

Program EVALPLOT  
(Version 2017-1)

by

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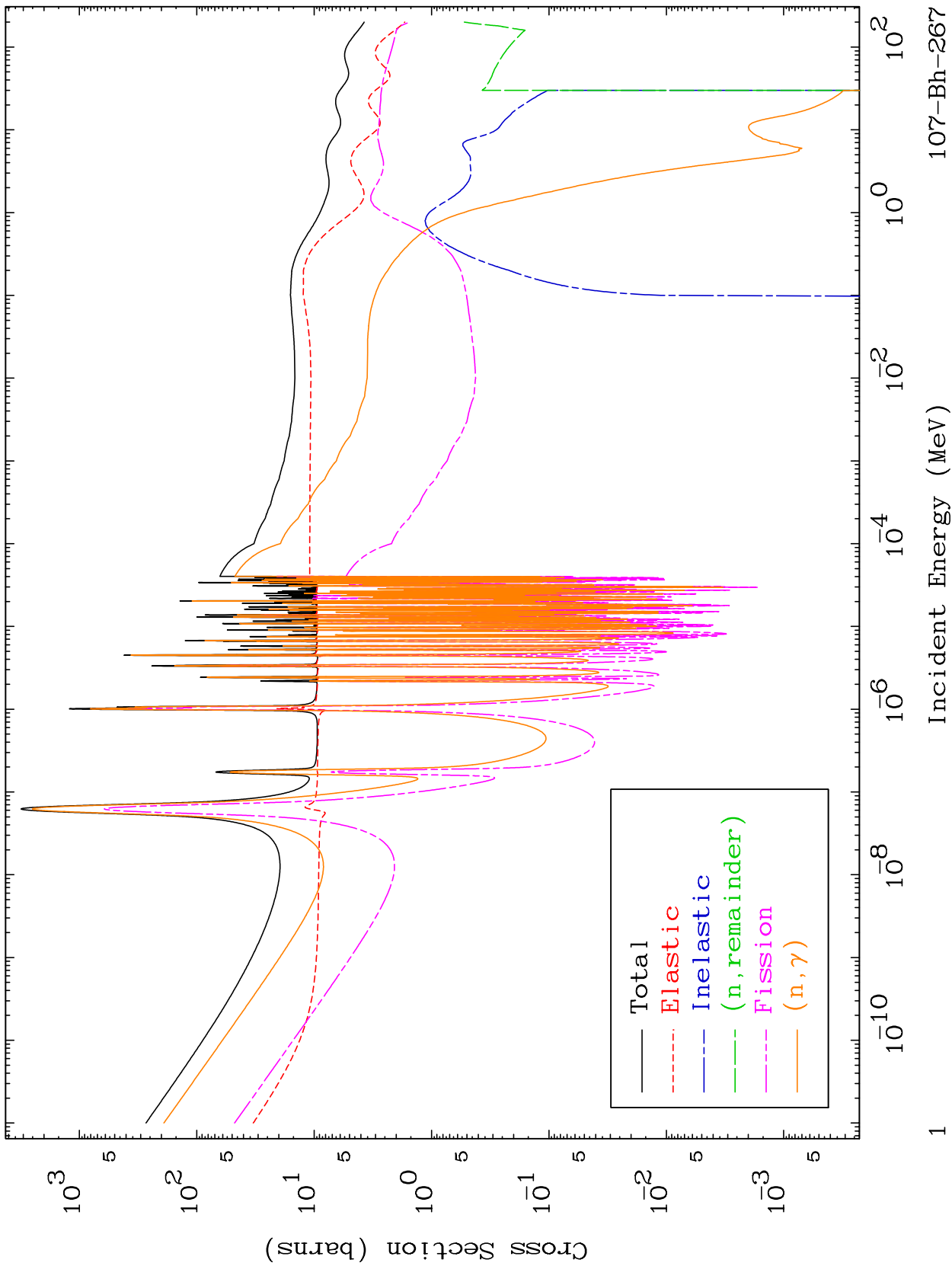
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 767

