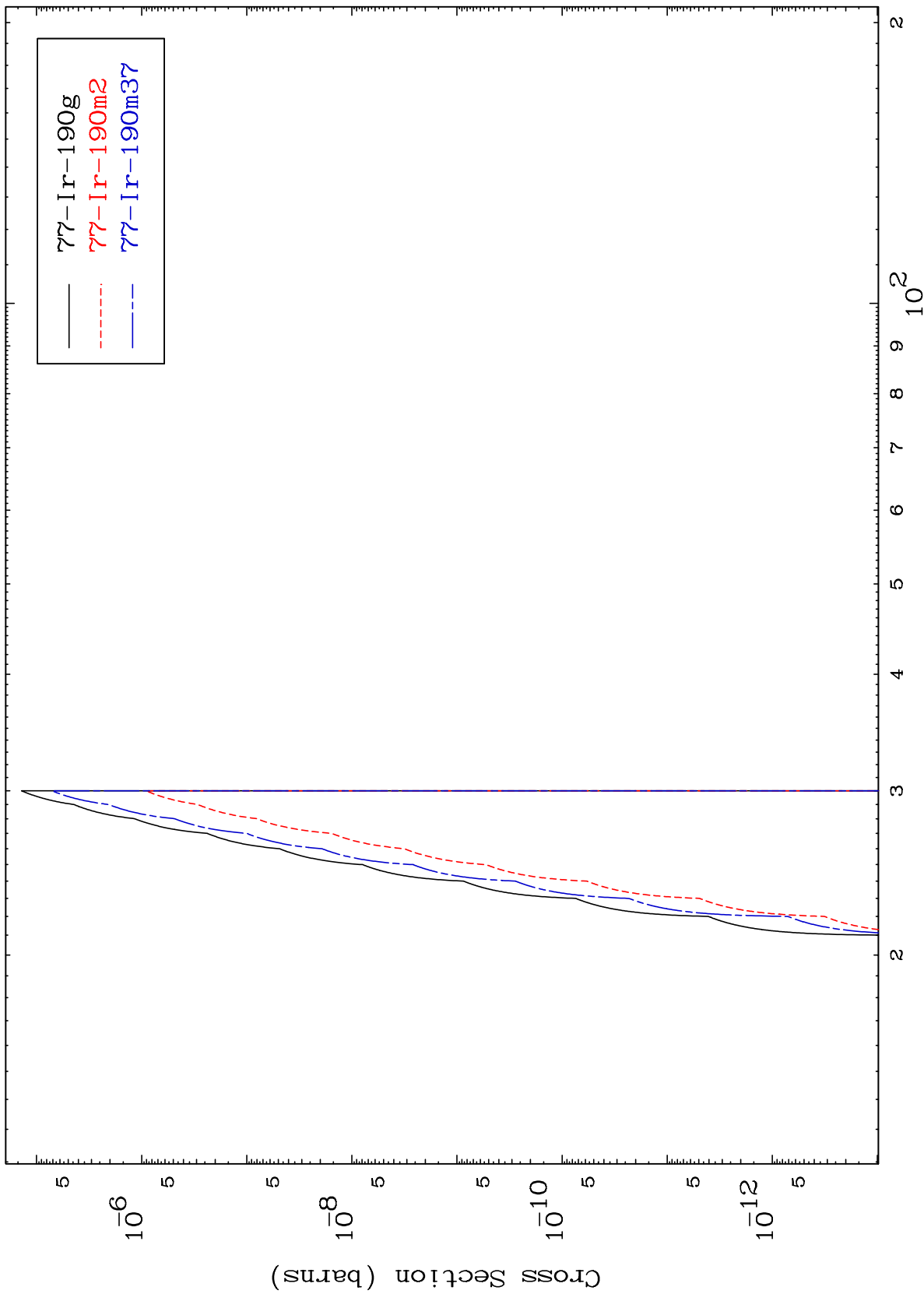


(n,2n)  $\alpha$

Radionuclide Production Cross Section



Radionuclide Production Cross Section

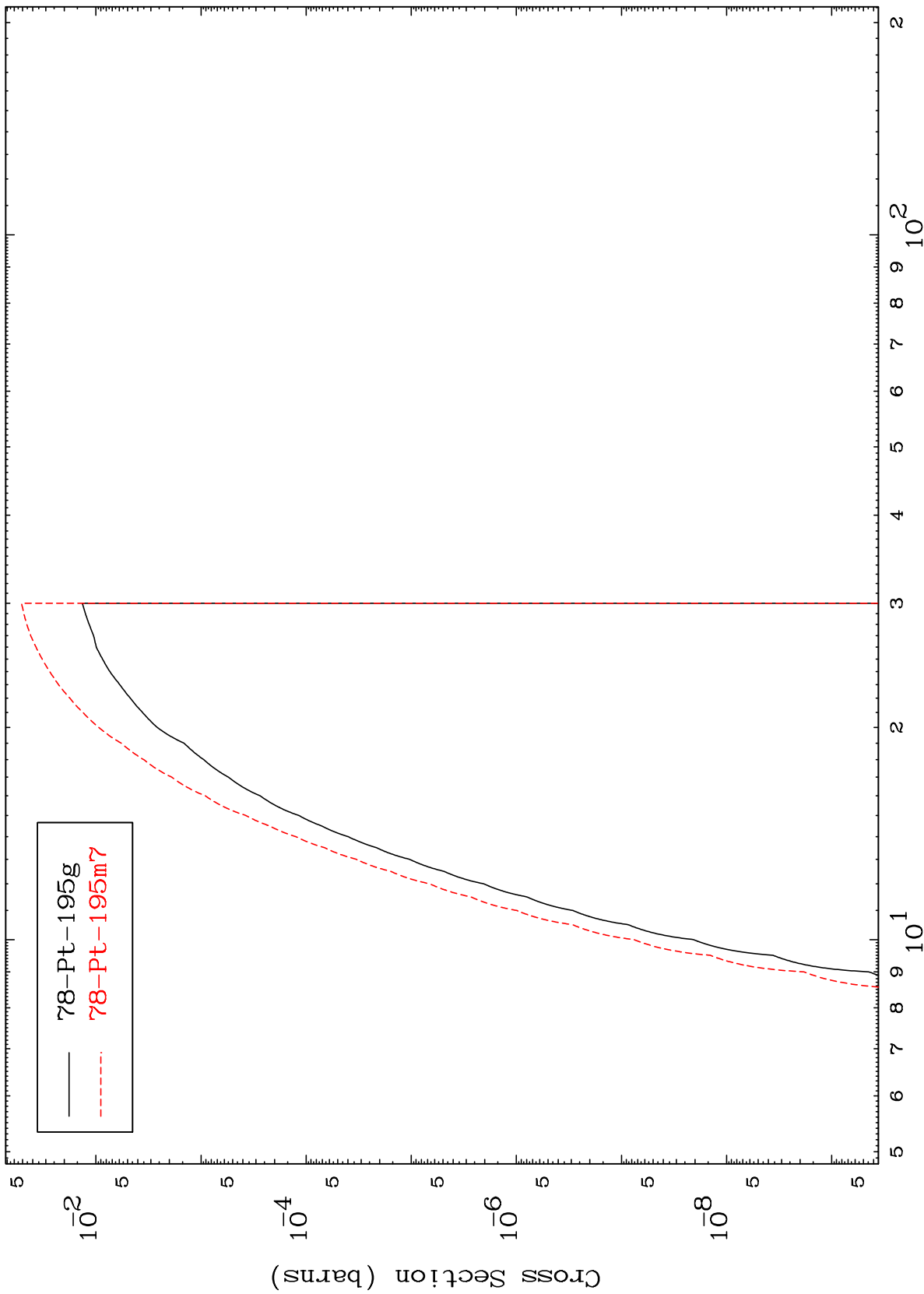


$^{77}\text{Ir-190g}$   
 $^{77}\text{Ir-190m2}$   
