

Program EVALPLOT  
(Version 2017-1)

by

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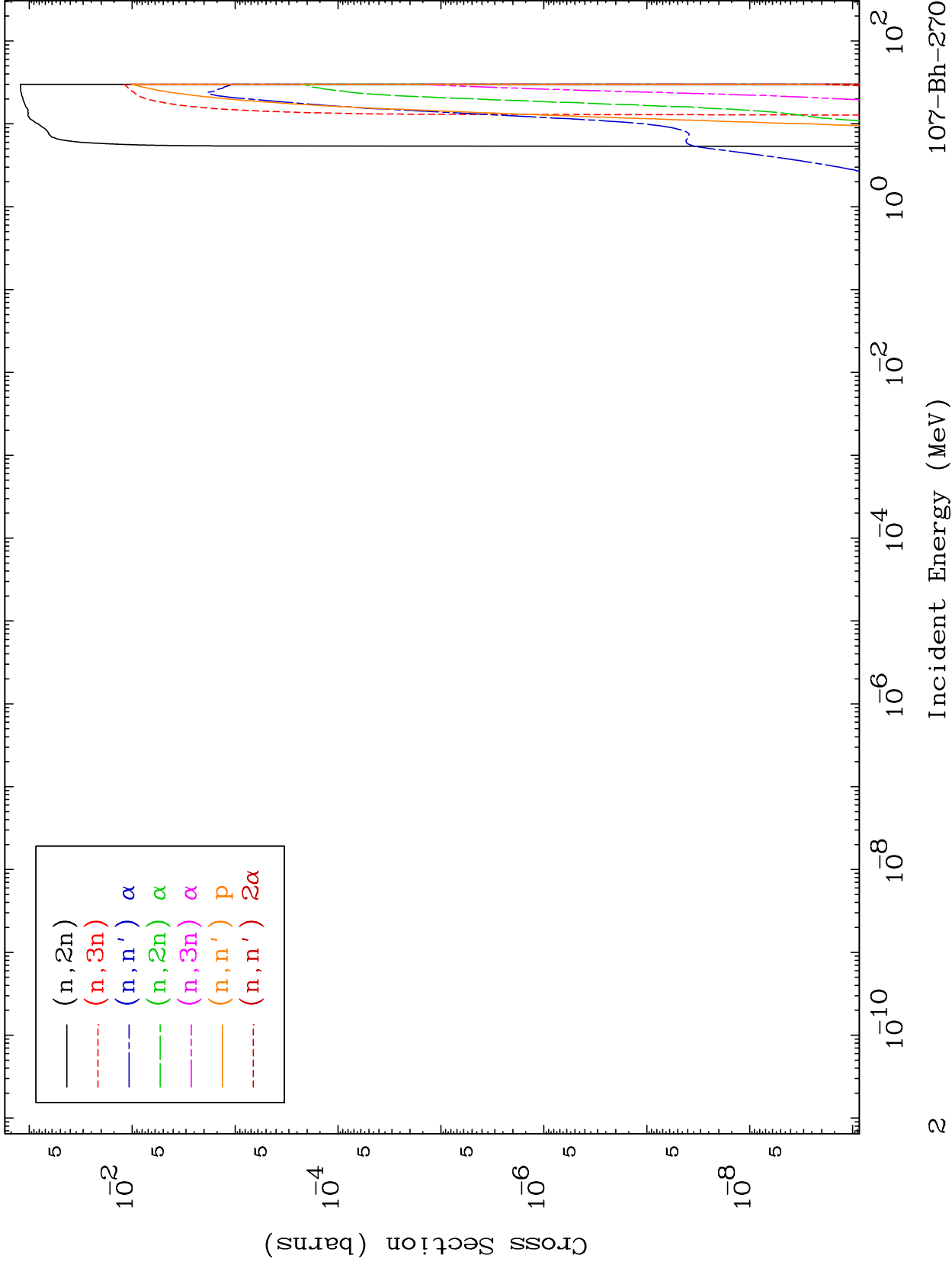
Press Mouse Button to Start



