

Program EVALPLOT
(Version 2017-1)

by

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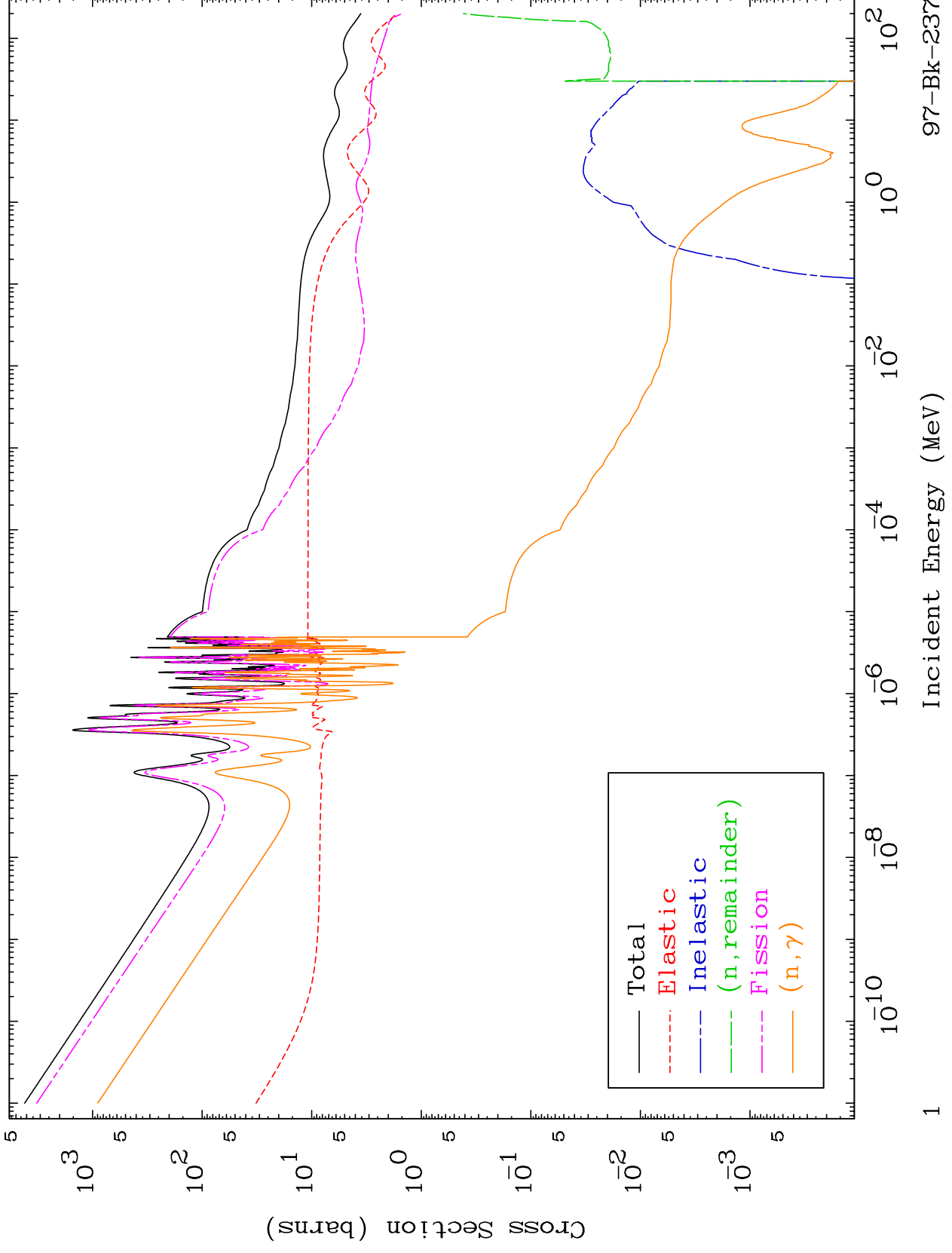
Press Mouse Button to Start

MAT 9716

Major

293 Kelvin Cross Sections

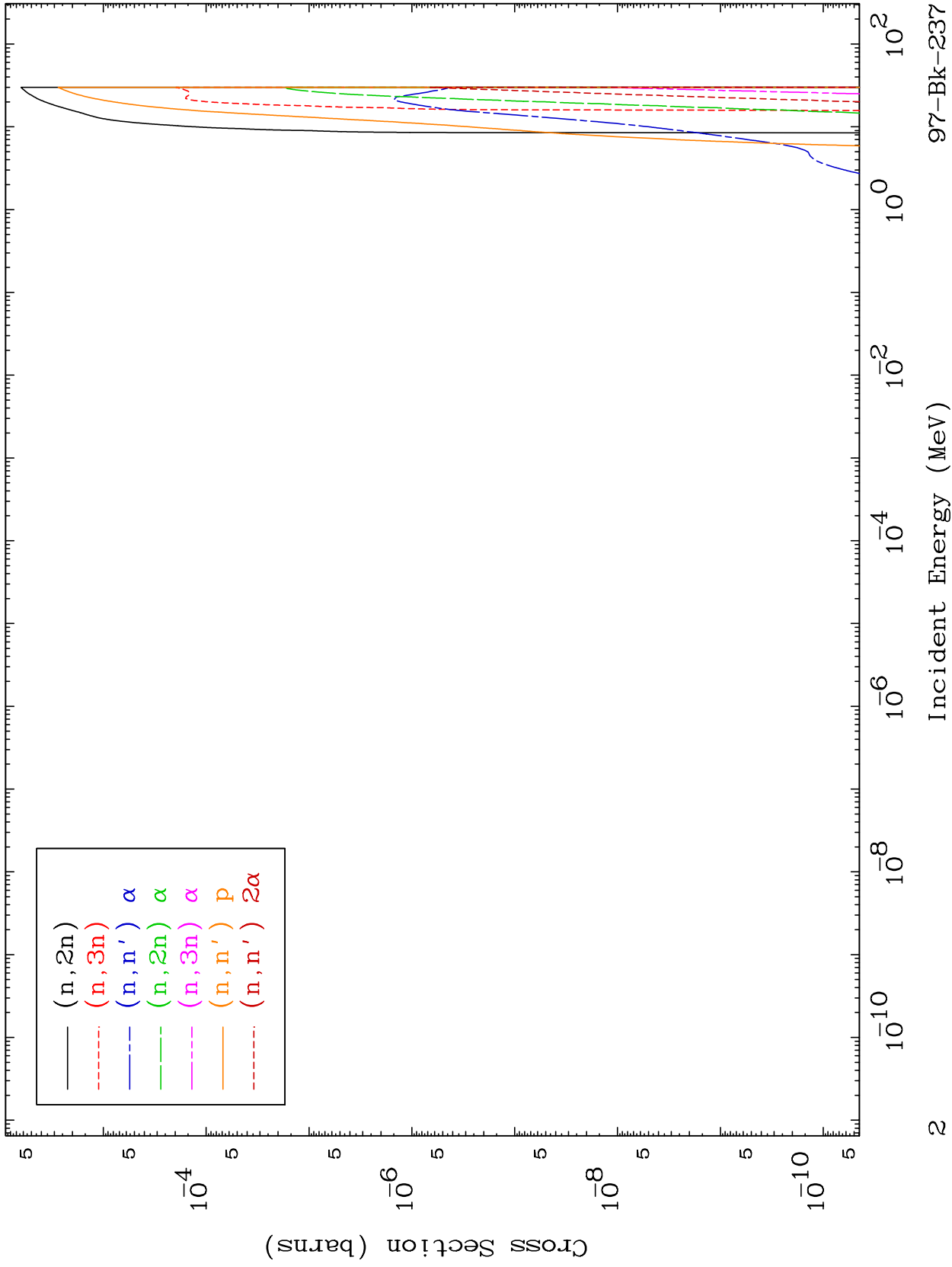
97-Bk-237



MAT 9716

Neutron Production
293 Kelvin Cross Sections

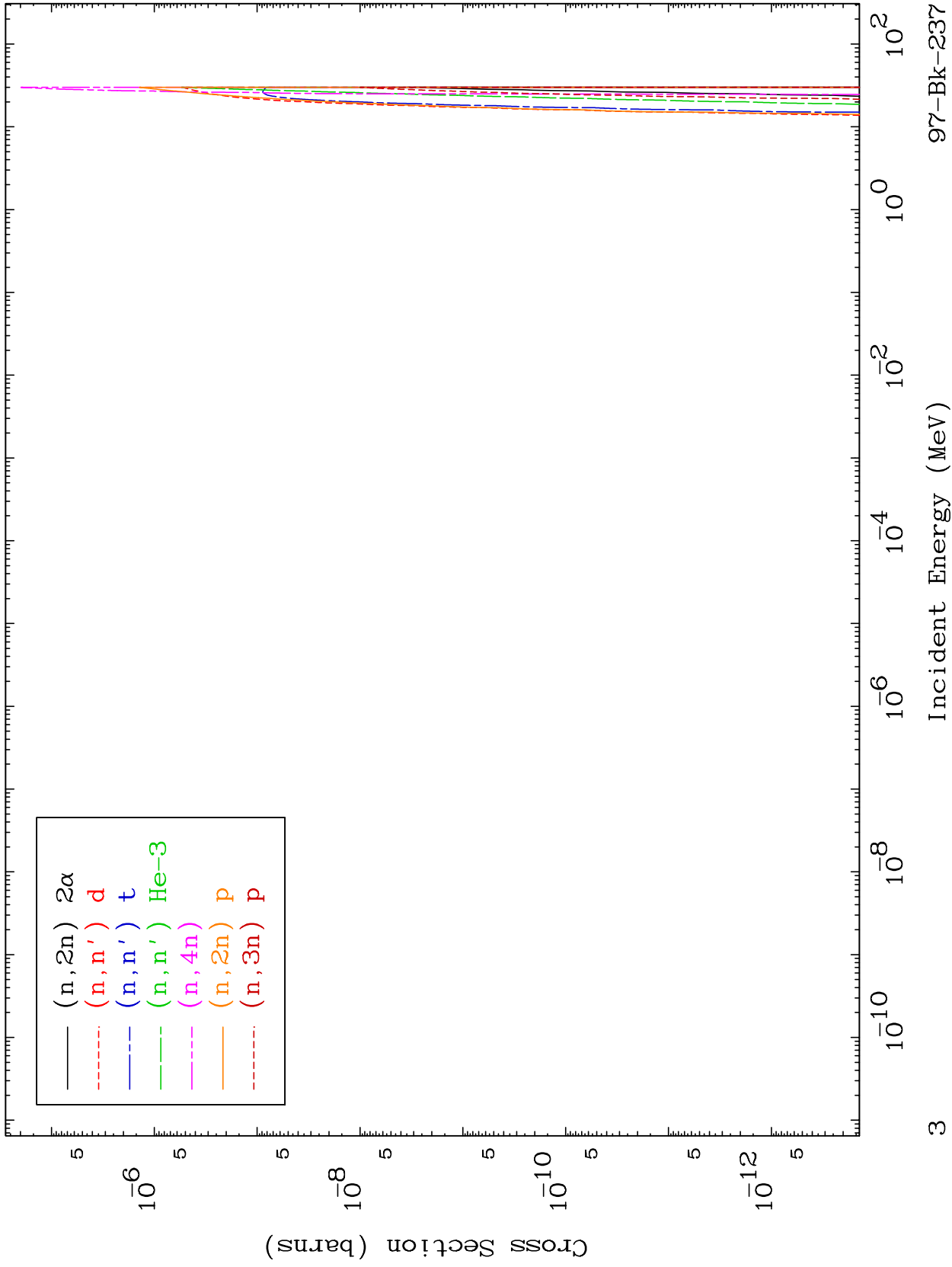
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MAT 9716