 $^{77}\text{Ir-190m37}$

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79-Au-196

(n,n') p  
Radionuclide Production Cross Section

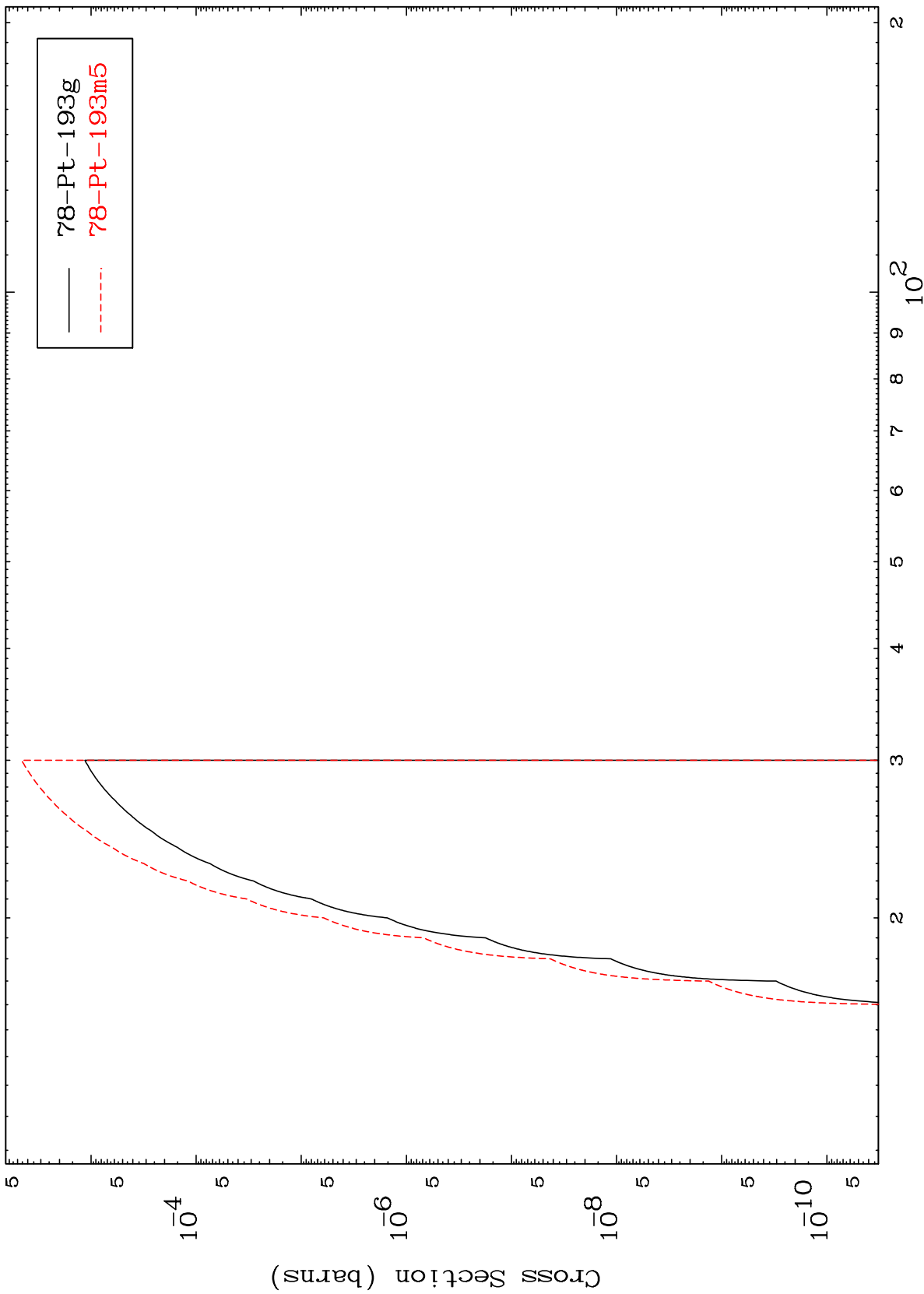


79-Au-196

Incident Energy (MeV)

