

Program EVALPLOT
(Version 2017-1)

by

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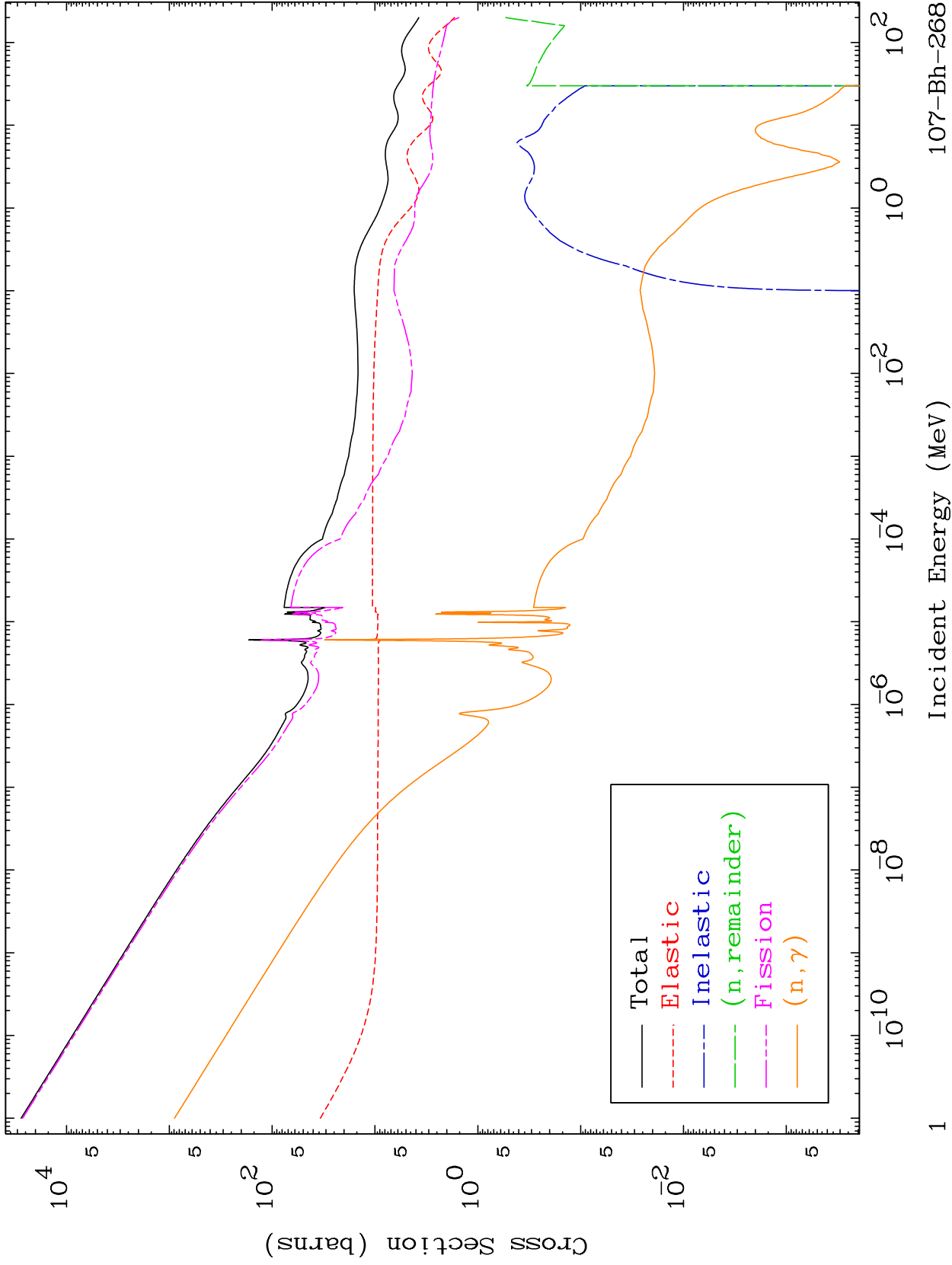
Press Mouse Button to Start

