

Program EVALPLOT  
(Version 2017-1)

by

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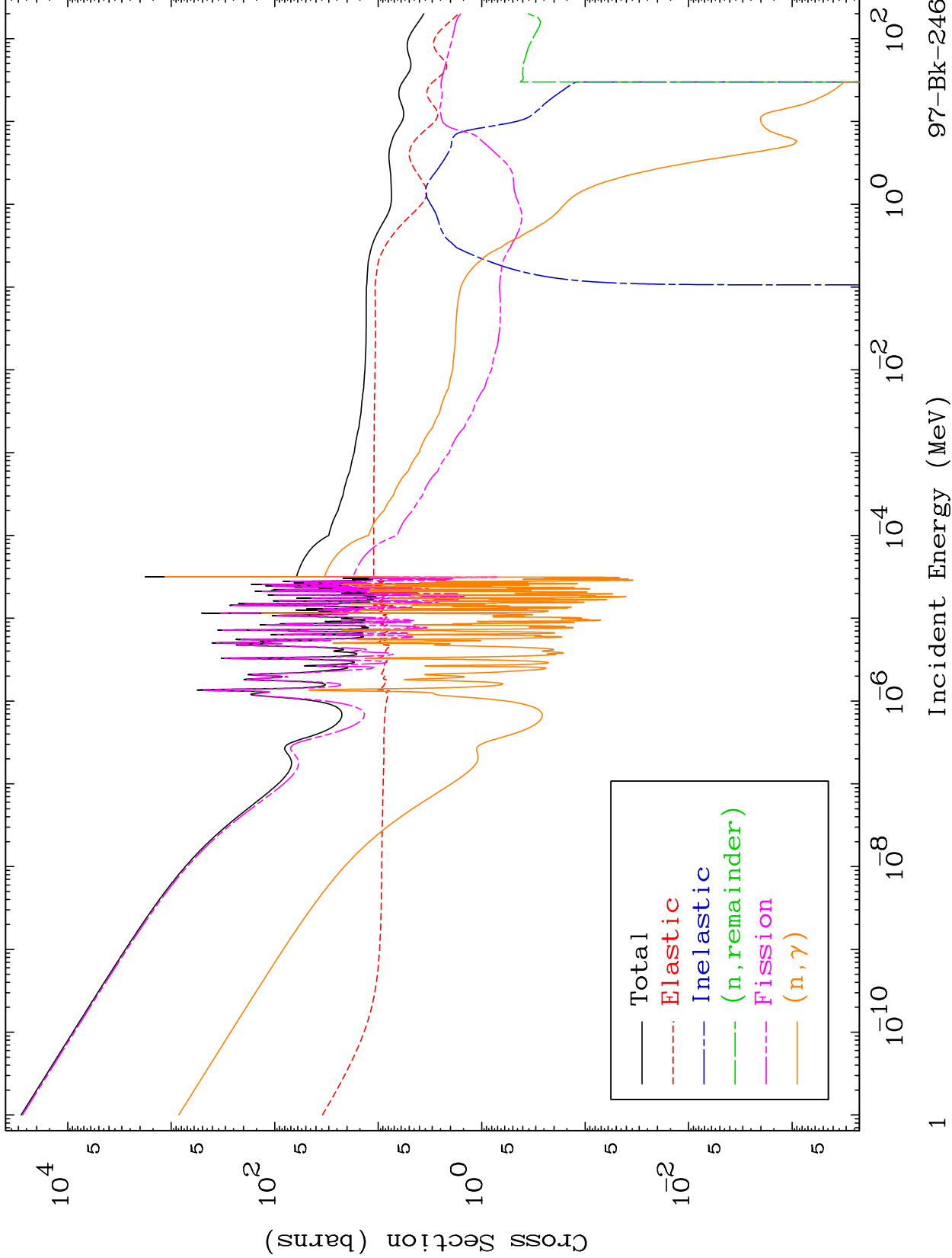
Press Mouse Button to Start