MAT 770

Neutron Production  
293 Kelvin Cross Sections

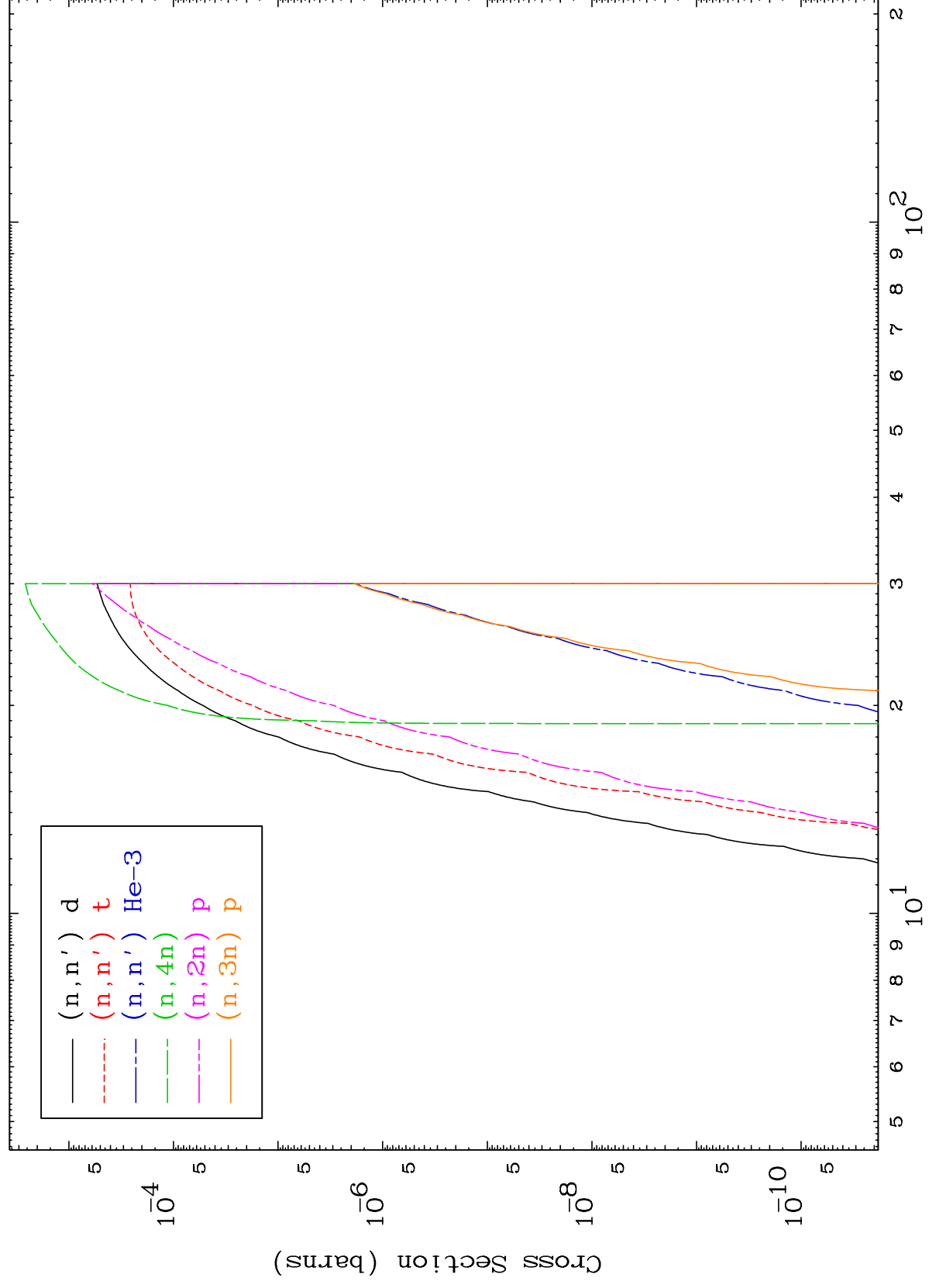
107-Bh-270



MAT 770

Neutron Production  
293 Kelvin Cross Sections

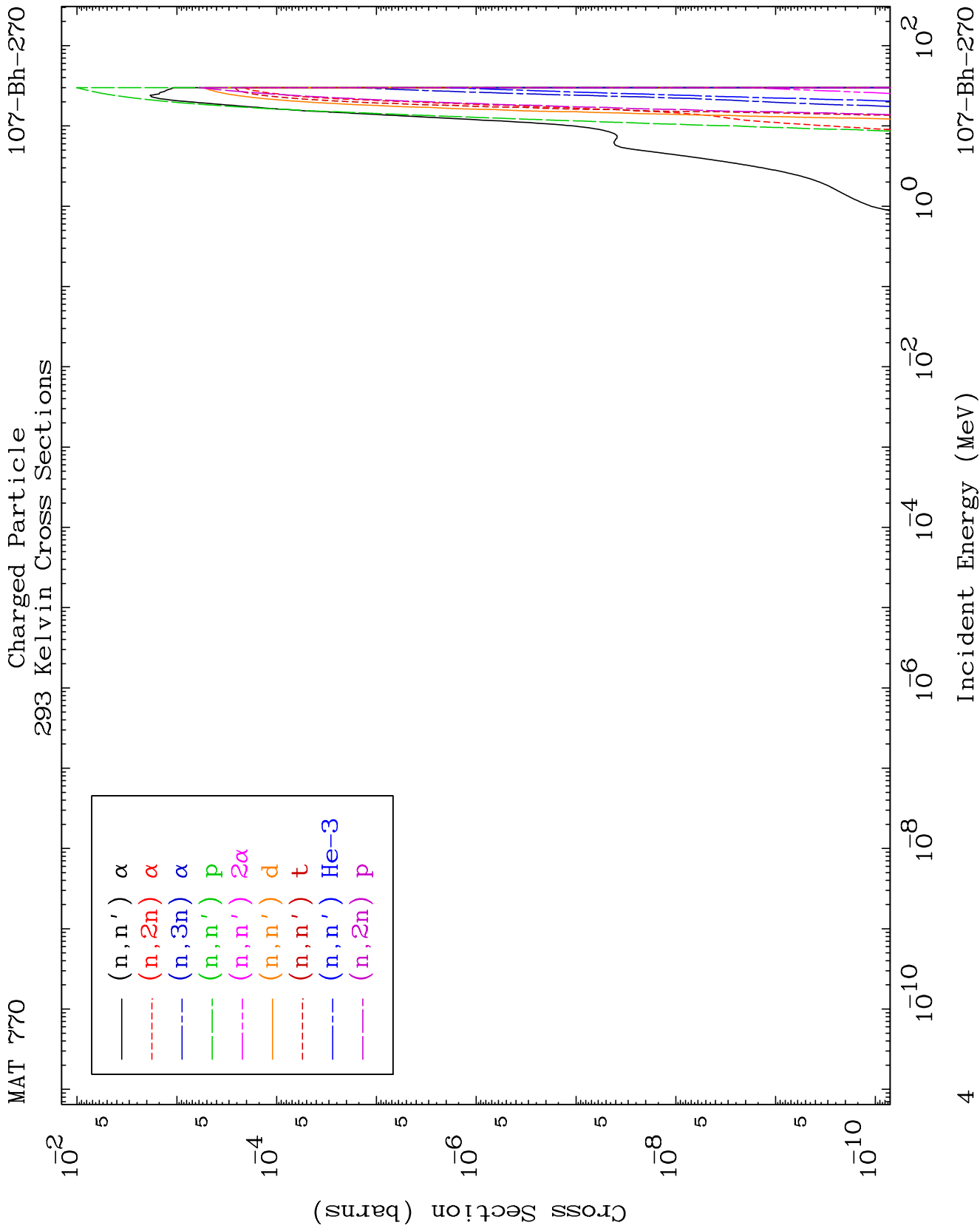
107-Bh-270

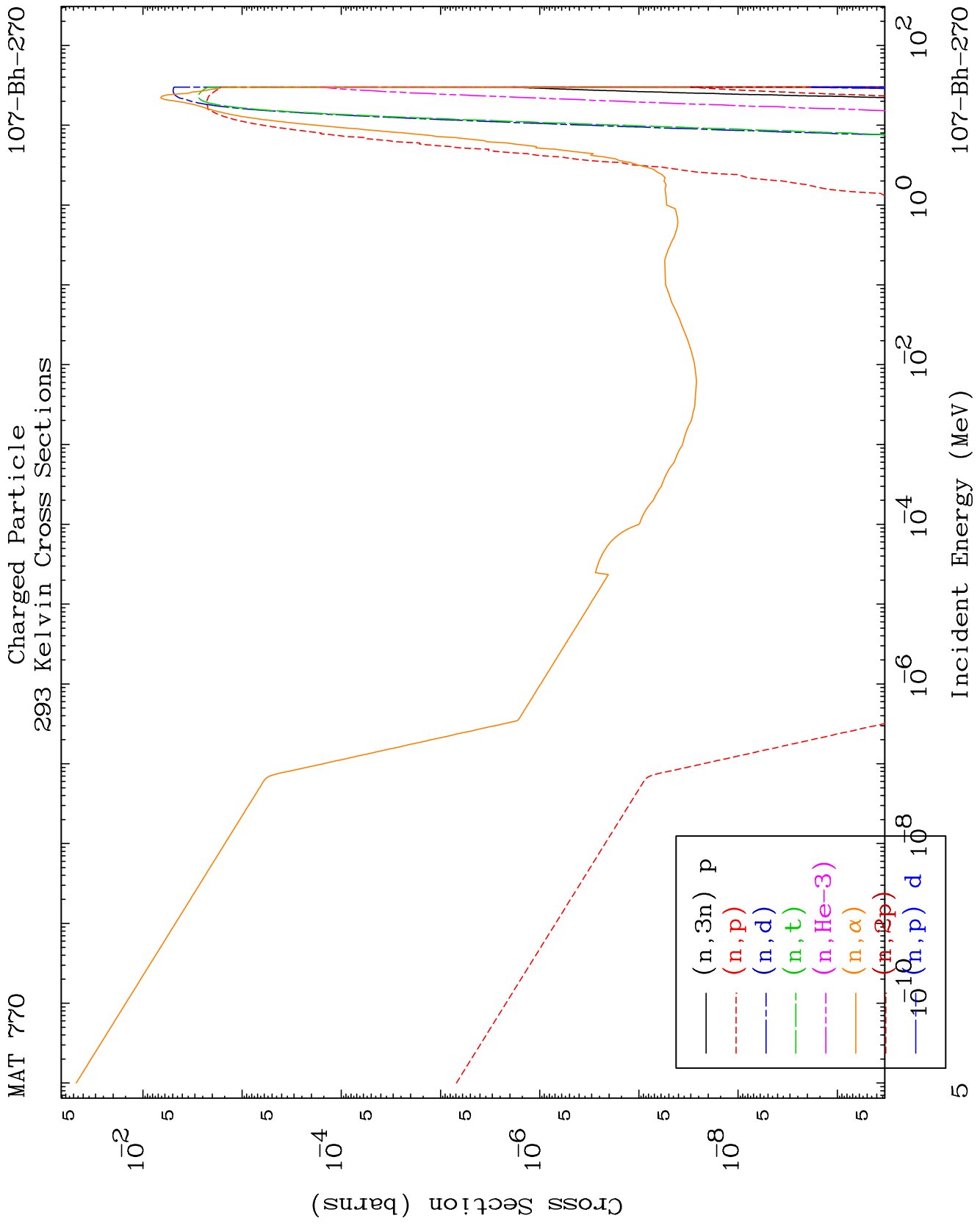


Incident Energy (MeV)