MAT 768

Major

293 Kelvin Cross Sections

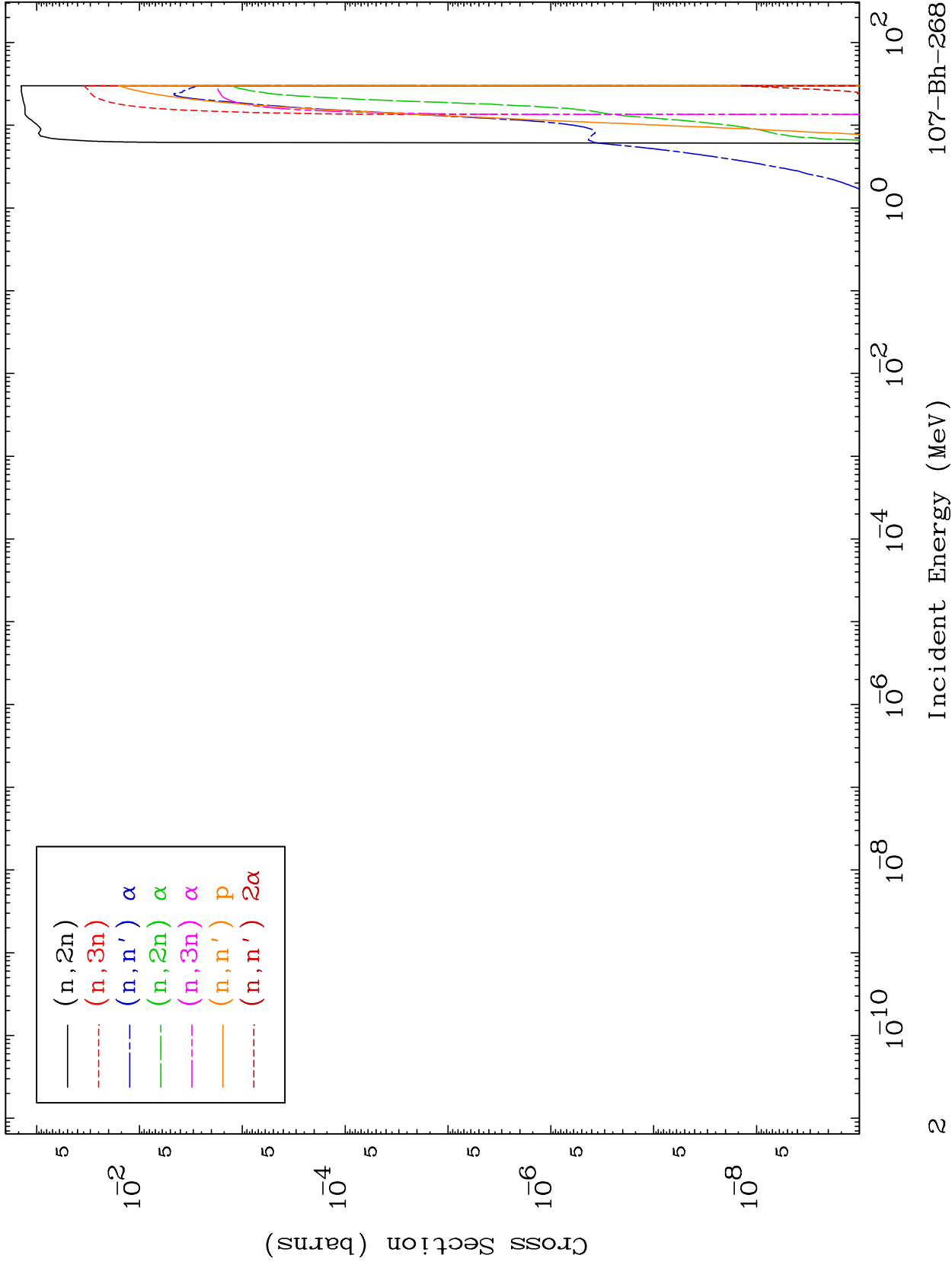
107-Bh-268



MAT 768

Neutron Production
293 Kelvin Cross Sections

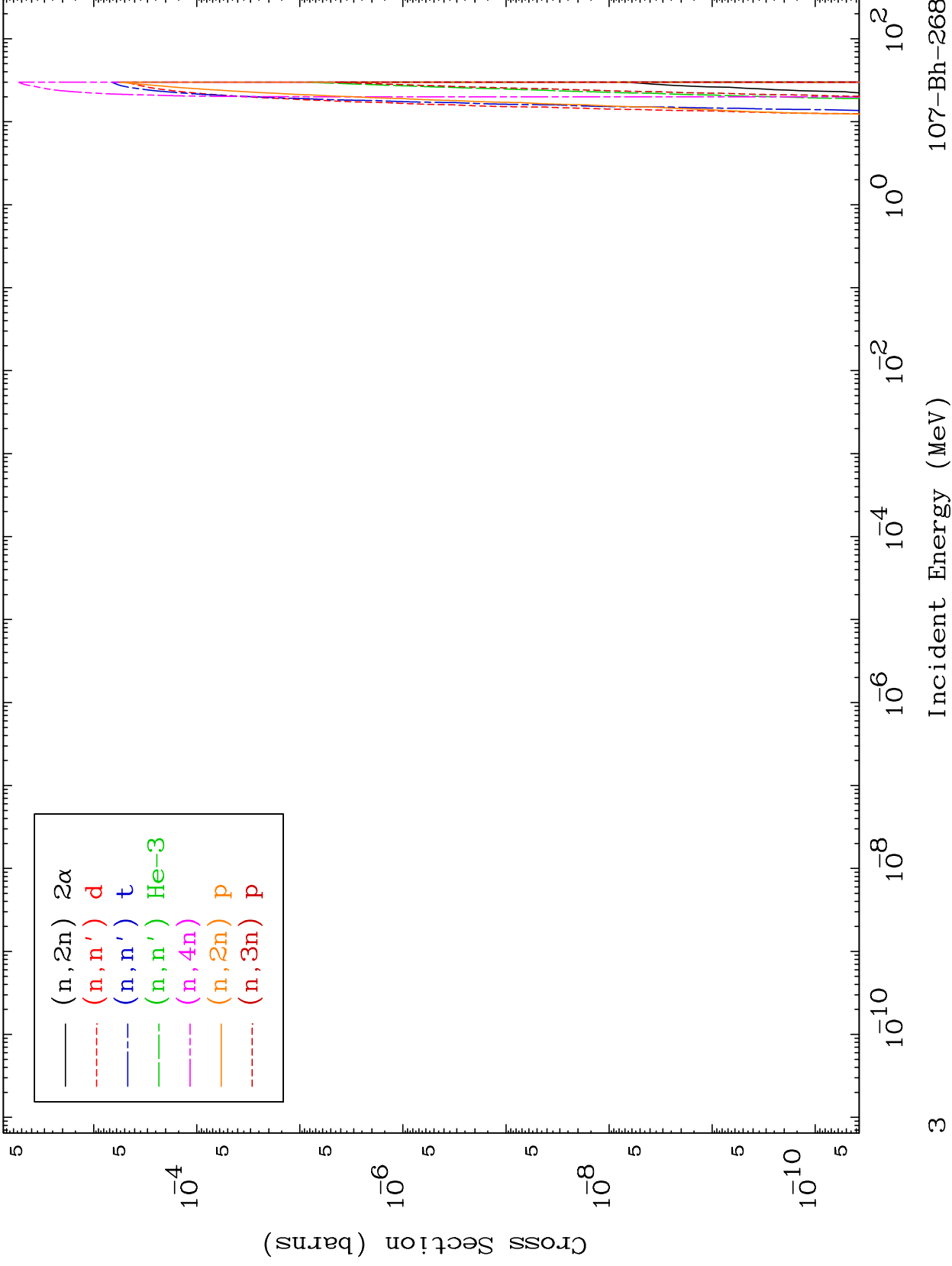
107-Bh-268

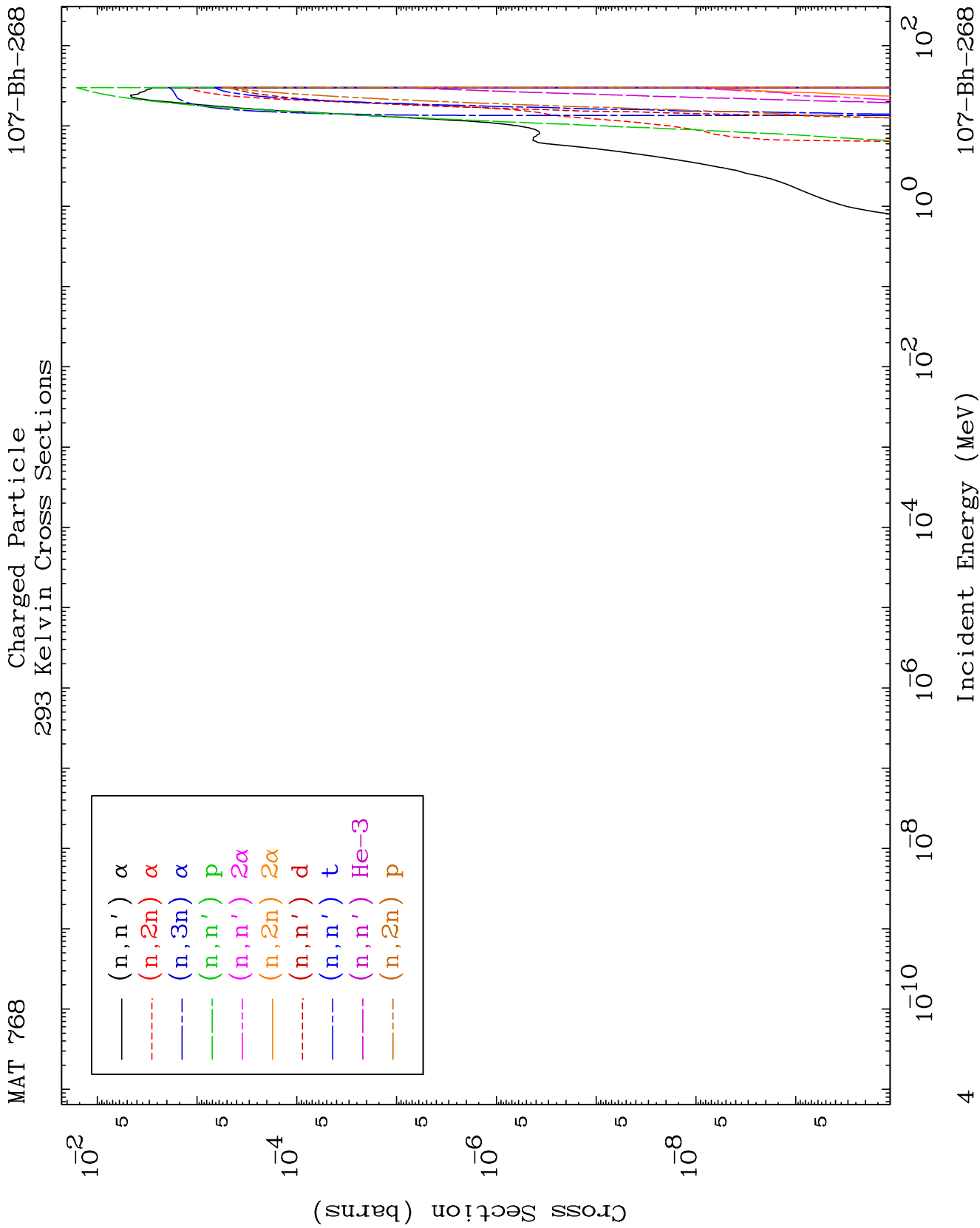


MAT 768

Neutron Production
293 Kelvin Cross Sections

107-Bh-268

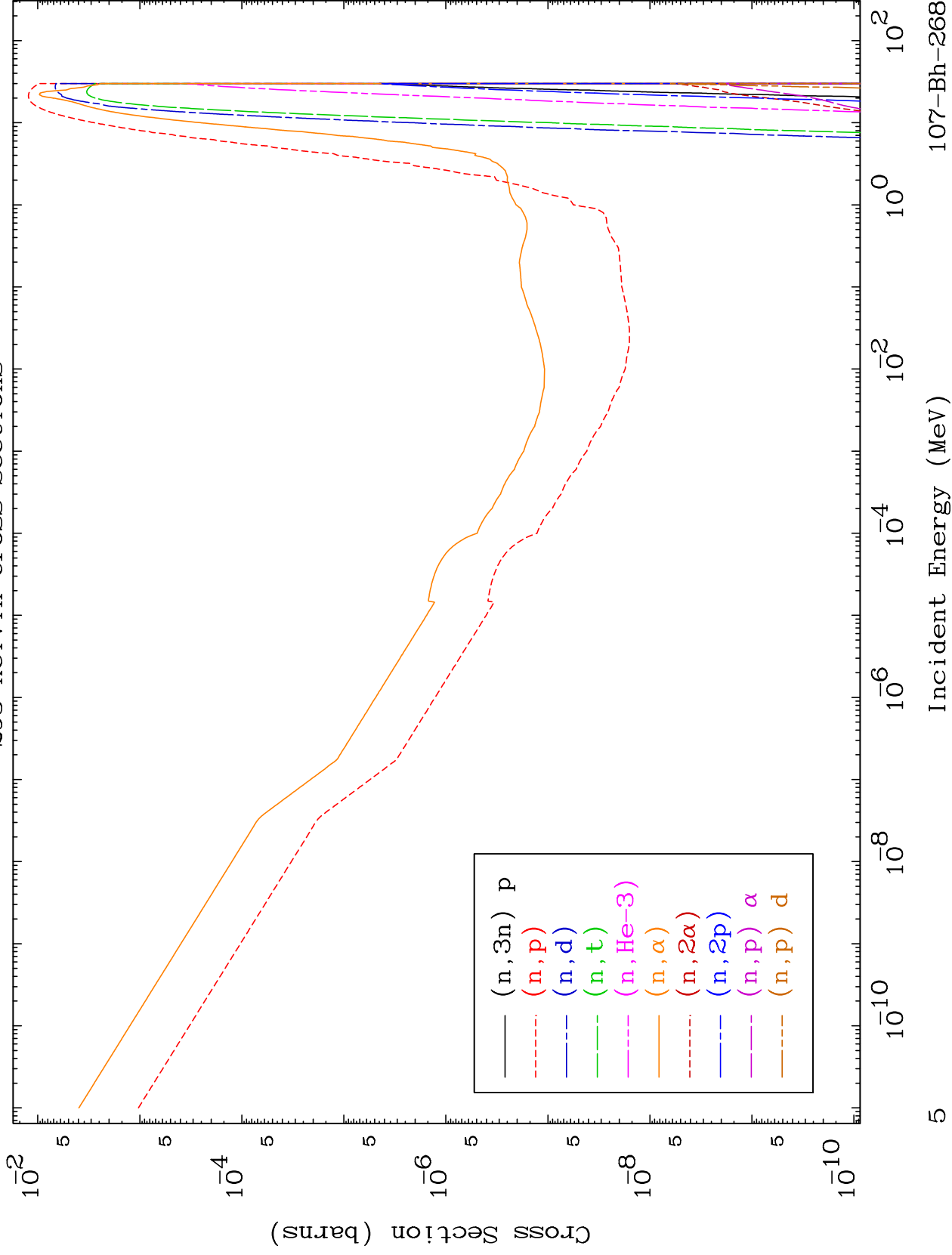