Neutron Production
293 Kelvin Cross Sections

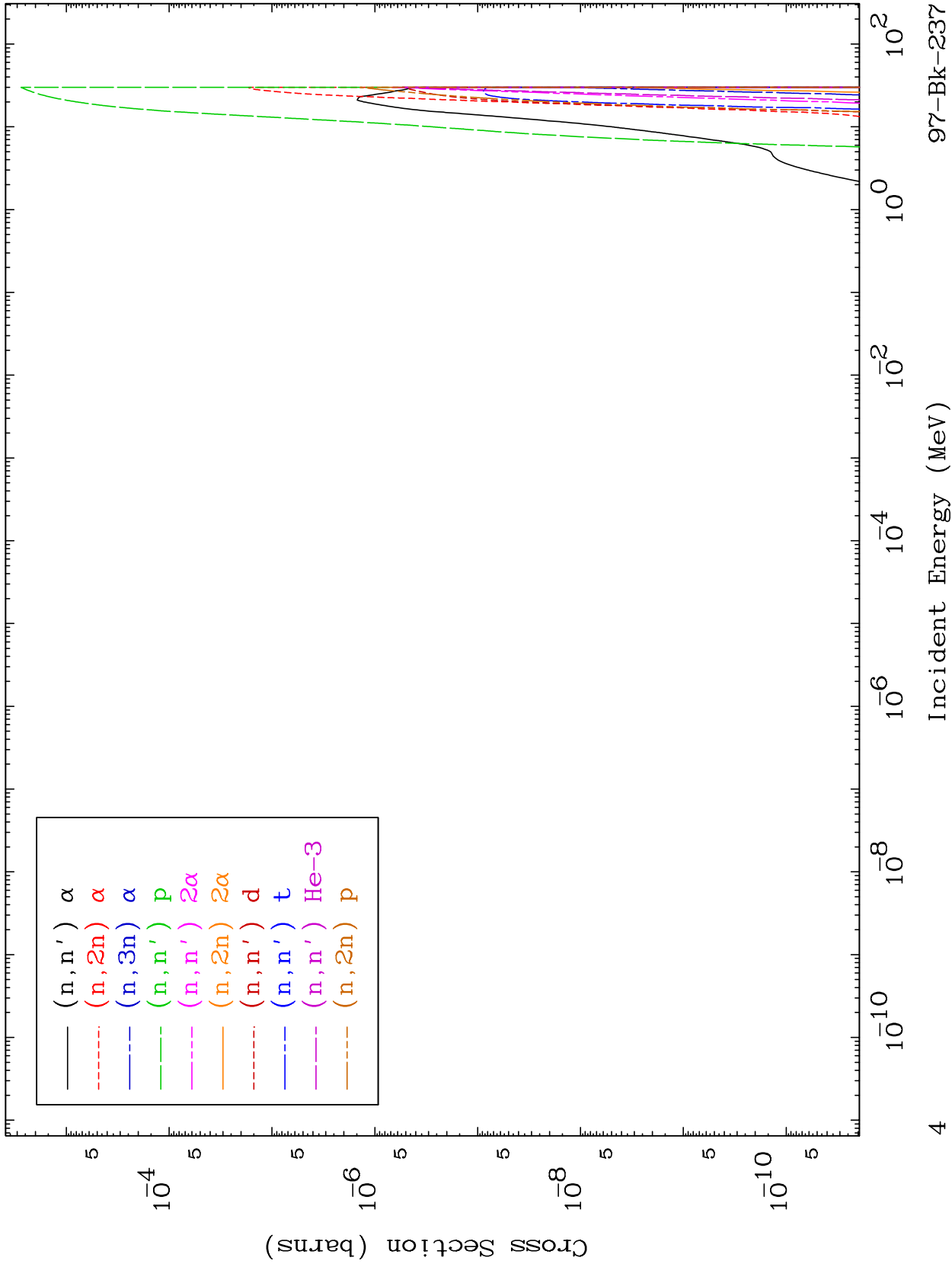
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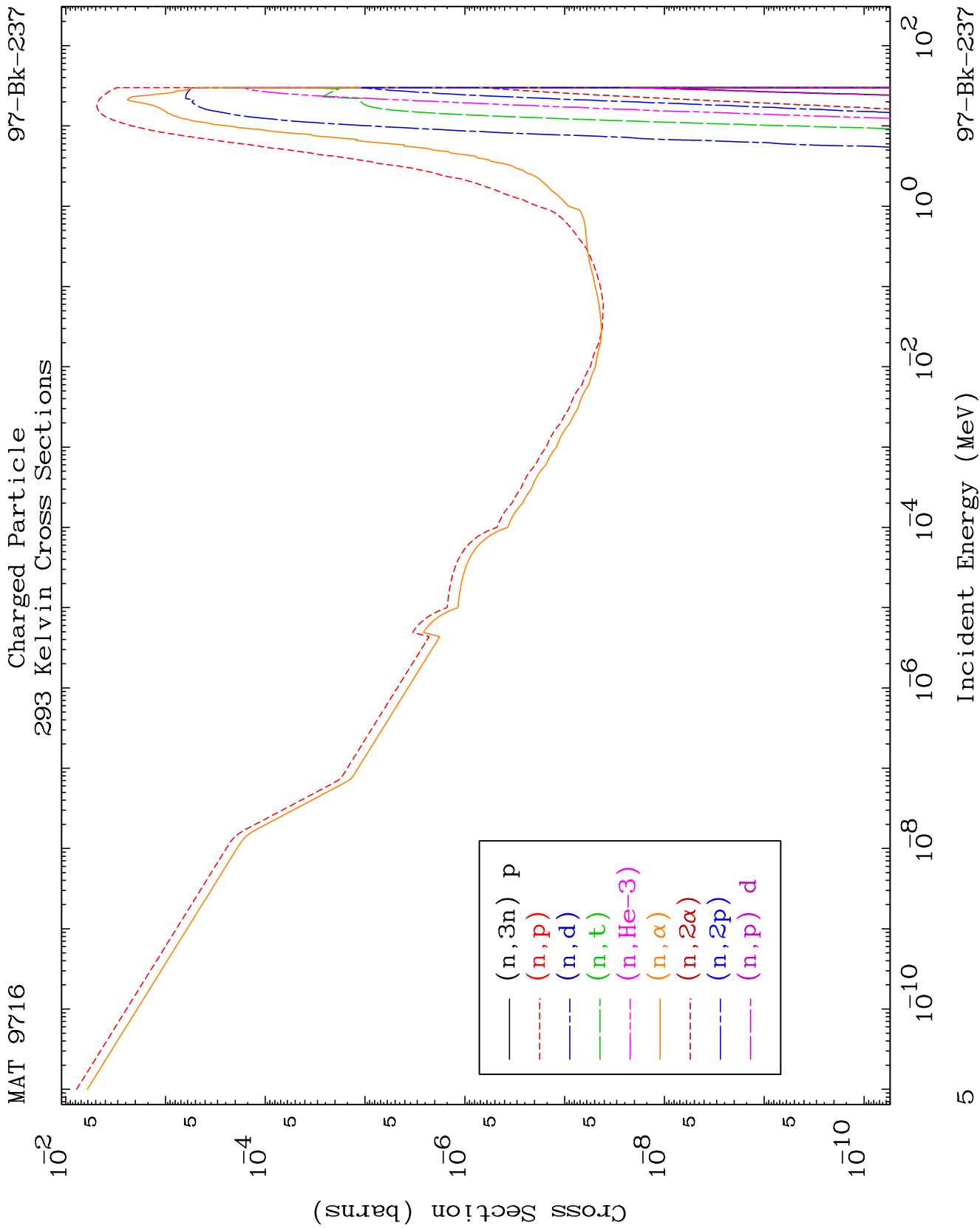


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Charged Particle
293 Kelvin Cross Sections

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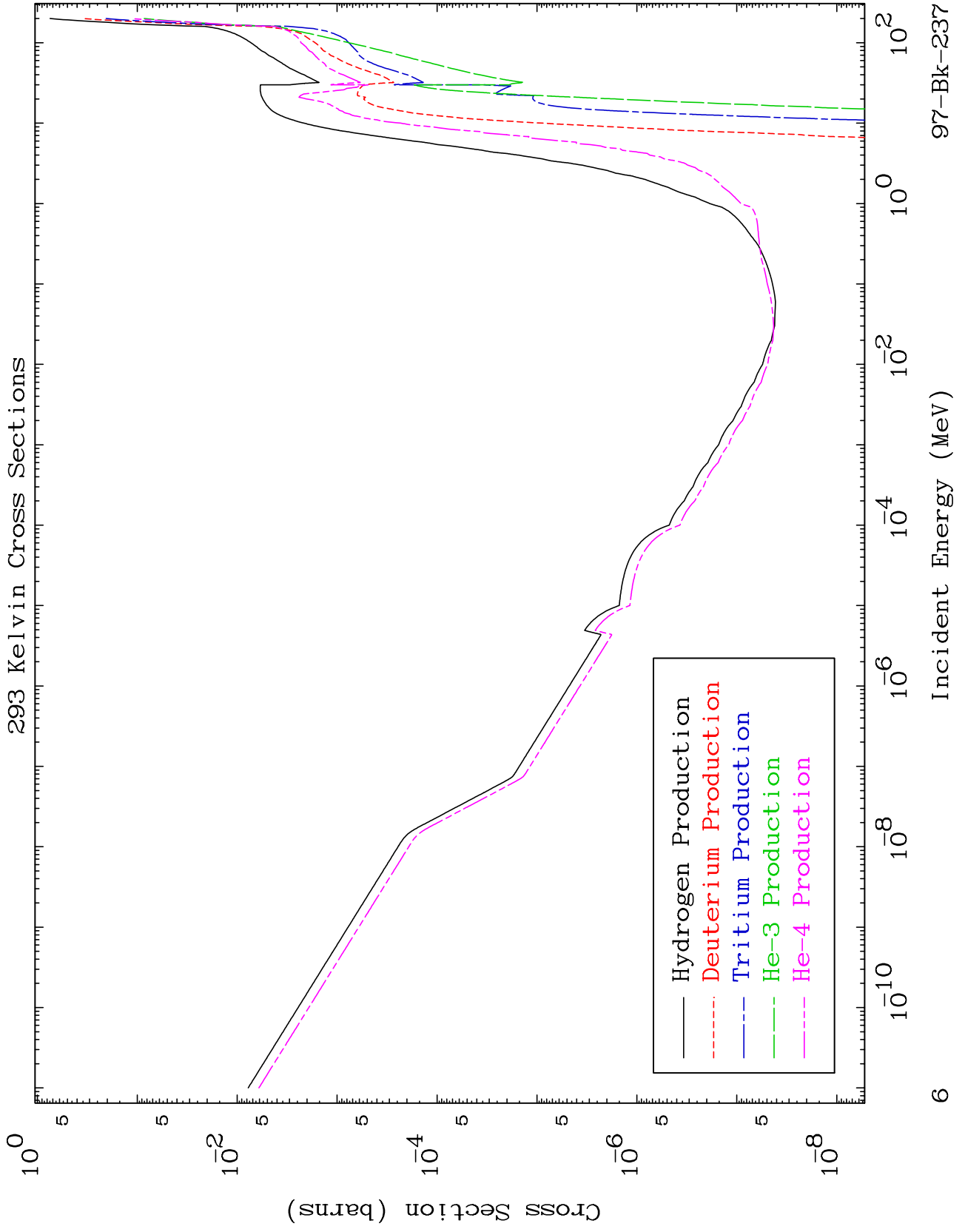




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Particle Production
293 Kelvin Cross Sections