Major  
293 Kelvin Cross Sections

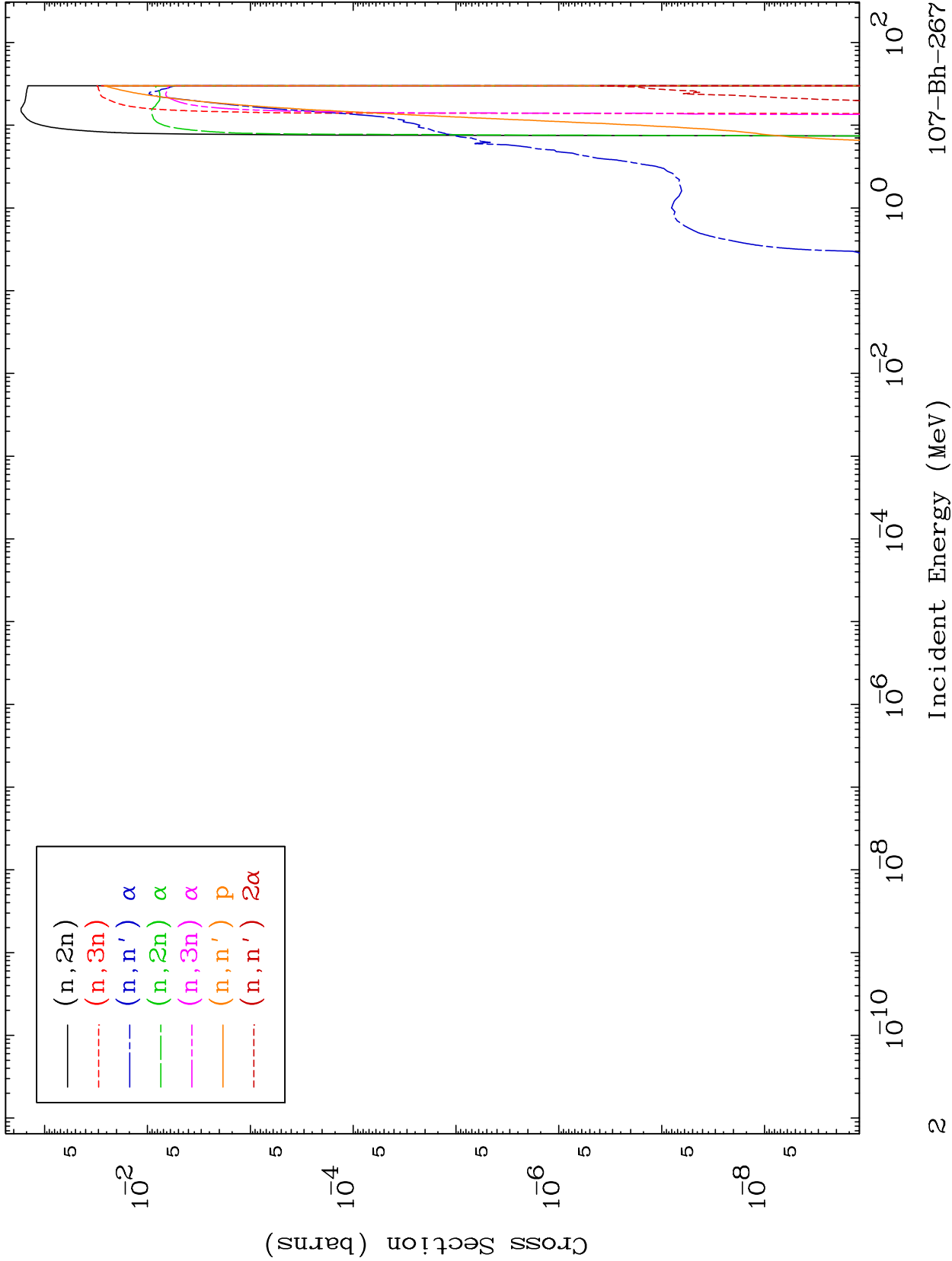
107-Bh-267



MAT 767

Neutron Production  
293 Kelvin Cross Sections

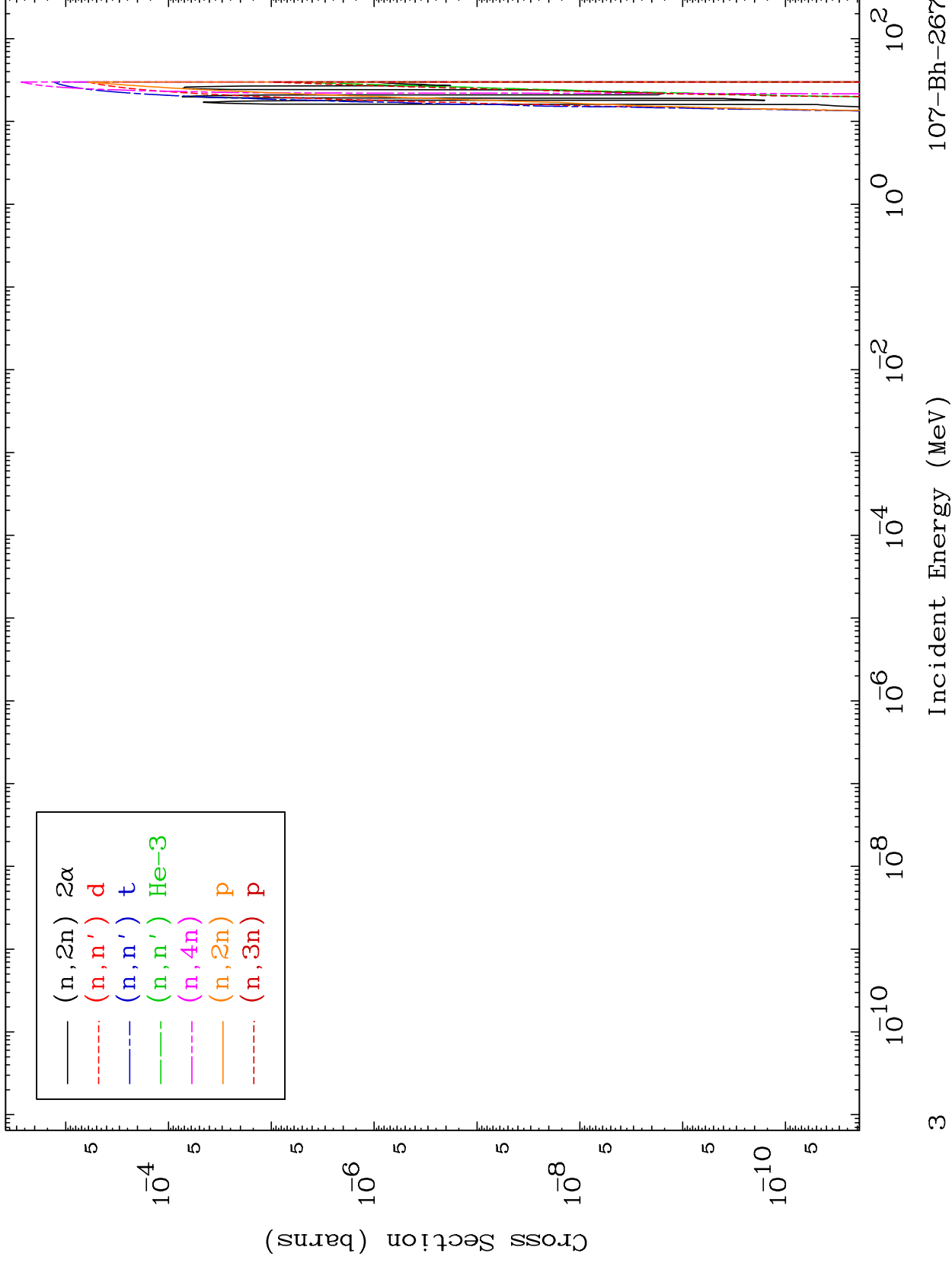
107-Bh-267



MAT 767

Neutron Production  
293 Kelvin Cross Sections

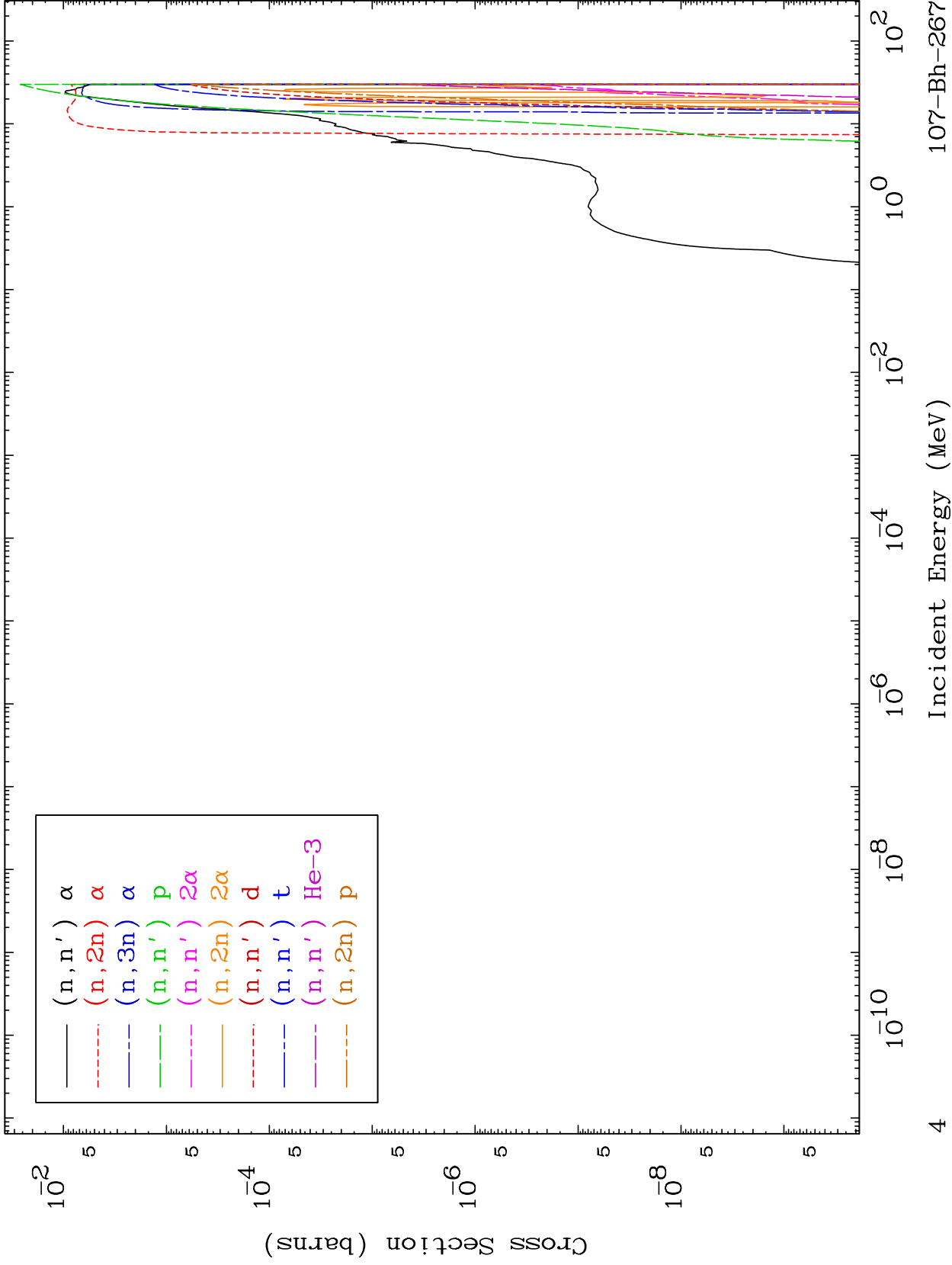
107-Bh-267



MAT 767

Charged Particle  
293 Kelvin Cross Sections

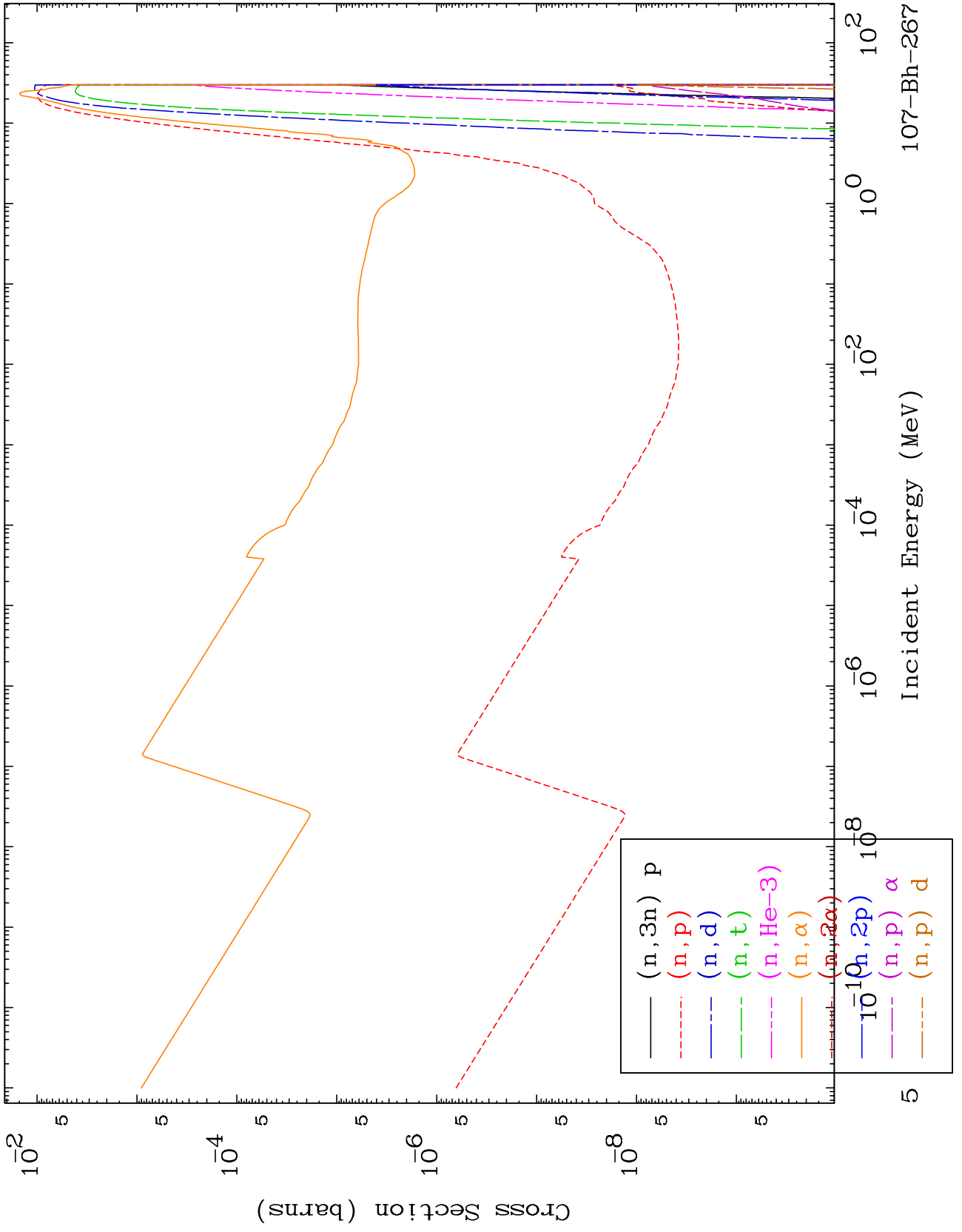
107-Bh-267

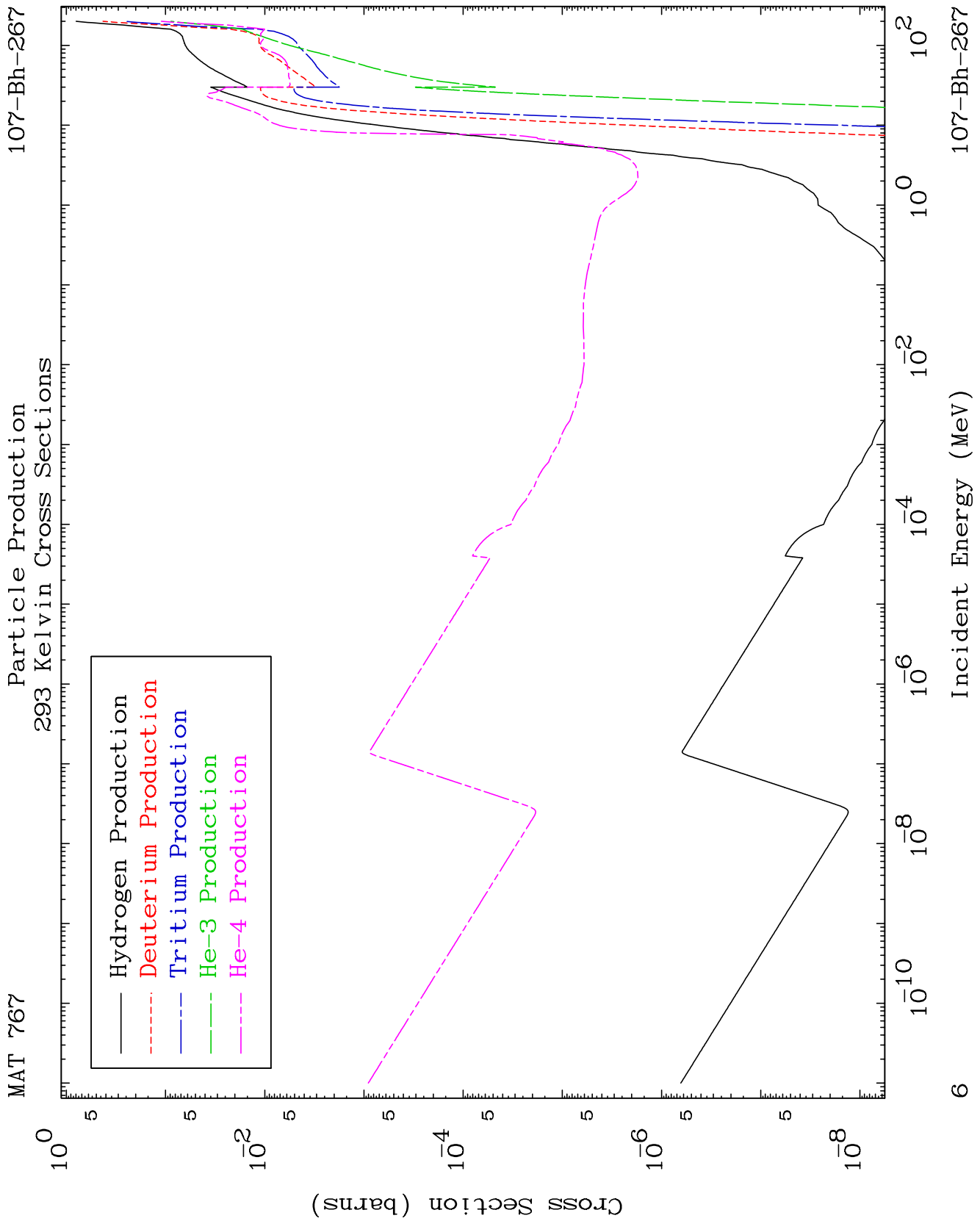


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Charged Particle  
293 Kelvin Cross Sections

107-Bh-267





MAT 767

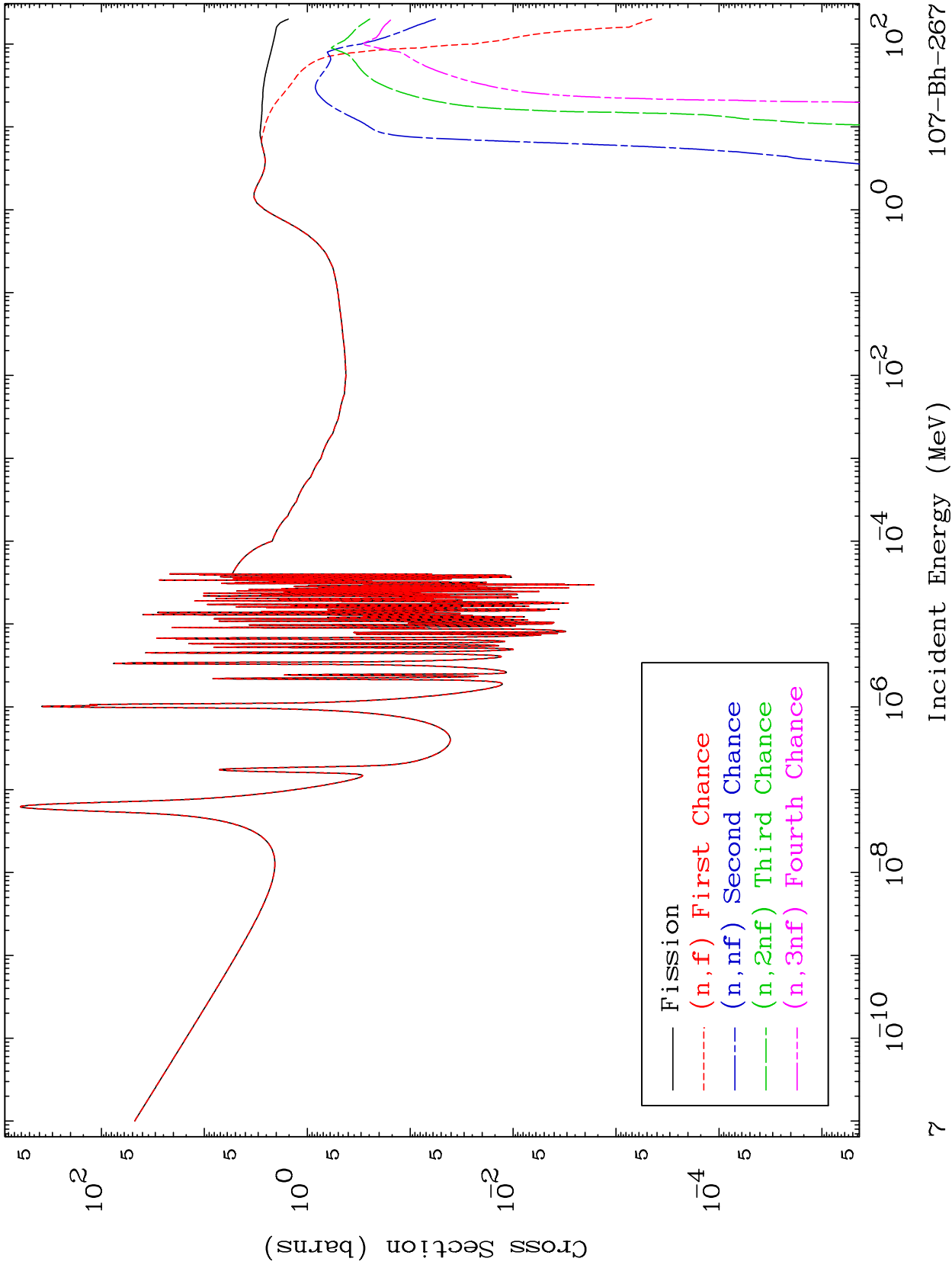
293 Kelvin Cross Sections

107-Bh-267

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Fission  
293 Kelvin Cross Sections

