

Program EVALPLOT  
(Version 2017-1)

by

Dermott E. Cullen  
(Present Contact Information)

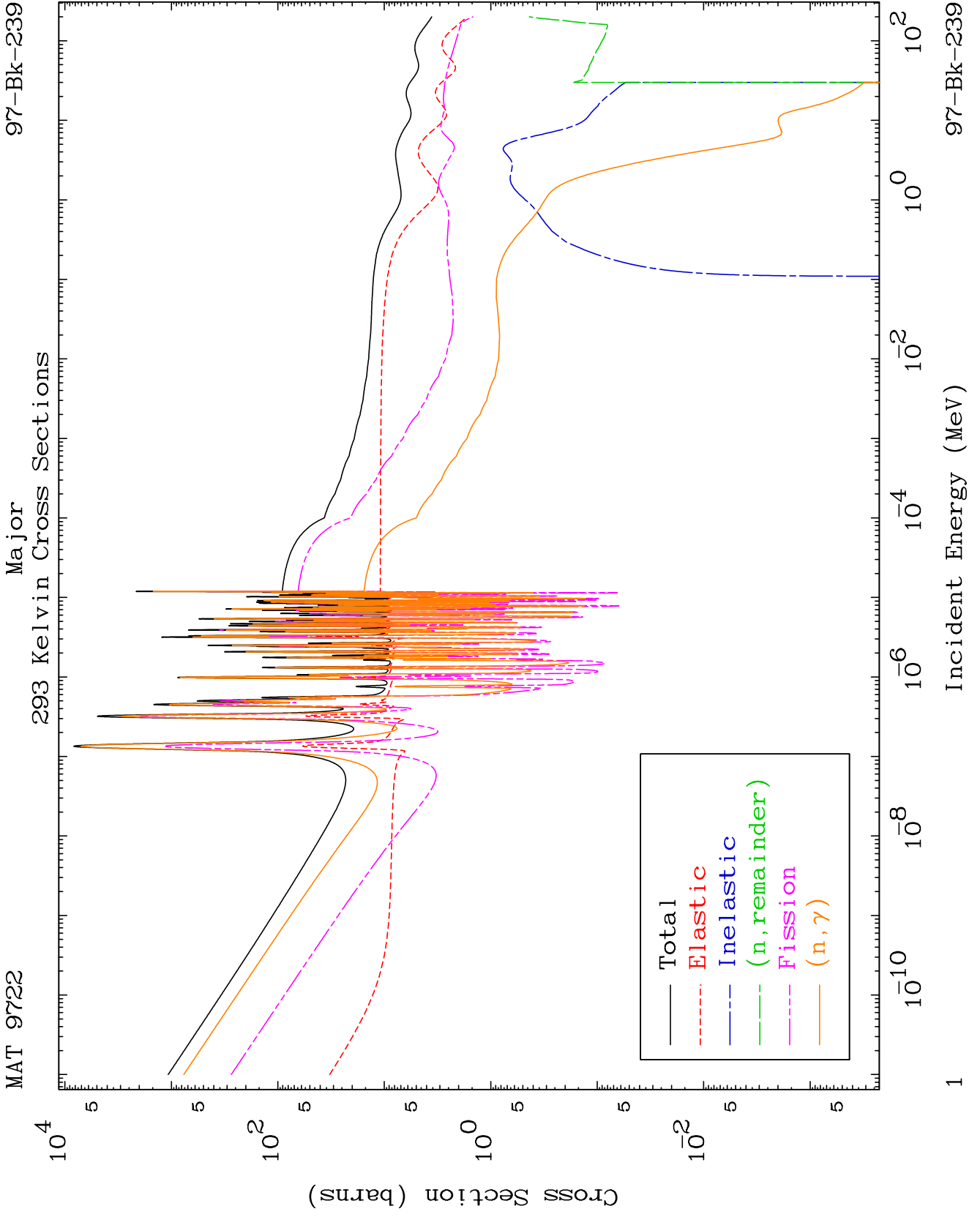
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Web:redcullen1.net/HOMEPAGE.NEW

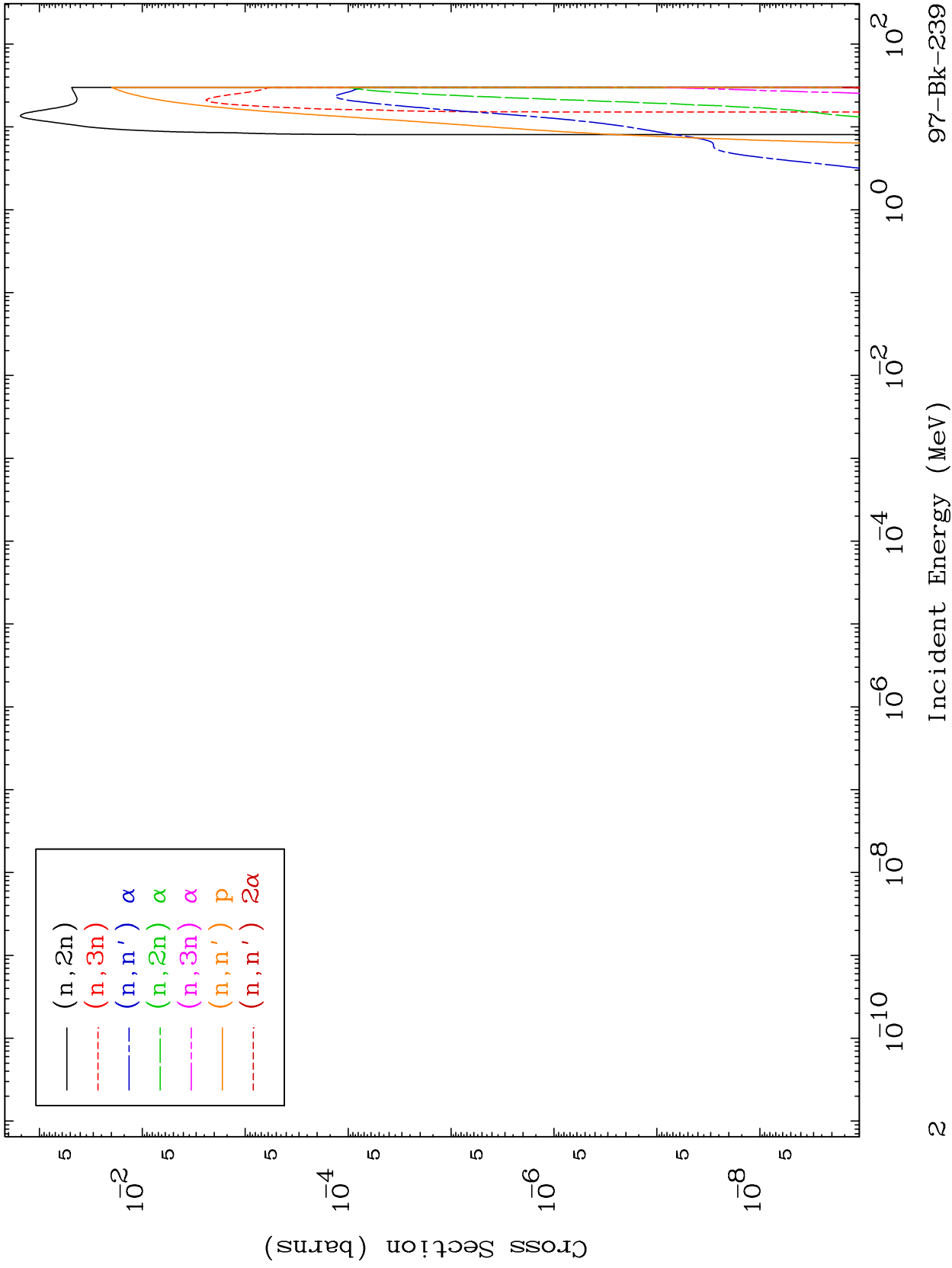
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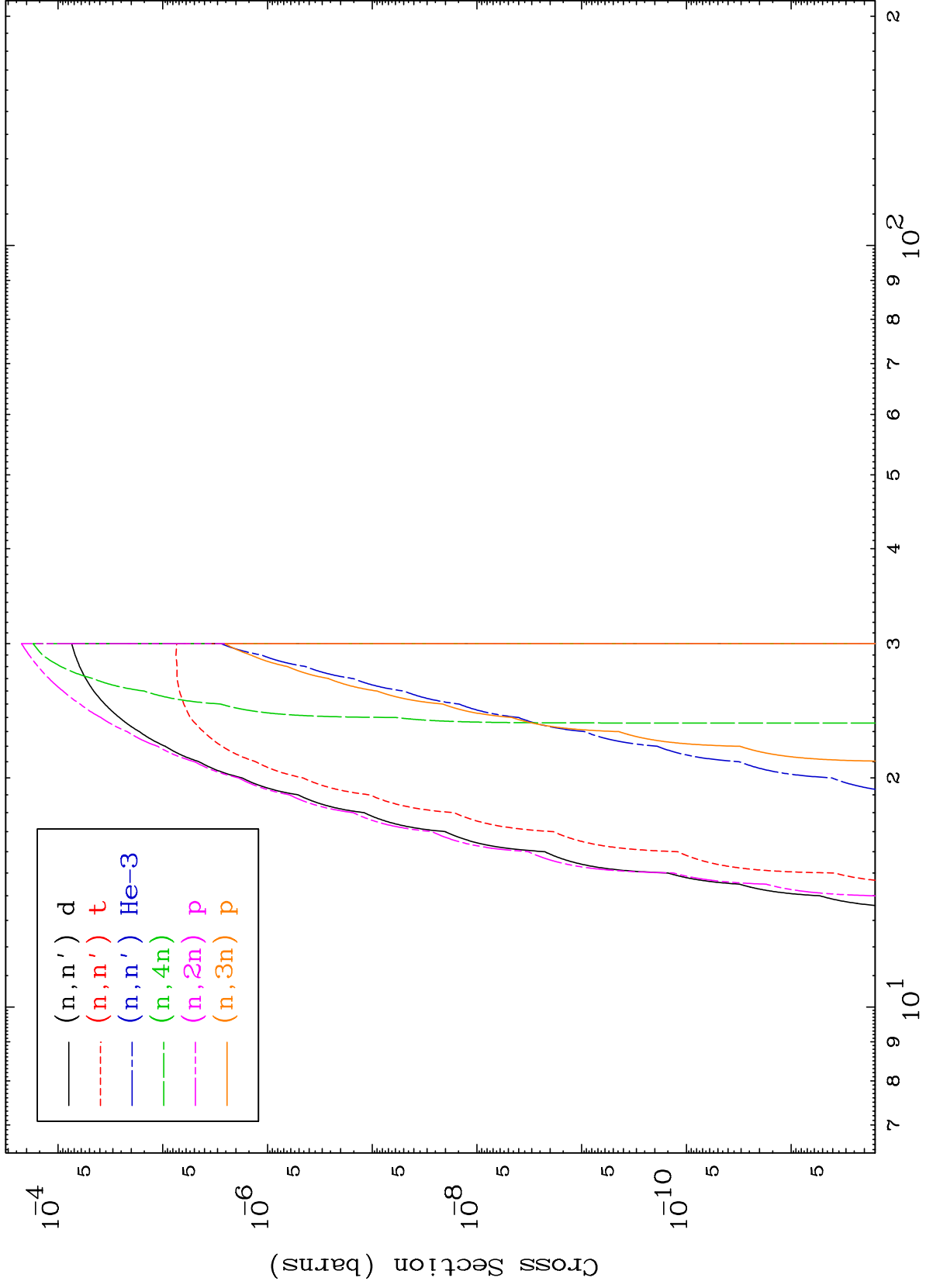