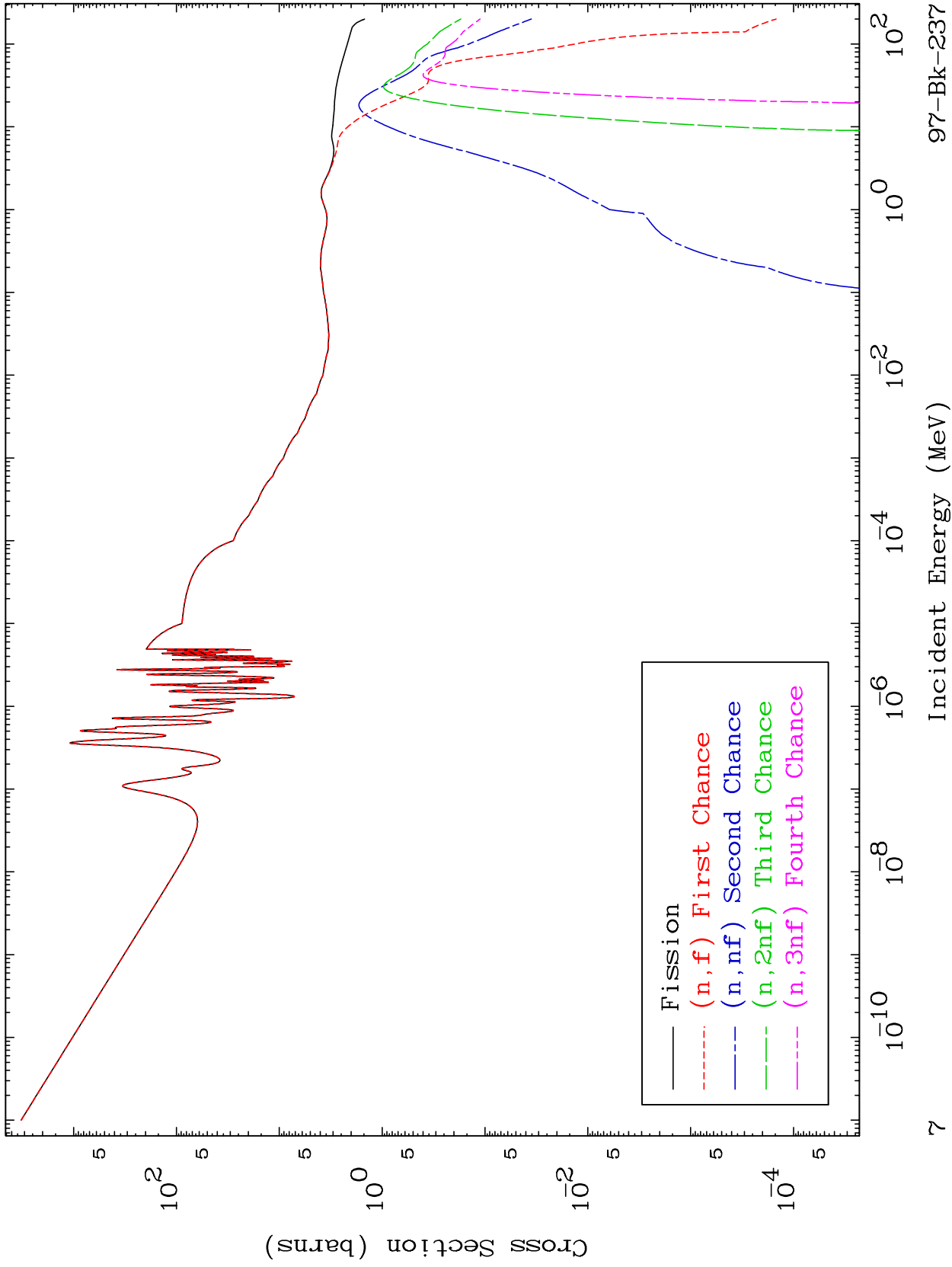
97-Bk-237



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Fission
293 Kelvin Cross Sections

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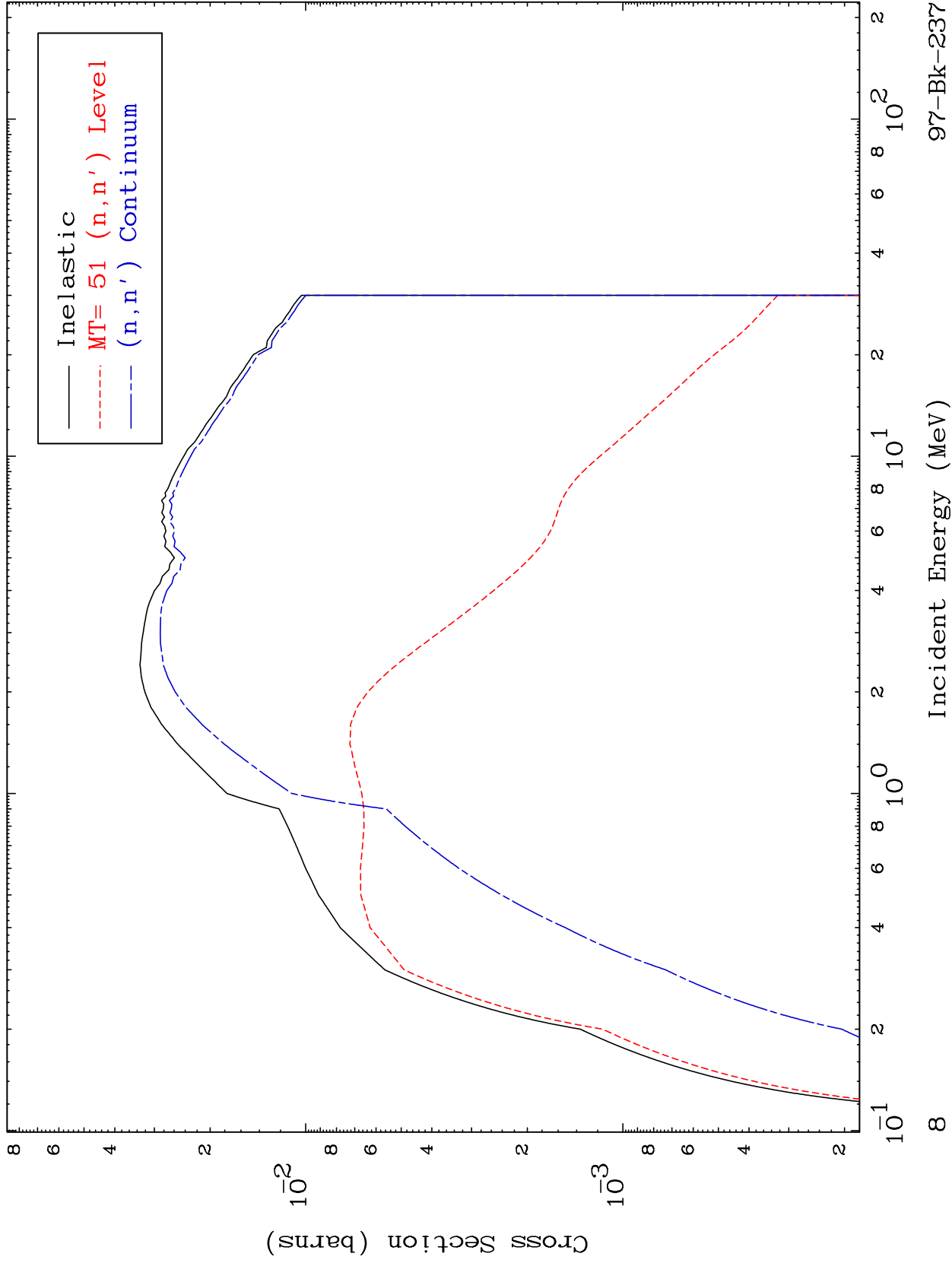


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(n,n') Level

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293 Kelvin Cross Sections



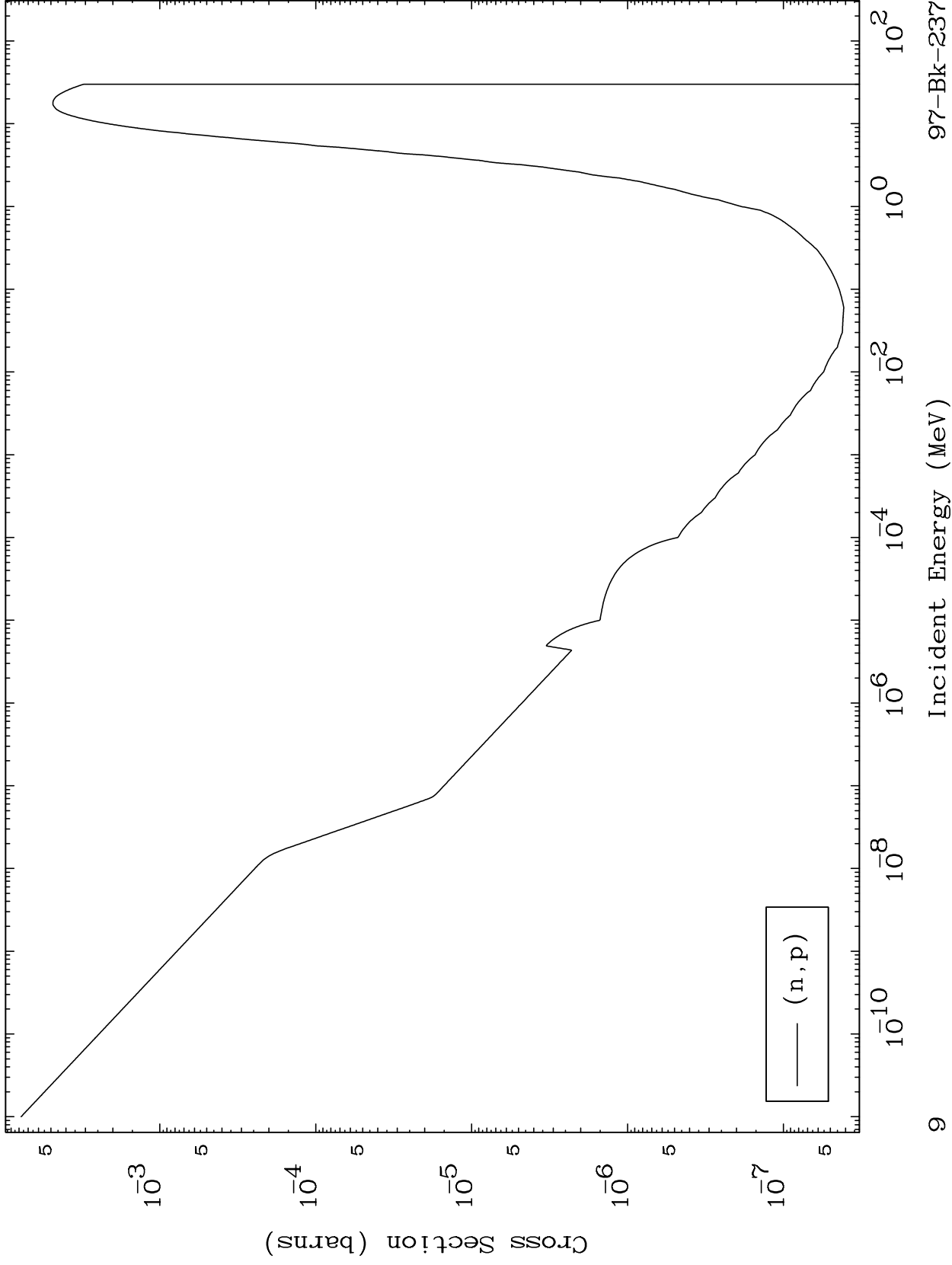
Incident Energy (MeV)

