

Program EVALPLOT
(Version 2017-1)

by

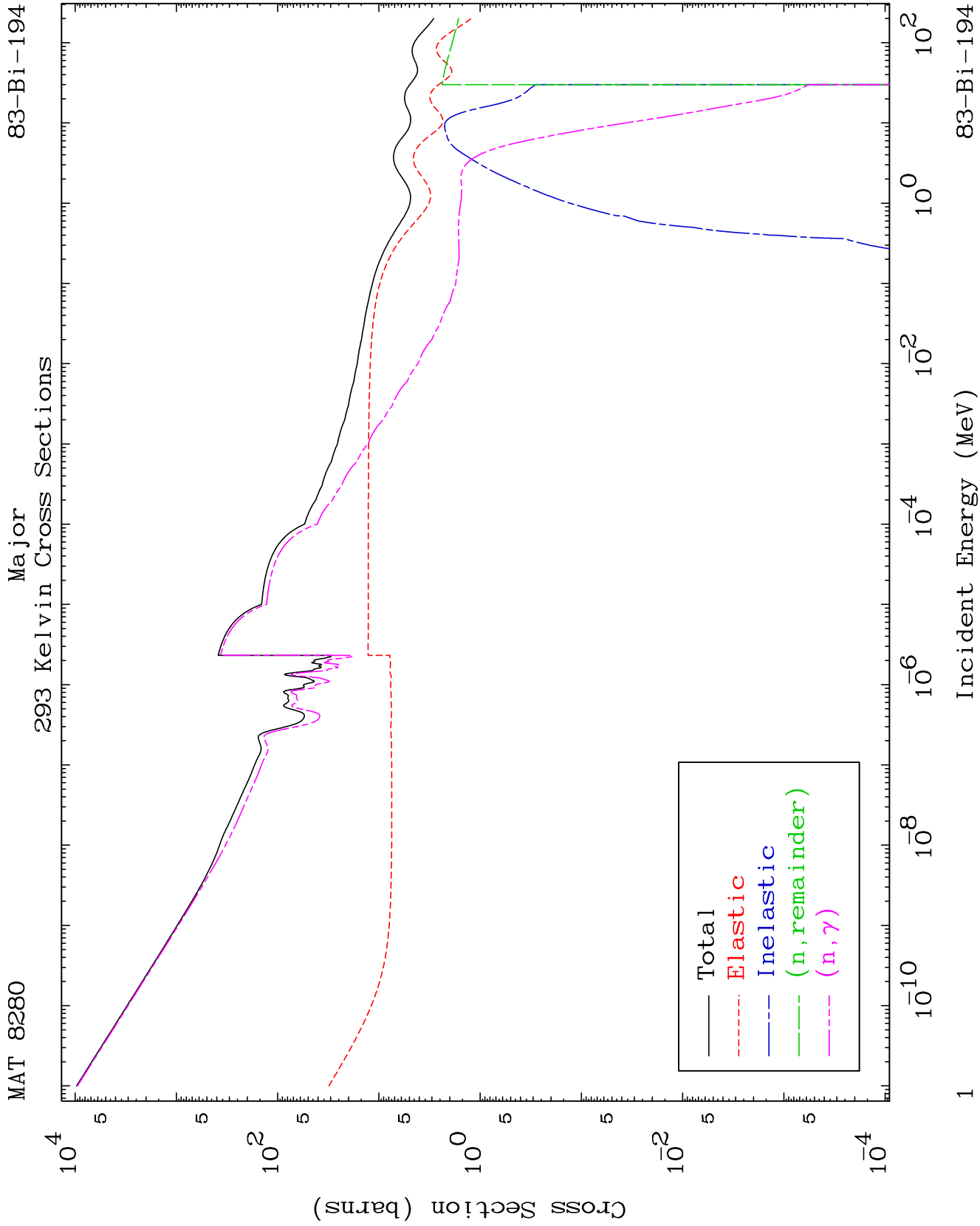
Dermott E. Cullen
(Present Contact Information)

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Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
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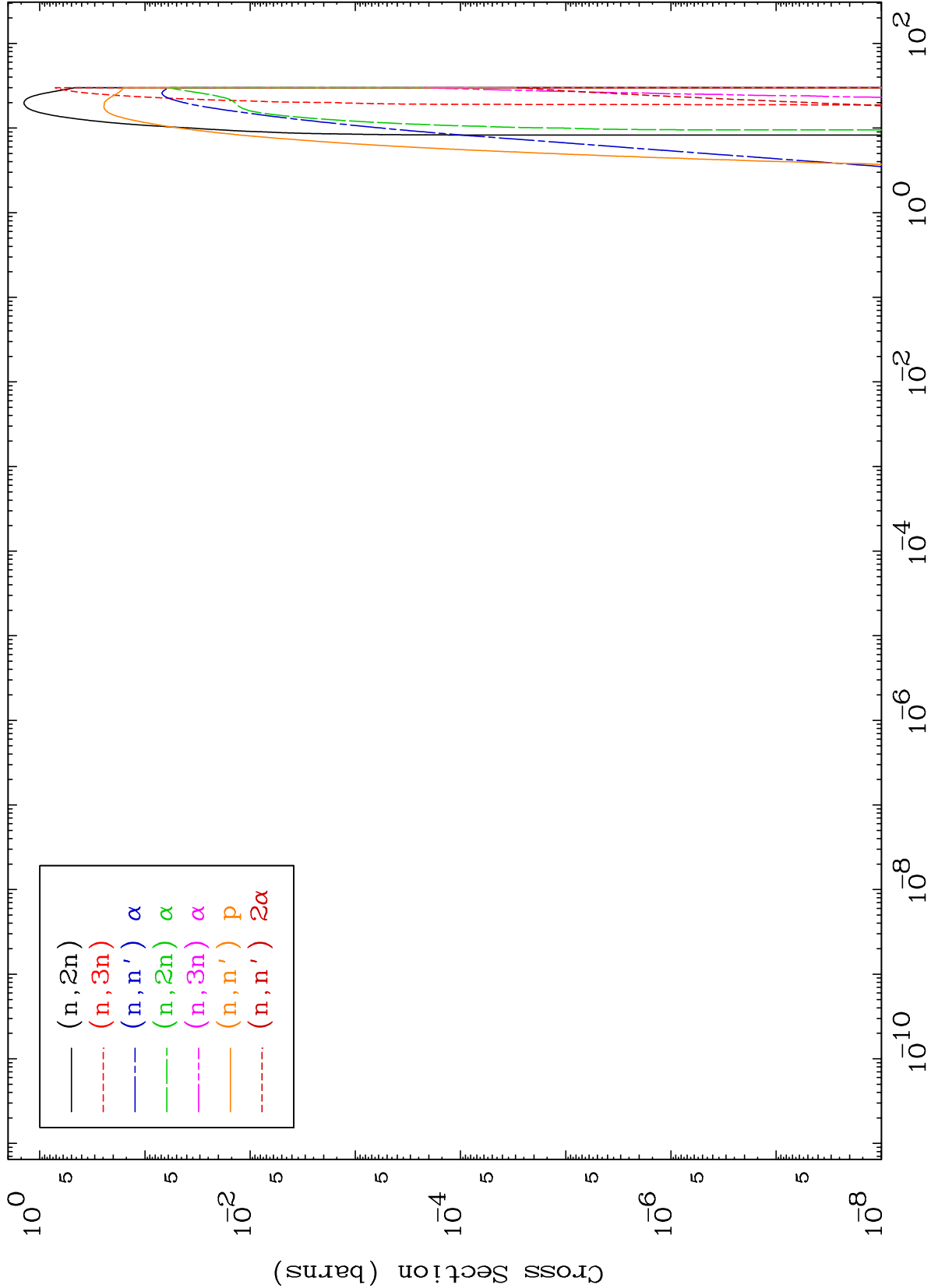
Press Mouse Button to Start

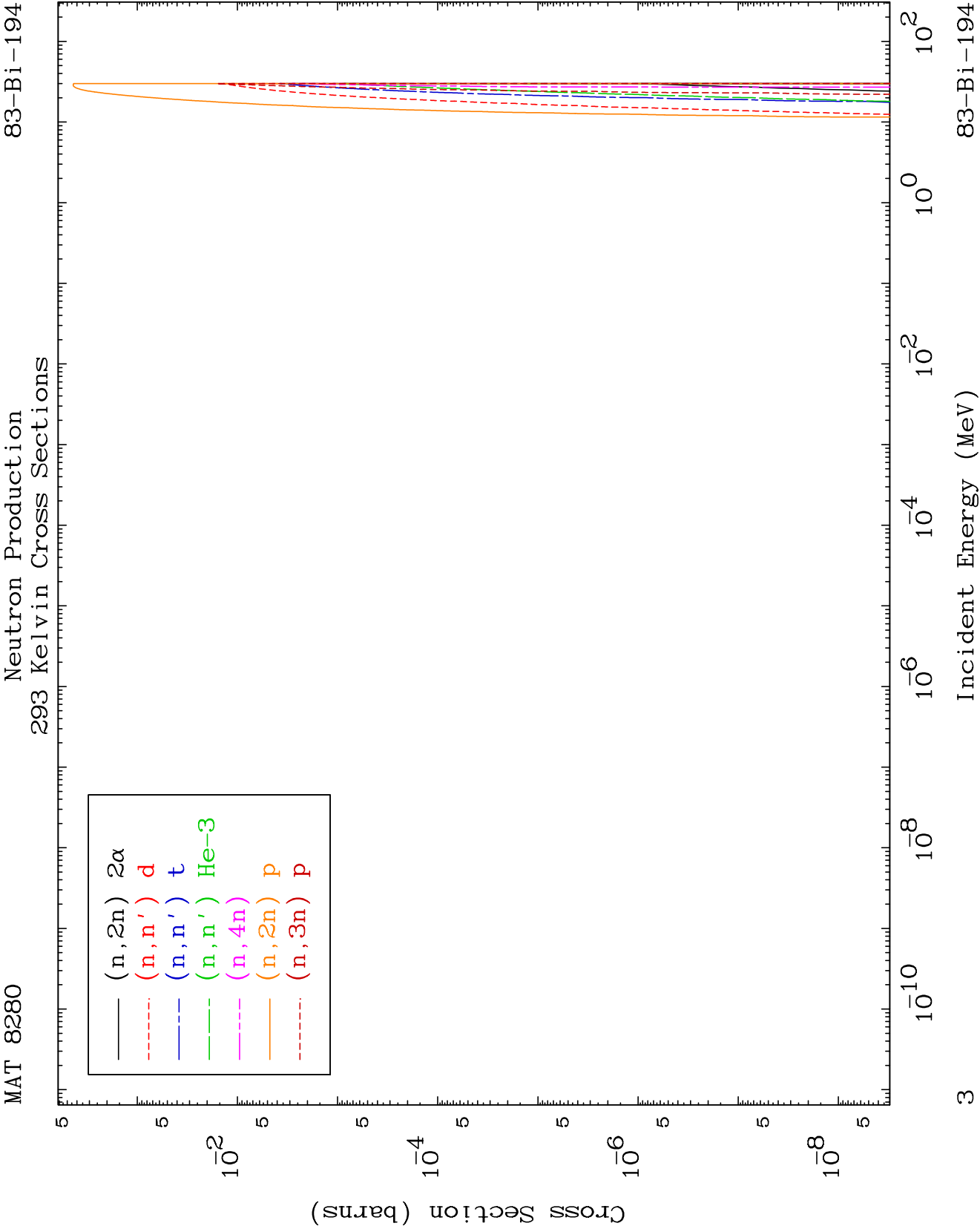


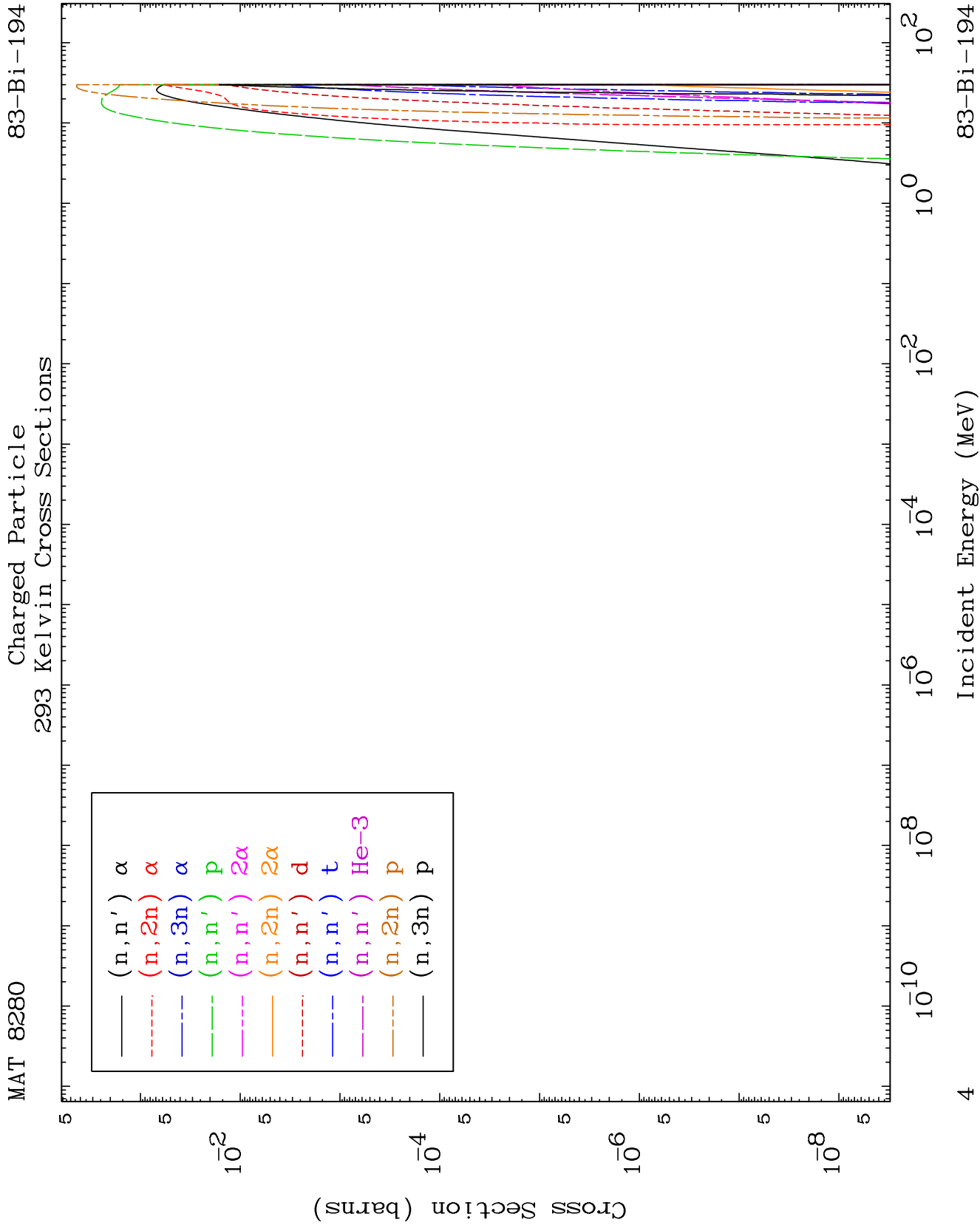
MAT 8280

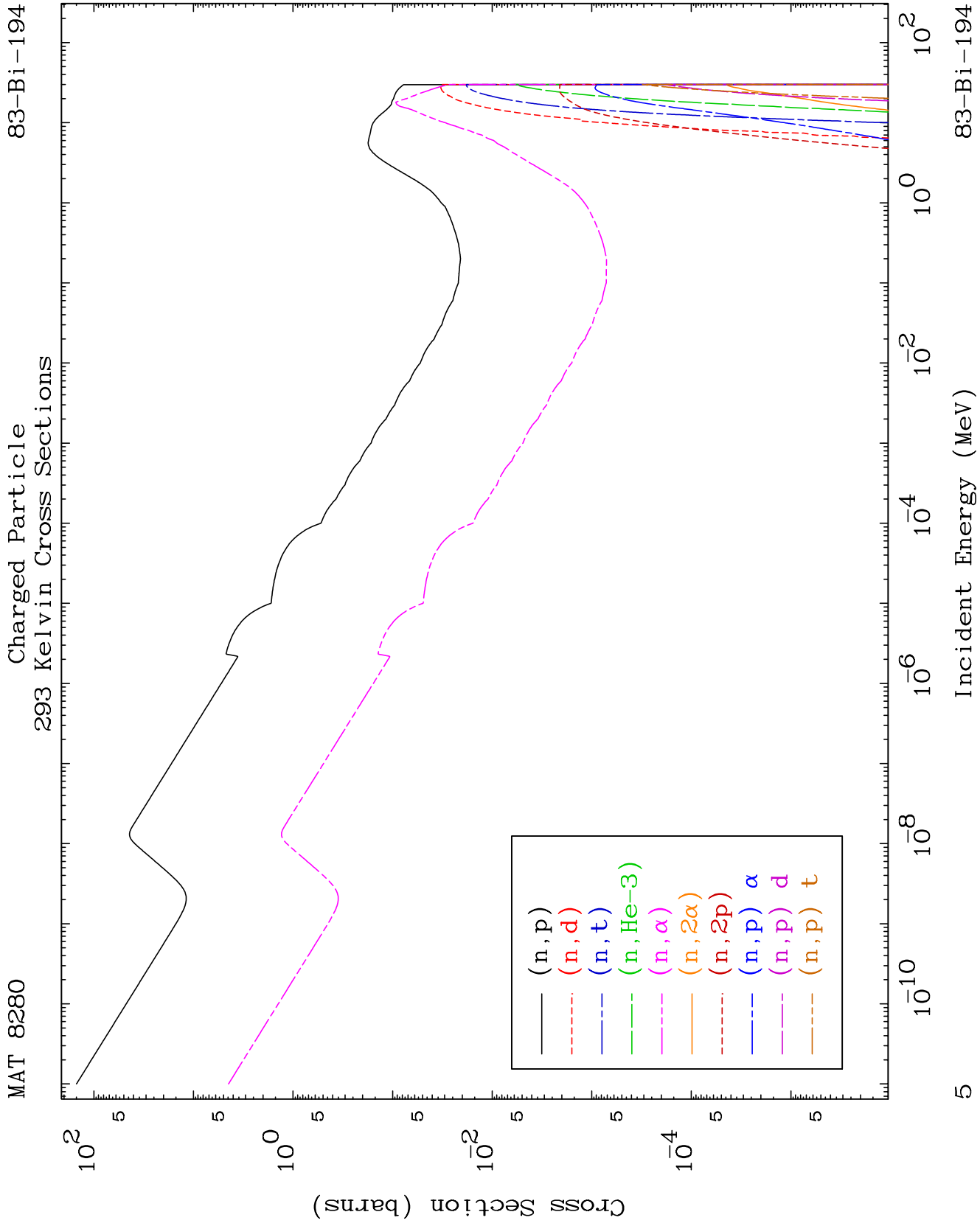
Neutron Production
293 Kelvin Cross Sections

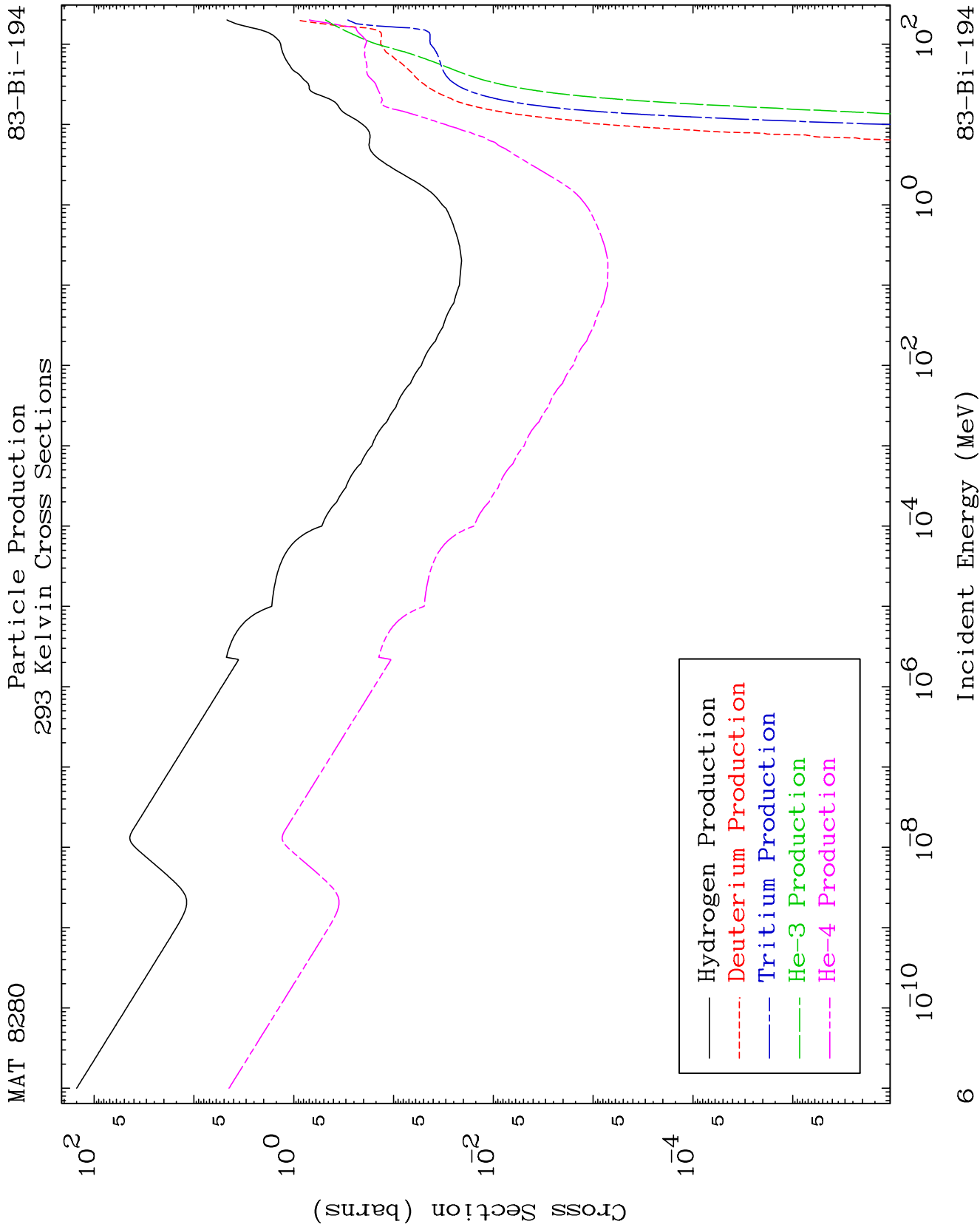
83-Bi-194

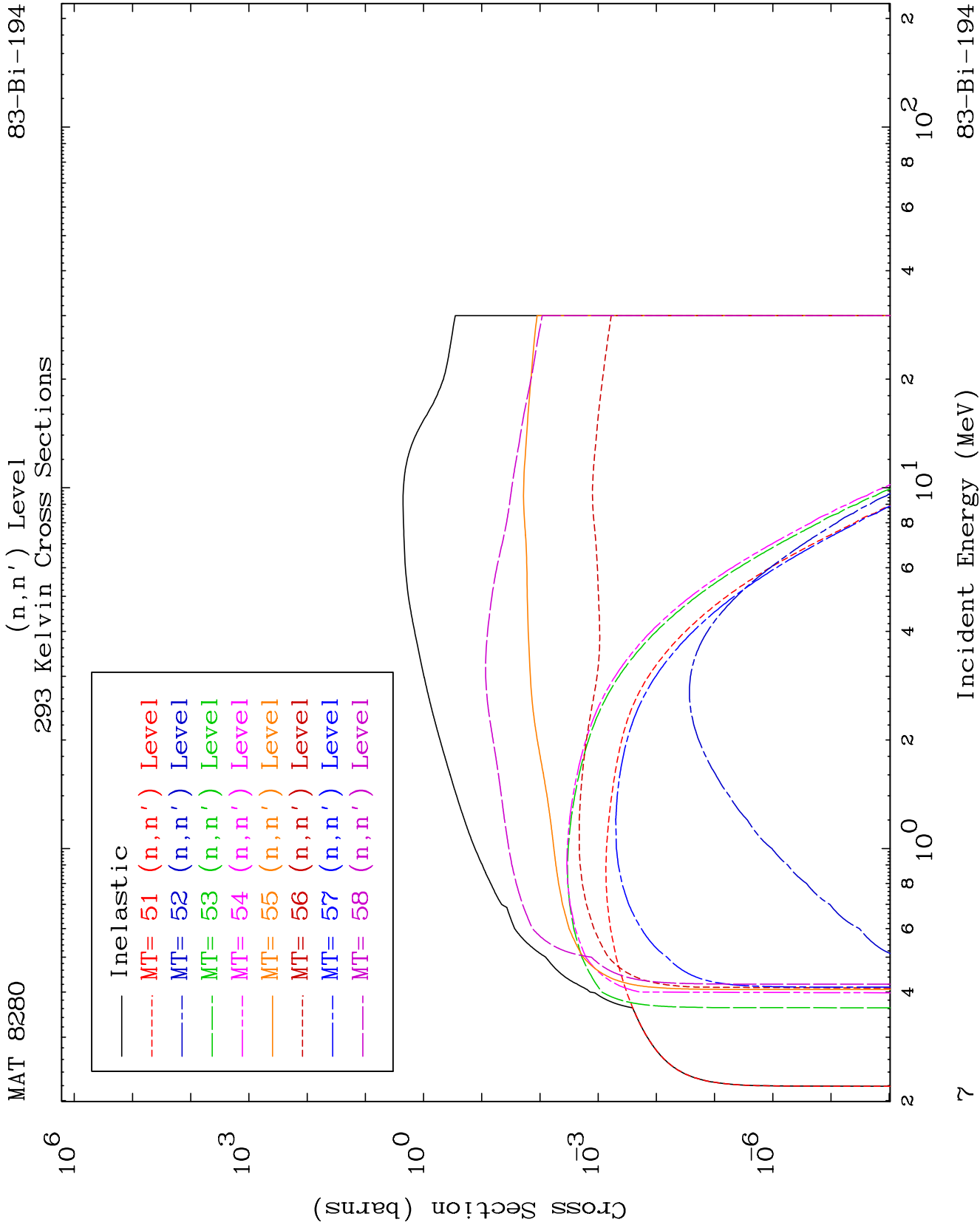










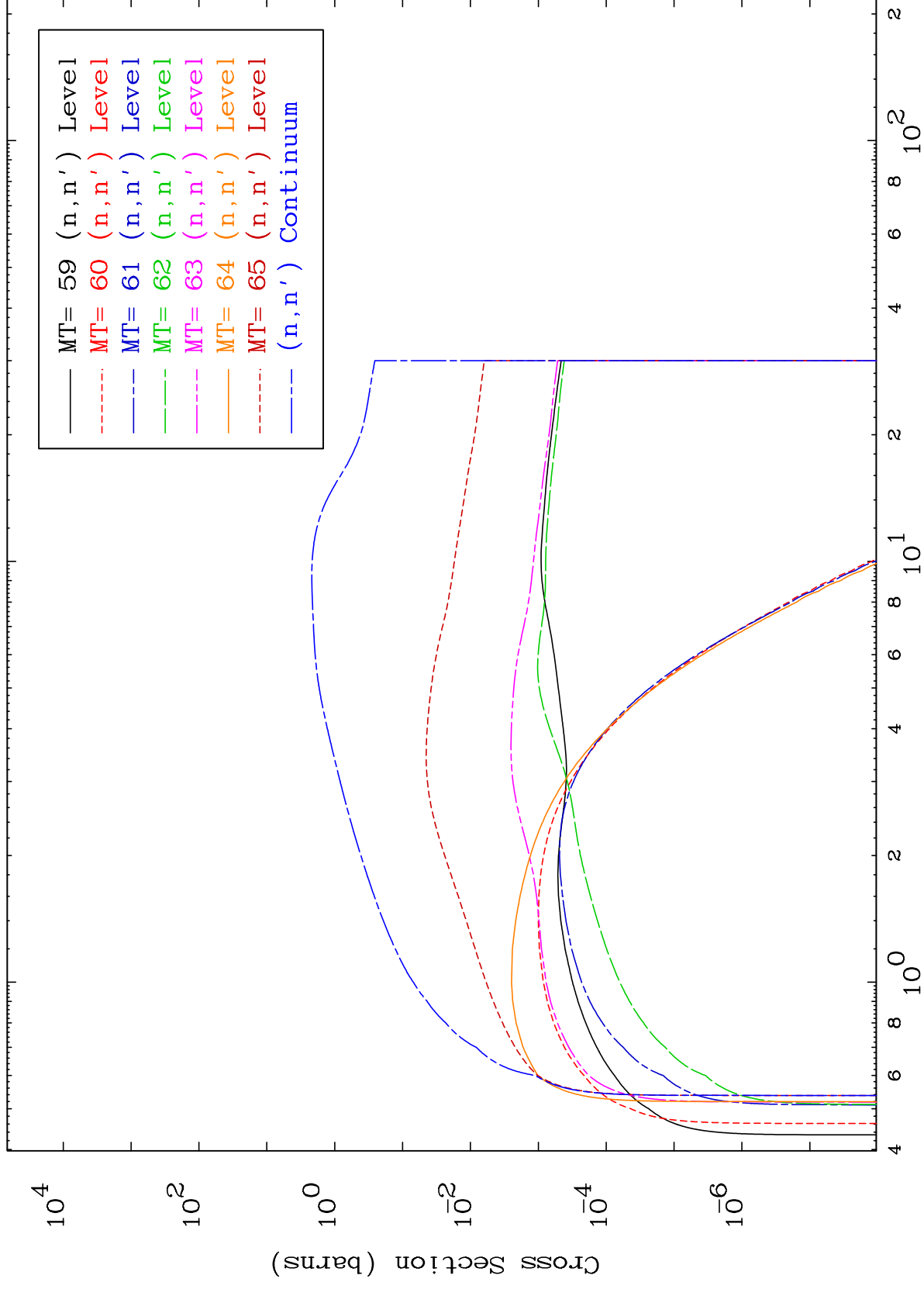


MAT 8280

(n,n') Level

293 Kelvin Cross Sections

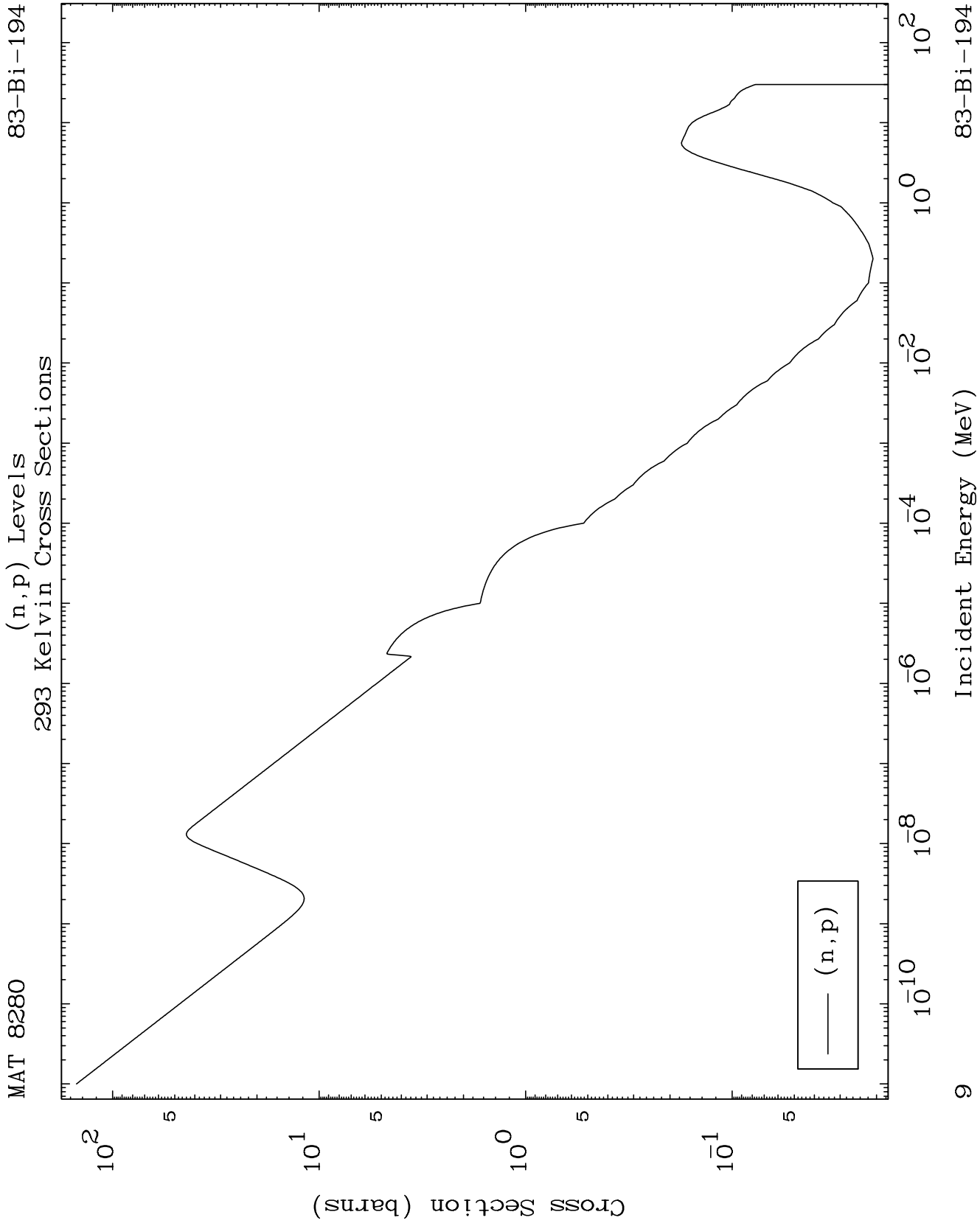
83-Bi-194



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Incident Energy (MeV)

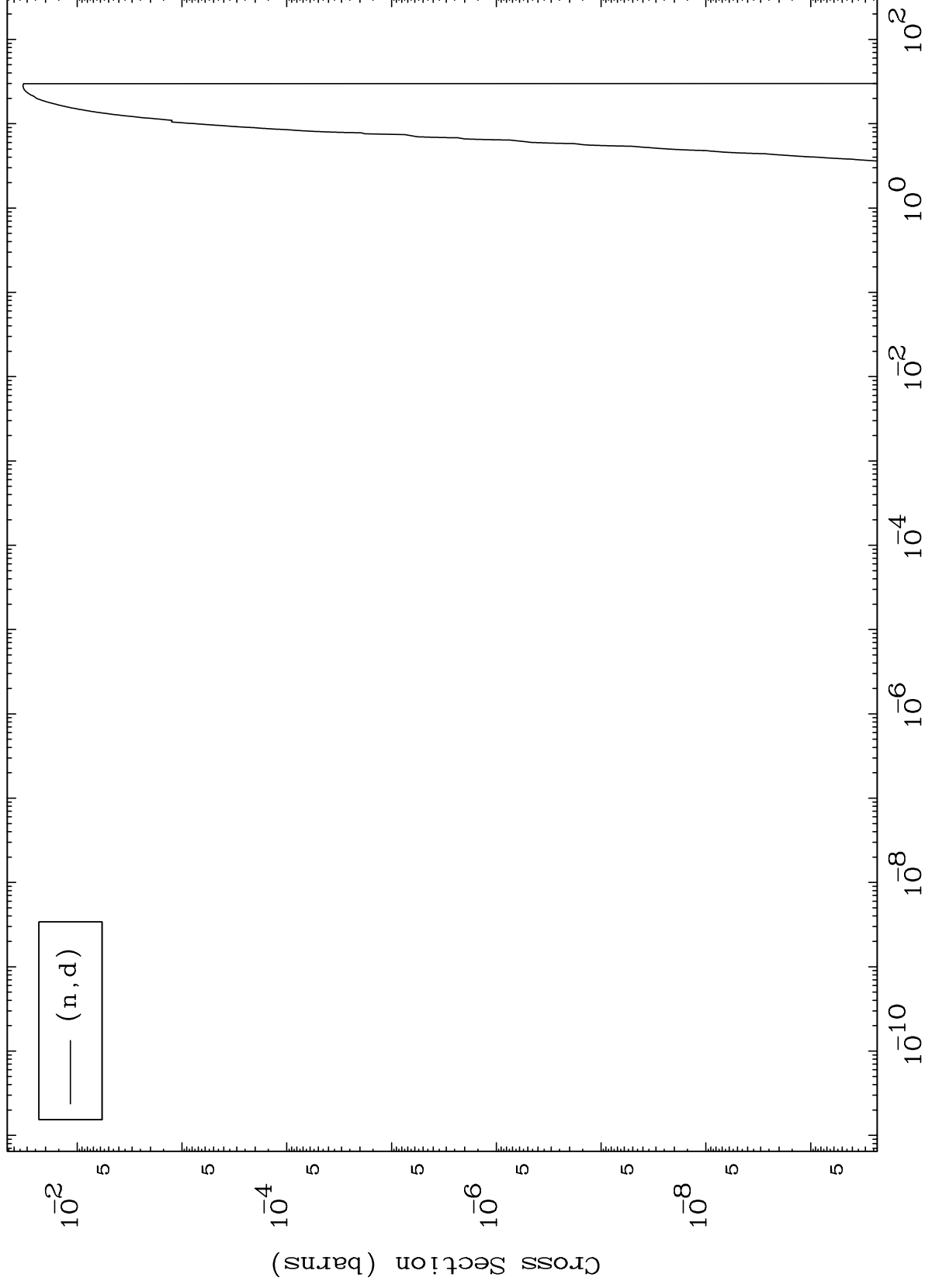
83-Bi-194



MAT 8280

(n,d) Levels
293 Kelvin Cross Sections

83-Bi-194



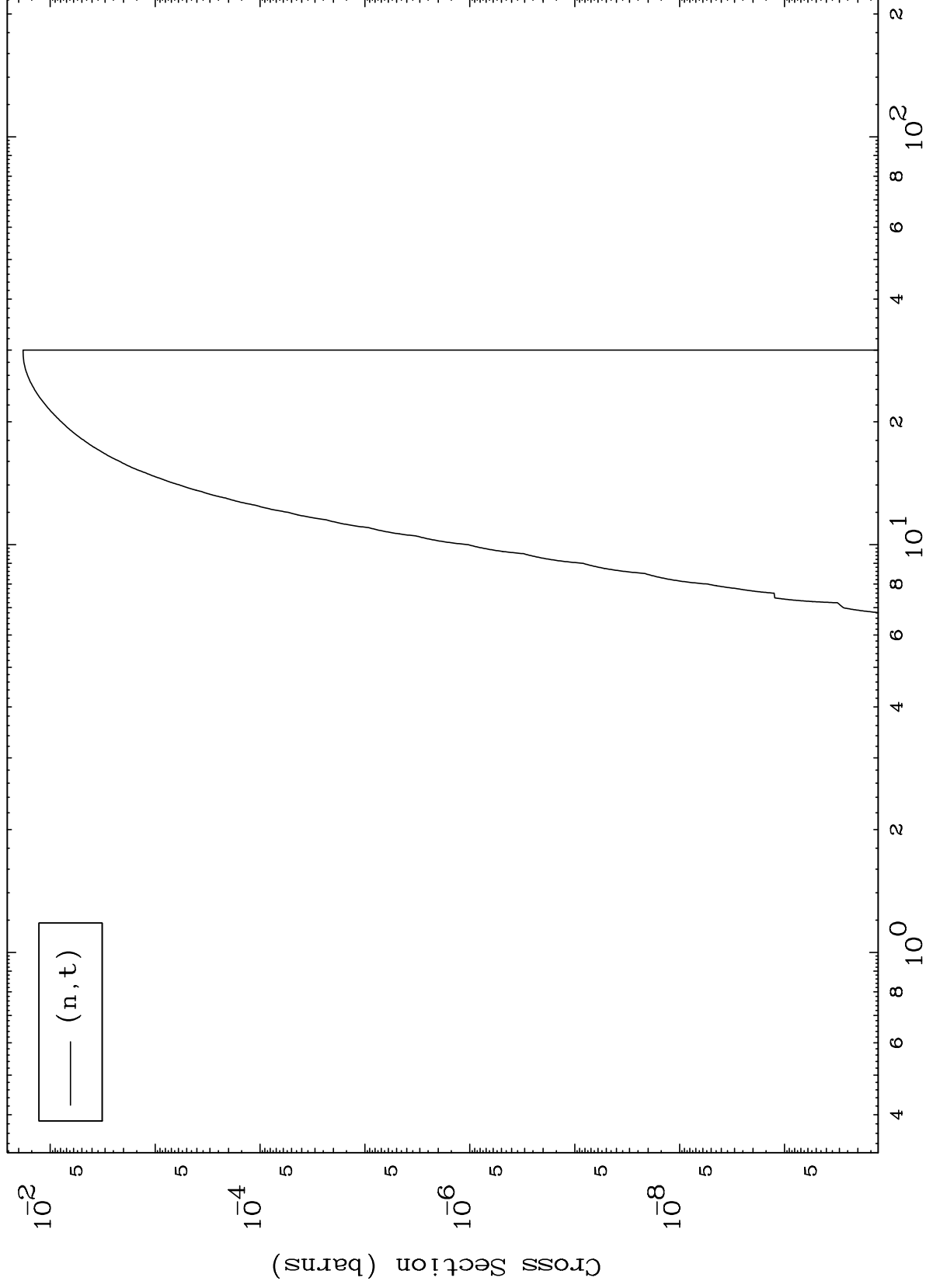
83-Bi-194

Incident Energy (MeV)