27

Radionuclide Production Cross Section

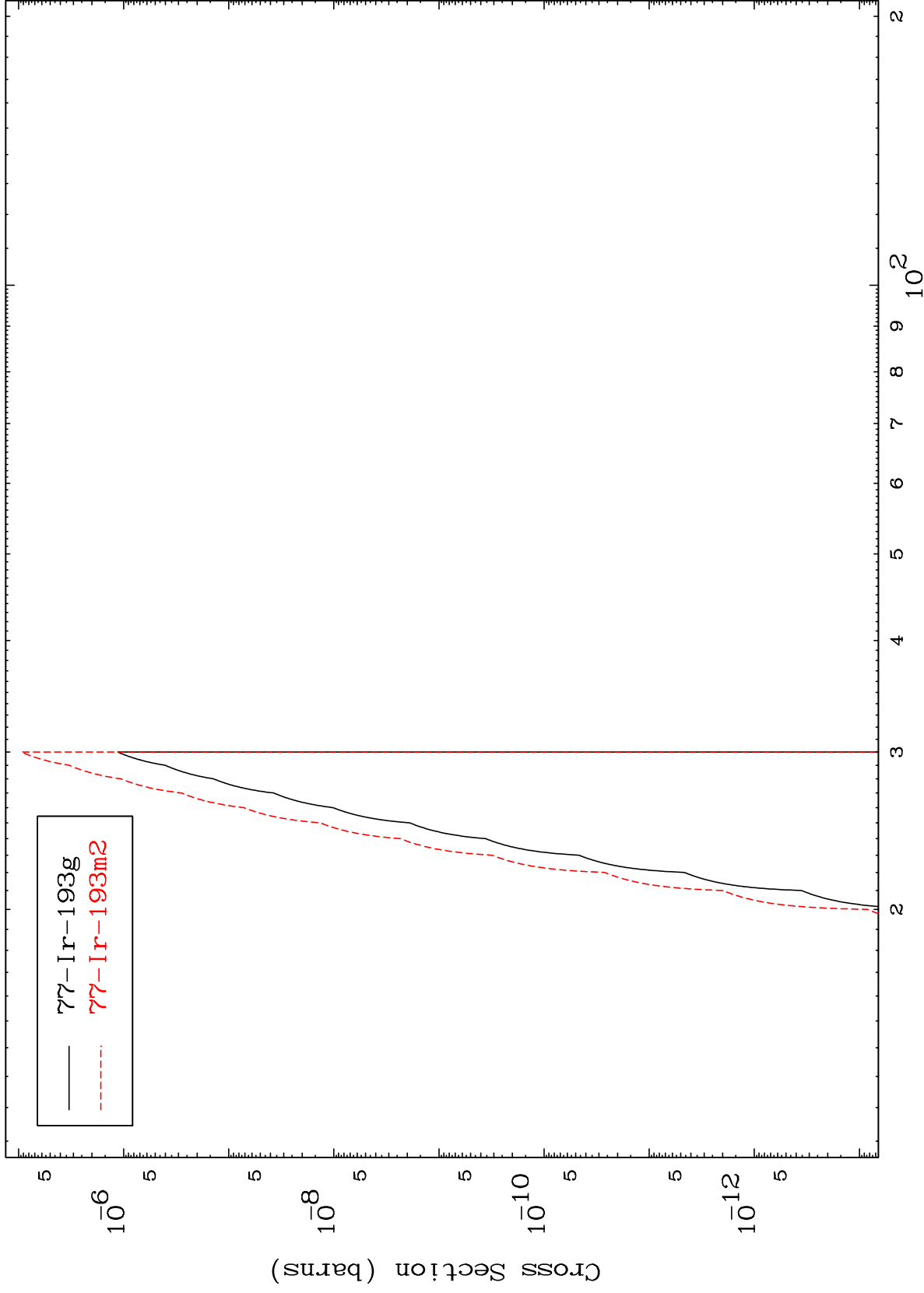


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(n,n') He-3

79-Au-196

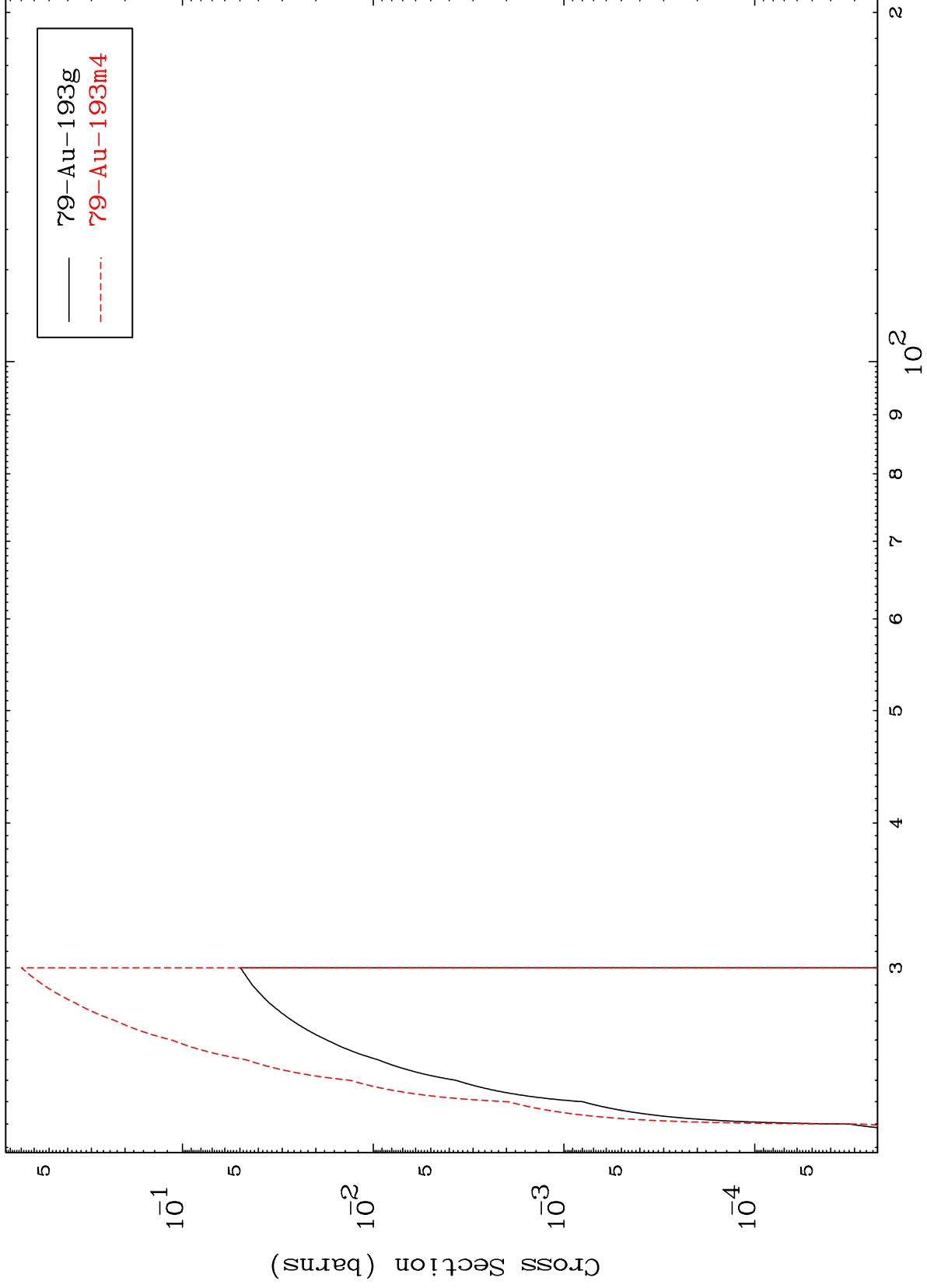