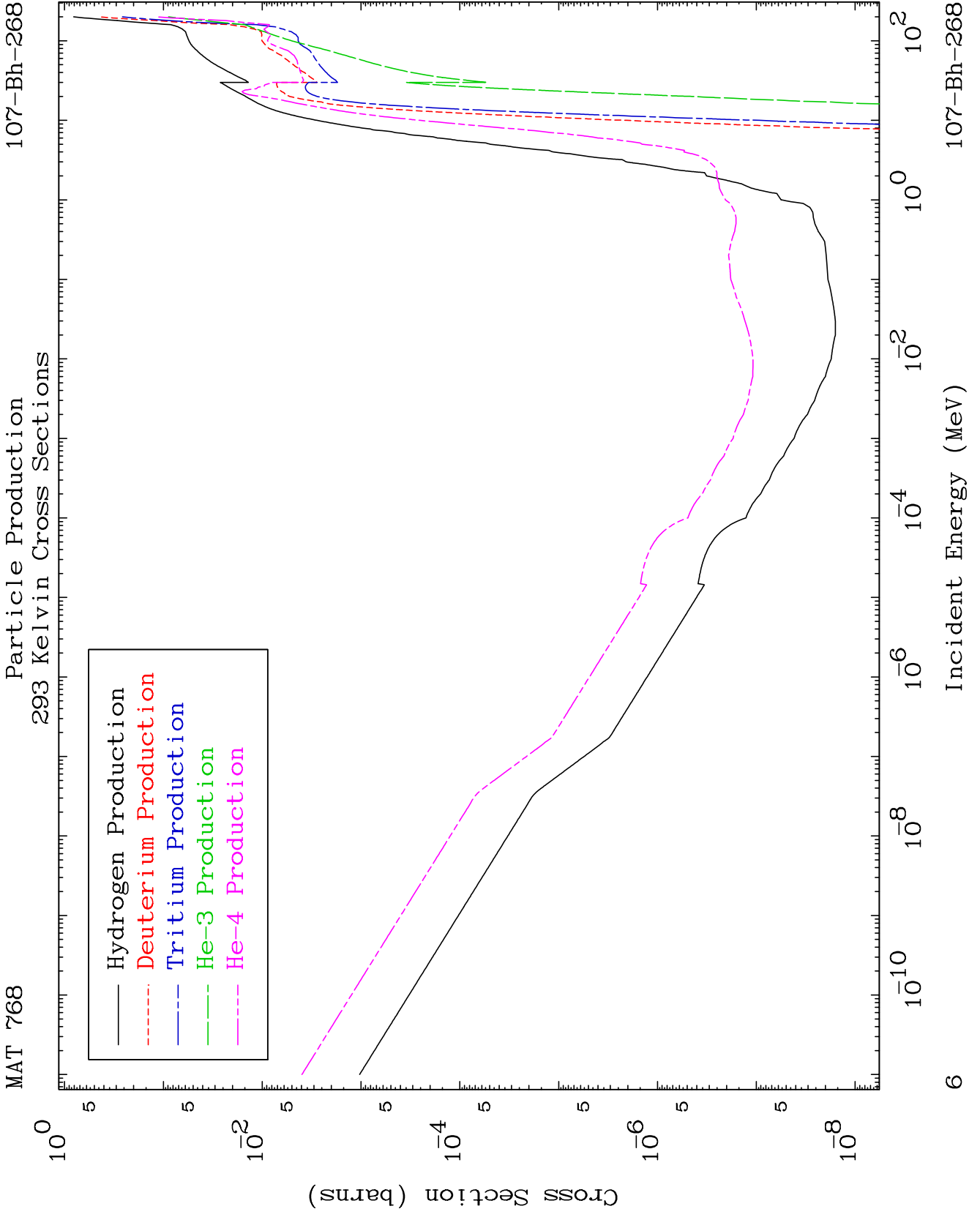


MAT 768

Charged Particle
293 Kelvin Cross Sections

107-Bh-268

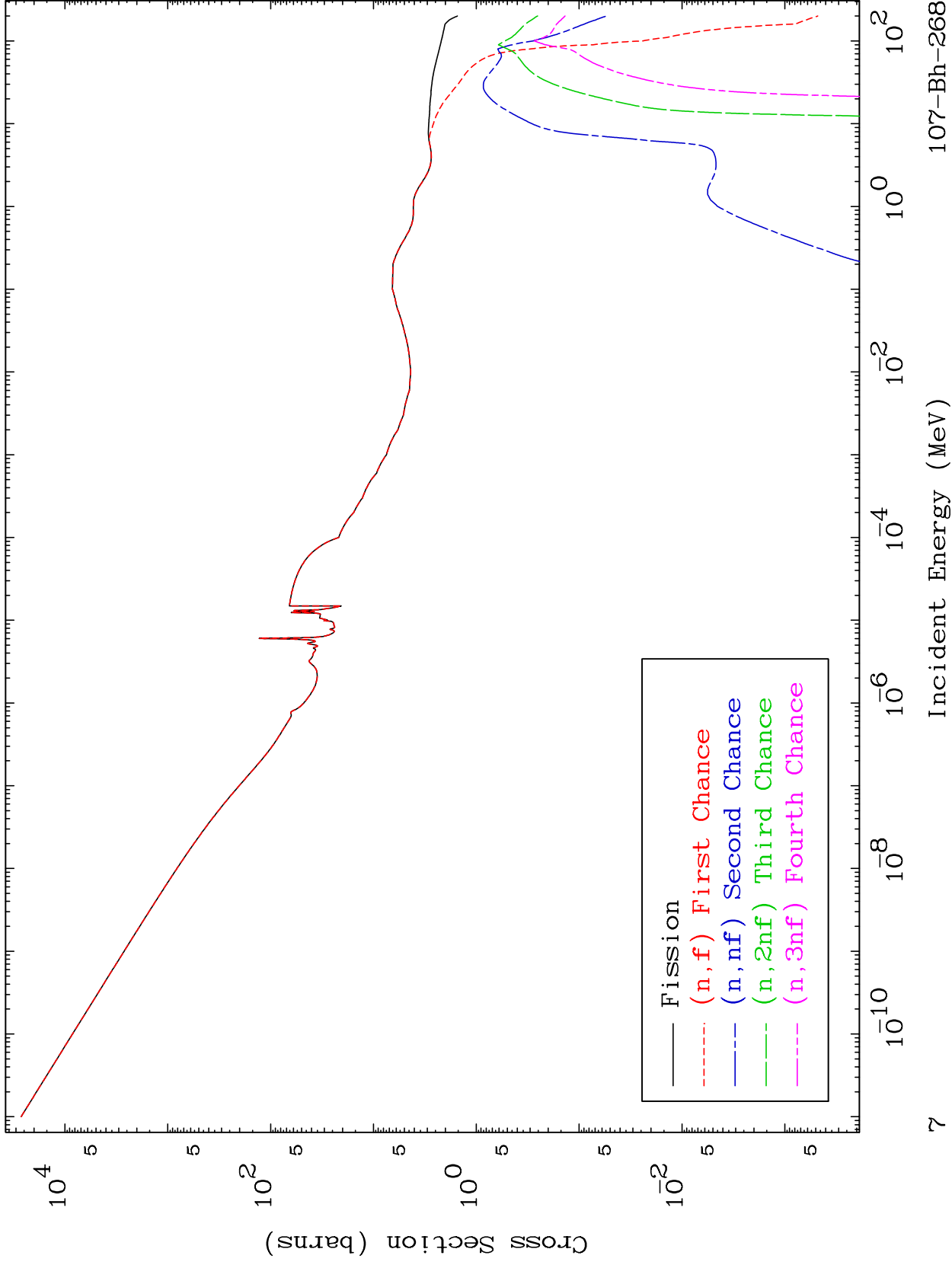




MAT 768

Fission
293 Kelvin Cross Sections

107-Bh-268

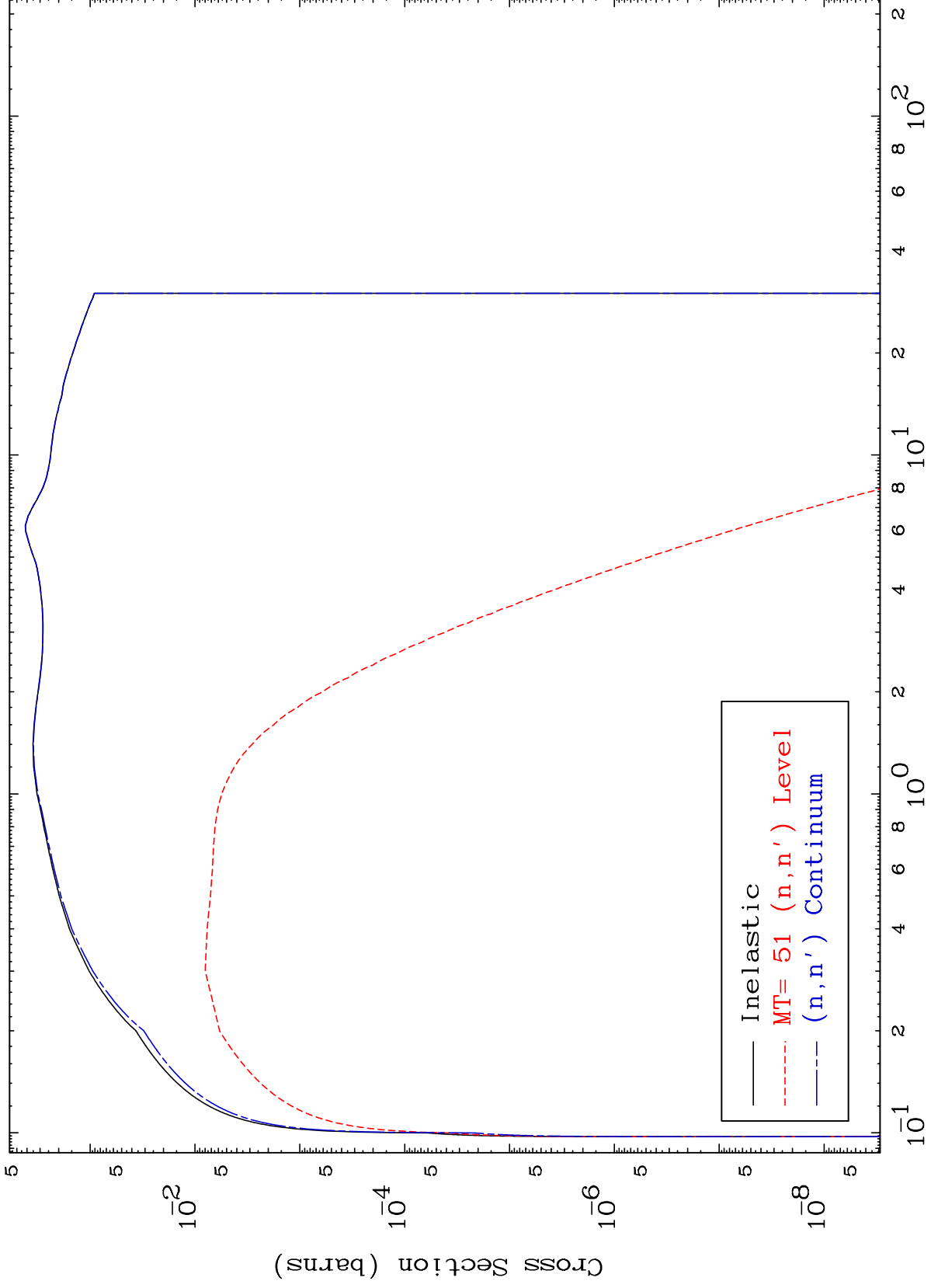


MAT 768

(n,n') Level

107-Bh-268

293 Kelvin Cross Sections



Incident Energy (MeV)