MAT 9722

Neutron Production  
293 Kelvin Cross Sections

97-Bk-239

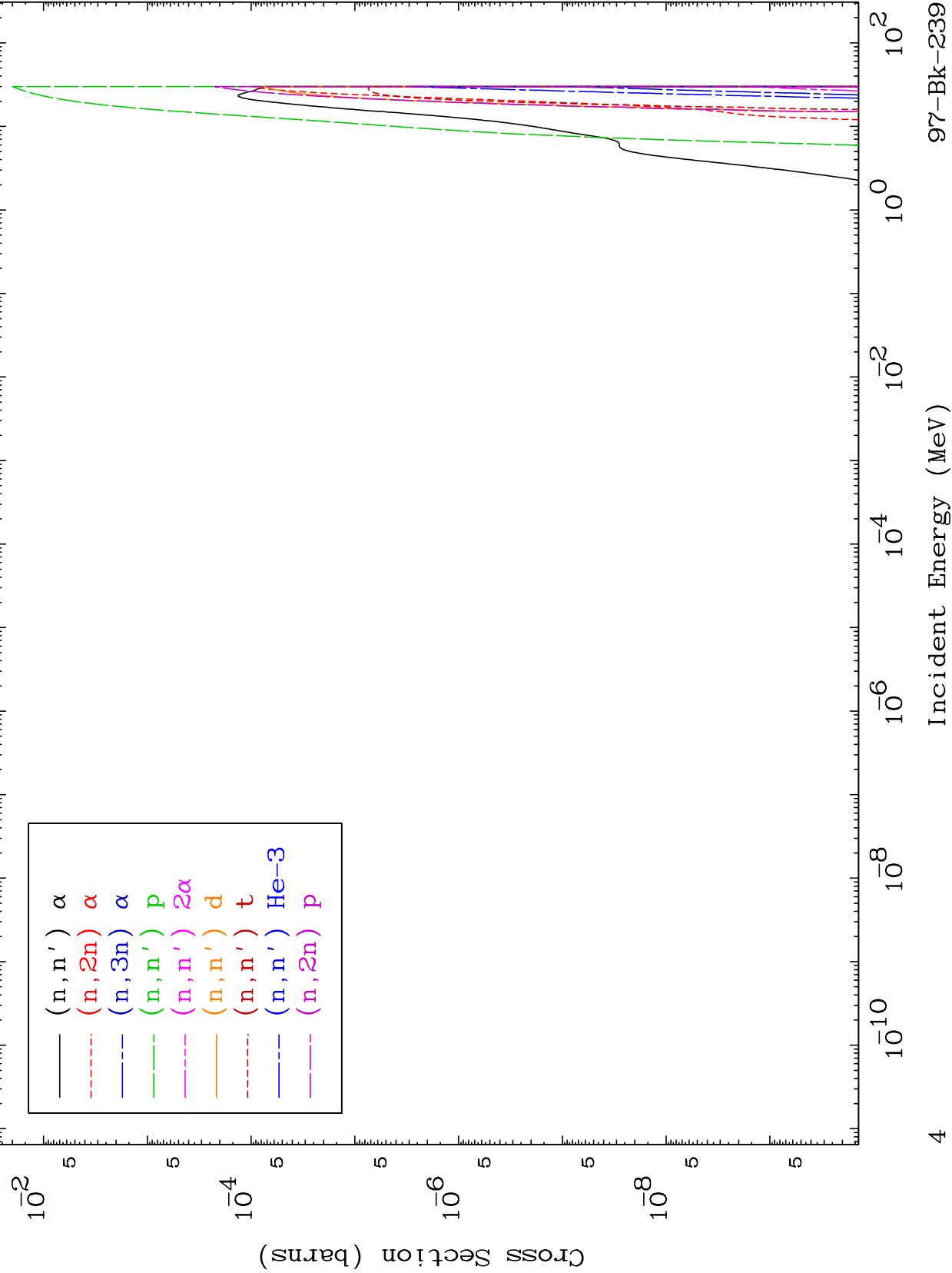




MAT 9722

Charged Particle  
293 Kelvin Cross Sections

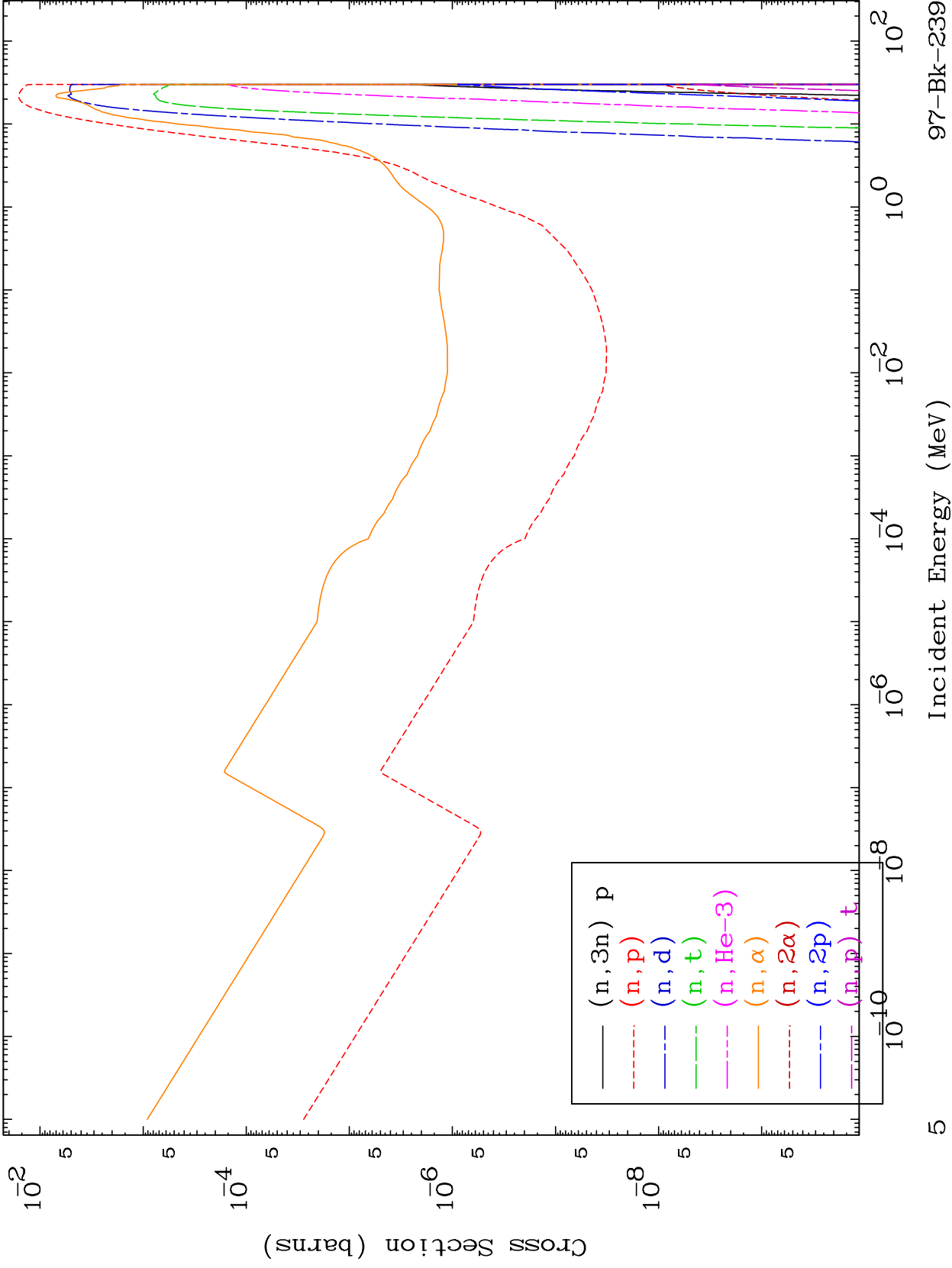
97-Bk-239



MAT 9722

Charged Particle  
293 Kelvin Cross Sections

97-Bk-239

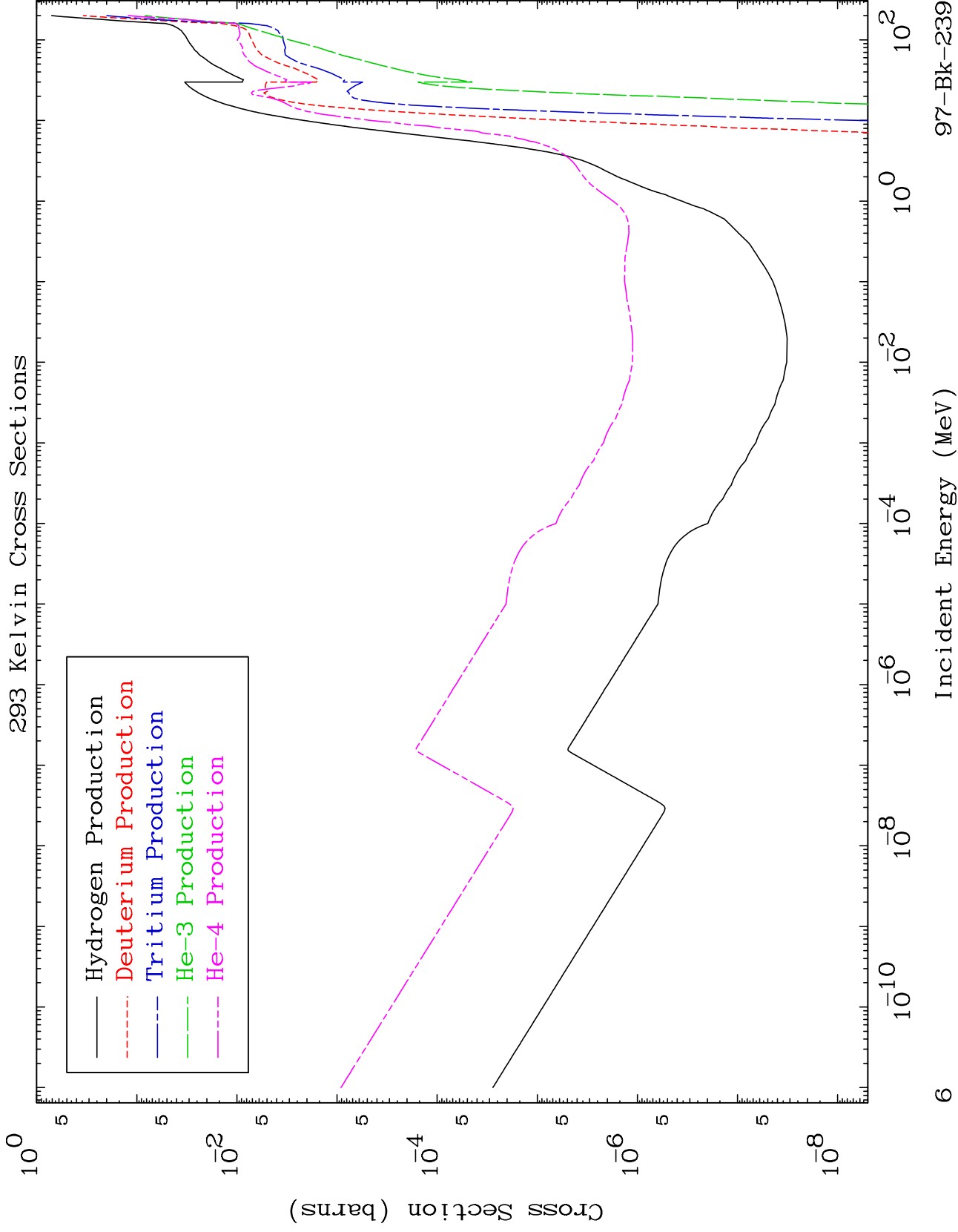


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MAT 9722

Particle Production  
293 Kelvin Cross Sections

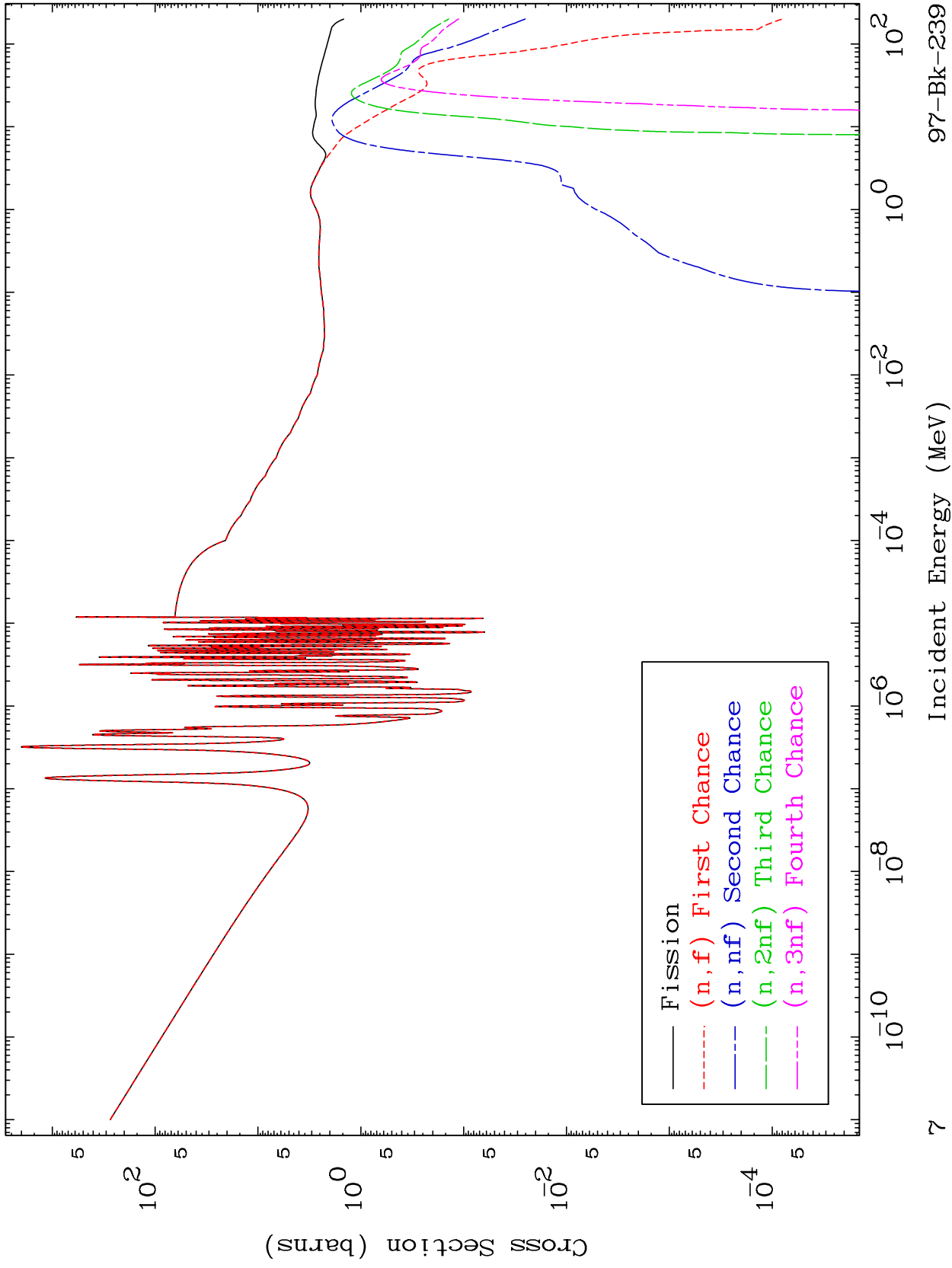
97-Bk-239



MAT 9722

Fission  
293 Kelvin Cross Sections

97-Bk-239

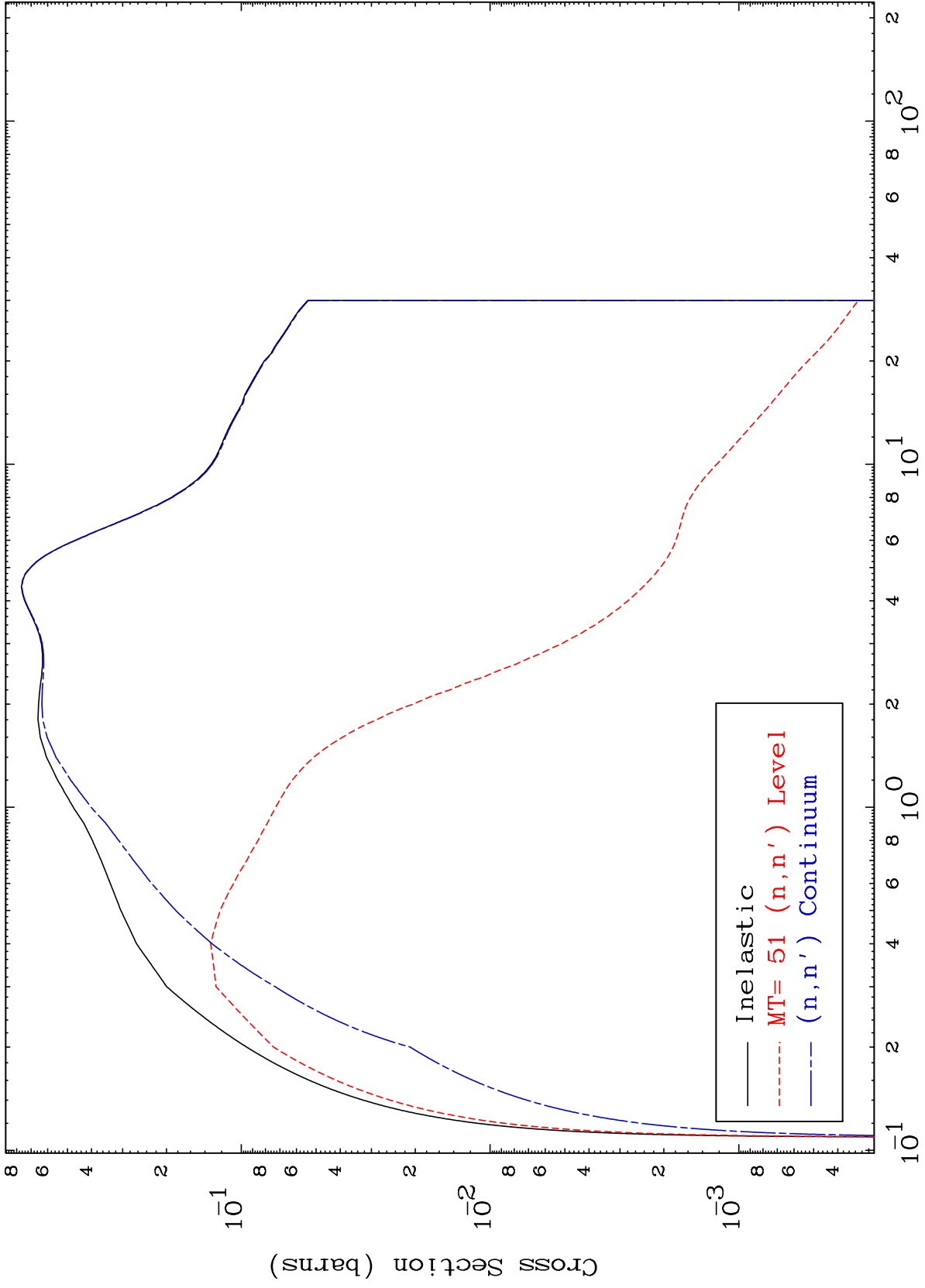




MAT 9722

293 (n,n') Level

97-Bk-239



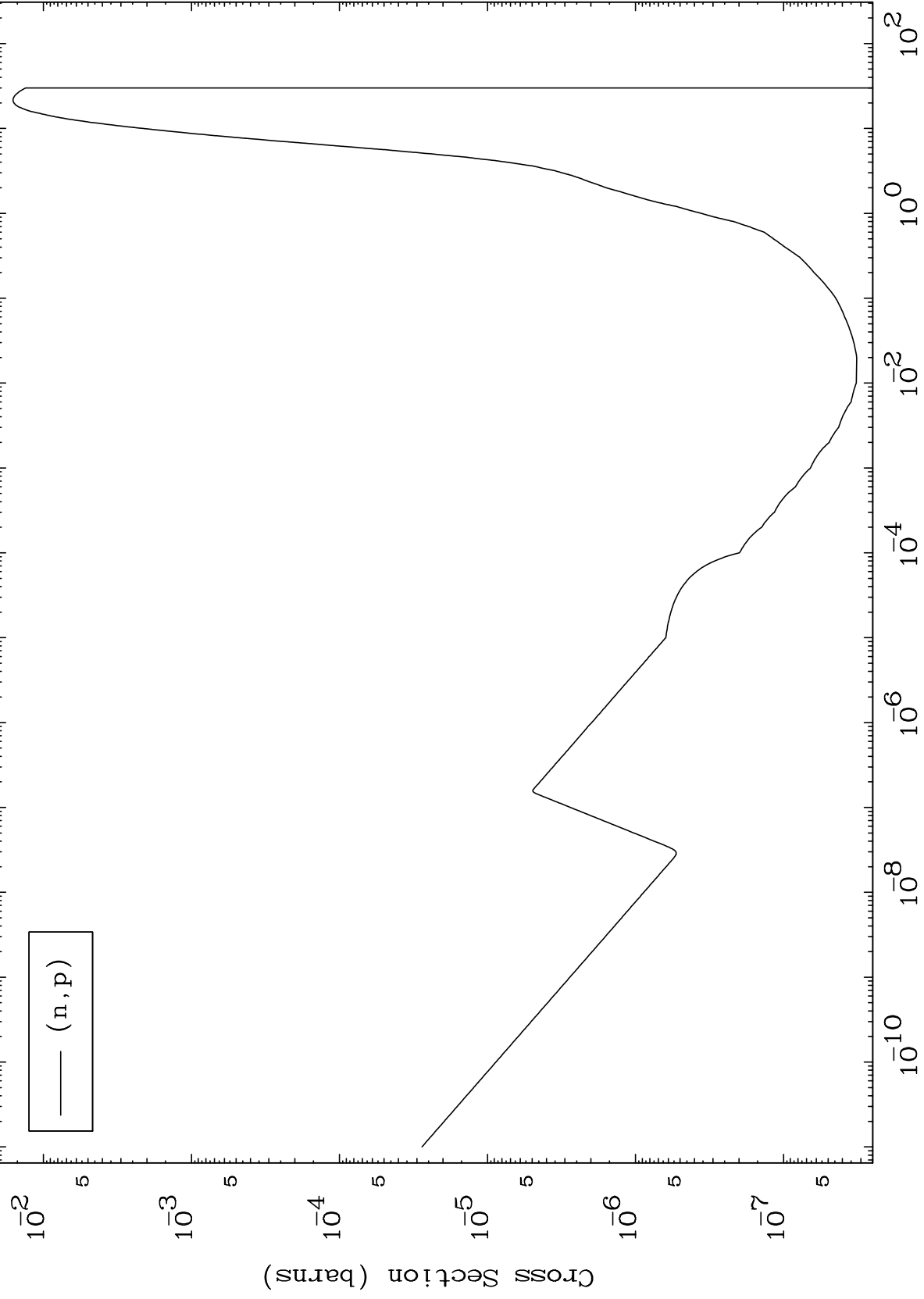
Incident Energy (MeV)