107-Bh-270

3

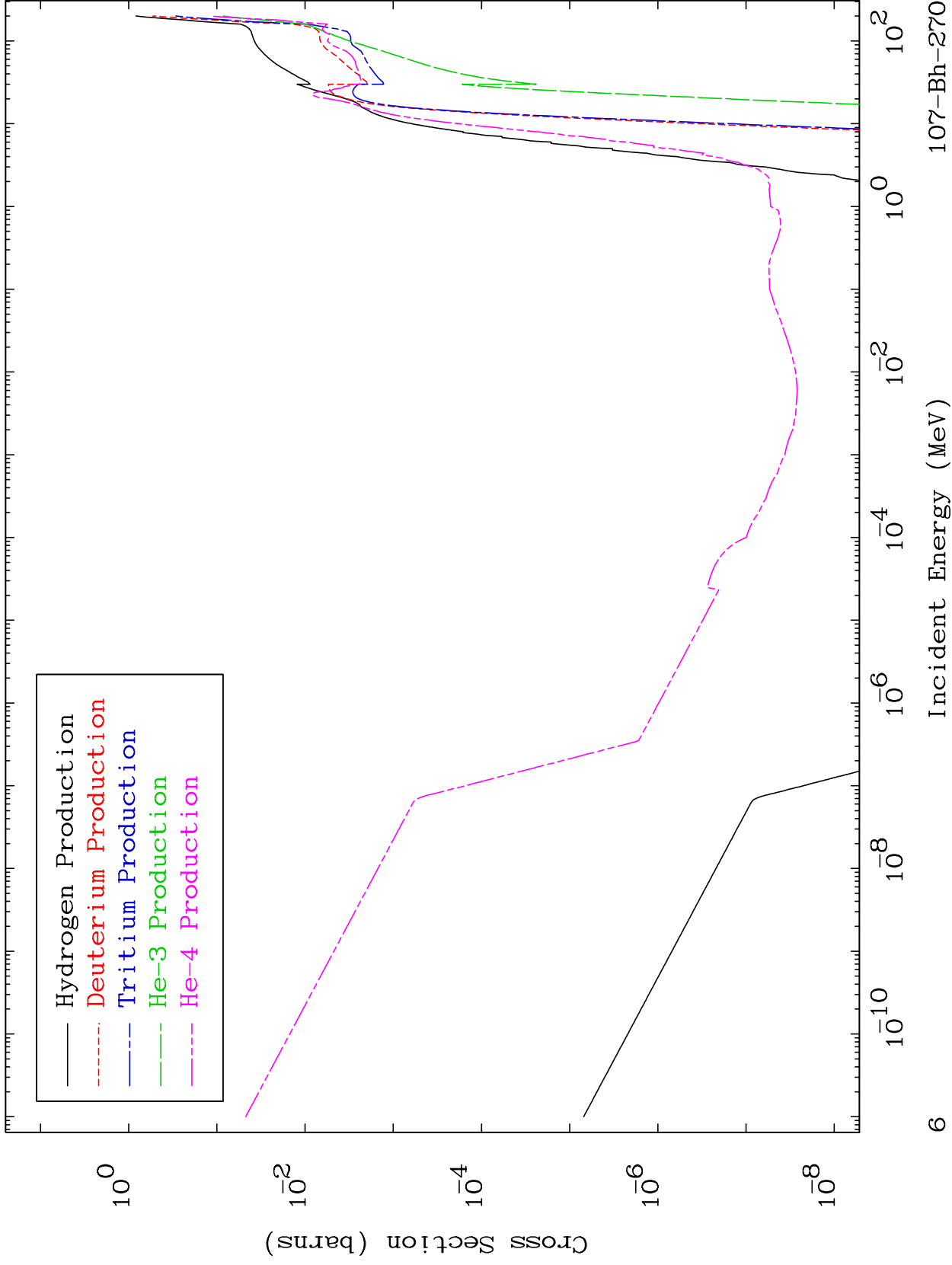


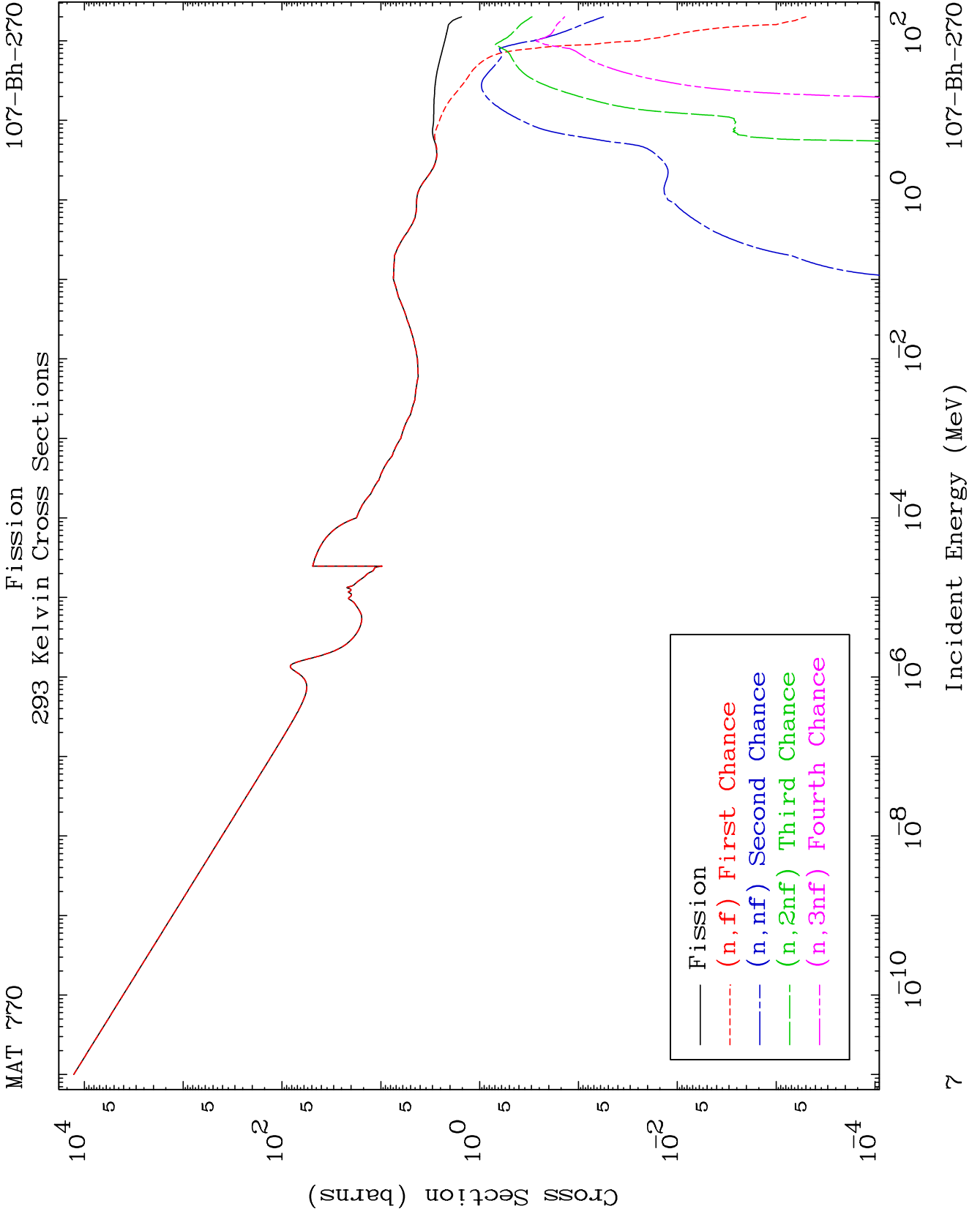


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Particle Production  
293 Kelvin Cross Sections

107-Bh-270



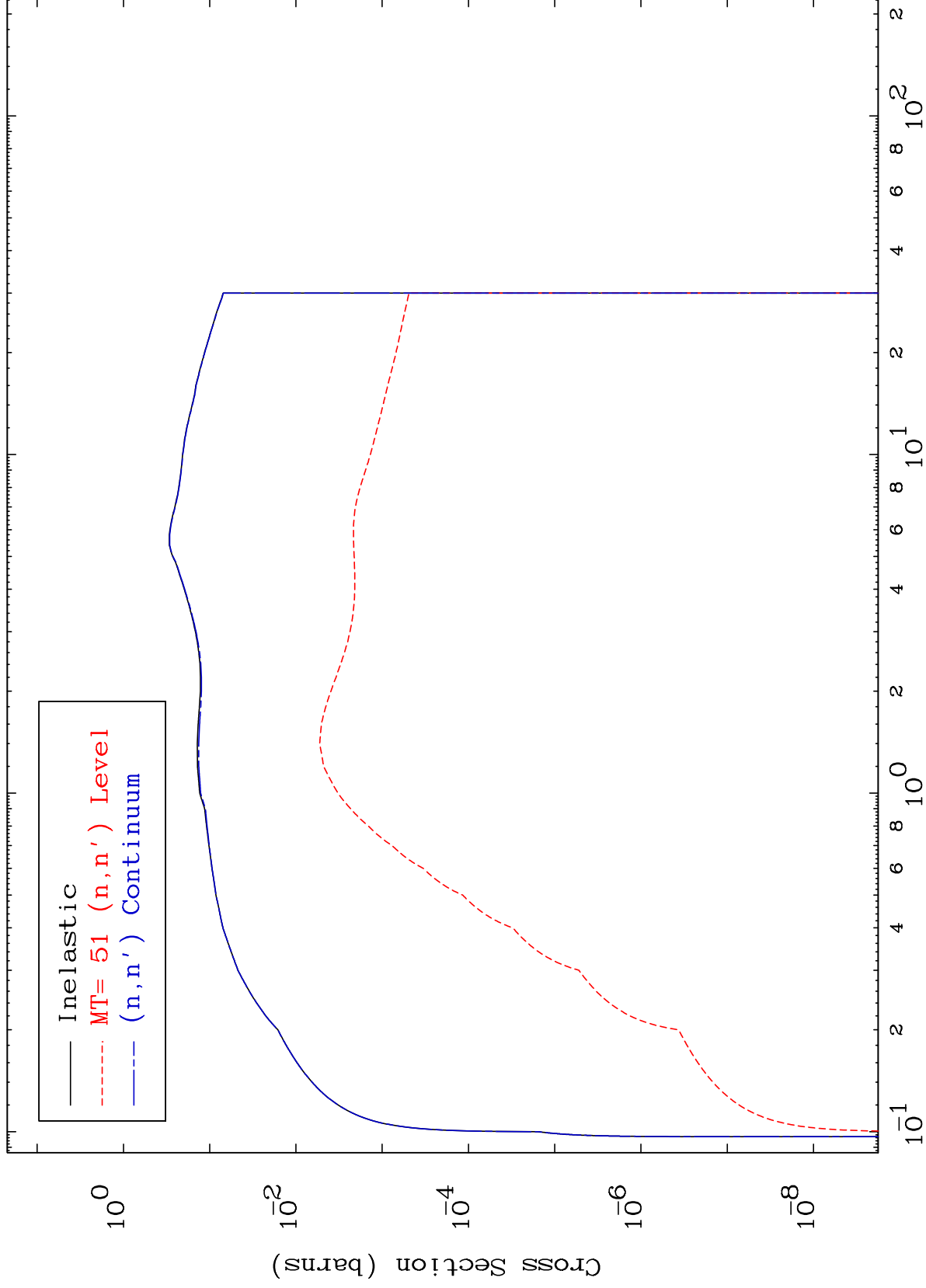




MAT 770

(n,n') Level  
293 Kelvin Cross Sections

107-Bh-270



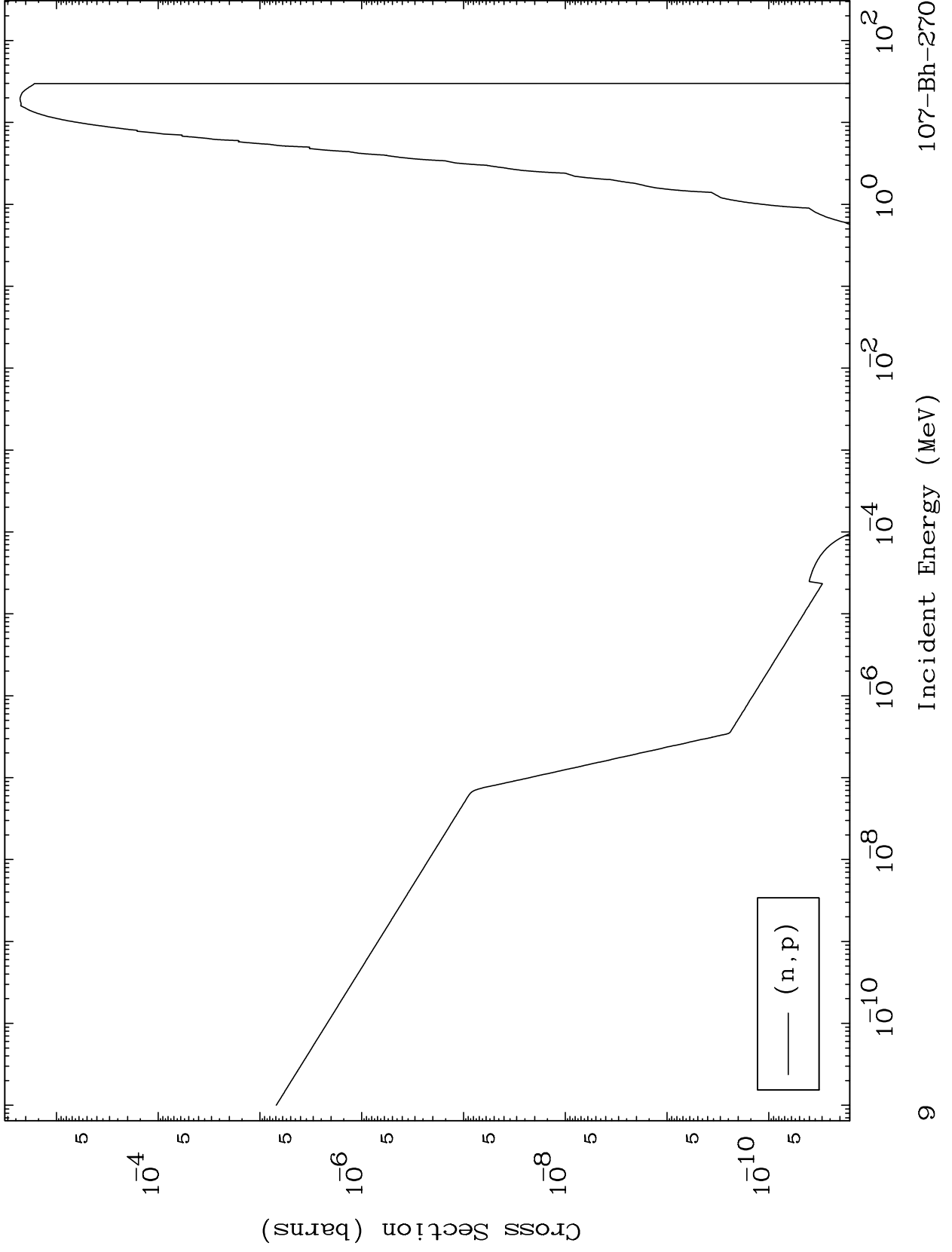
Incident Energy (MeV)