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(n,p) Levels
293 Kelvin Cross Sections

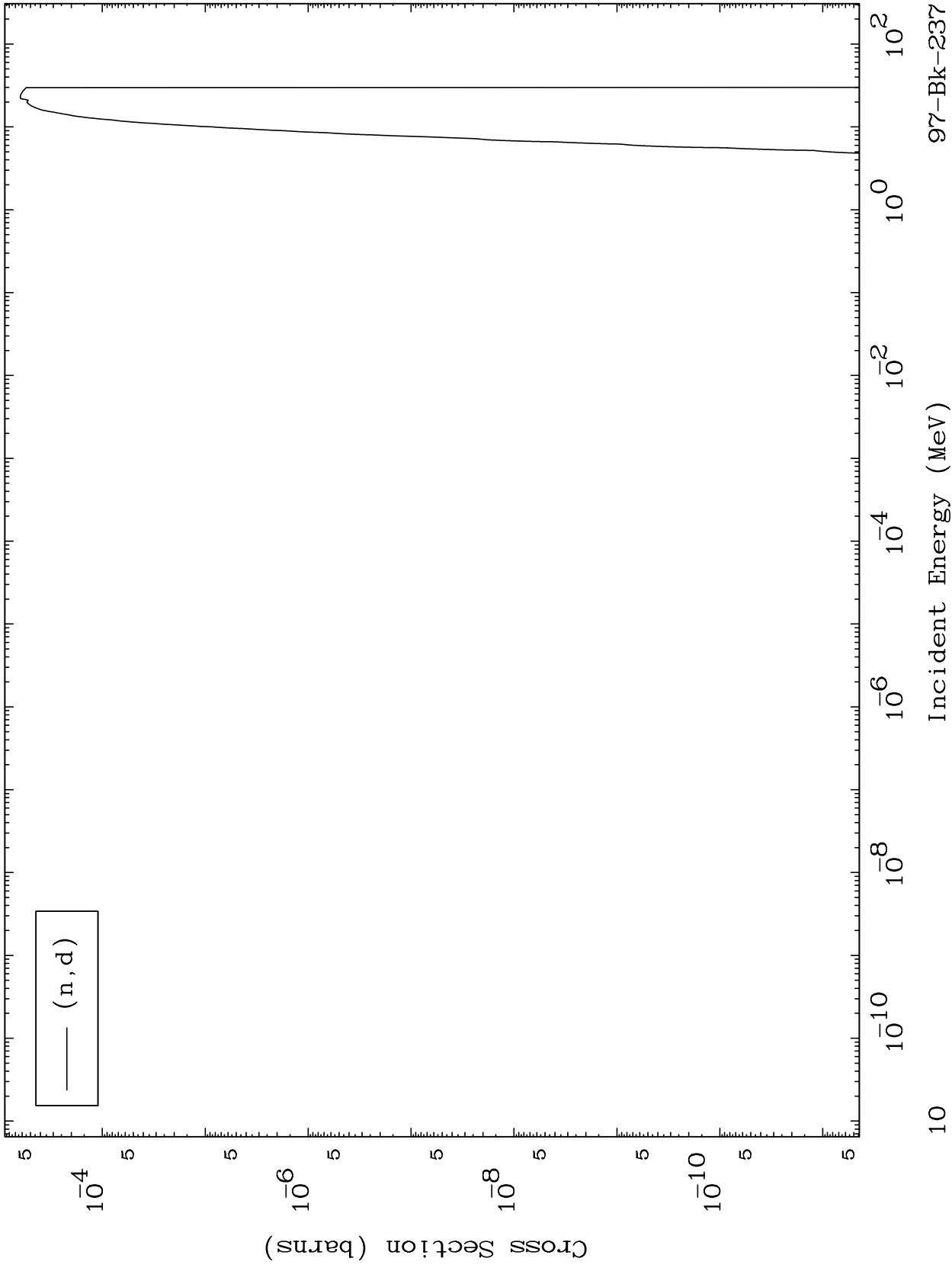
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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-237



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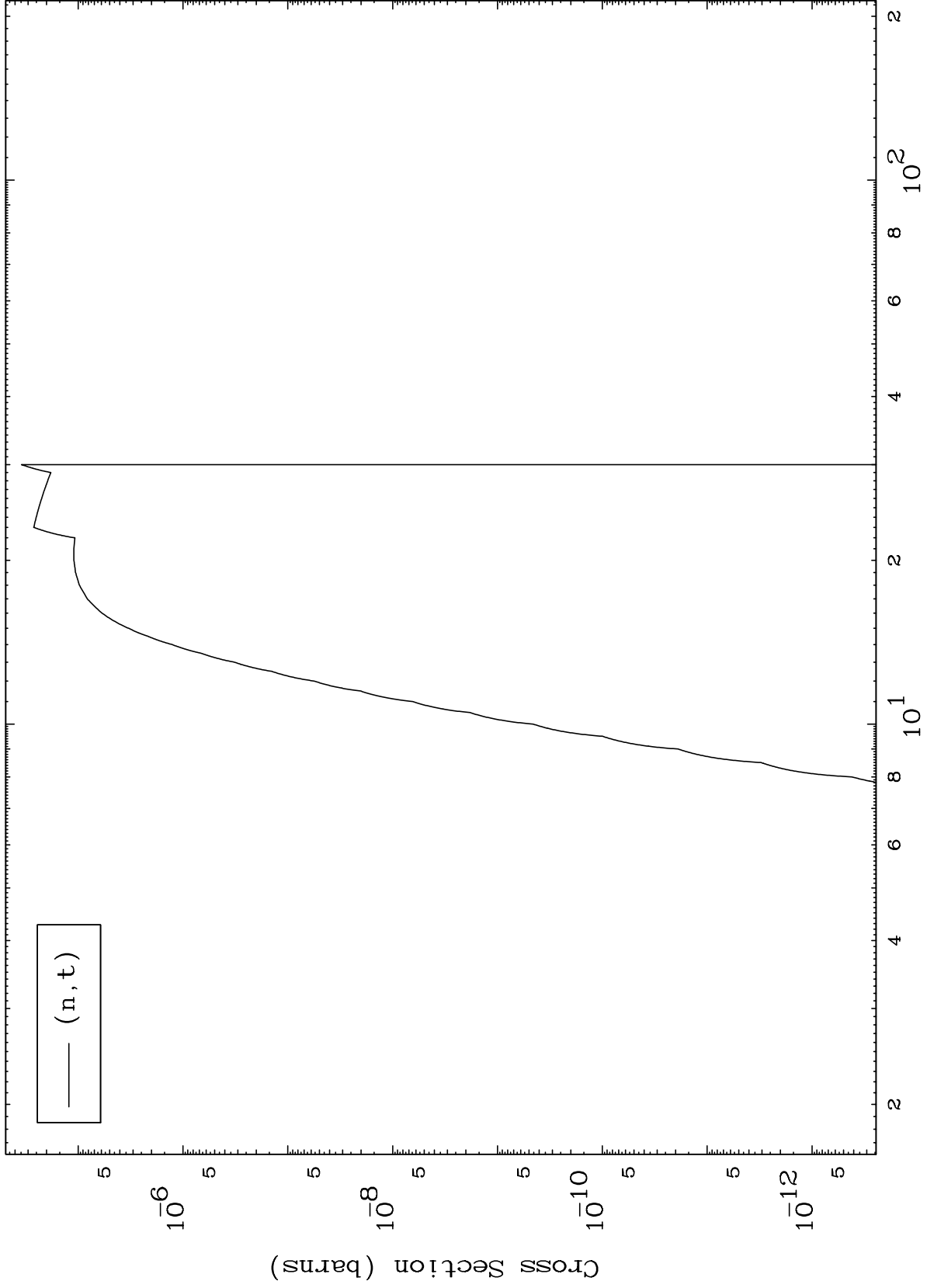
Incident Energy (MeV)

97-Bk-237

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(n,t) Levels
293 Kelvin Cross Sections

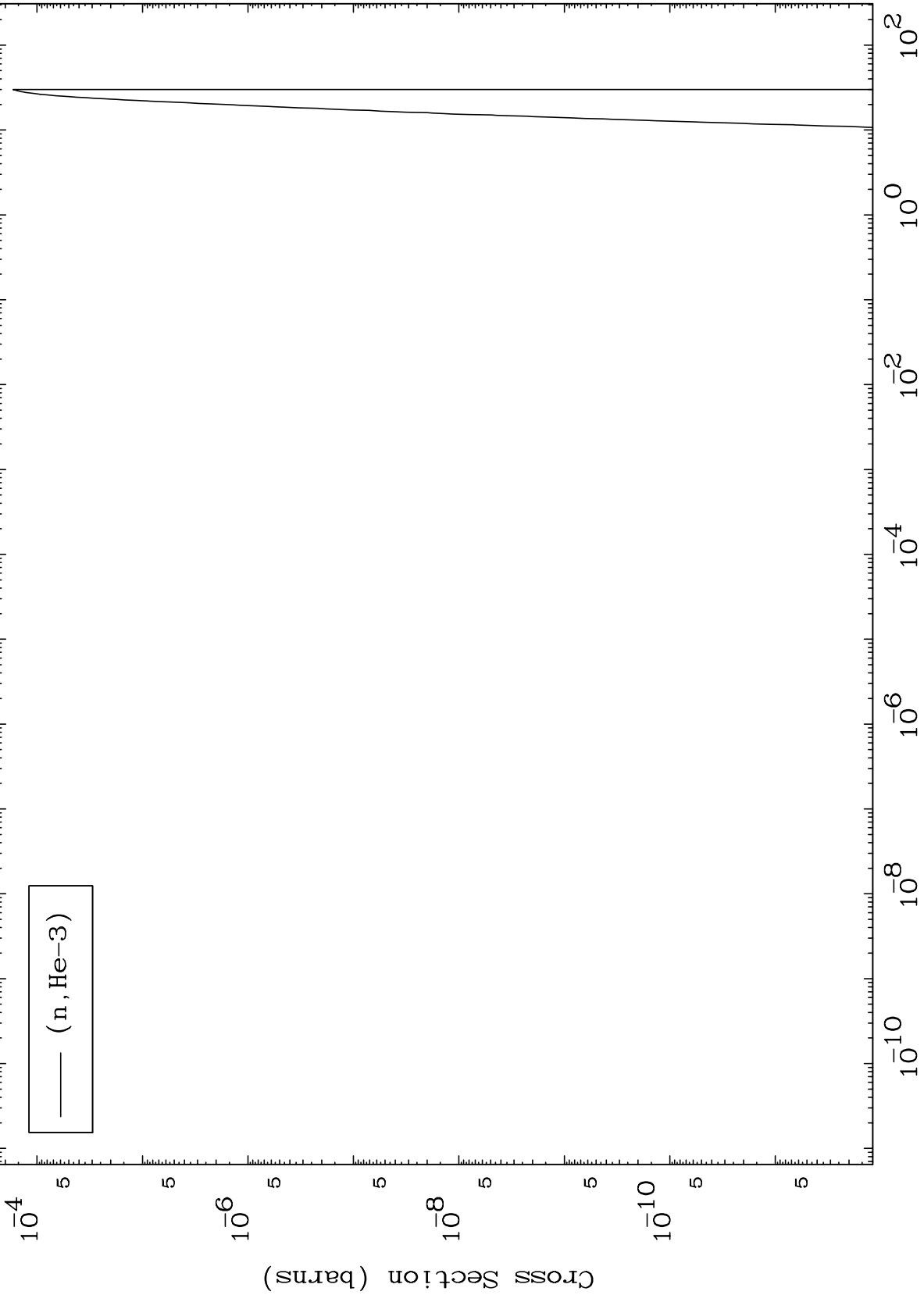
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(n,He3) Levels
293 Kelvin Cross Sections