97-Bk-239

MAT 9722

(n,p) Levels  
293 Kelvin Cross Sections

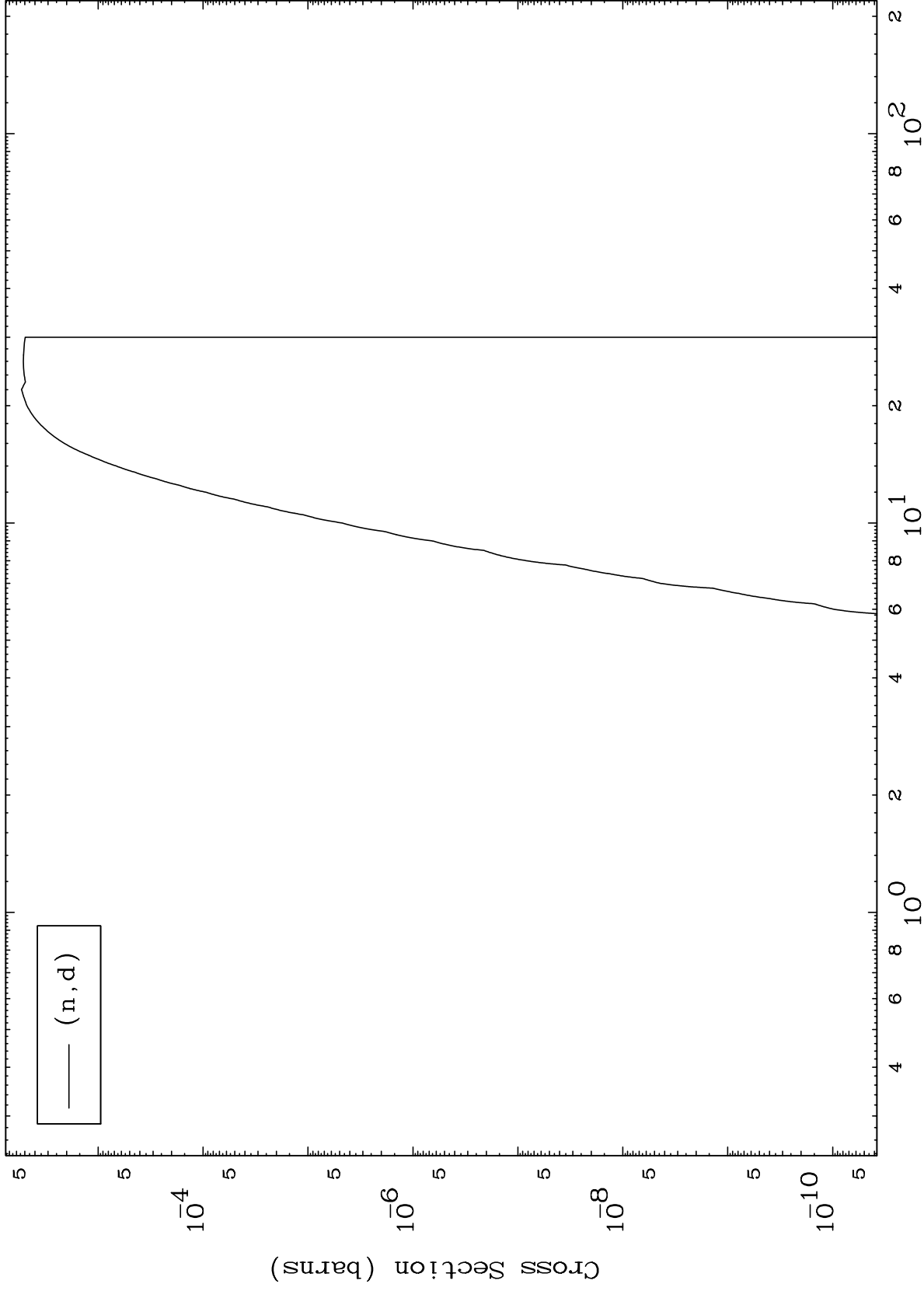
97-Bk-239



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-239



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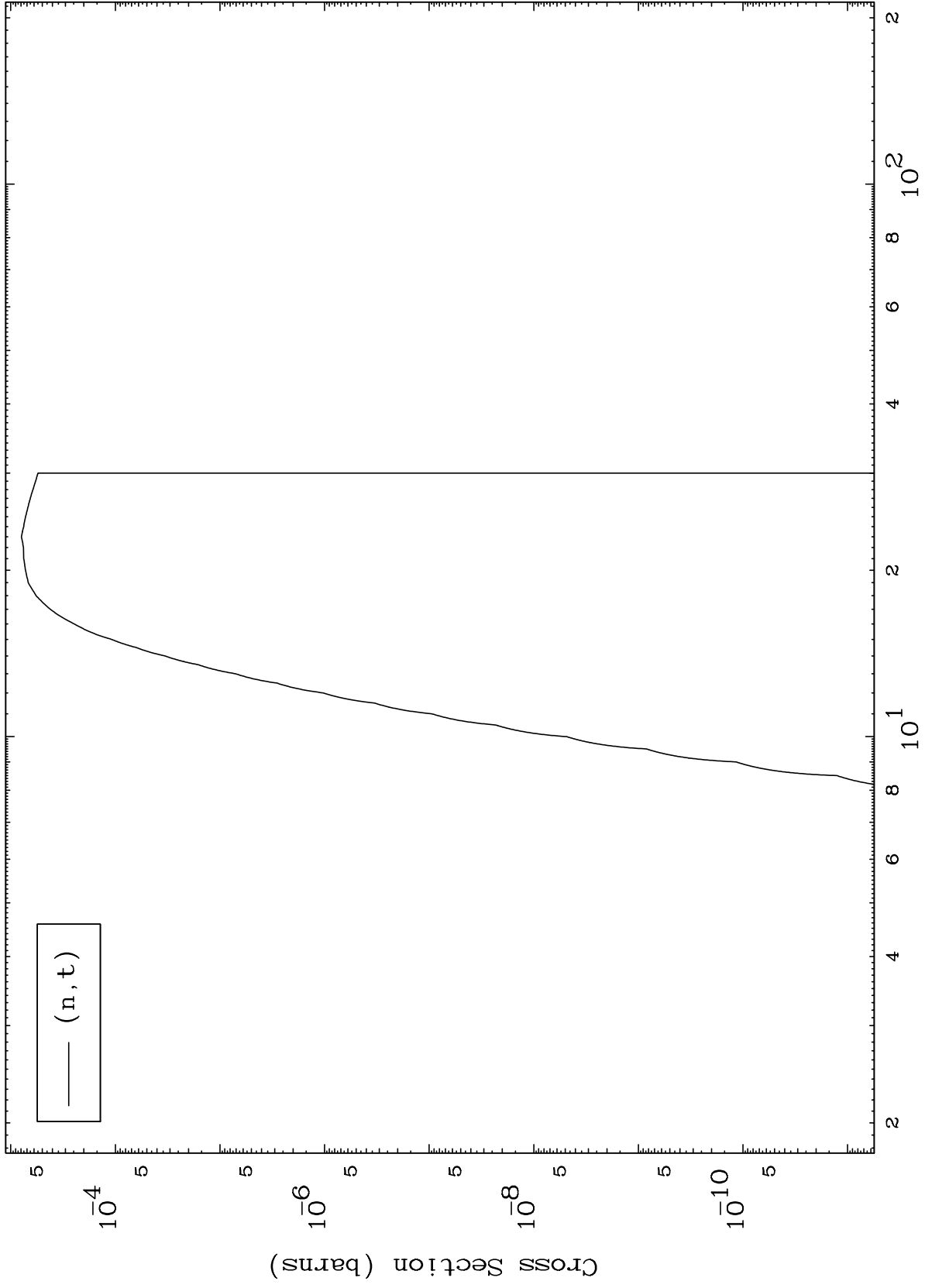
Incident Energy (MeV)

97-Bk-239

MAT 9722

(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-239



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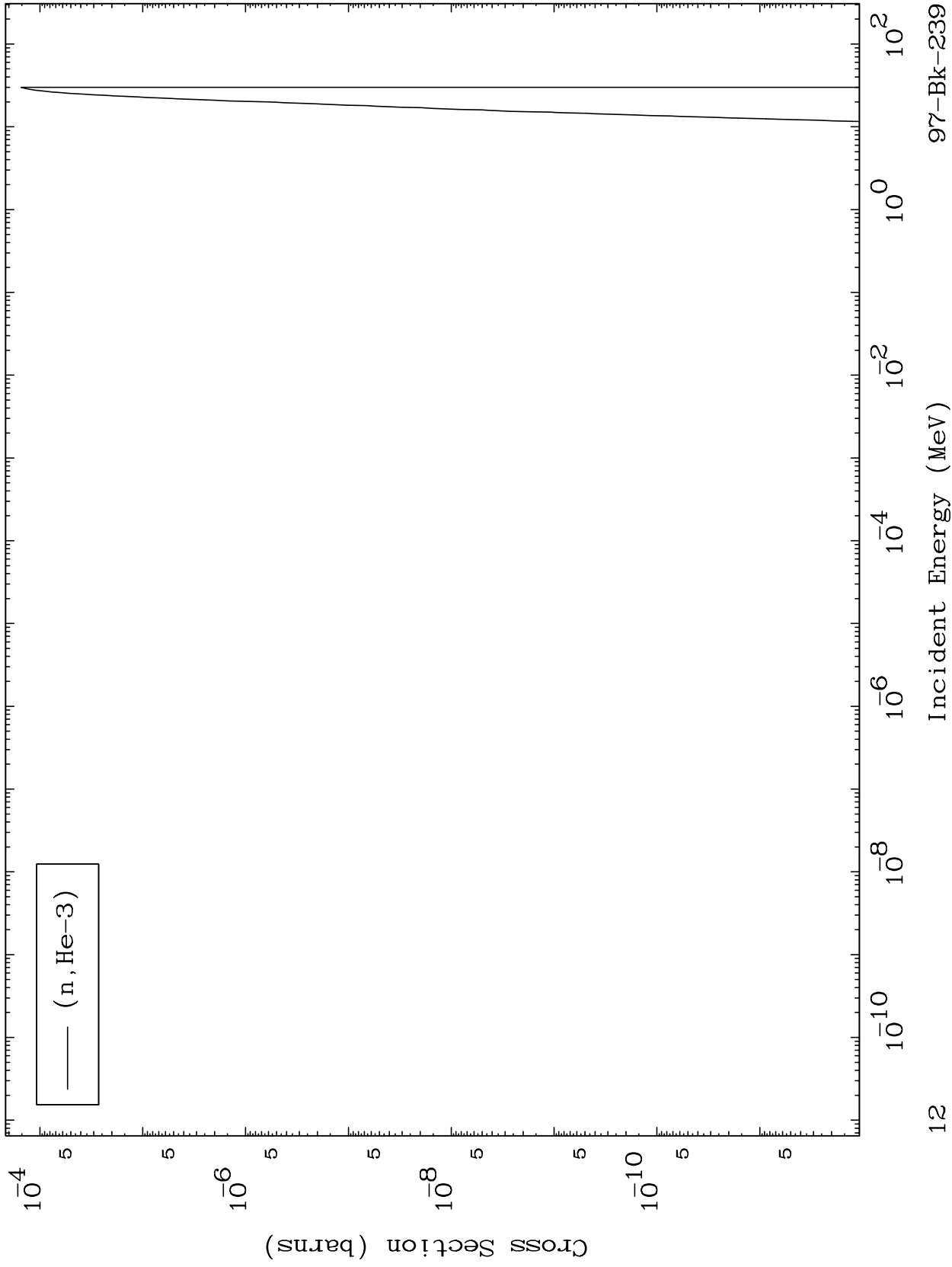
Incident Energy (MeV)

97-Bk-239

MAT 9722

(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-239



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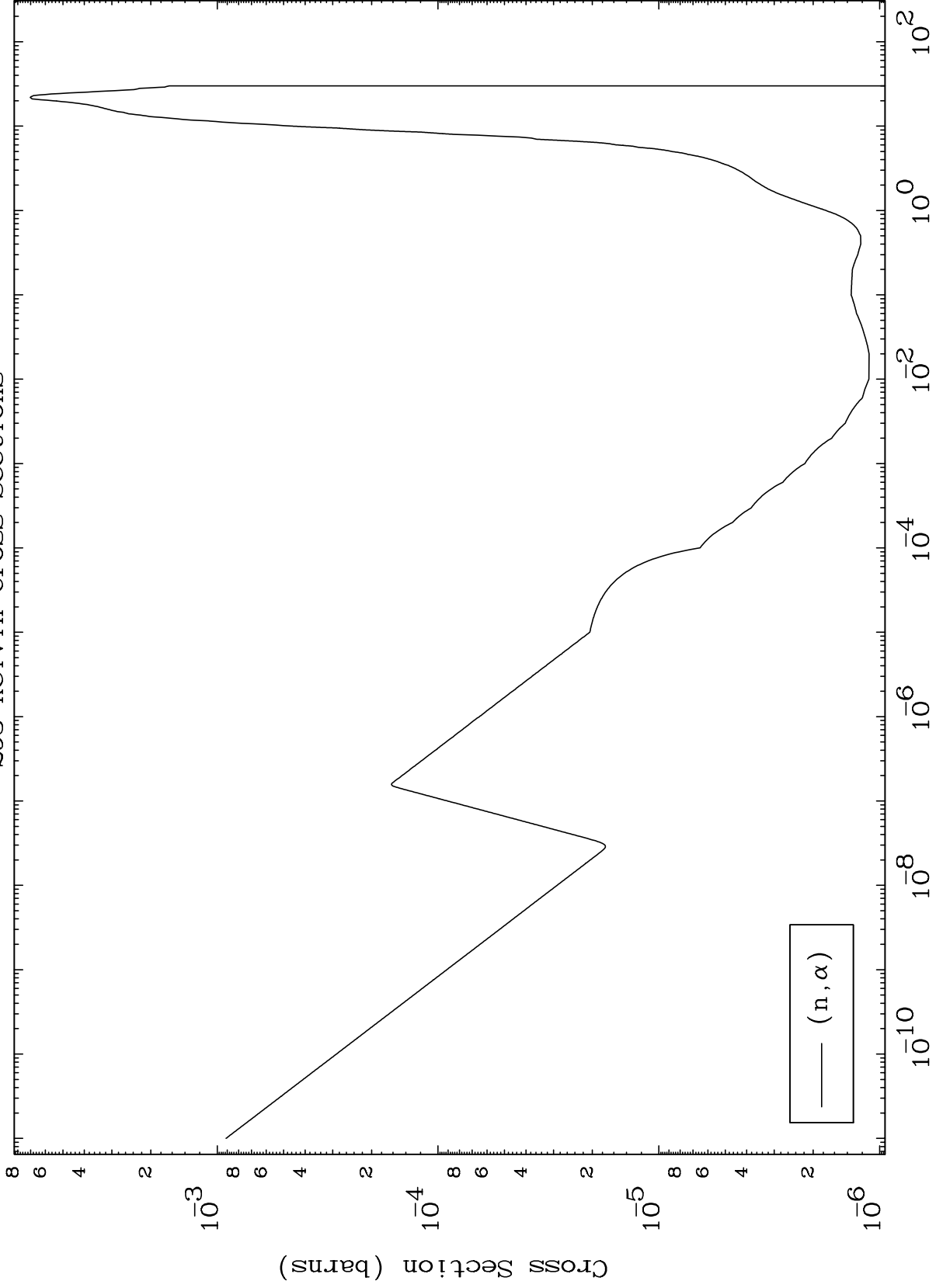
Incident Energy (MeV)