107-Bh-270

MAT 770

(n,p) Levels  
293 Kelvin Cross Sections

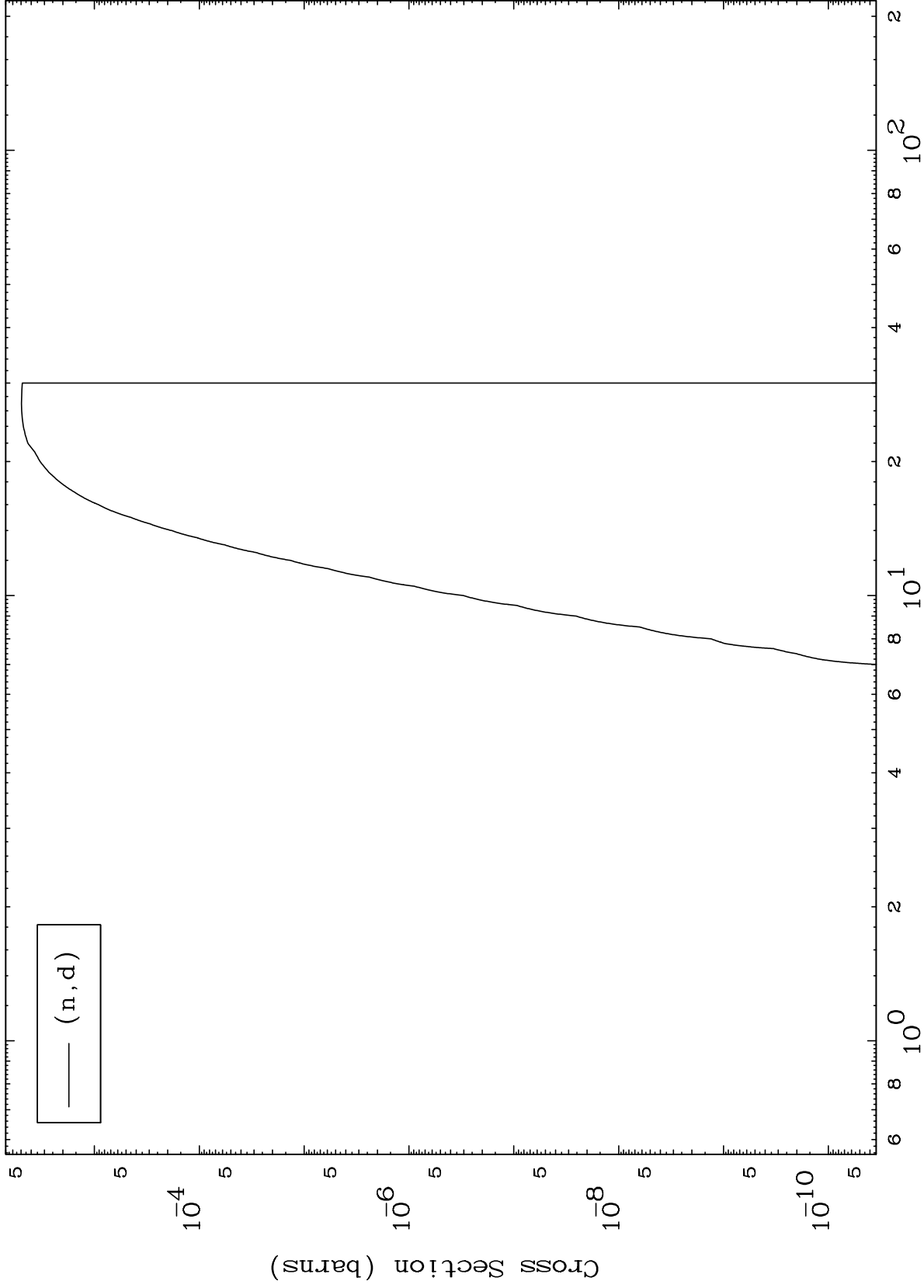
107-Bh-270



MAT 770

(n,d) Levels  
293 Kelvin Cross Sections

107-Bh-270



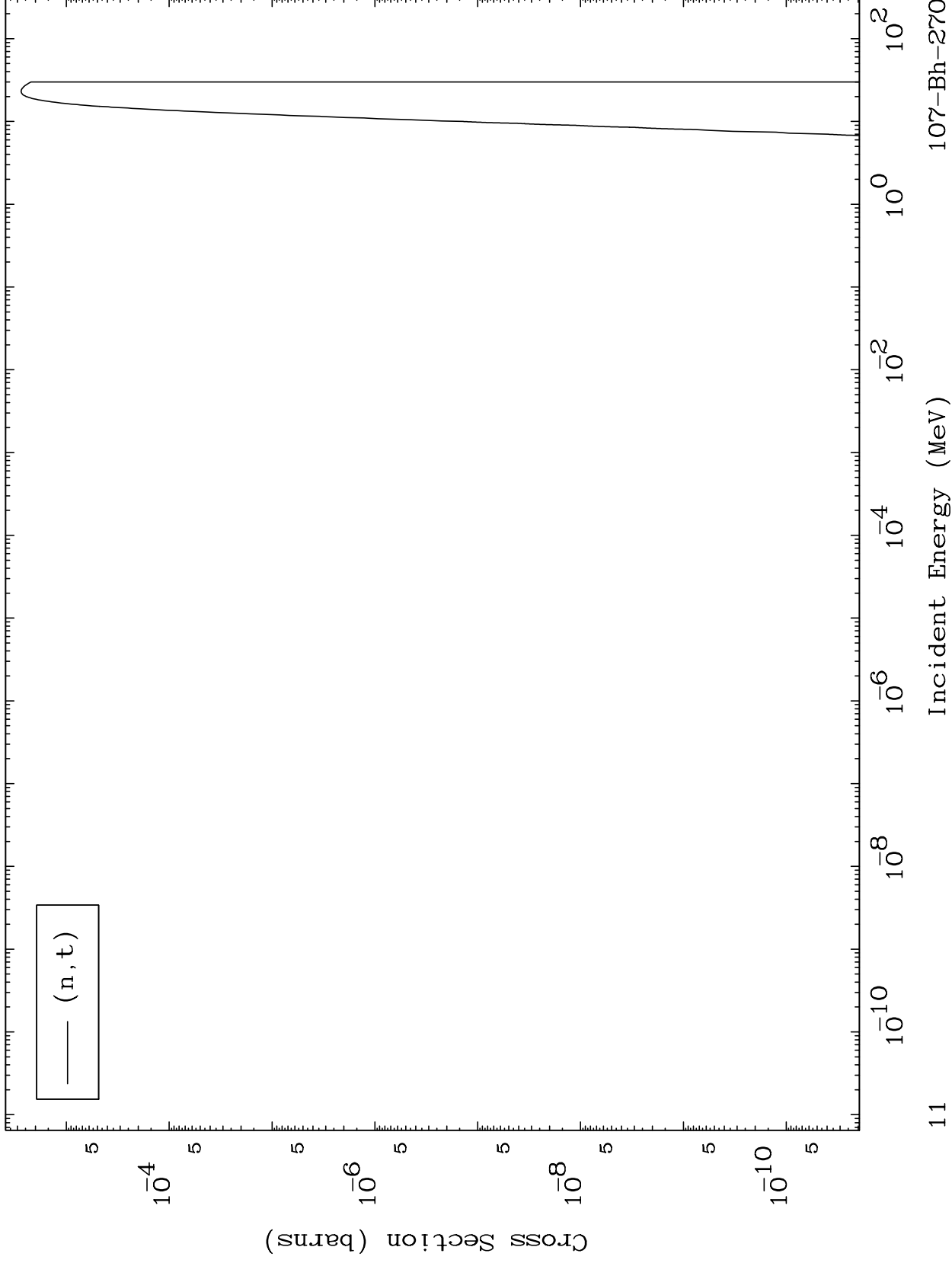
Incident Energy (MeV)