MAT 9743

Major

293 Kelvin Cross Sections

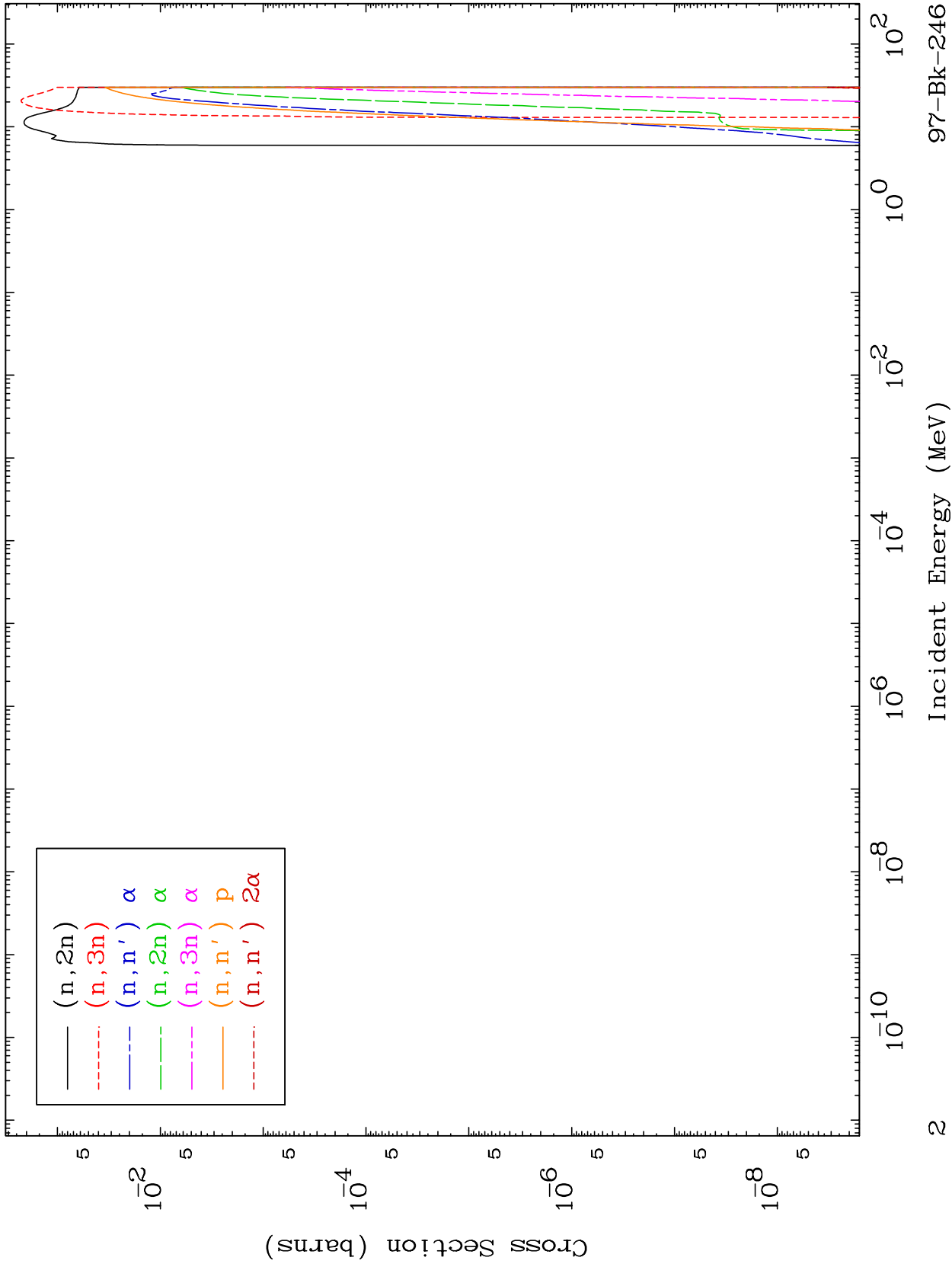
97-Bk-246