97-Bk-239

MAT 9722

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

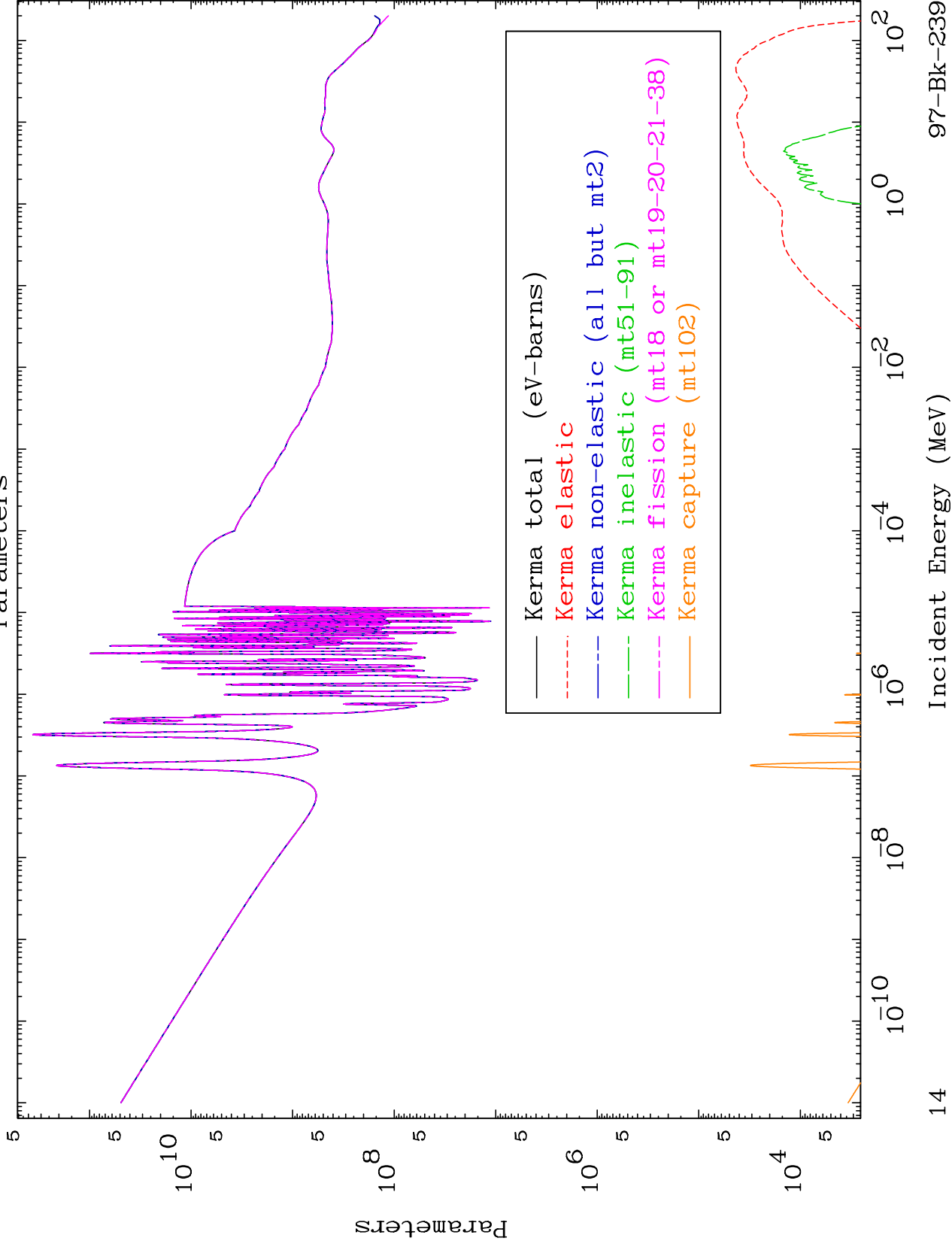
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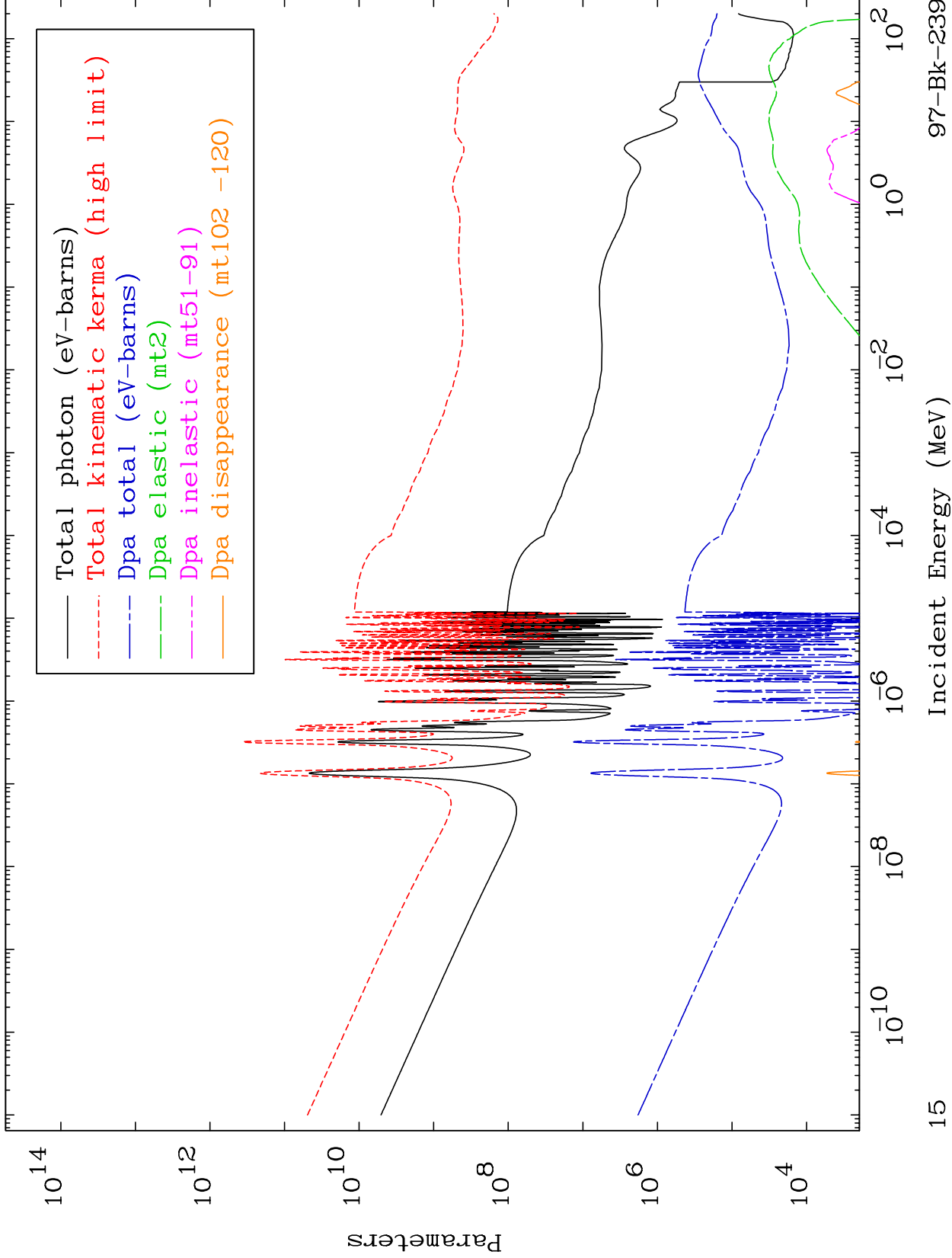


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Incident Energy (MeV)

97-Bk-239



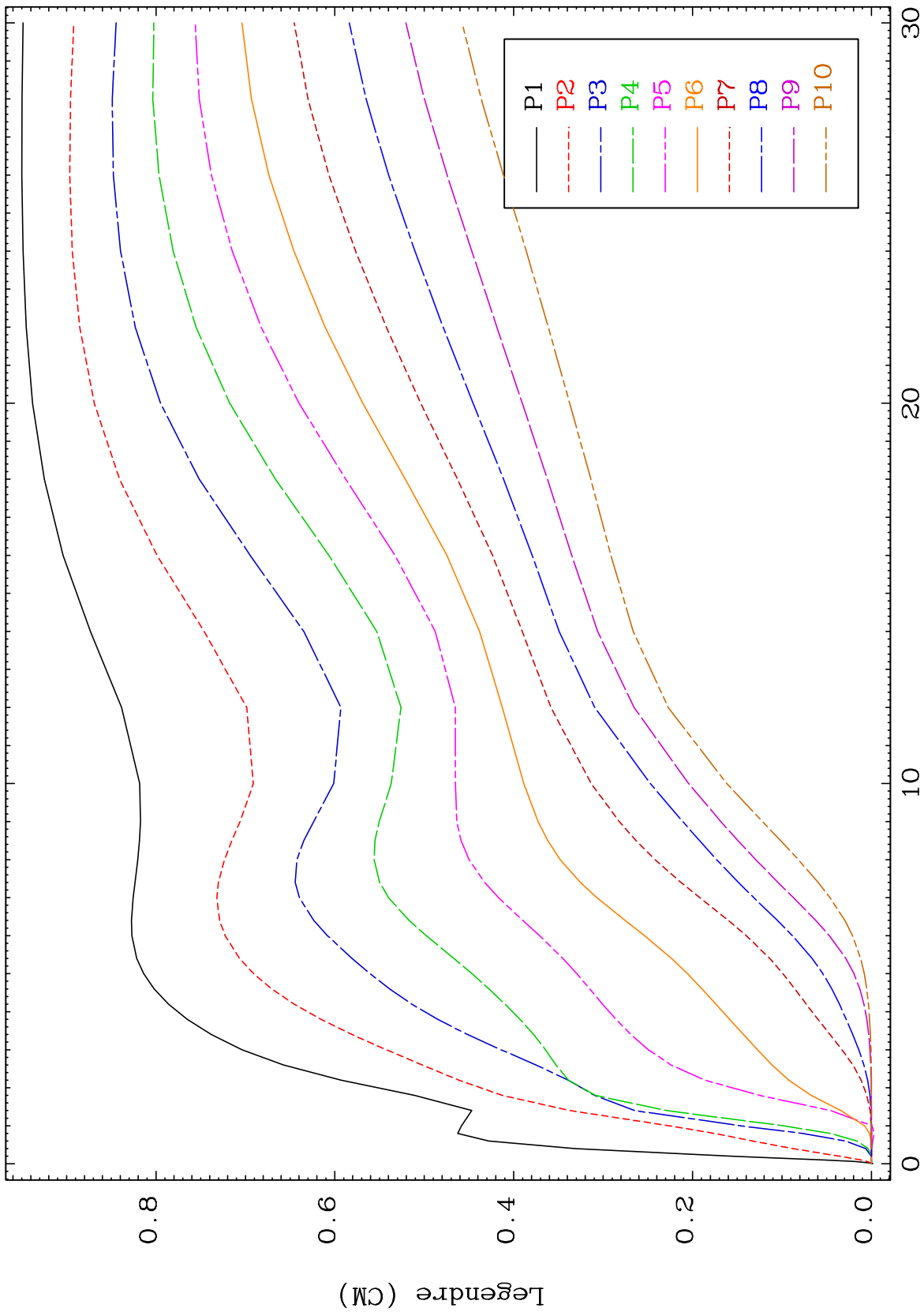




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Elastic Legendre Coefficients

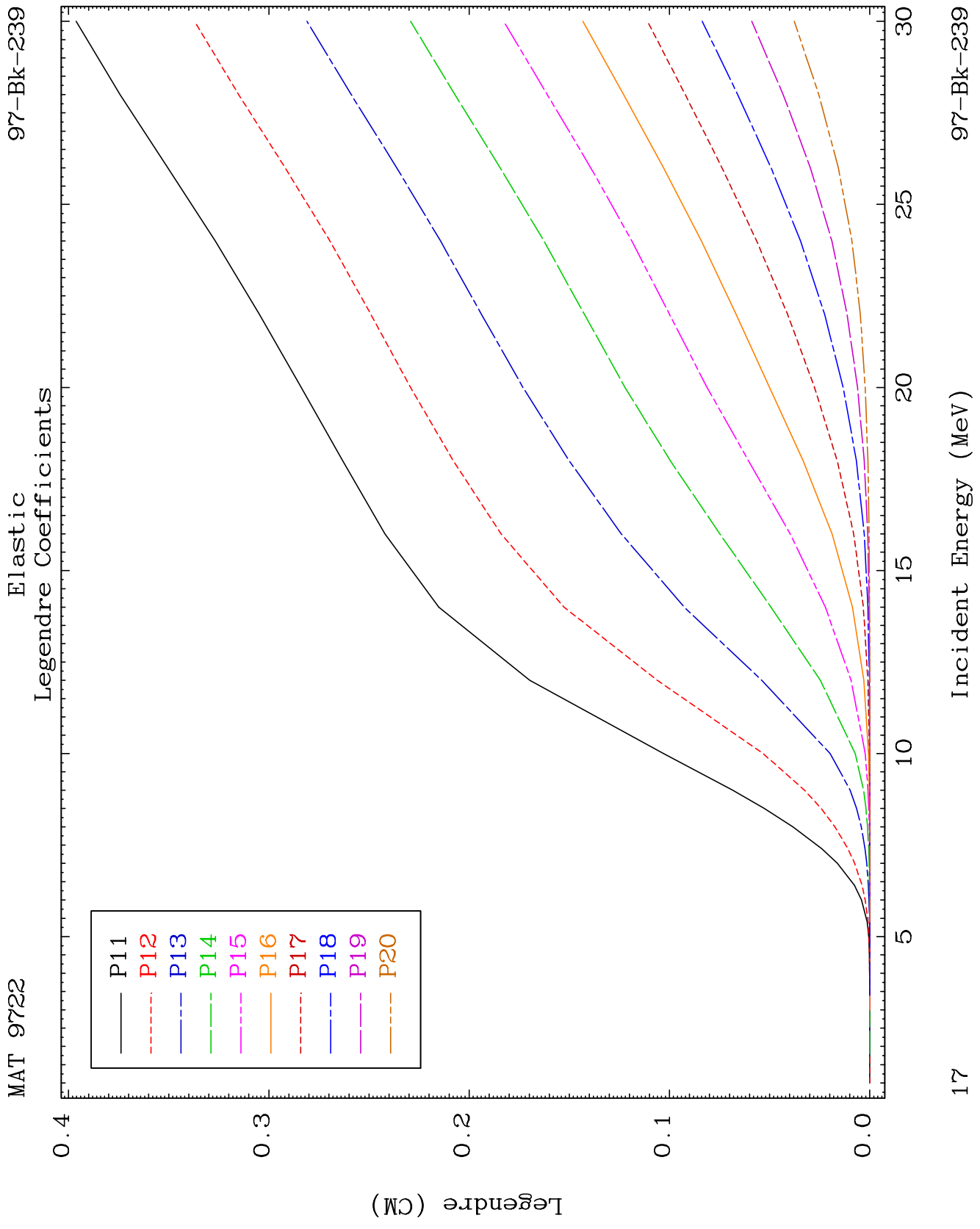
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Incident Energy (MeV)