107-Bh-270

MAT 770

(n,t) Levels  
293 Kelvin Cross Sections

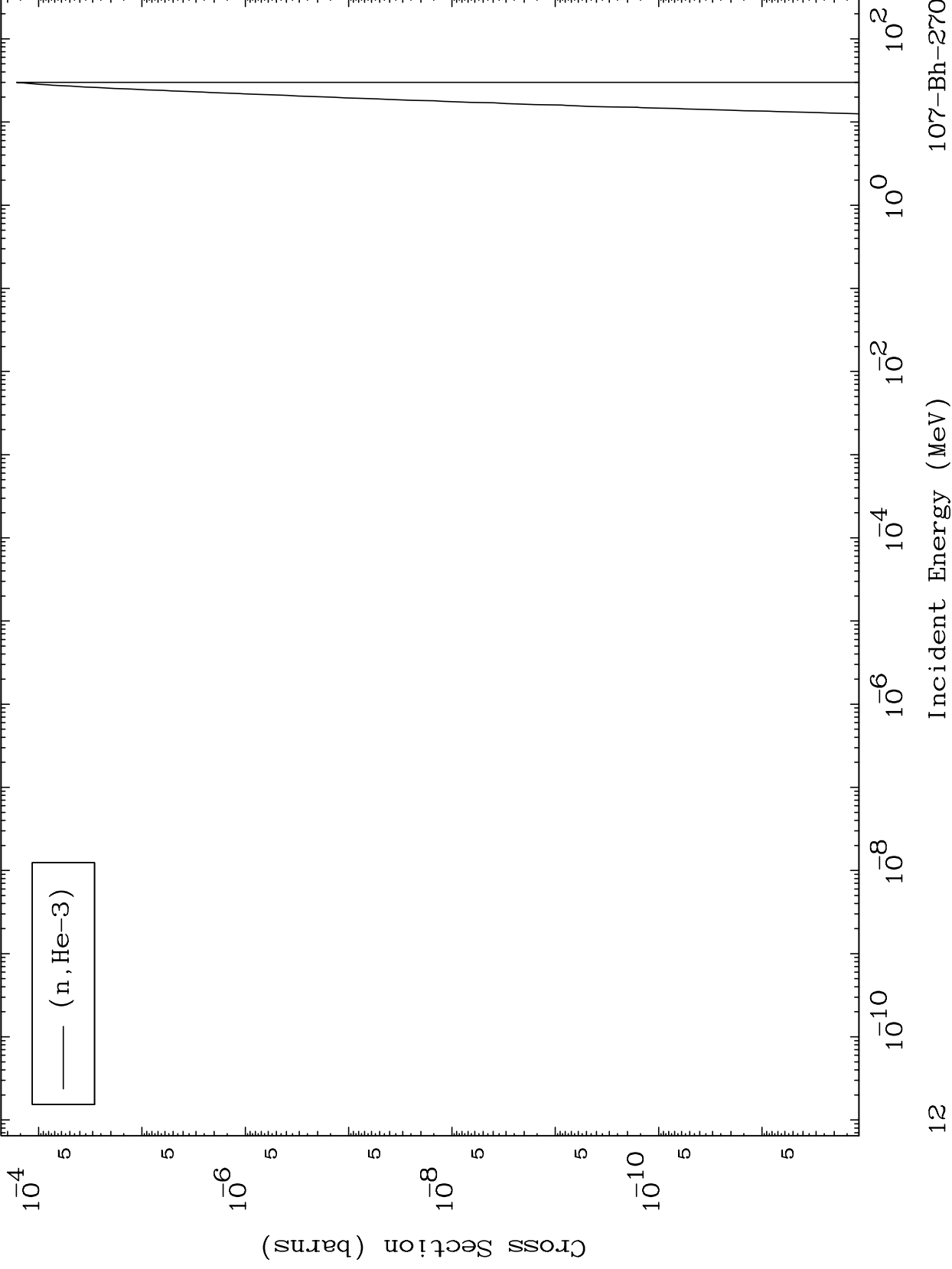
107-Bh-270

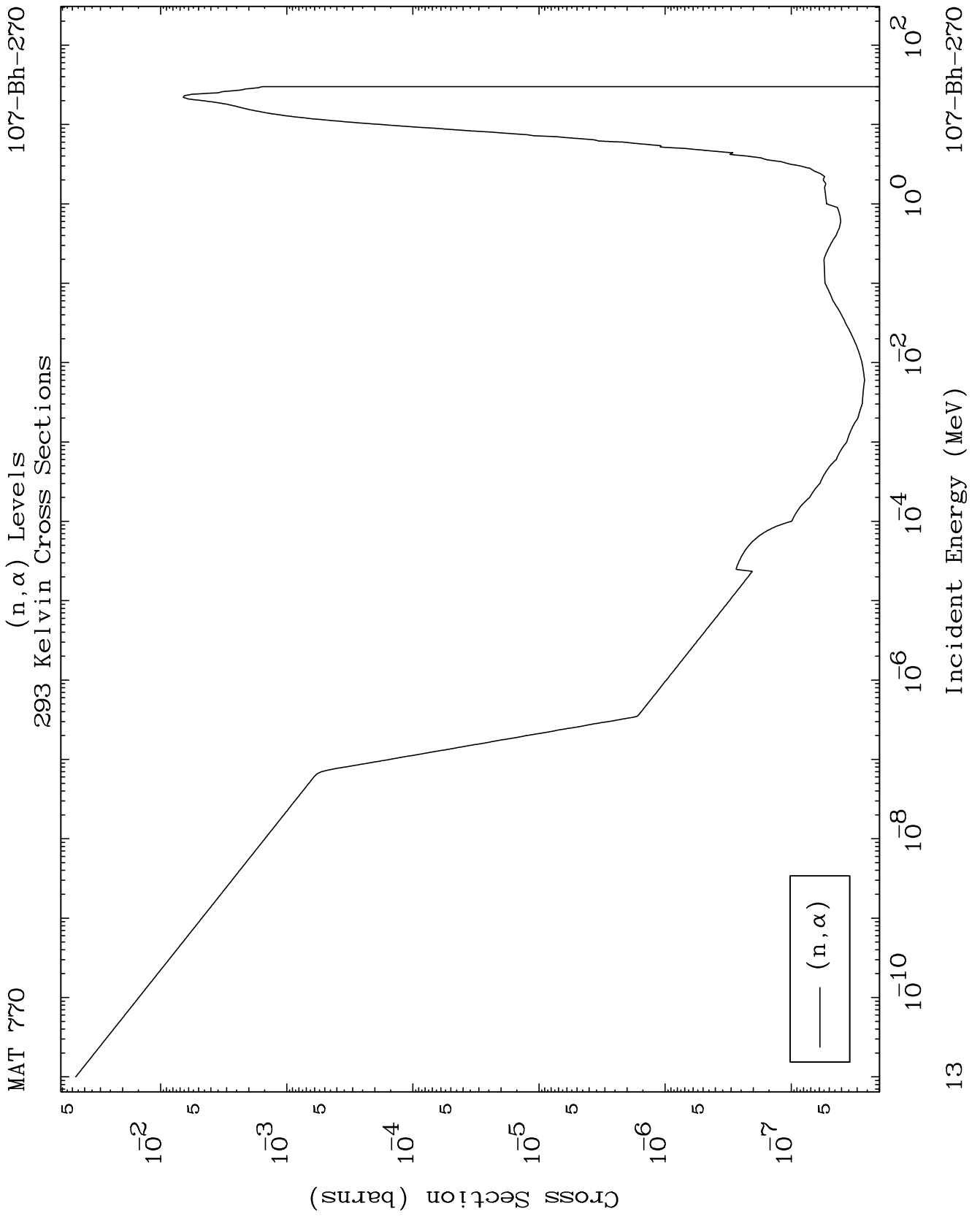


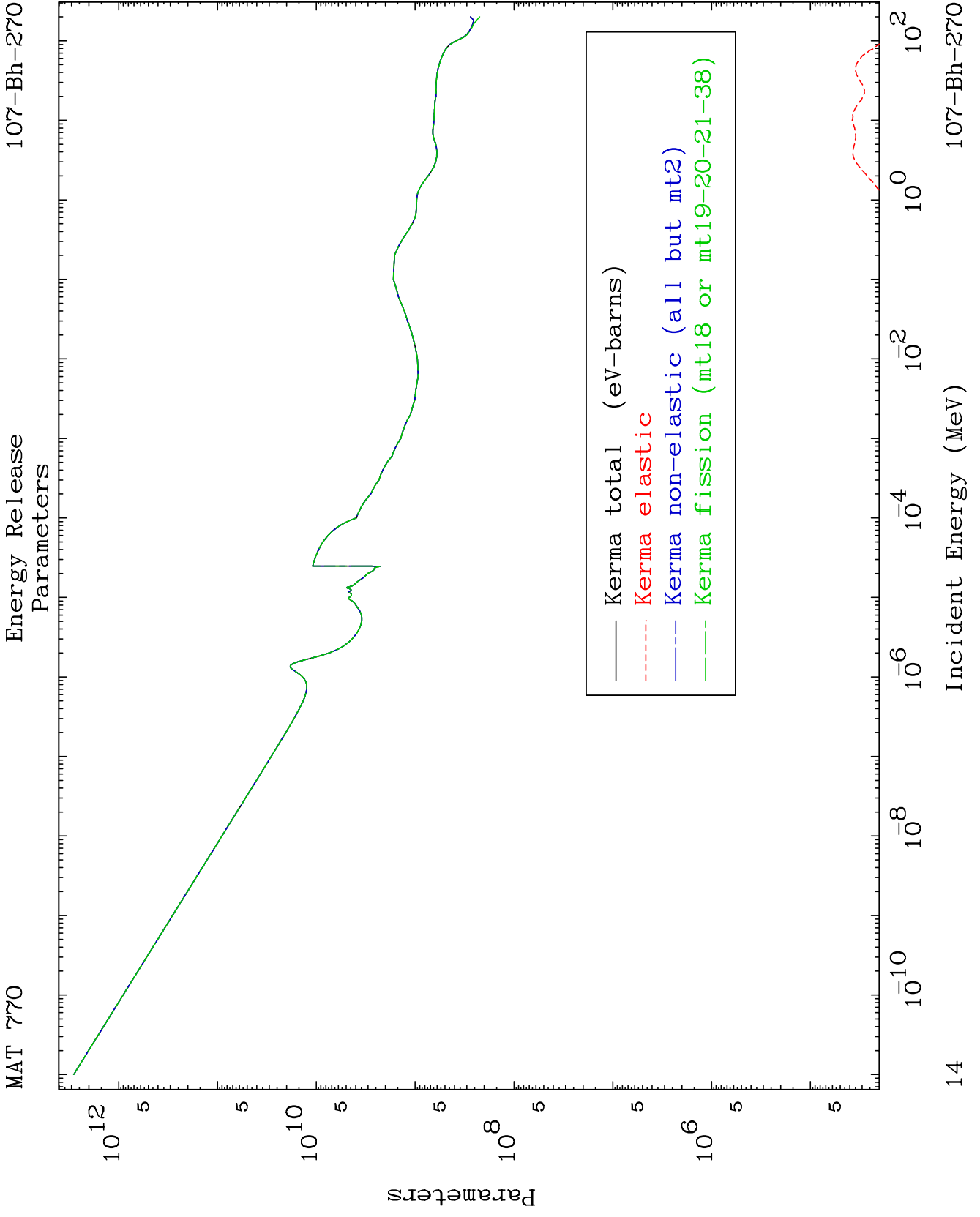
MAT 770

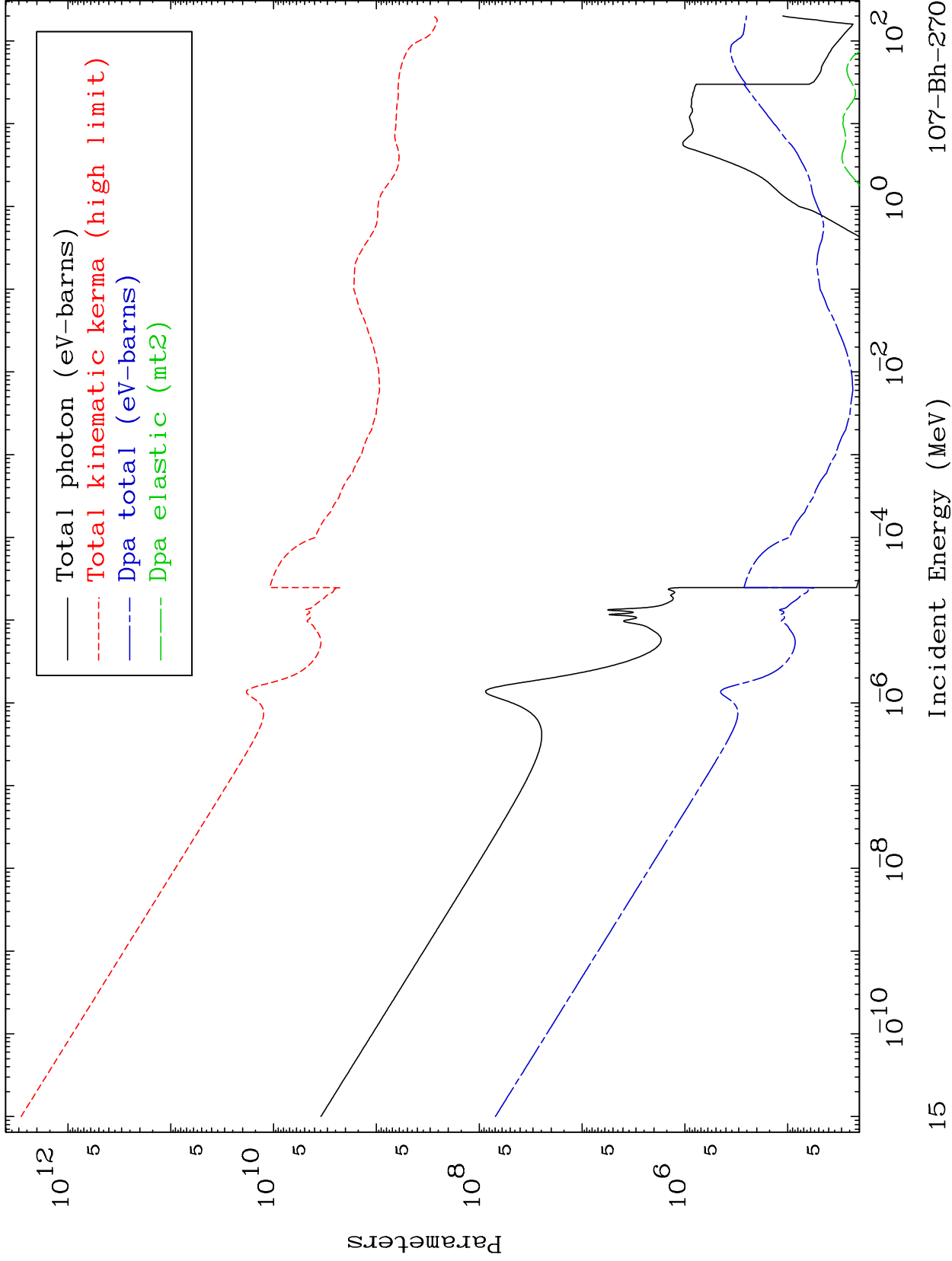
(n,He3) Levels  
293 Kelvin Cross Sections