MAT 8280

83-Bi-194

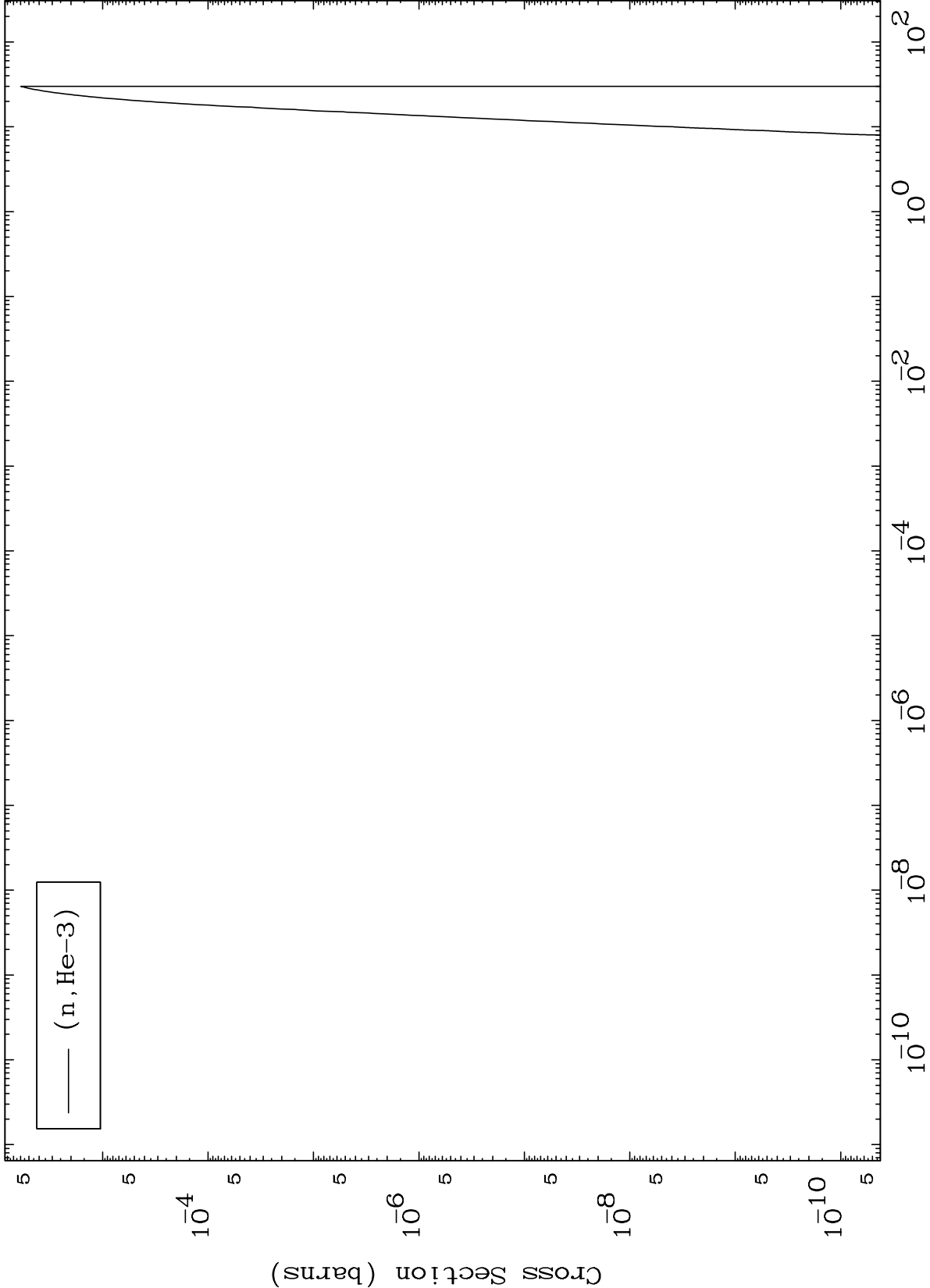
(n,t) Levels
293 Kelvin Cross Sections



MAT 8280

(n,He3) Levels
293 Kelvin Cross Sections

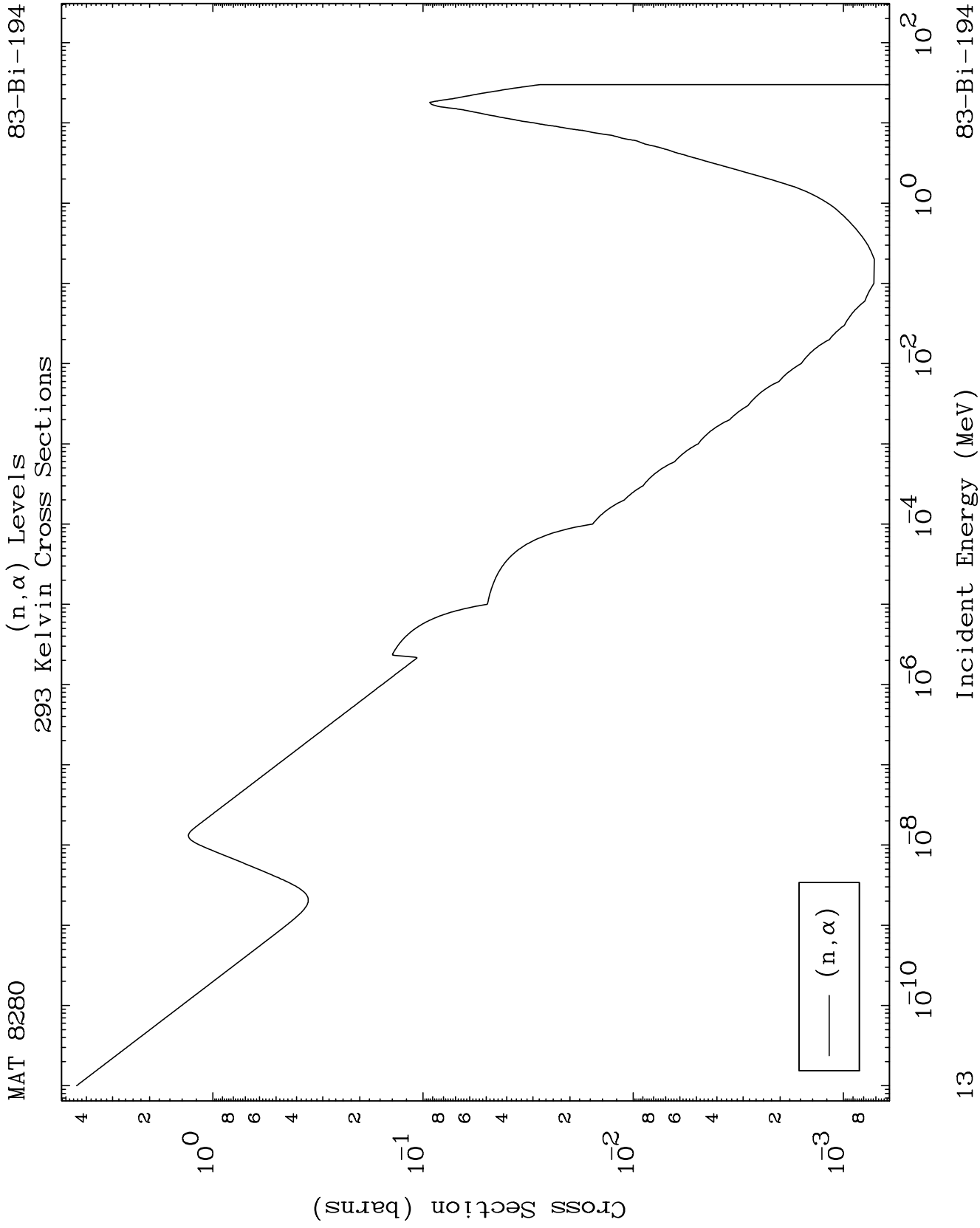
83-Bi-194

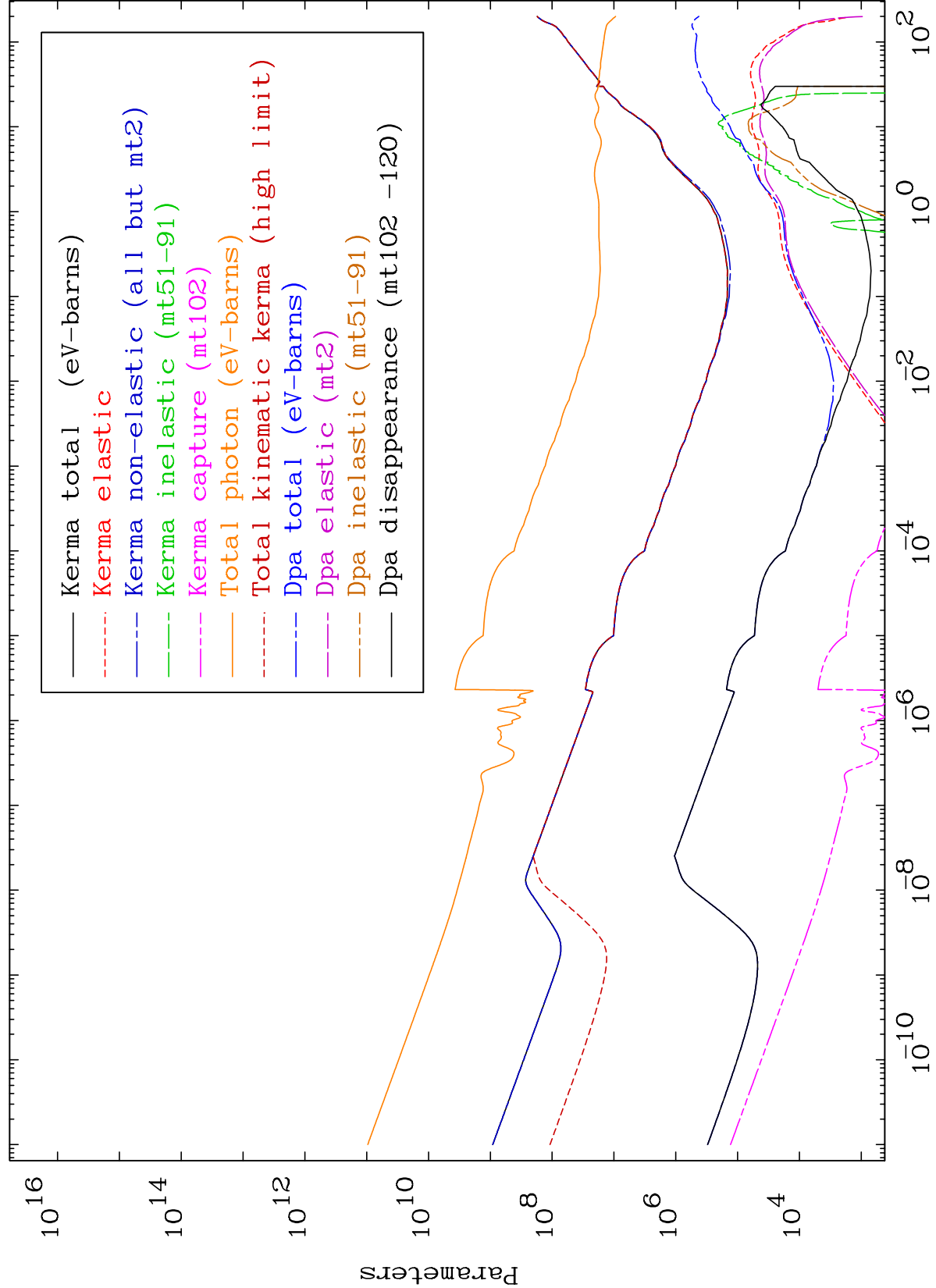


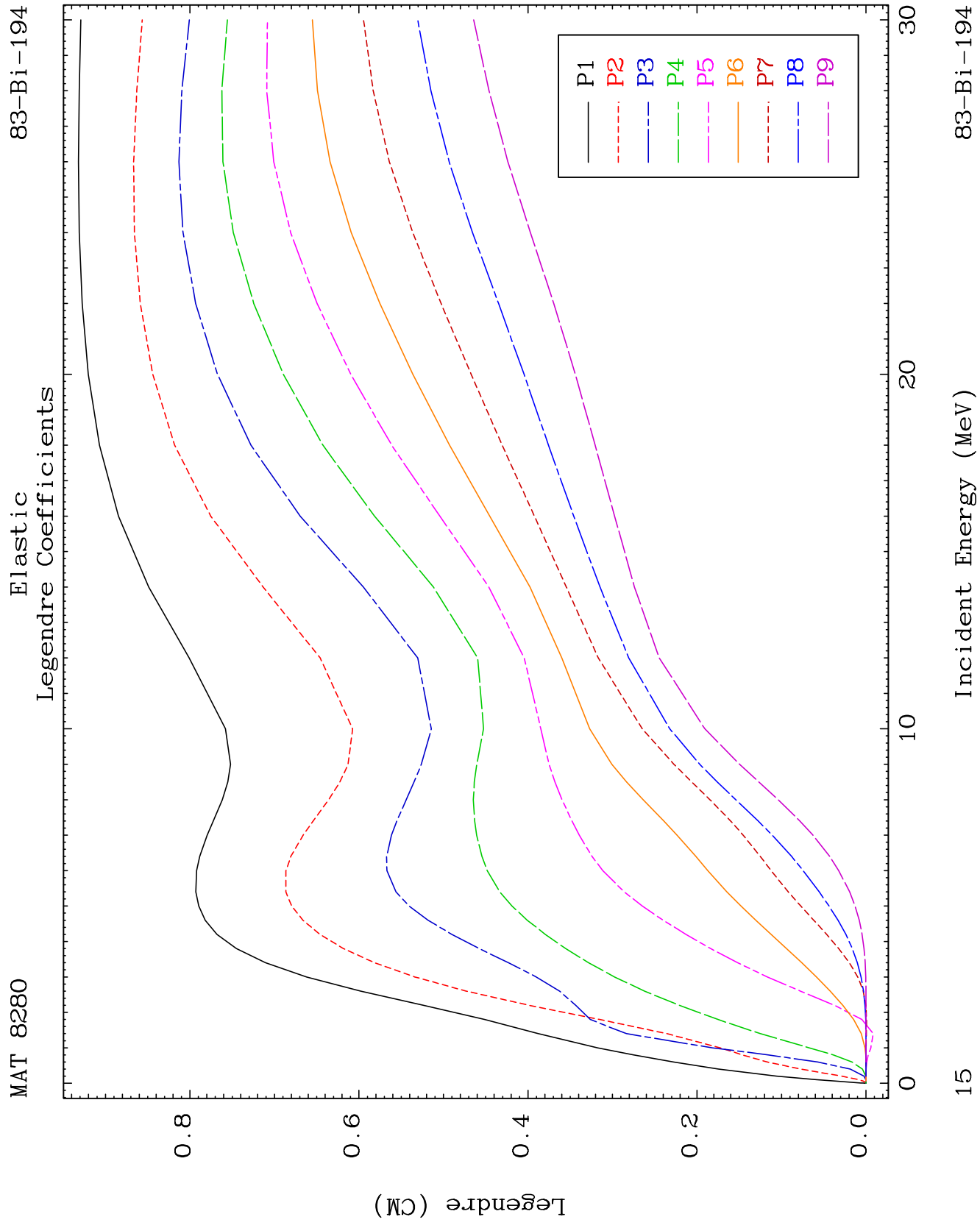
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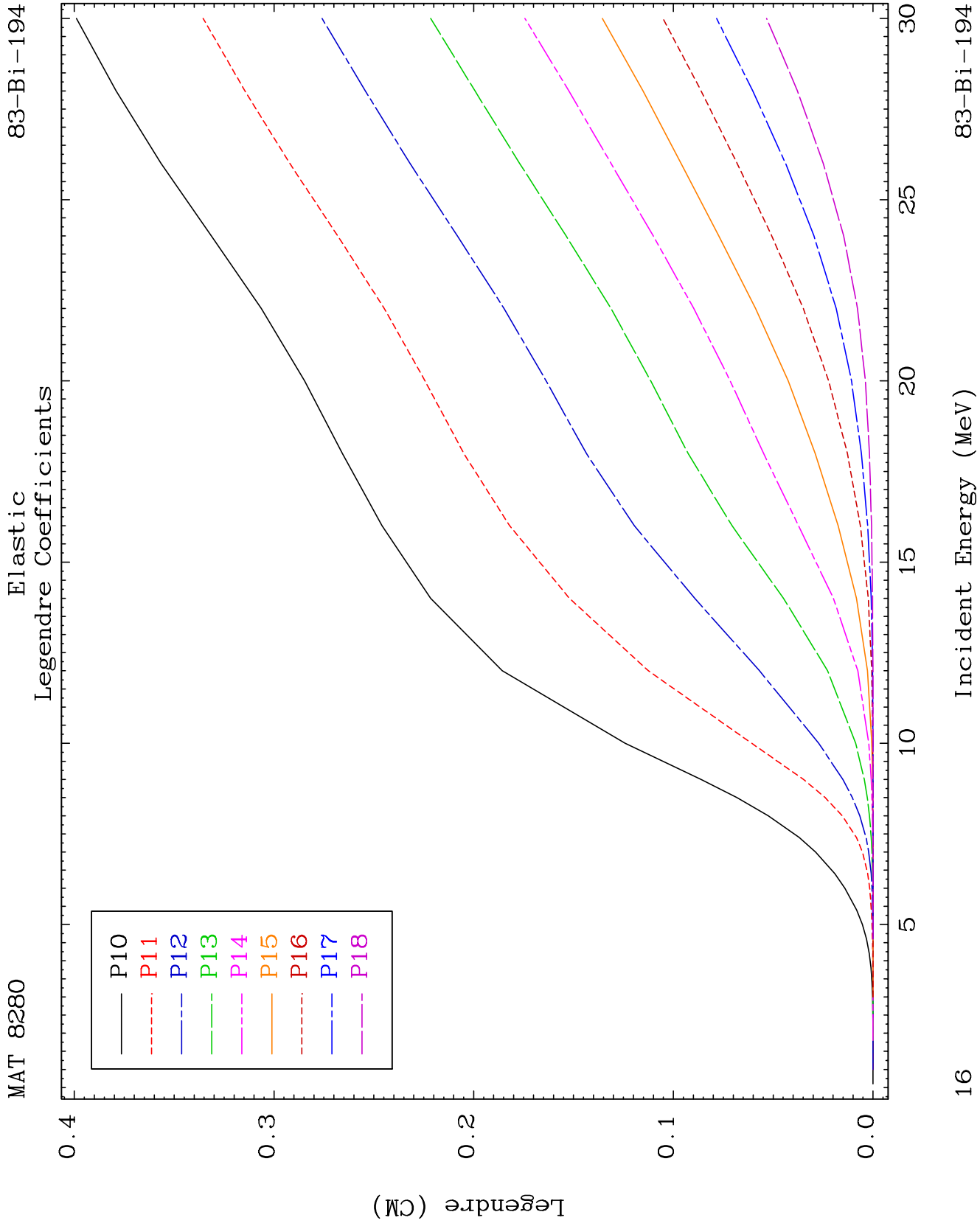
Incident Energy (MeV)

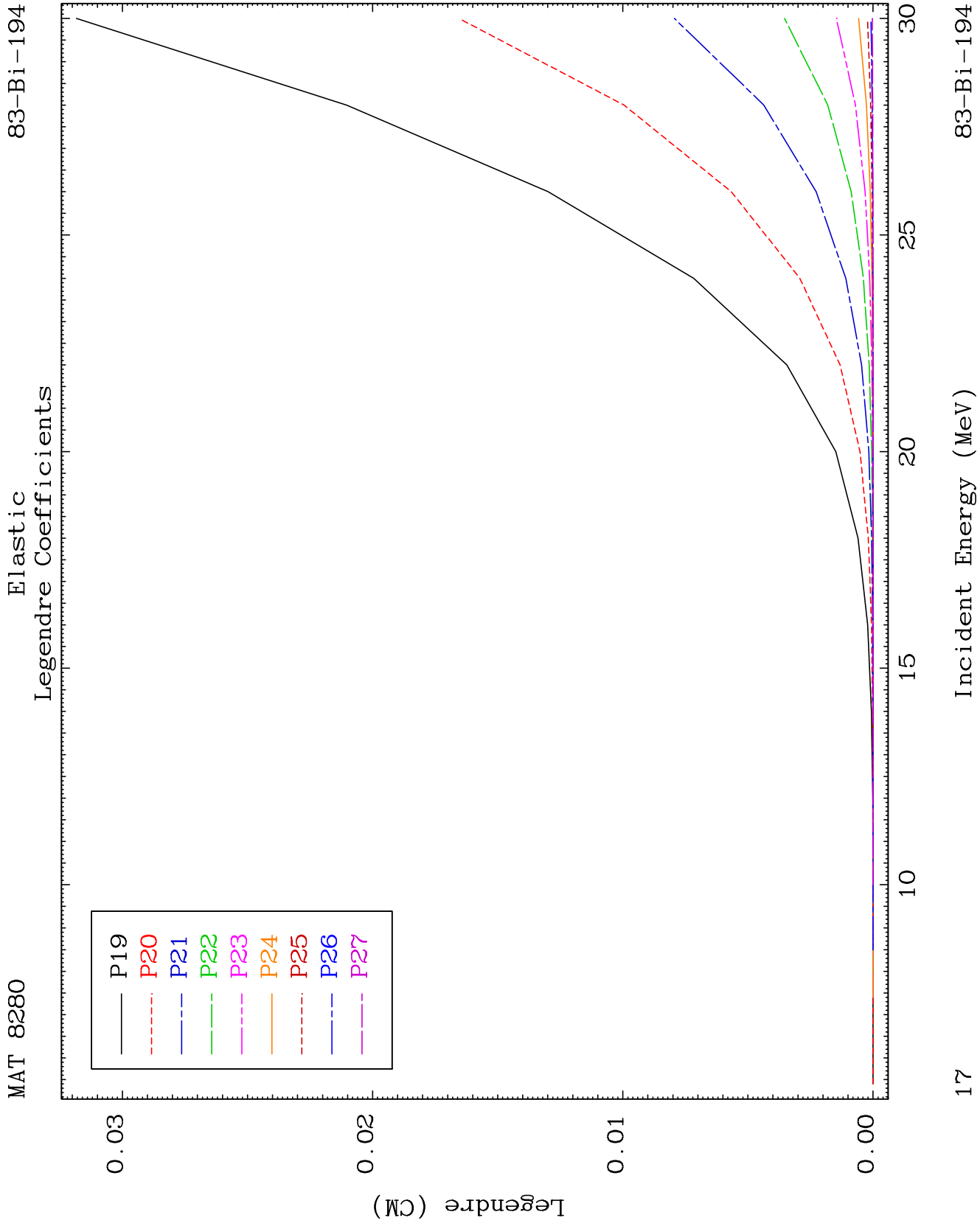
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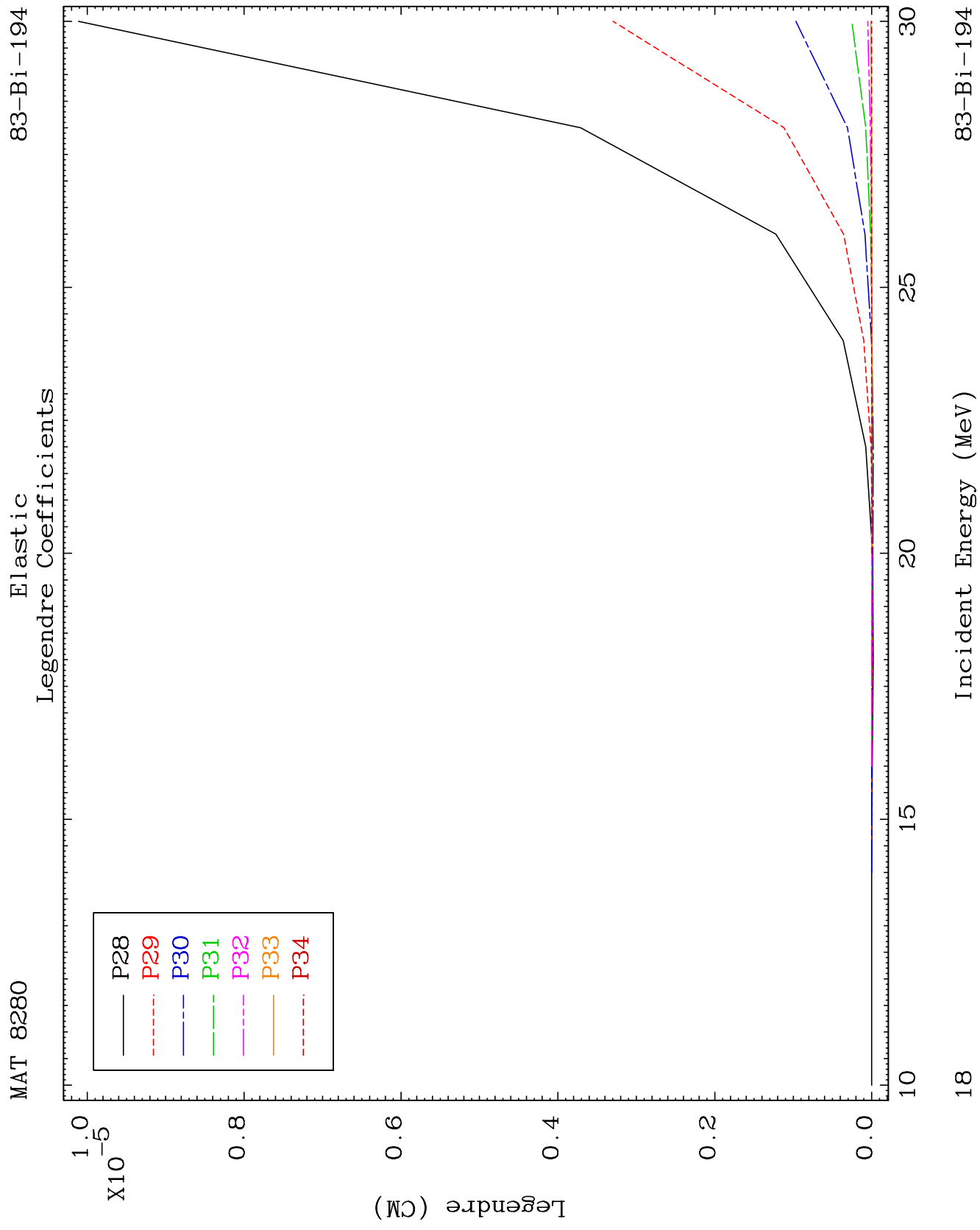