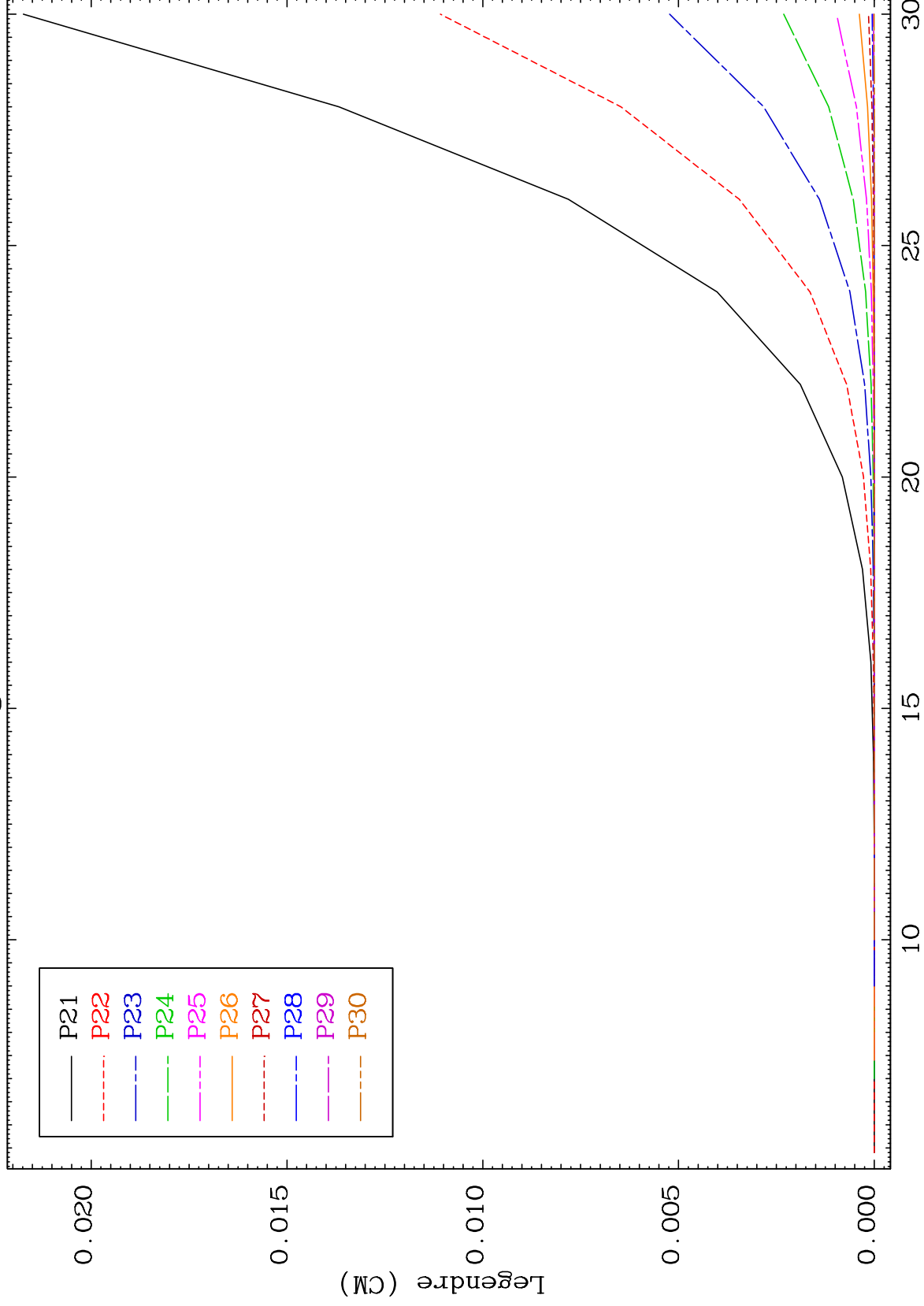
97-Bk-239



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### Elastic Legendre Coefficients

97-Bk-239



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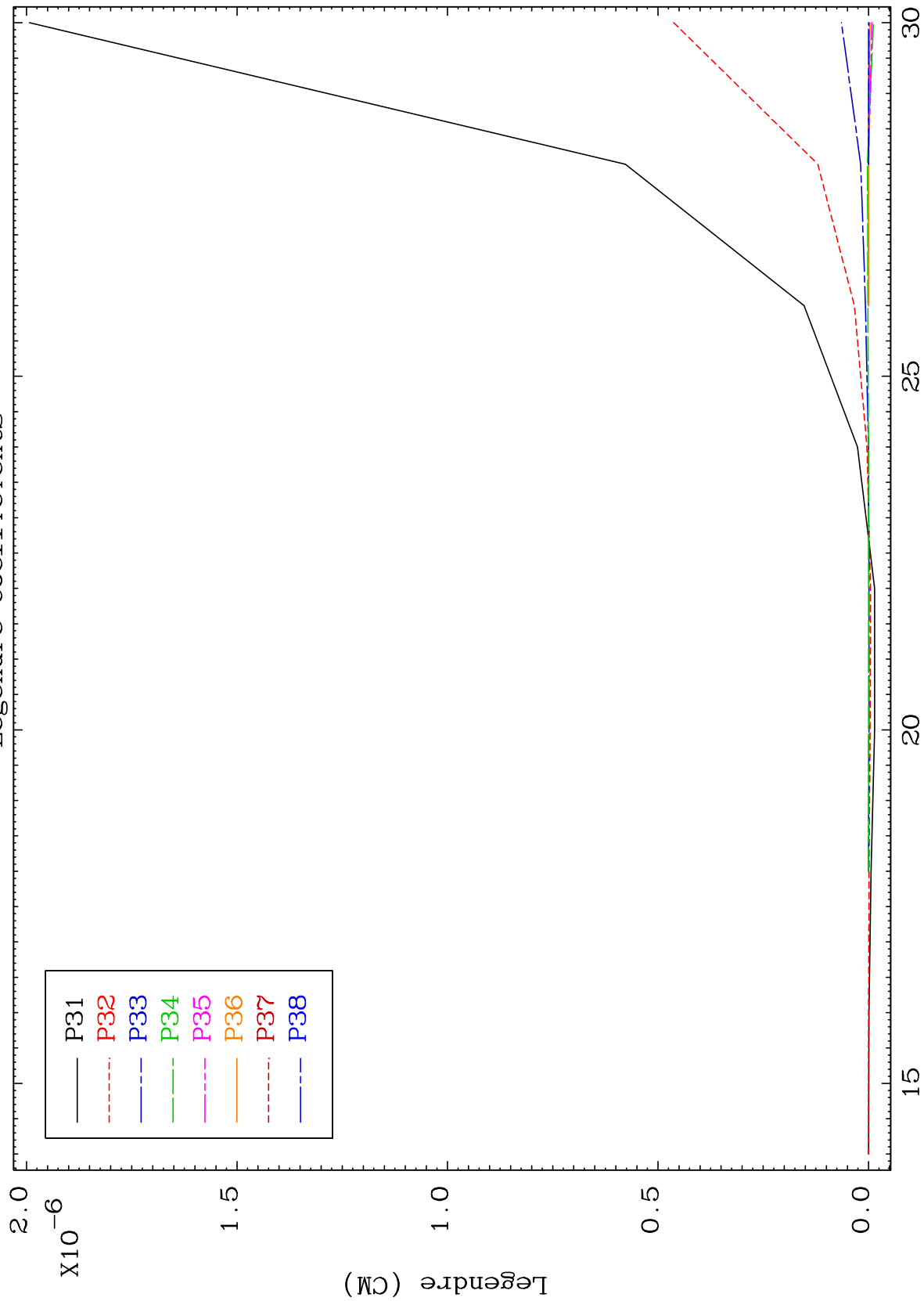
Incident Energy (MeV)

97-Bk-239

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Elastic Legendre Coefficients