Radionuclide Production Cross Section



MAT 7924

79-Au-196

(n,4n)  
Radionuclide Production Cross Section

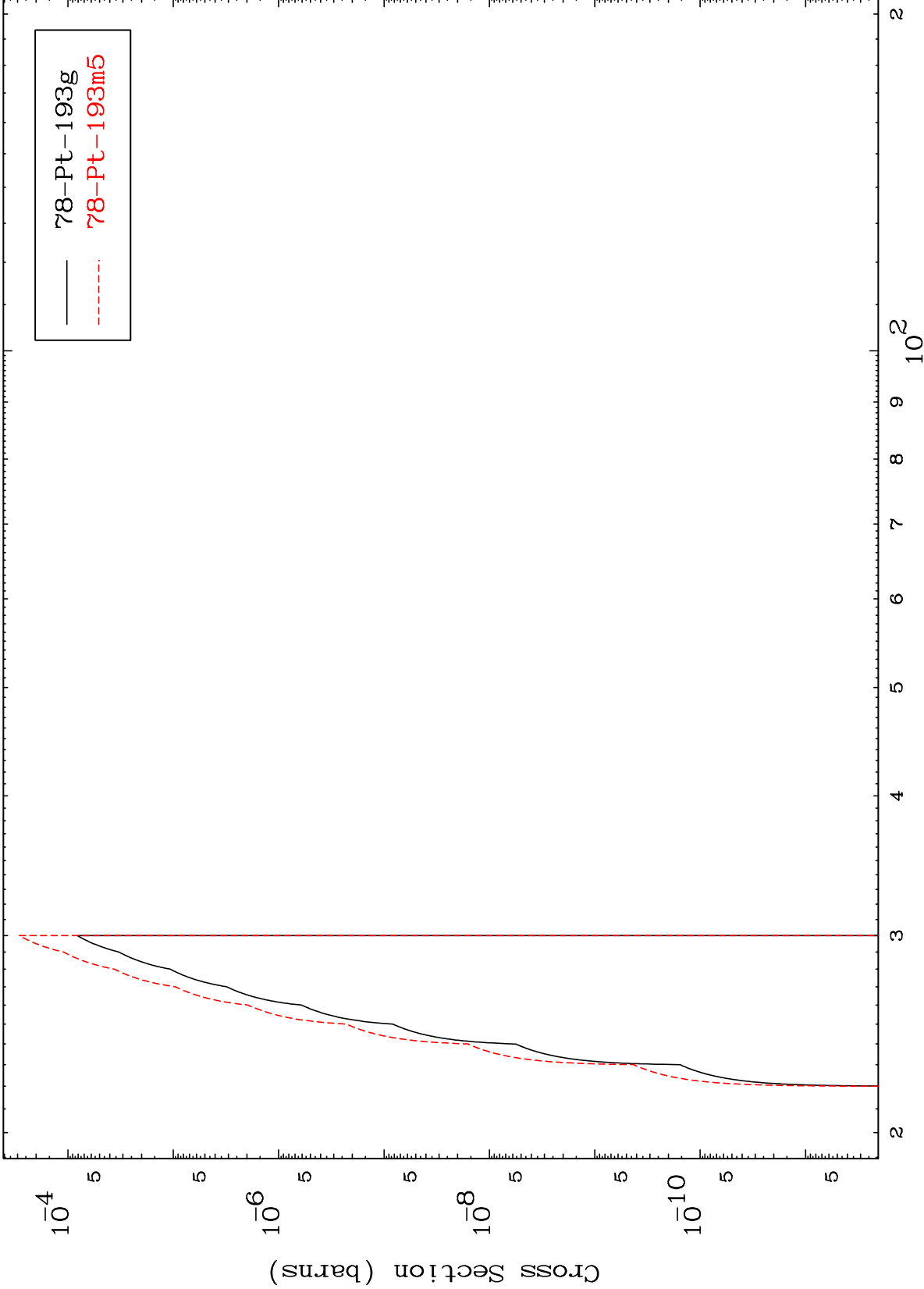


30

Incident Energy (MeV)