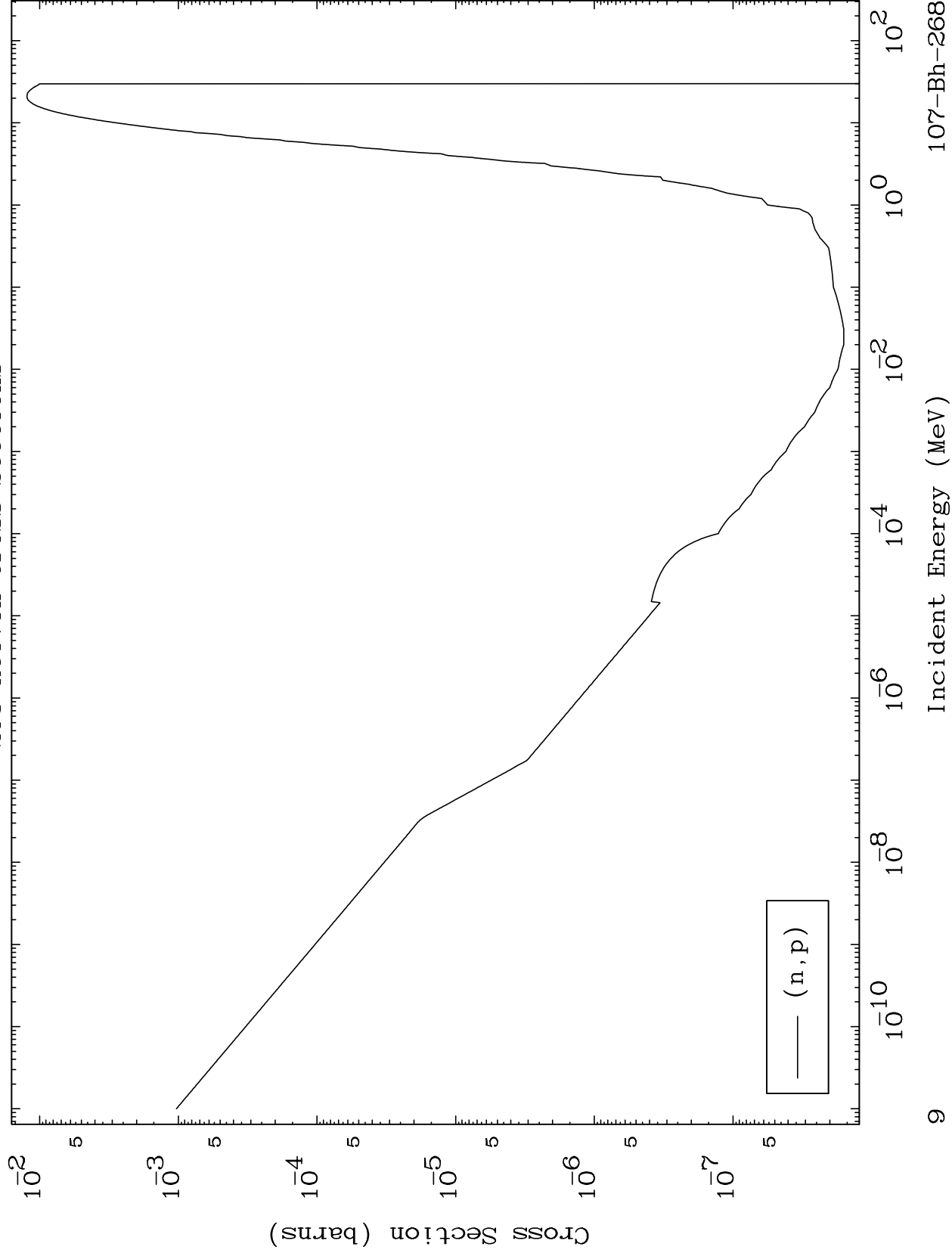
107-Bh-268

8

MAT 768

(n,p) Levels
293 Kelvin Cross Sections

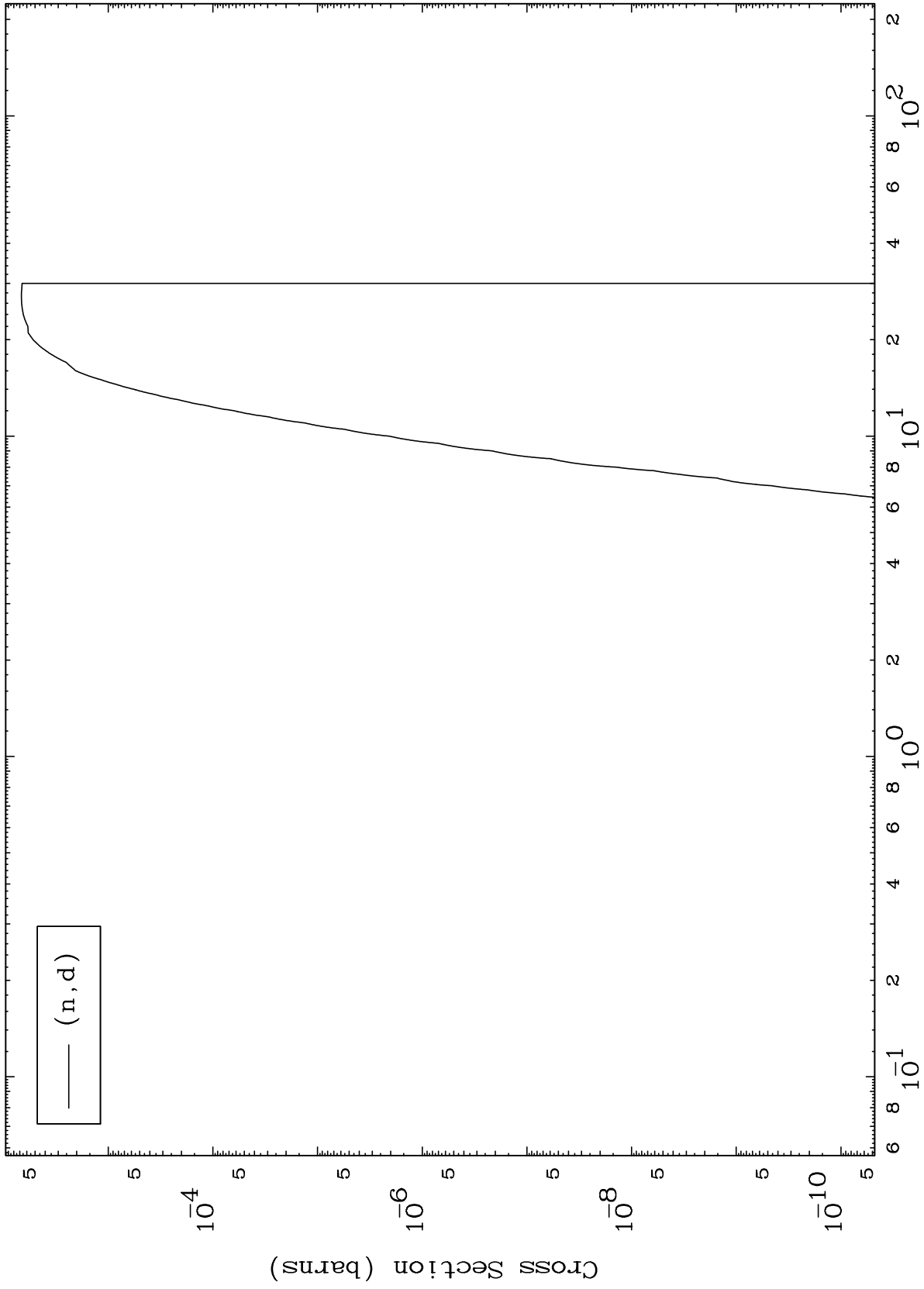
107-Bh-268



MAT 768

(n,d) Levels
293 Kelvin Cross Sections

107-Bh-268



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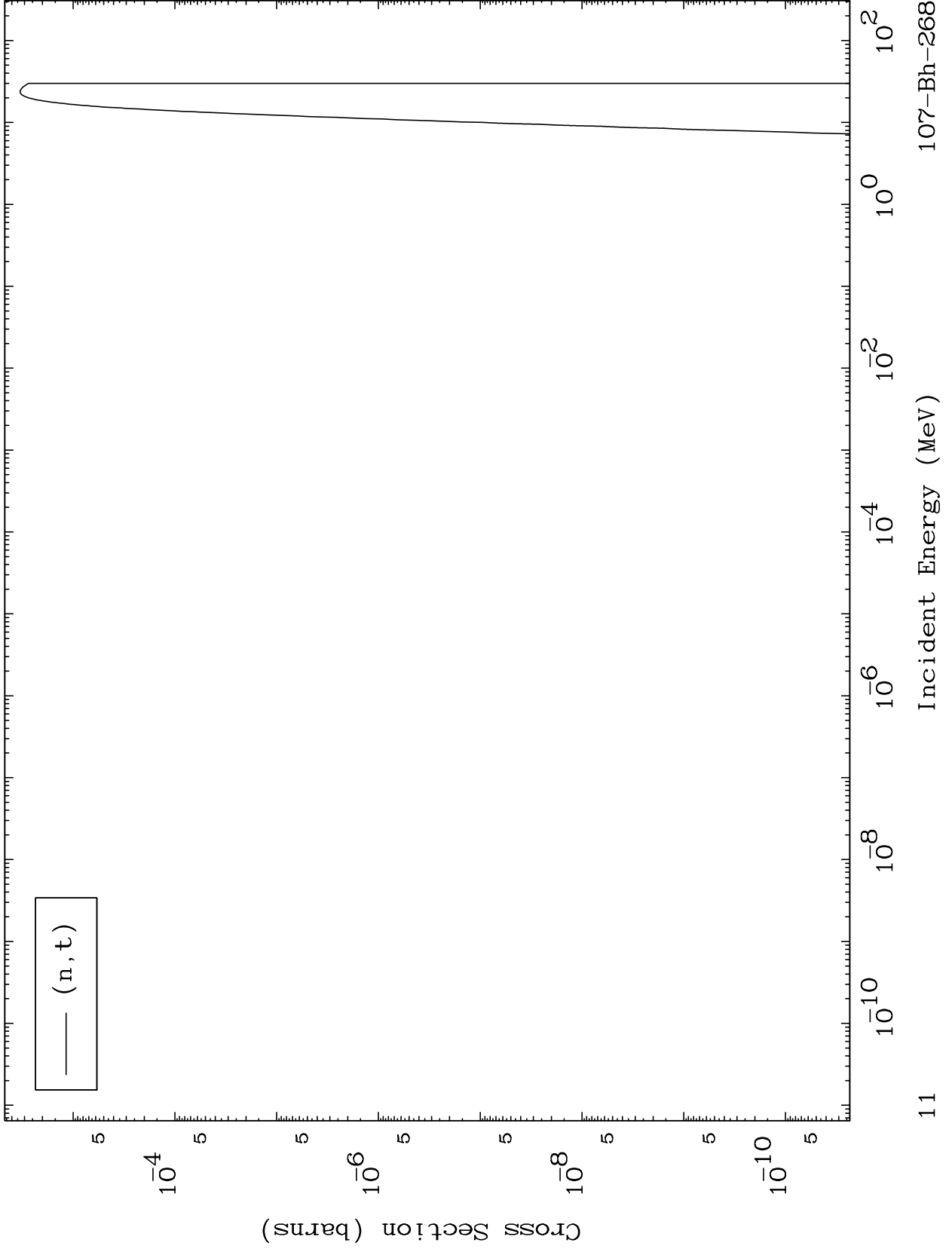
Incident Energy (MeV)