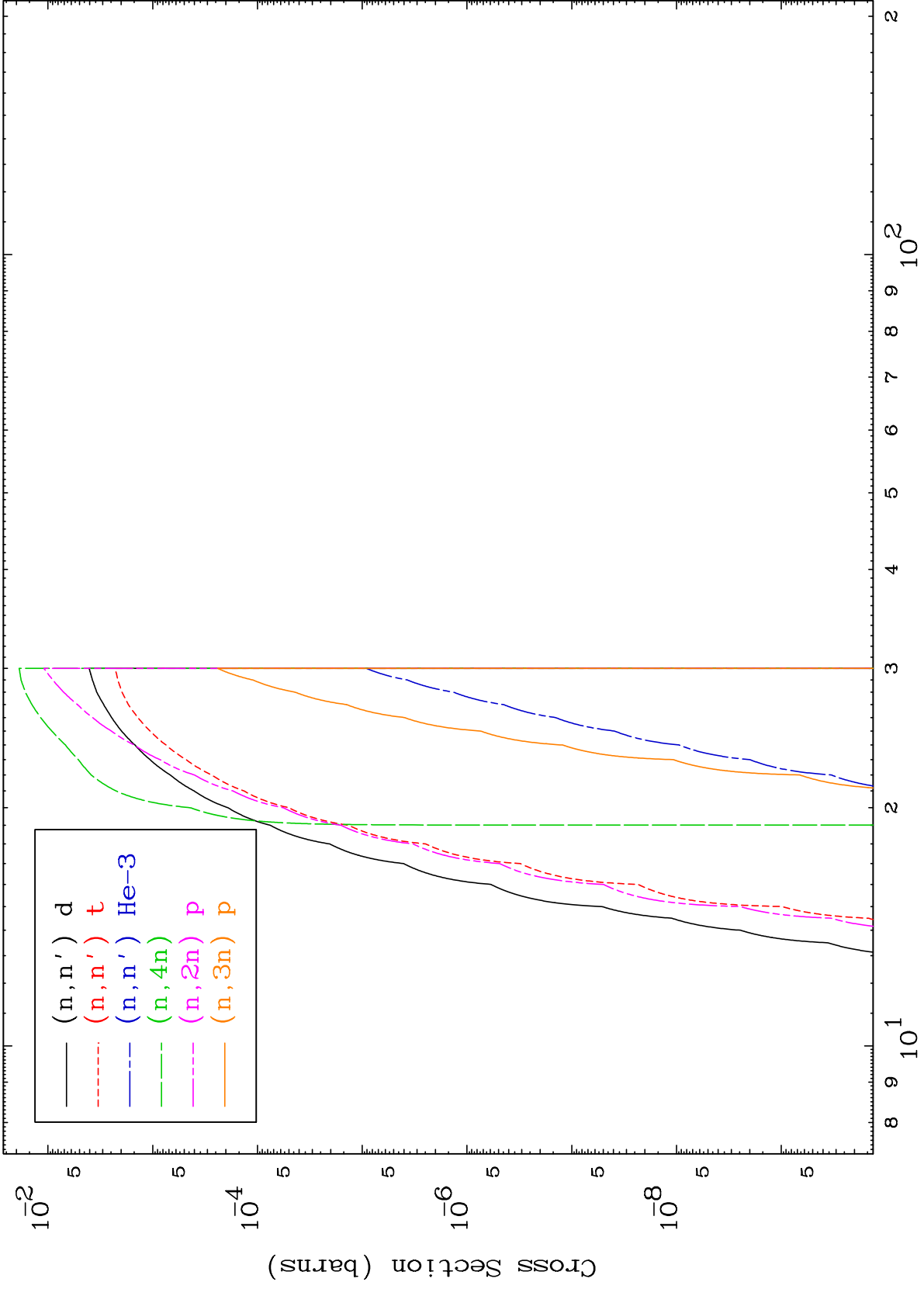


MAT 9743

Neutron Production  
293 Kelvin Cross Sections

97-Bk-246

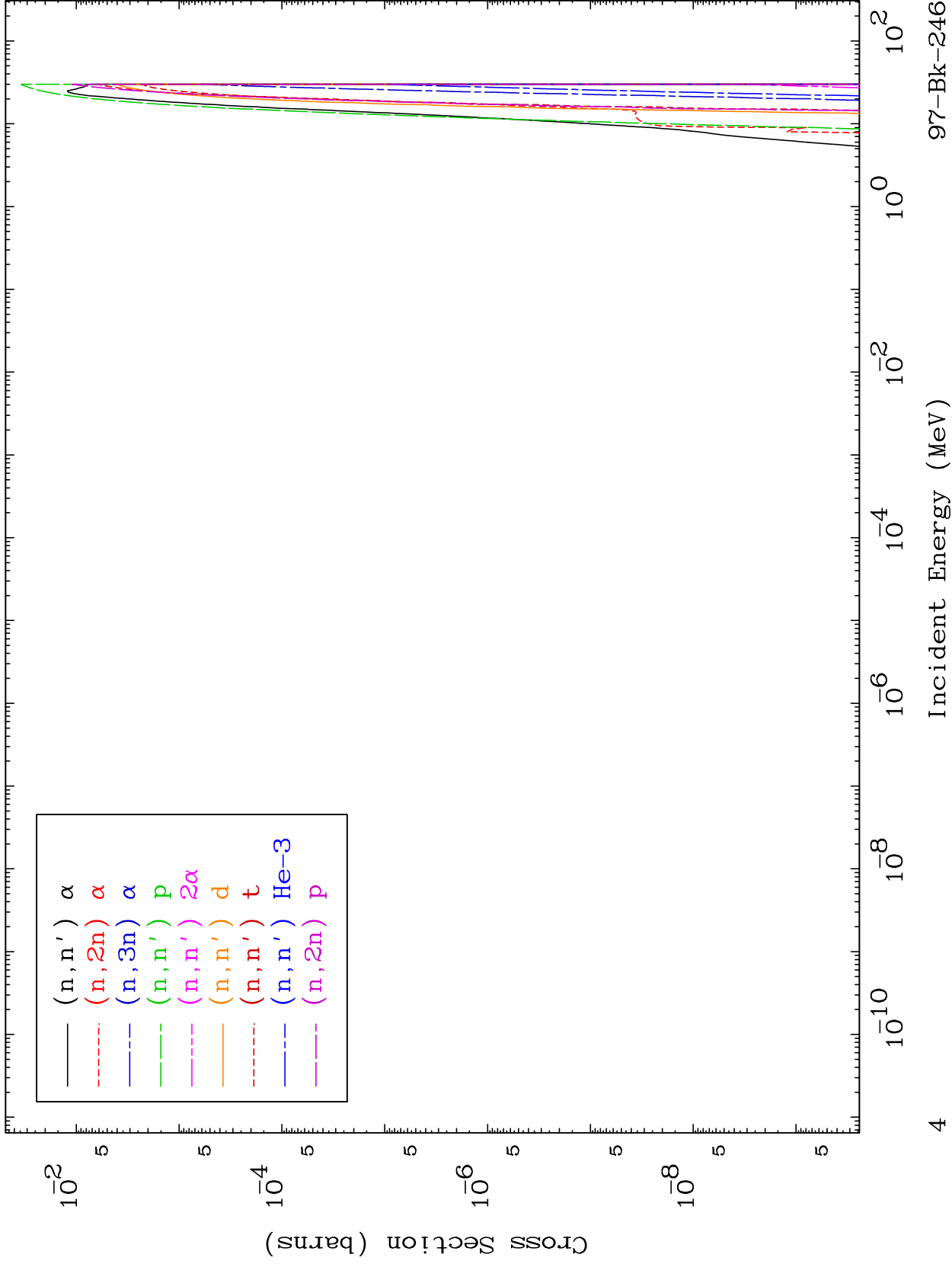