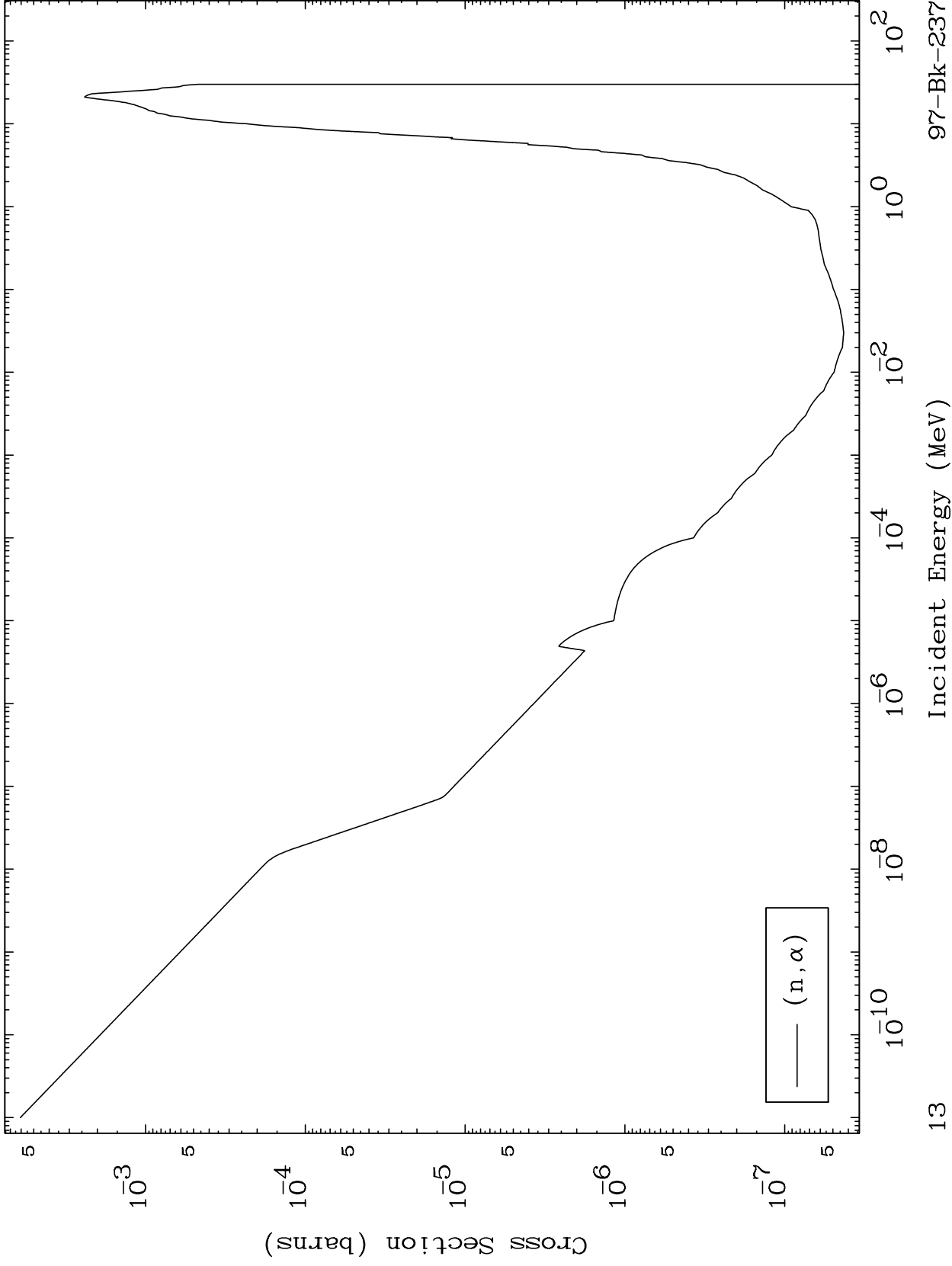
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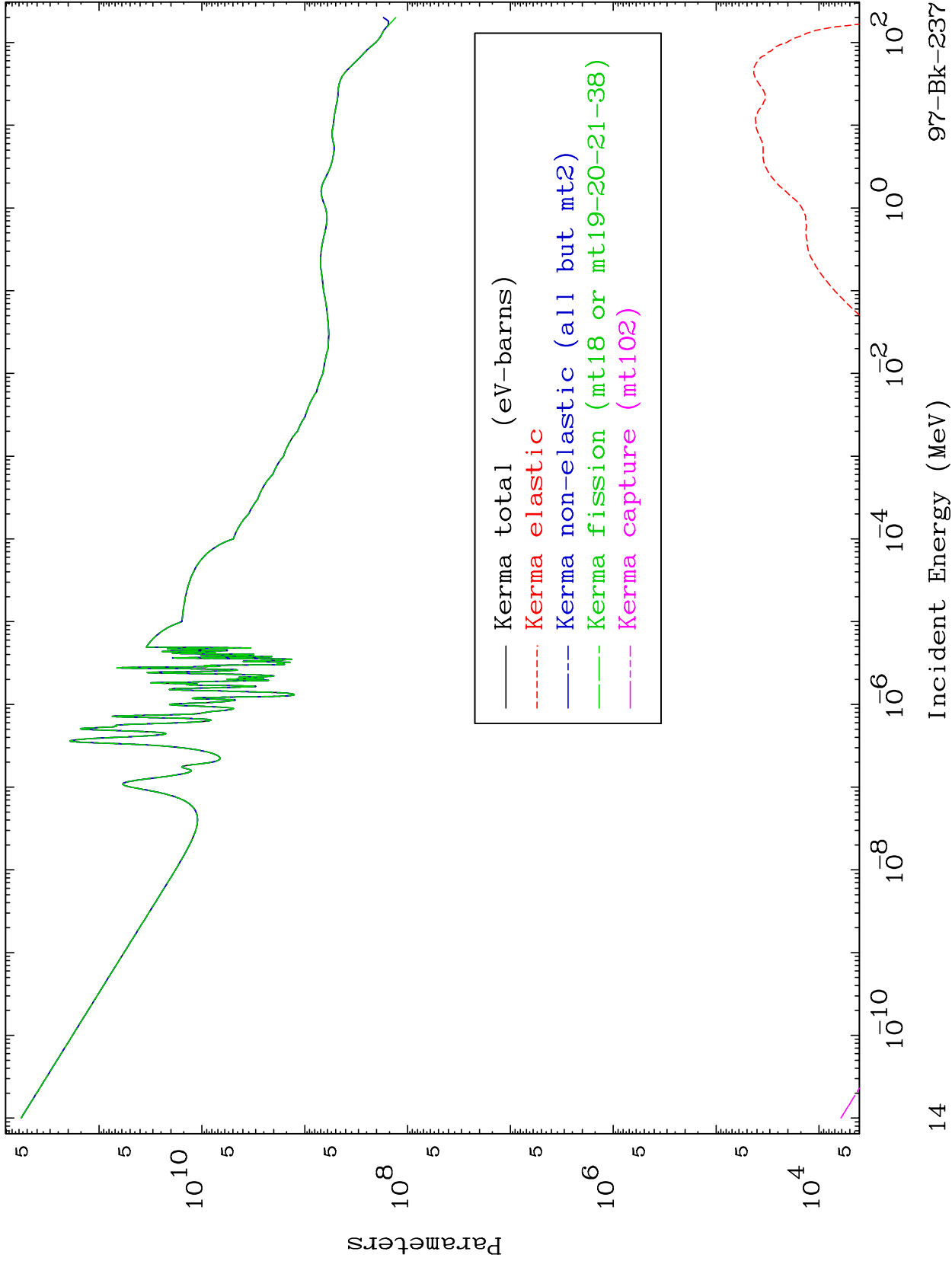


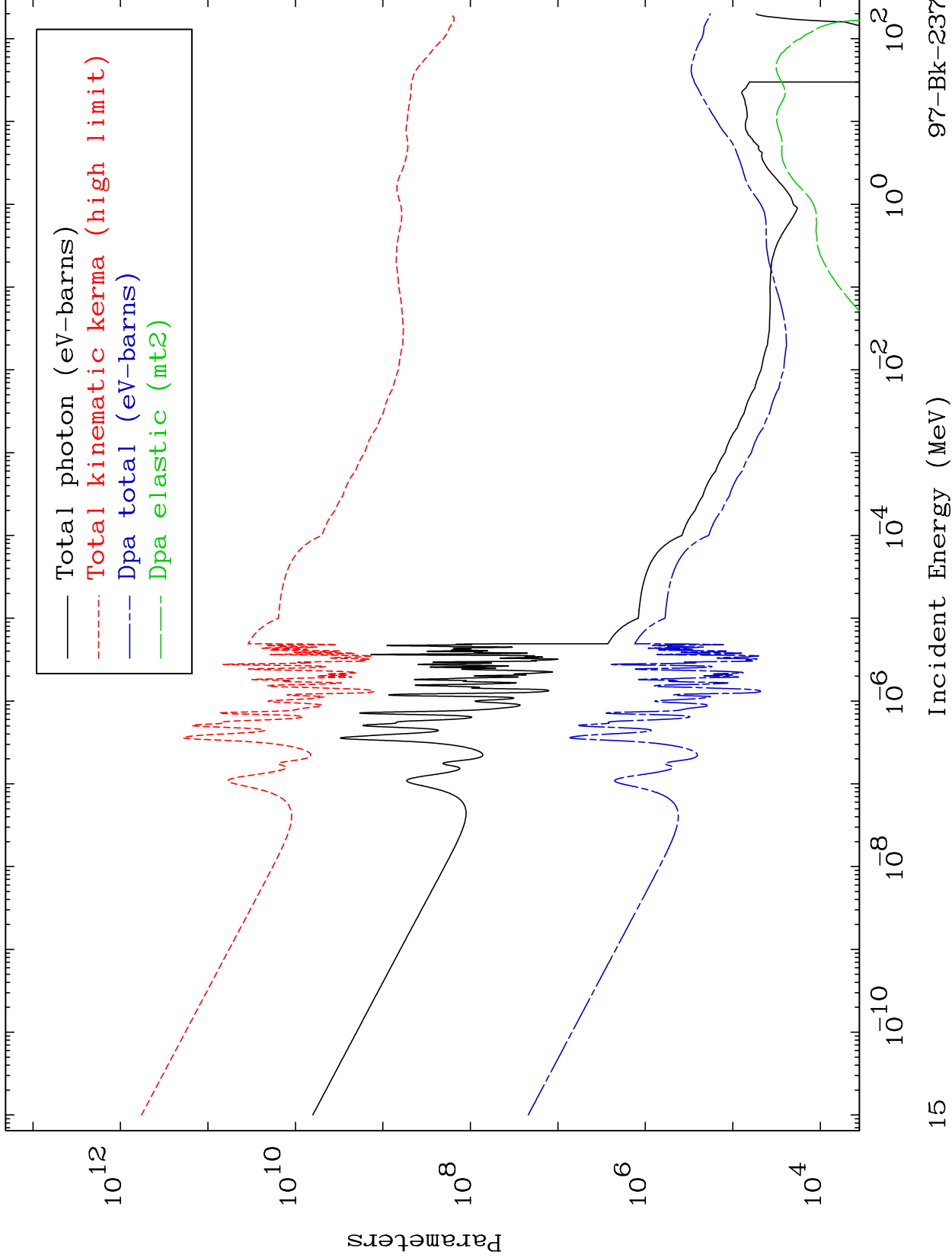
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(n,α) Levels
293 Kelvin Cross Sections