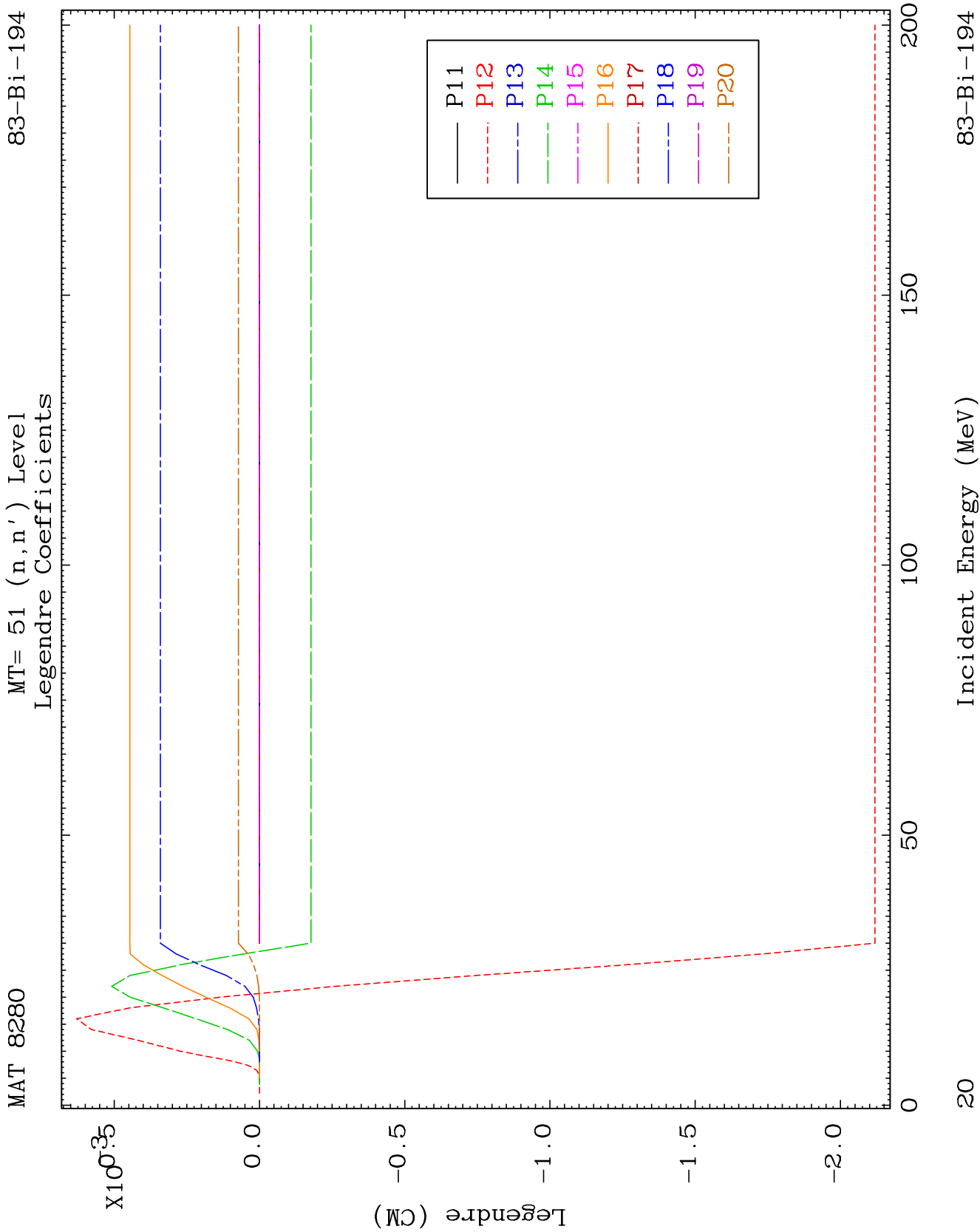


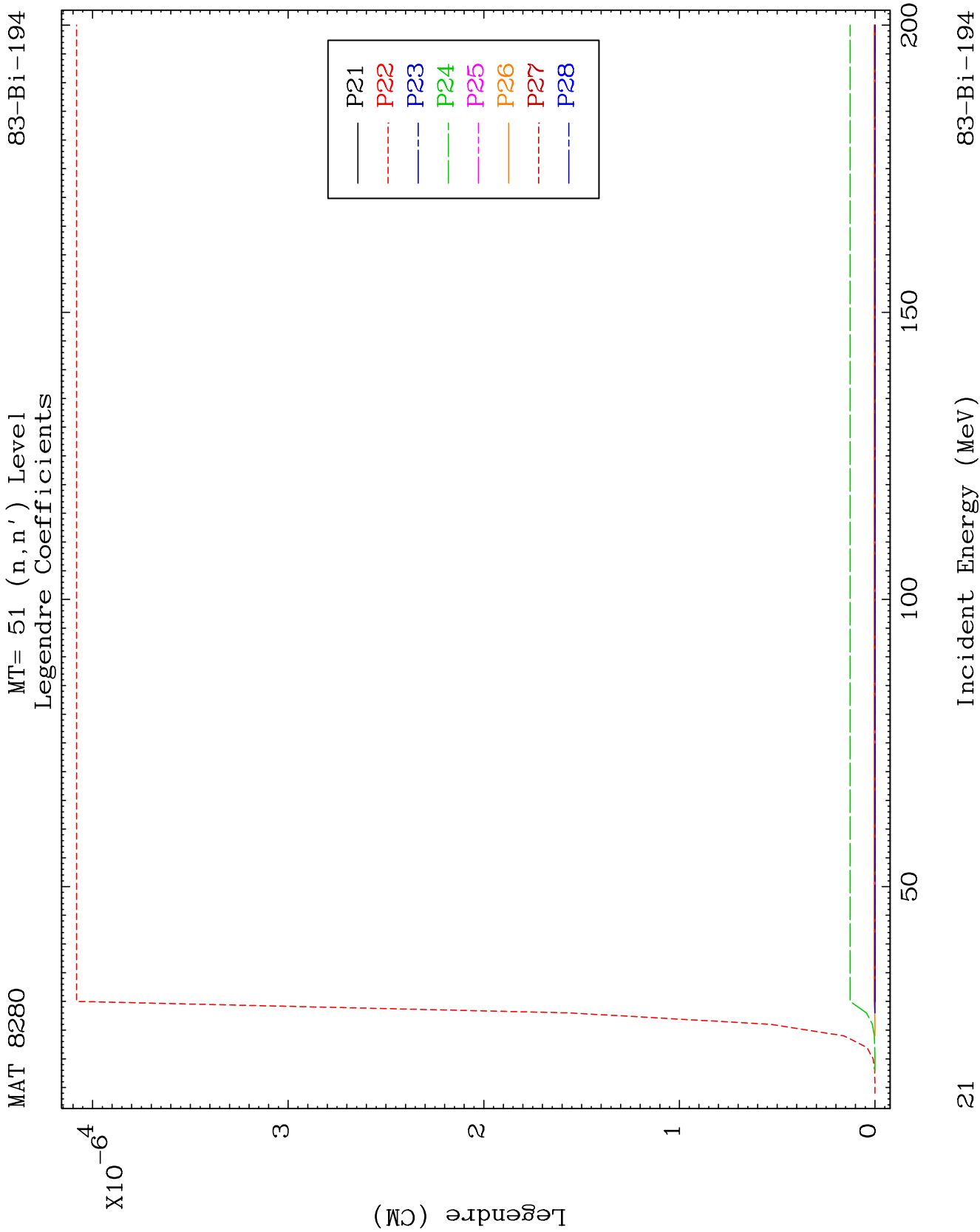


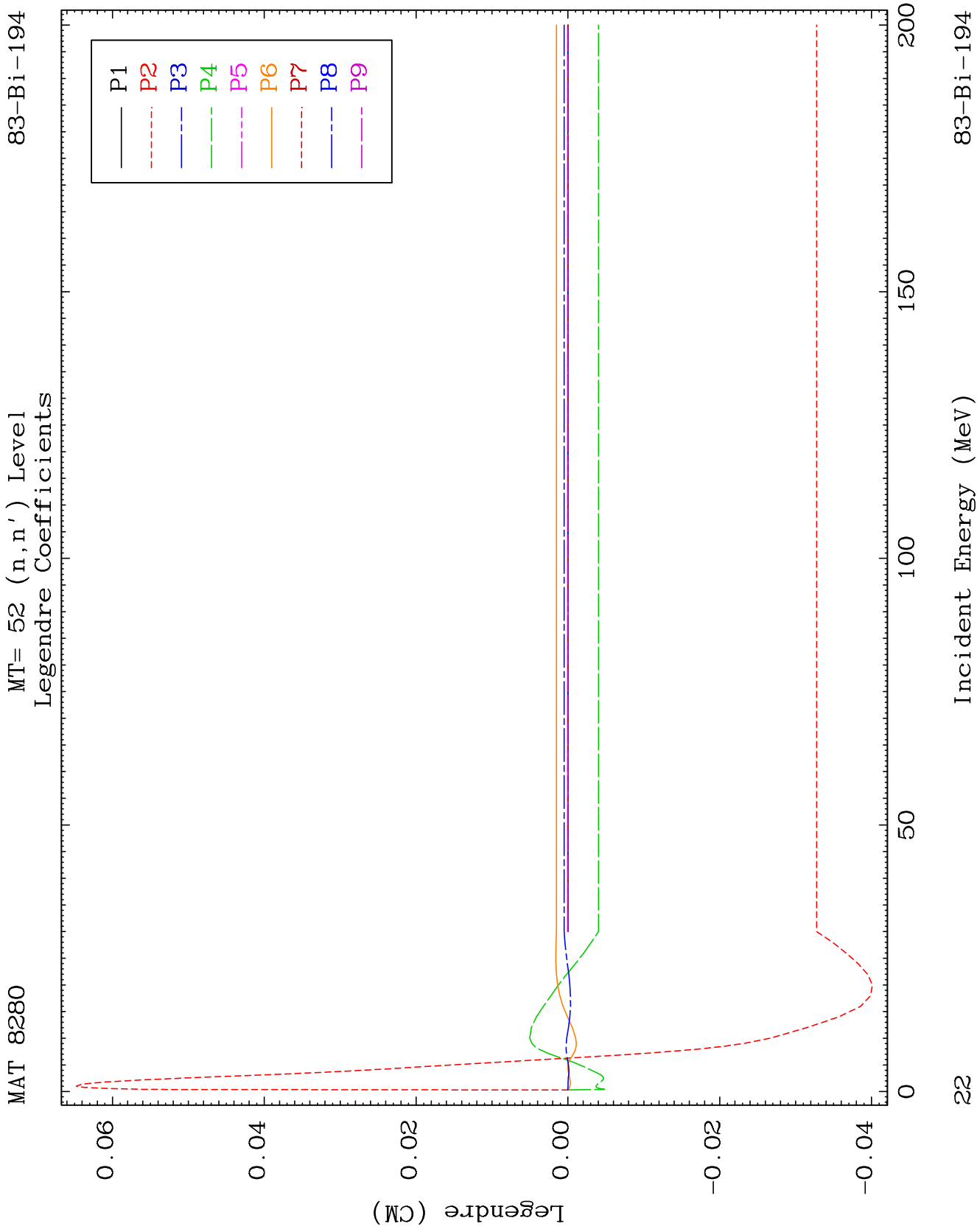








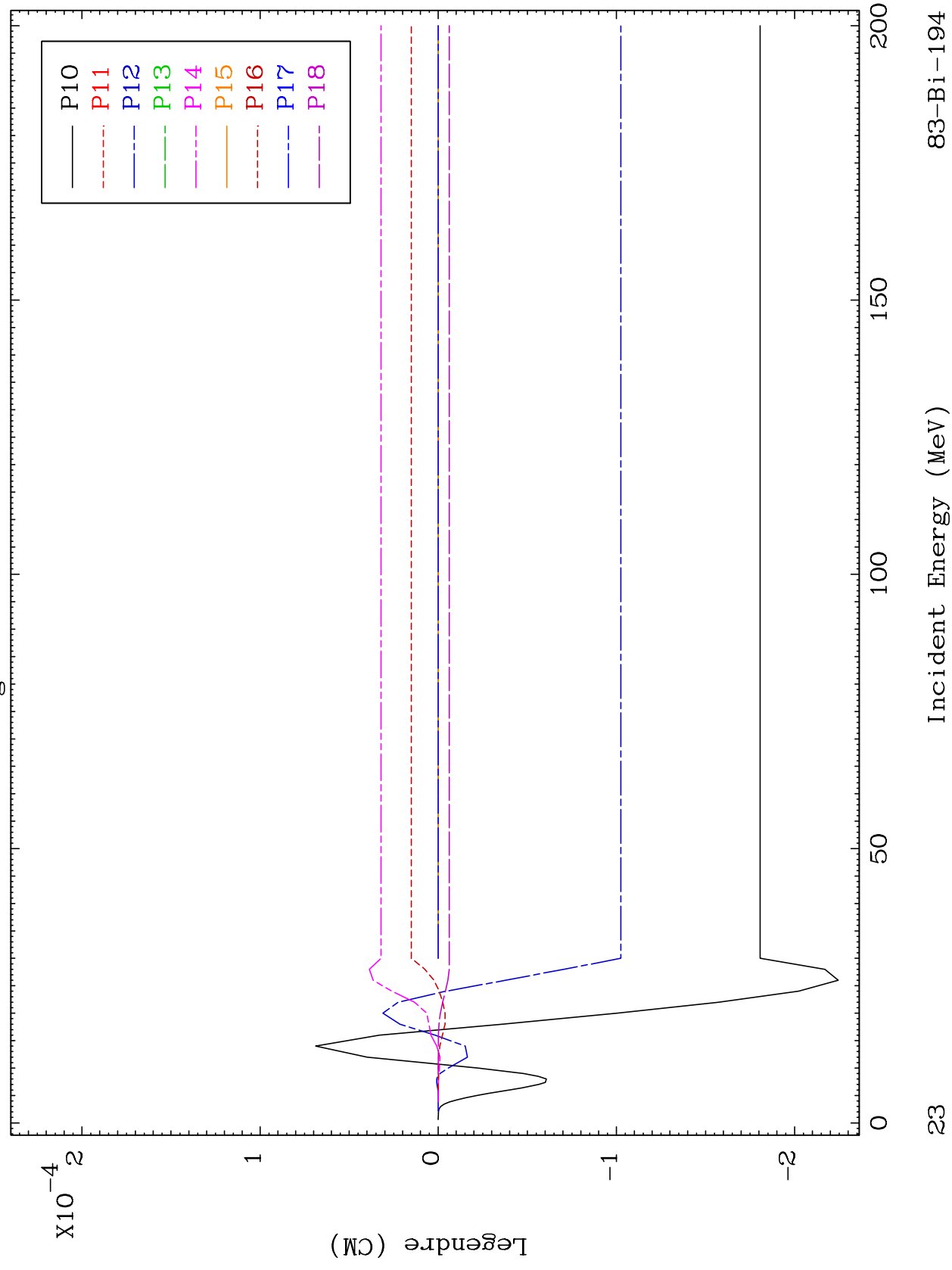


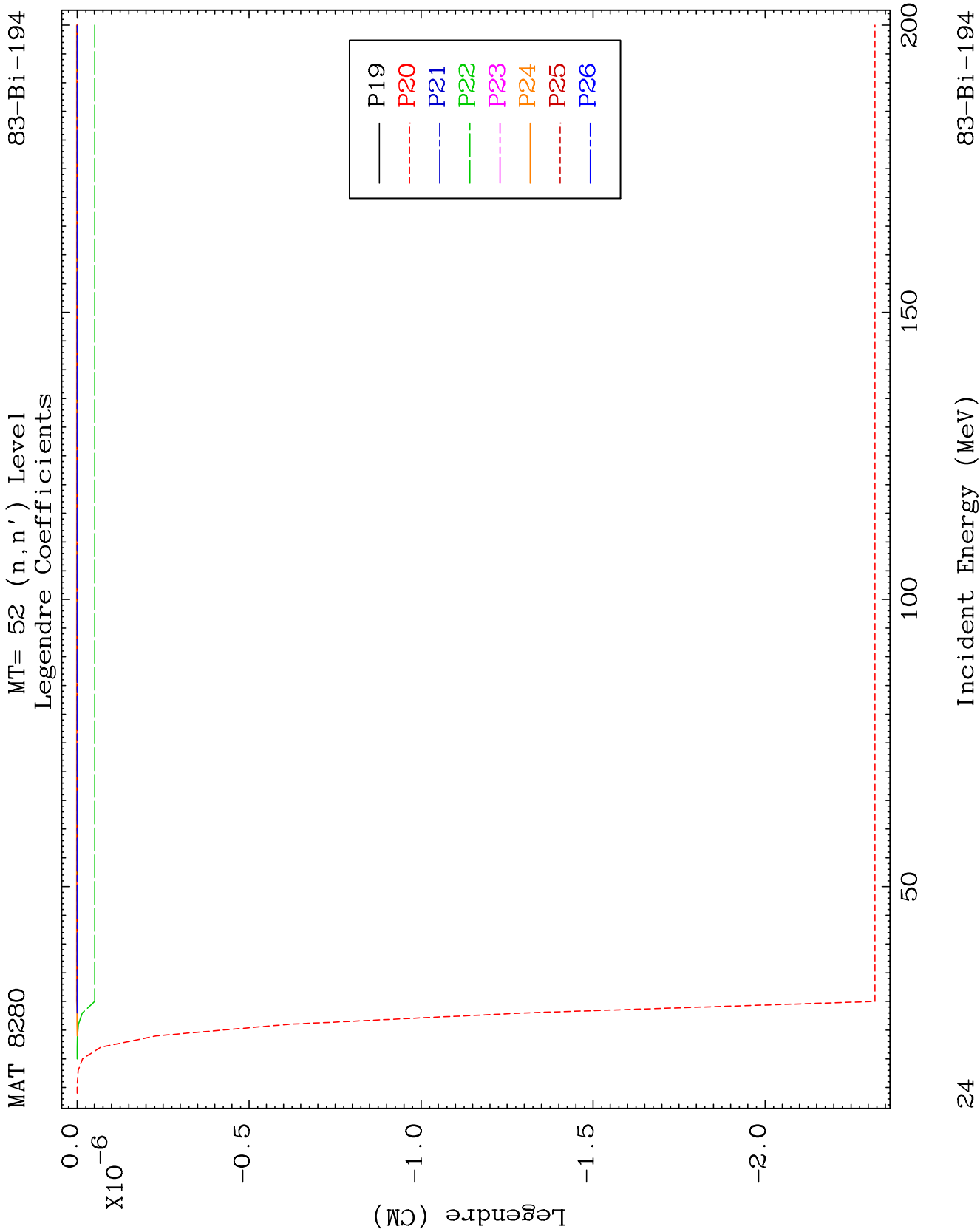


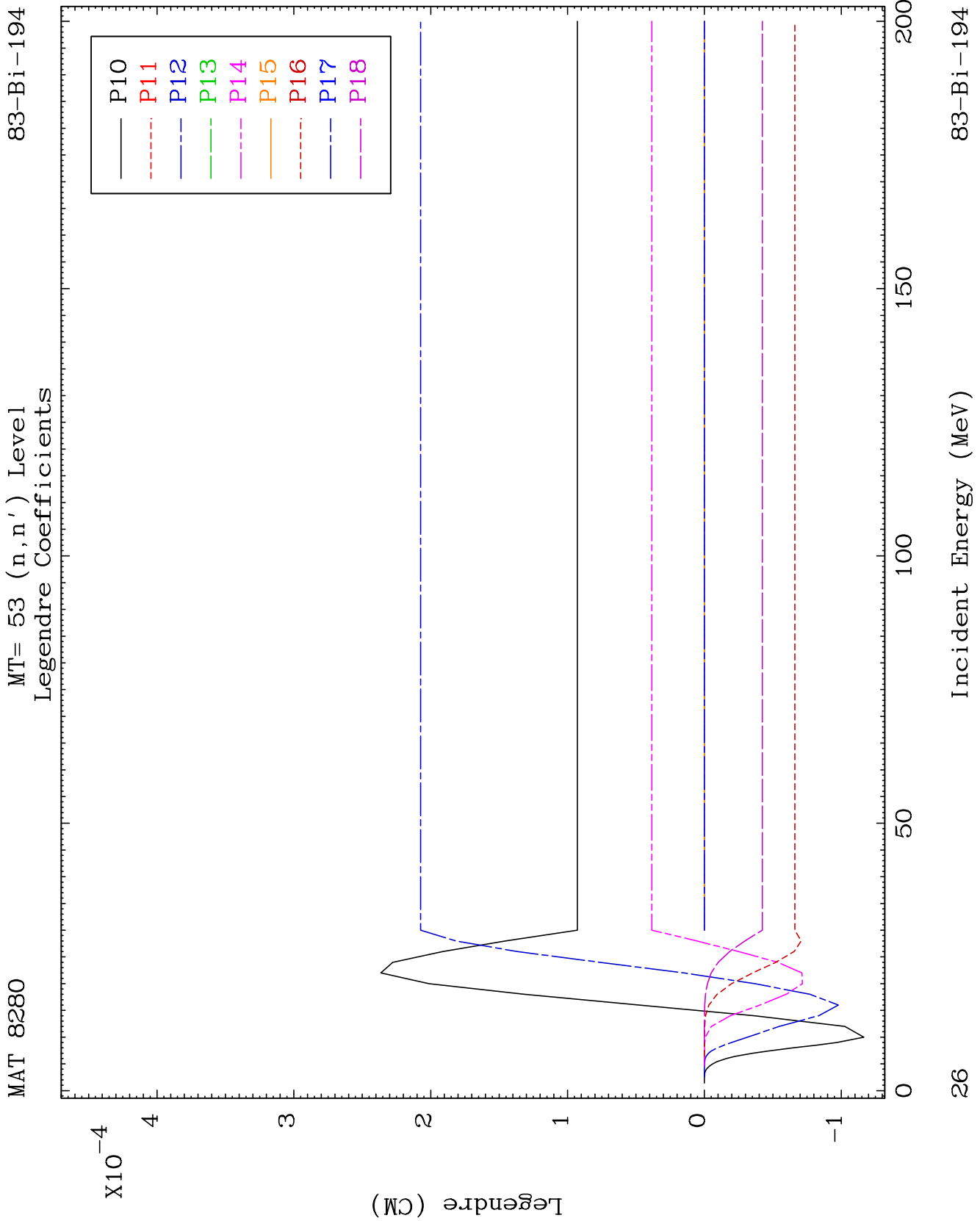
MAT 8280

MT= 52 (n, n') Level
Legendre Coefficients

83-Bi-194



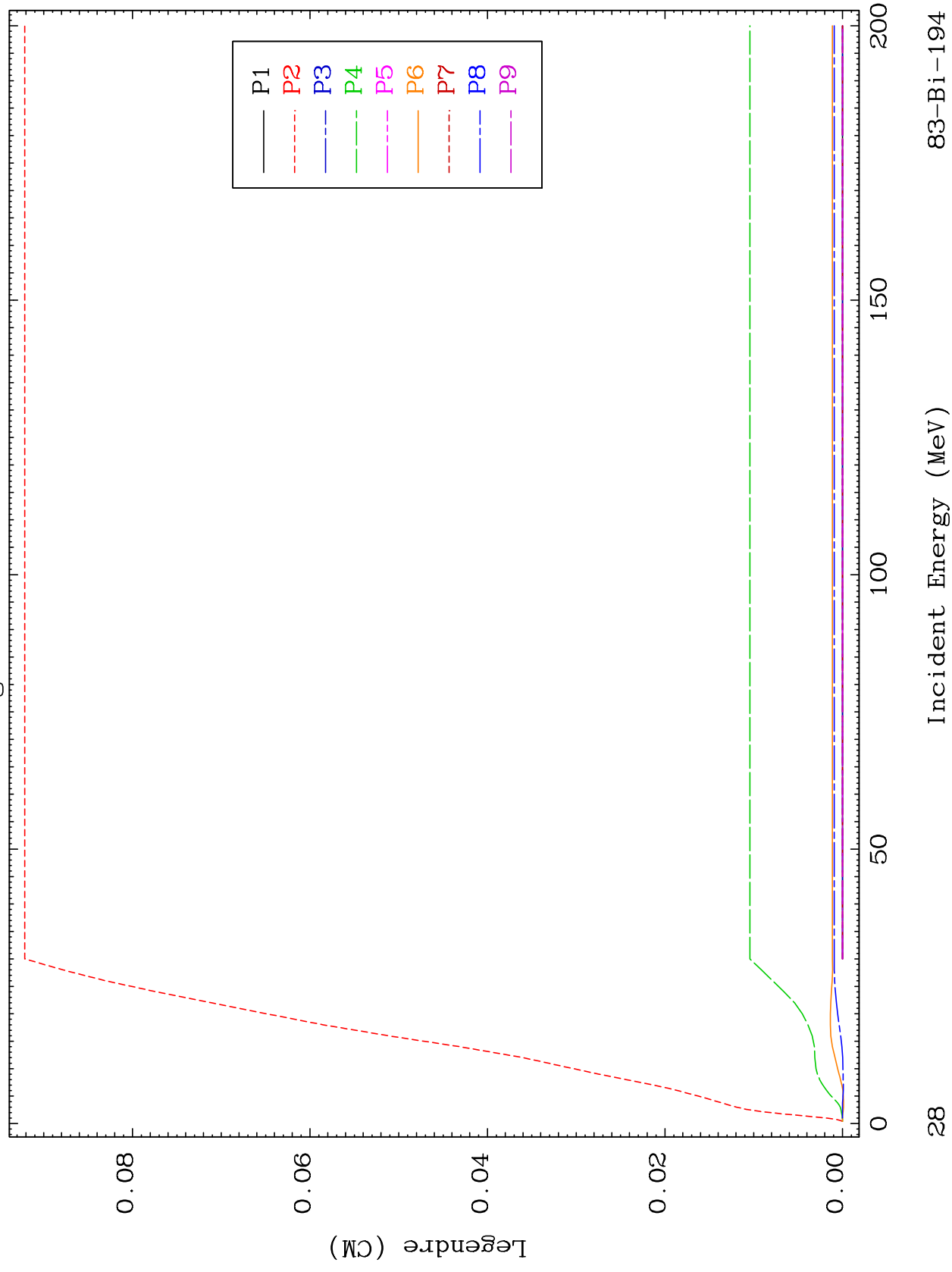


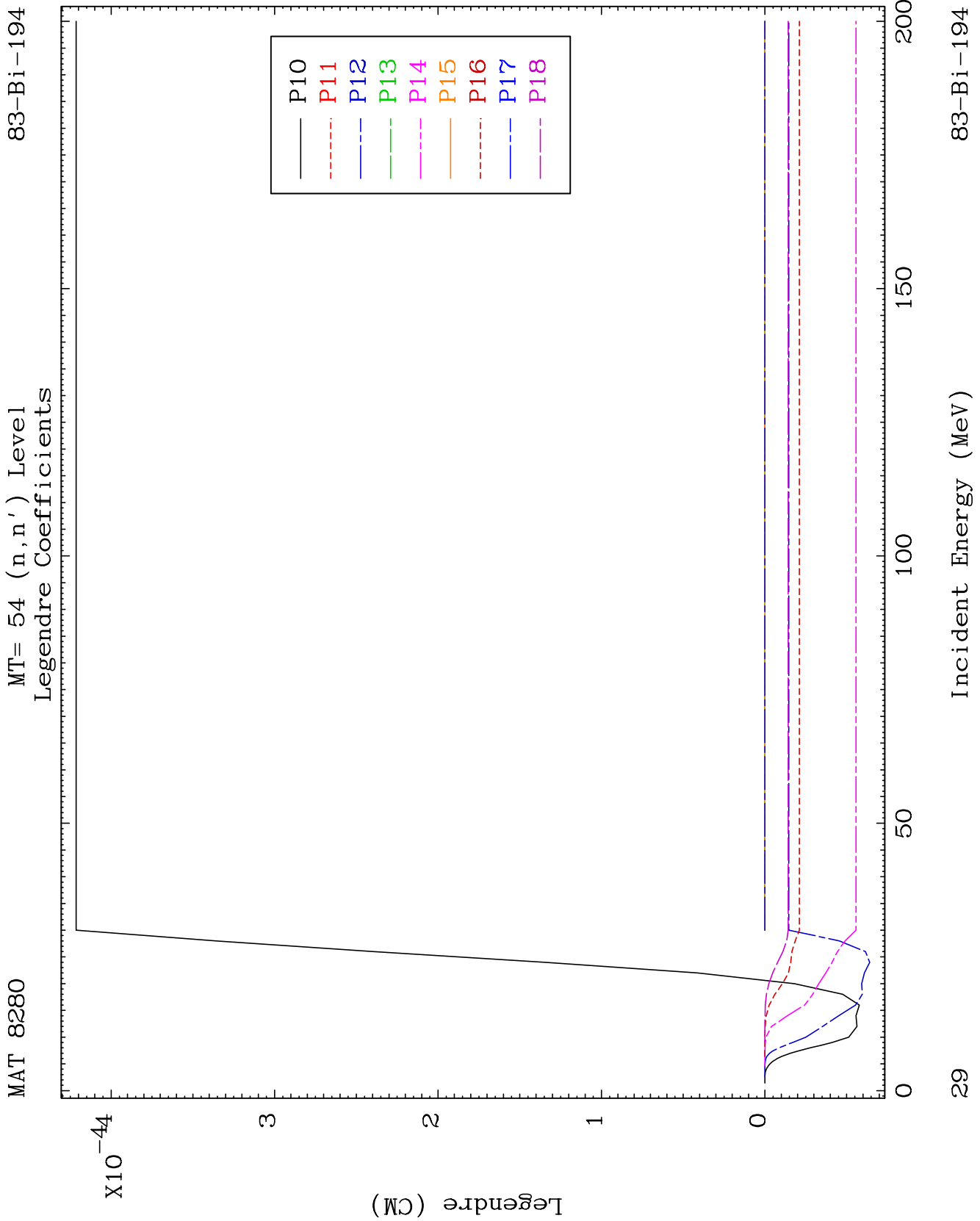


MAT 8280

MT= 54 (n, n') Level
Legendre Coefficients

83-Bi-194

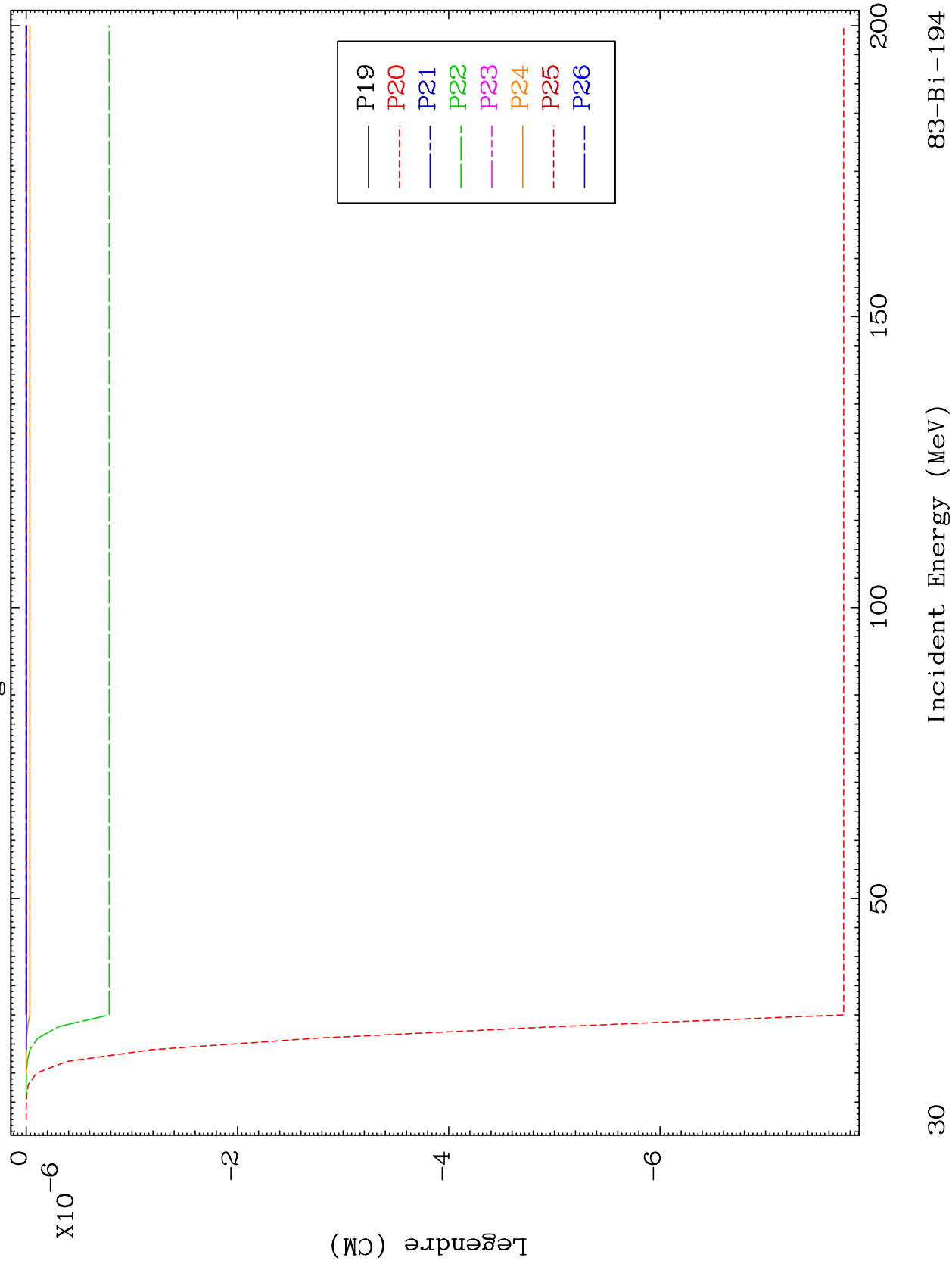


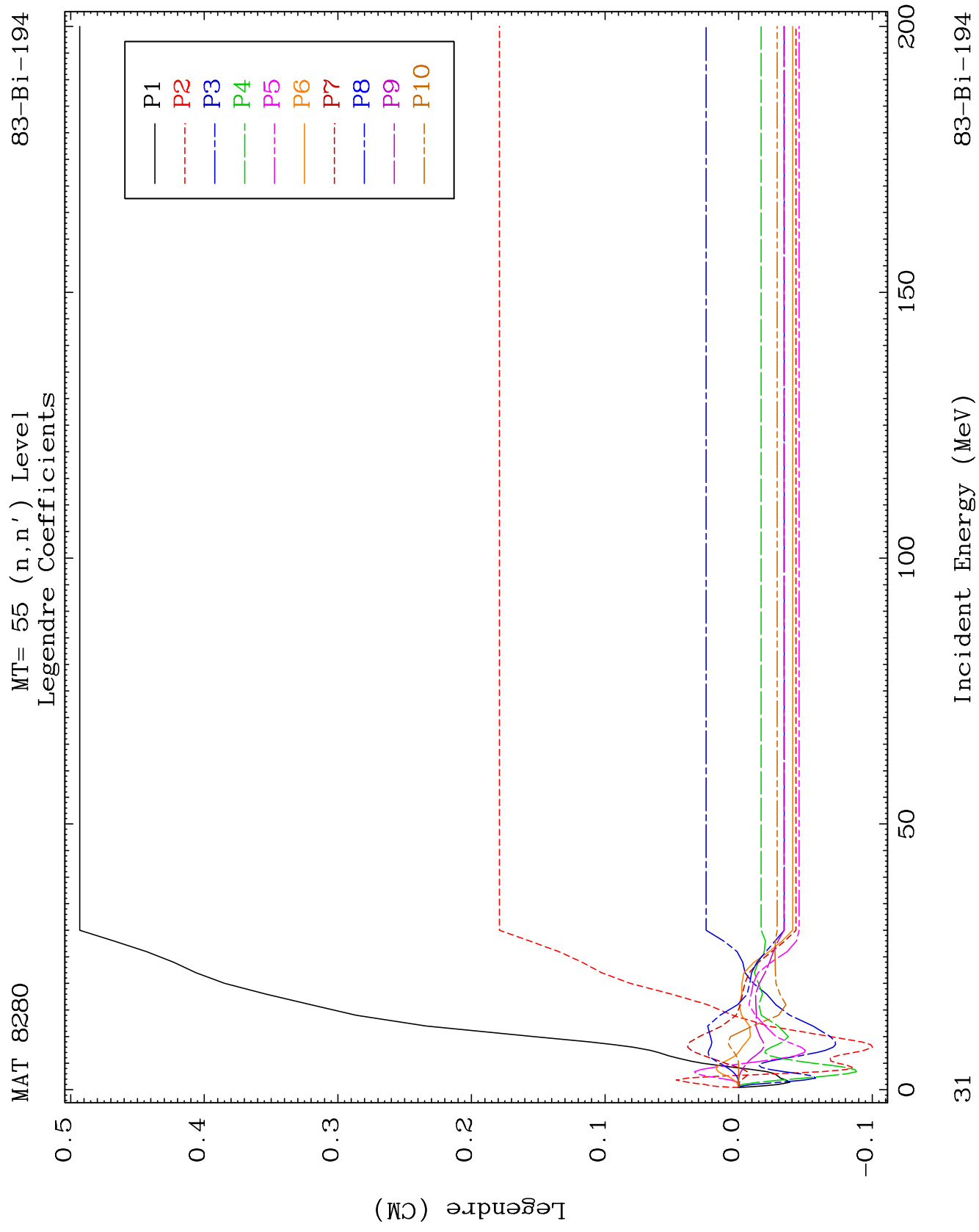


MAT 8280

MT= 54 (n, n') Level
Legendre Coefficients

83-Bi-194

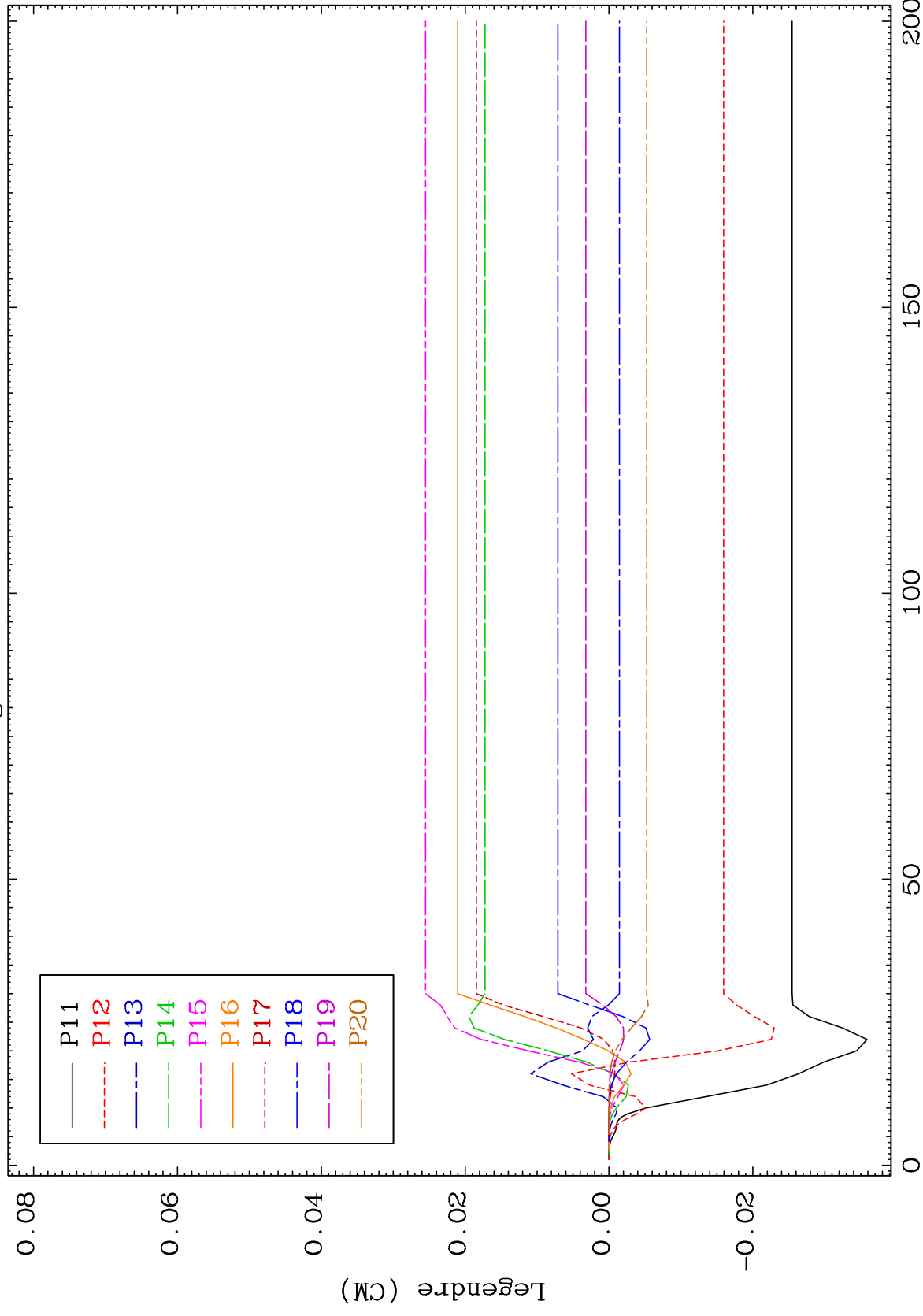




MAT 8280

MT= 55 (n, n') Level
Legendre Coefficients

83-Bi-194



23

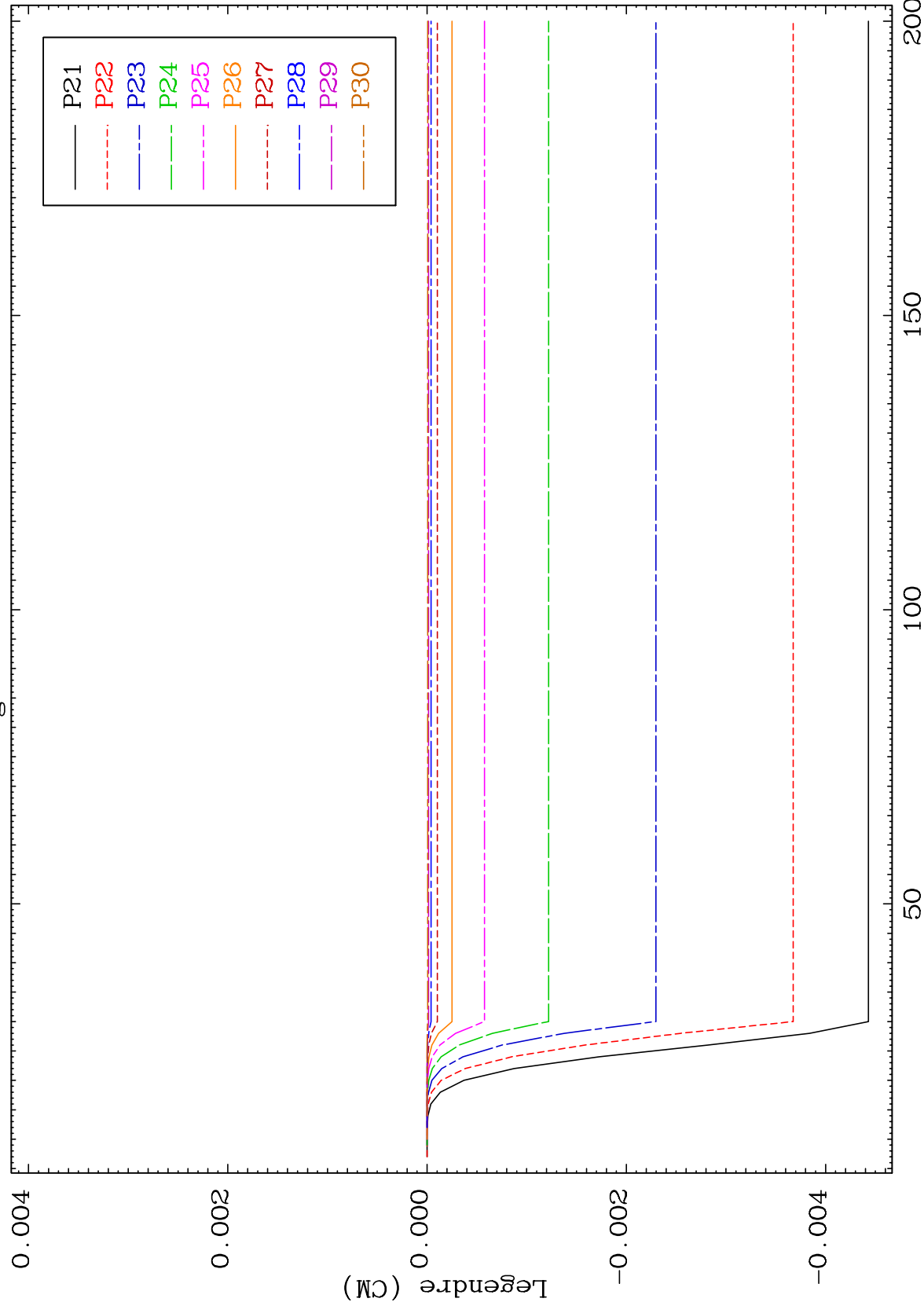
Incident Energy (MeV)

83-Bi-194

MAT 8280

MT= 55 (n, n') Level
Legendre Coefficients

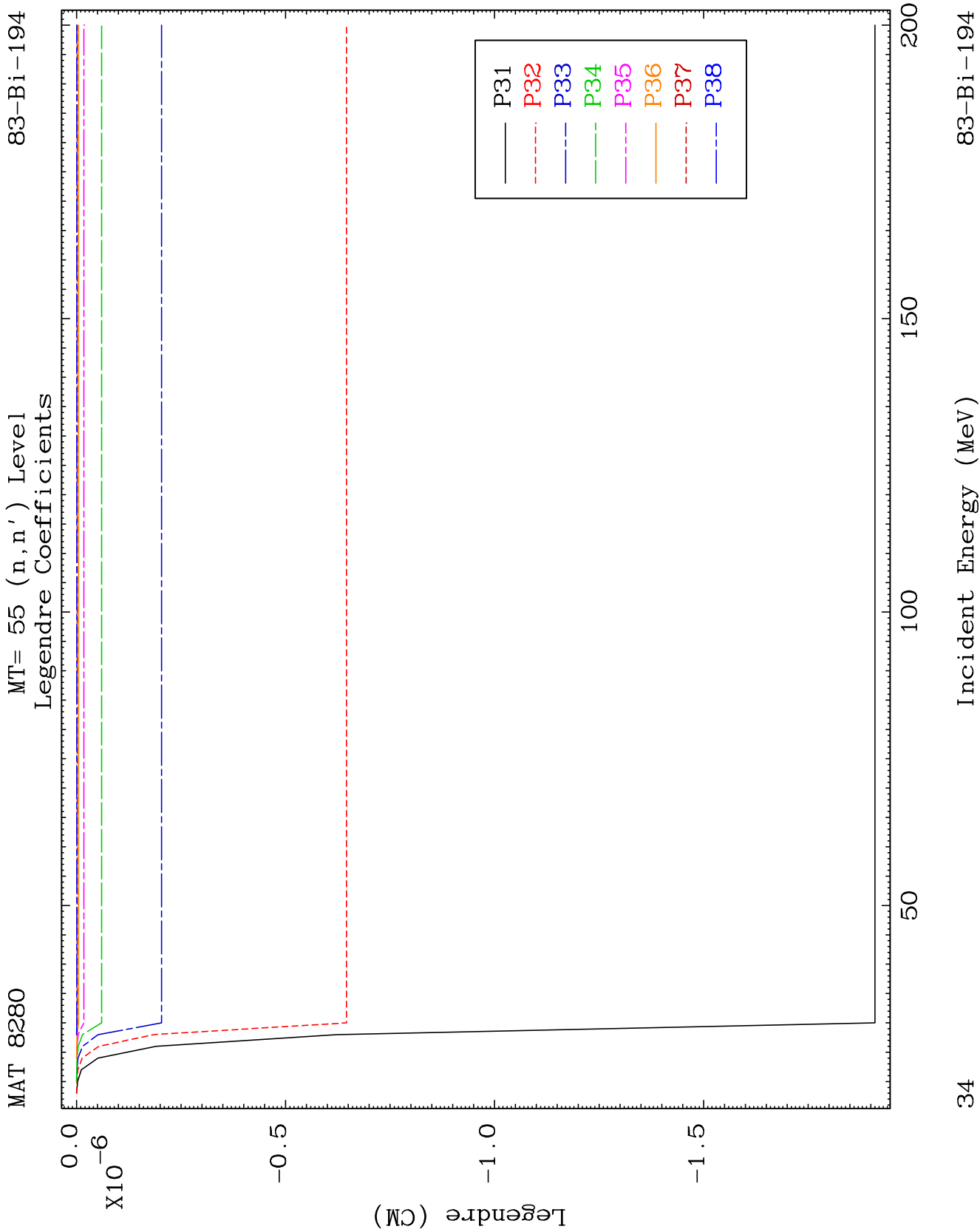
83-Bi-194



33

Incident Energy (MeV)

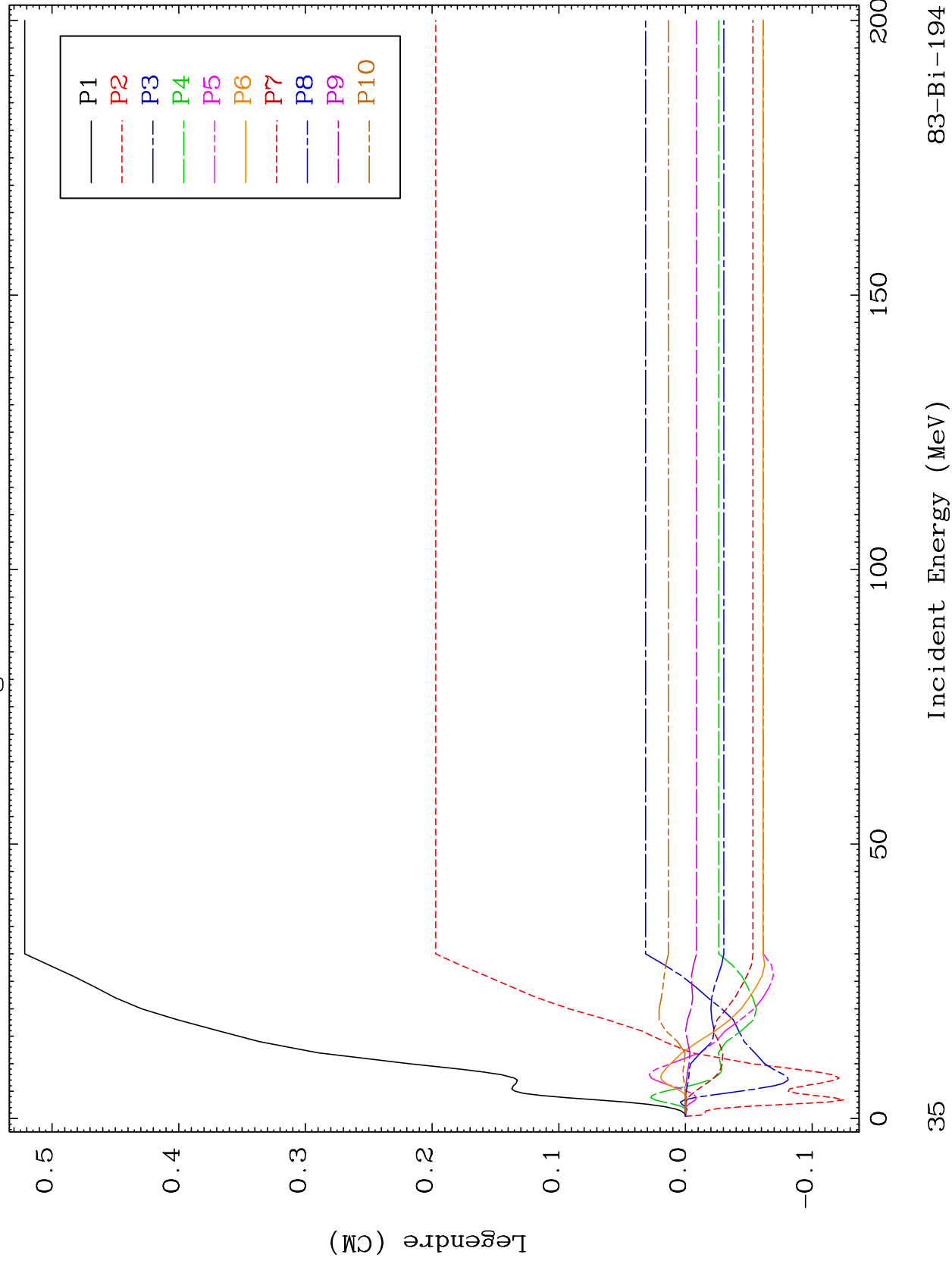
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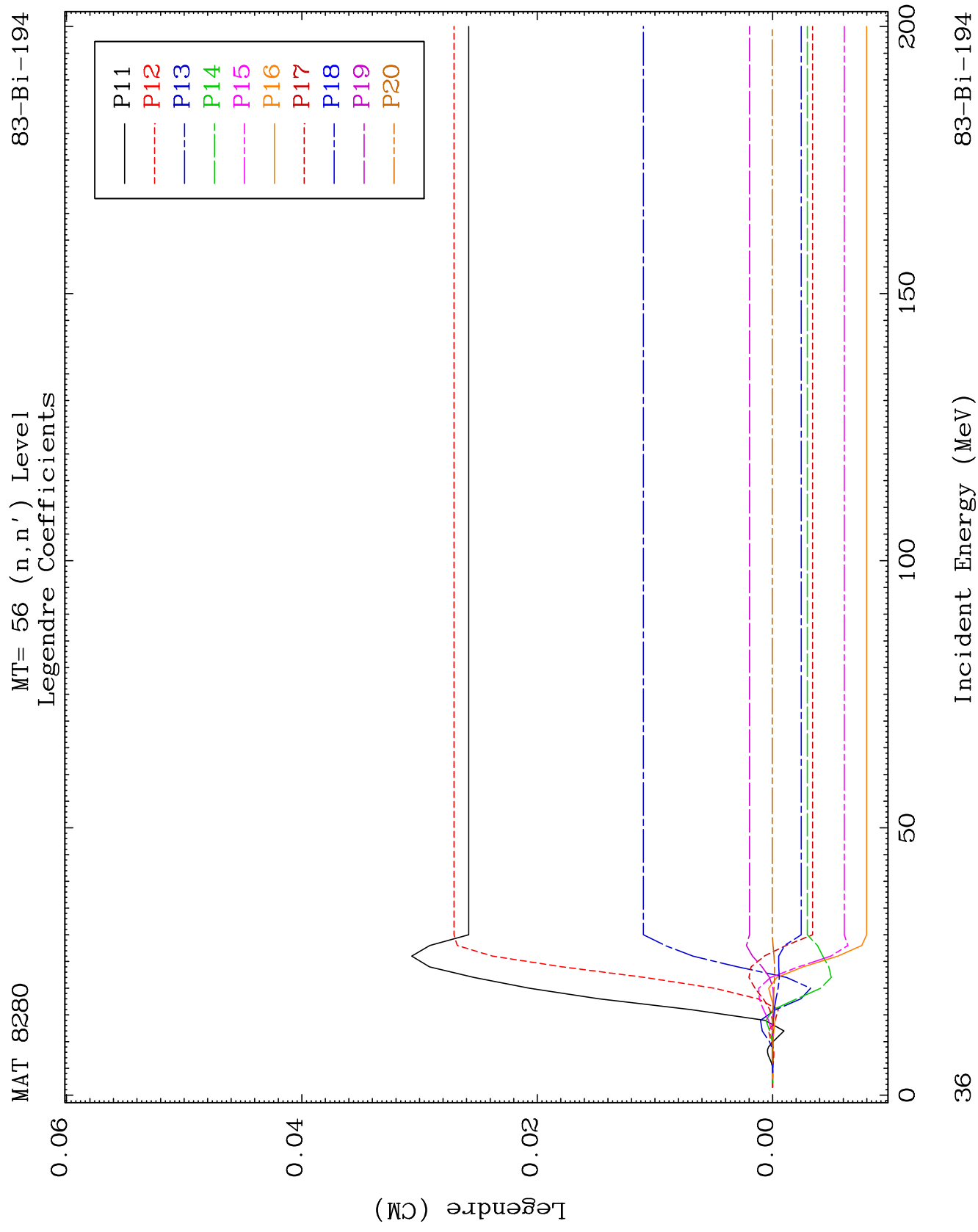