MAT 9743

Charged Particle  
293 Kelvin Cross Sections

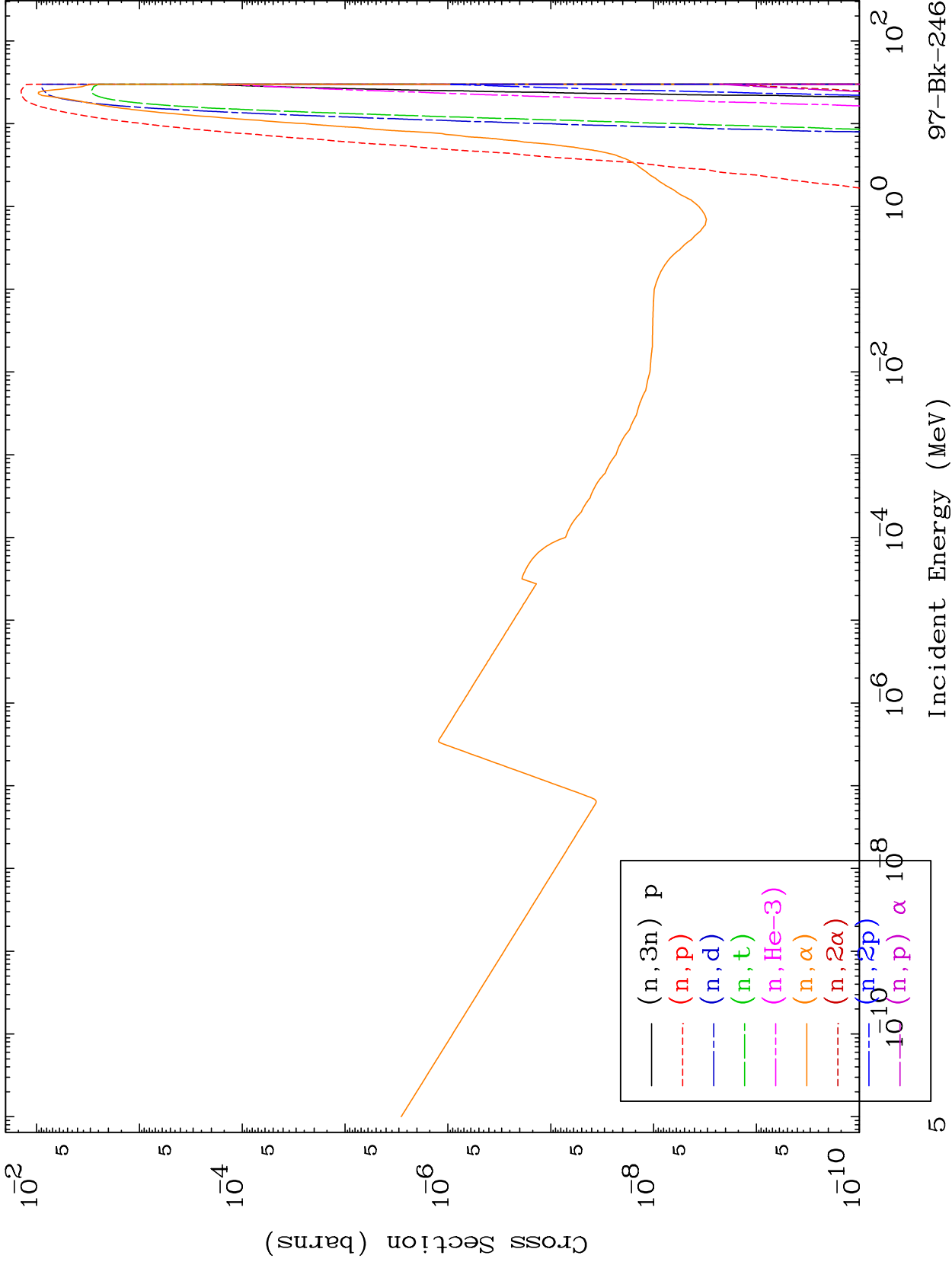
97-Bk-246



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Charged Particle  
293 Kelvin Cross Sections