107-Bh-267

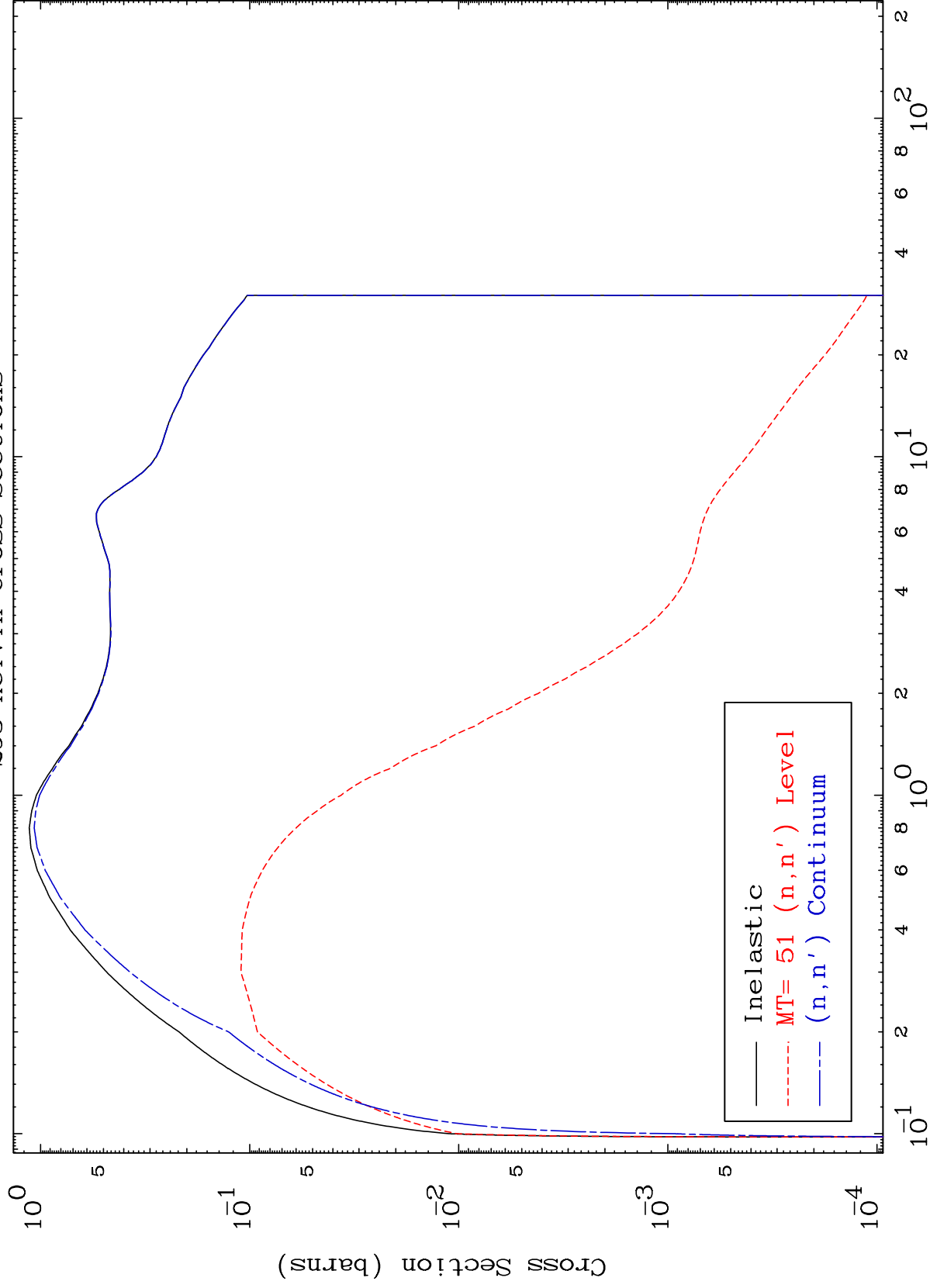




MAT 767

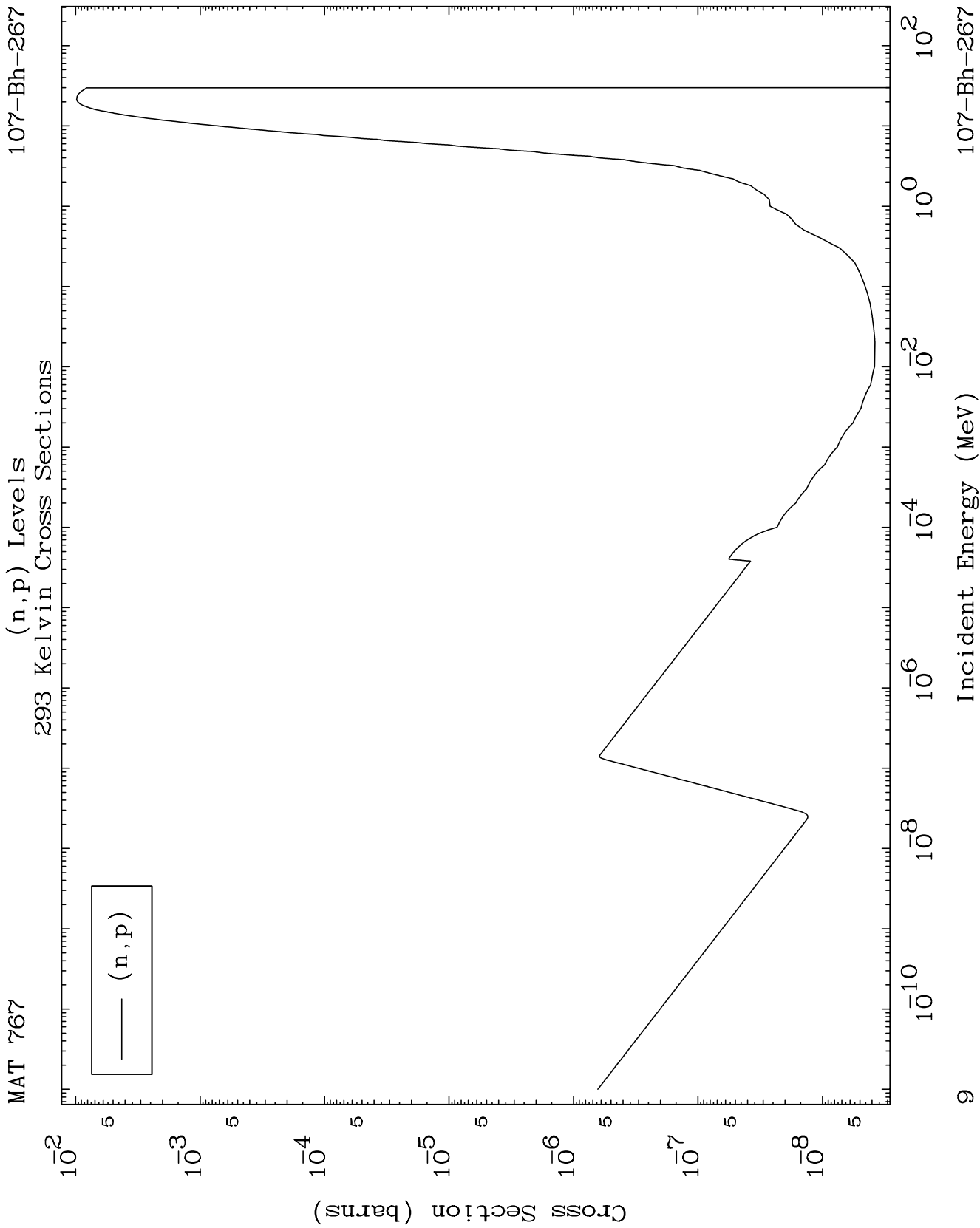
293 Kelvin Cross Sections  
(n,n') Level

107-Bh-267



Incident Energy (MeV)

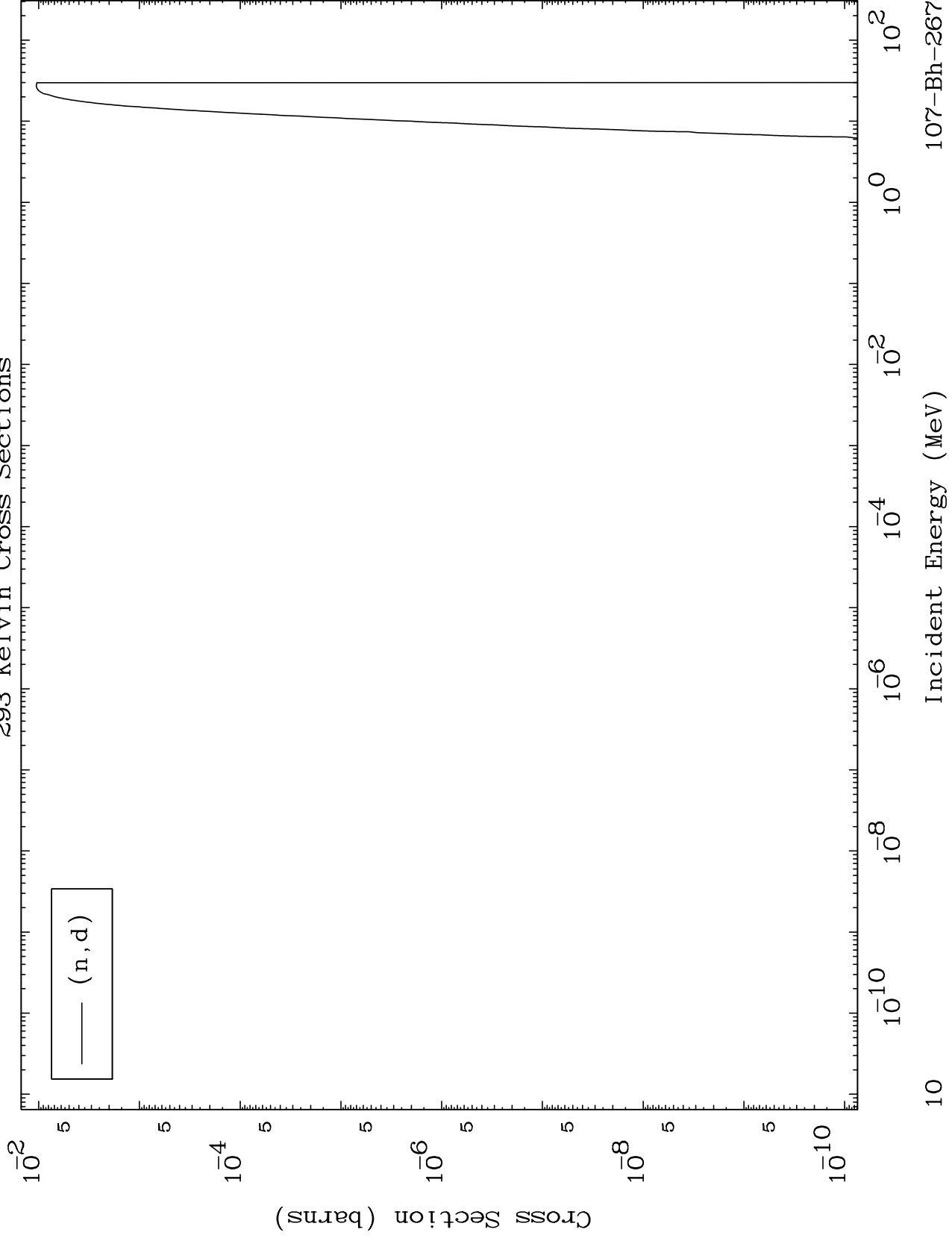
107-Bh-267



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(n,d) Levels  
293 Kelvin Cross Sections