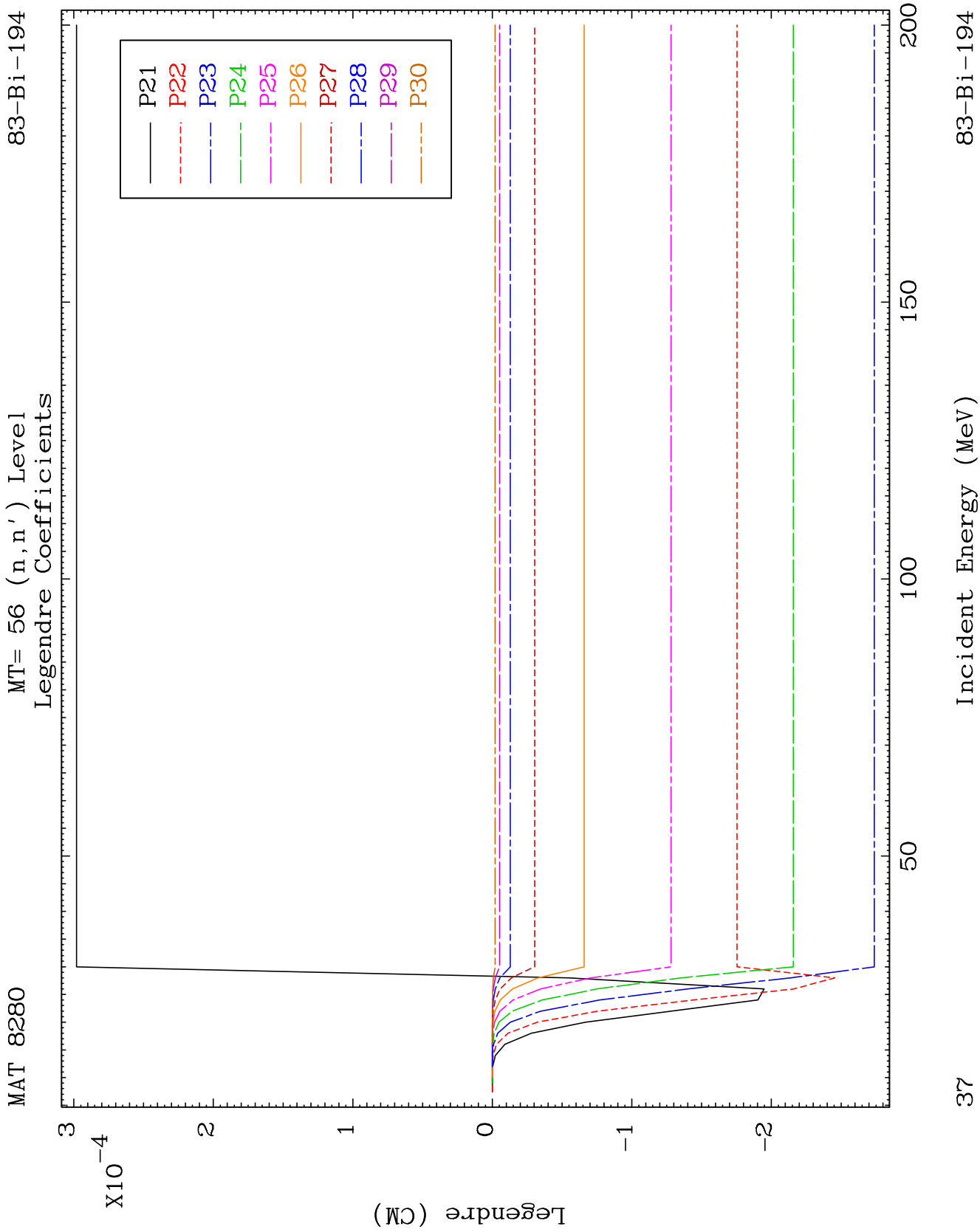
MAT 8280

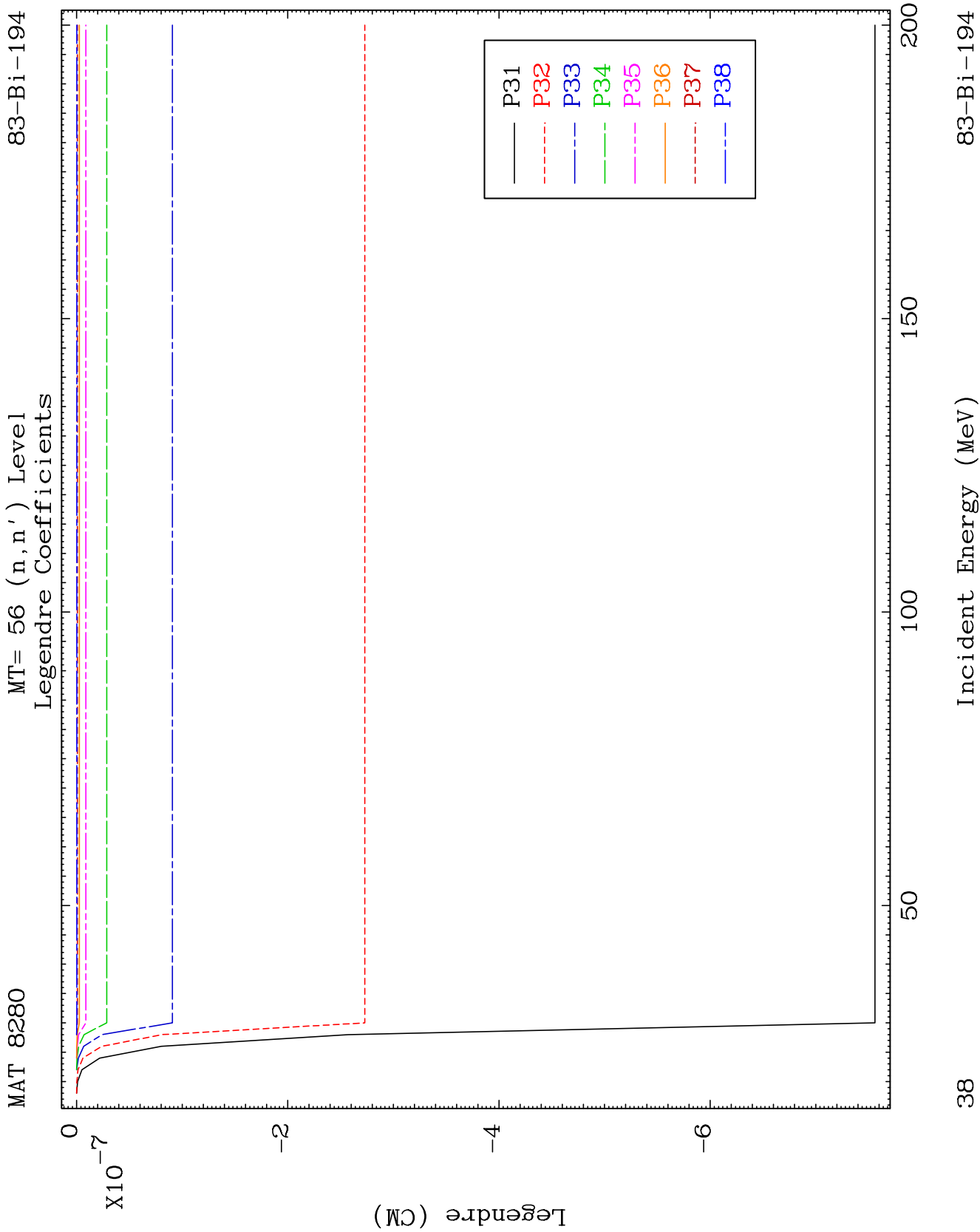
MT= 56 (n, n') Level
Legendre Coefficients

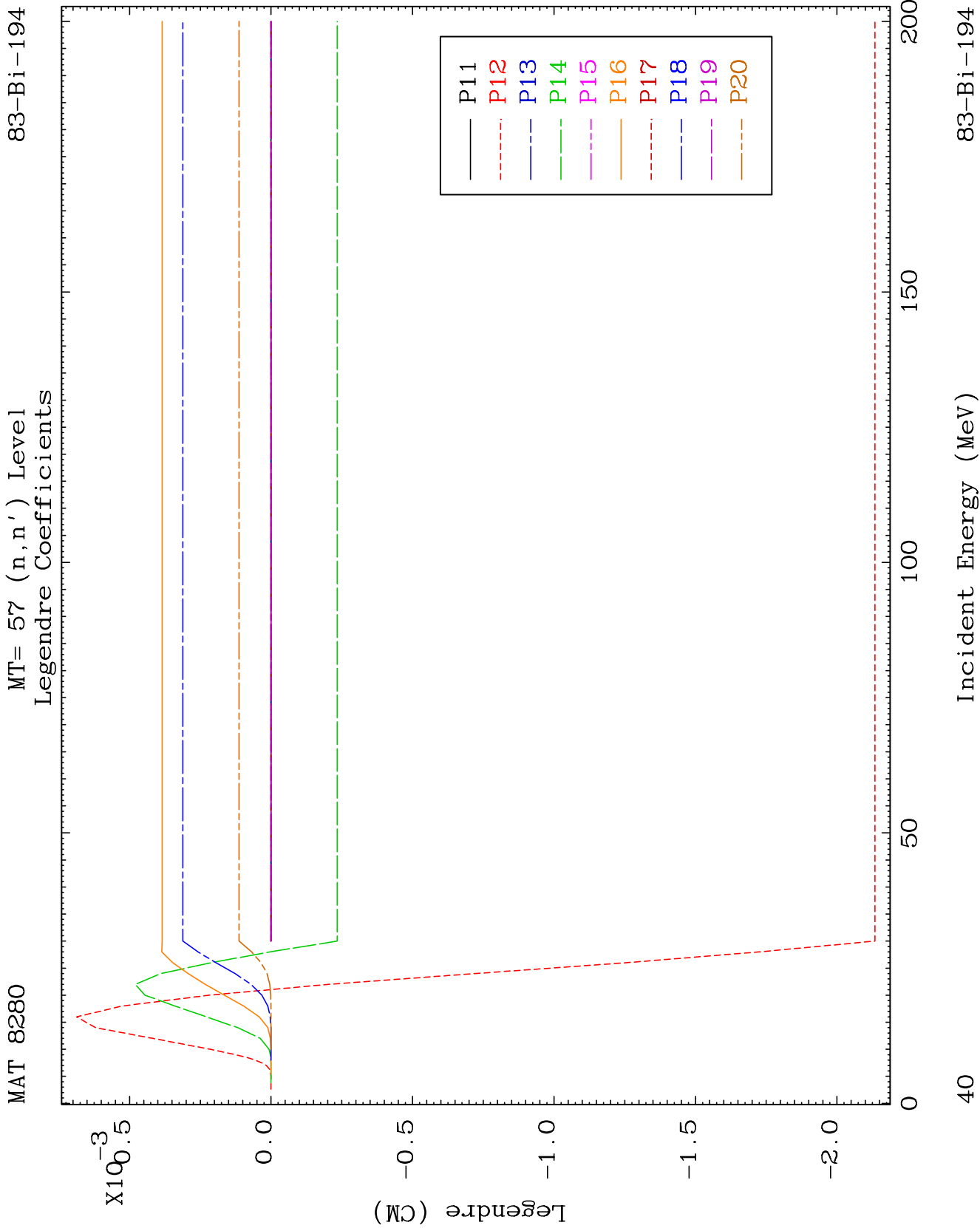
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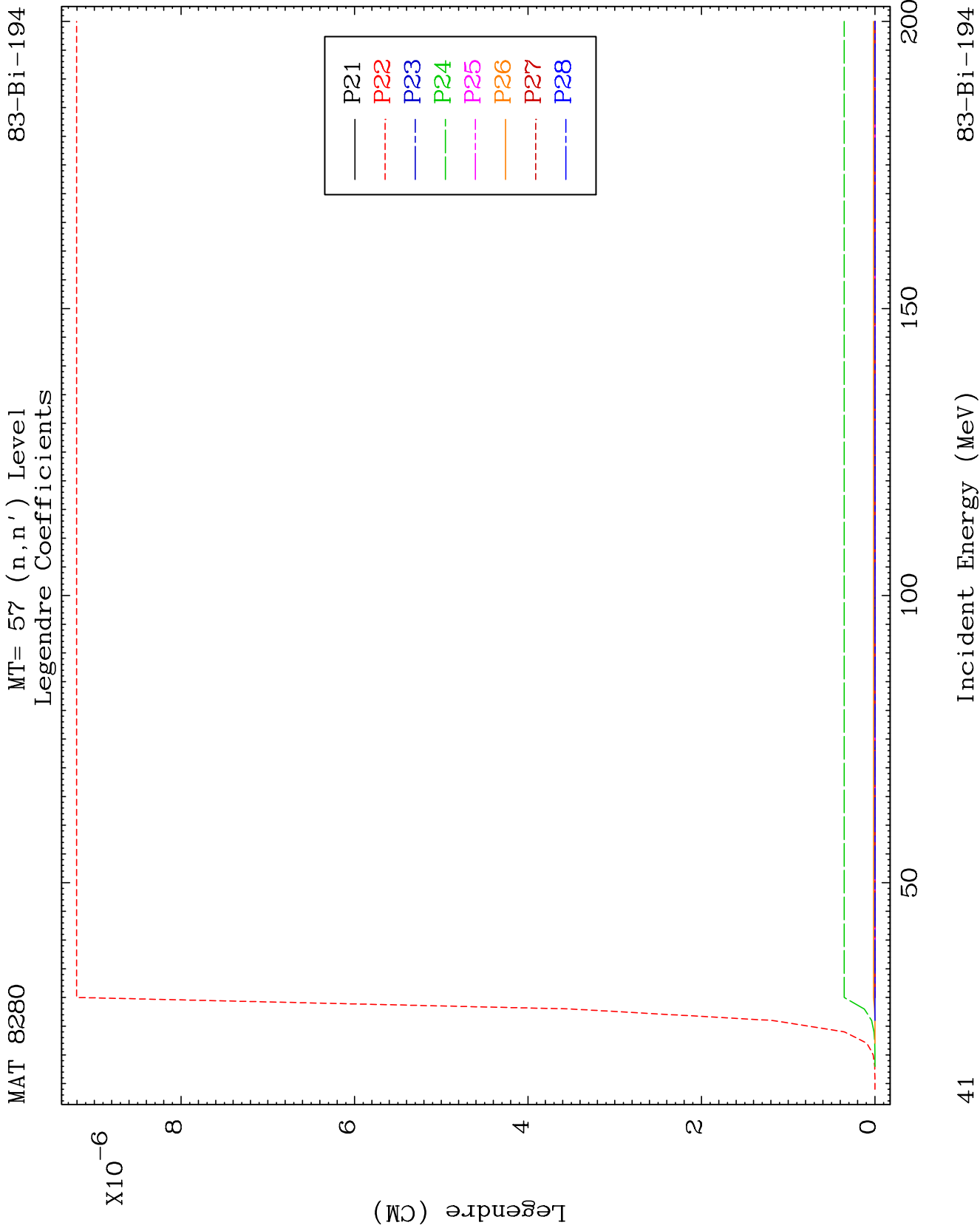








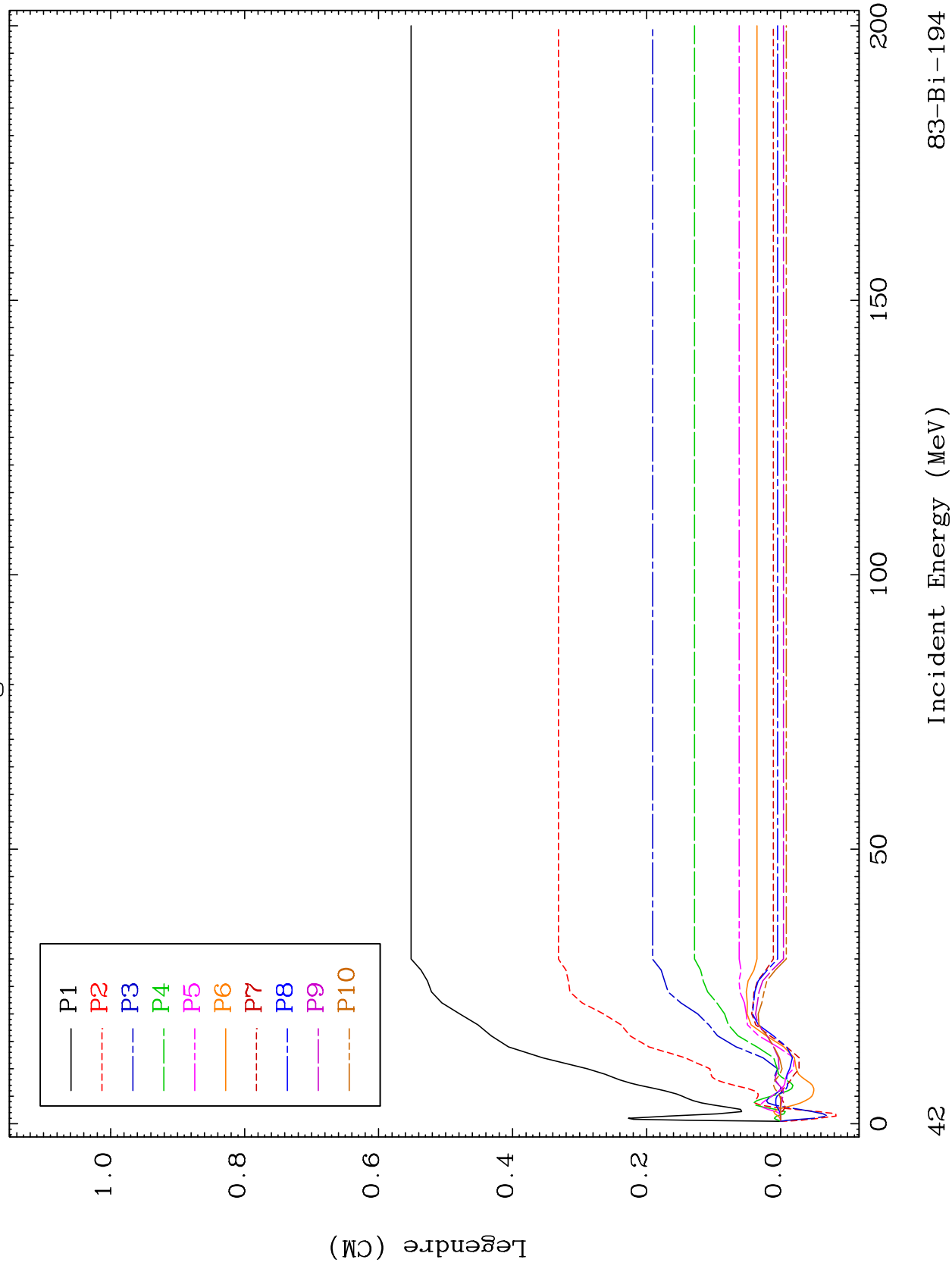




MAT 8280

MT= 58 (n, n') Level
Legendre Coefficients

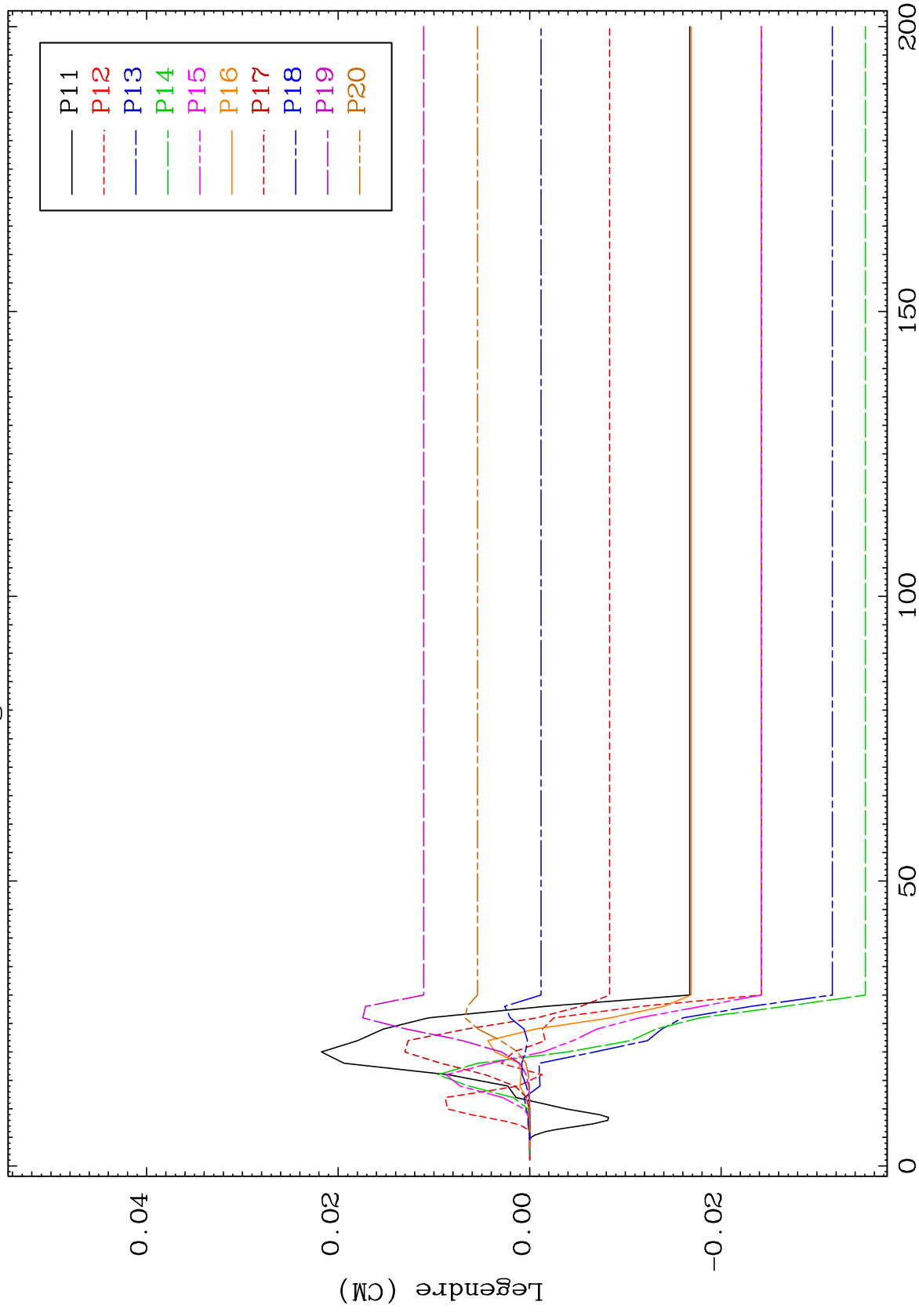
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MAT 8280

MT= 58 (n, n') Level
Legendre Coefficients

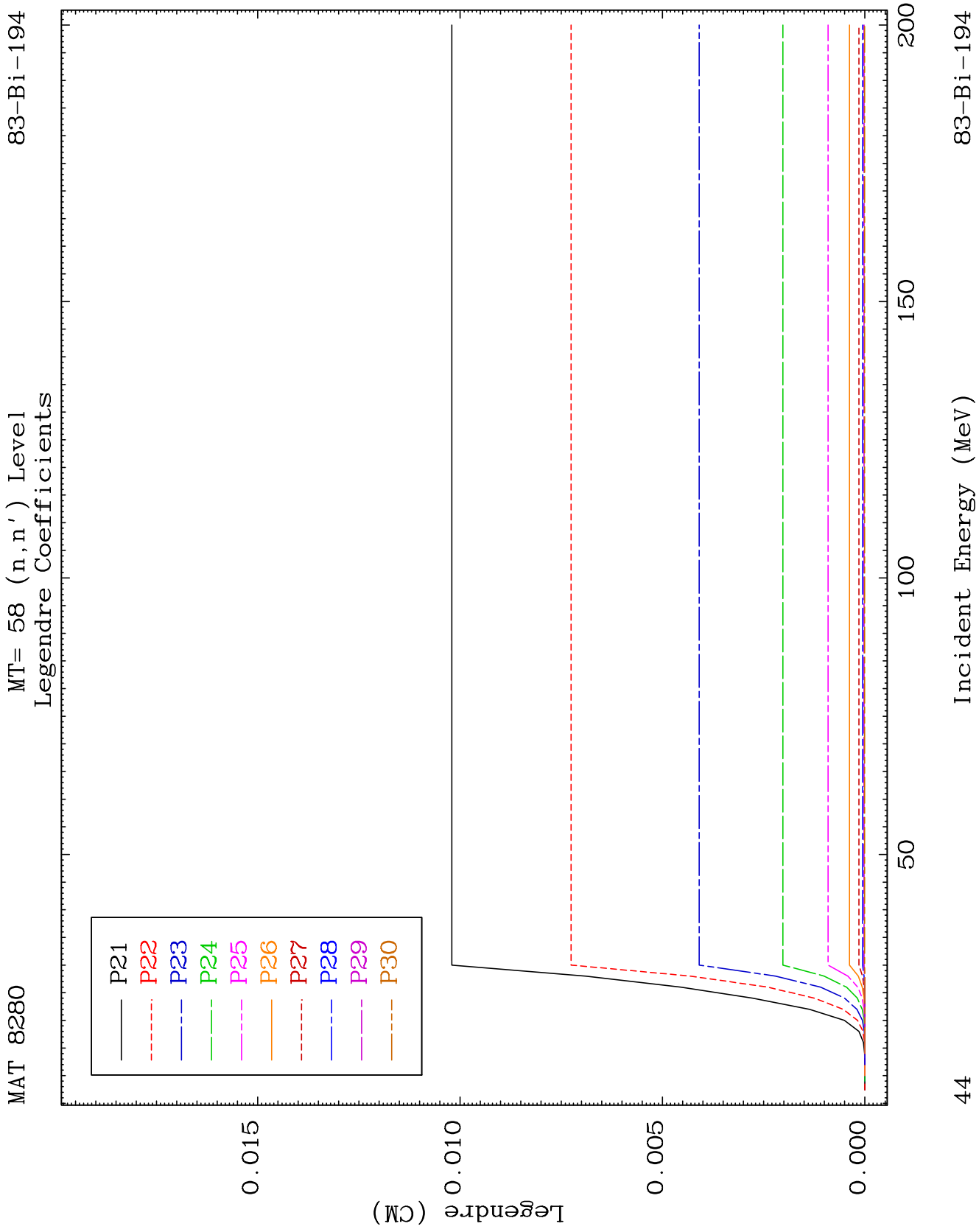
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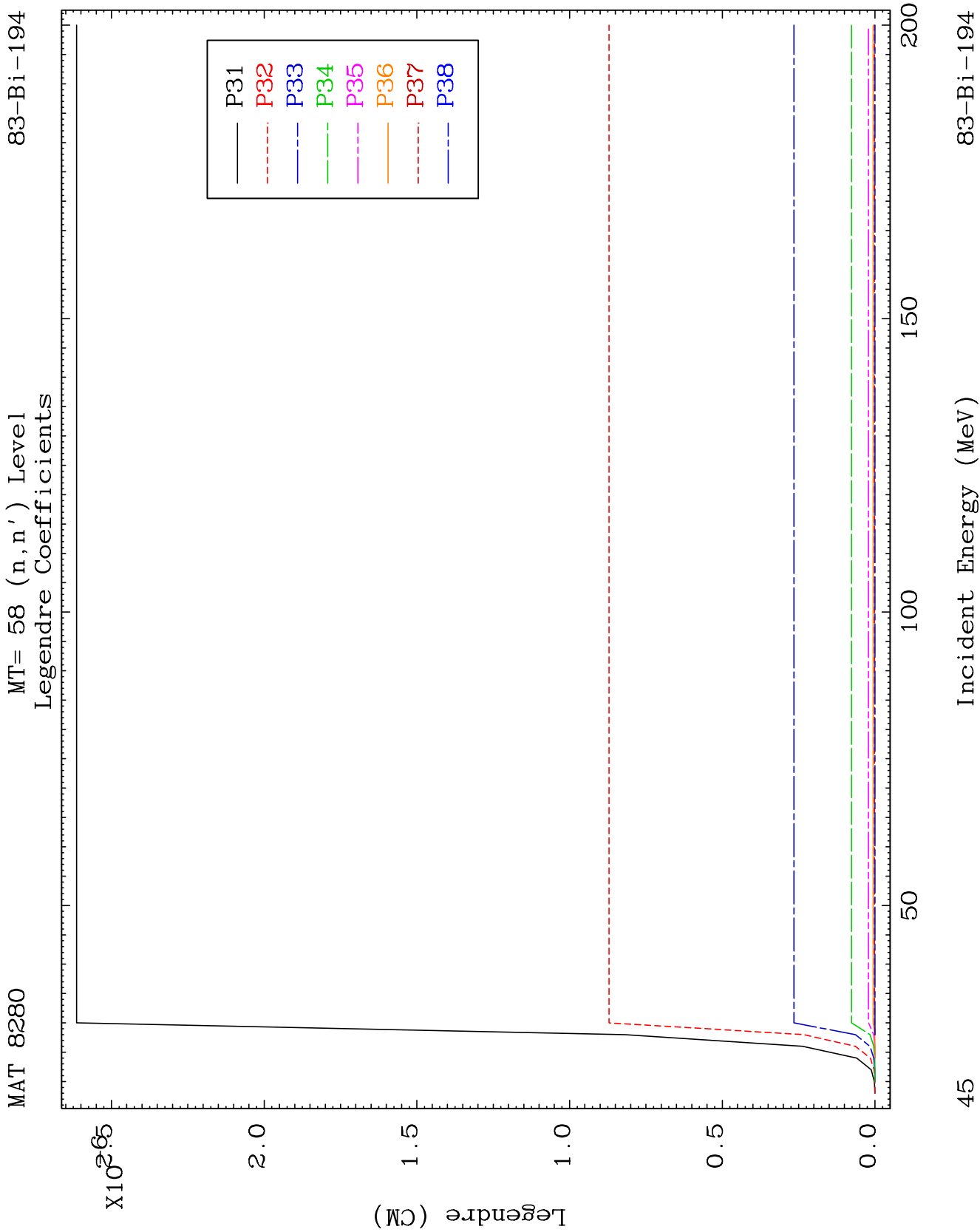


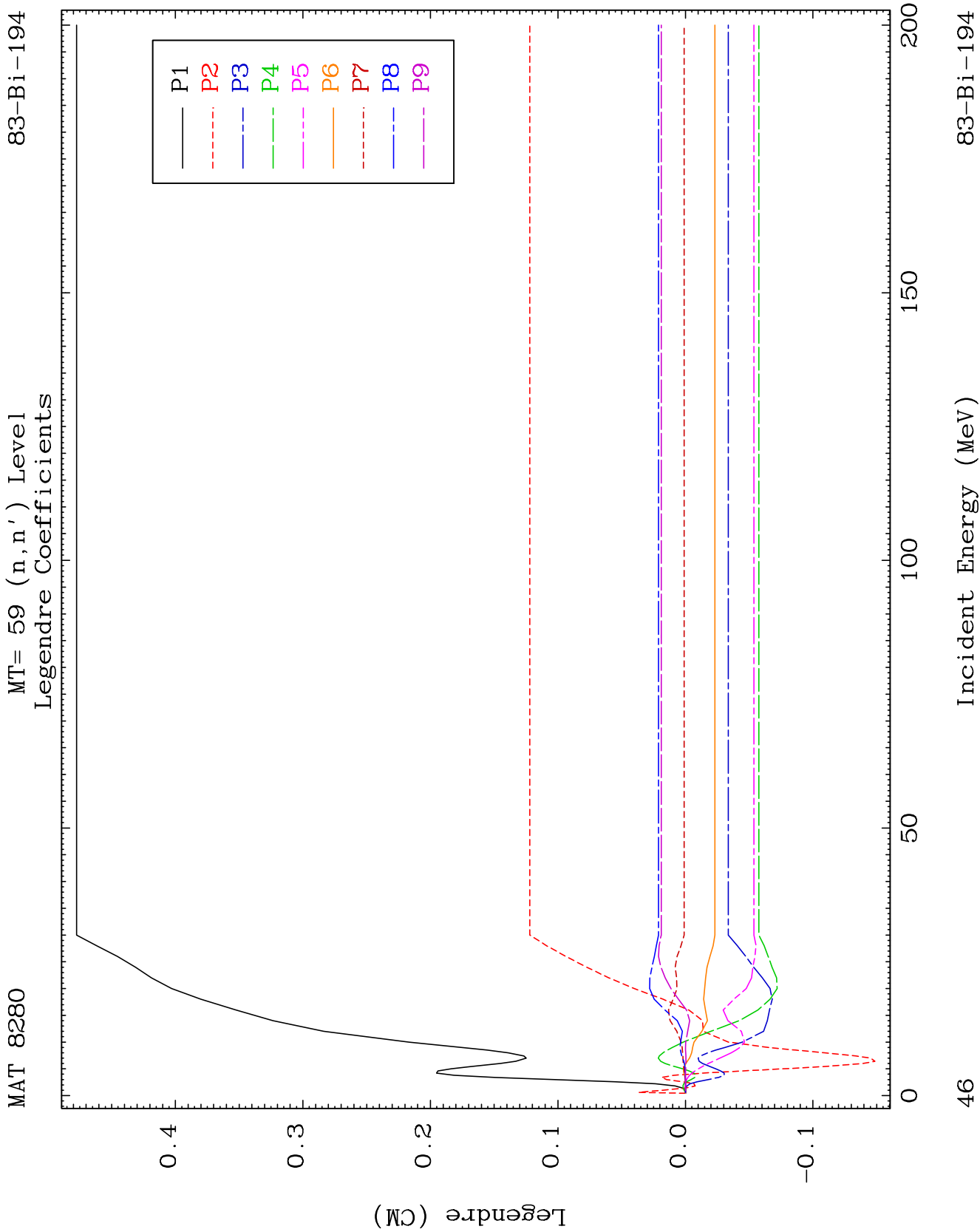
43

Incident Energy (MeV)

83-Bi-194



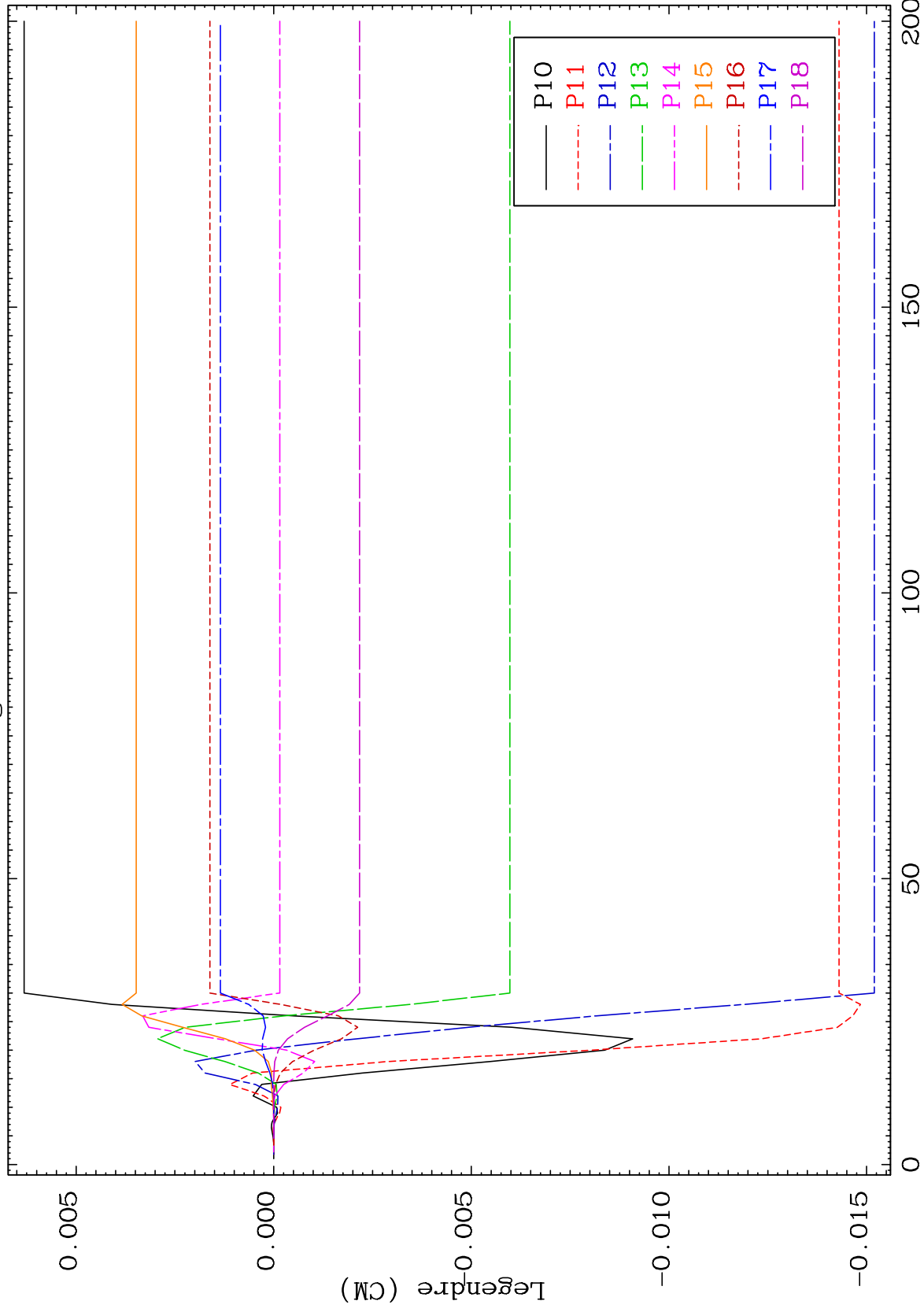




MAT 8280

MT= 59 (n,n') Level
Legendre Coefficients

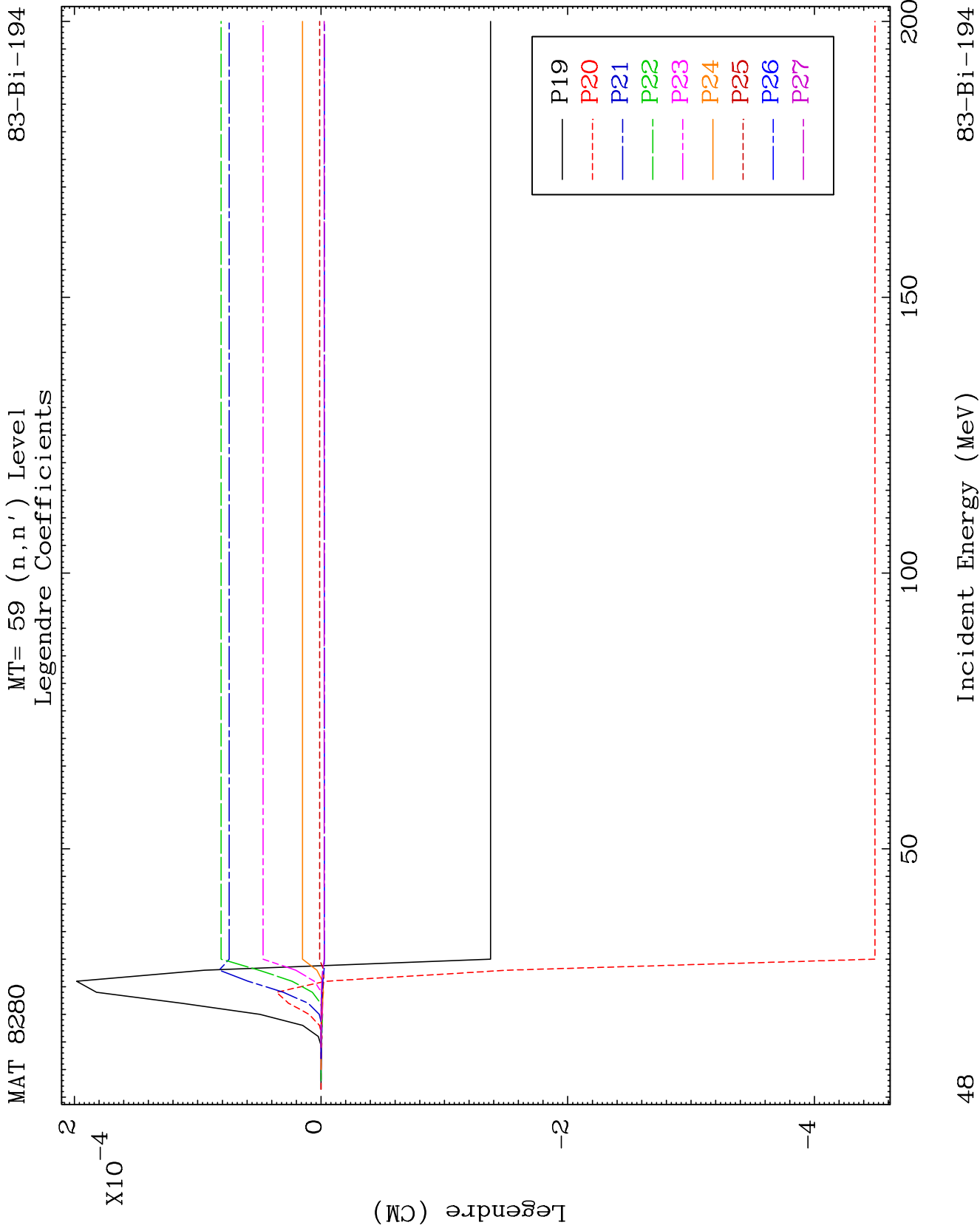
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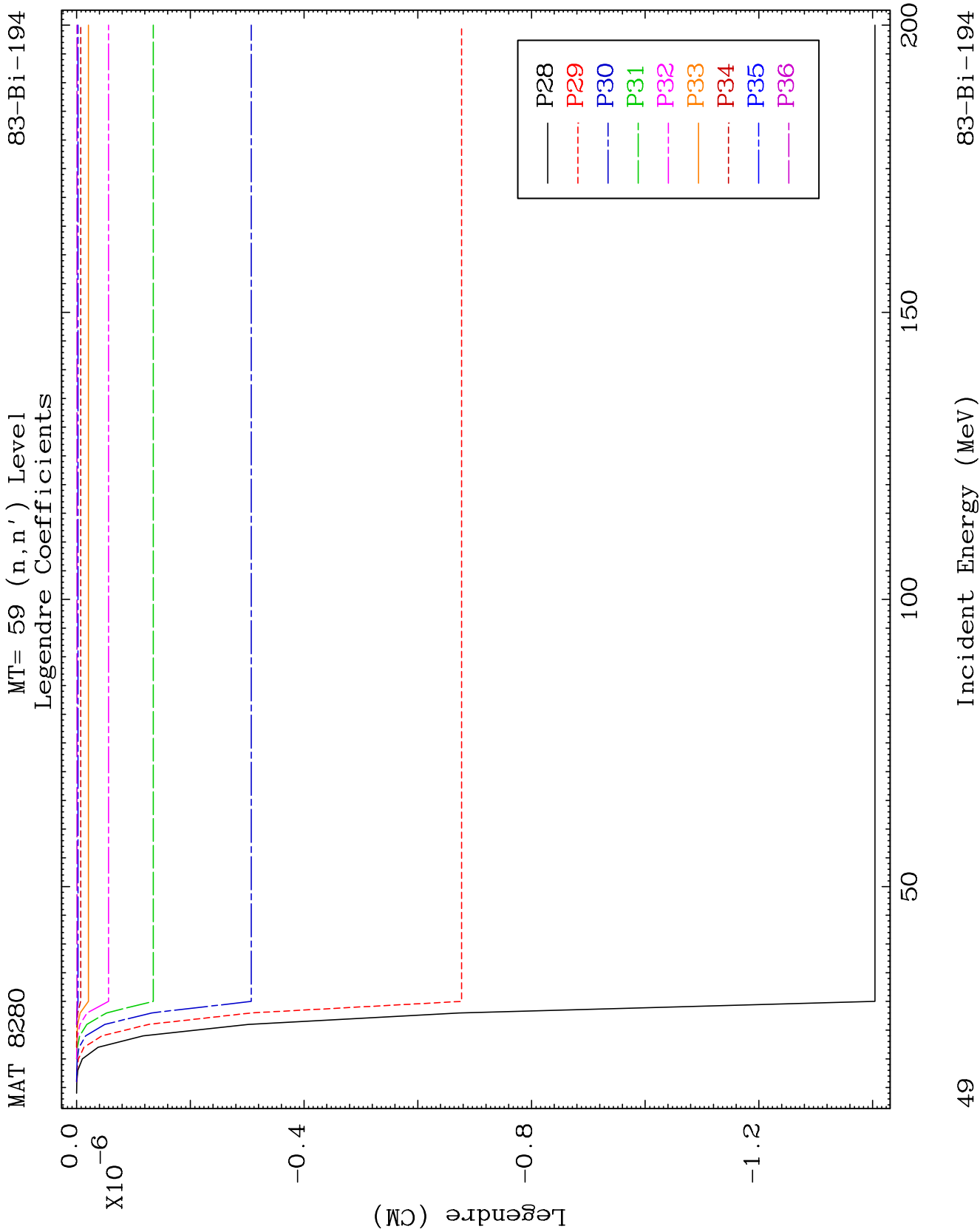