97-Bk-239

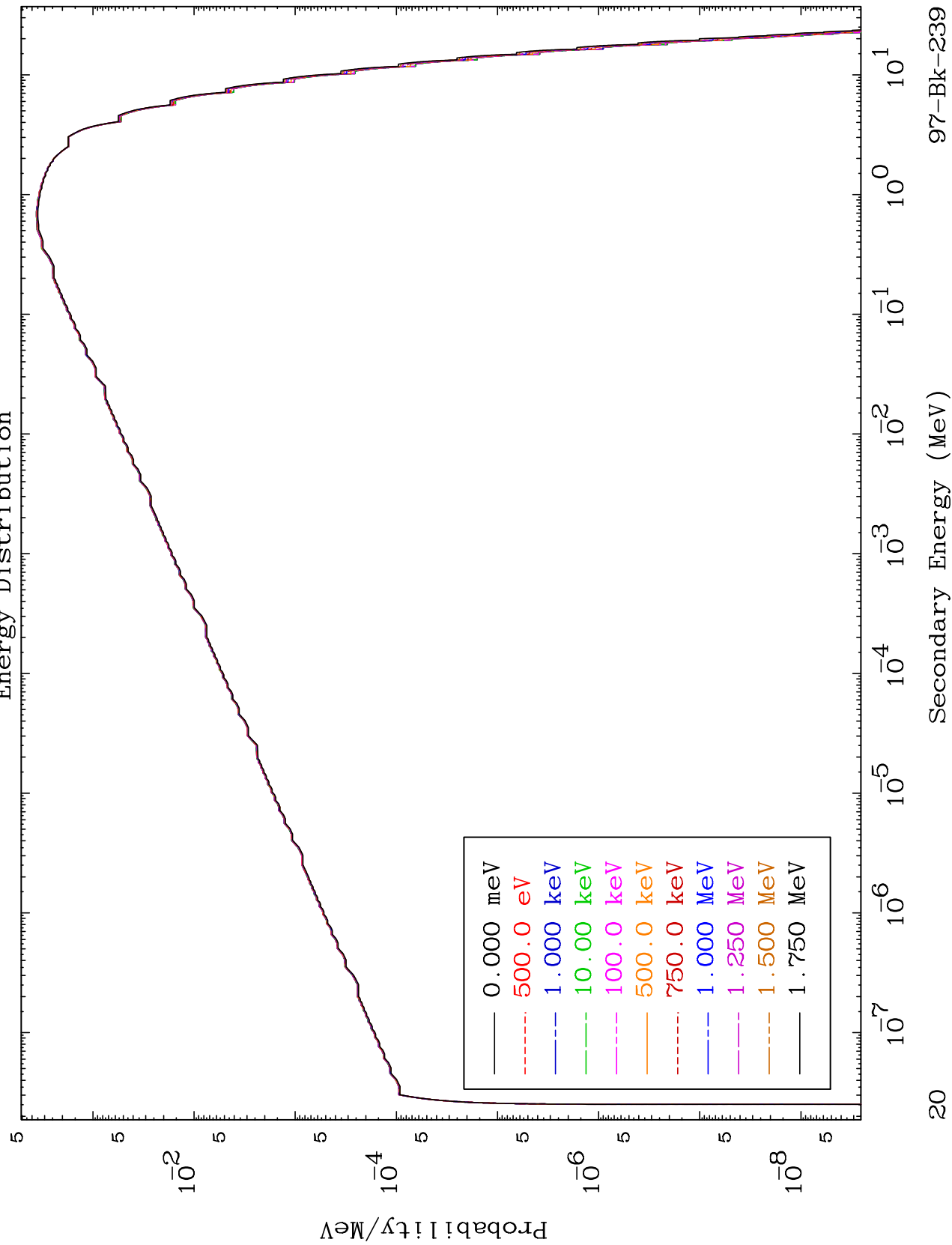


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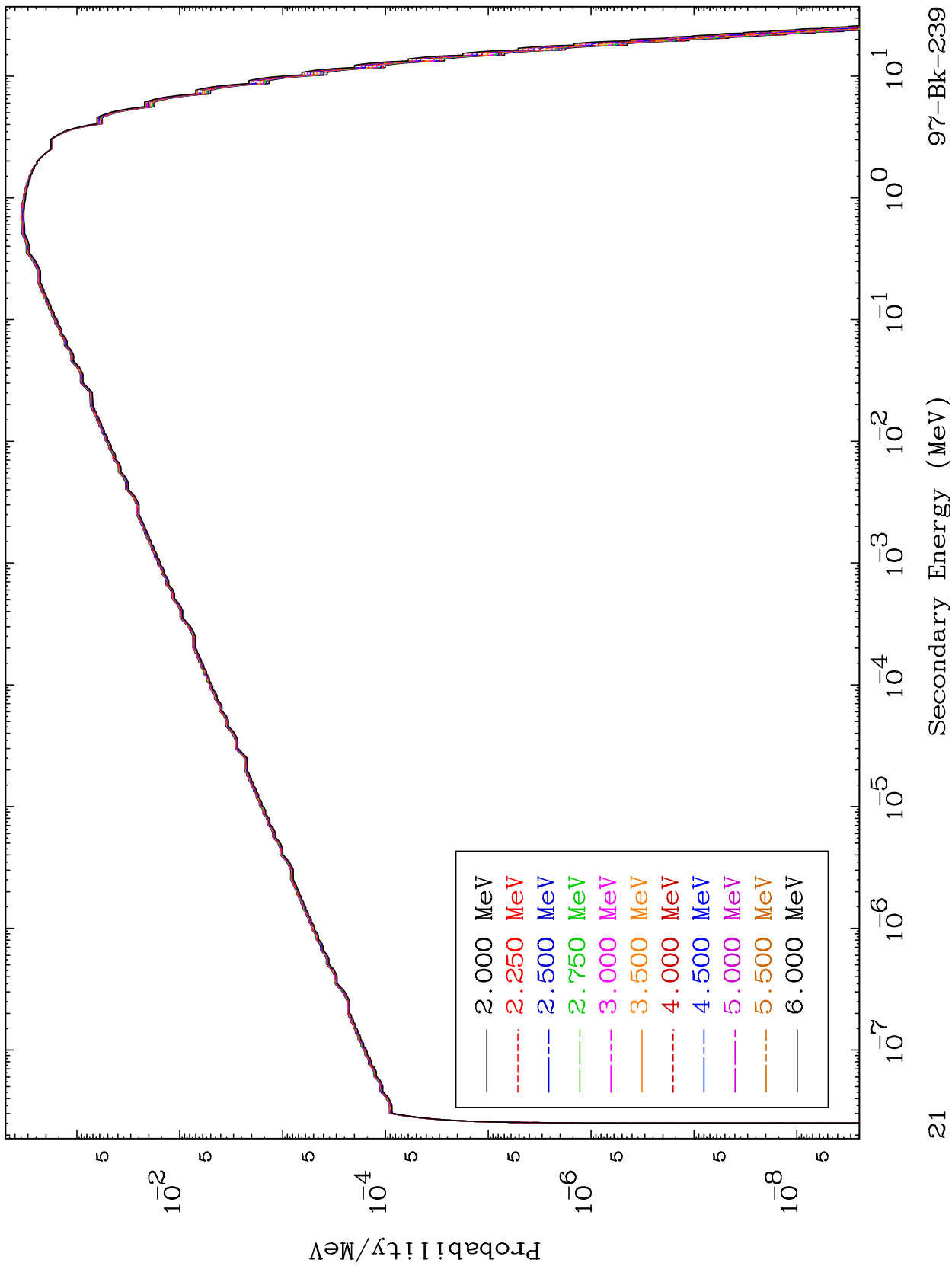
Incident Energy (MeV)