107-Bh-267



10

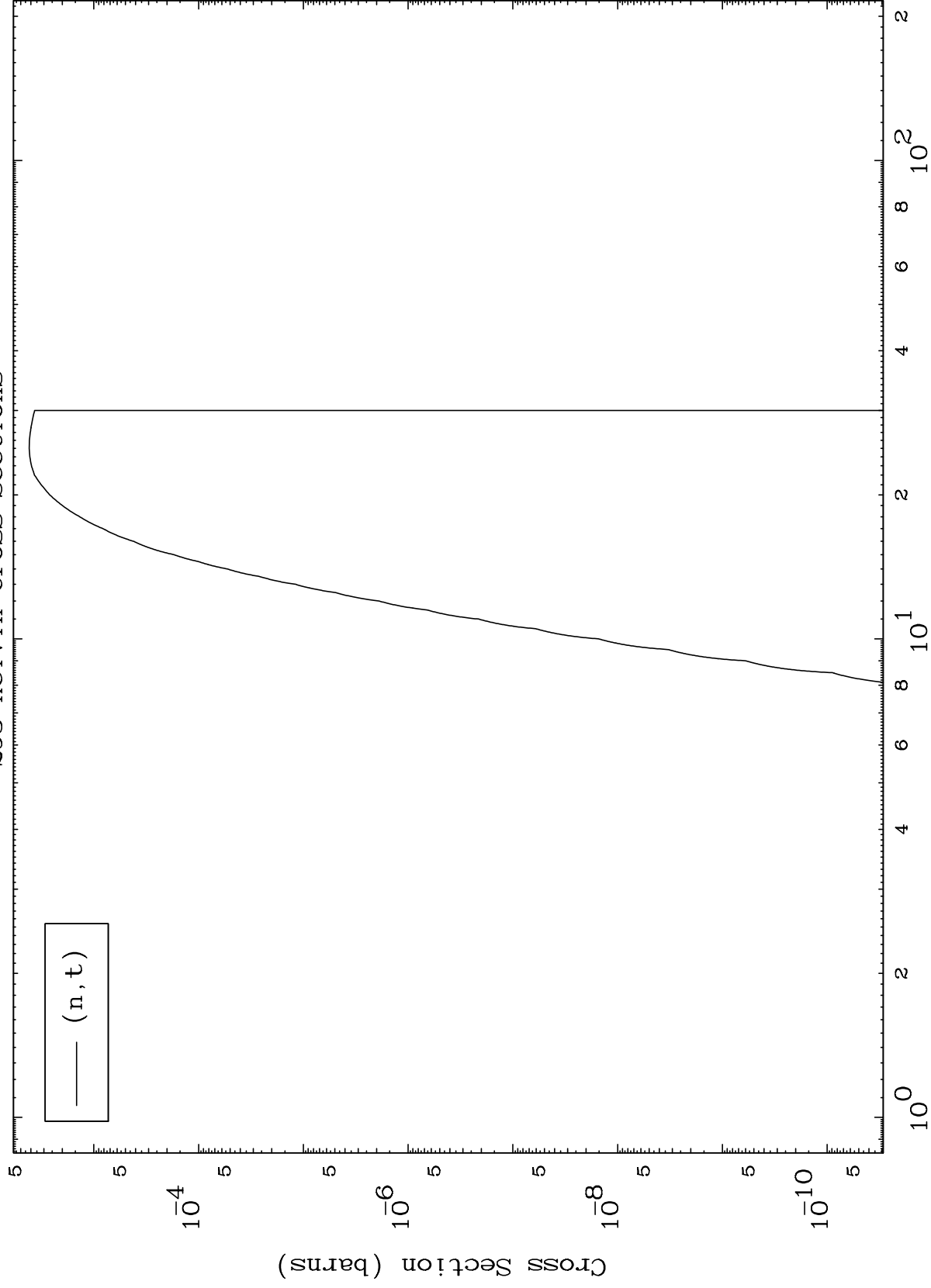
Incident Energy (MeV)

107-Bh-267

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(n,t) Levels  
293 Kelvin Cross Sections

107-Bh-267



11

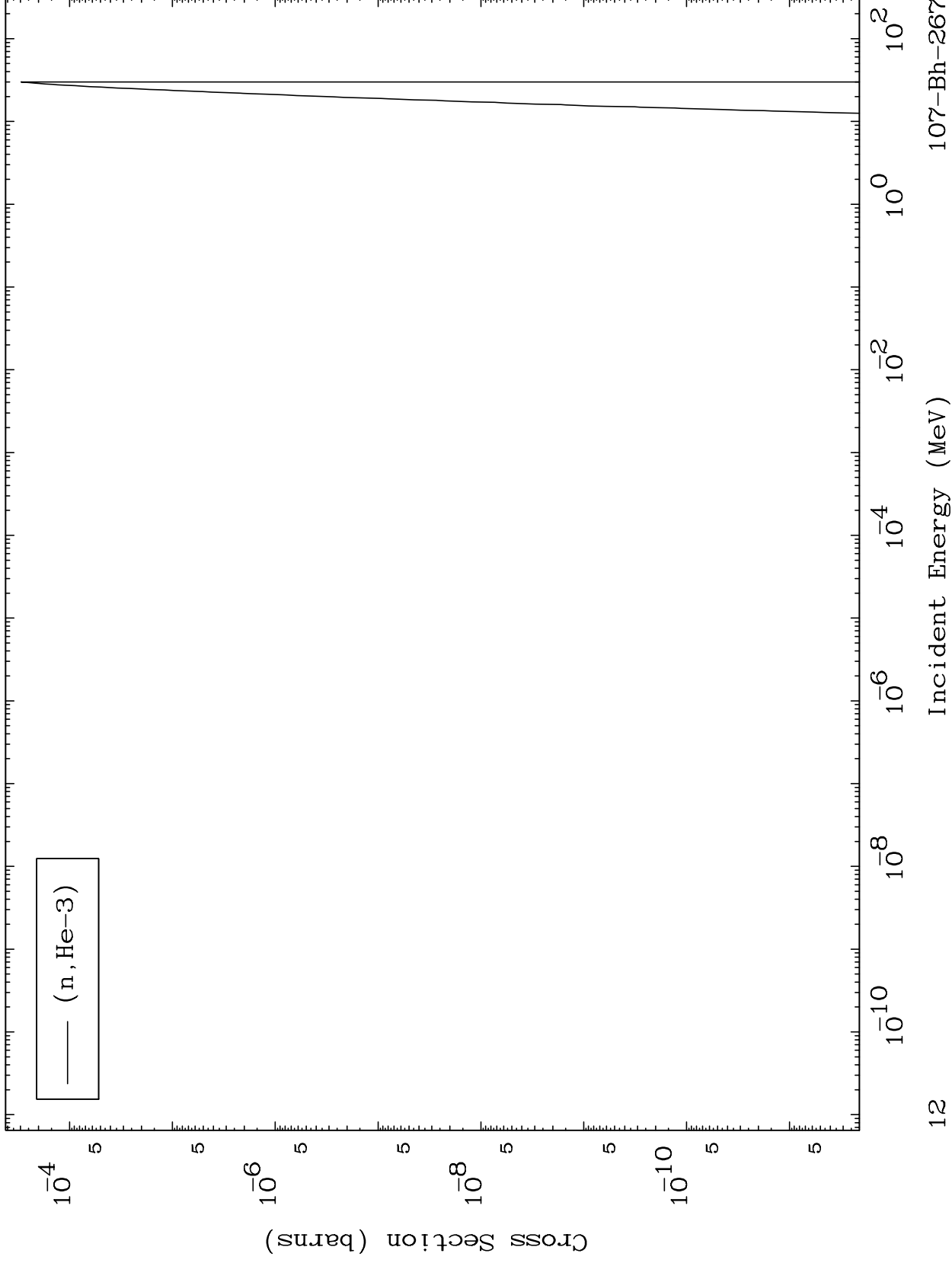
Incident Energy (MeV)