107-Bh-270

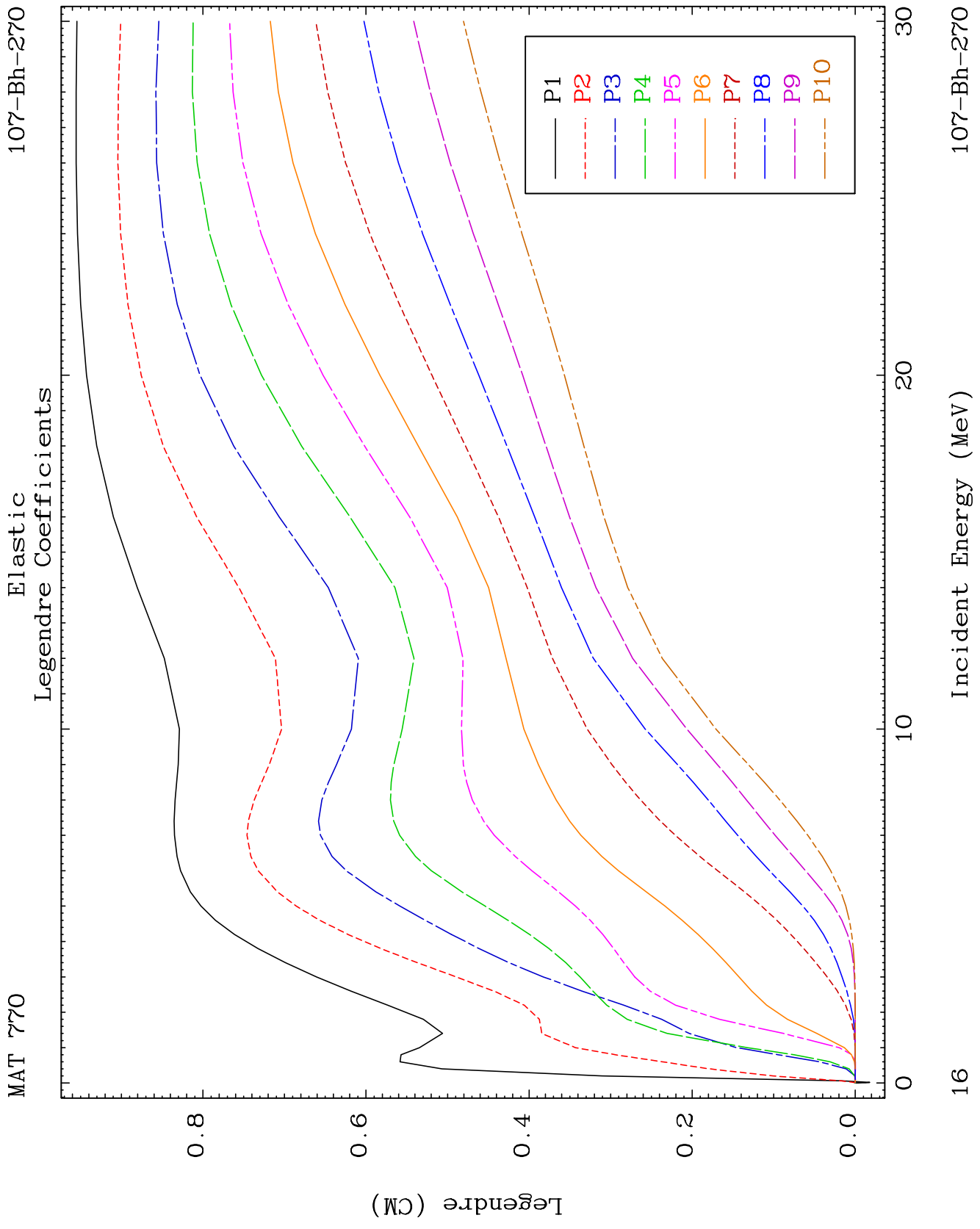


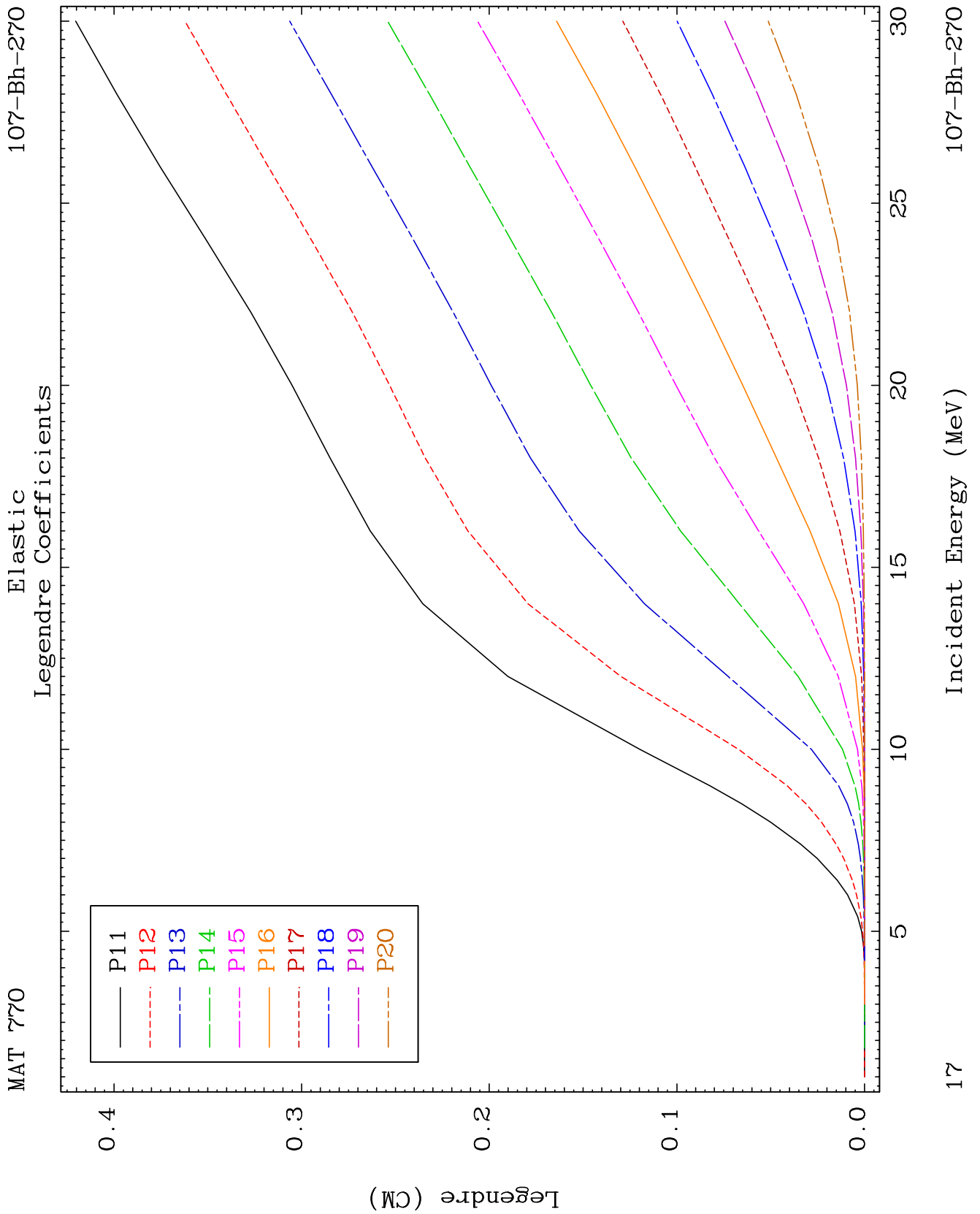




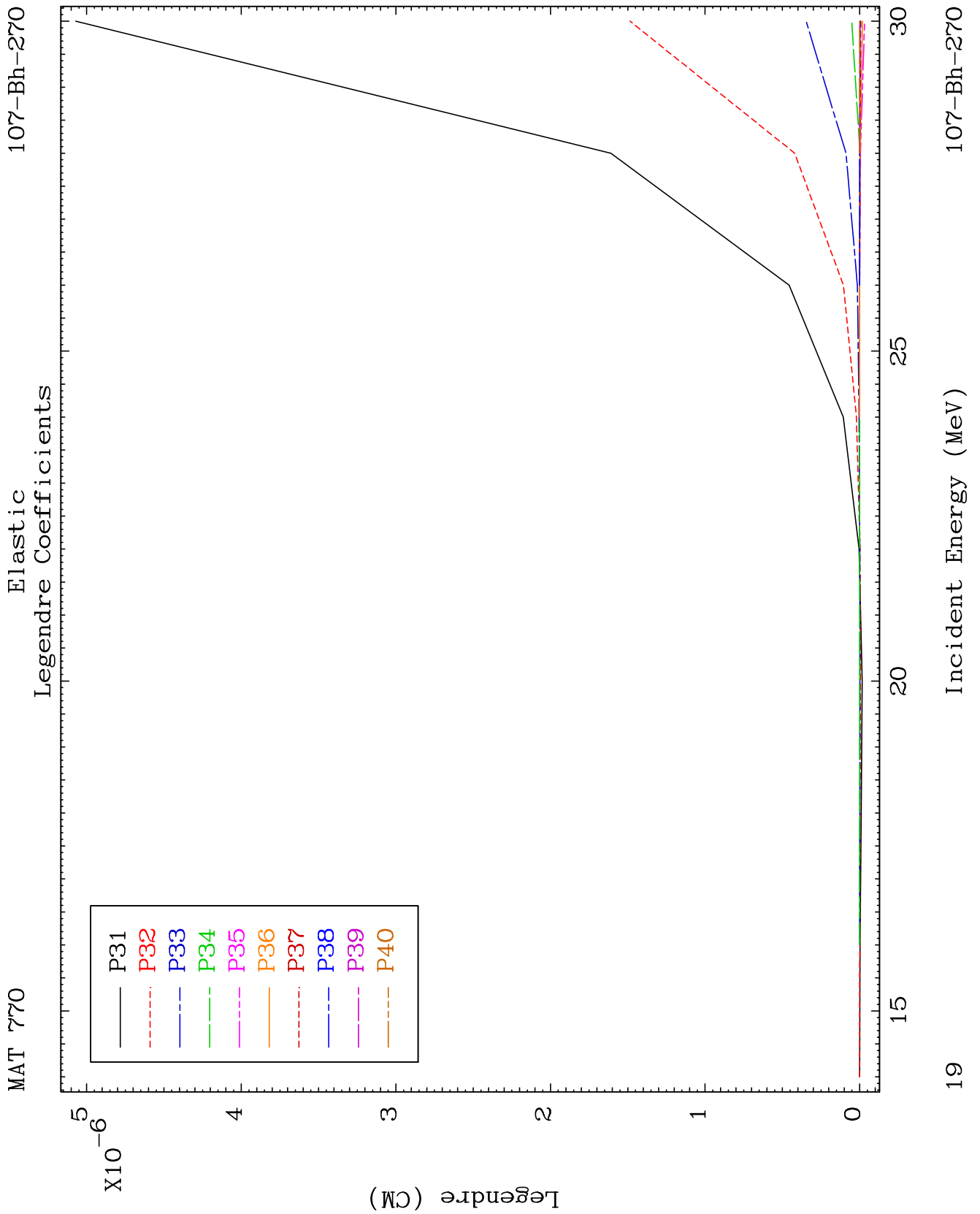












MAT 770