107-Bh-268

MAT 768

(n,t) Levels
293 Kelvin Cross Sections

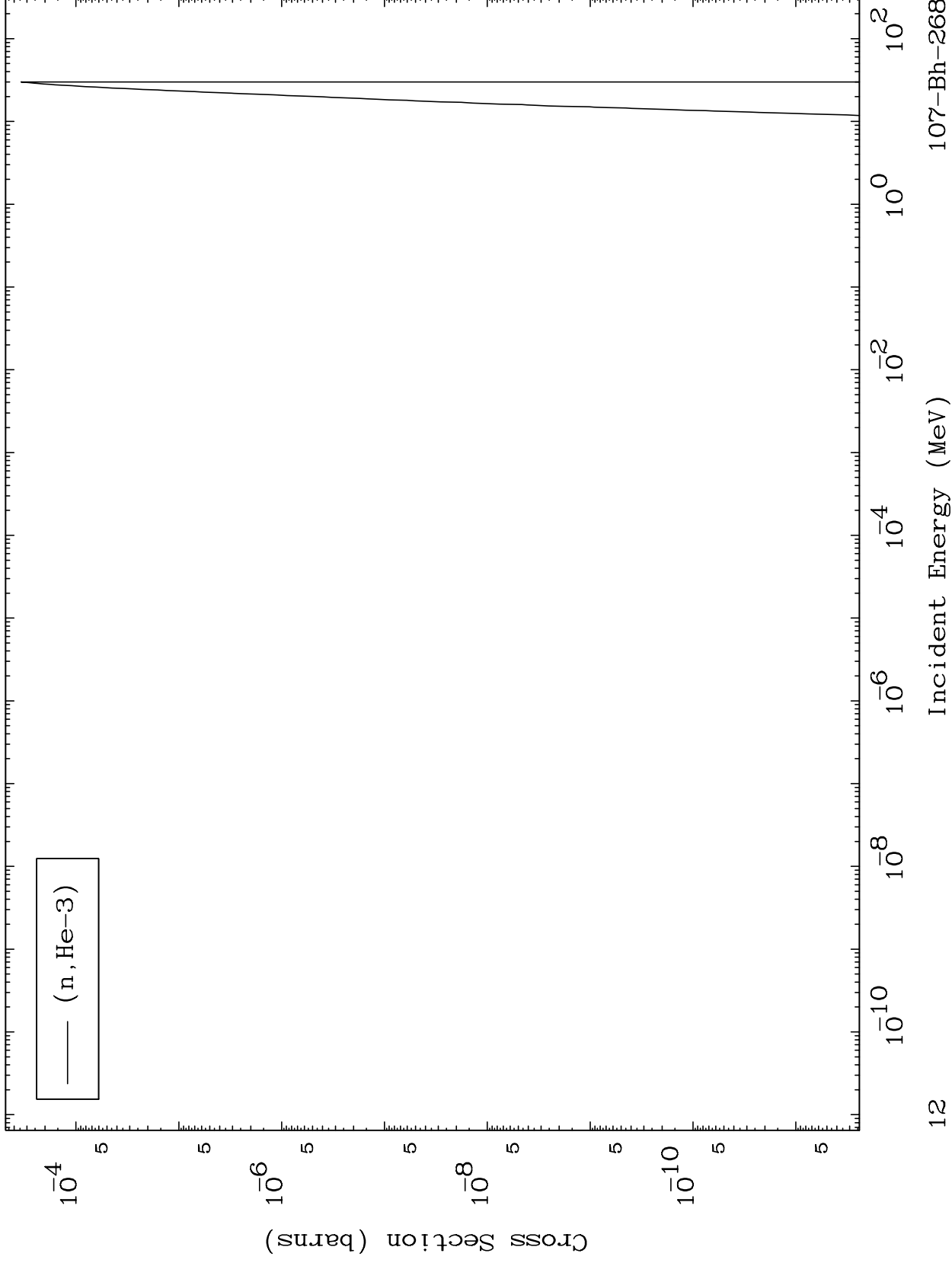
107-Bh-268

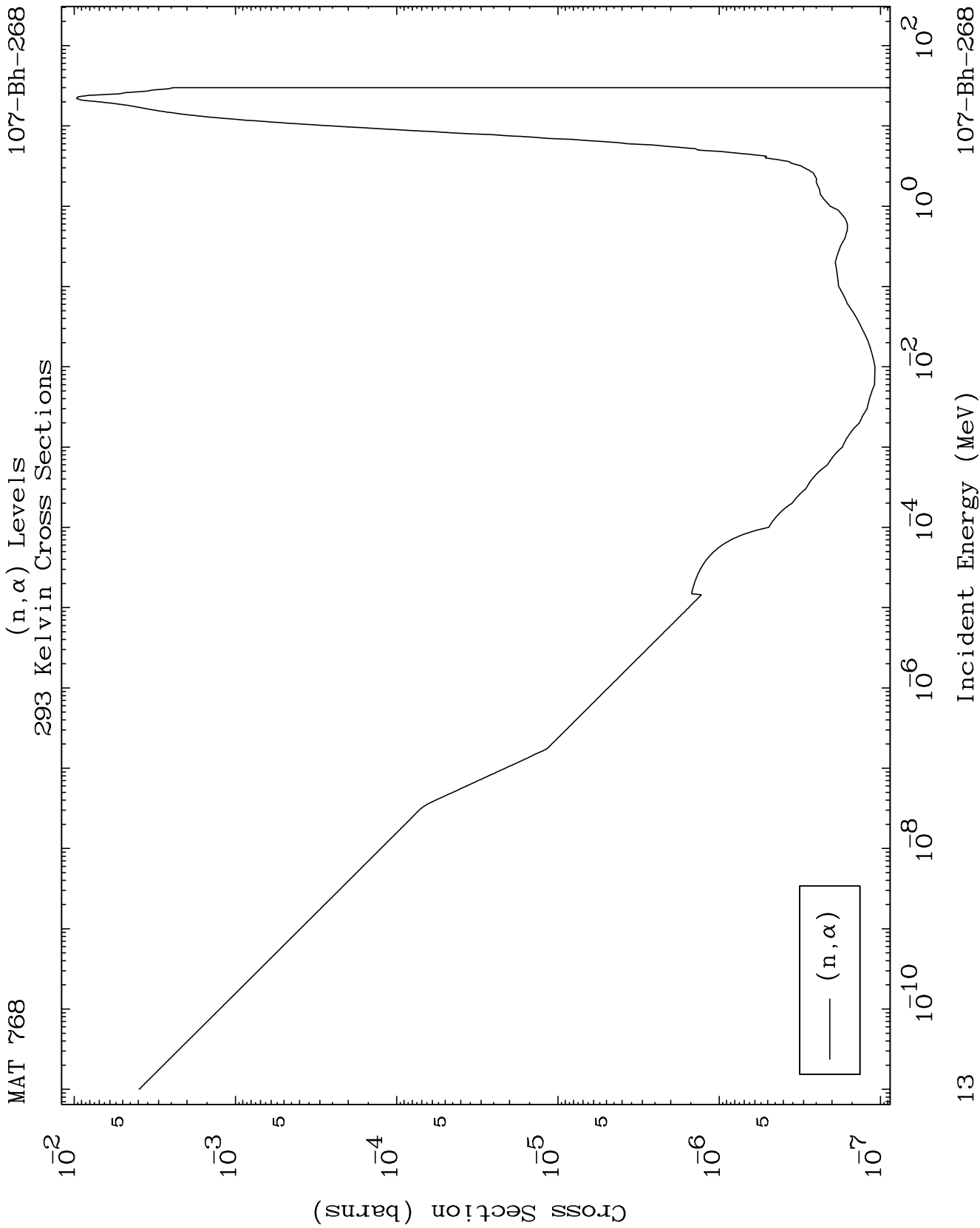


MAT 768

(n,He3) Levels
293 Kelvin Cross Sections

107-Bh-268

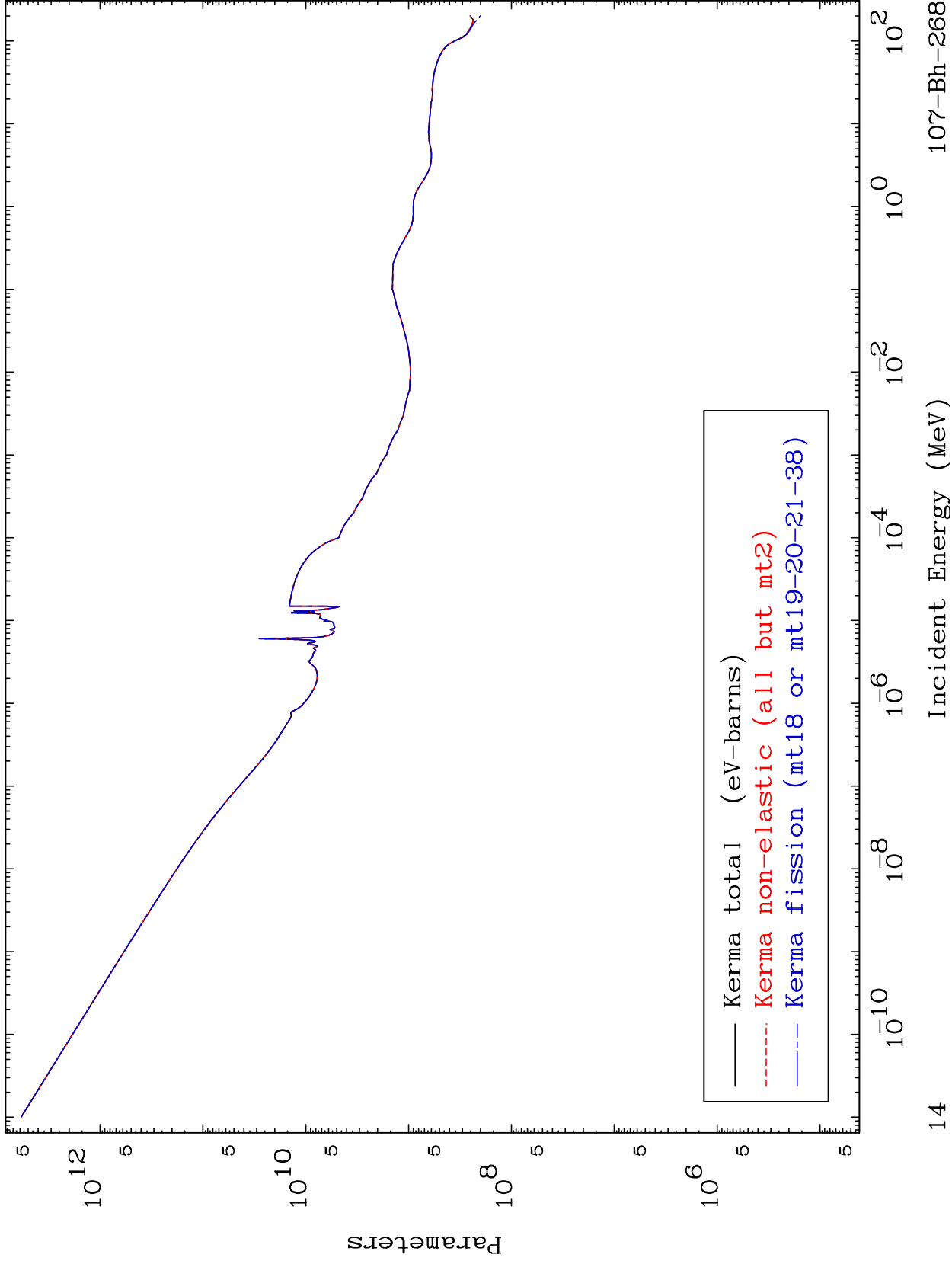


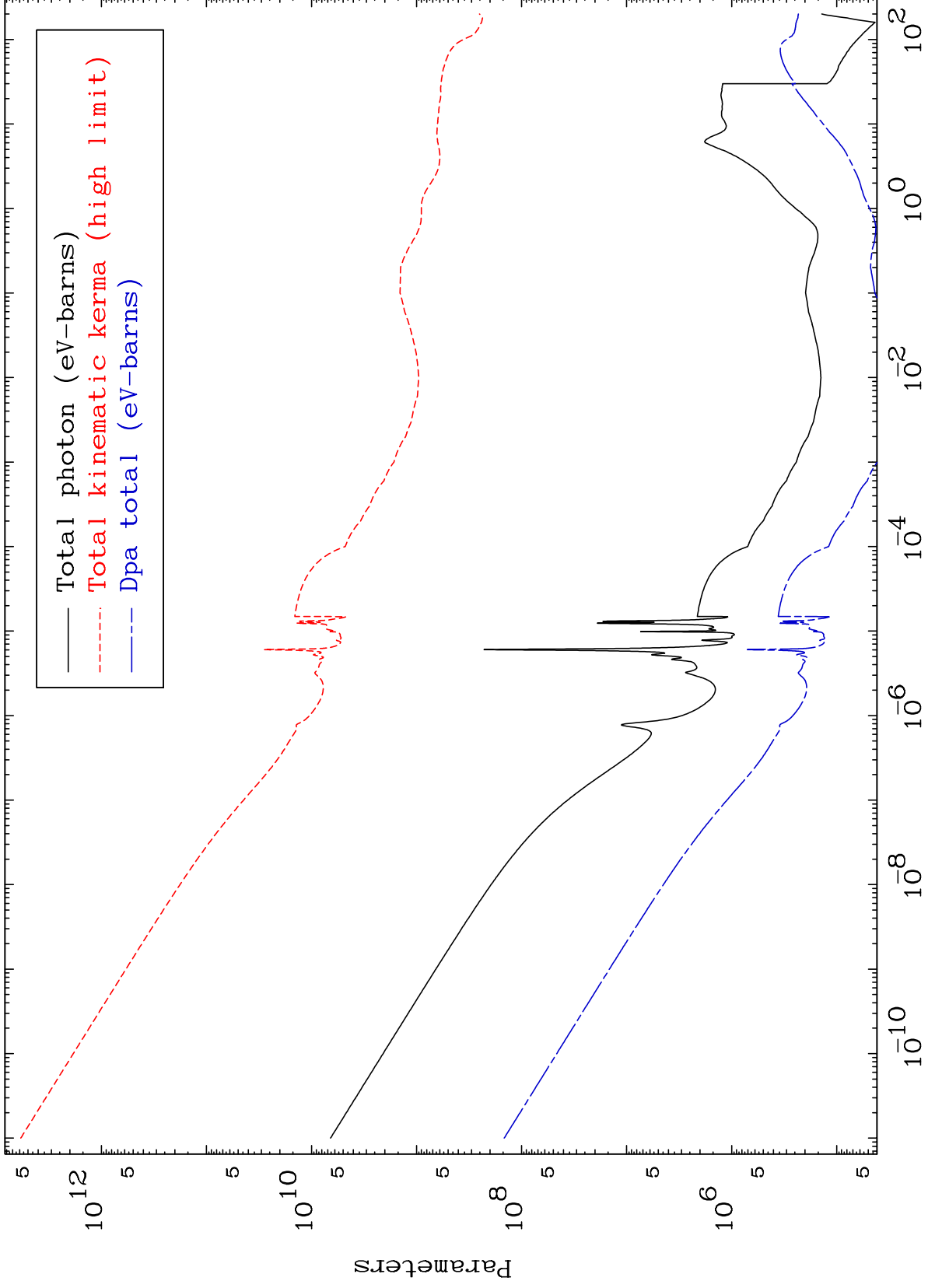


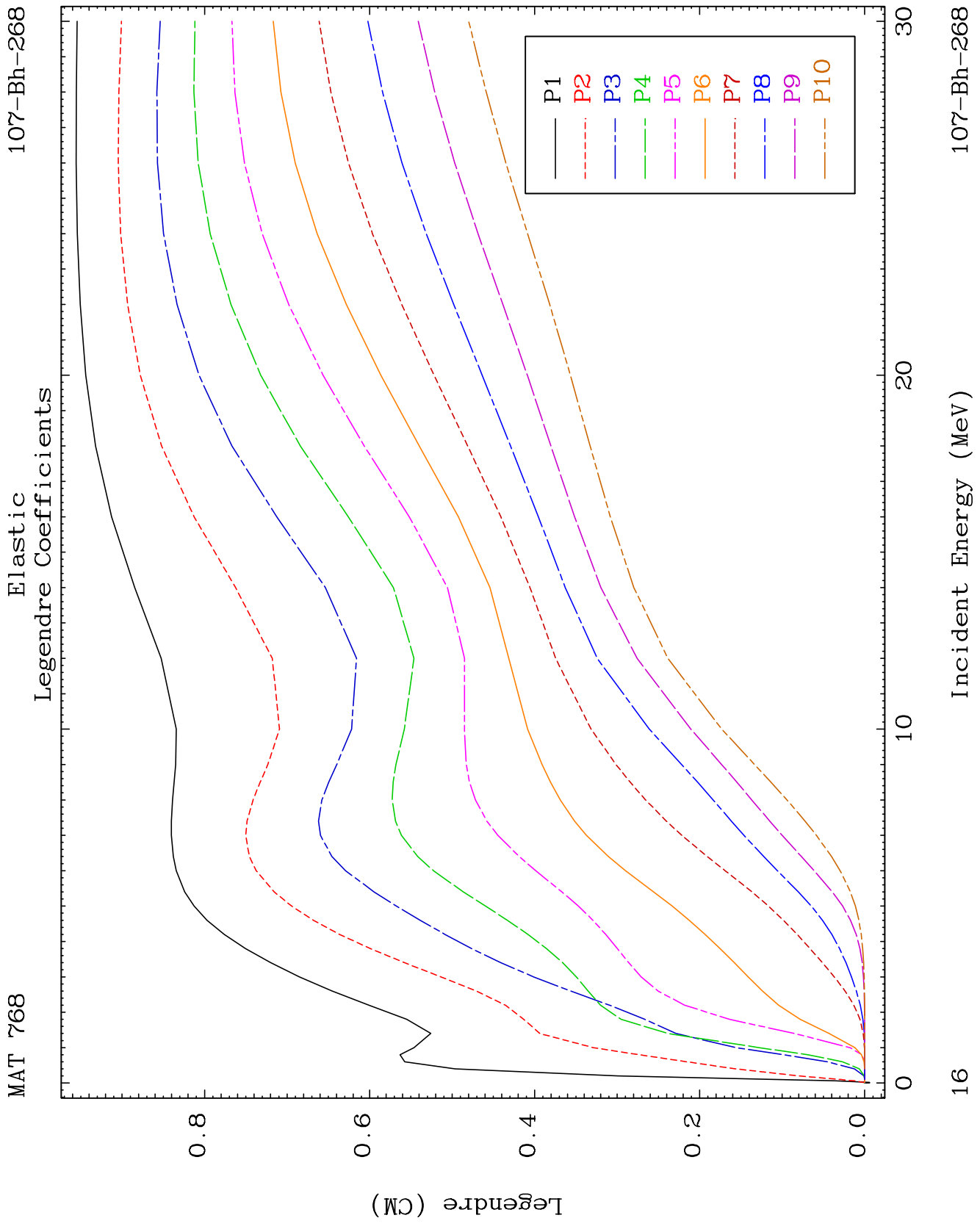