107-Bh-267

MAT 767

(n,He3) Levels  
293 Kelvin Cross Sections

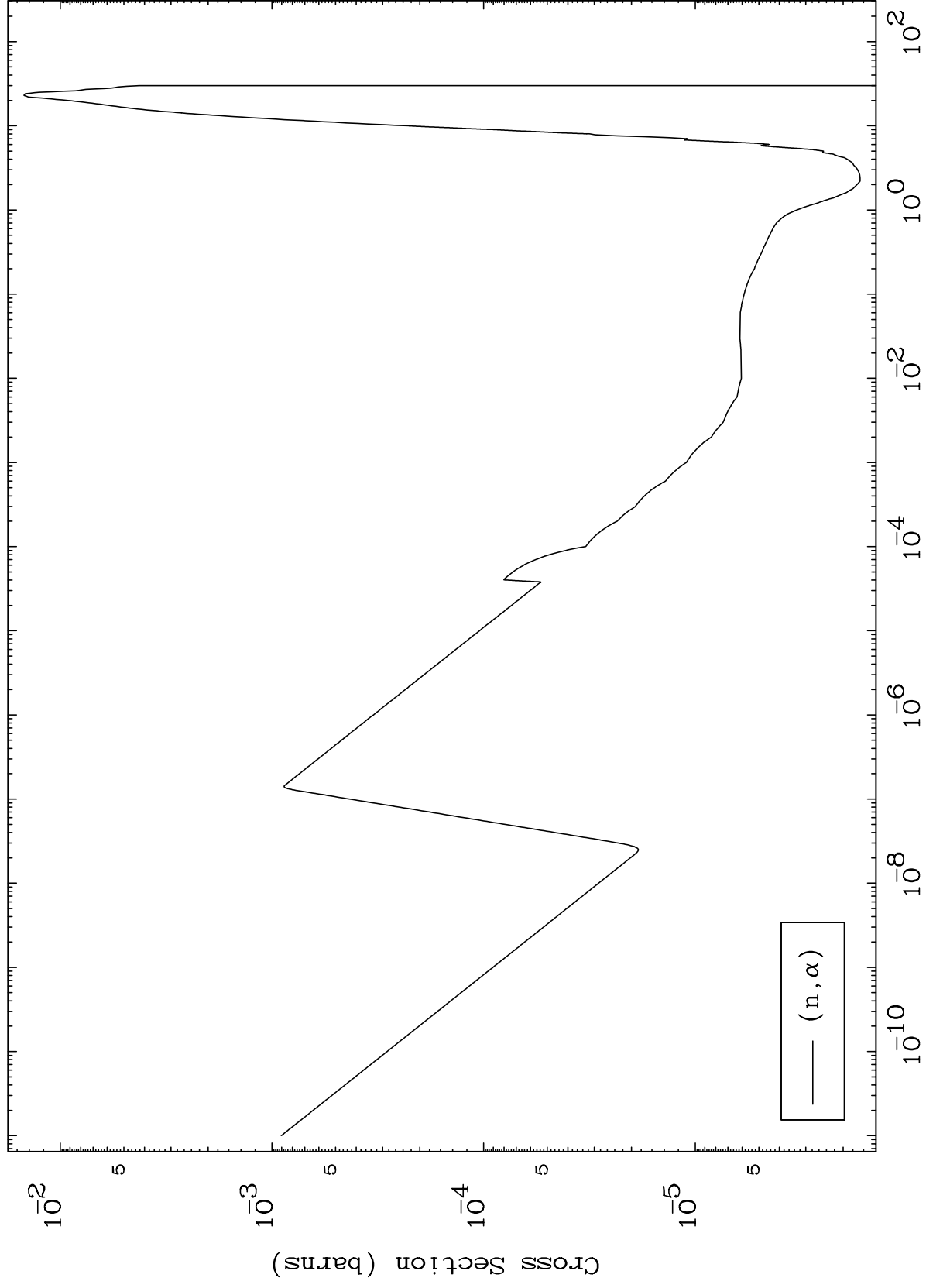
107-Bh-267



MAT 767

(n,α) Levels  
293 Kelvin Cross Sections

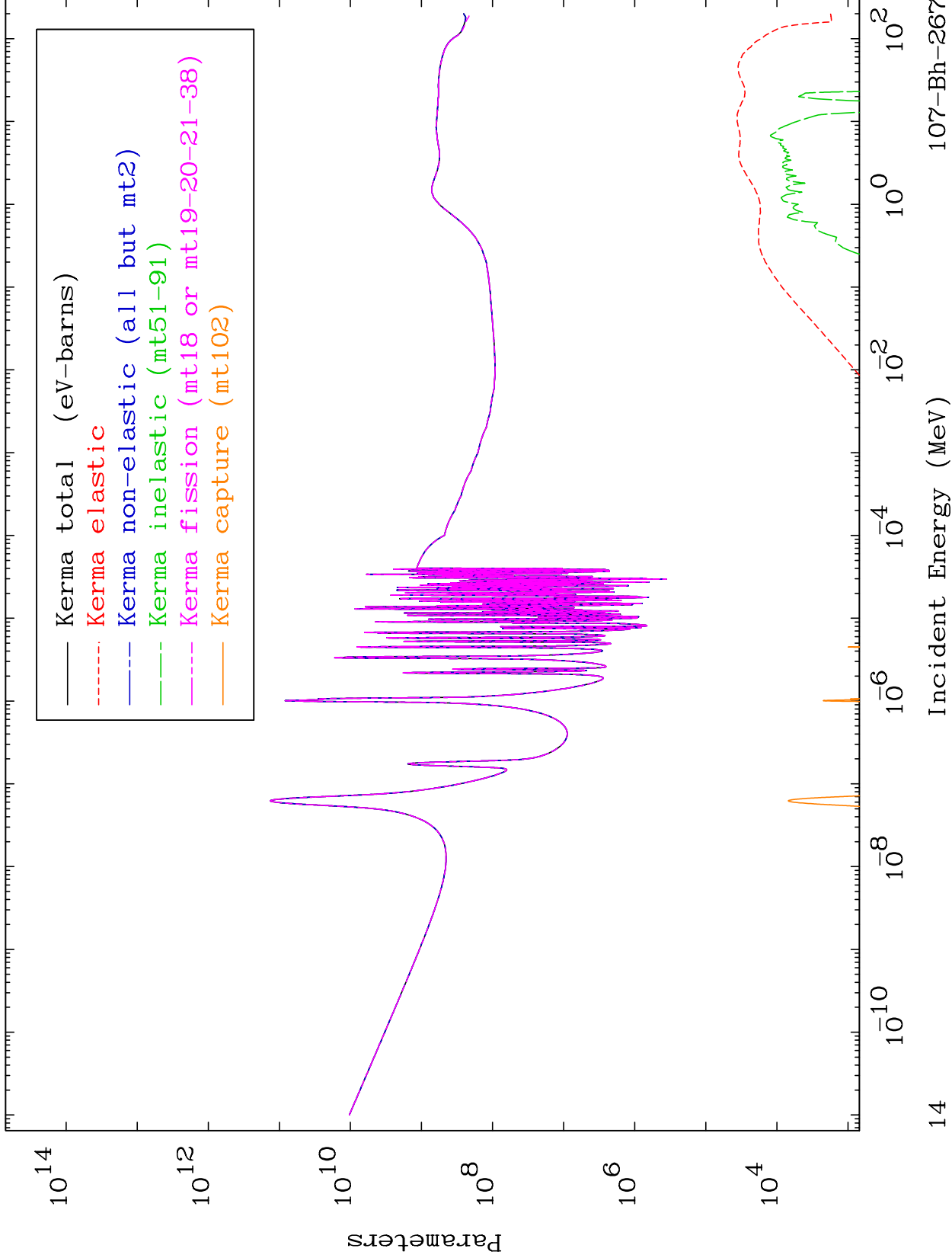
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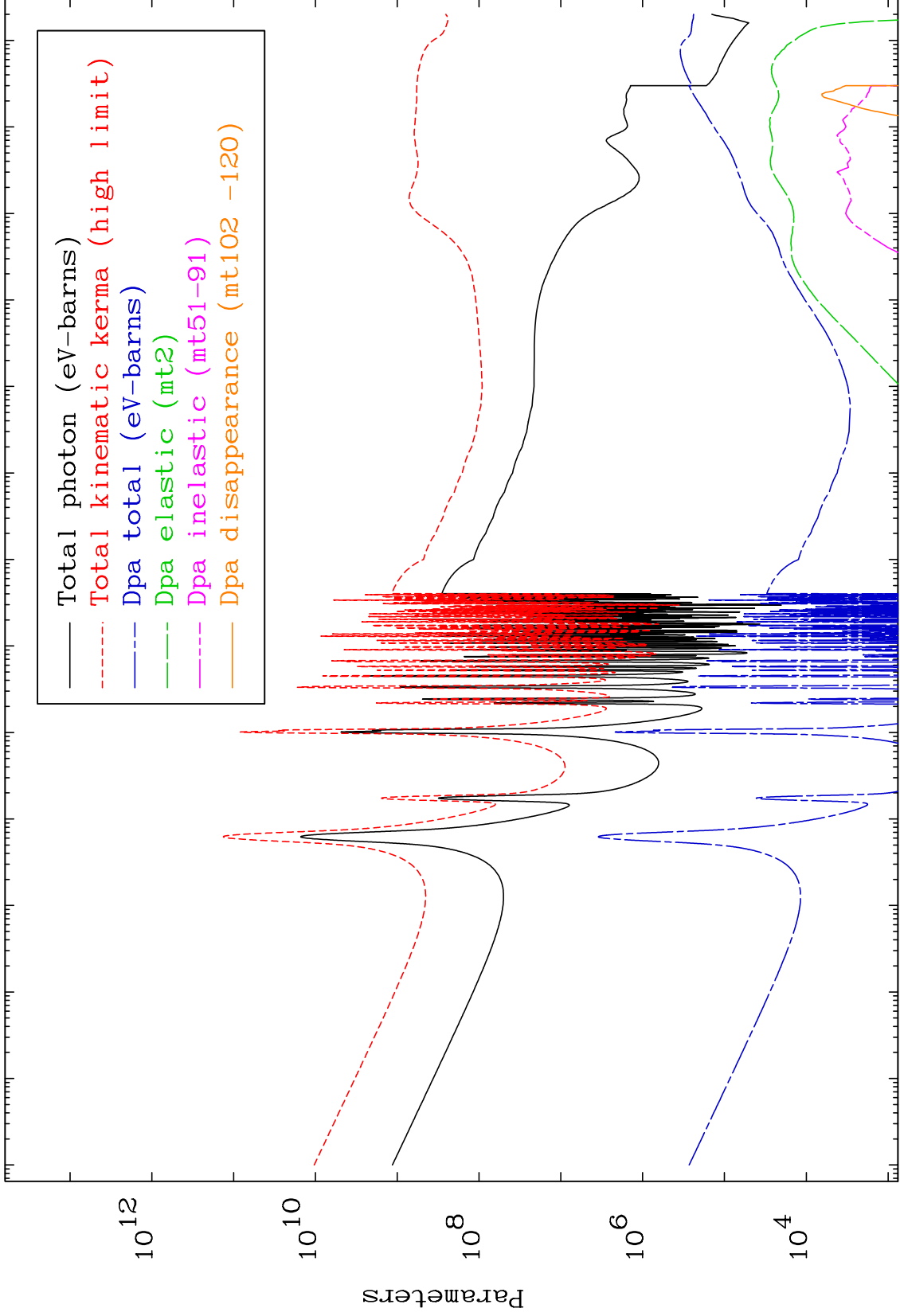


13

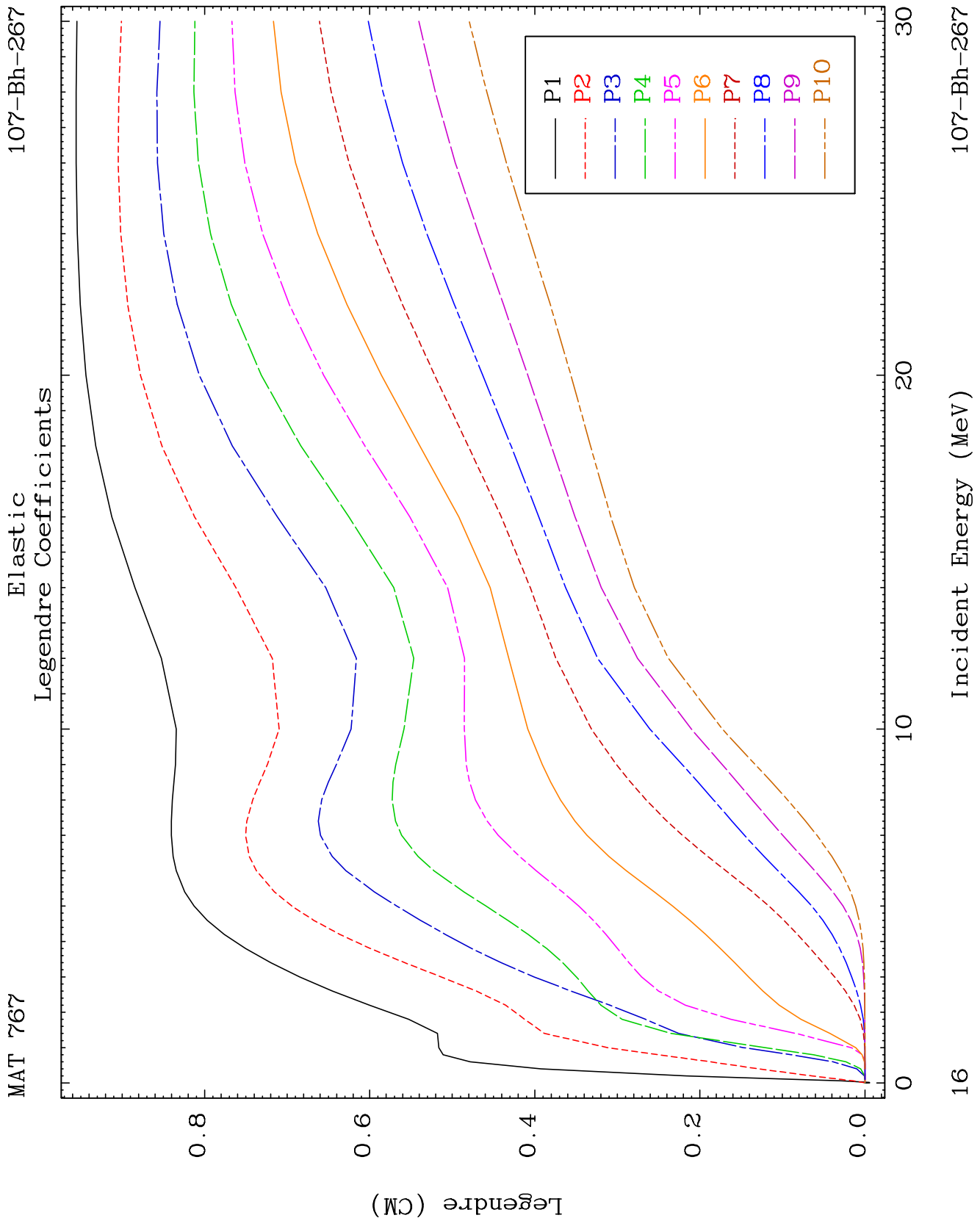
Incident Energy (MeV)