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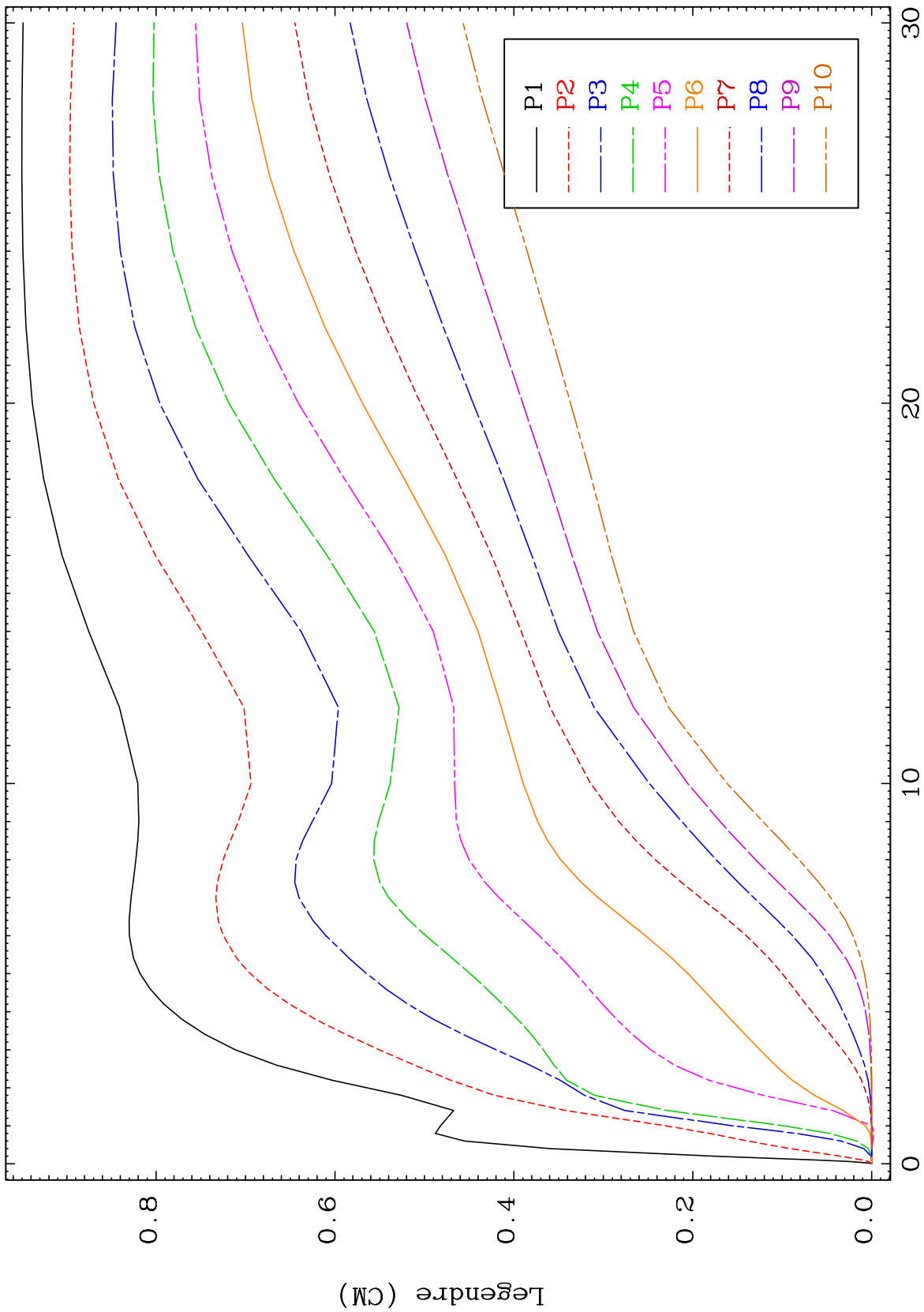




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Elastic Legendre Coefficients

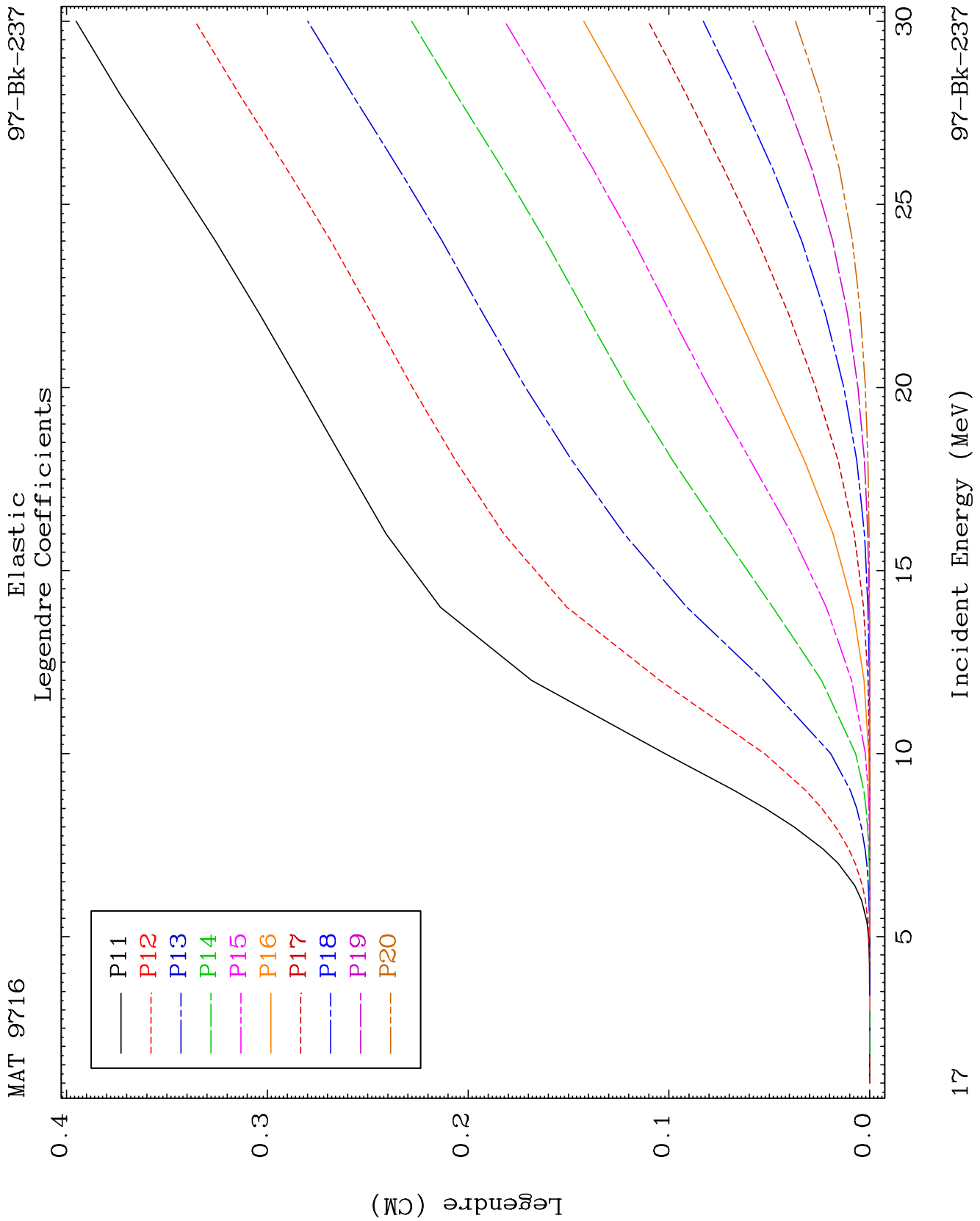
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Incident Energy (MeV)

97-Bk-237

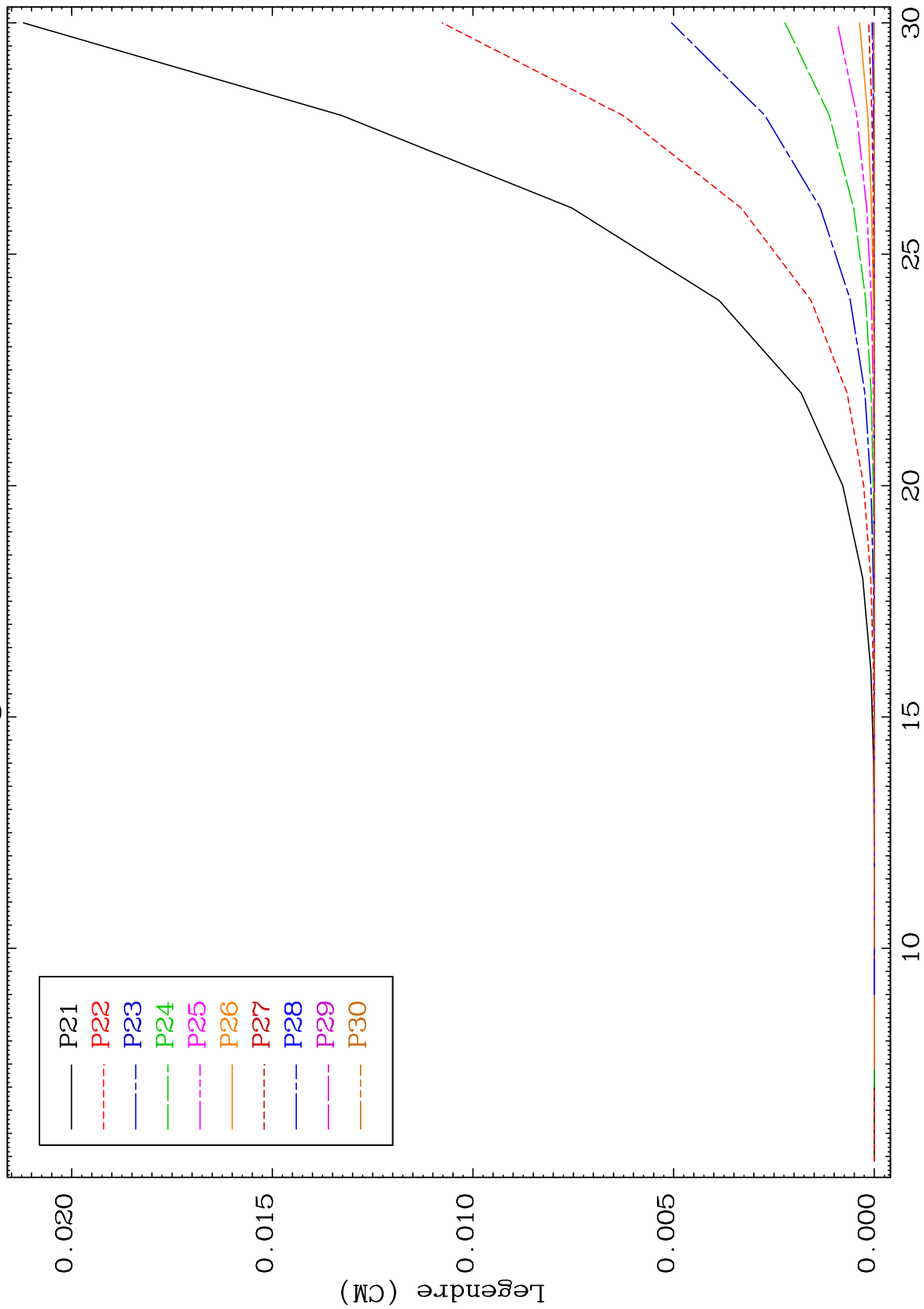
16



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Elastic Legendre Coefficients