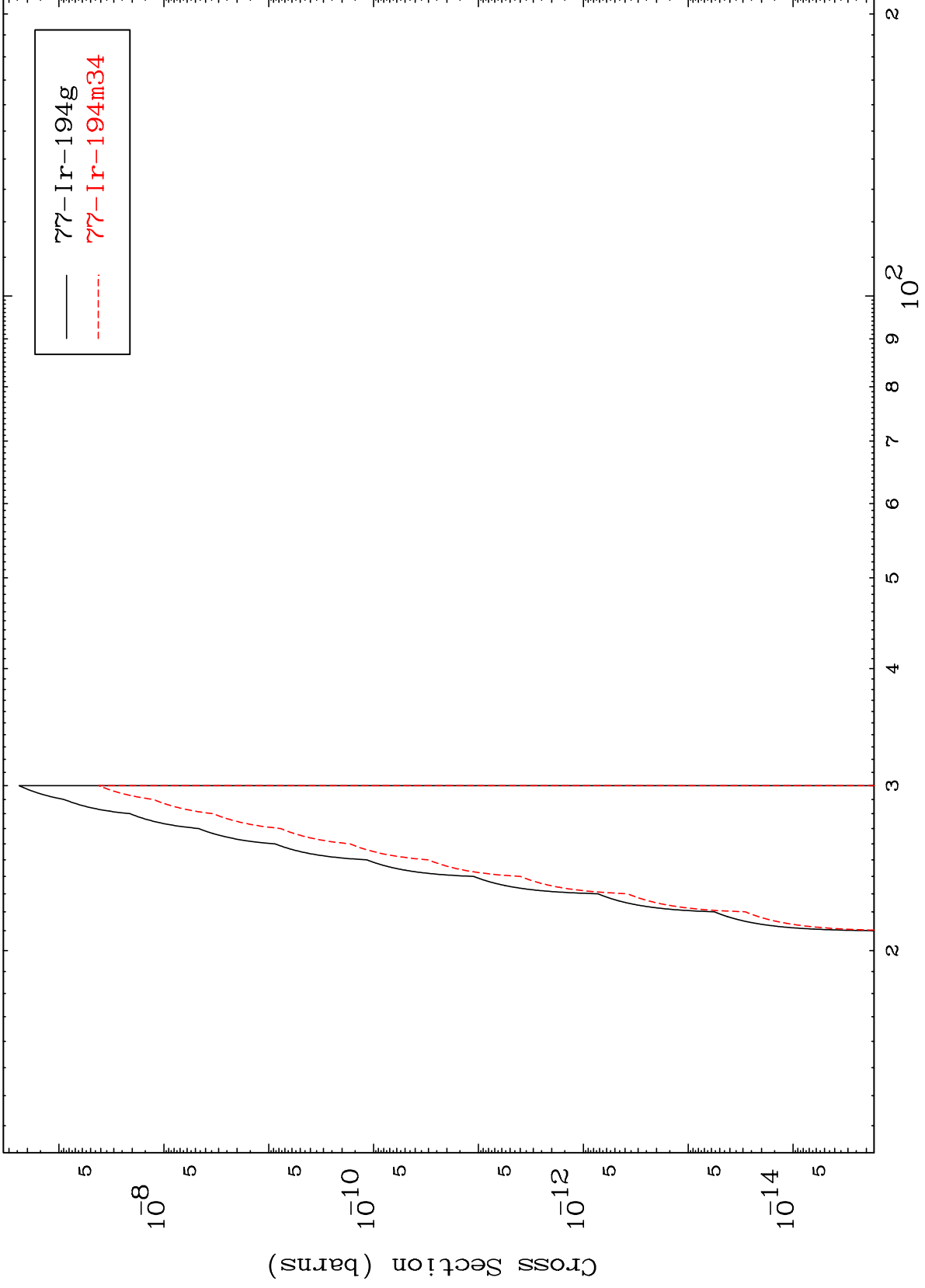
79-Au-196

Radionuclide Production Cross Section





Radionuclide Production Cross Section

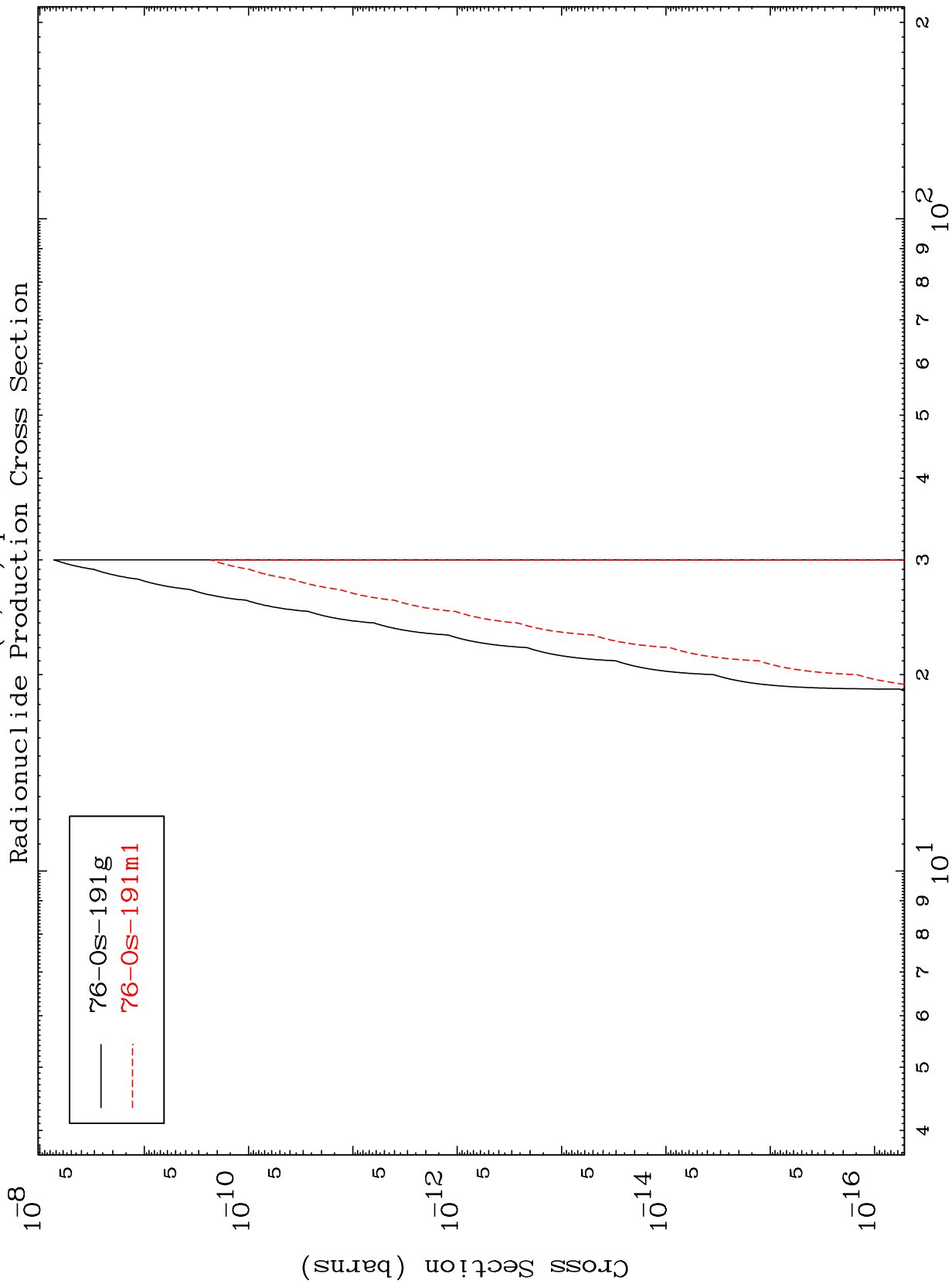


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(n,n') p  $\alpha$

79-Au-196

Radionuclide Production Cross Section



33

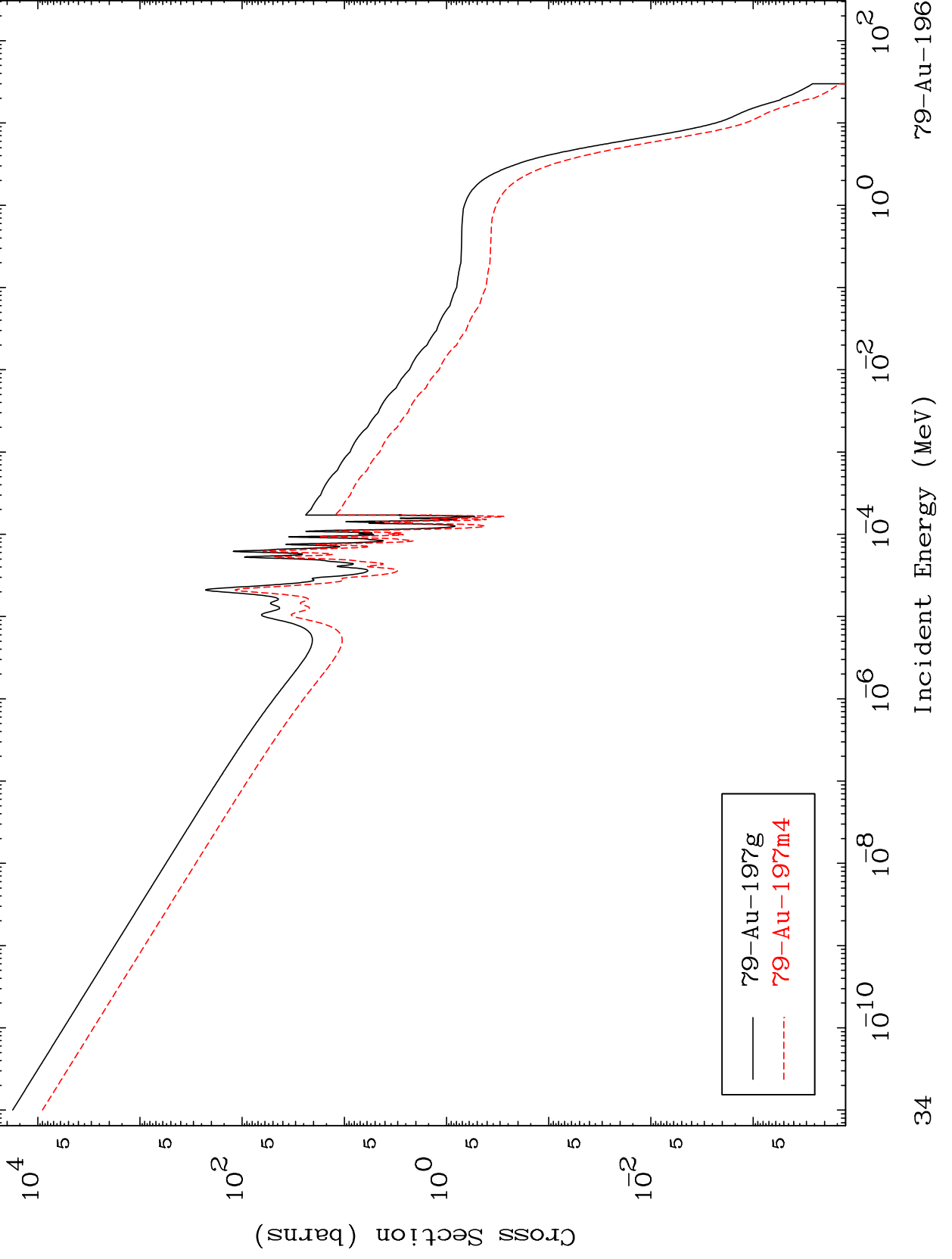
Incident Energy (MeV)