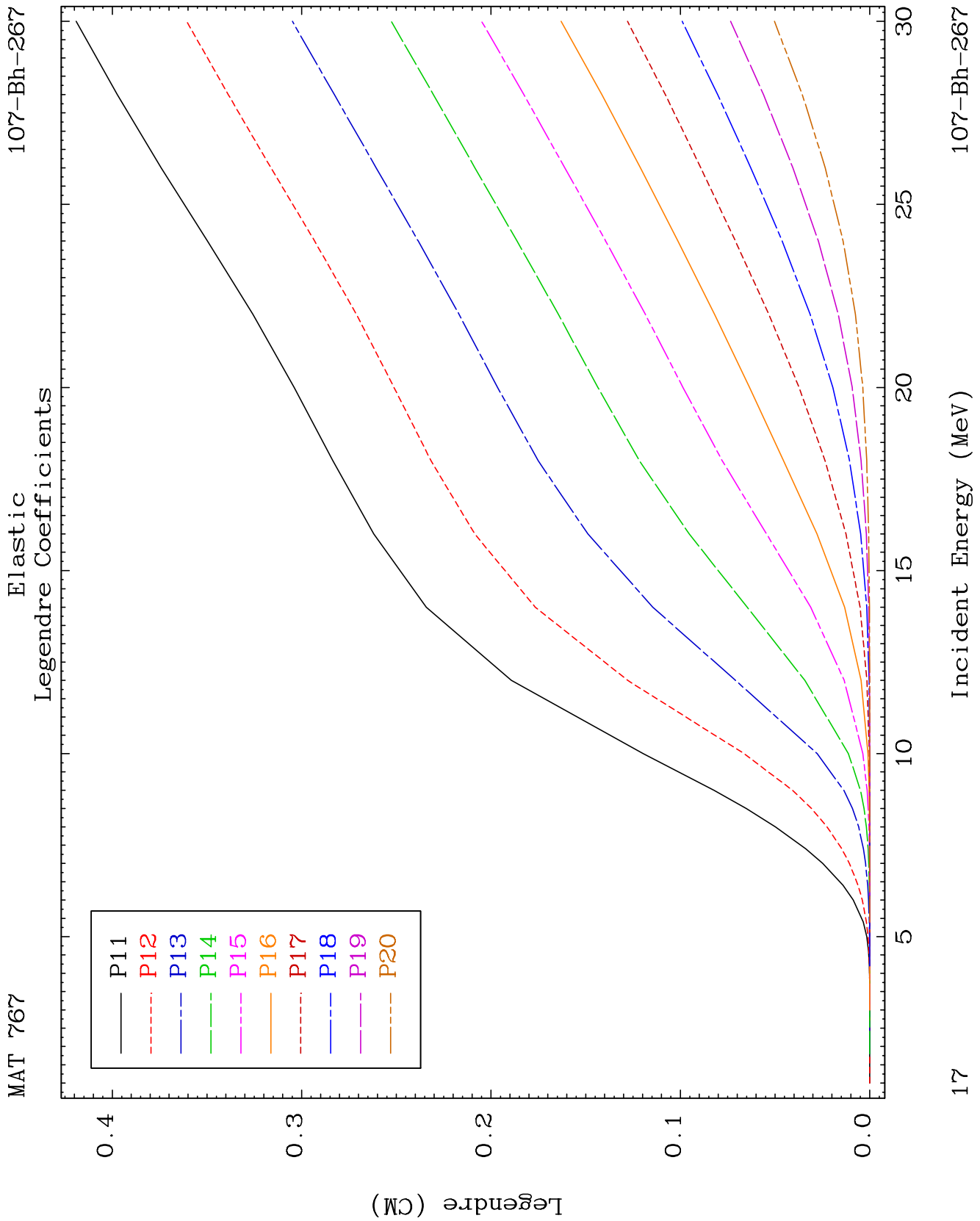
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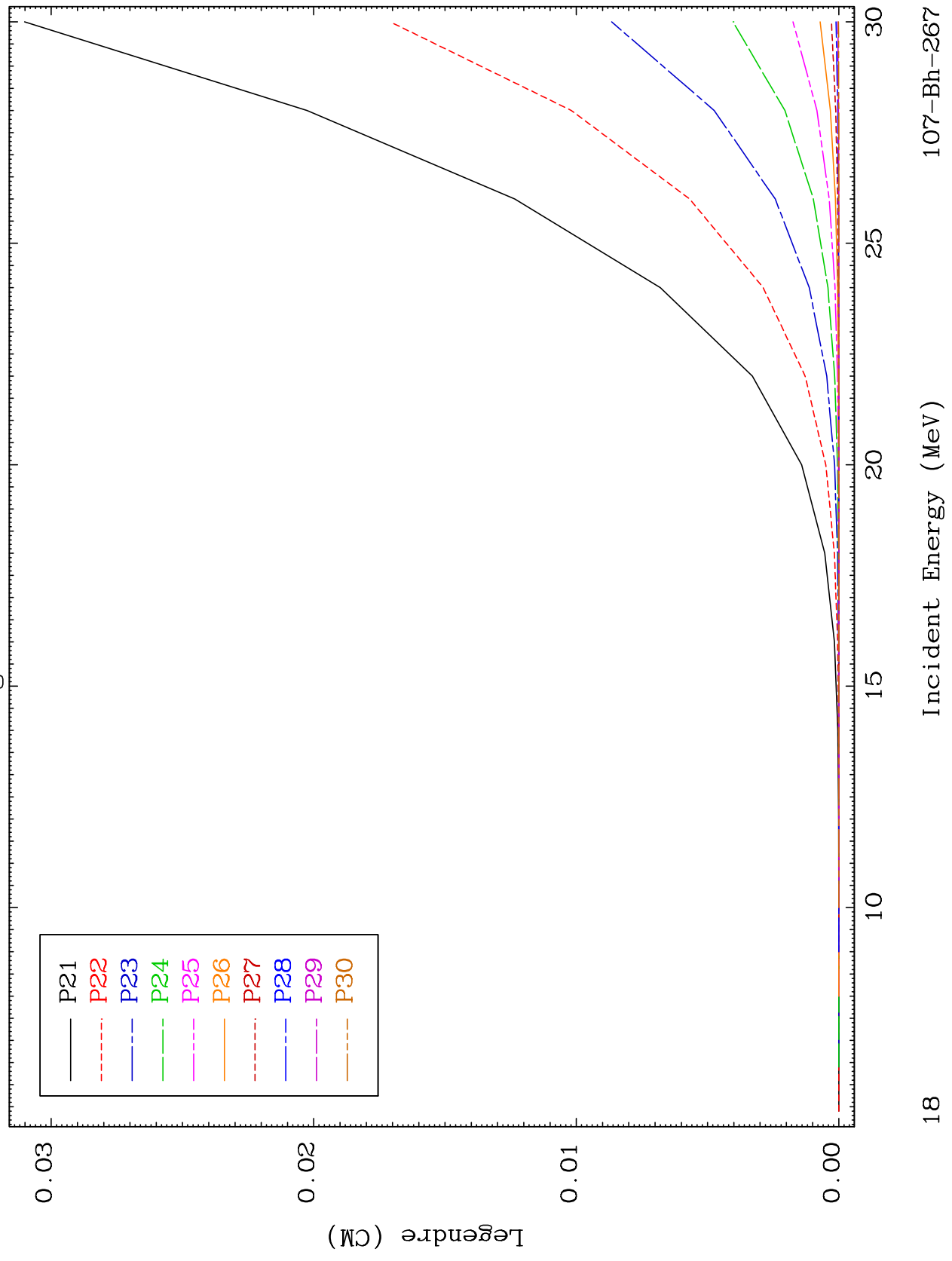




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### Elastic Legendre Coefficients

107-Bh-267

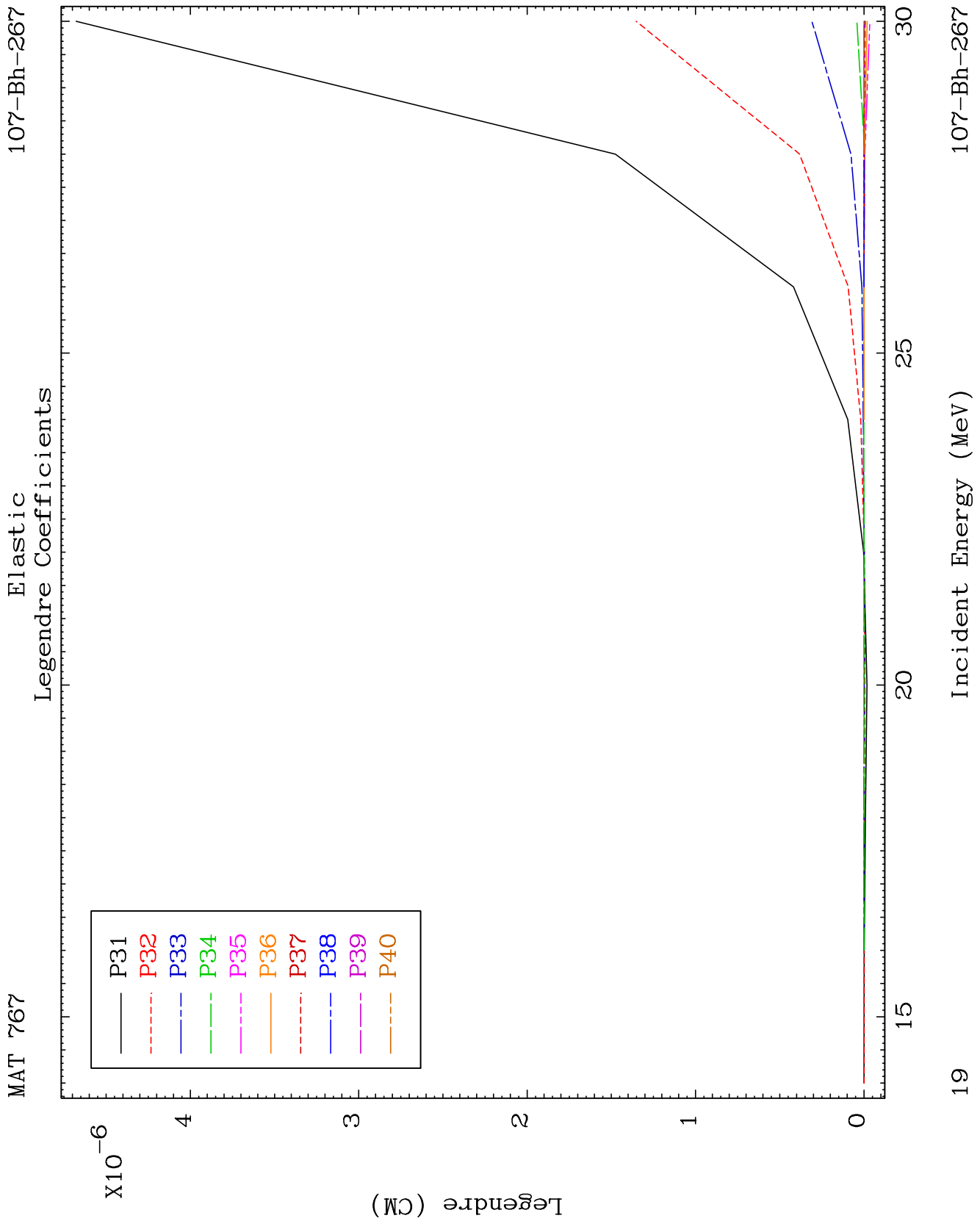


18

Incident Energy (MeV)

107-Bh-267

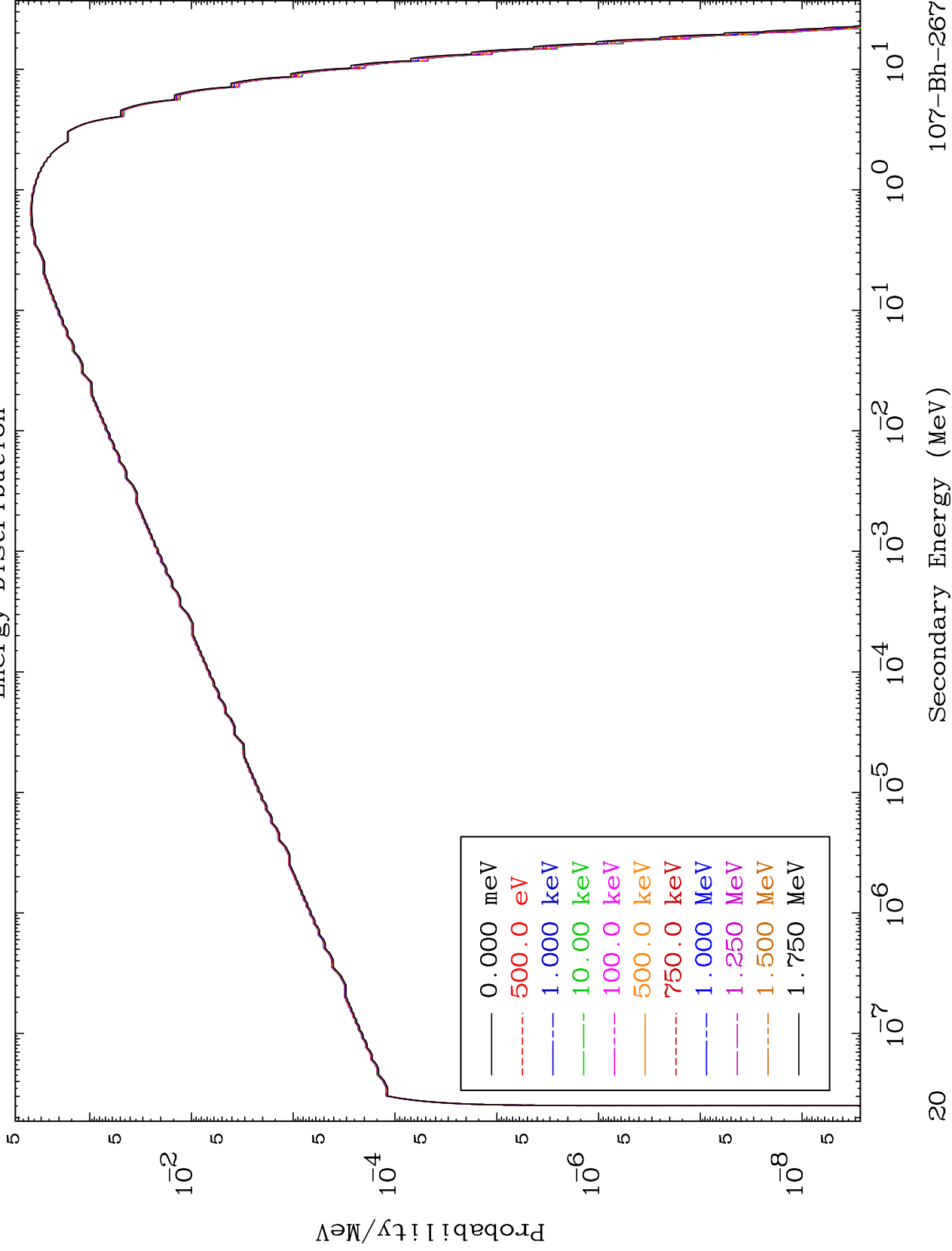
30



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### Fission Energy Distribution