47

Incident Energy (MeV)

83-Bi-194

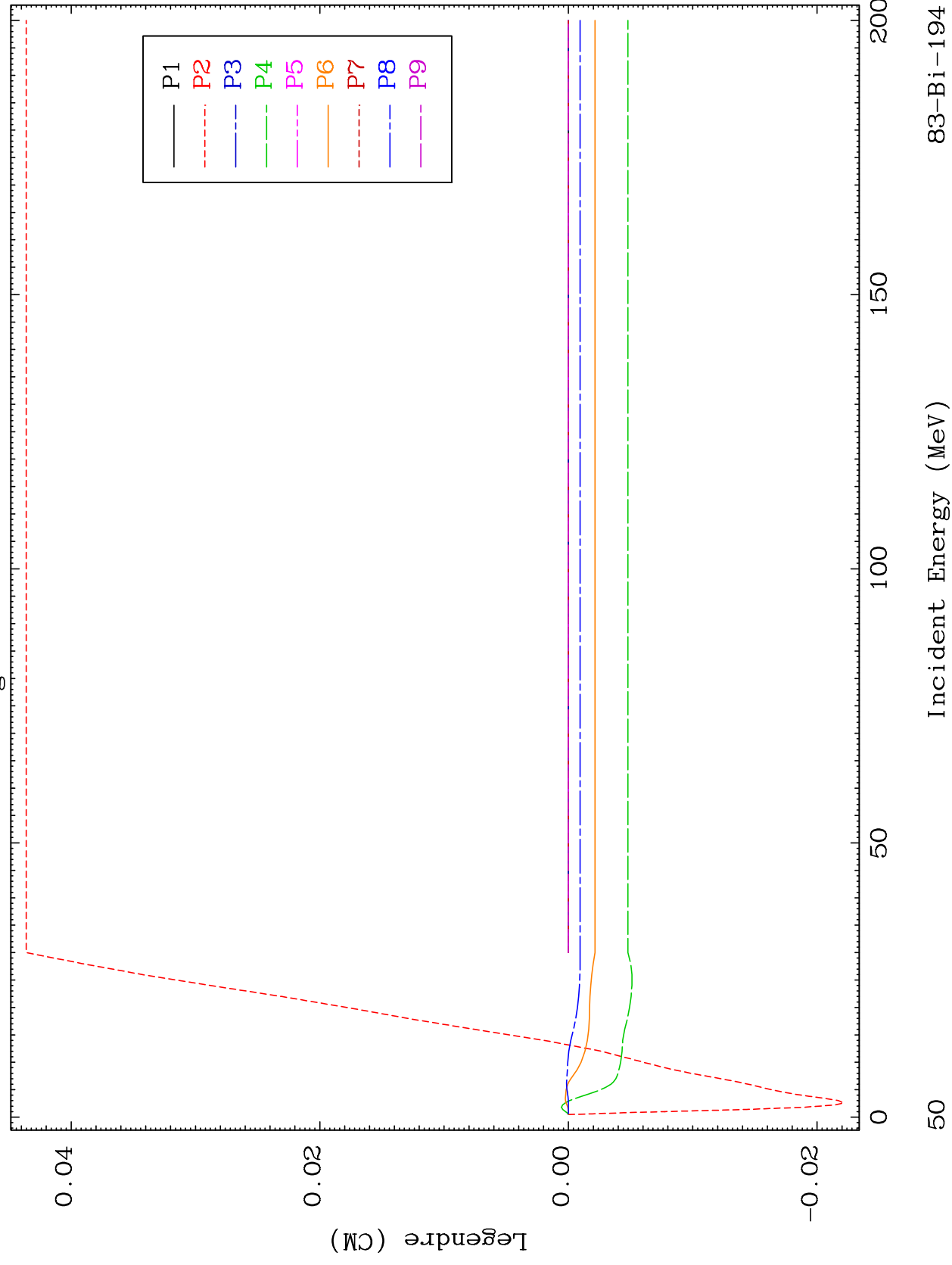




MAT 8280

MT= 60 (n, n') Level
Legendre Coefficients

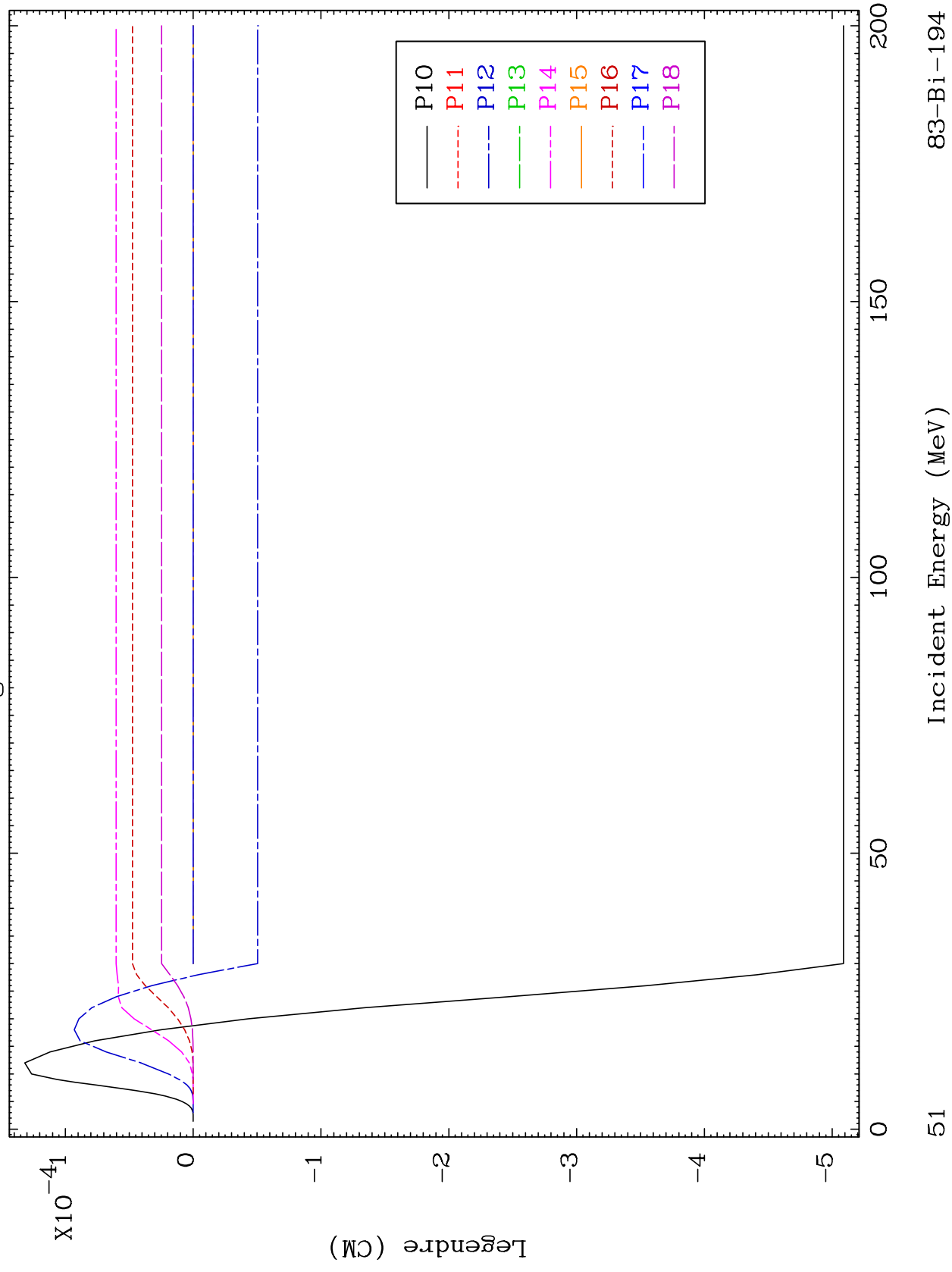
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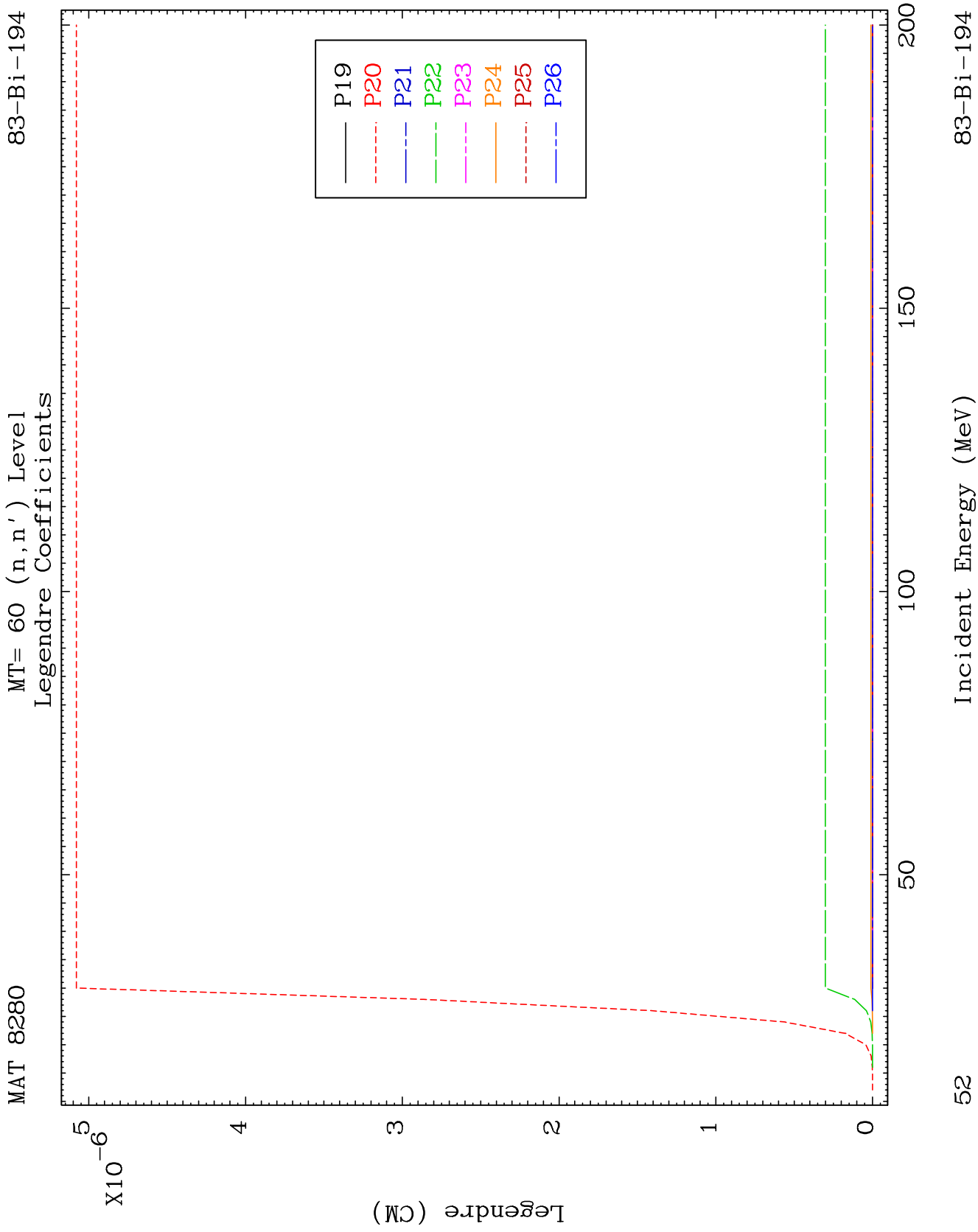


MAT 8280

83-Bi-194

MT= 60 (n, n') Level
Legendre Coefficients

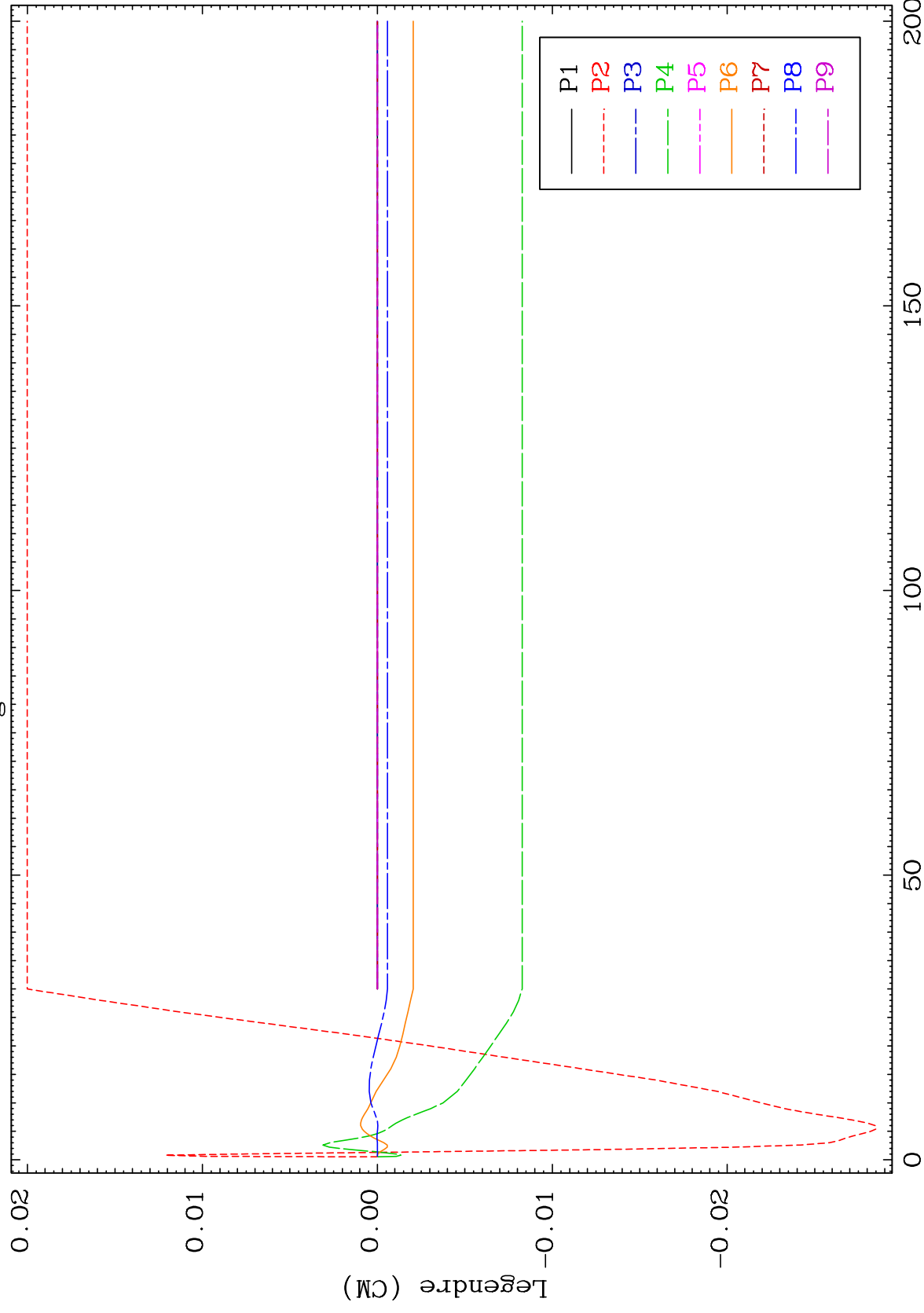




MAT 8280

MT= 61 (n, n') Level
Legendre Coefficients

83-Bi-194



53

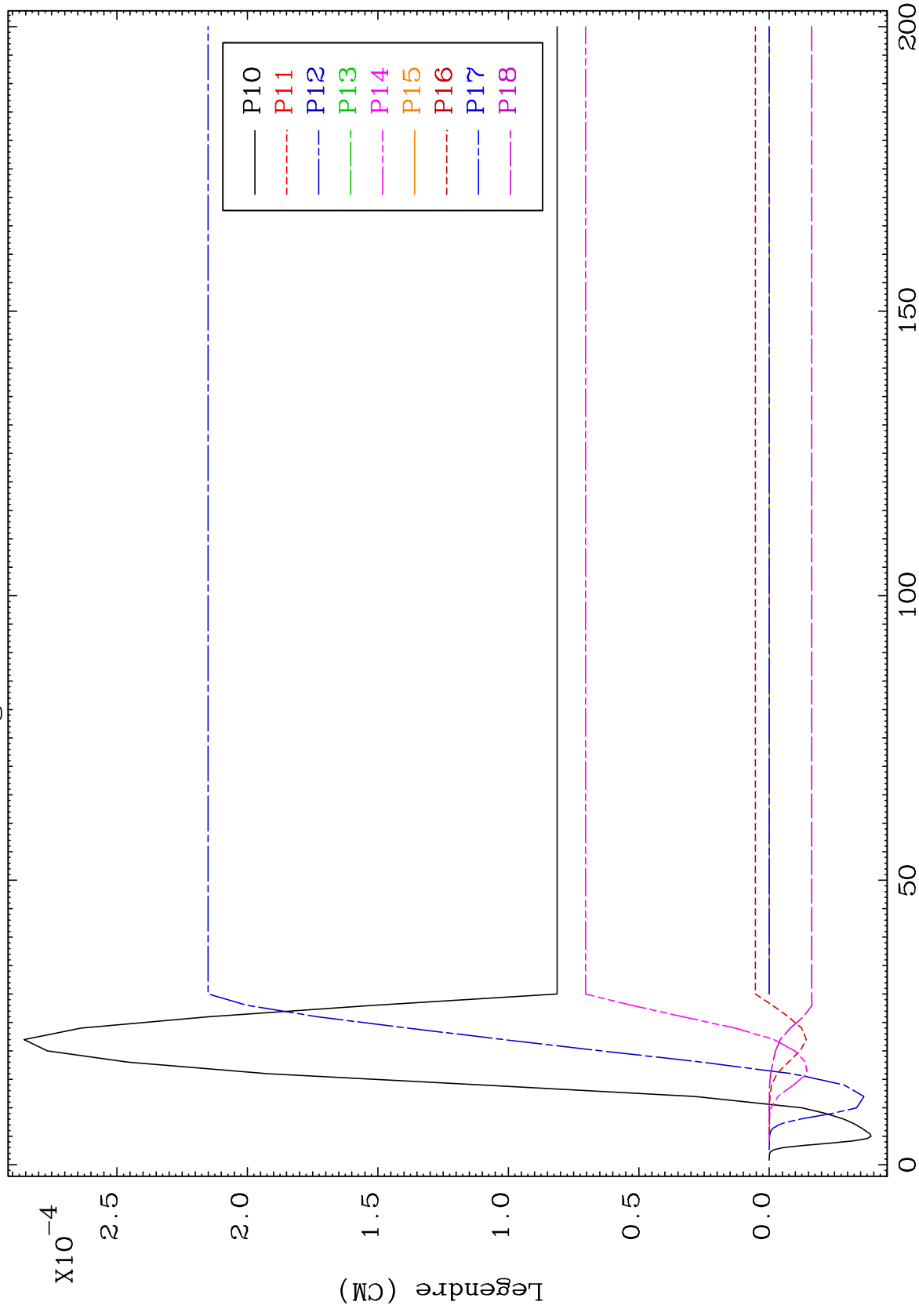
Incident Energy (MeV)

83-Bi-194

MAT 8280

MT= 61 (n, n') Level
Legendre Coefficients

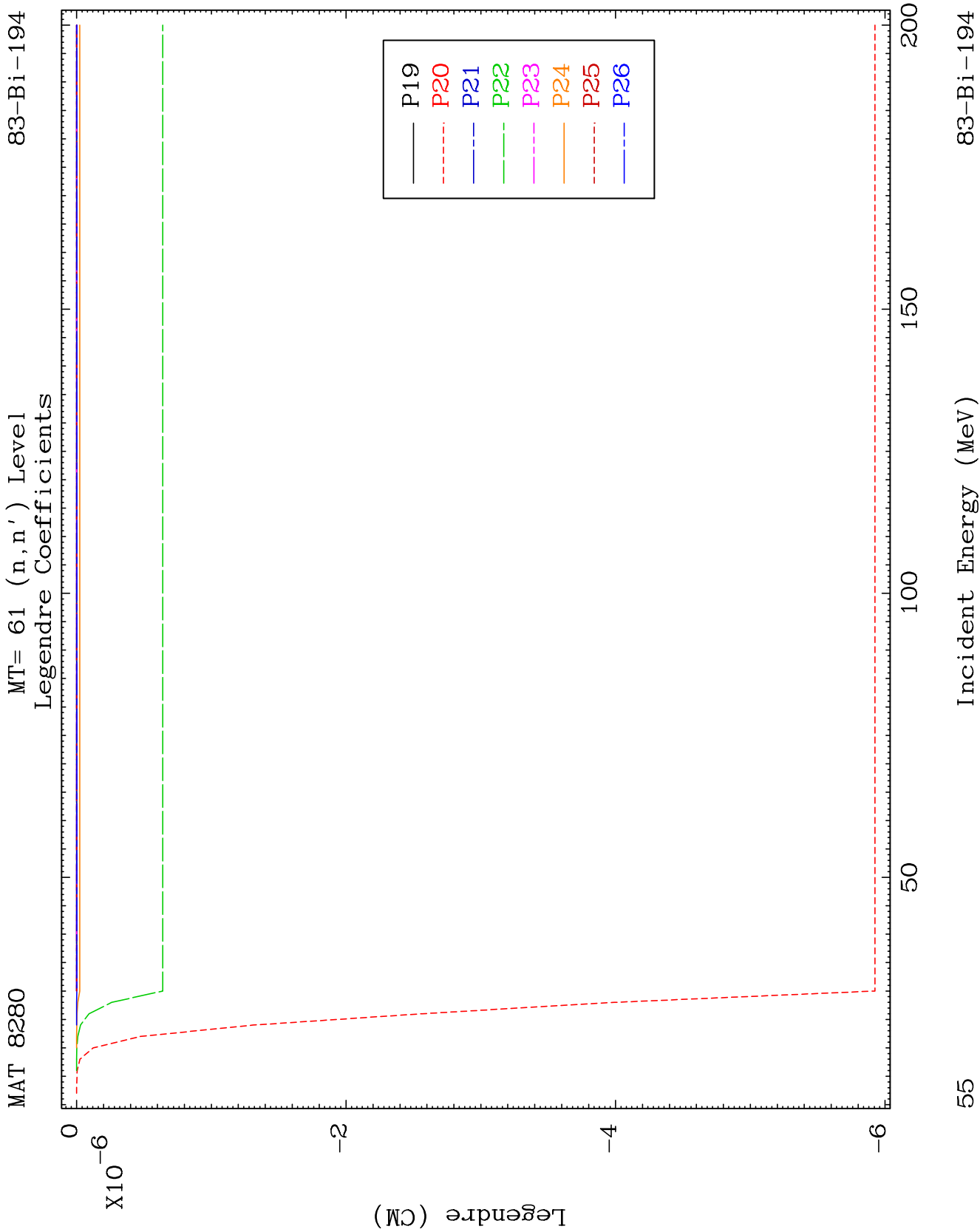
83-Bi-194



83-Bi-194

Incident Energy (MeV)

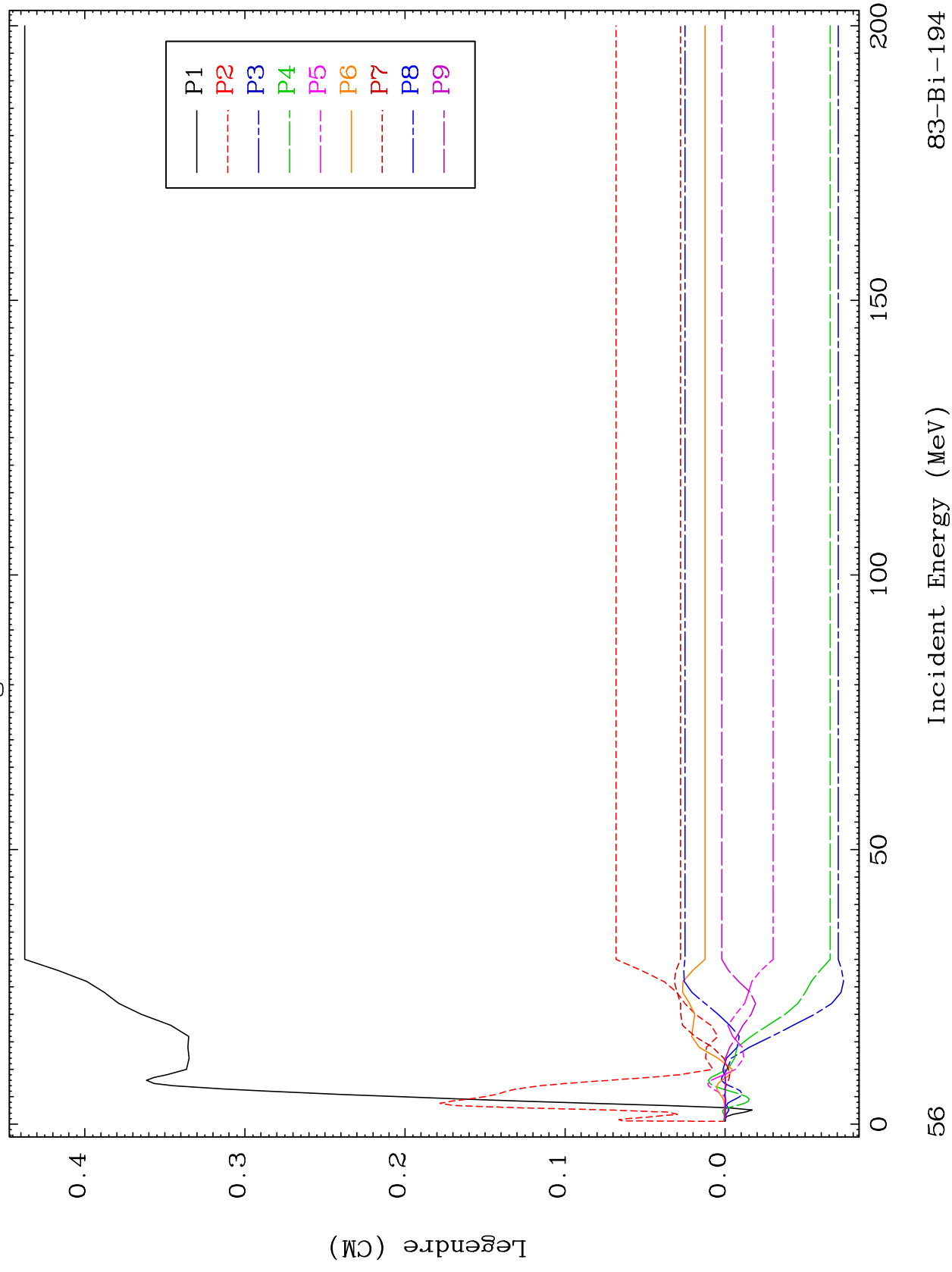
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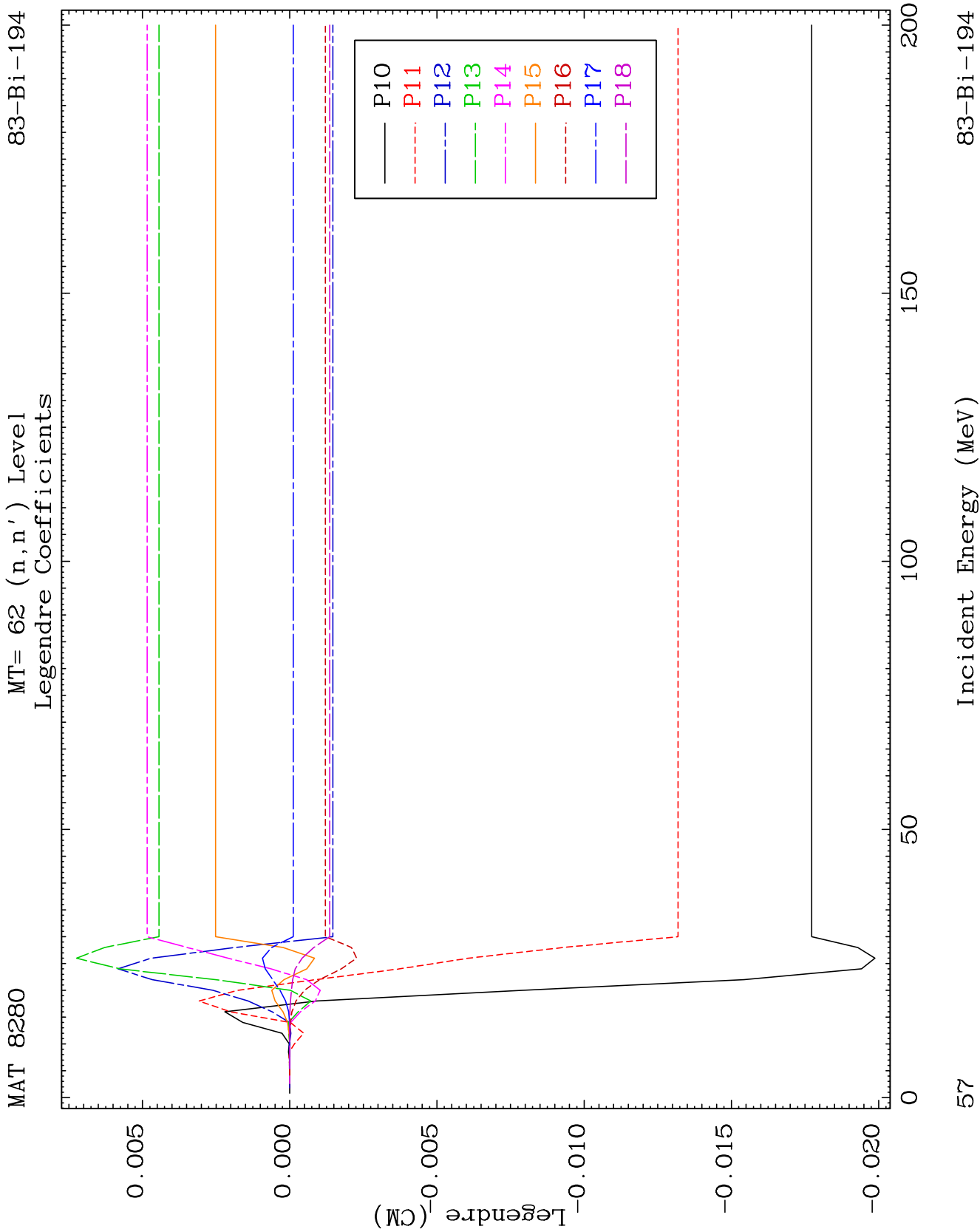