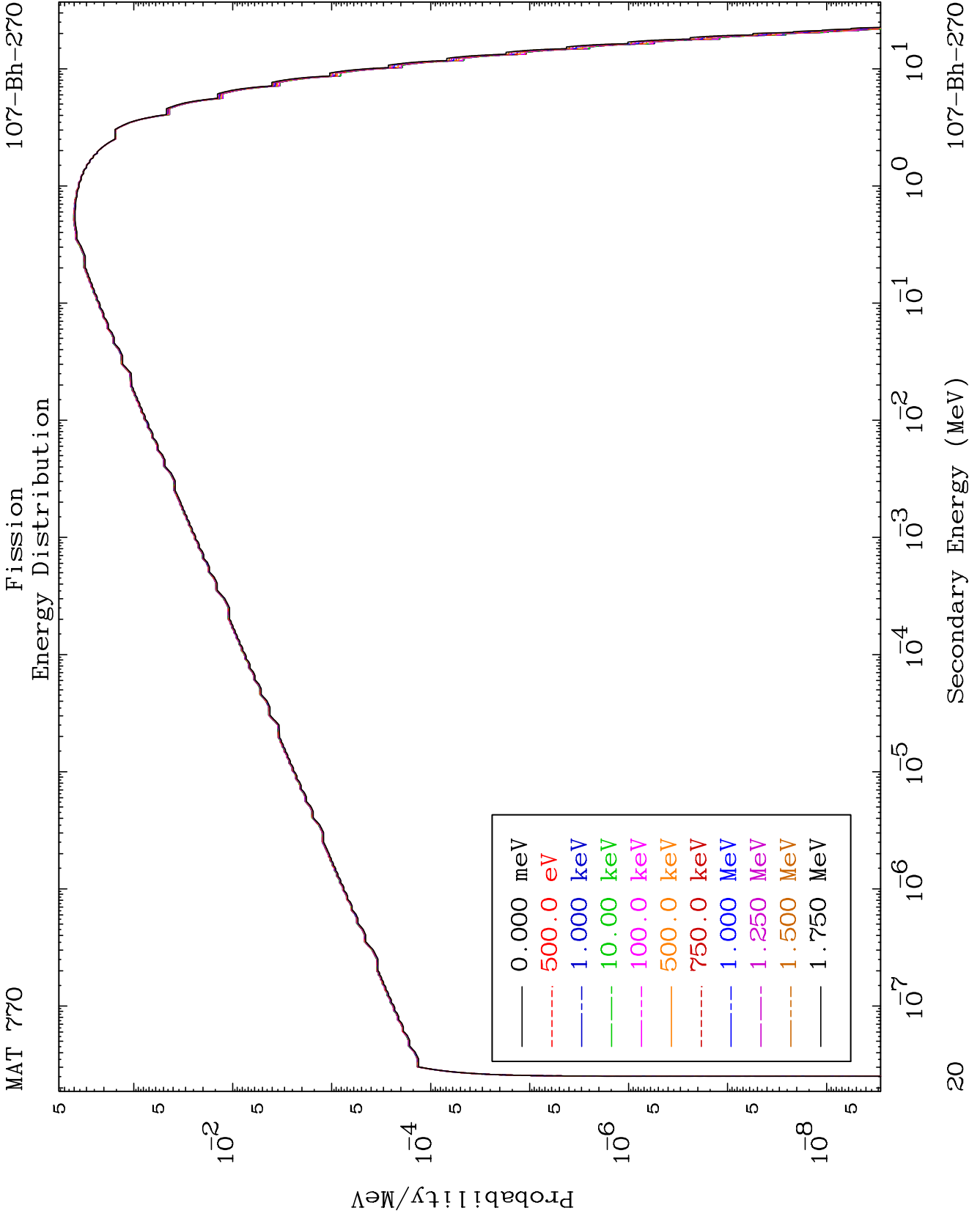
Elastic Legendre Coefficients

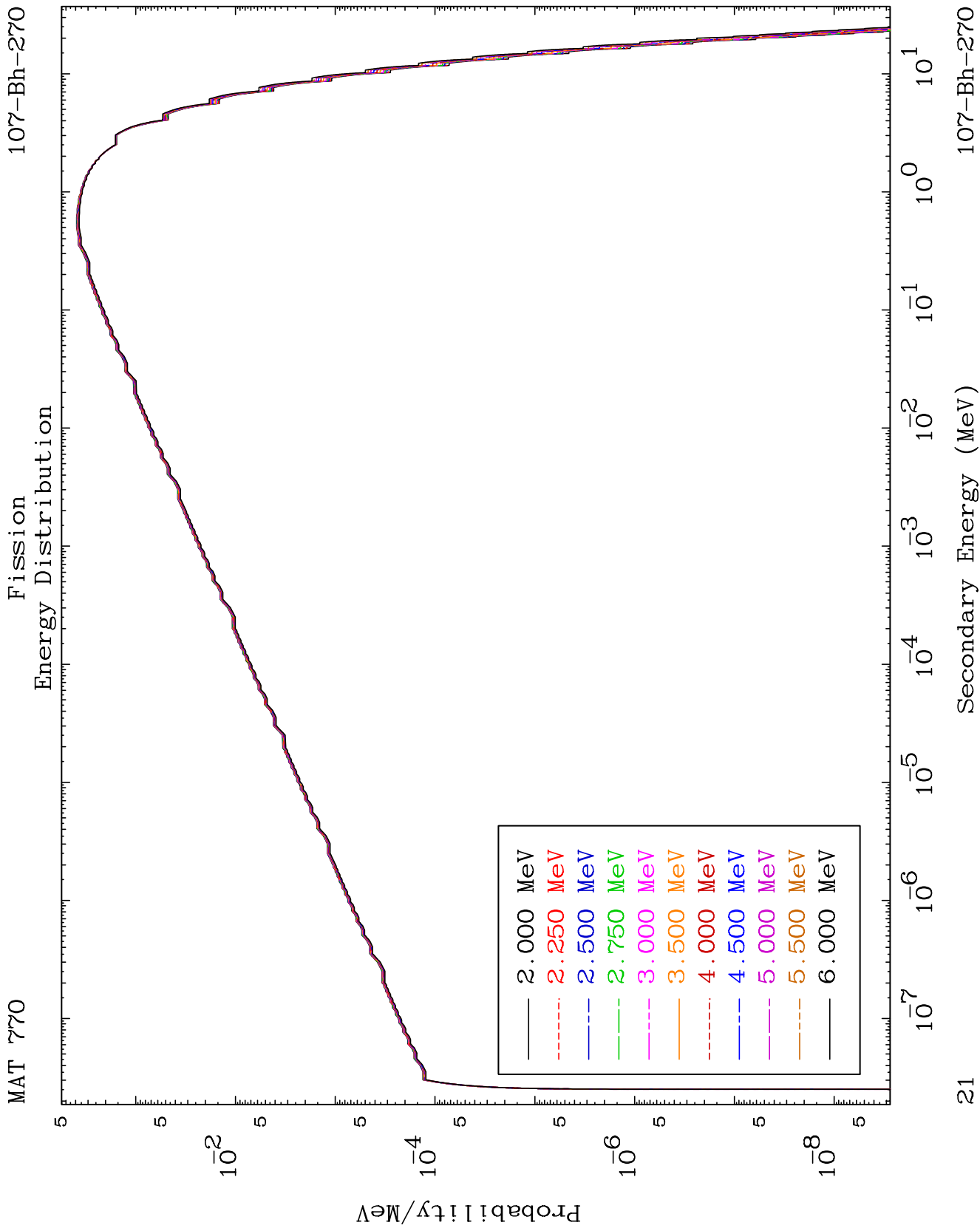
107-Bh-270

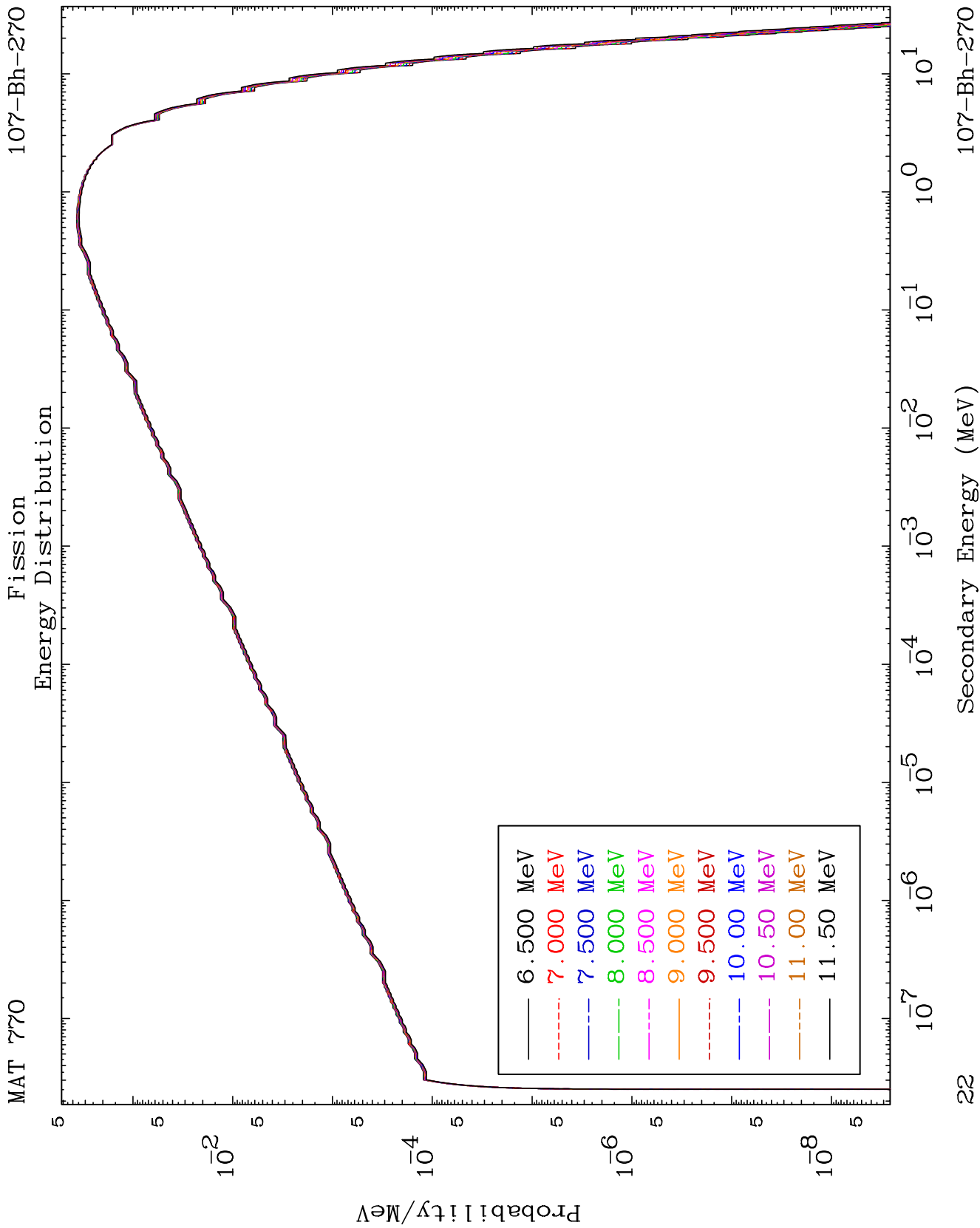
19

Incident Energy (MeV)