107-Bh-267



20

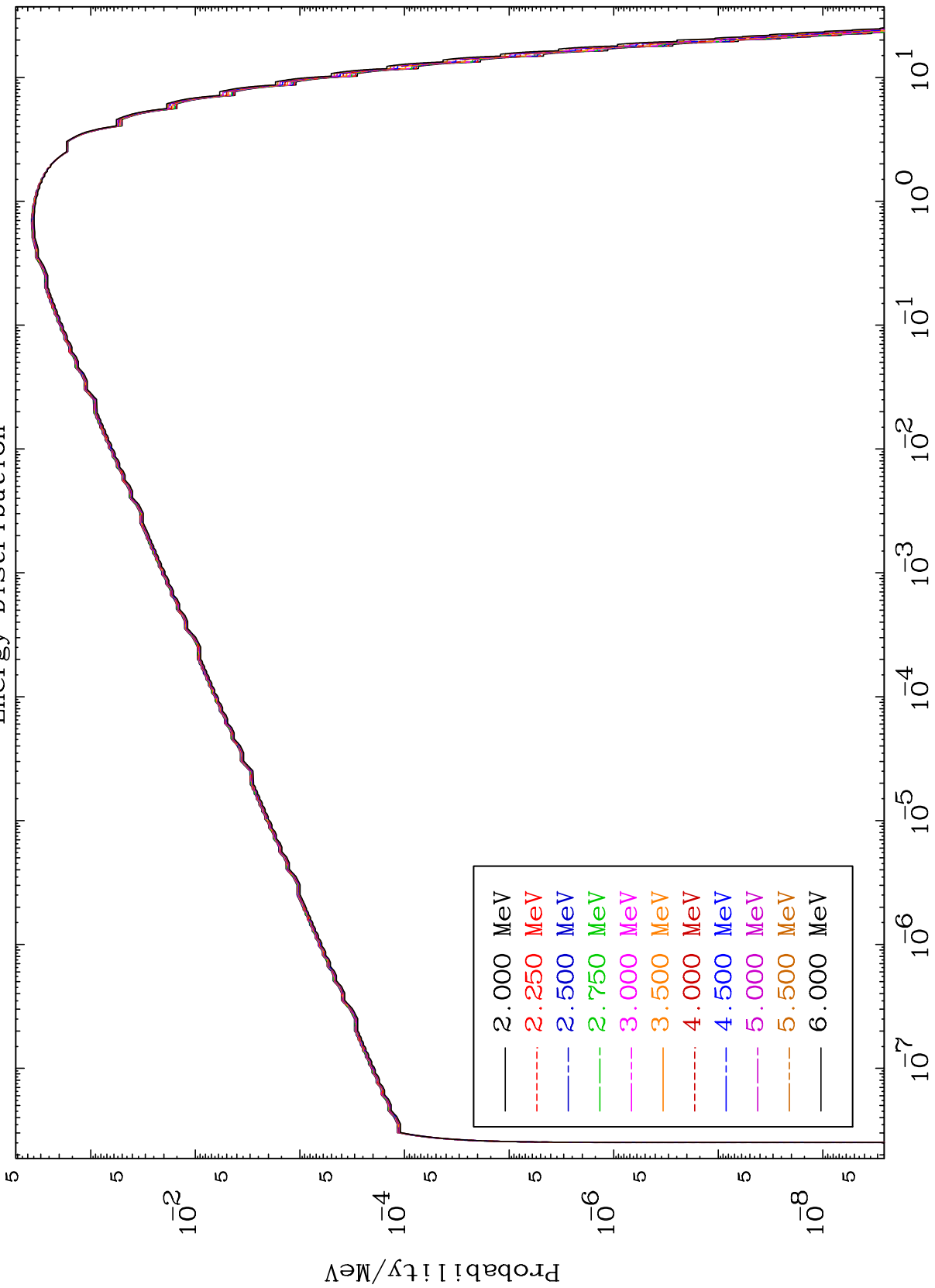
Secondary Energy (MeV)

107-Bh-267

MAT 767

### Fission Energy Distribution

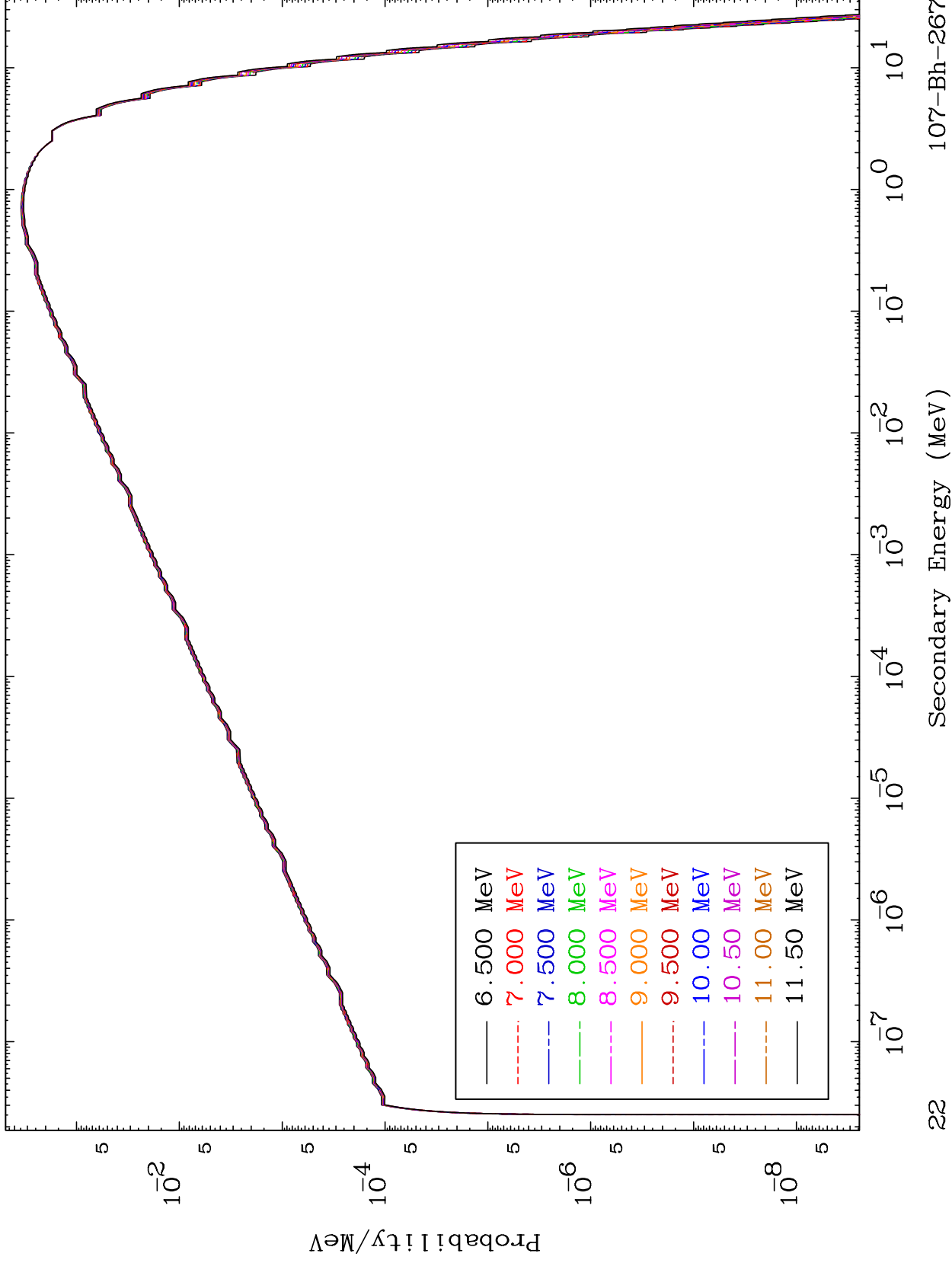
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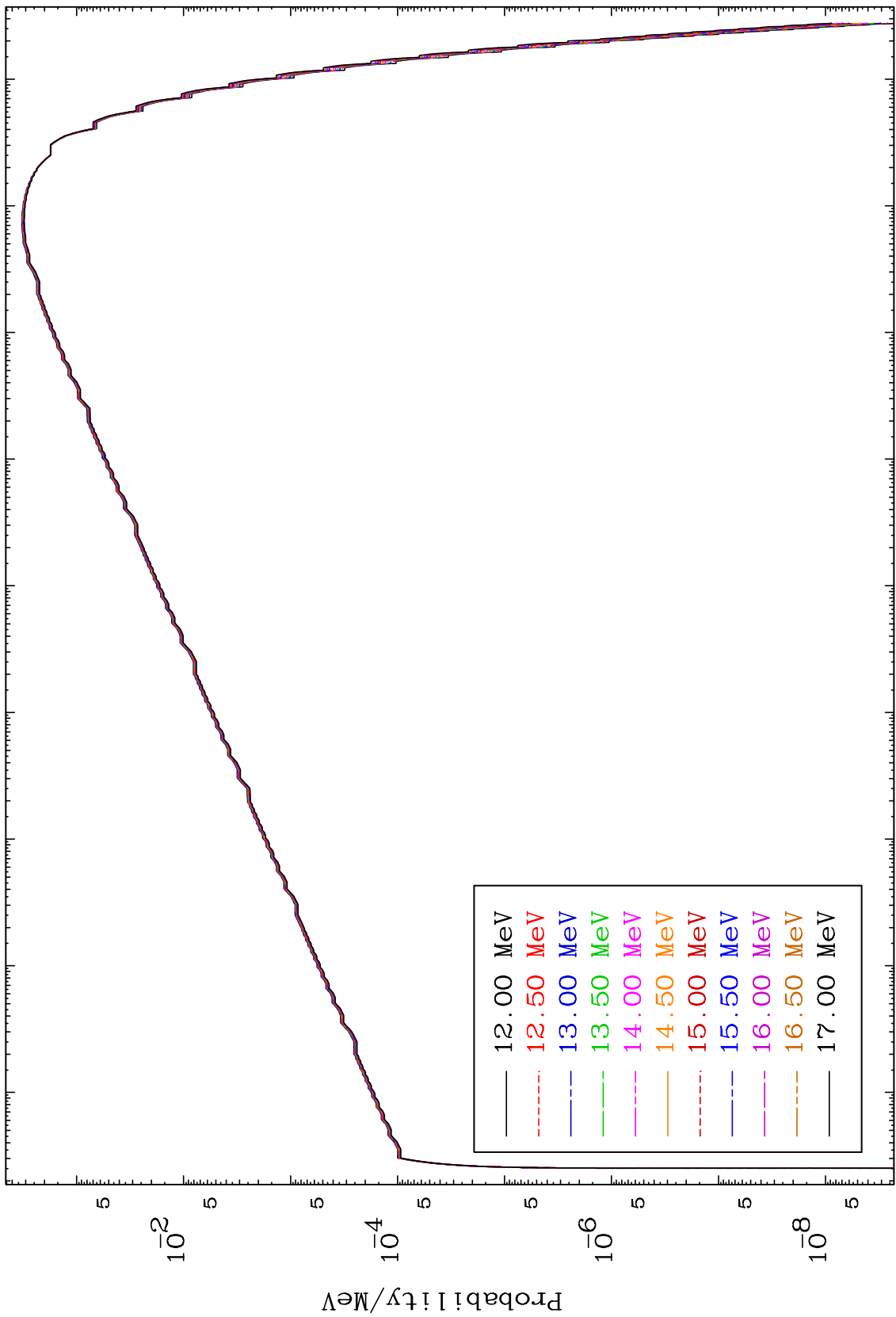
107-Bh-267

Secondary Energy (MeV)

Fission  
Energy Distribution



Fission Energy Distribution

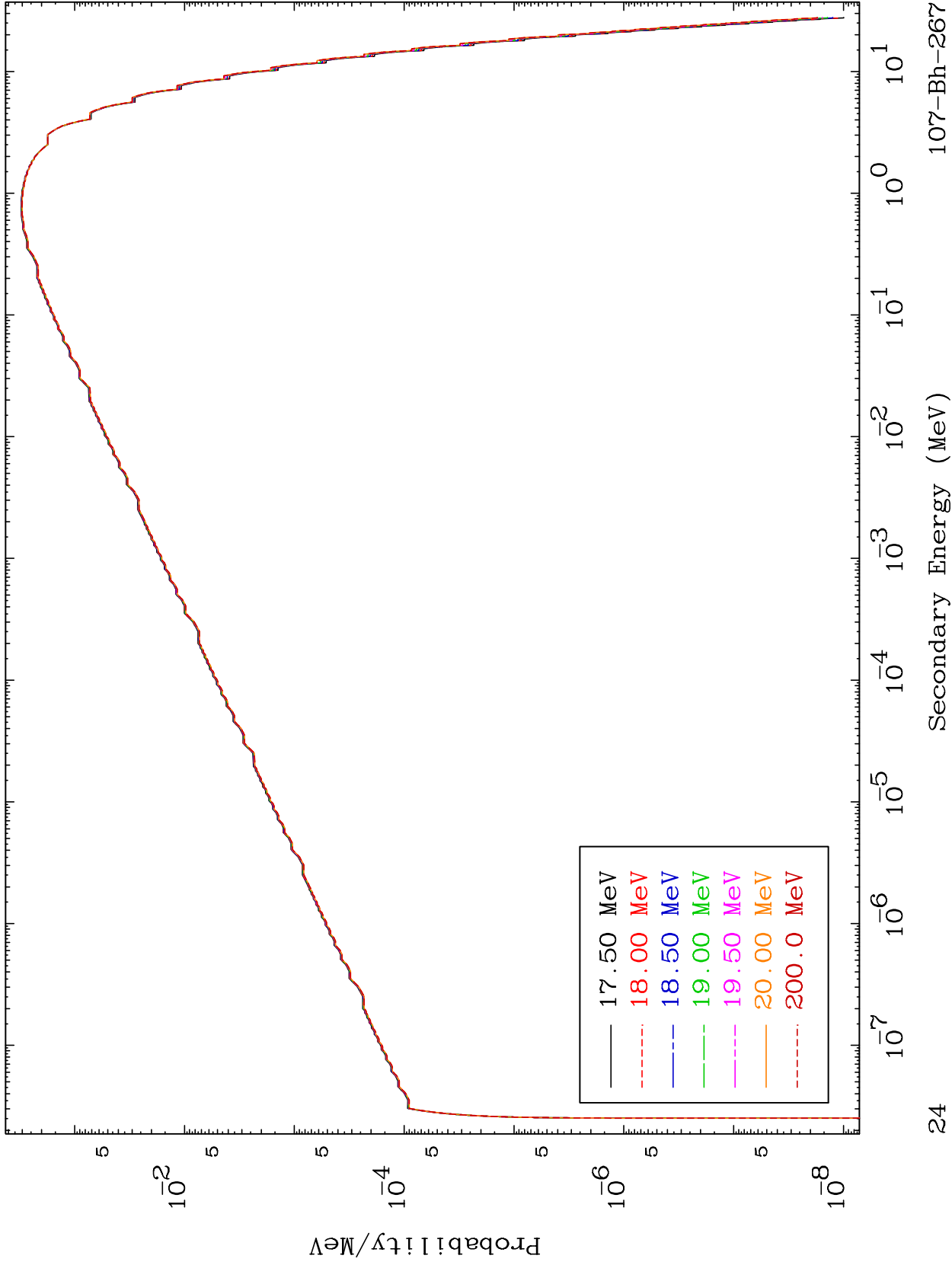




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### Fission Energy Distribution