97-Bk-239

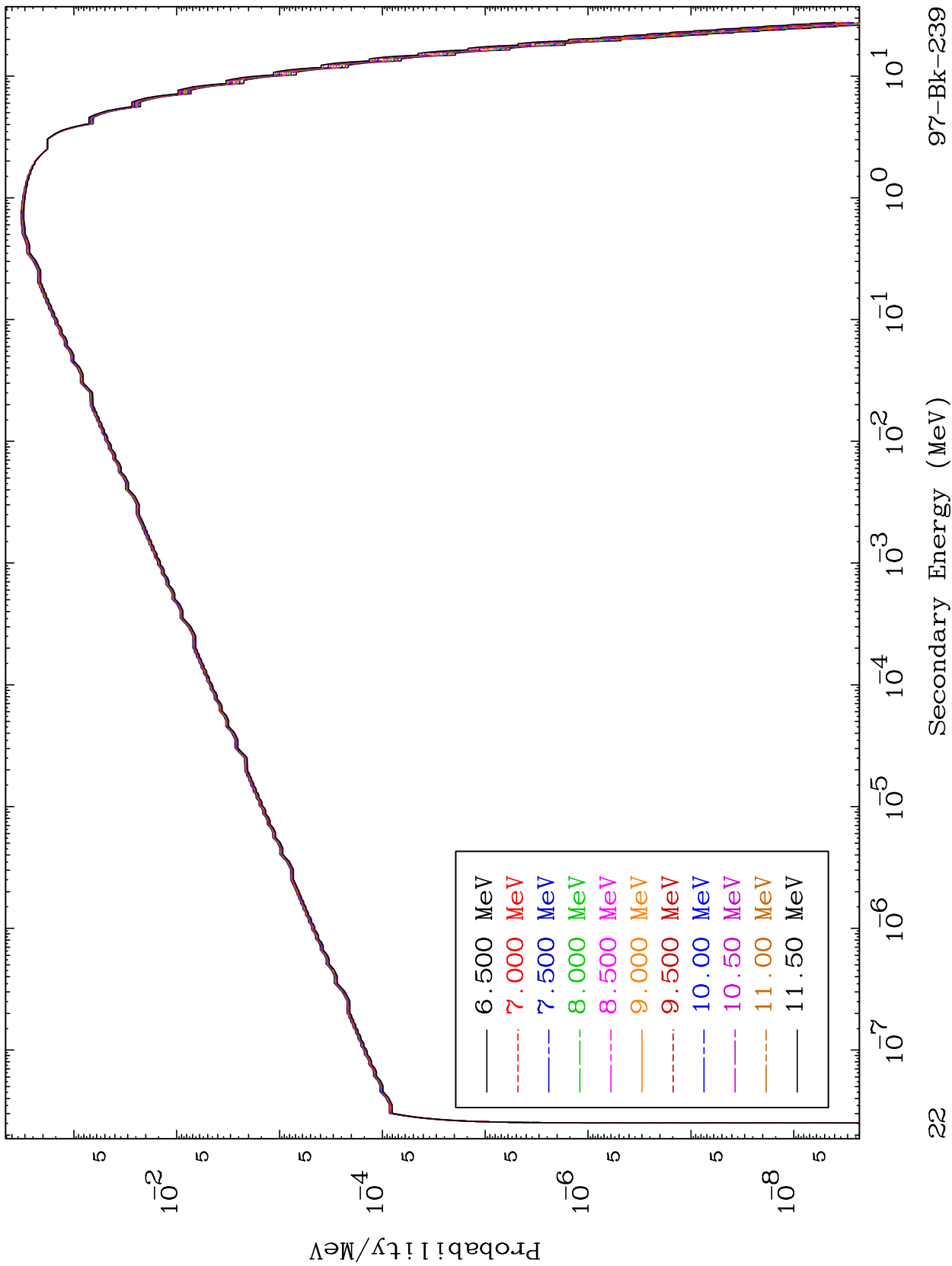
Fission Energy Distribution



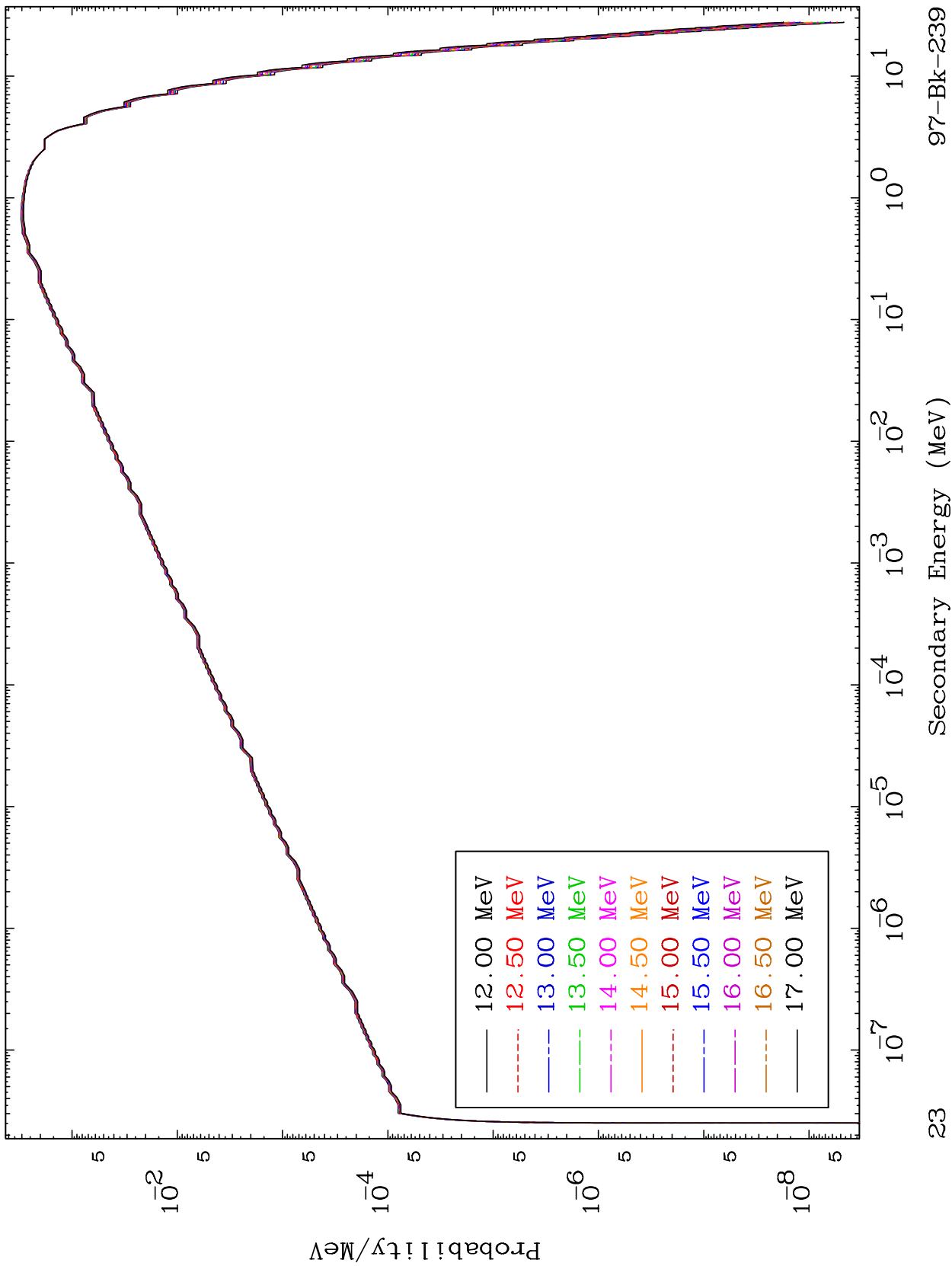
Fission  
Energy Distribution



Fission  
Energy Distribution

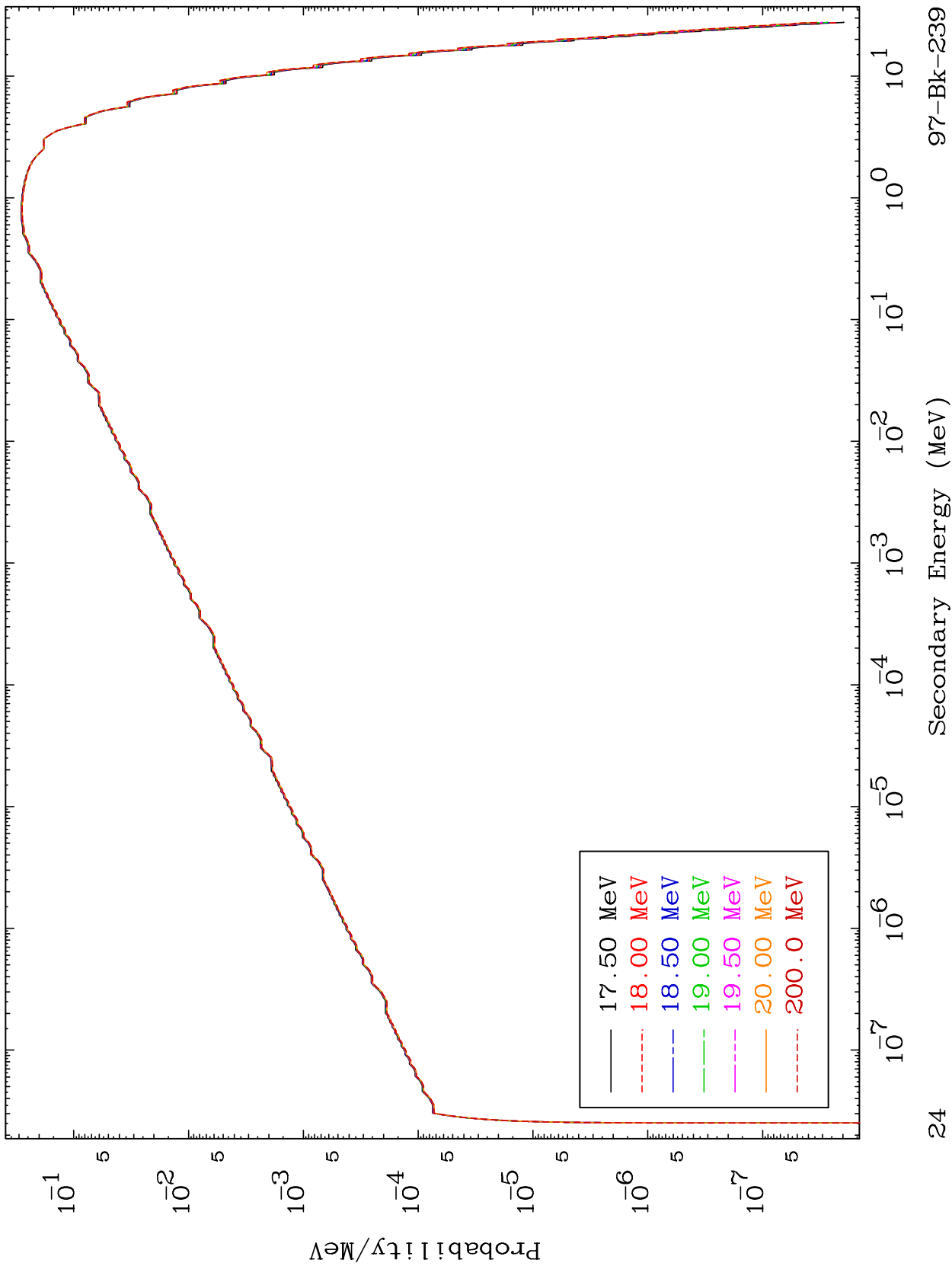


Fission  
Energy Distribution





### Fission Energy Distribution

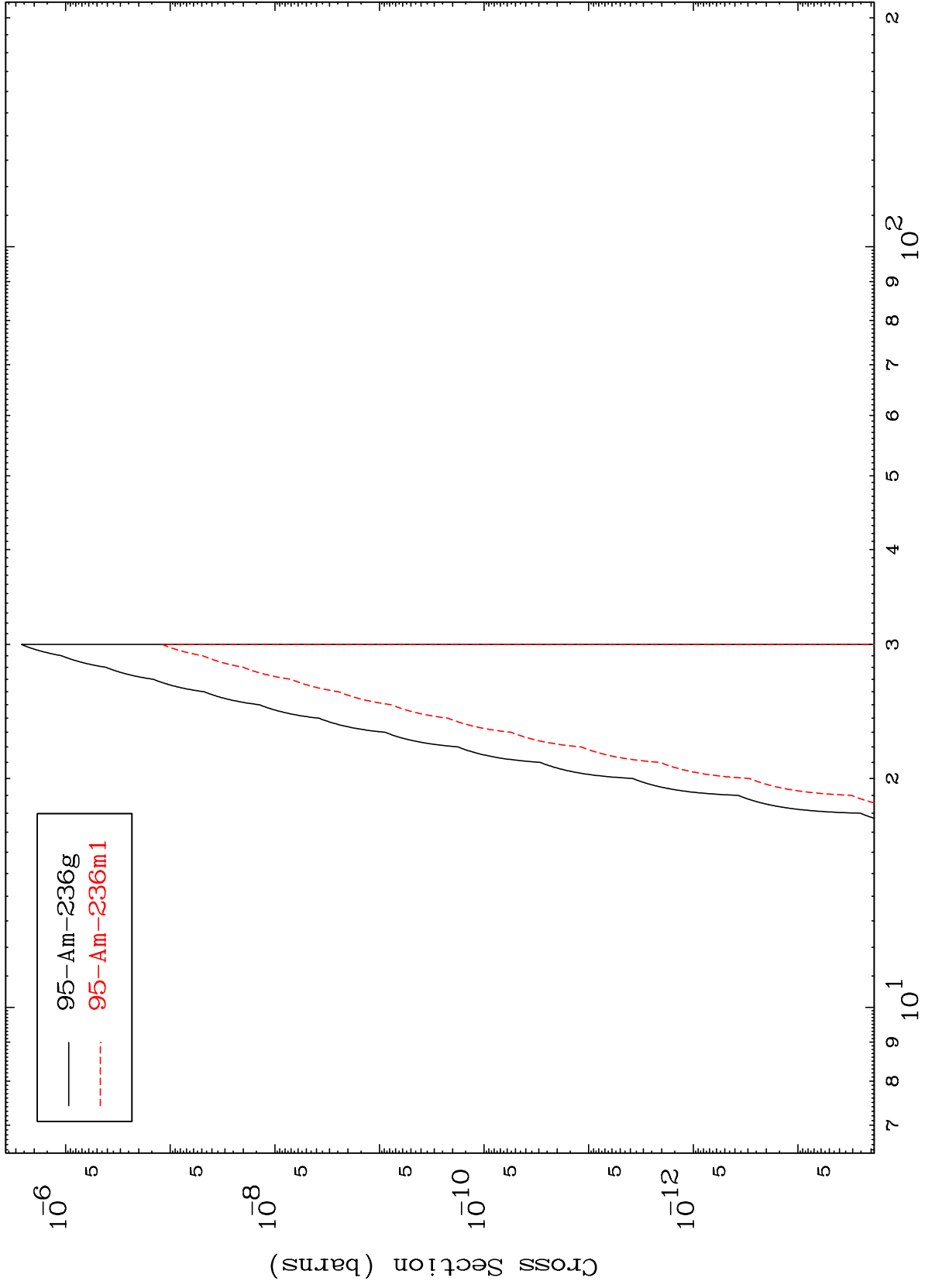


MAT 9722

(n,n') He-3

97-Bk-239

Radionuclide Production Cross Section



— 95-Am-236g  
- - - 95-Am-236m1