MAT 8280

MT= 62 (n, n') Level
Legendre Coefficients

83-Bi-194

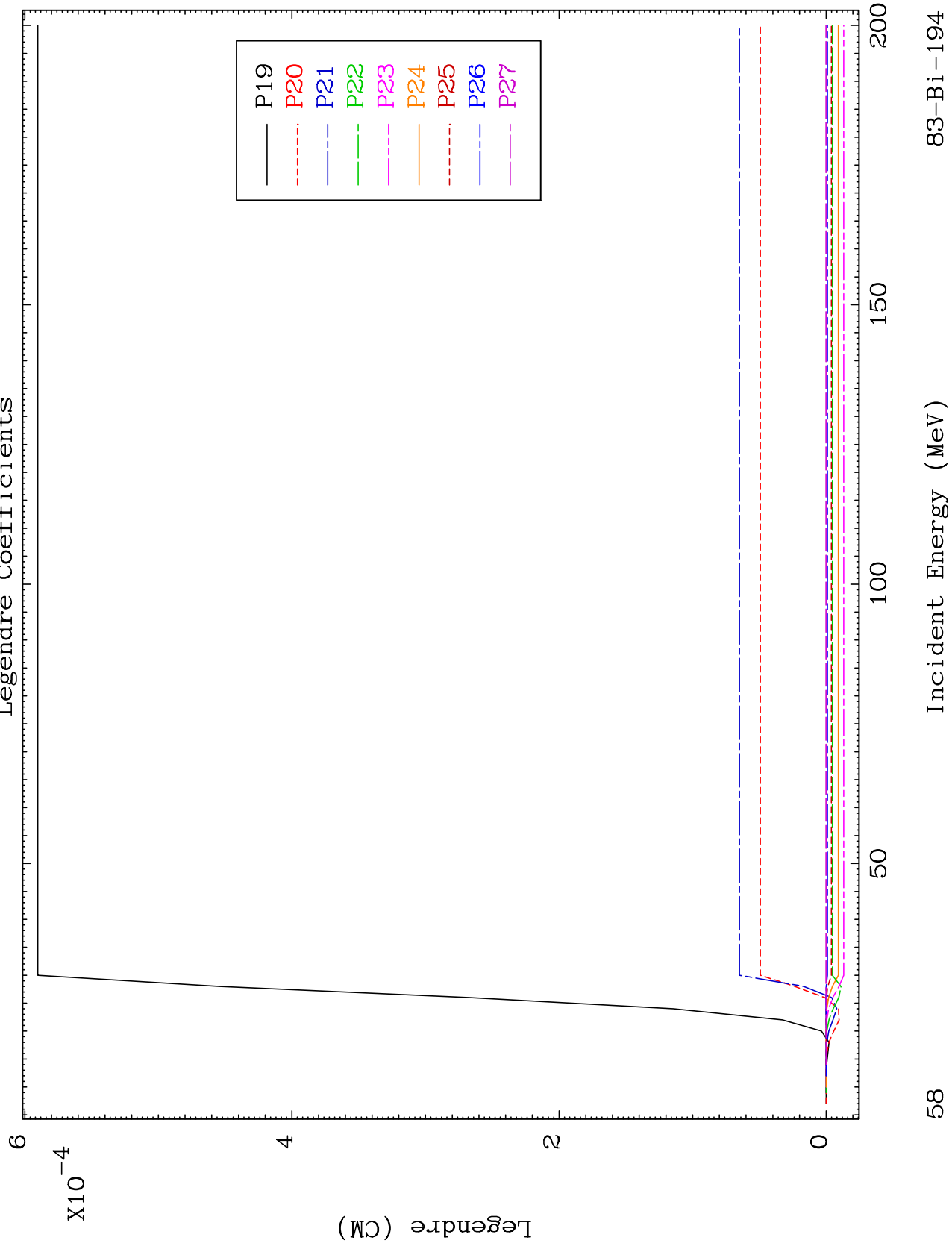


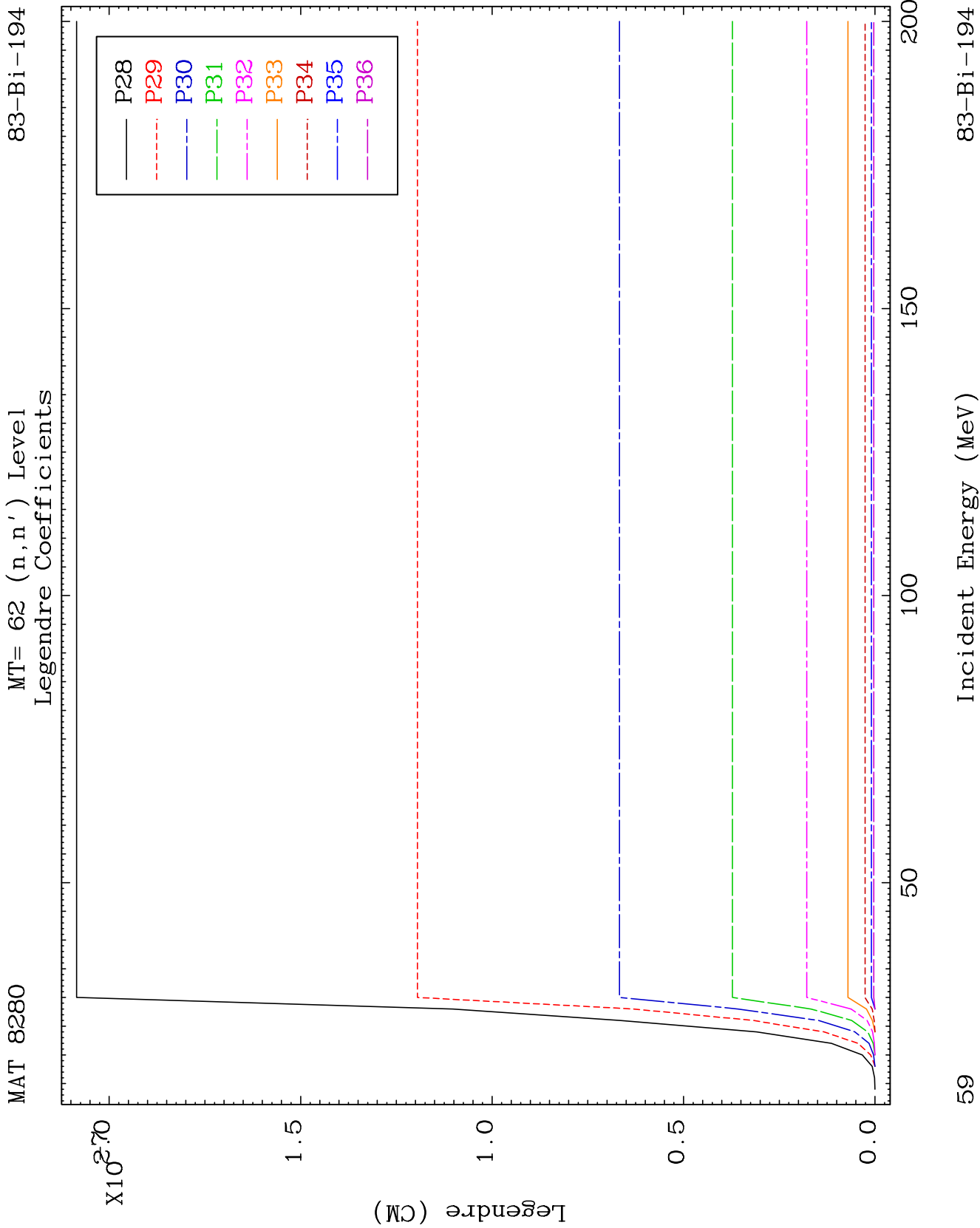


MAT 8280

MT= 62 (n, n') Level
Legendre Coefficients

83-Bi-194

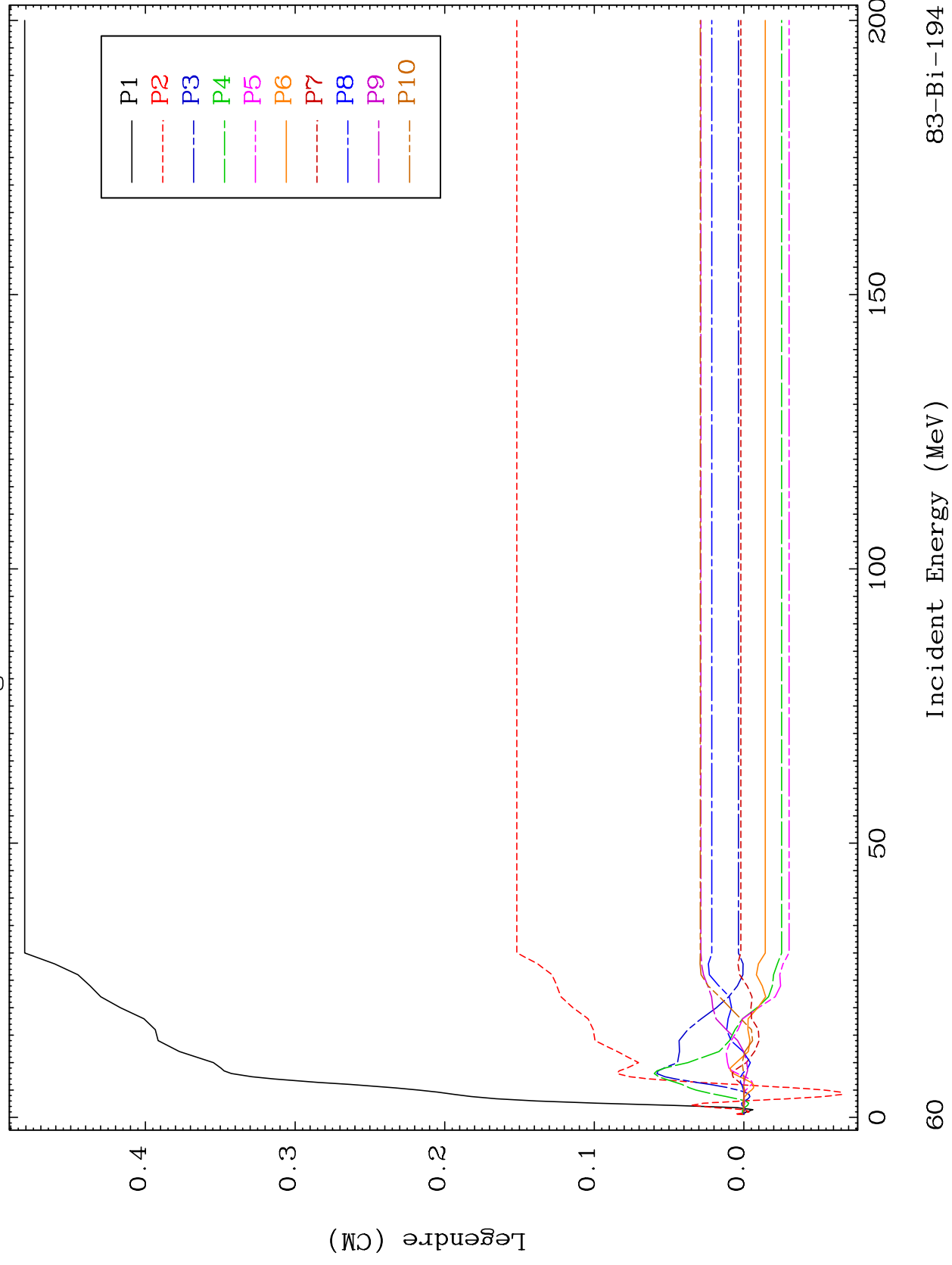


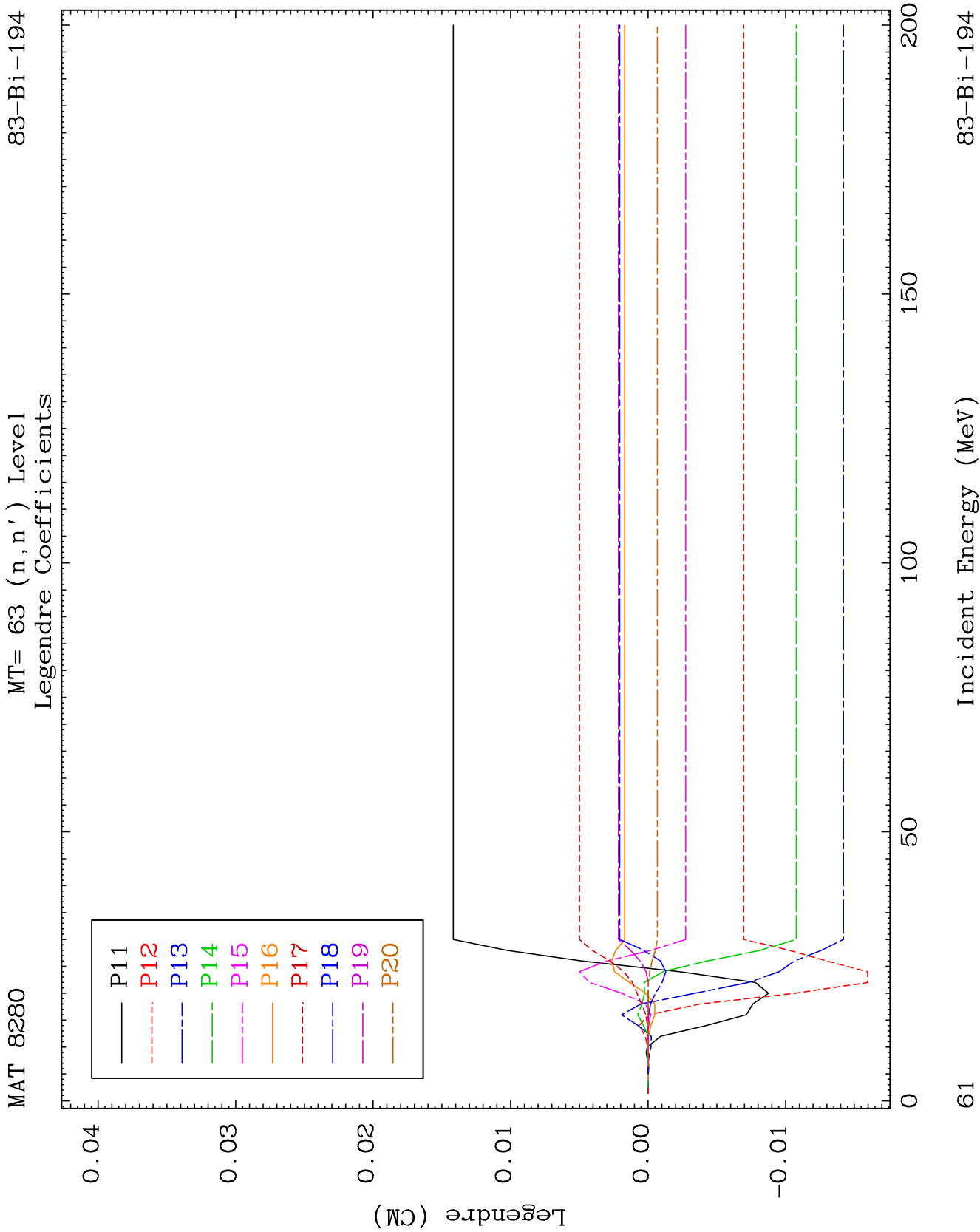


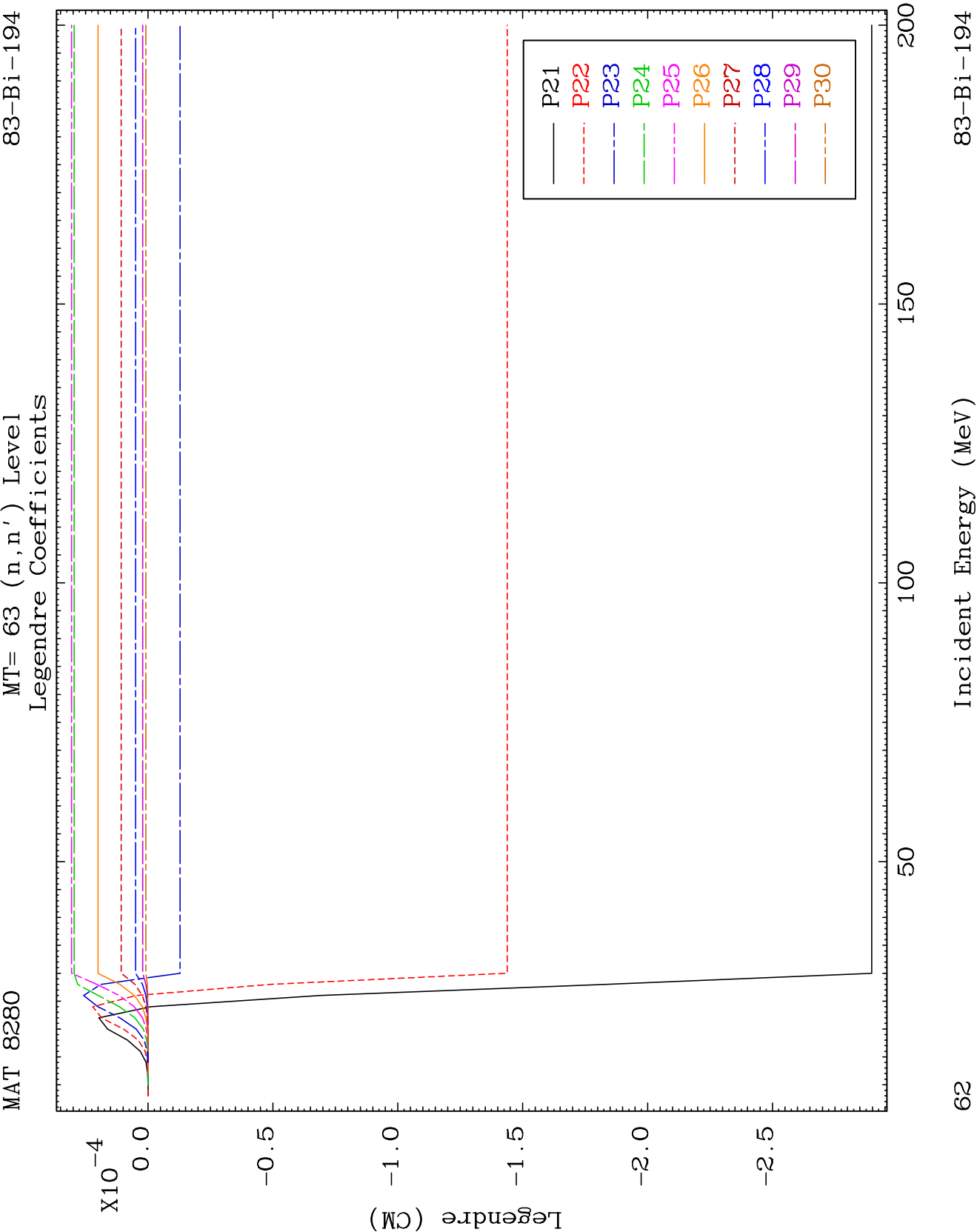
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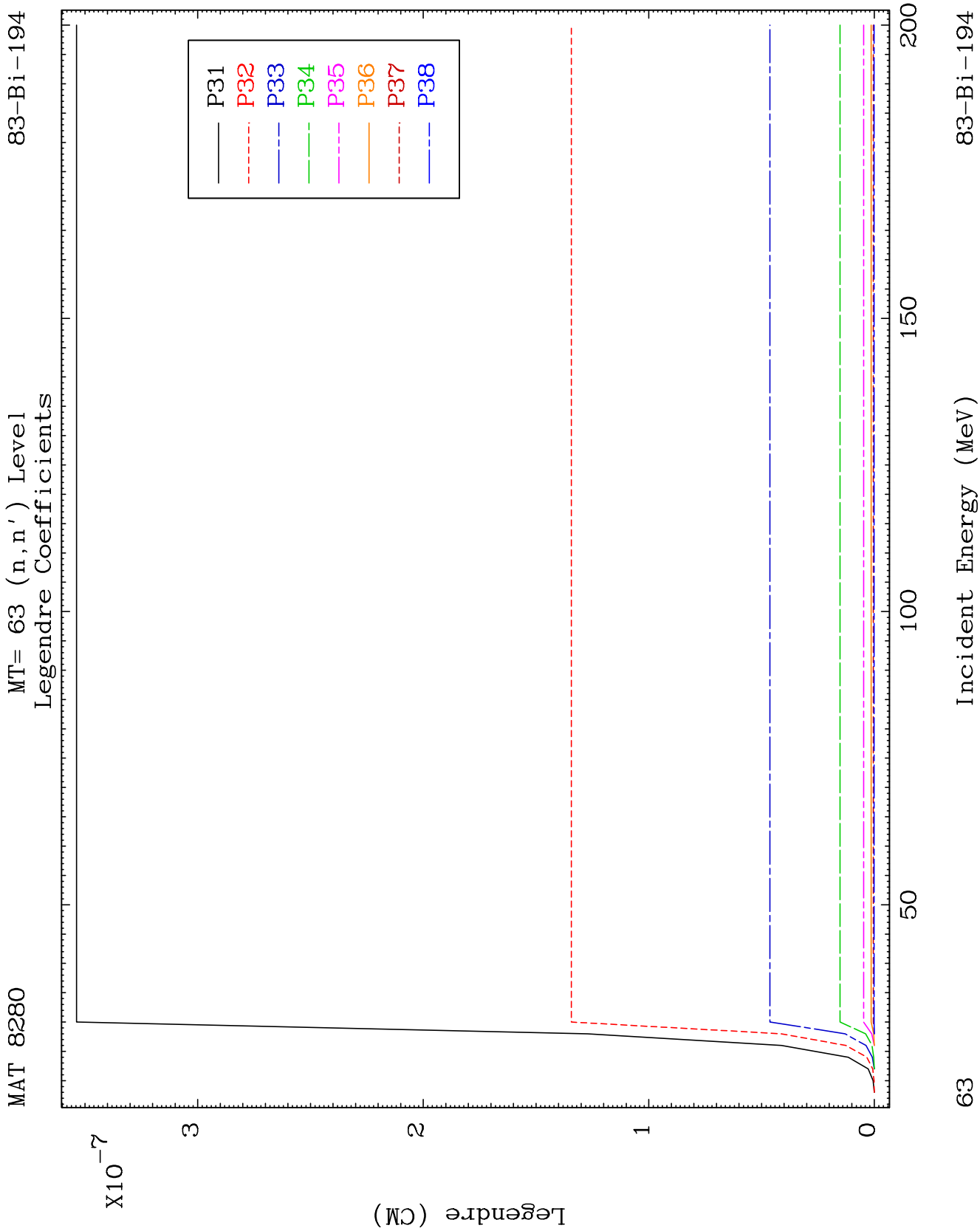
MT= 63 (n, n') Level
Legendre Coefficients

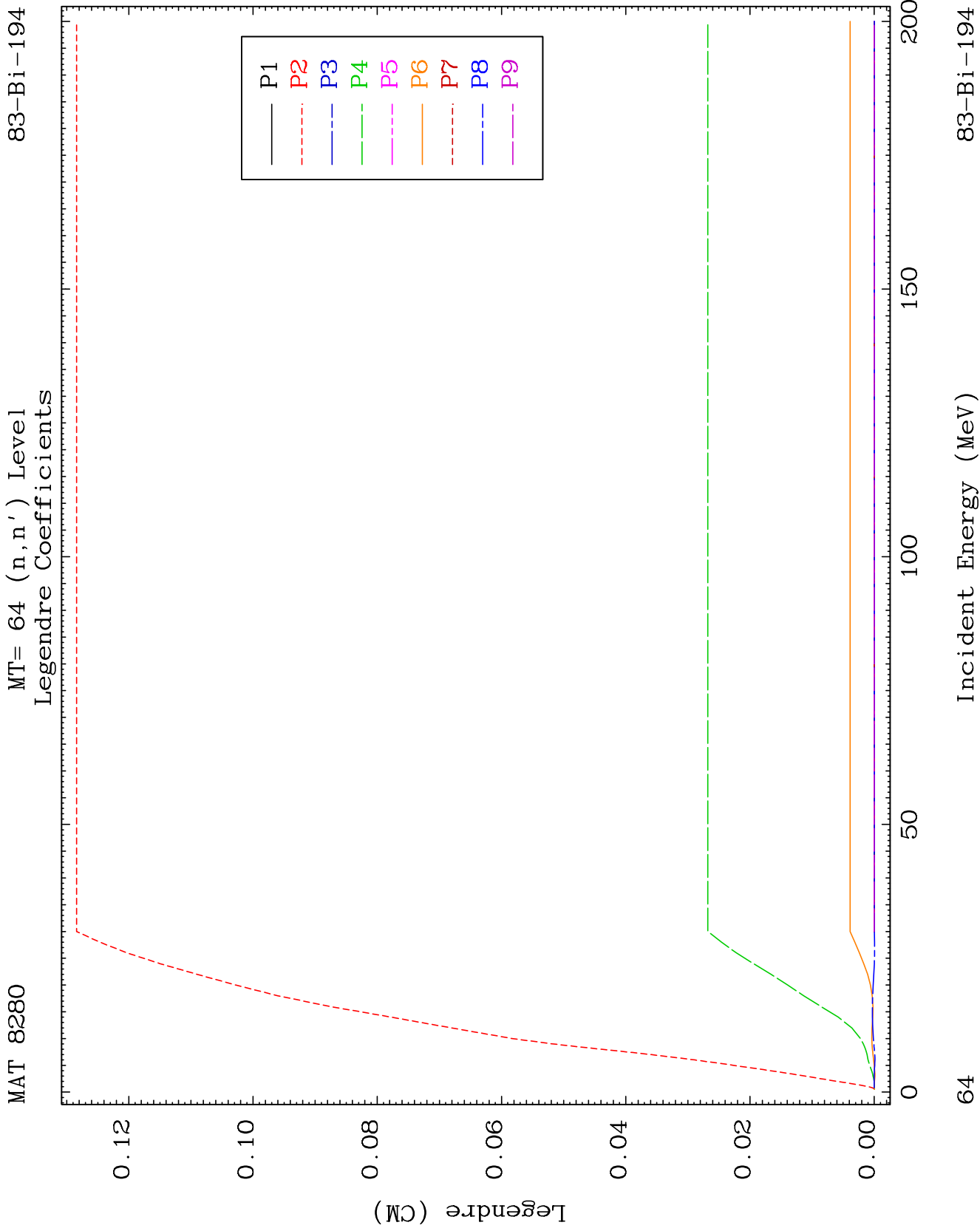
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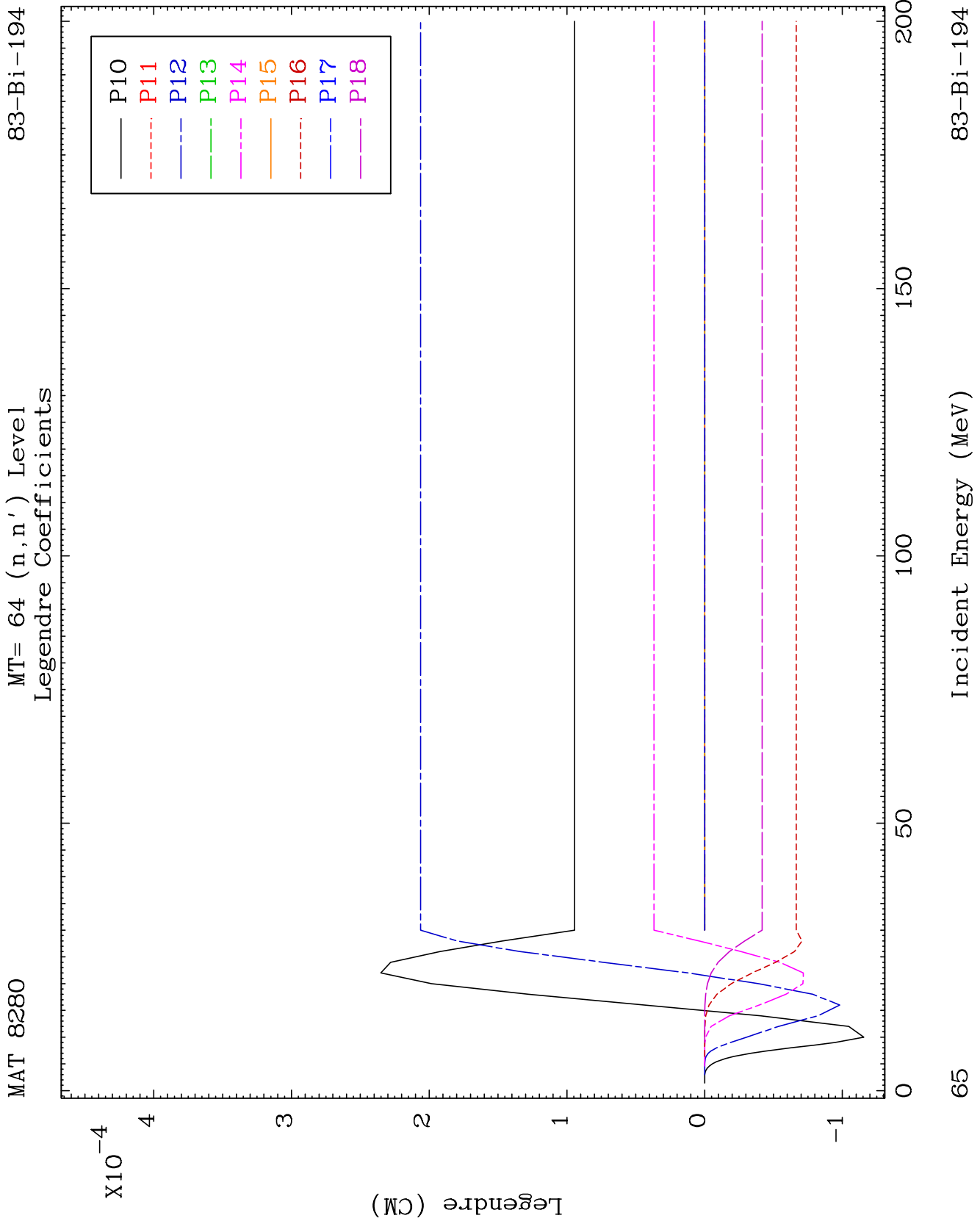


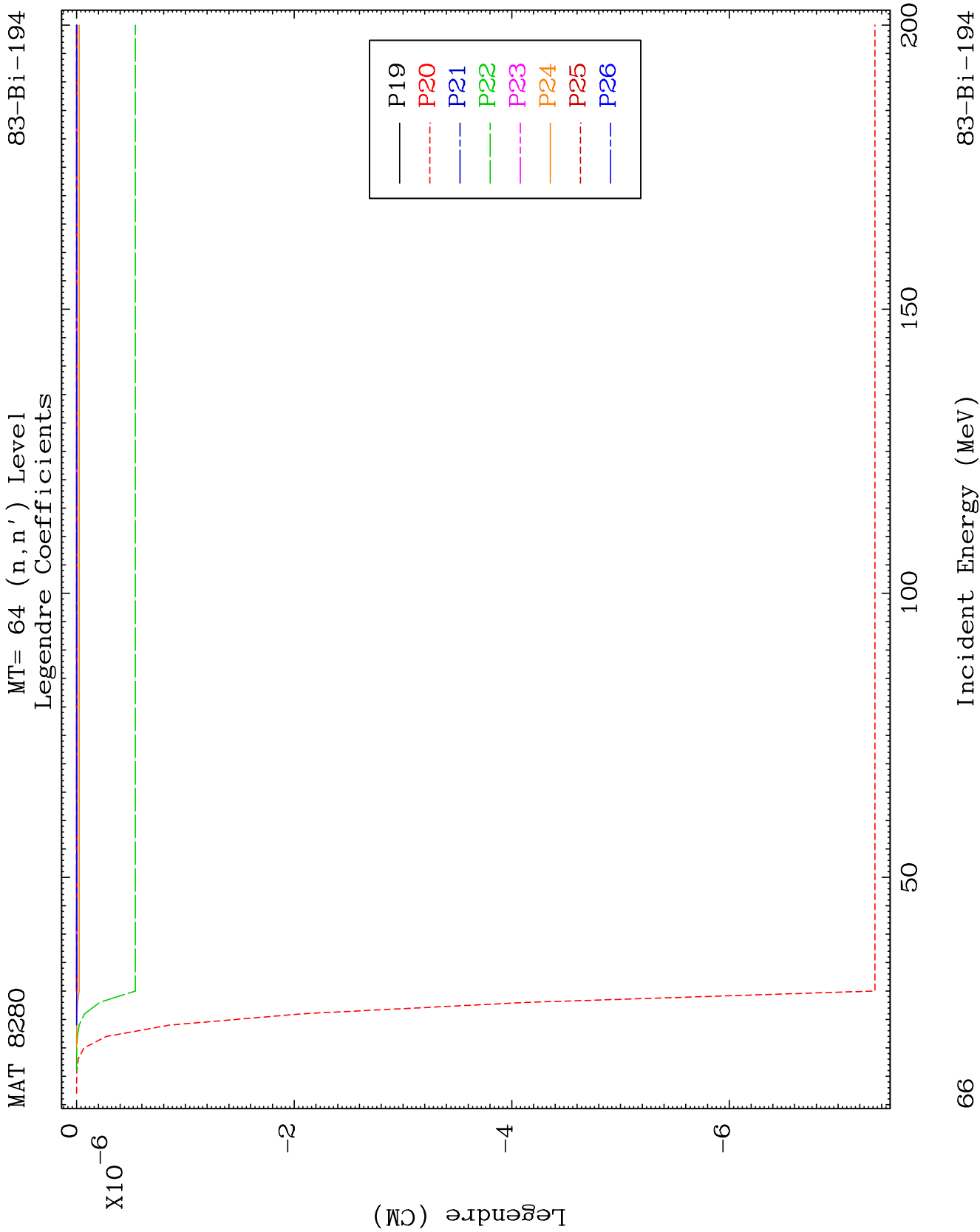








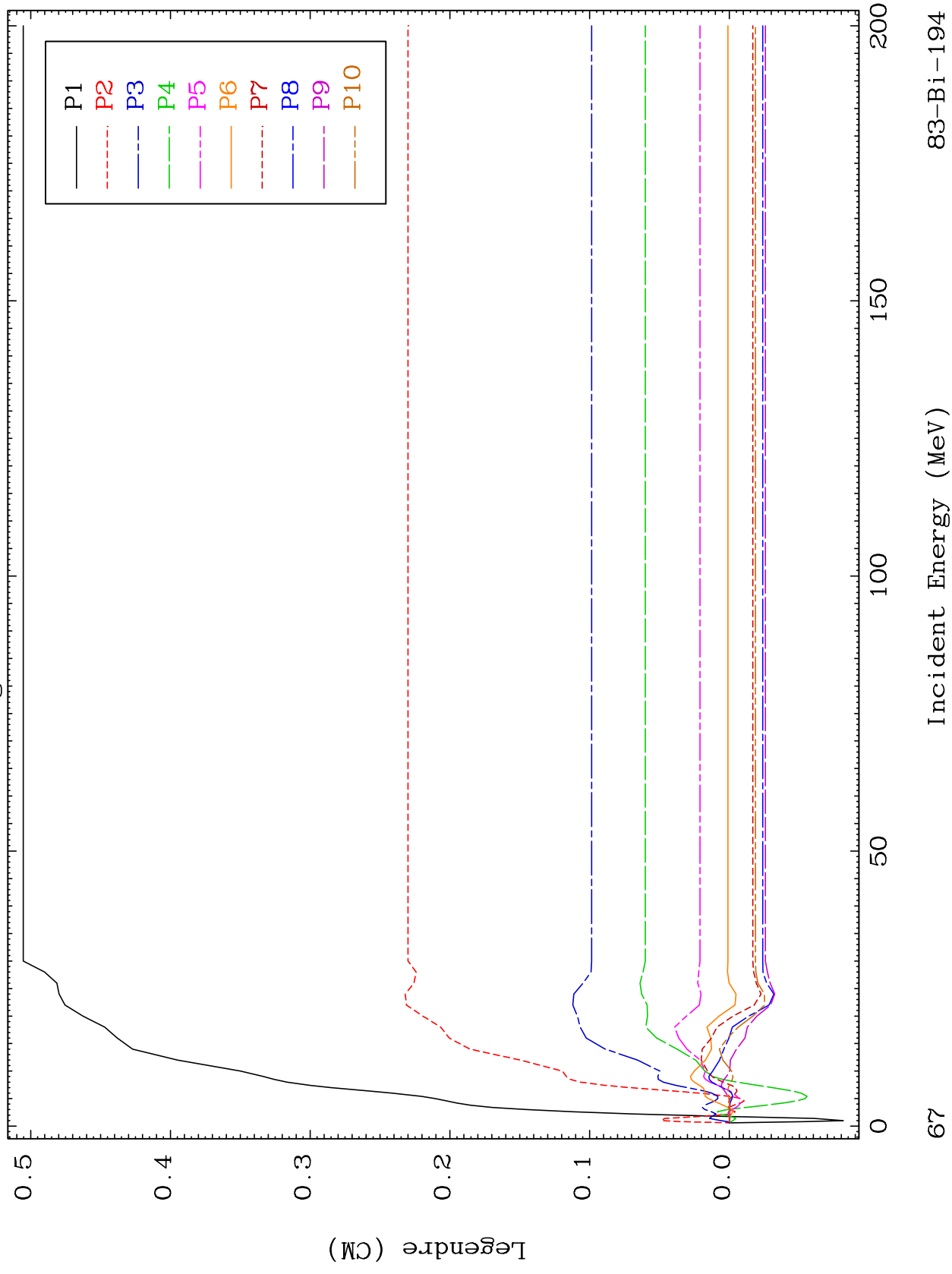




MAT 8280

MT= 65 (n, n') Level
Legendre Coefficients

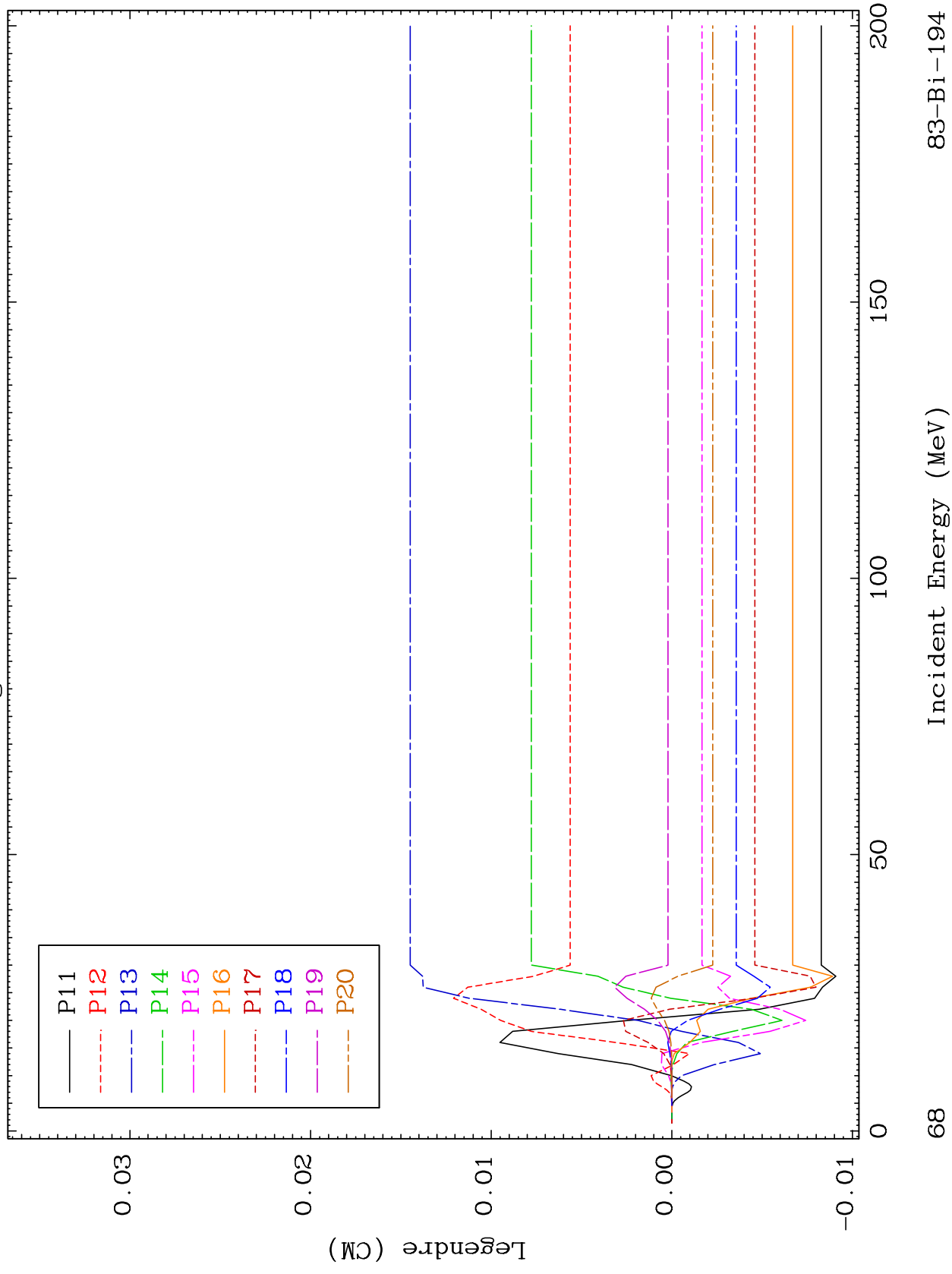
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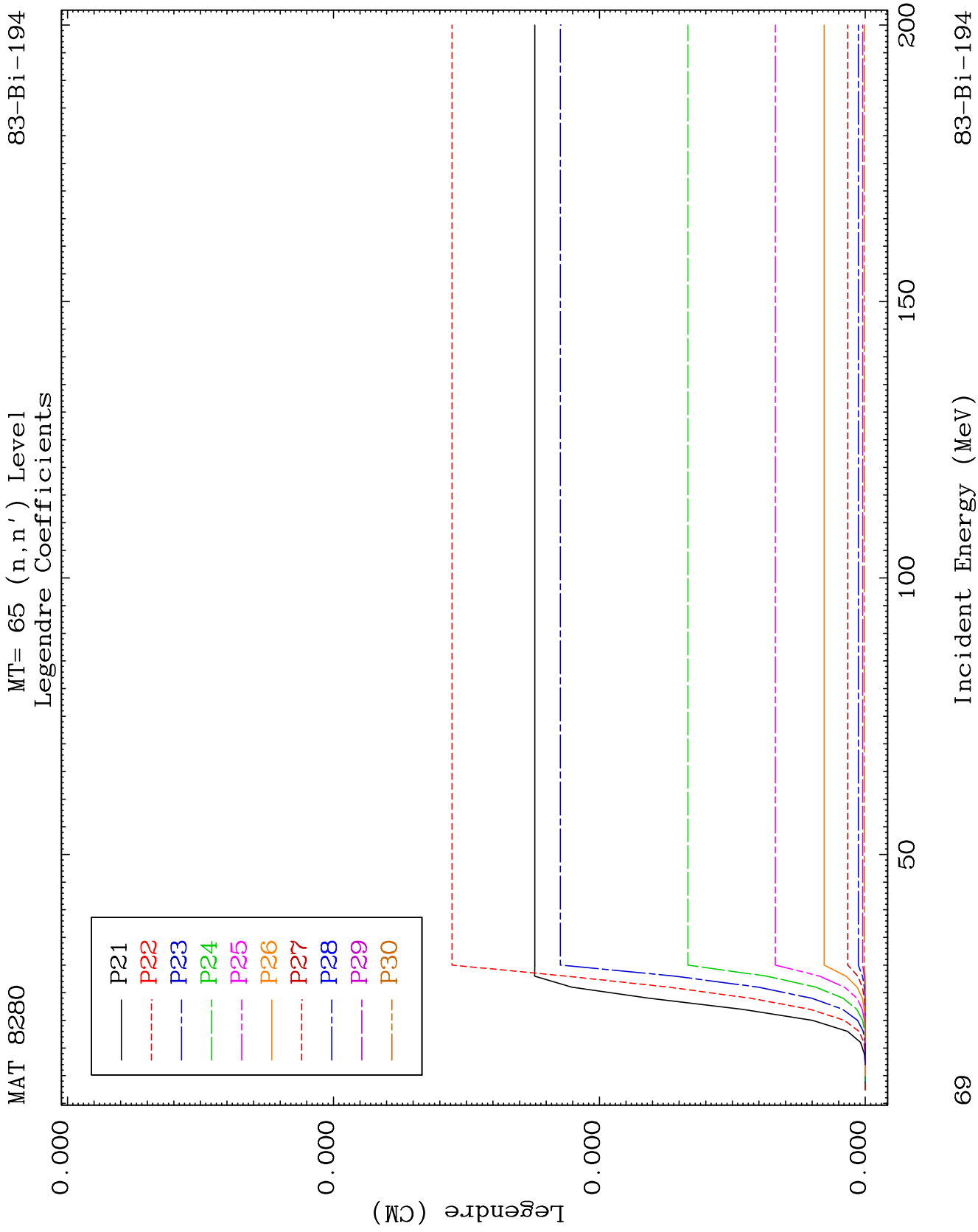