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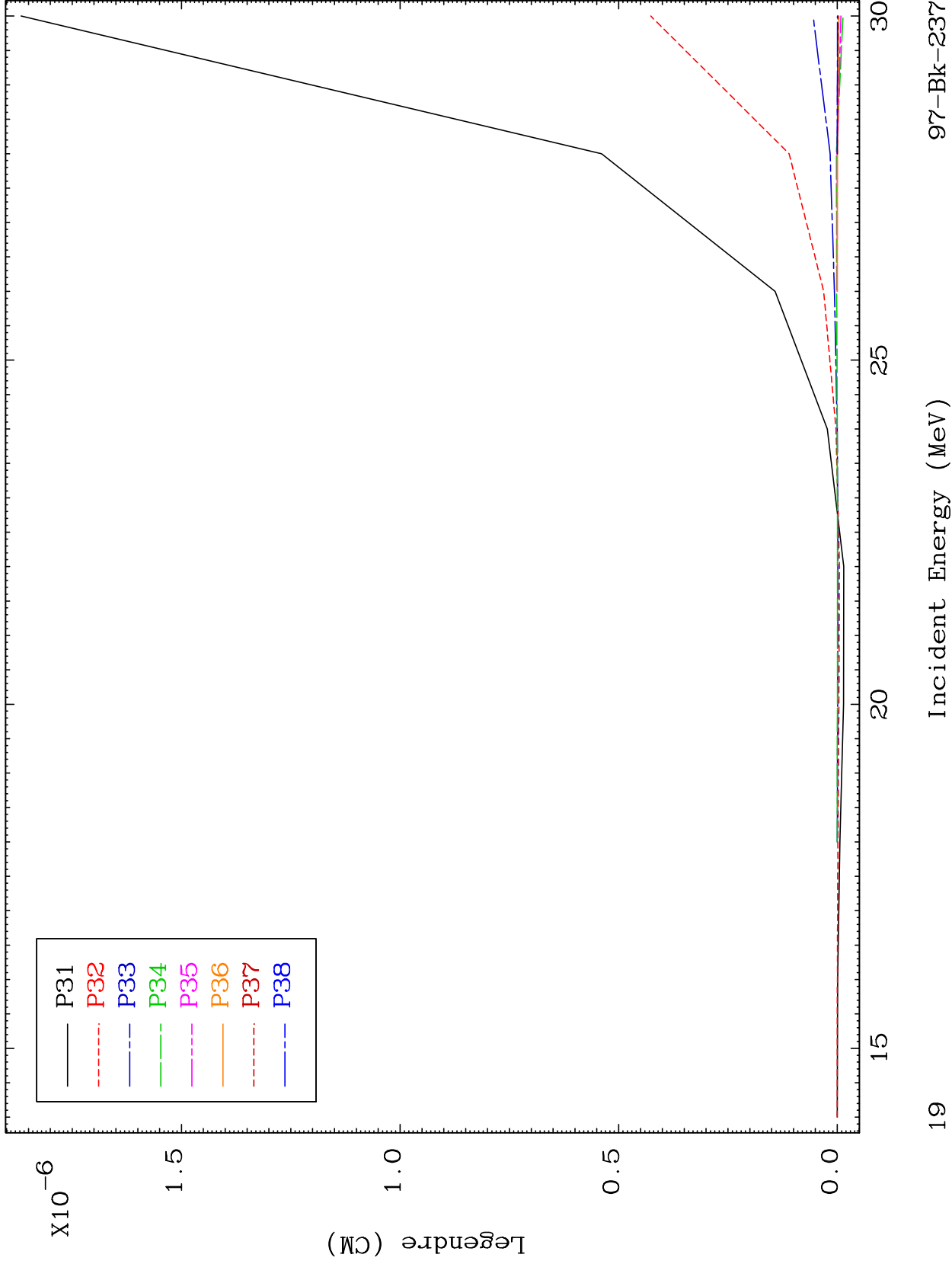
Incident Energy (MeV)

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Elastic
Legendre Coefficients

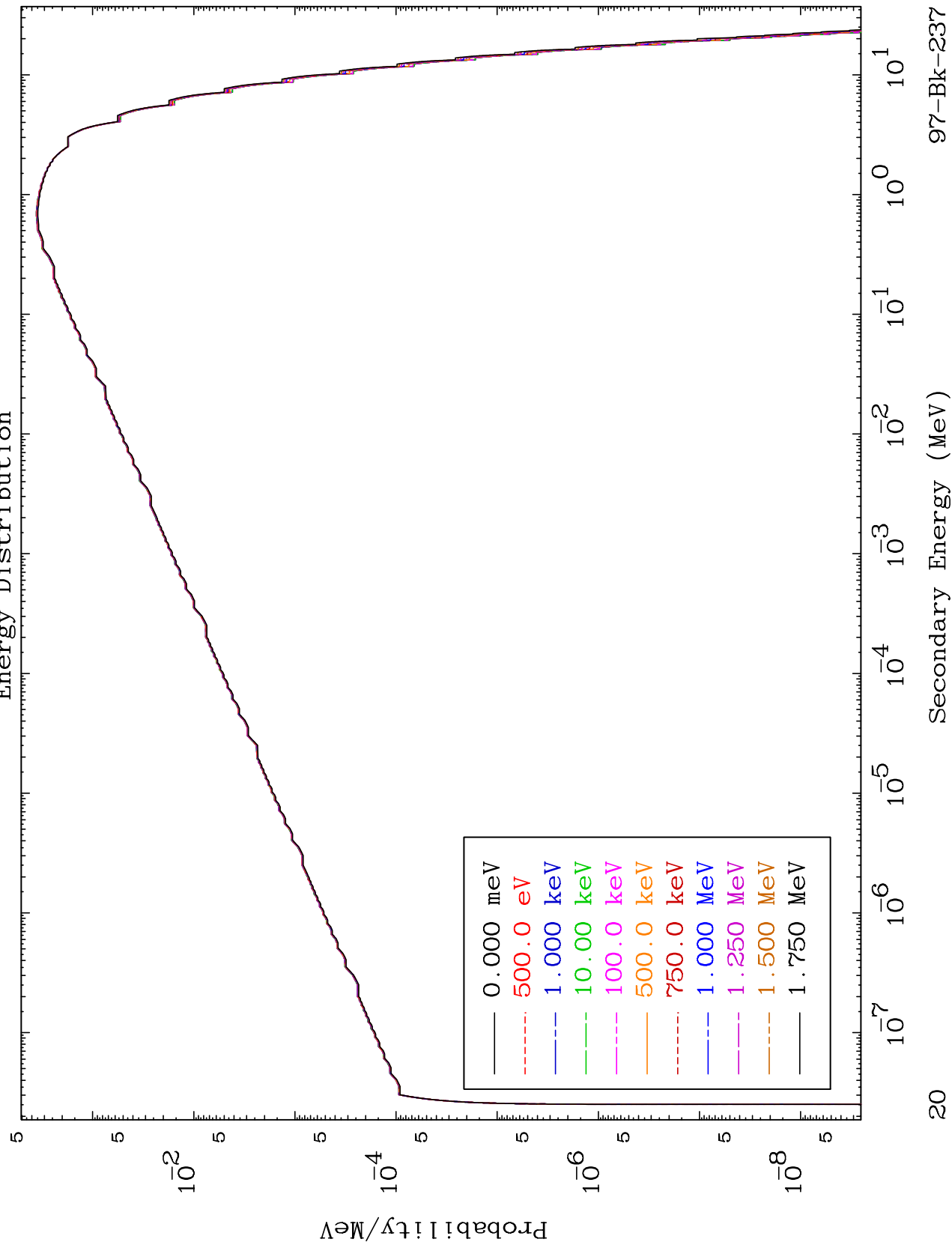
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Fission Energy Distribution

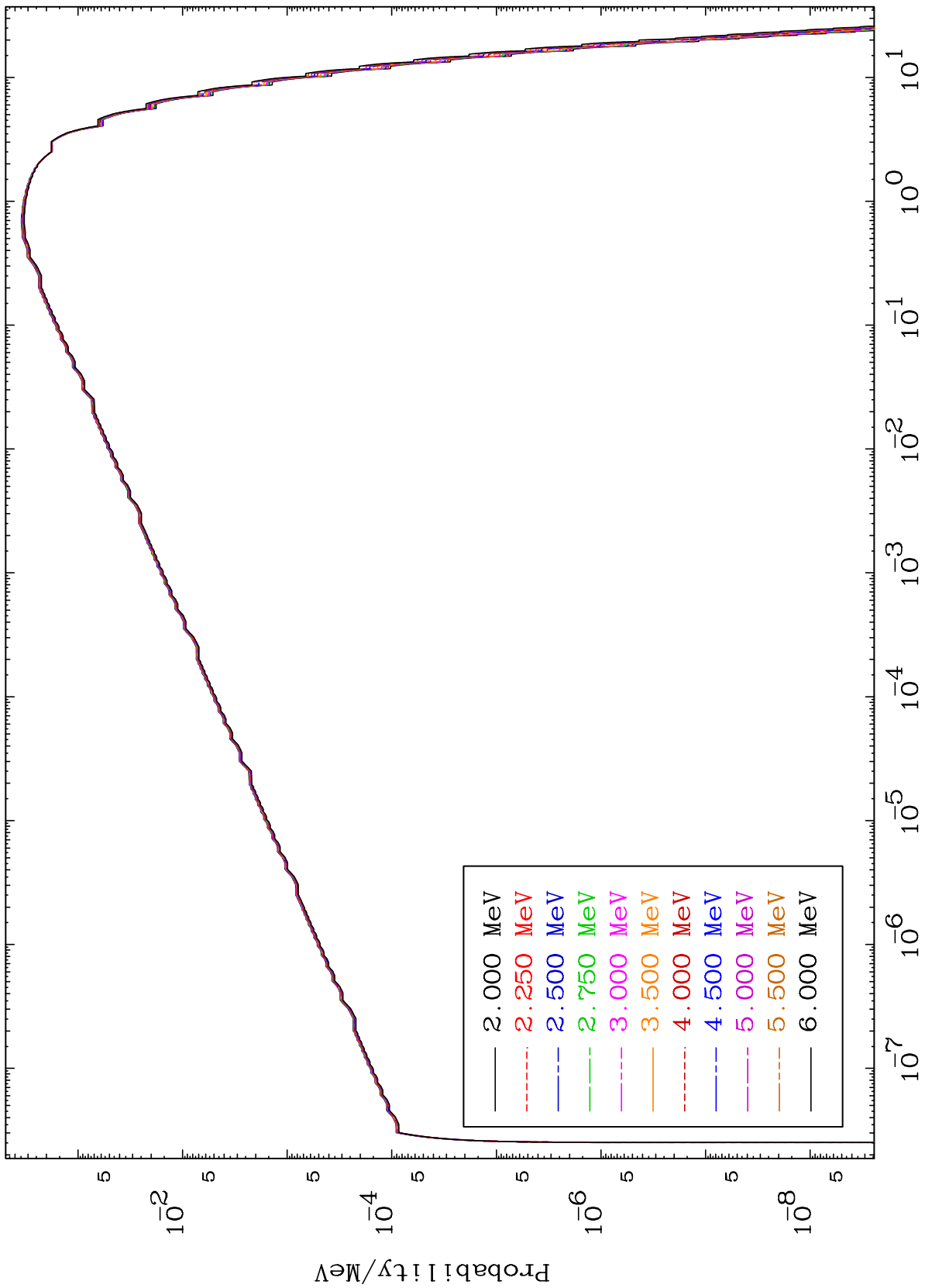
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Fission Energy Distribution