MAT 768

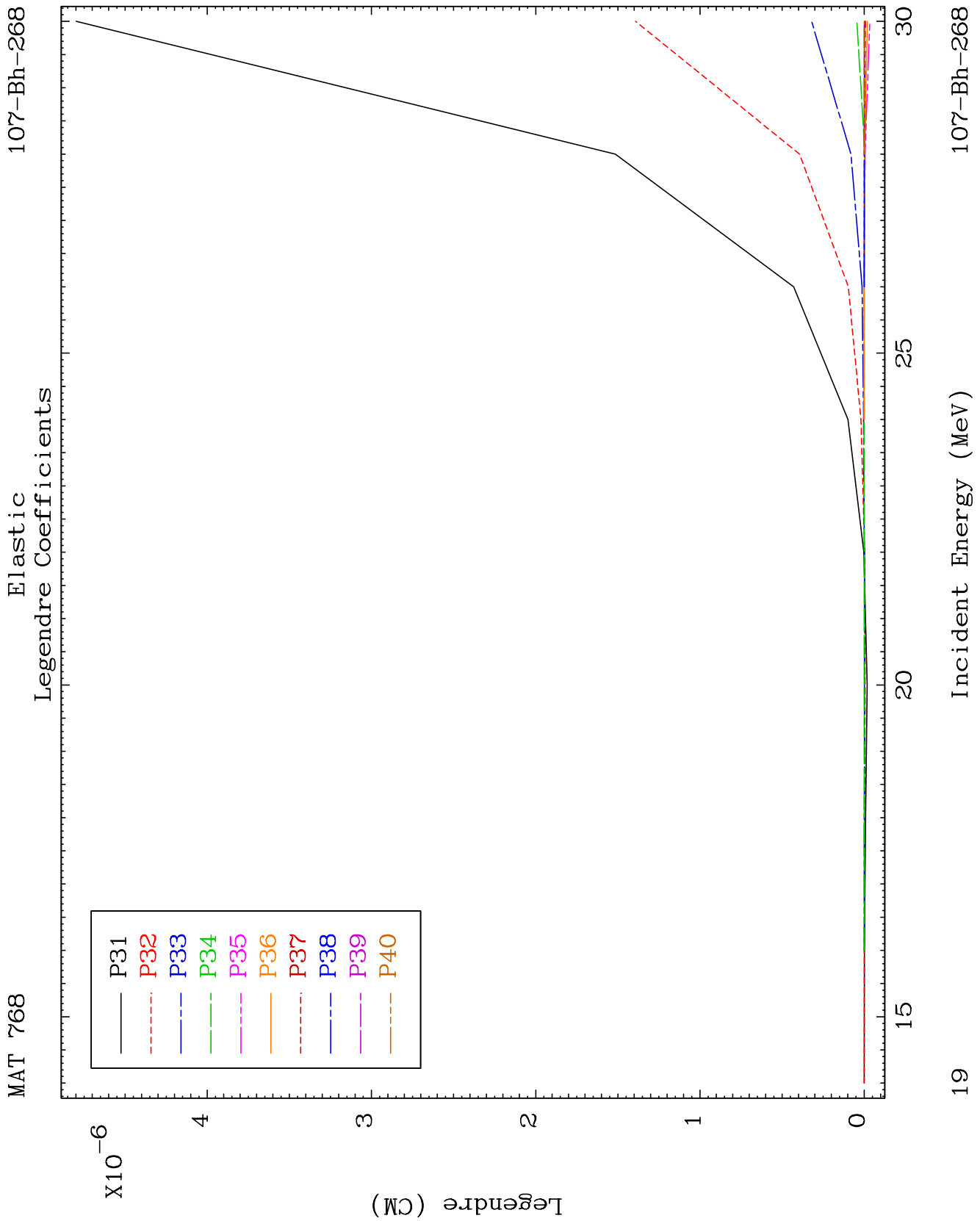
Energy Release
Parameters

107-Bh-268





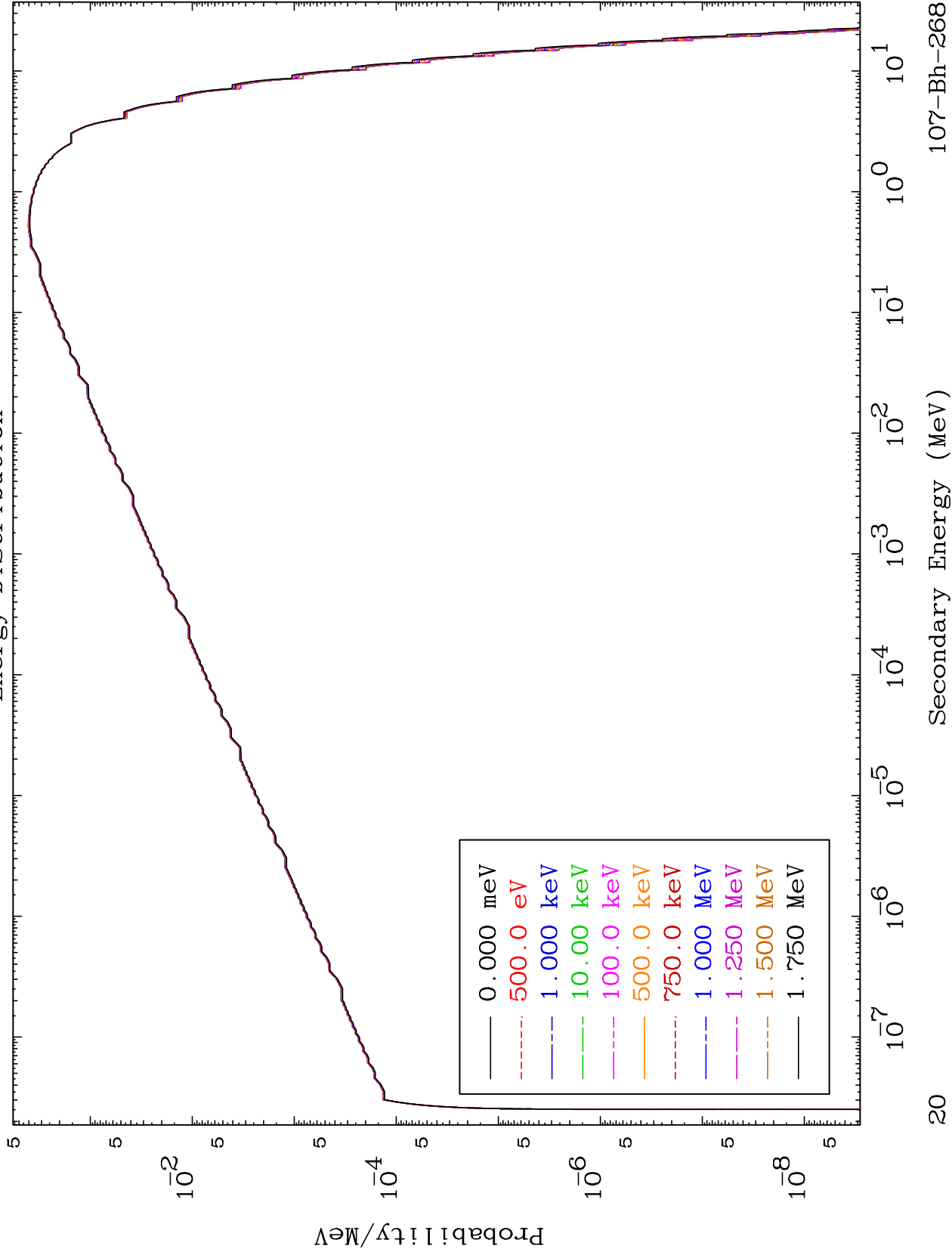




MAT 768

Fission Energy Distribution

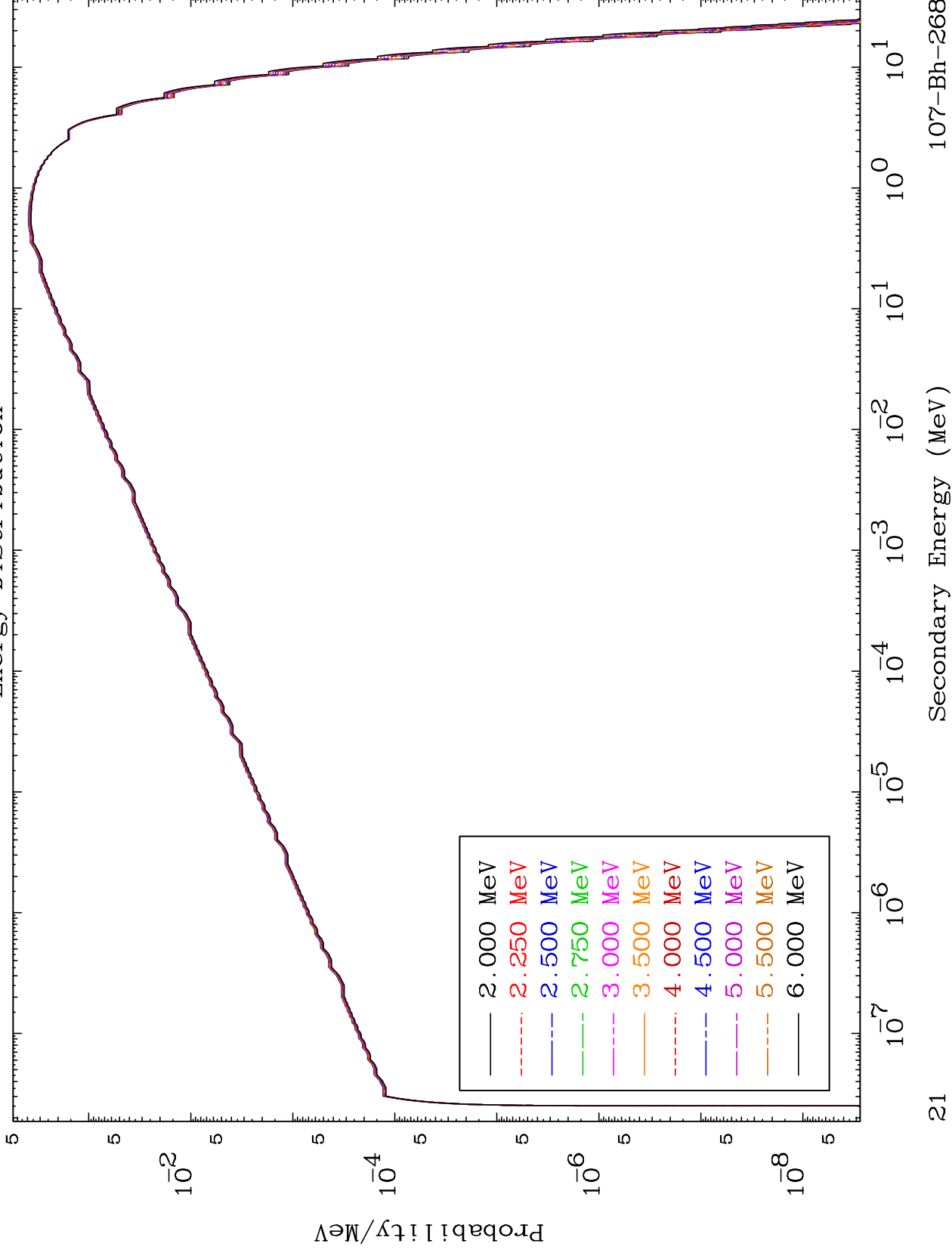
107-Bh-268



MAT 768

Fission Energy Distribution

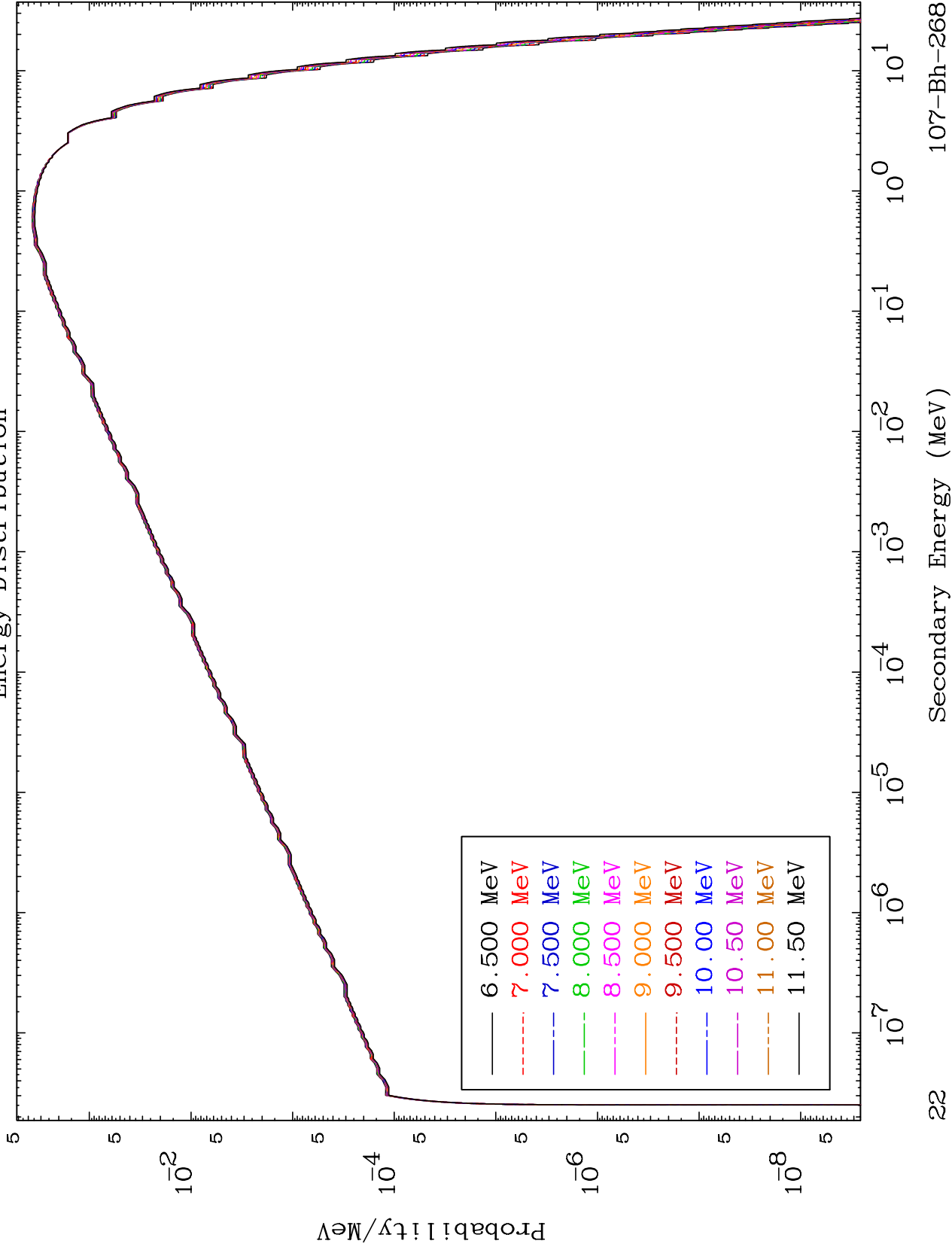
107-Bh-268