25

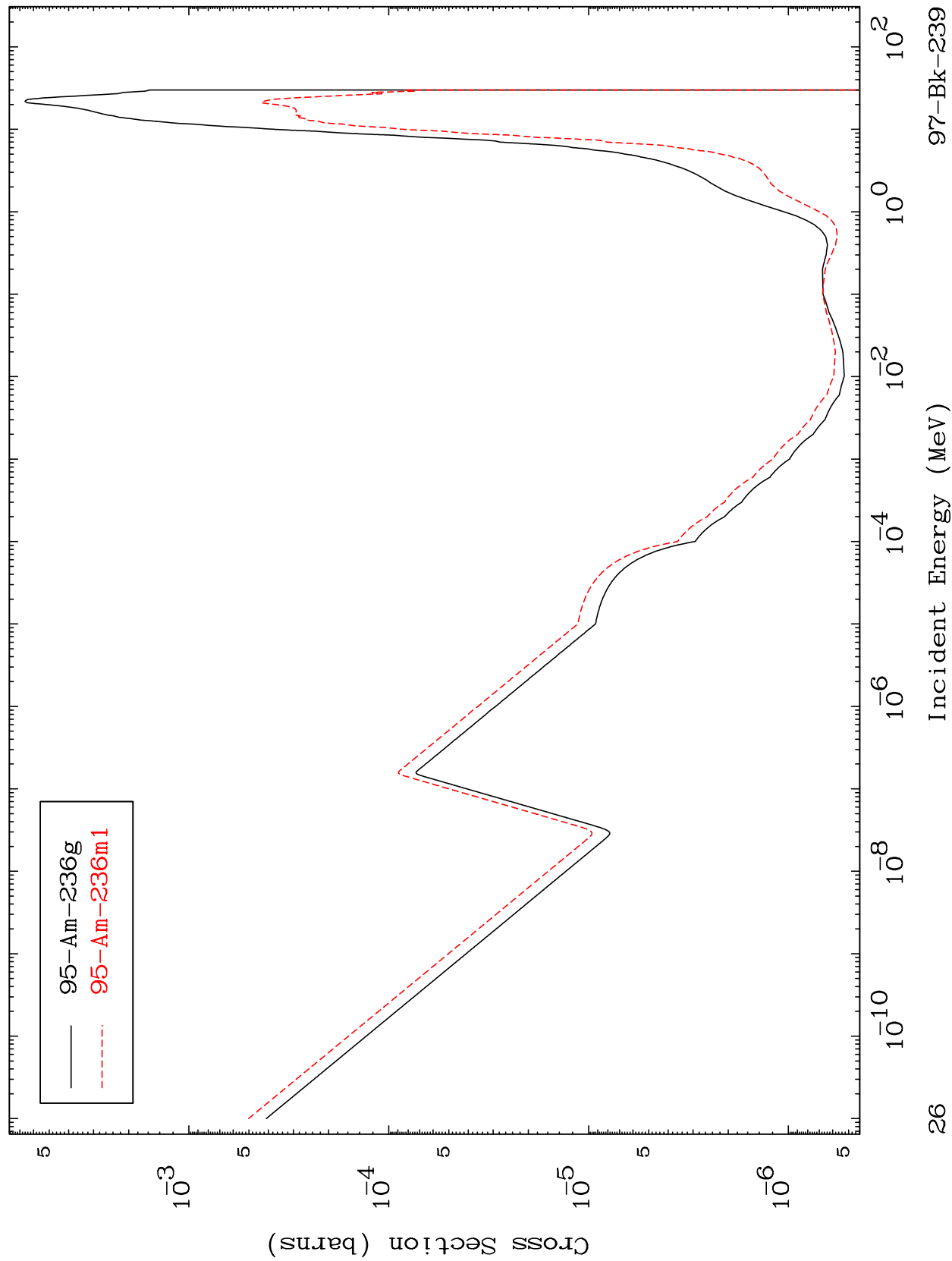
Incident Energy (MeV)

97-Bk-239

MAT 9722

97-Bk-239

Radionuclide Production Cross Section  
(n,  $\alpha$ )



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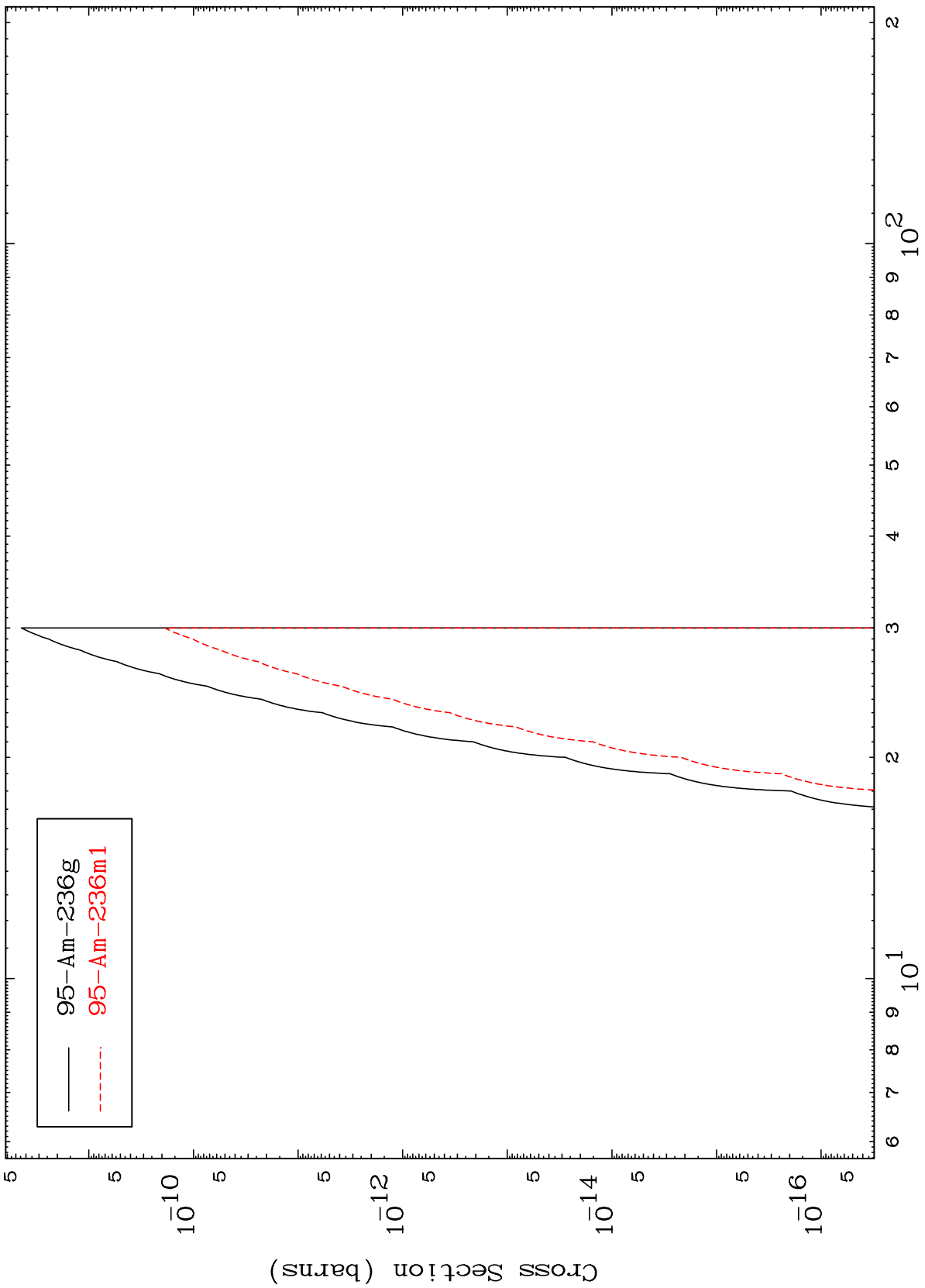
97-Bk-239

MAT 9722

(n,p) t

97-Bk-239

Radionuclide Production Cross Section



95-Am-236g  
95-Am-236m1

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Incident Energy (MeV)

97-Bk-239