97-Bk-237

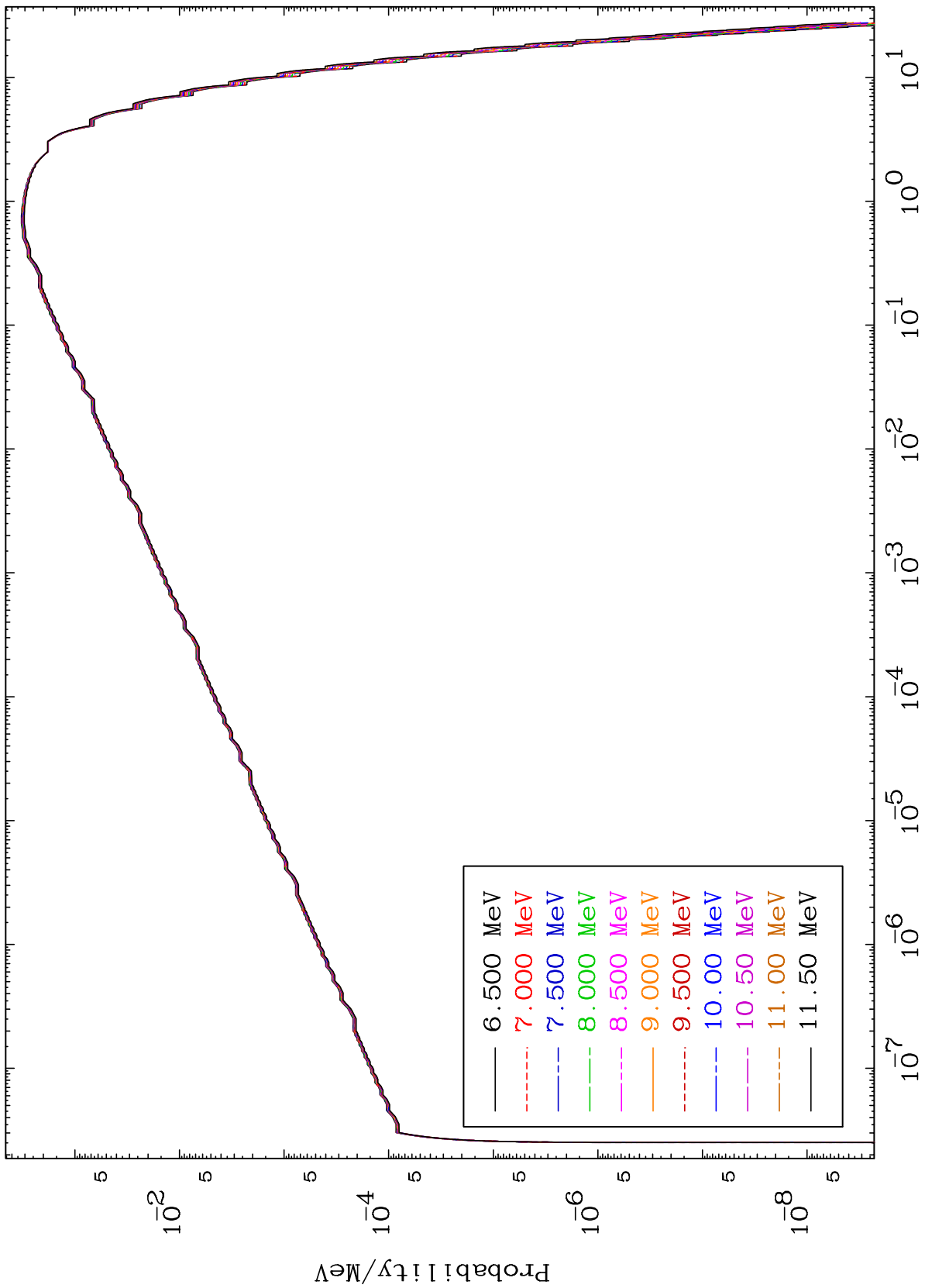


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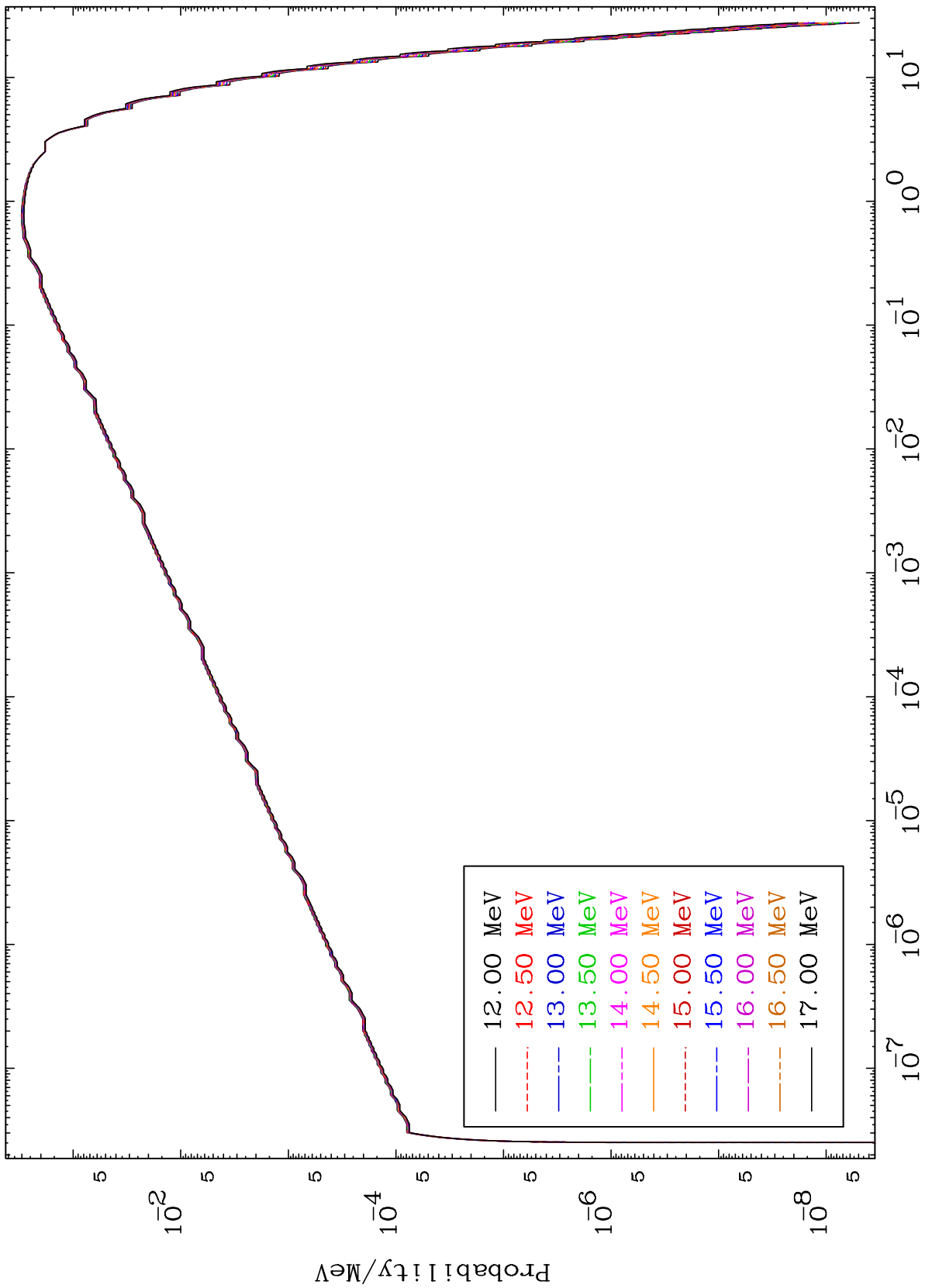
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Fission Energy Distribution

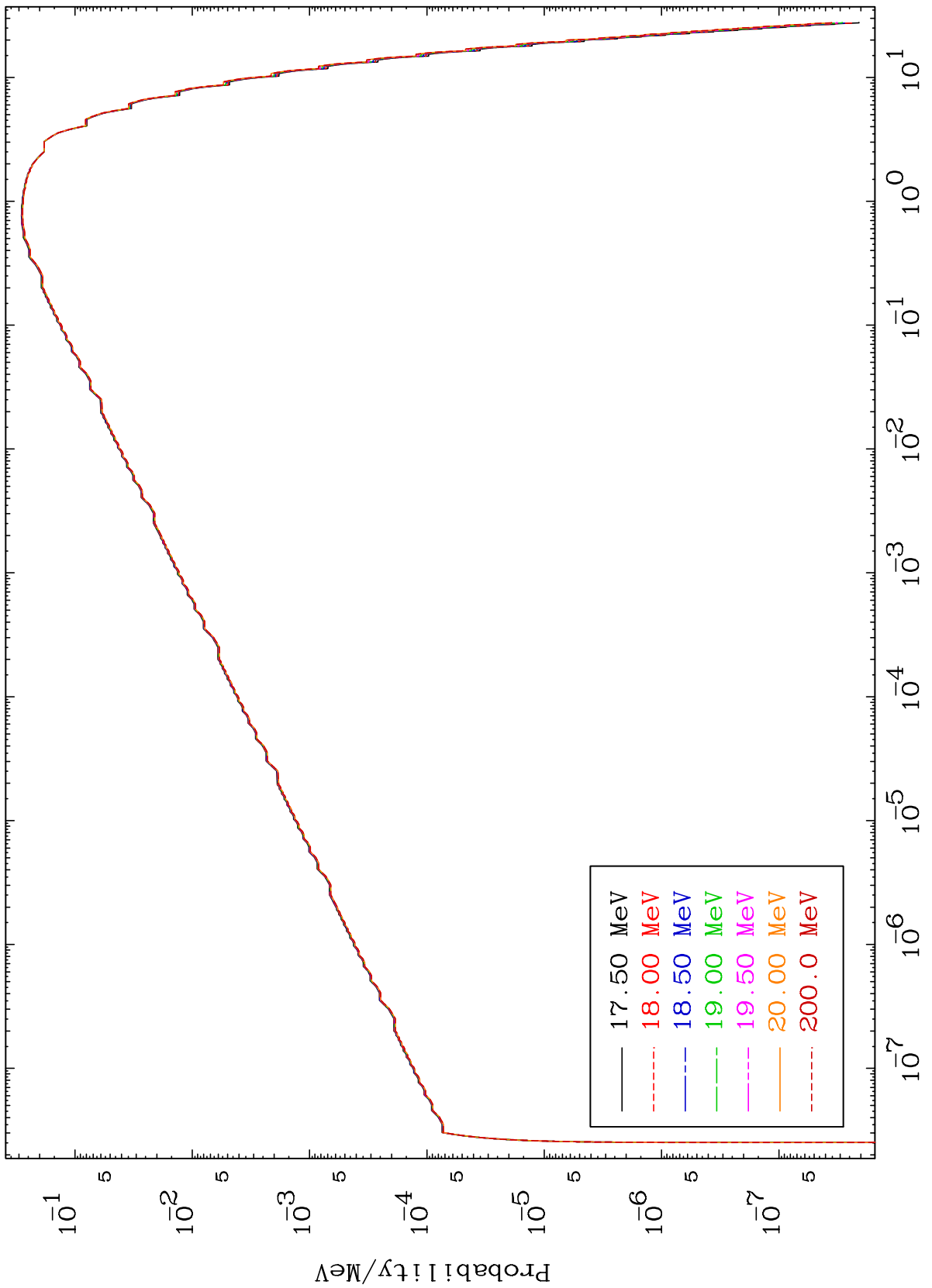
97-Bk-237



Fission
Energy Distribution



Fission Energy Distribution



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(n,2p)

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Radionuclide Production Cross Section