MAT 8280

MT= 65 (n, n') Level
Legendre Coefficients

83-Bi-194

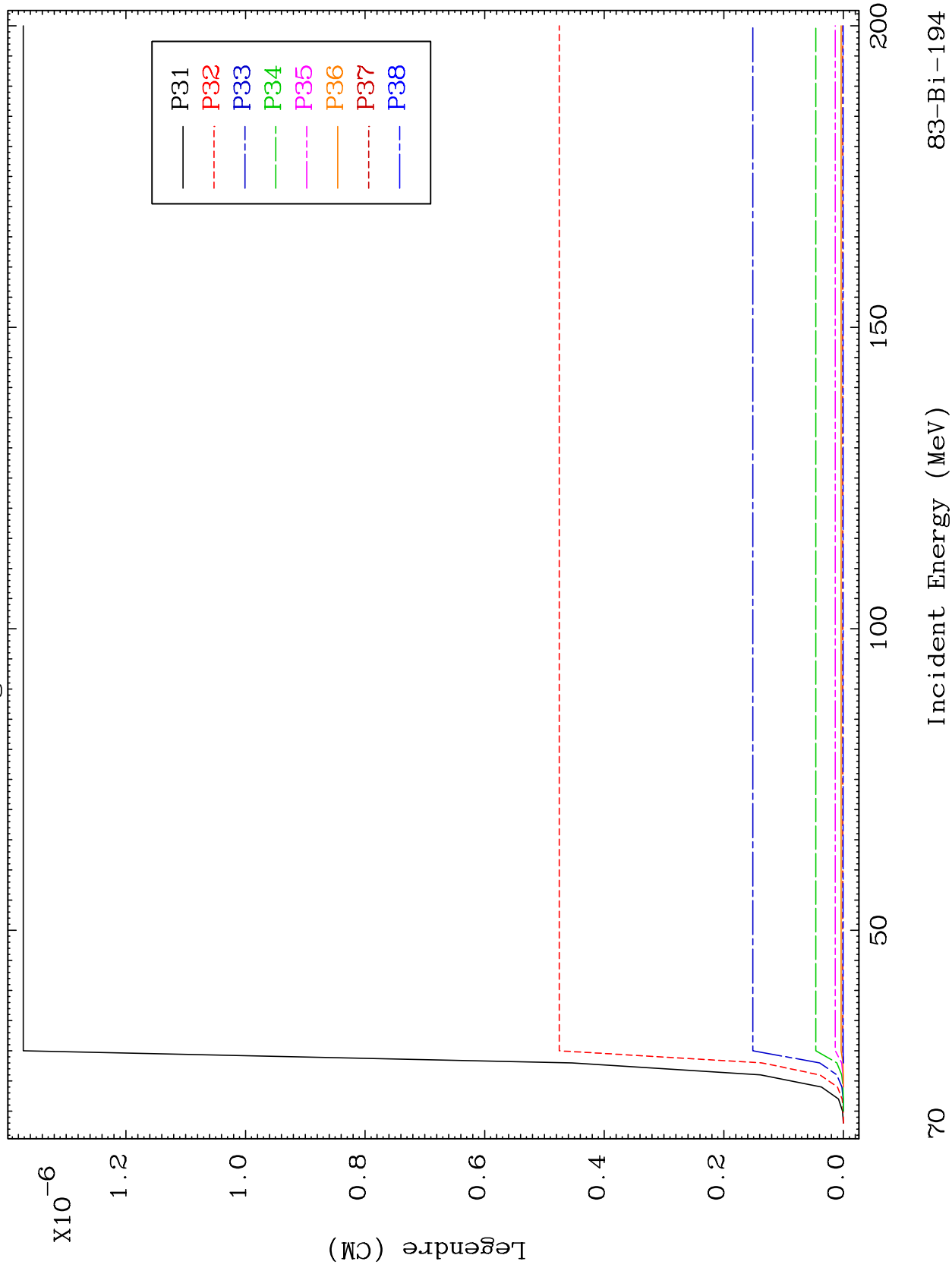


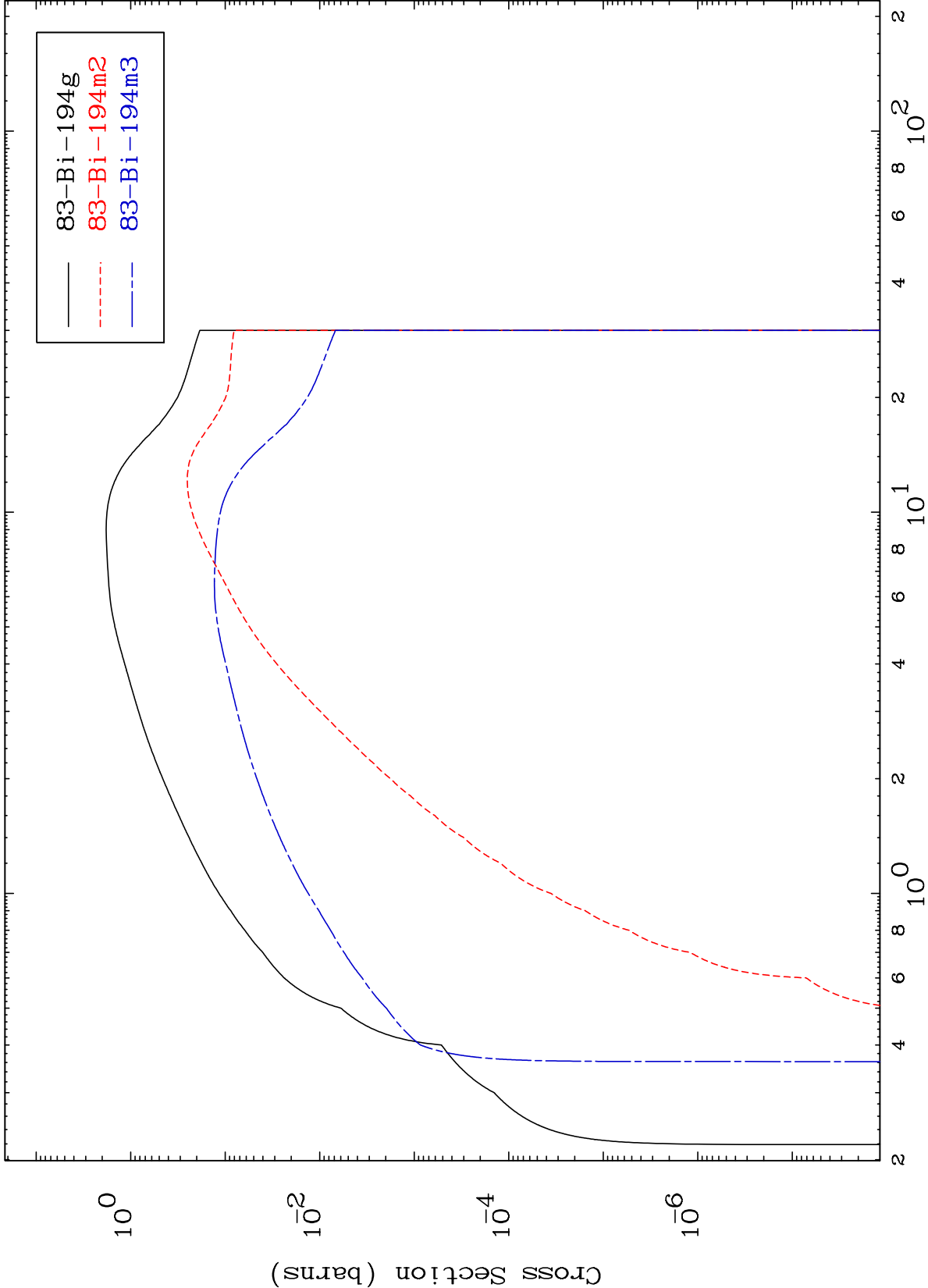


MAT 8280

MT= 65 (n, n') Level
Legendre Coefficients

83-Bi-194

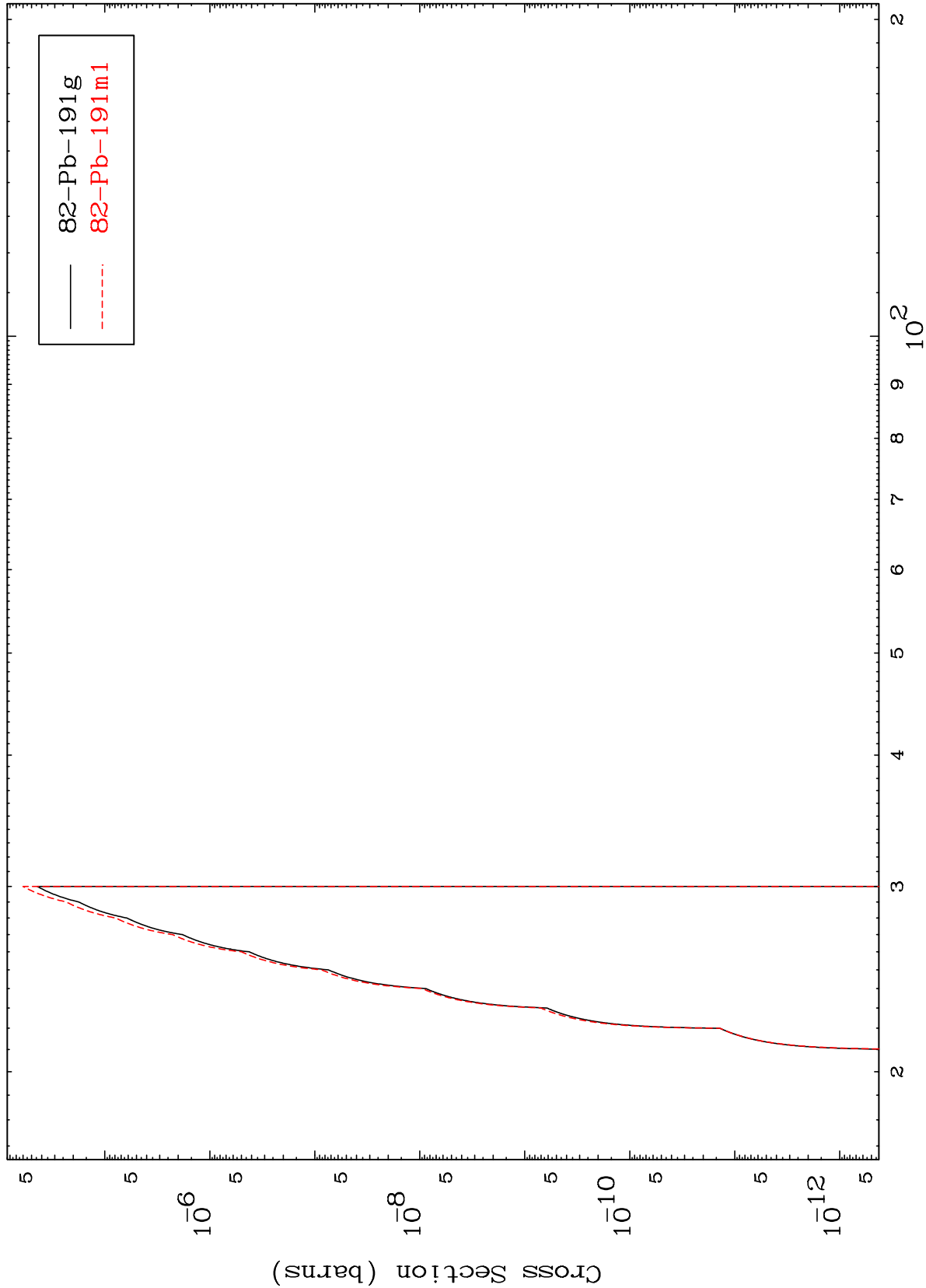




MAT 8280

83-Bi-194

(n,2n) d
Radionuclide Production Cross Section

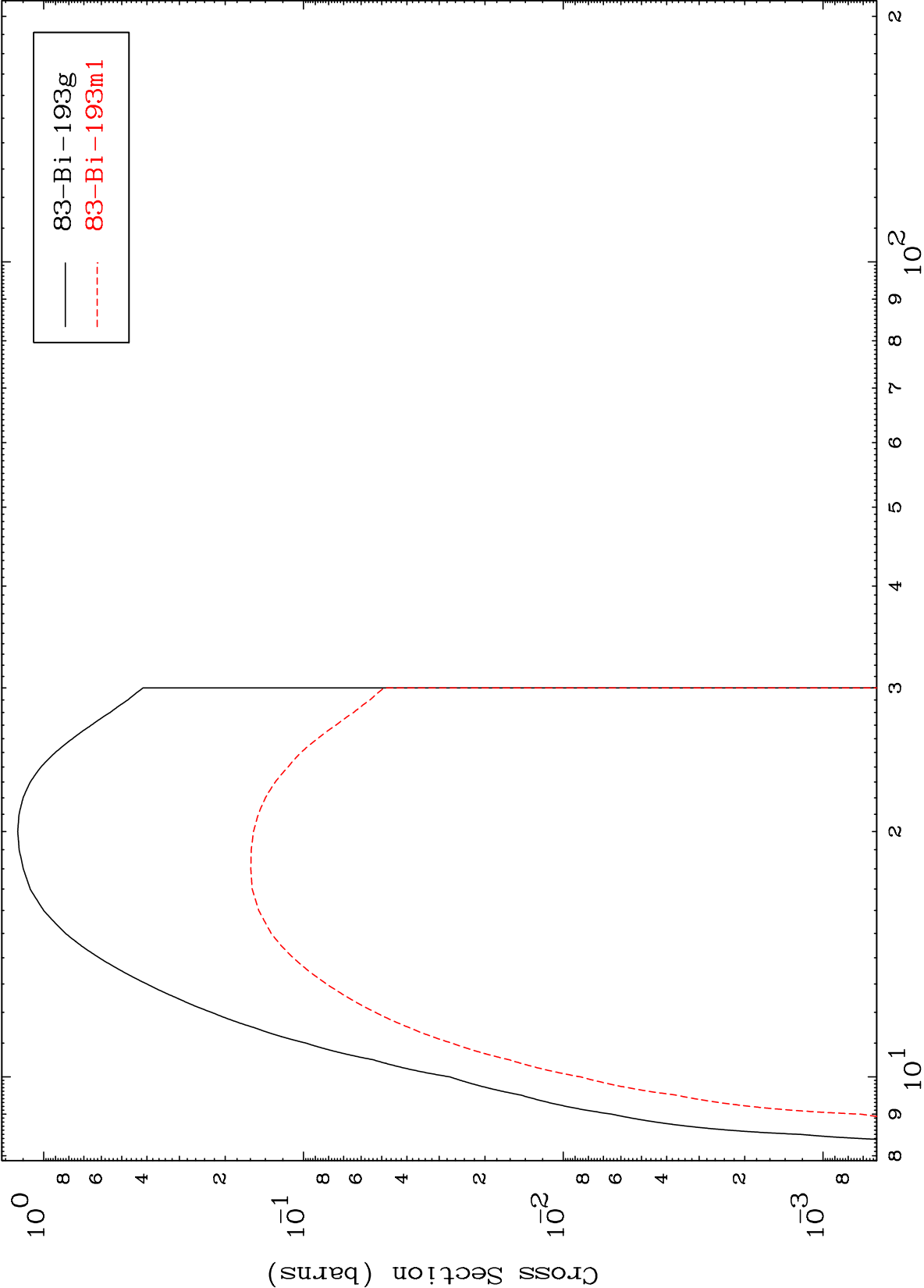


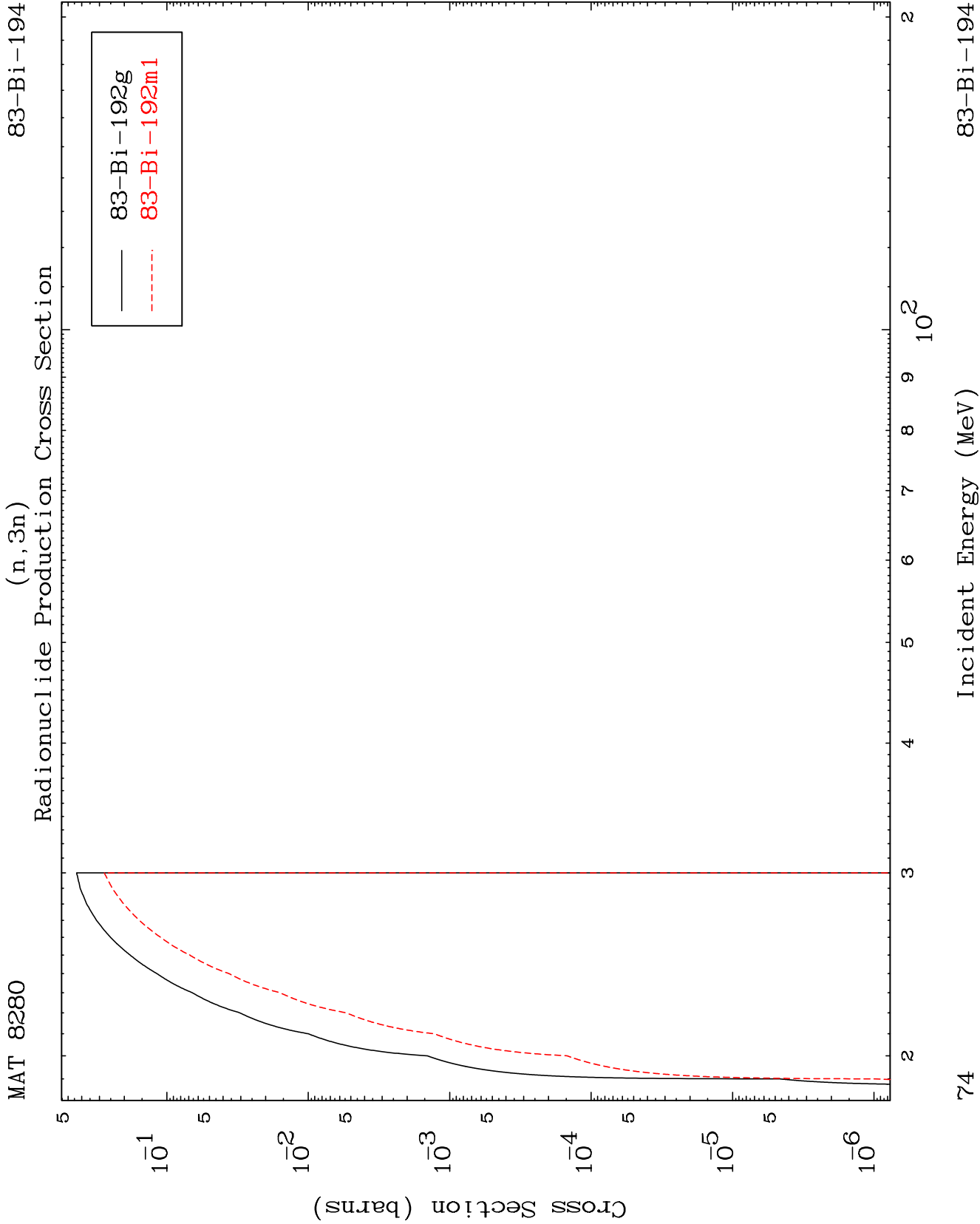
72

Incident Energy (MeV)

83-Bi-194

(n,2n)
Radionuclide Production Cross Section



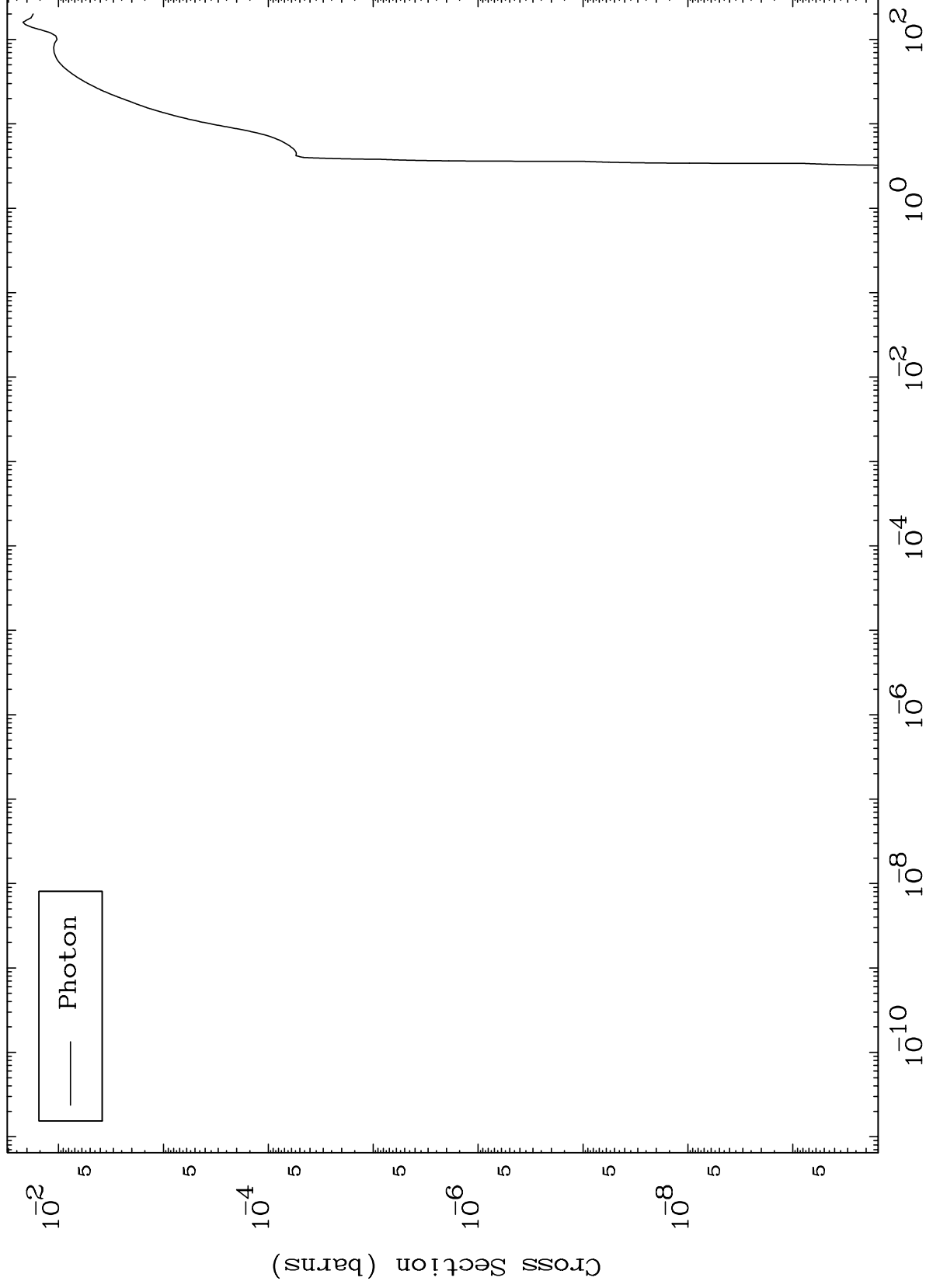


MAT 8280

Fission

83-Bi-194

Radionuclide Production Cross Section



75

Incident Energy (MeV)

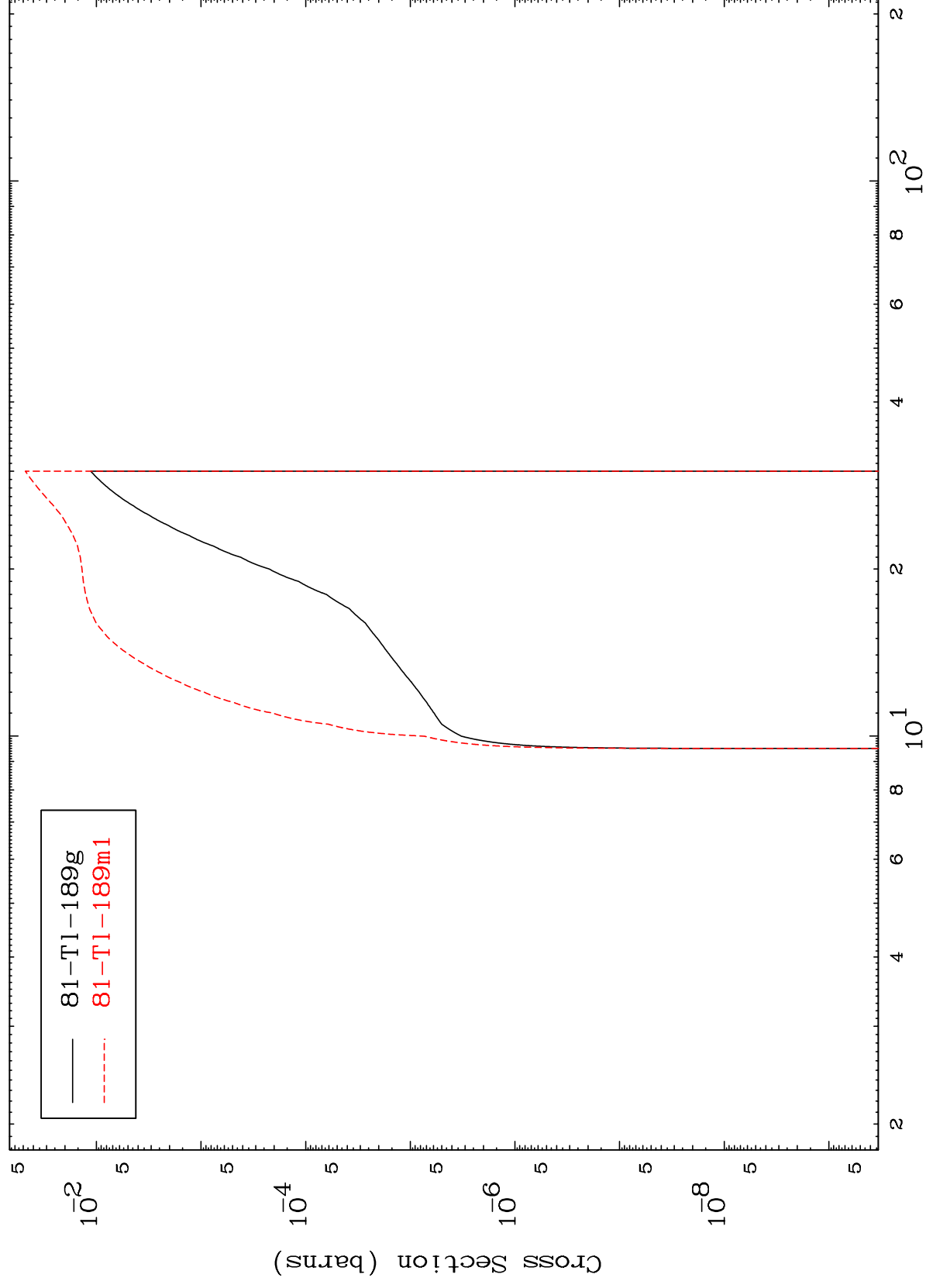
83-Bi-194

MAT 8280

$$(n, 2n) \propto$$

83-Bi-194

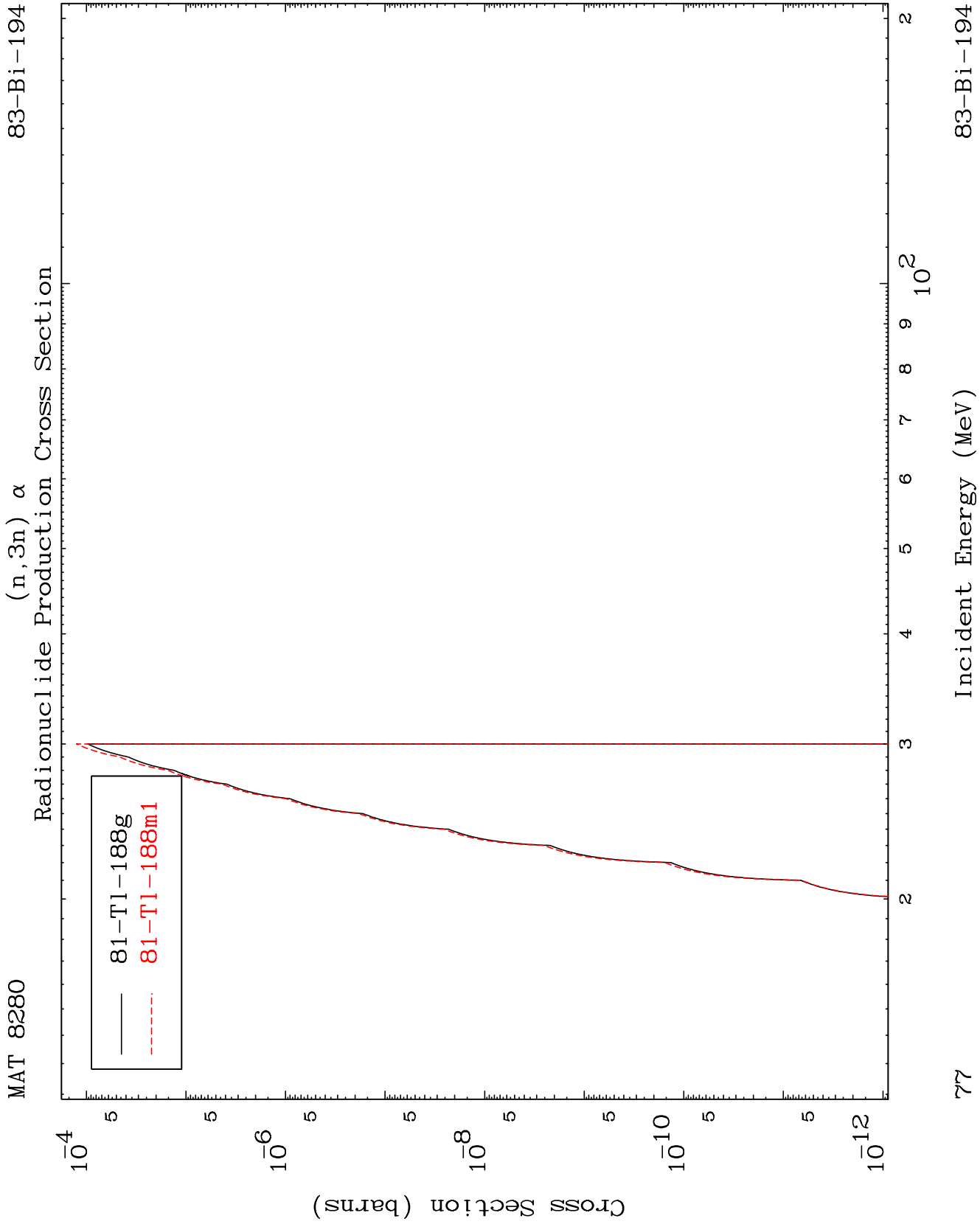
Radionuclide Production Cross Section



92

Incident Energy (MeV)

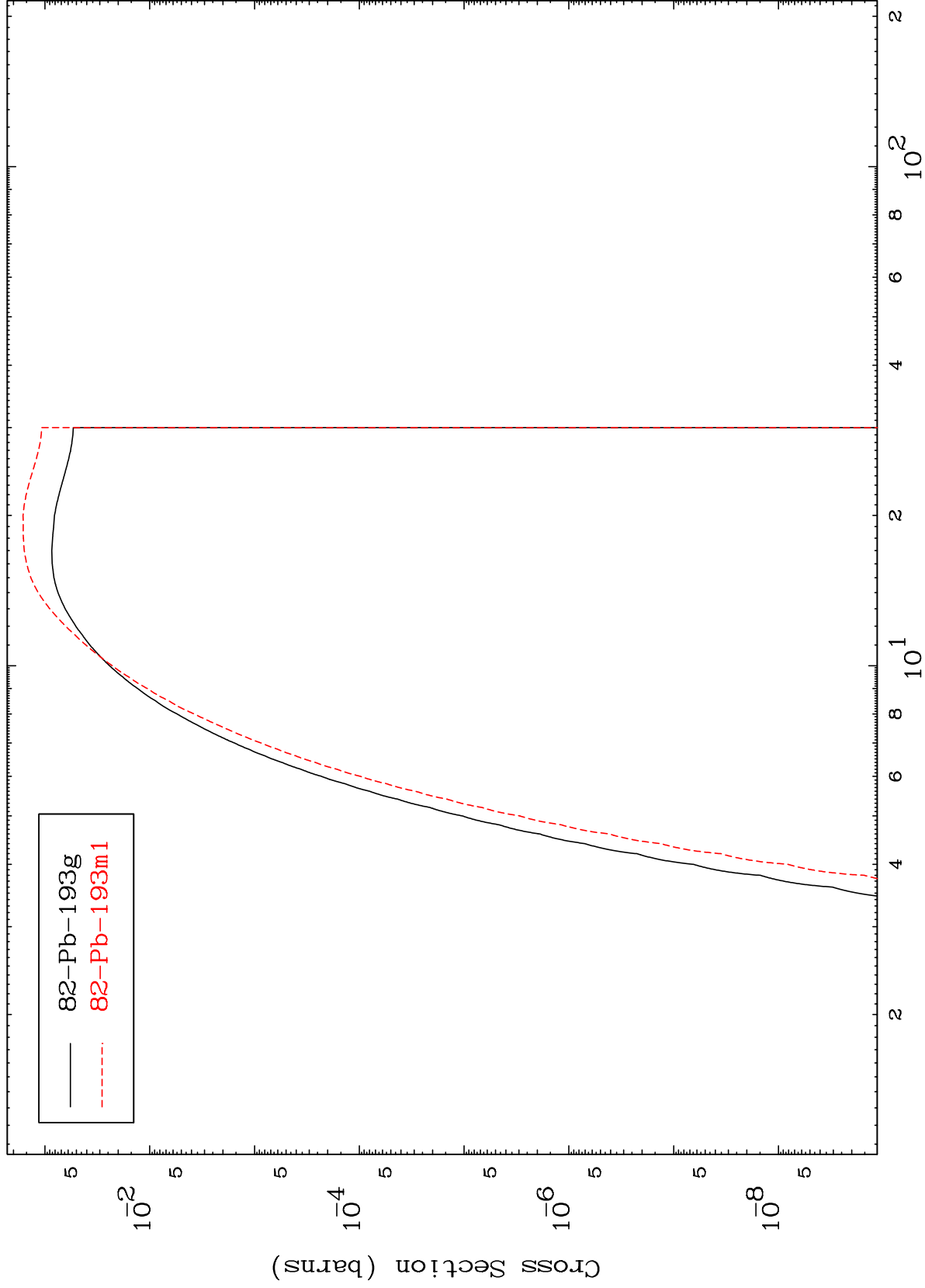
83-Bi-194



MAT 8280

83-Bi-194

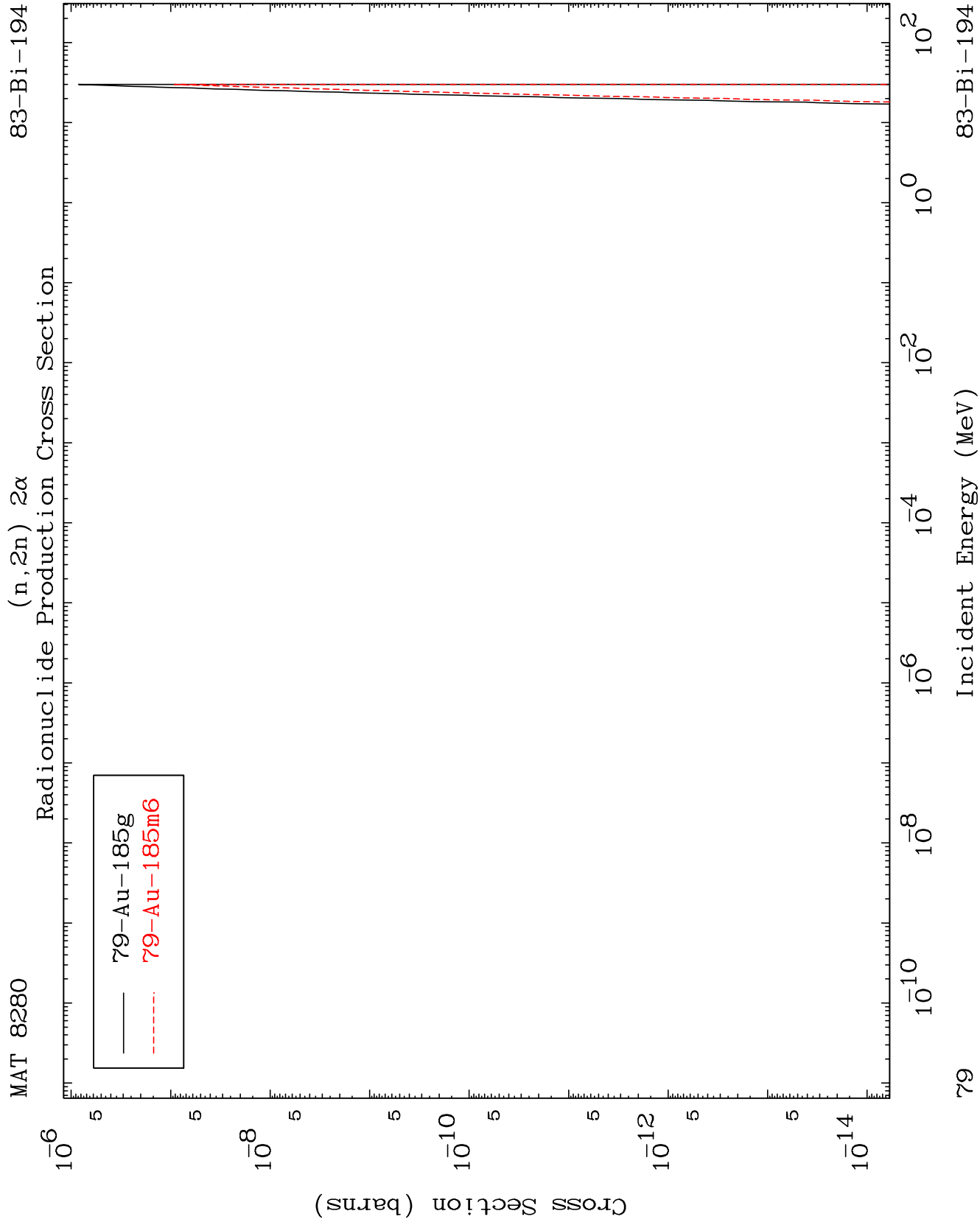
(n,n') p
Radionuclide Production Cross Section



78

Incident Energy (MeV)

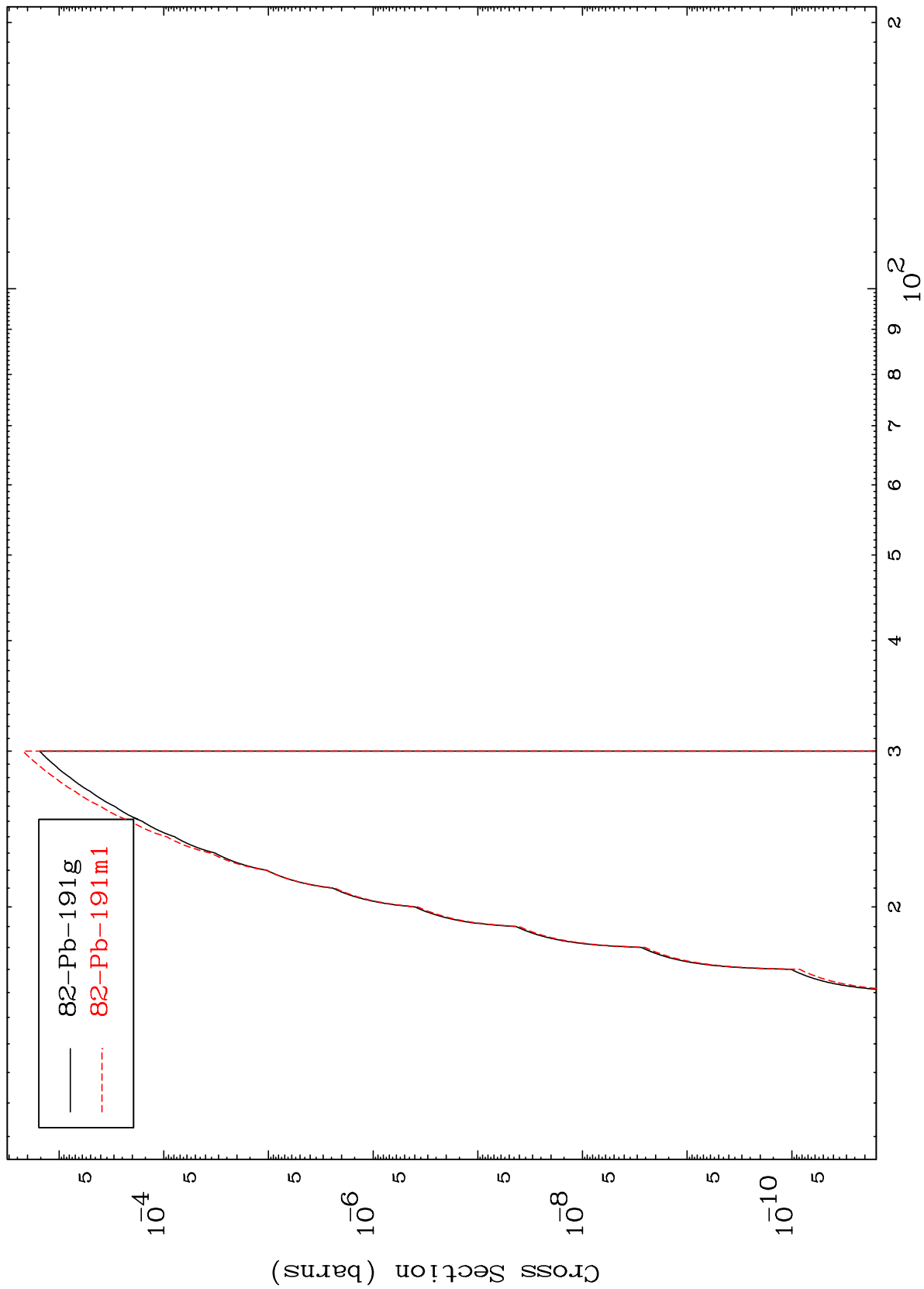
83-Bi-194



MAT 8280

83-Bi-194

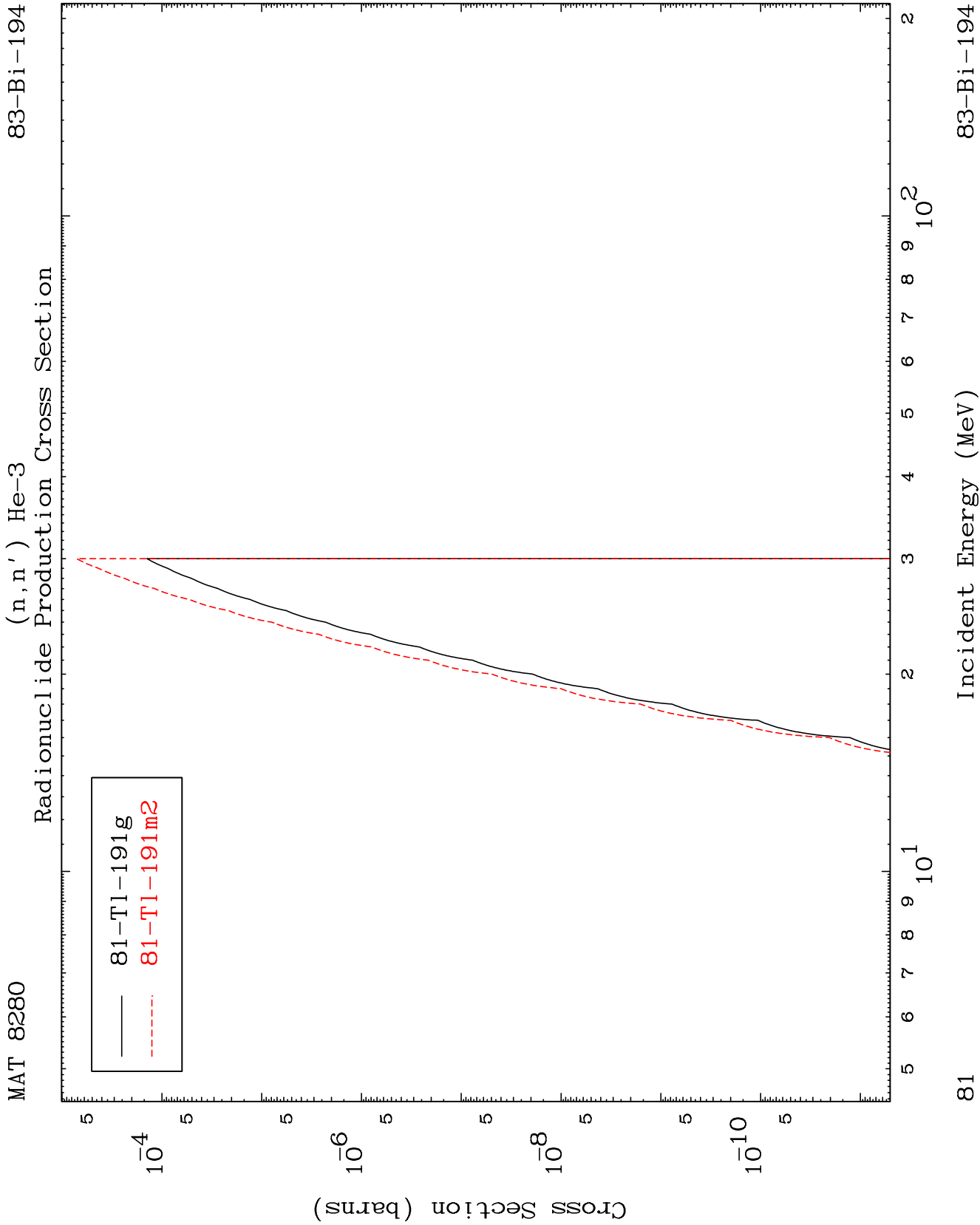
Radionuclide Production Cross Section
(n,n') t



80

Incident Energy (MeV)