79-Au-196

MAT 7924

79-Au-196

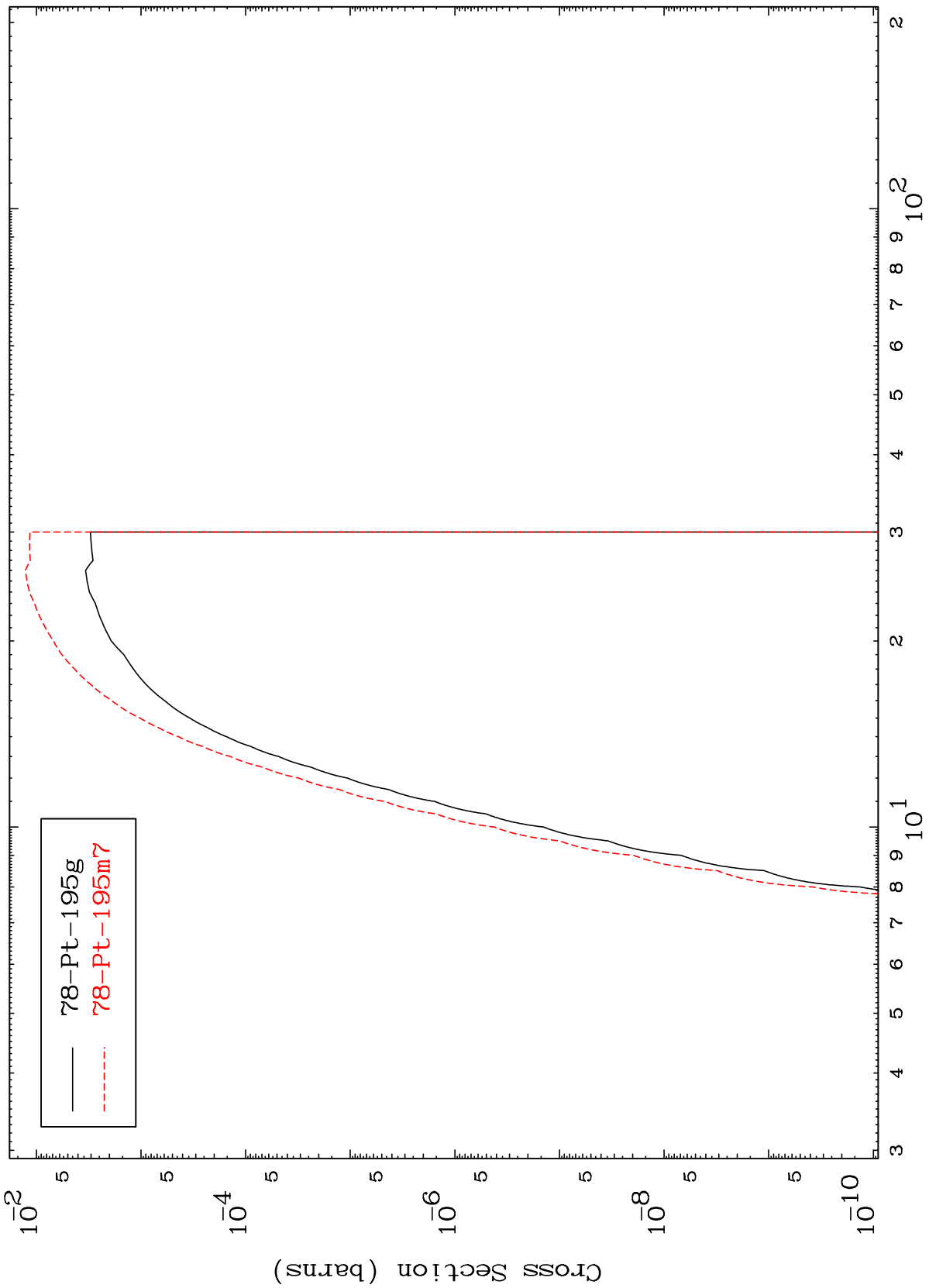
(n,γ)  
Radionuclide Production Cross Section



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79-Au-196

(n,d)  
Radionuclide Production Cross Section



35

Incident Energy (MeV)

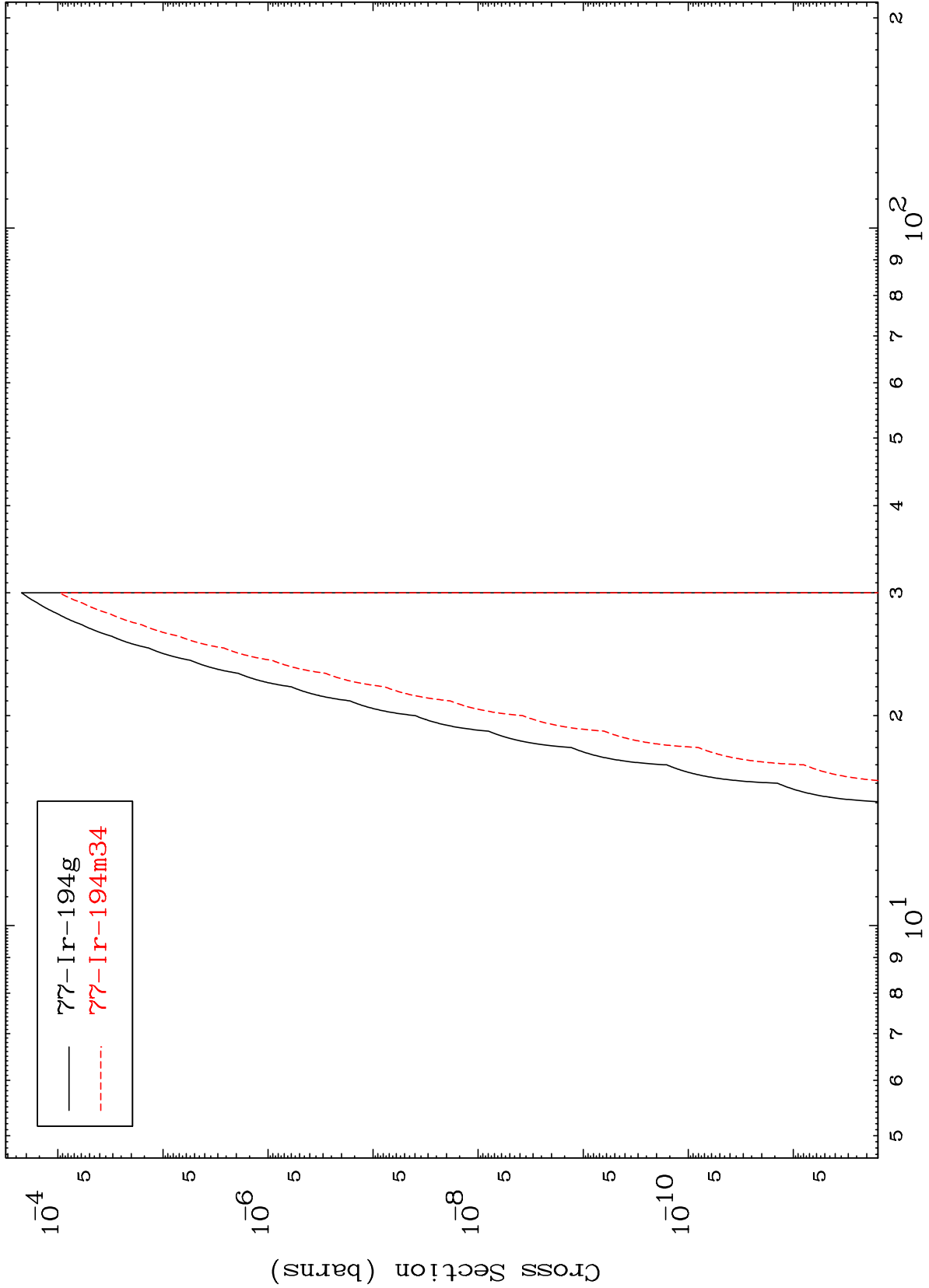
79-Au-196

MAT 7924

(n,He-3)

79-Au-196

Radionuclide Production Cross Section



36

Incident Energy (MeV)