107-Bh-267

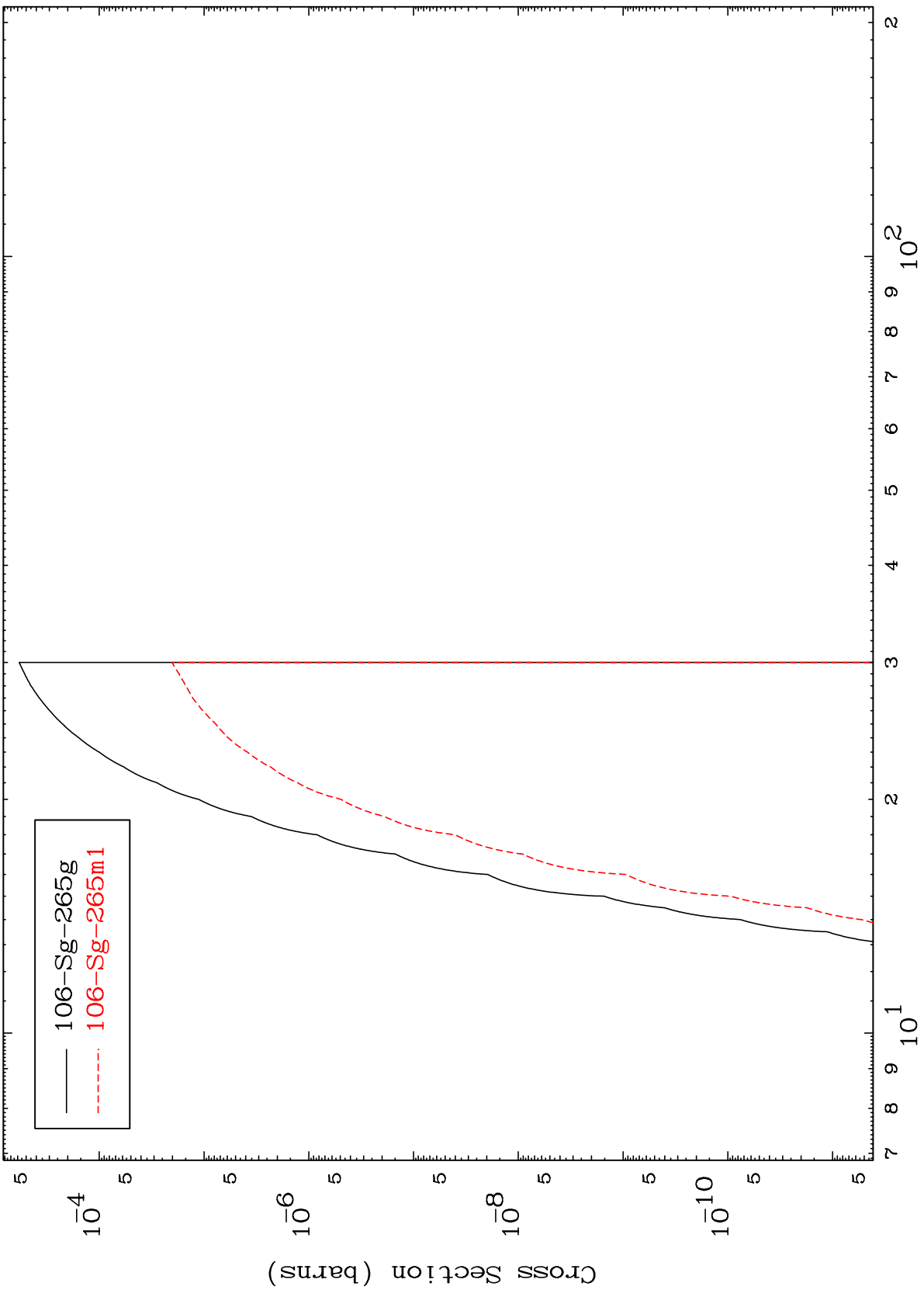


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(n,n') d

107-Bh-267

Radionuclide Production Cross Section



106-Sg-265g  
106-Sg-265m1

25

Incident Energy (MeV)

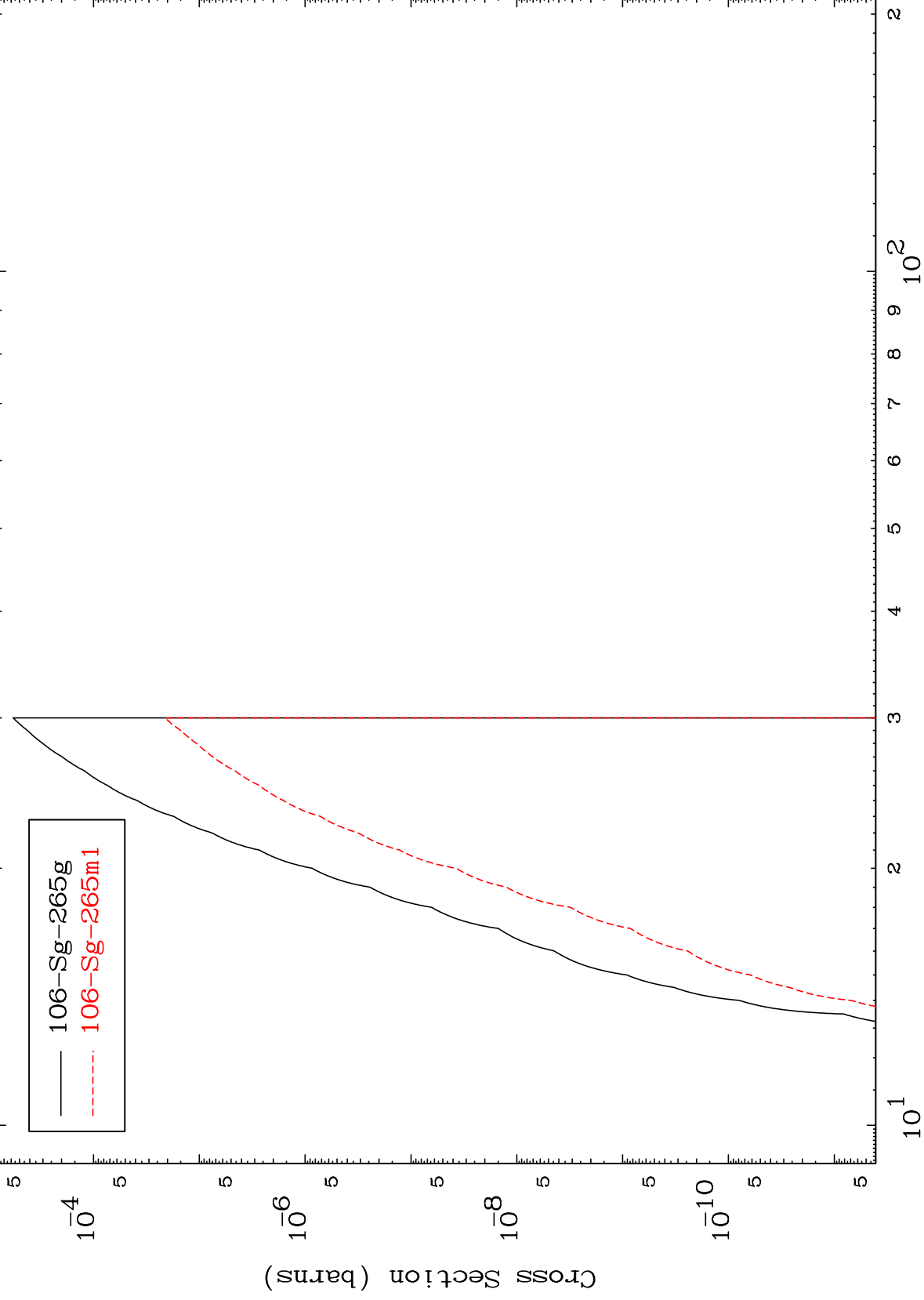
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(n,2n) p

107-Bh-267

Radionuclide Production Cross Section



26

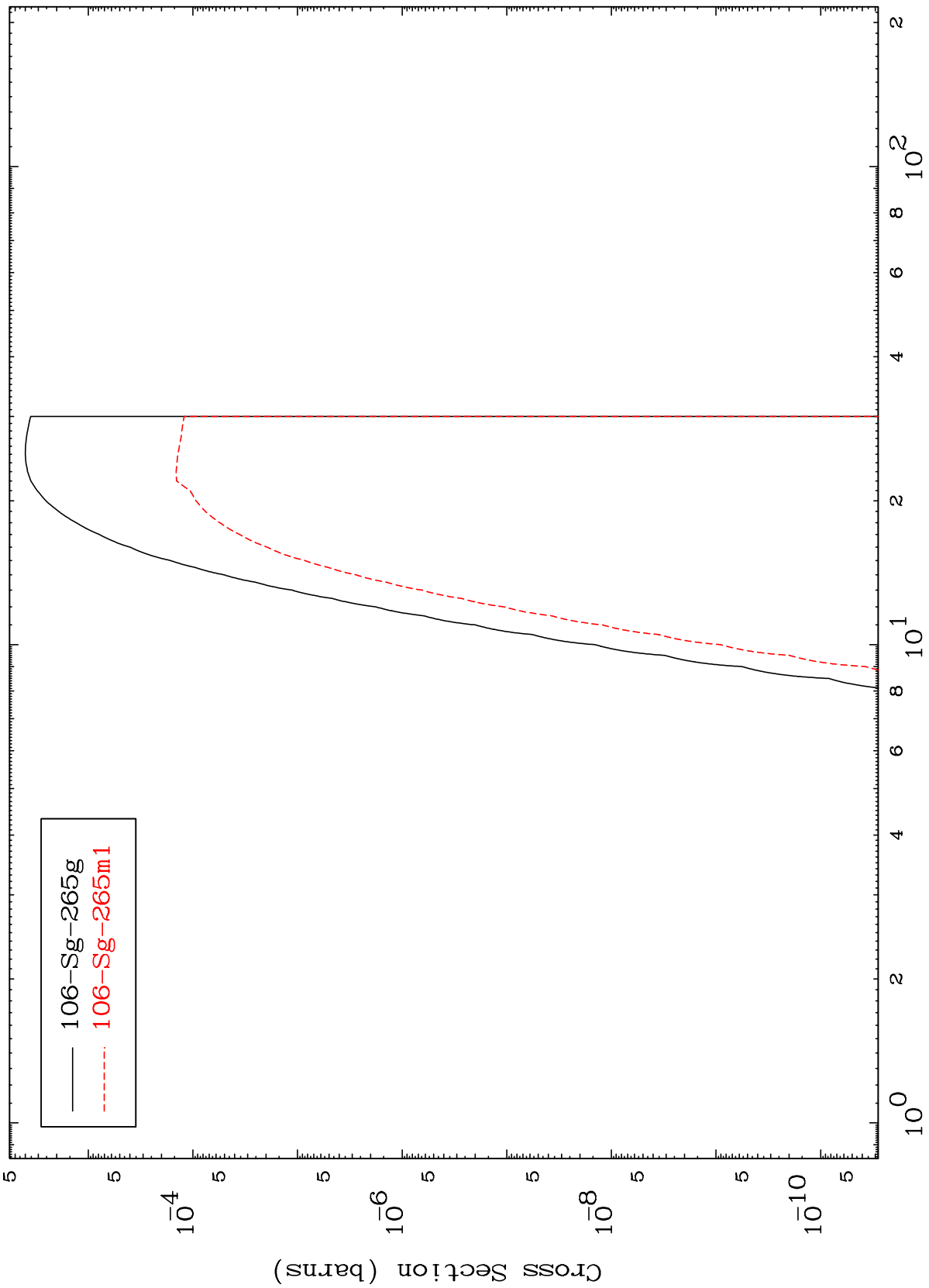
Incident Energy (MeV)

107-Bh-267

MAT 767

107-Bh-267

Radionuclide Production Cross Section  
(n, t)



— 106-Sg-265g  
- - - 106-Sg-265m1

107-Bh-267

Incident Energy (MeV)

27