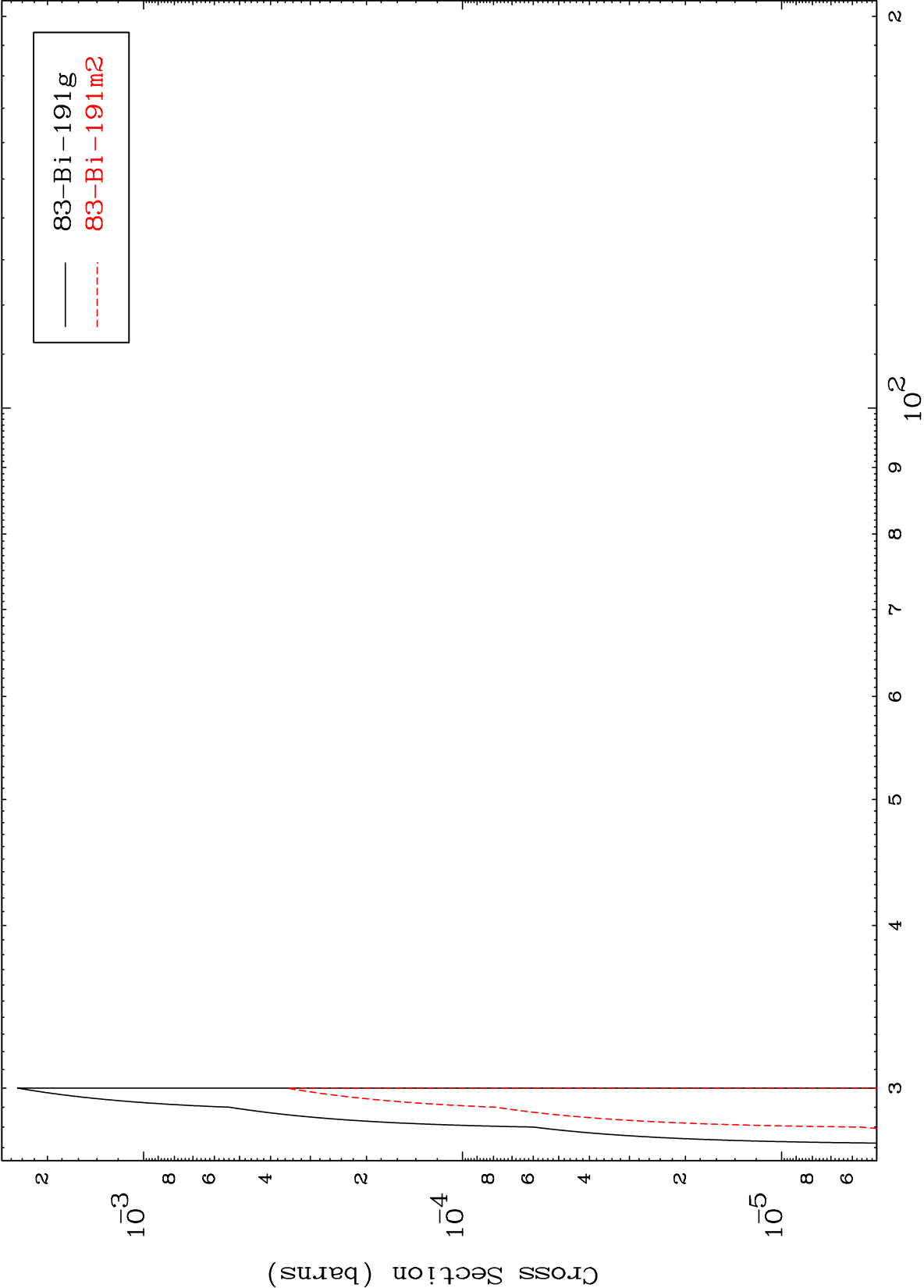
83-Bi-194



MAT 8280

83-Bi-194

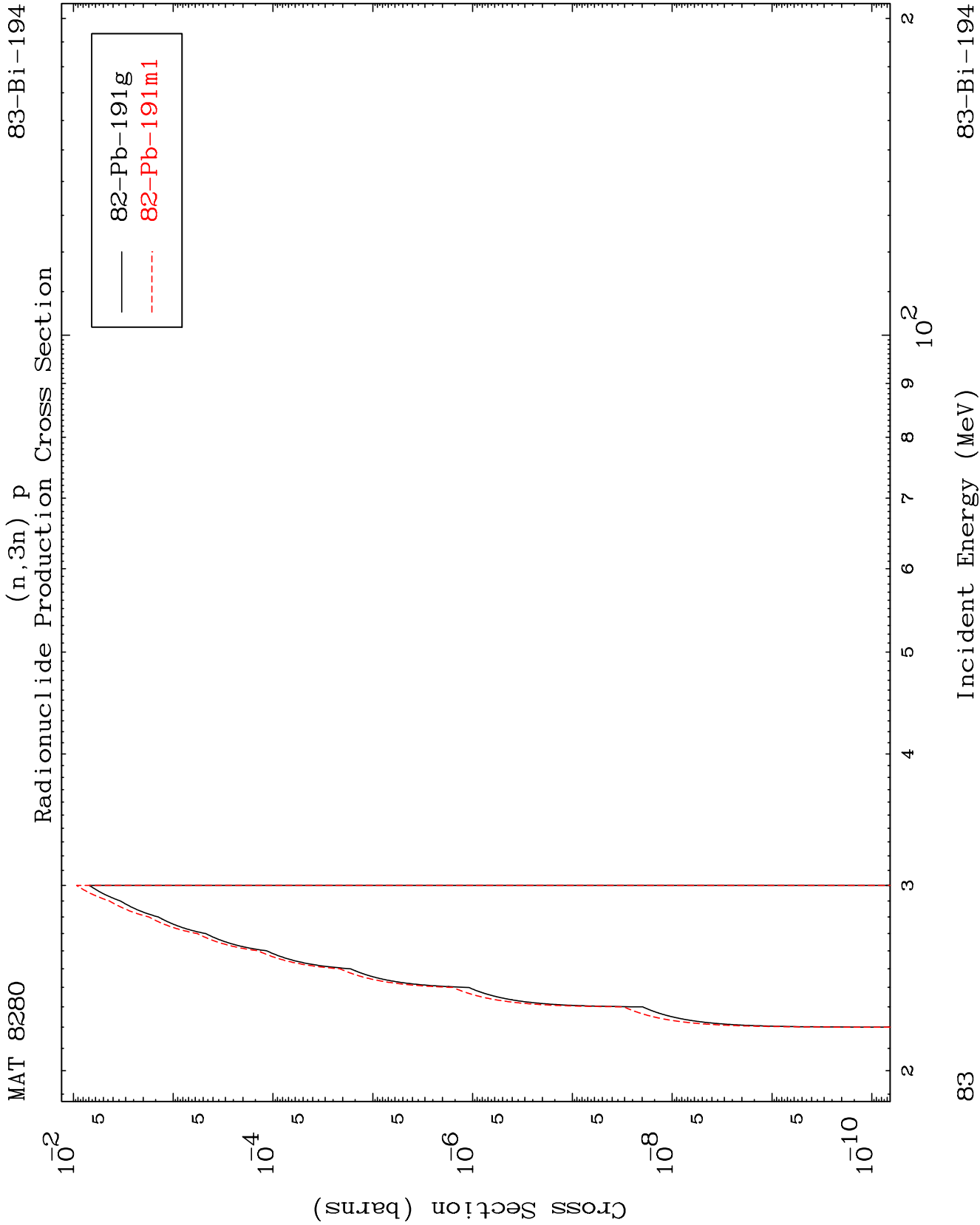
(n,4n)
Radionuclide Production Cross Section



82

Incident Energy (MeV)

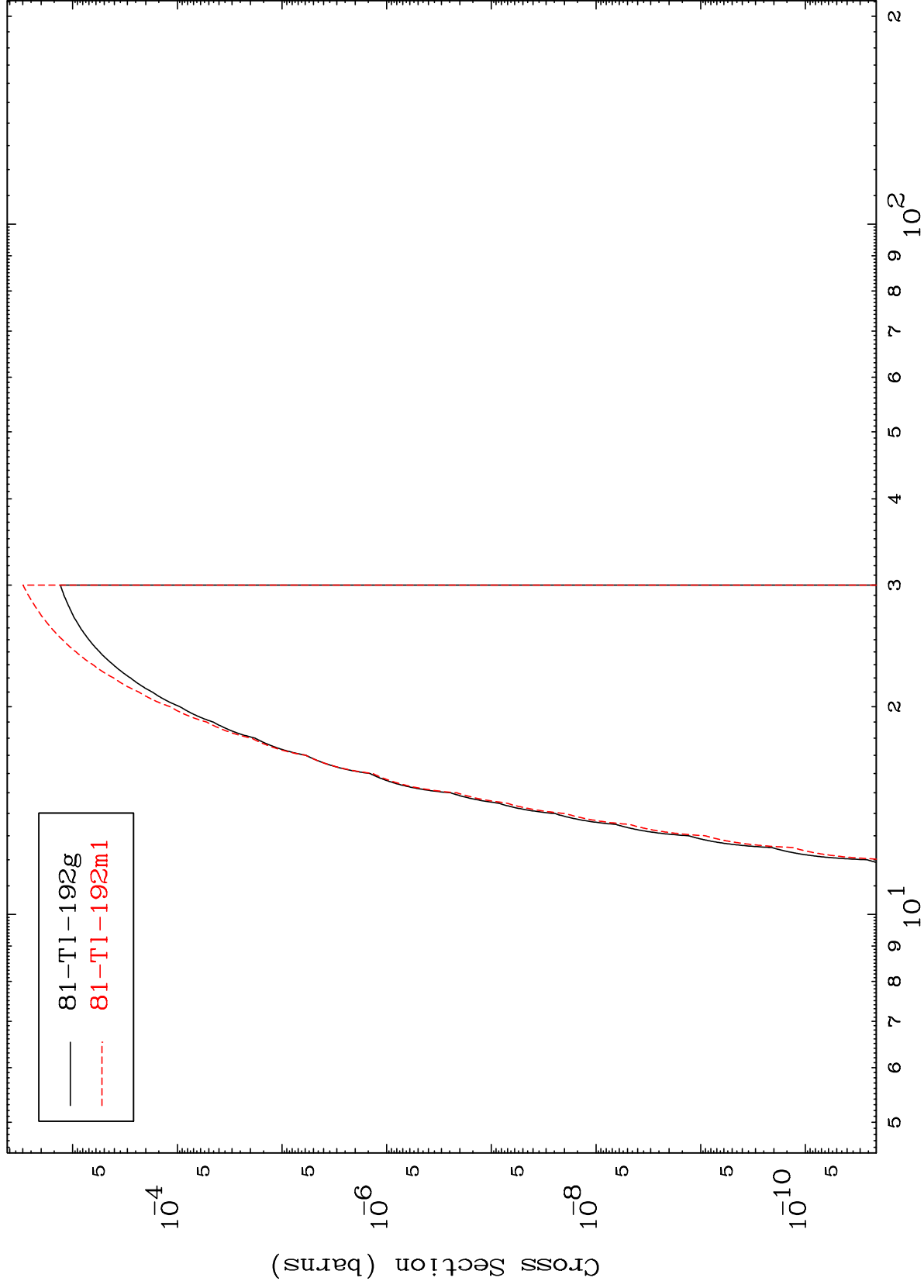
83-Bi-194



MAT 8280

83-Bi-194

(n,2n) p
Radionuclide Production Cross Section



84

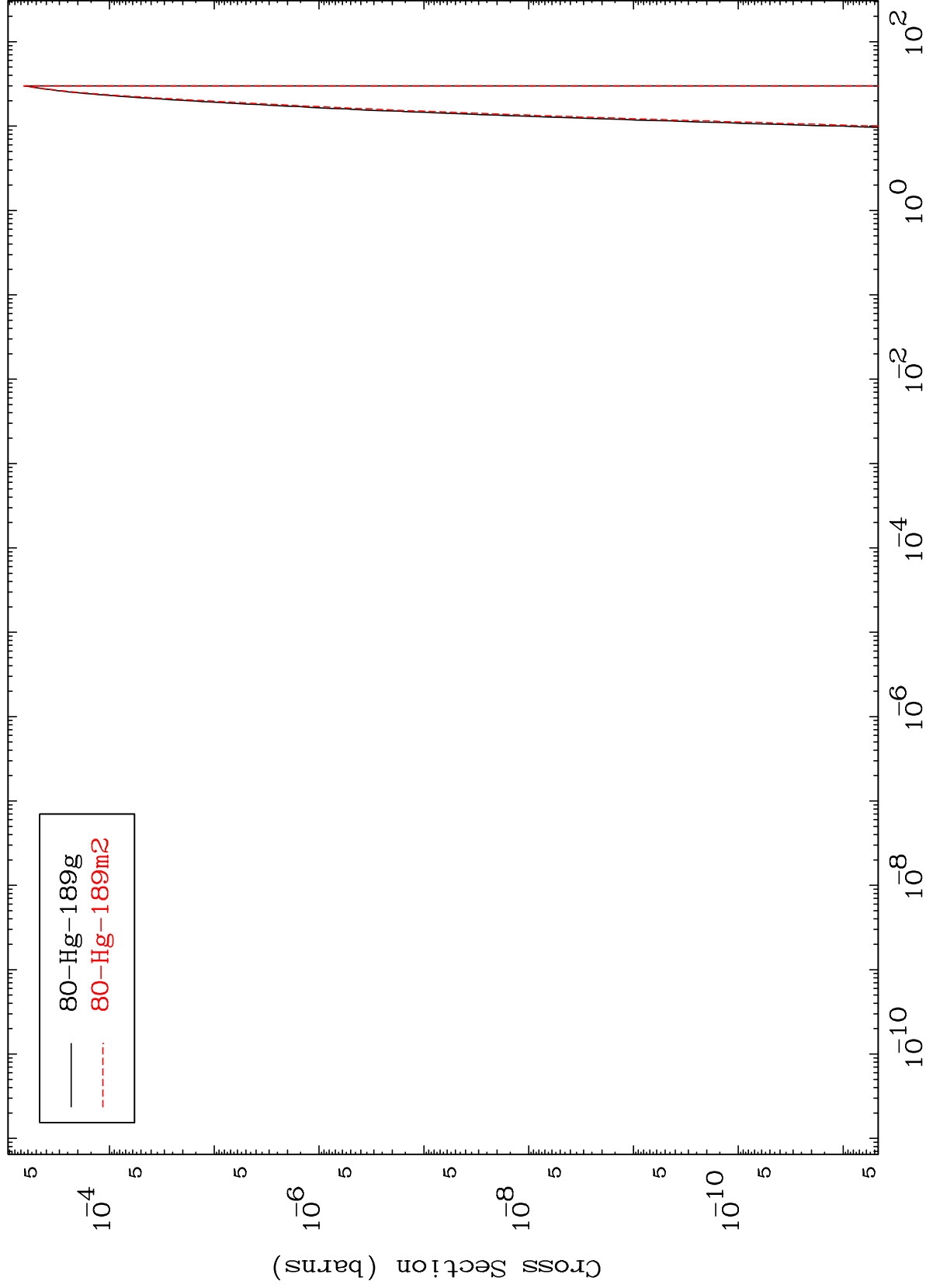
Incident Energy (MeV)

83-Bi-194

MAT 8280

83-Bi-194

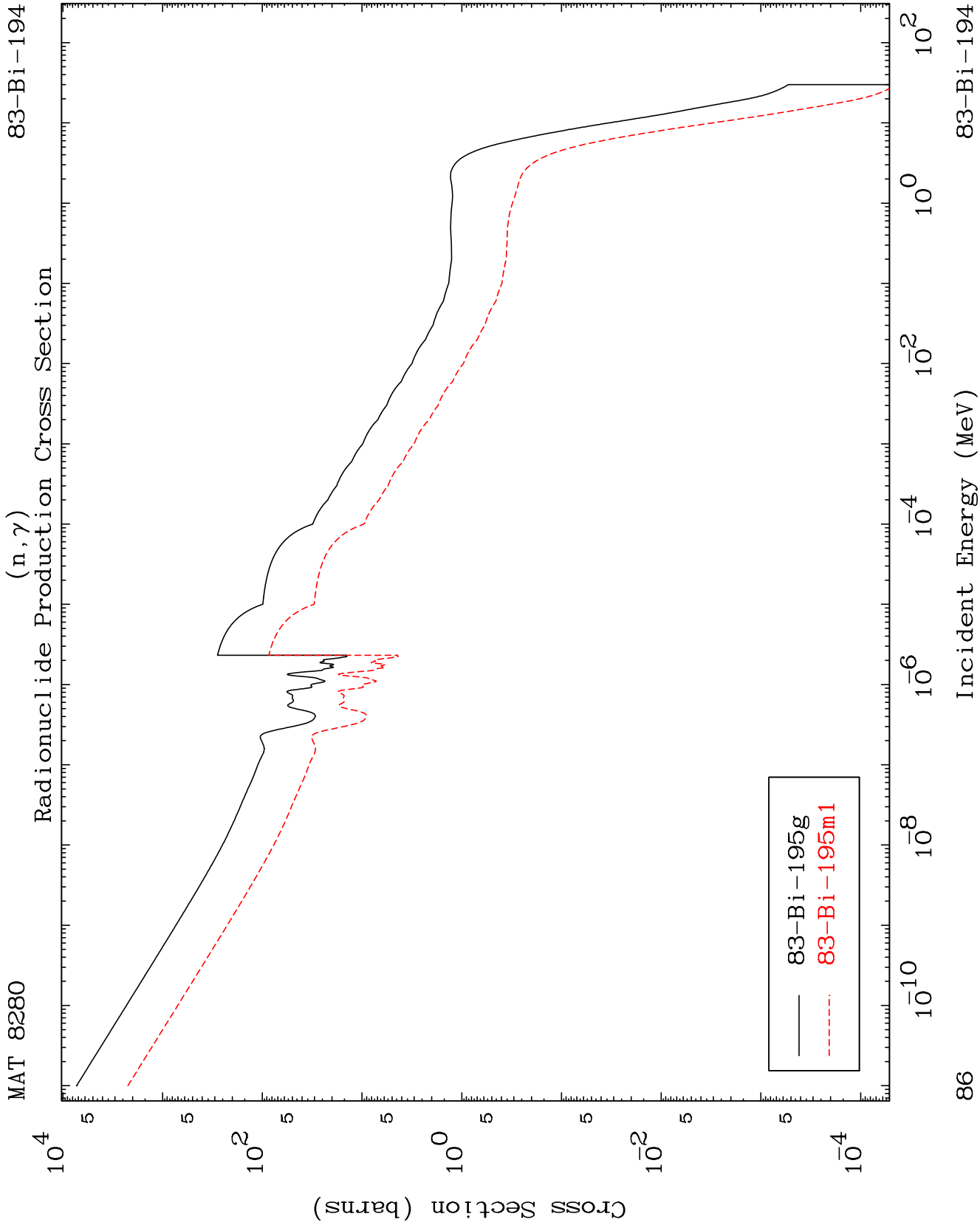
(n,n') p α
Radionuclide Production Cross Section



85

Incident Energy (MeV)

83-Bi-194

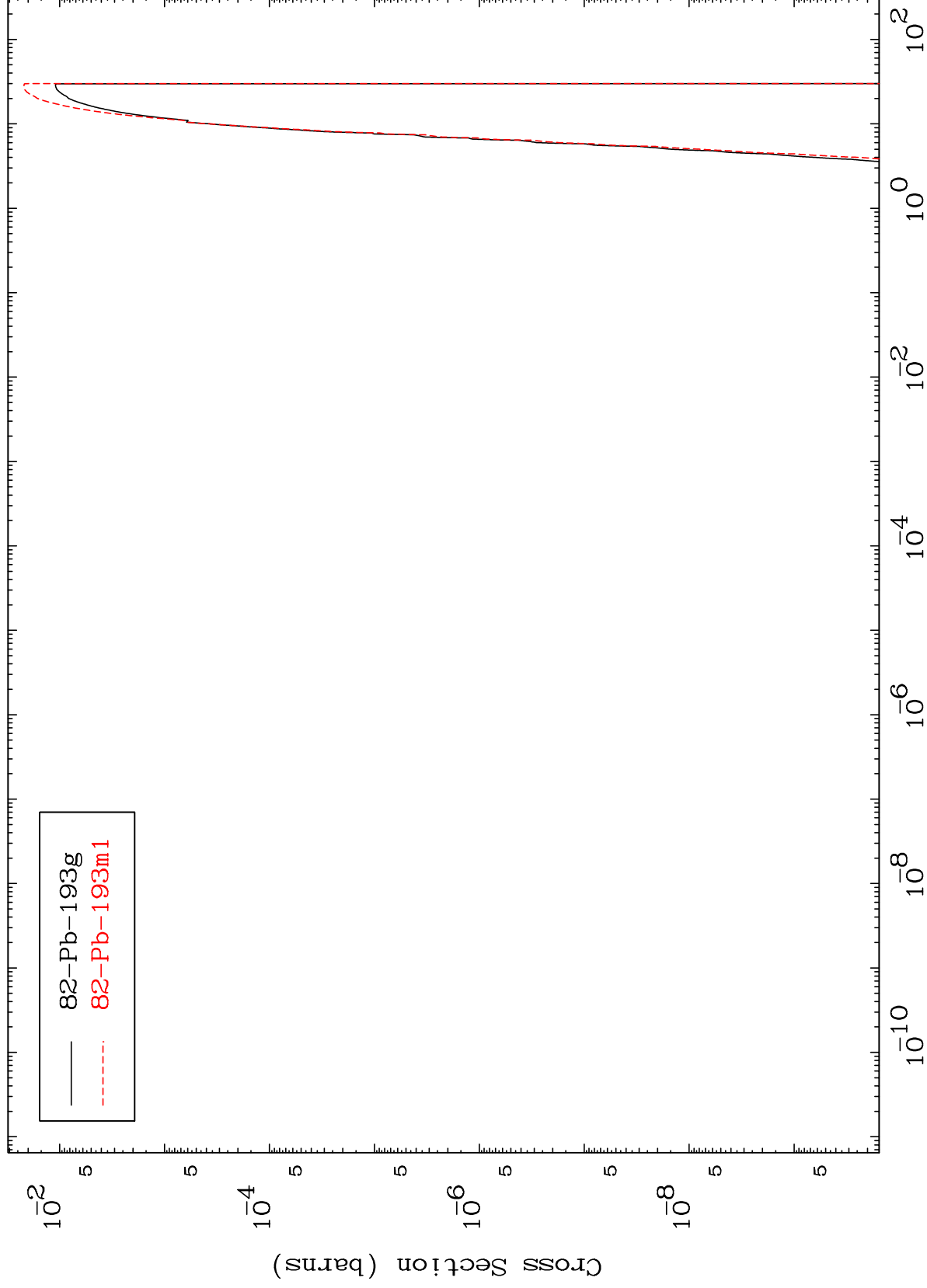
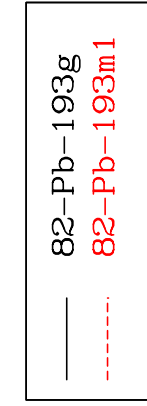


MAT 8280

(n,d)

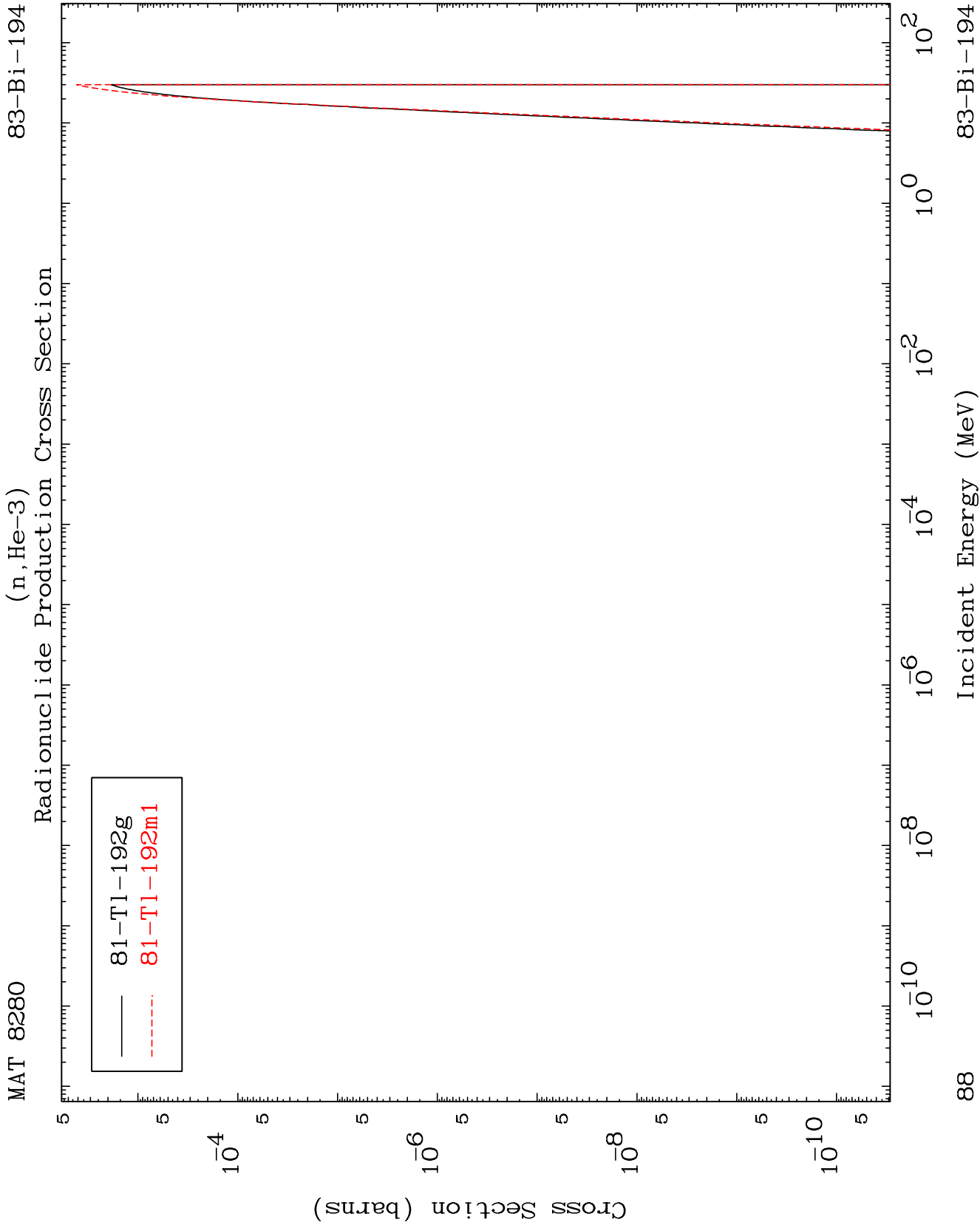
83-Bi-194

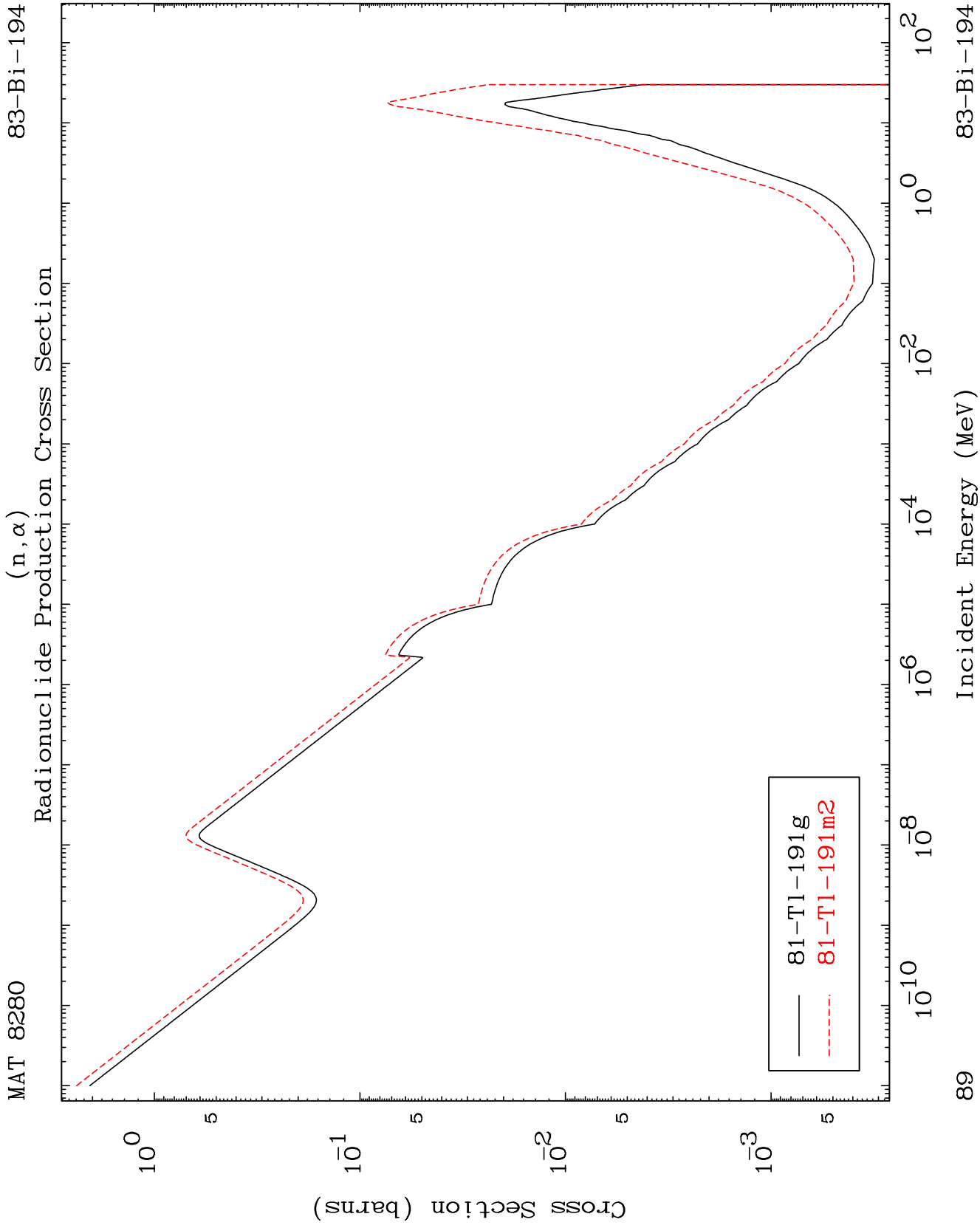
Radionuclide Production Cross Section

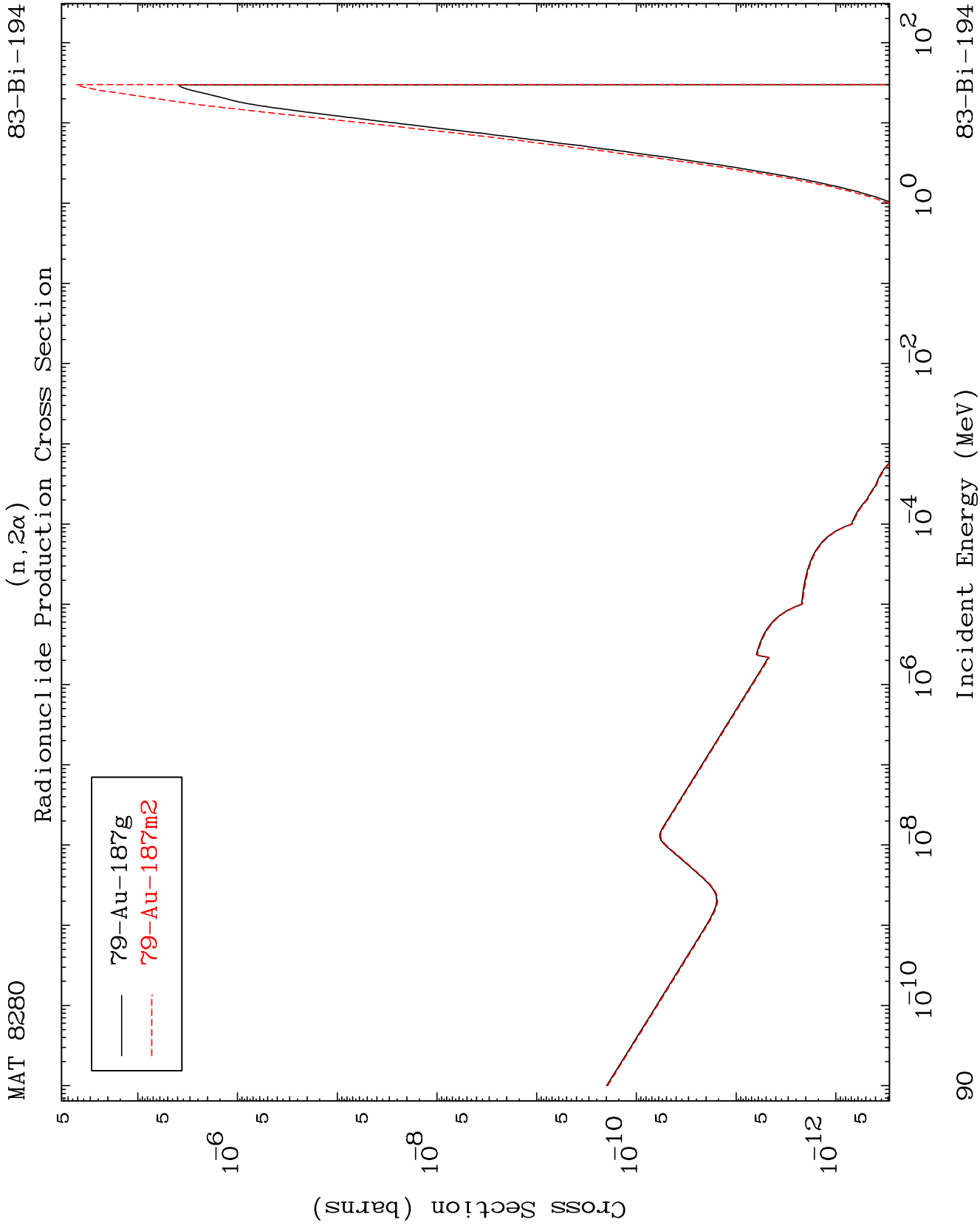


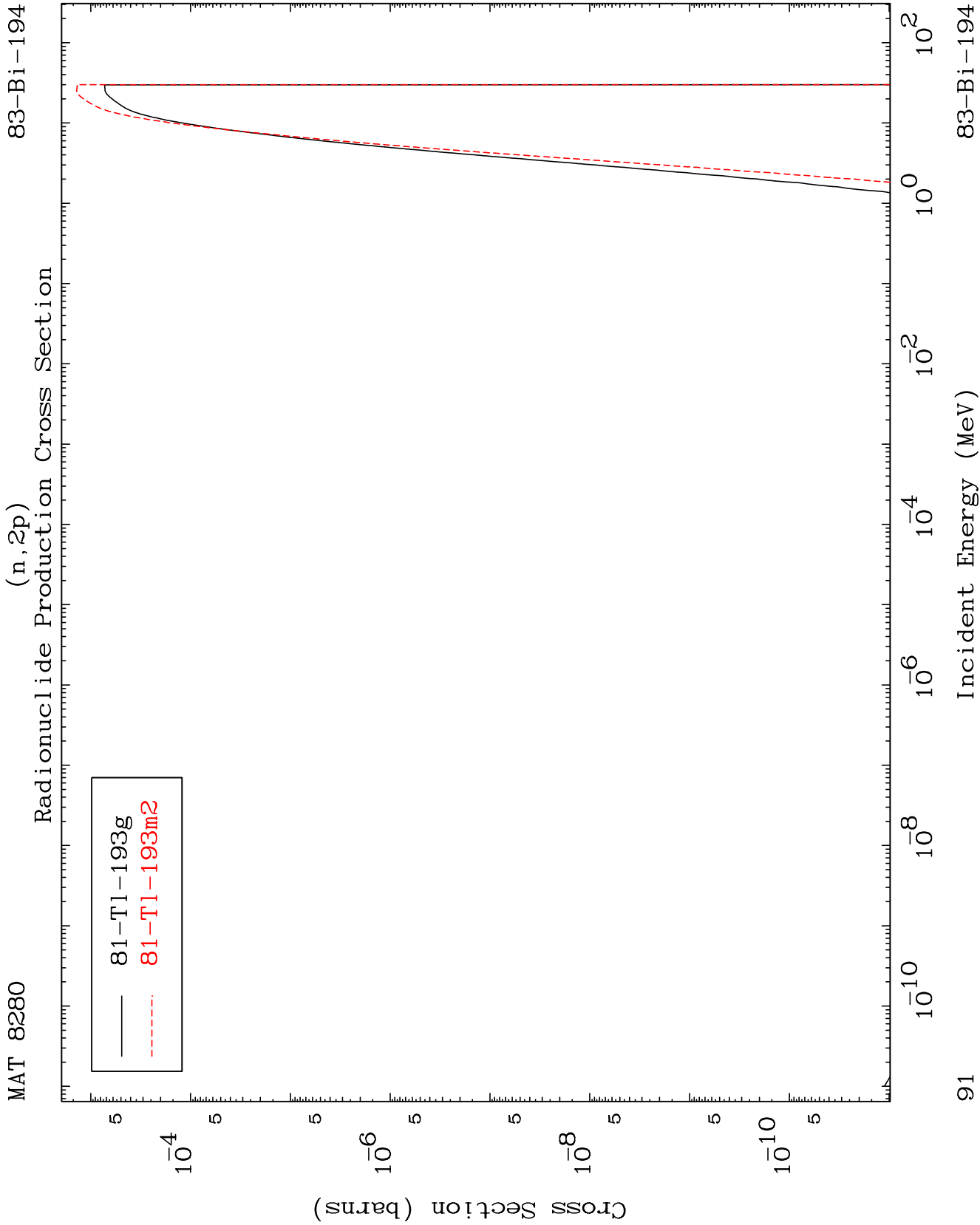
87

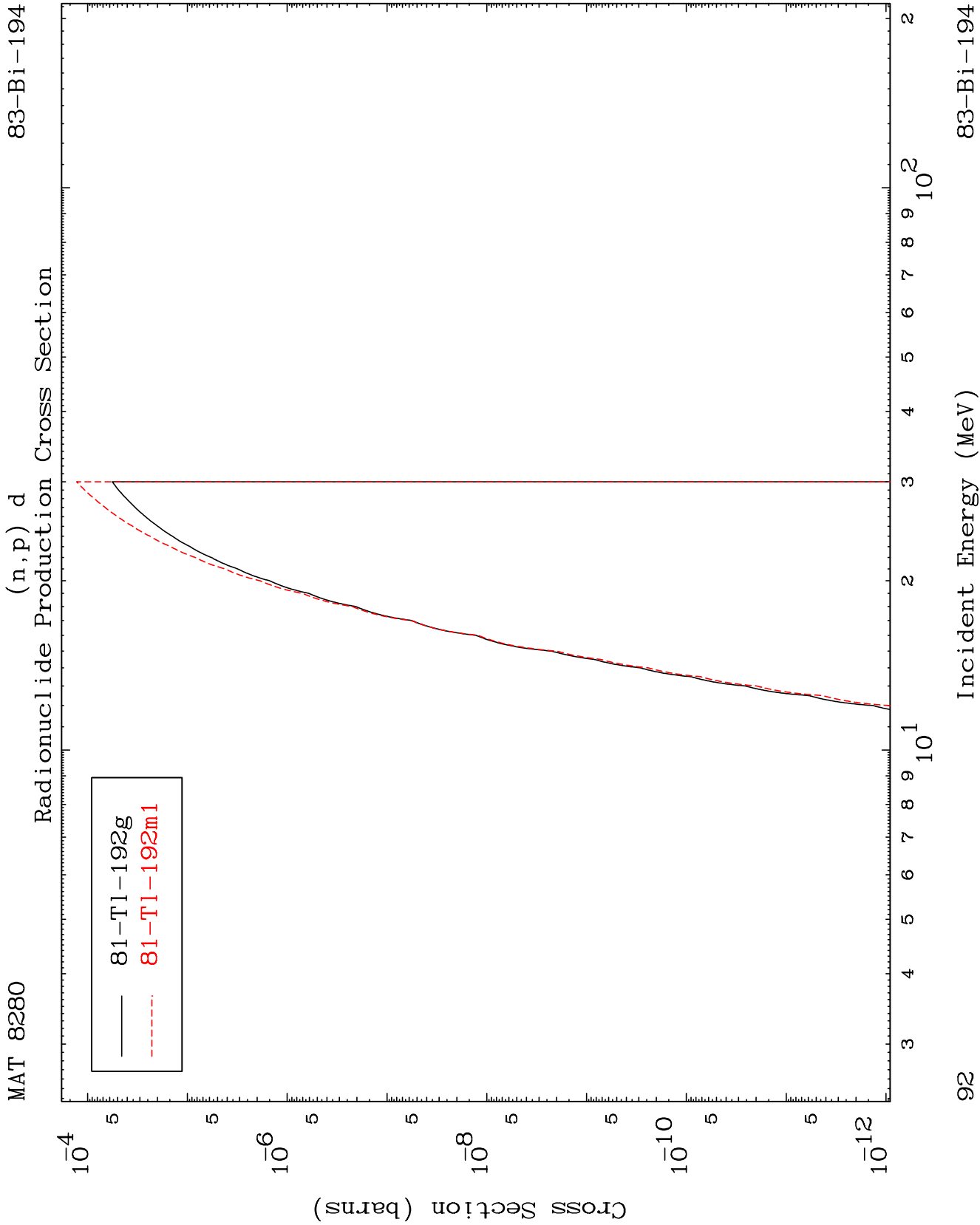
83-Bi-194









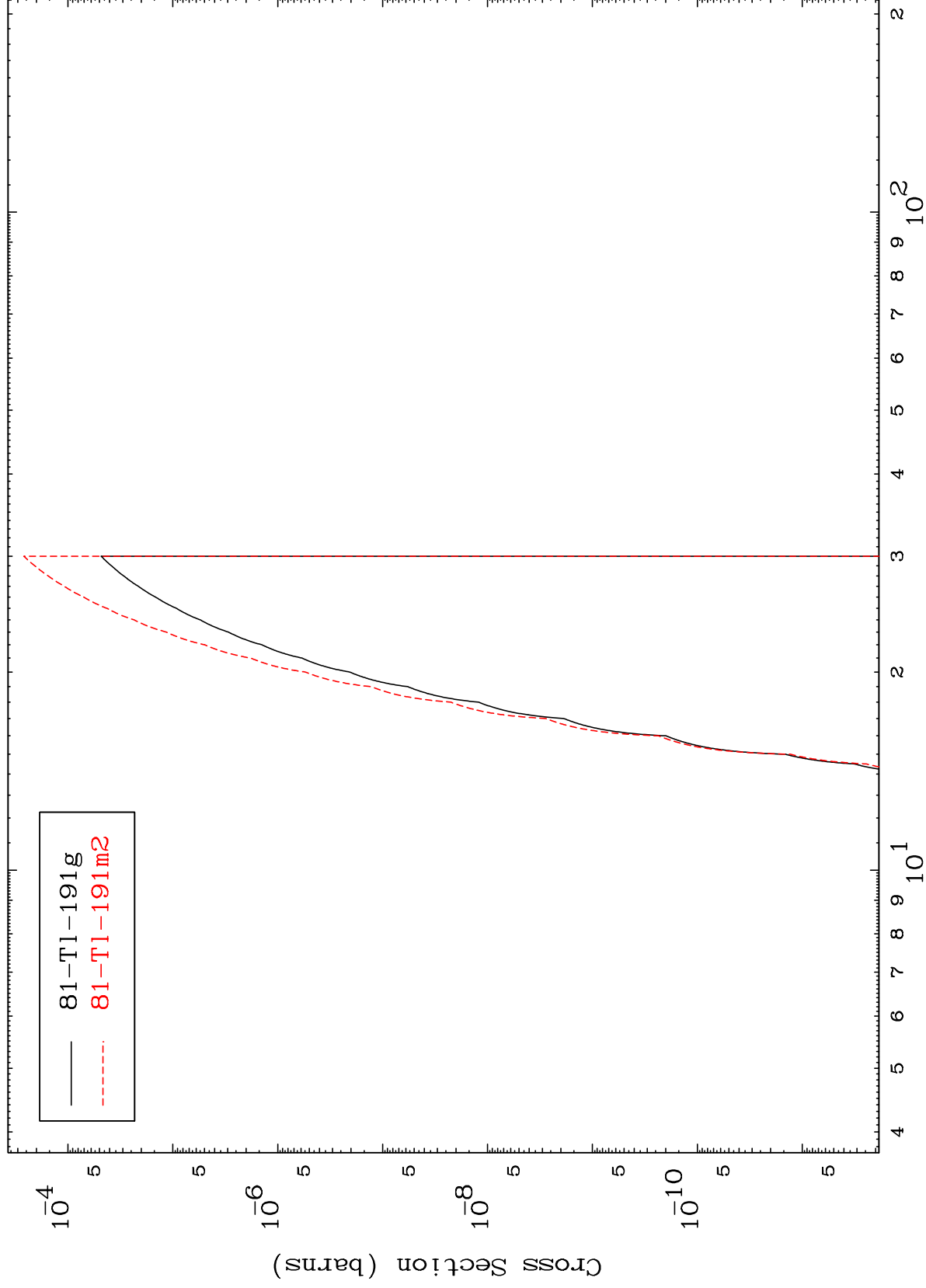


MAT 8280

(n,p) t

83-Bi-194

Radionuclide Production Cross Section



93

Incident Energy (MeV)

83-Bi-194

Radionuclide Production Cross Section