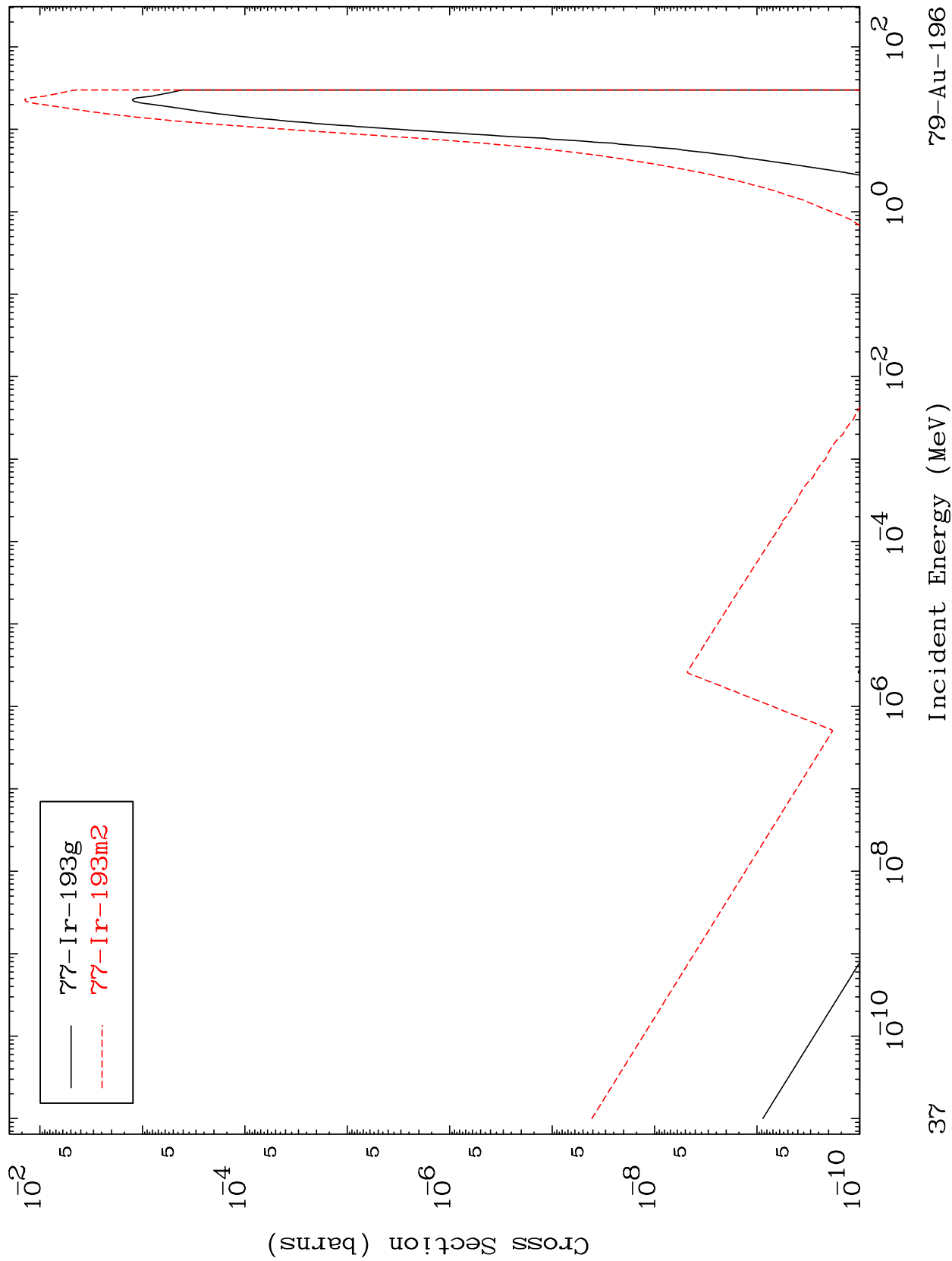
79-Au-196

MAT 7924

(n,  $\alpha$ )

79-Au-196

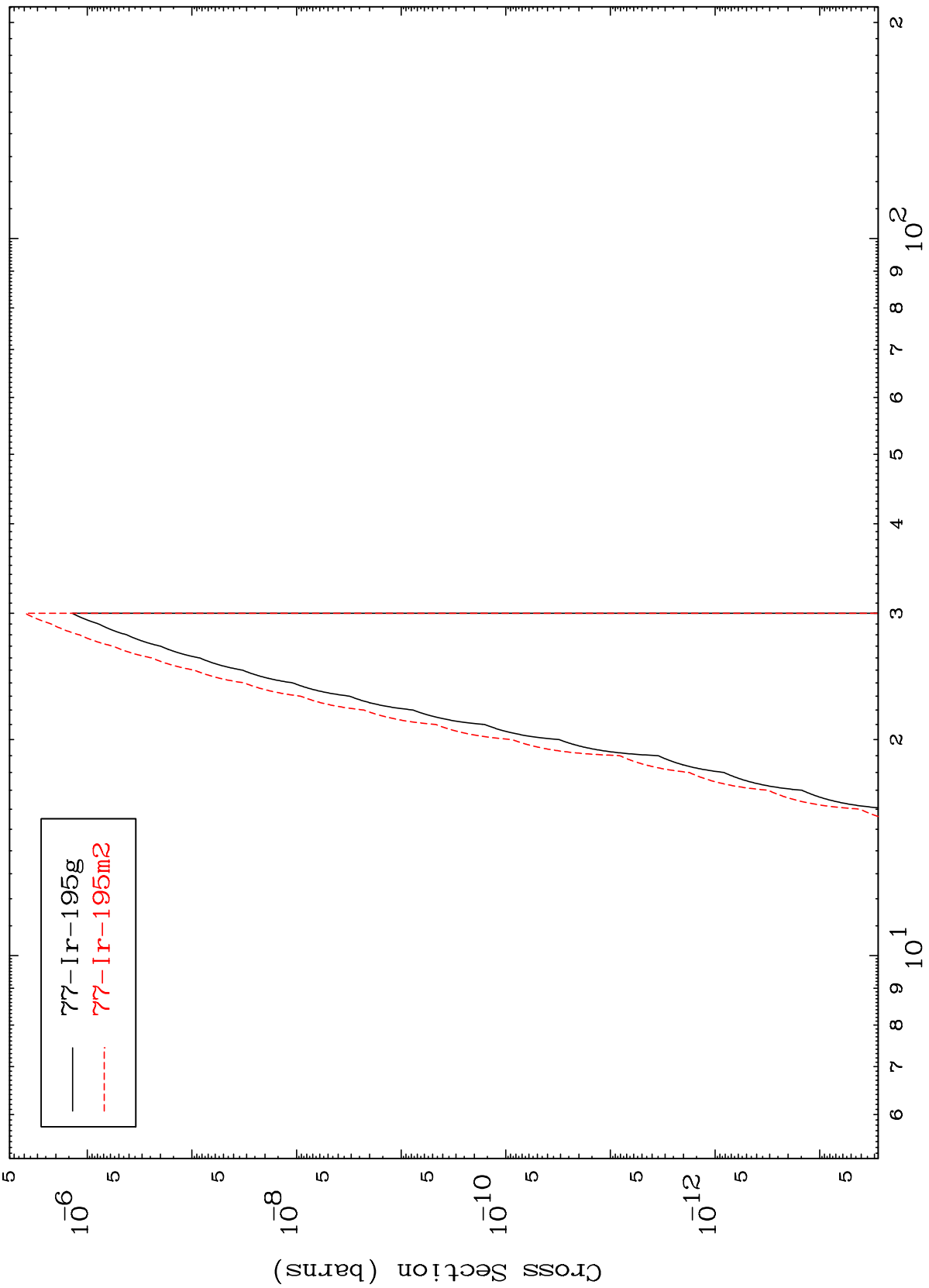
Radionuclide Production Cross Section



MAT 7924

79-Au-196

(n,2p)  
Radionuclide Production Cross Section



79-Au-196

Incident Energy (MeV)