107-Bh-270



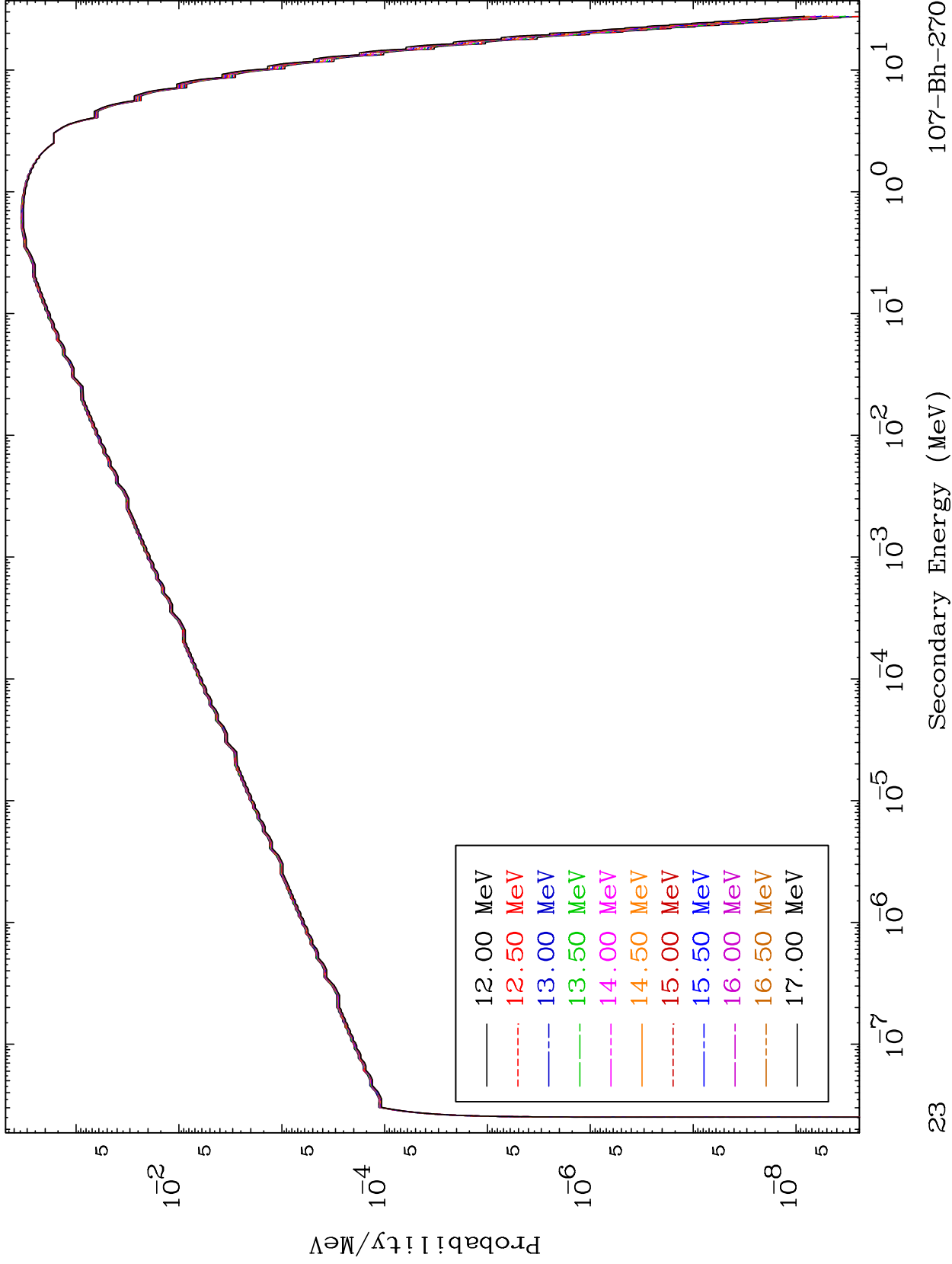




MAT 770

### Fission Energy Distribution

107-Bh-270





MAT 770

Fission  
Energy Distribution

107-Bh-270