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Fission Energy Distribution

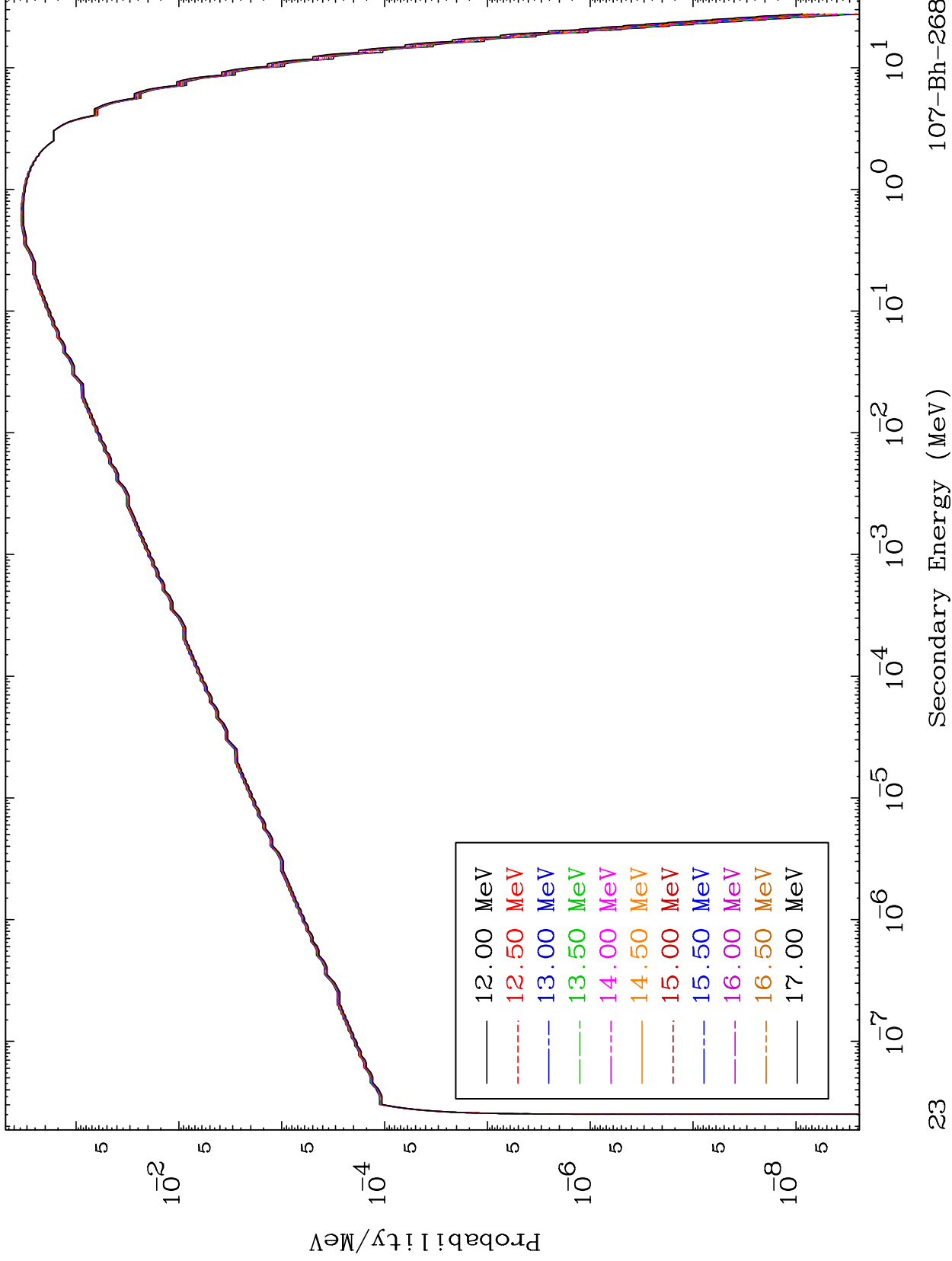
107-Bh-268



MAT 768

Fission Energy Distribution

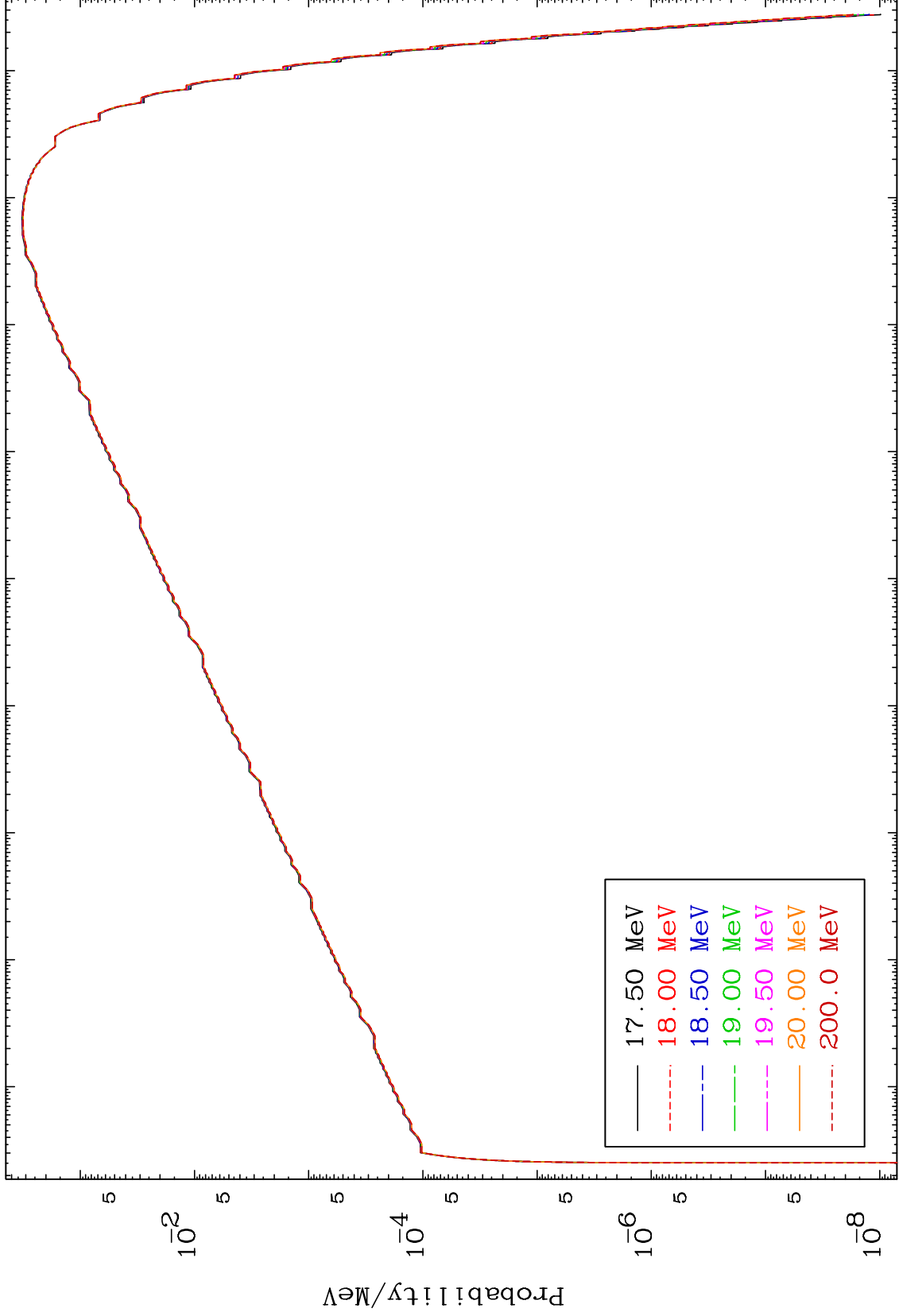
107-Bh-268



MAT 768

Fission Energy Distribution

107-Bh-268

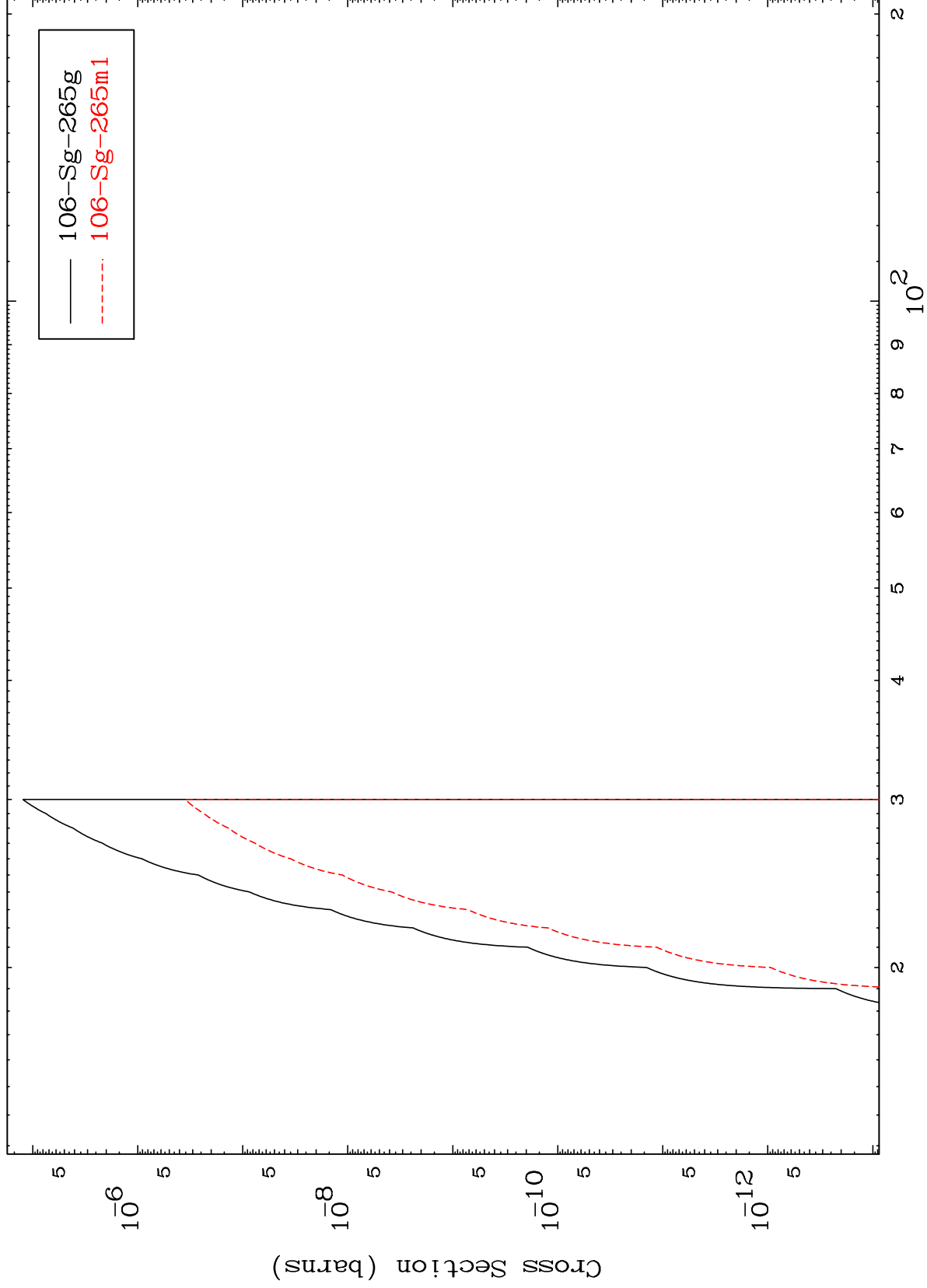


MAT 768

(n,2n) d

107-Bh-268