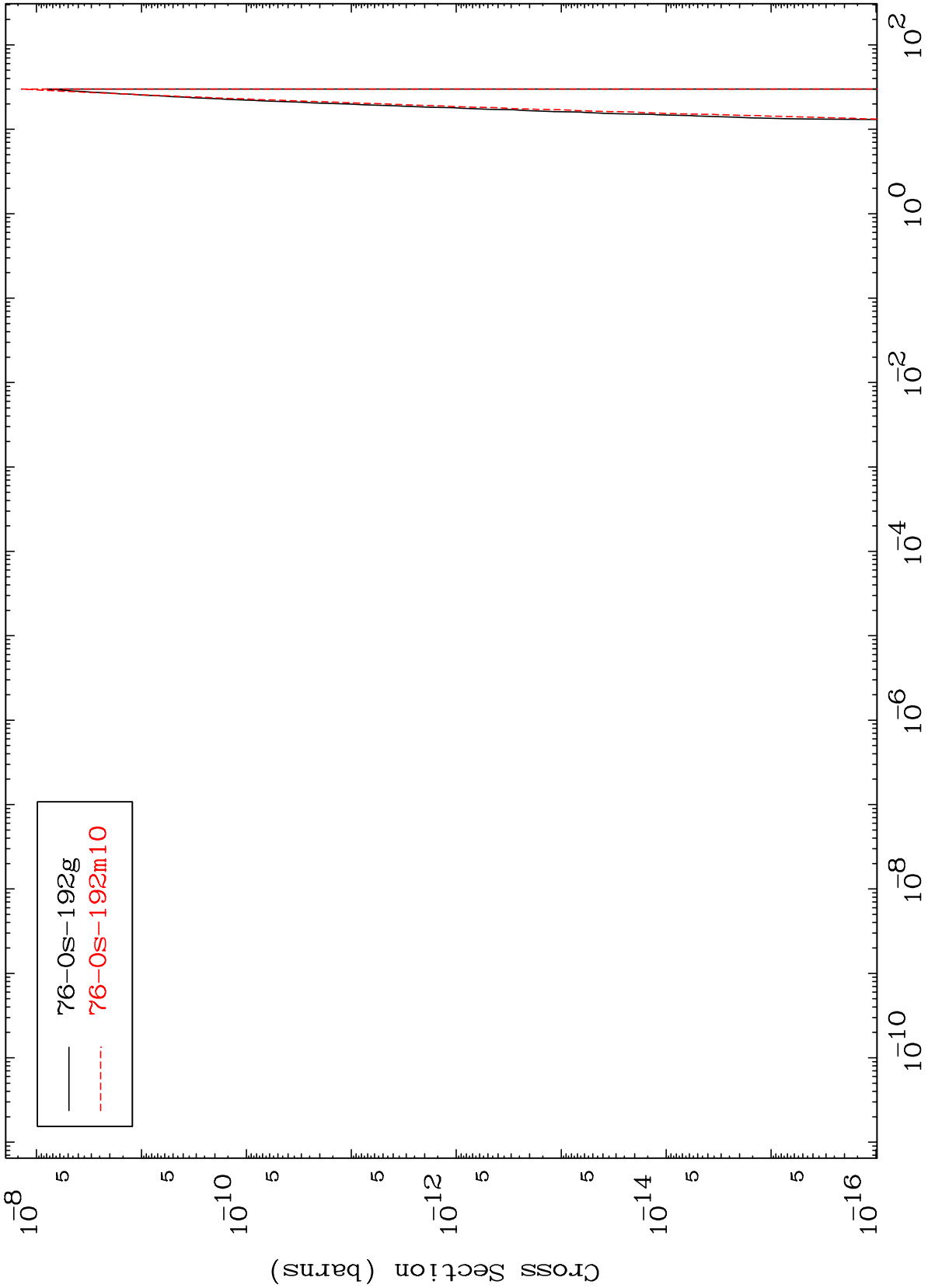
38

MAT 7924

(n,p)  $\alpha$

79-Au-196

Radionuclide Production Cross Section



76-0s-192g  
76-0s-192m10

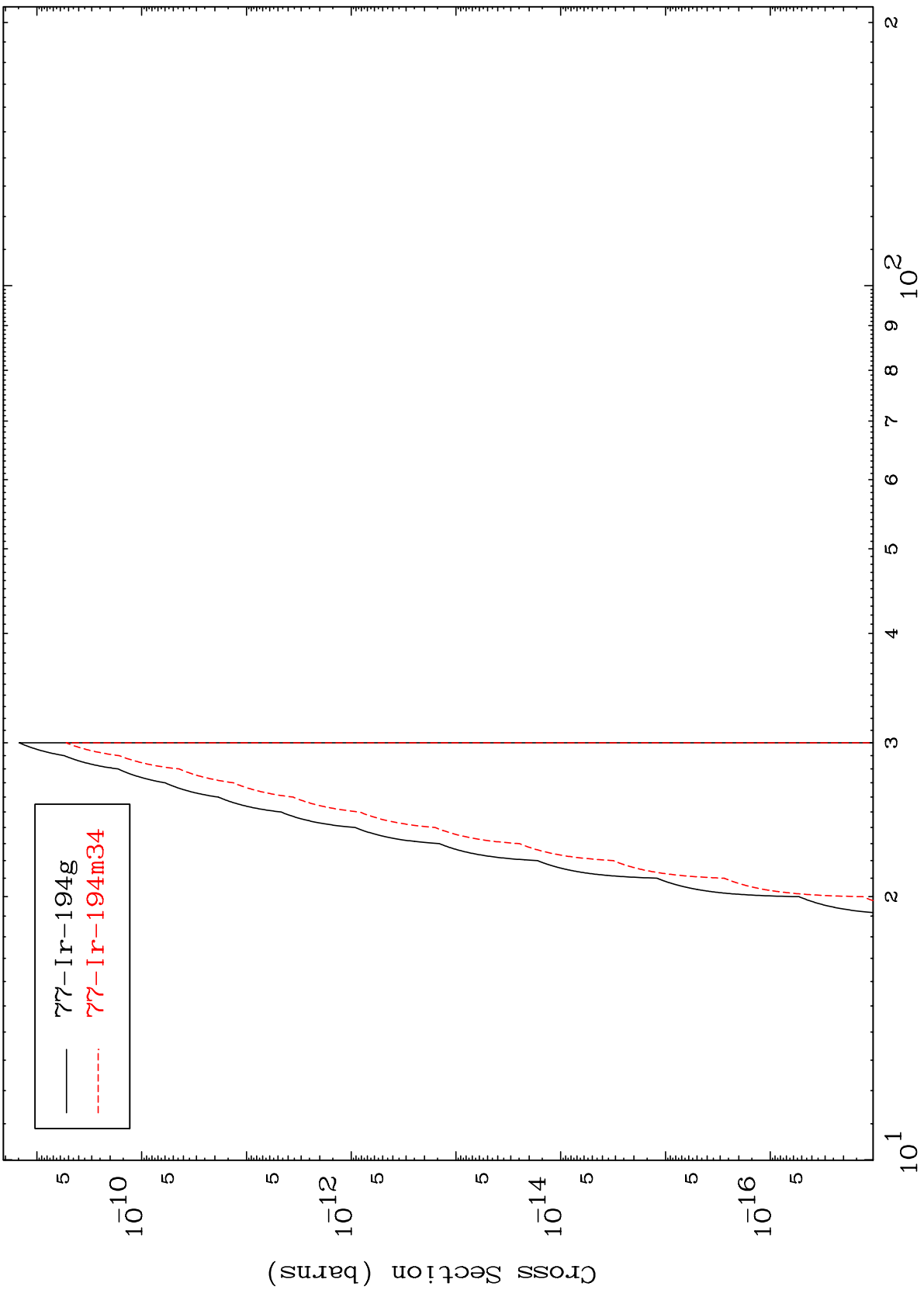


MAT 7924

(n,p) d

79-Au-196

Radionuclide Production Cross Section



—  $^{77}\text{Ir-194g}$   
- - -  $^{77}\text{Ir-194m34}$

Incident Energy (MeV)

79-Au-196

40