Radionuclide Production Cross Section



25

Incident Energy (MeV)

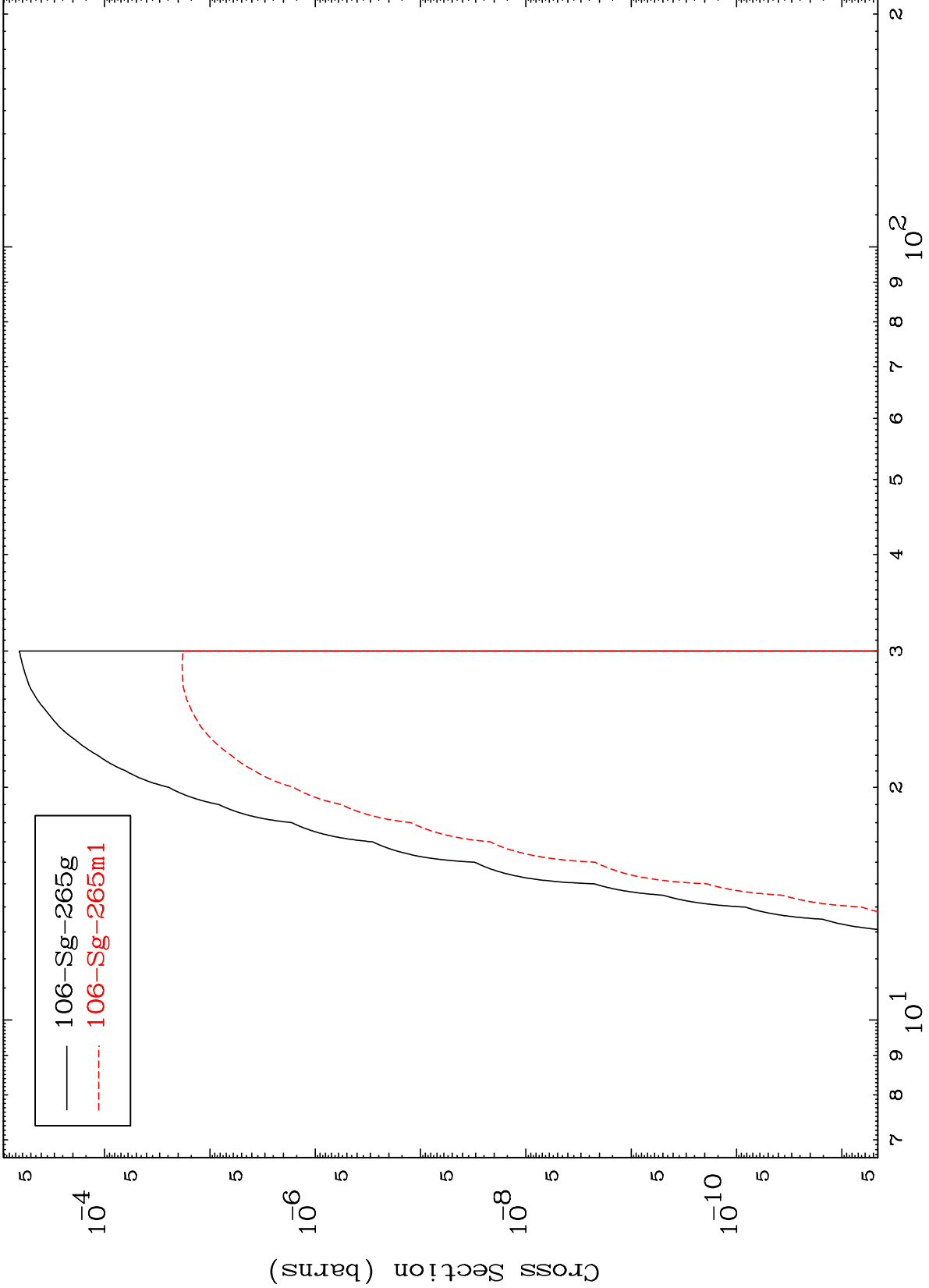
107-Bh-268

MAT 768

(n,n') t

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Radionuclide Production Cross Section



Legend:
— 106-Sg-265g
- - - 106-Sg-265m1

26

Incident Energy (MeV)

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