97-Bk-246

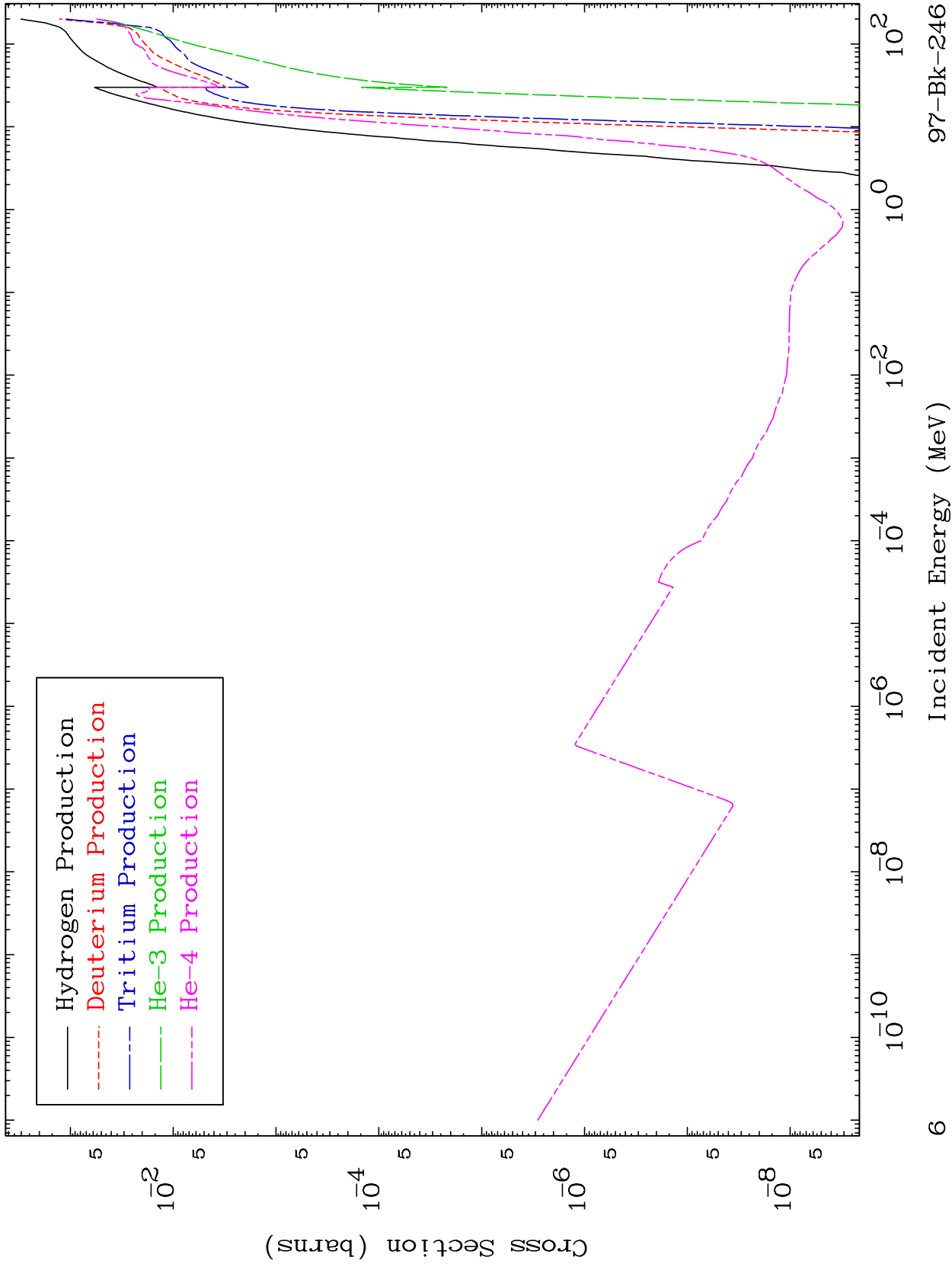


97-Bk-246

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Particle Production  
293 Kelvin Cross Sections

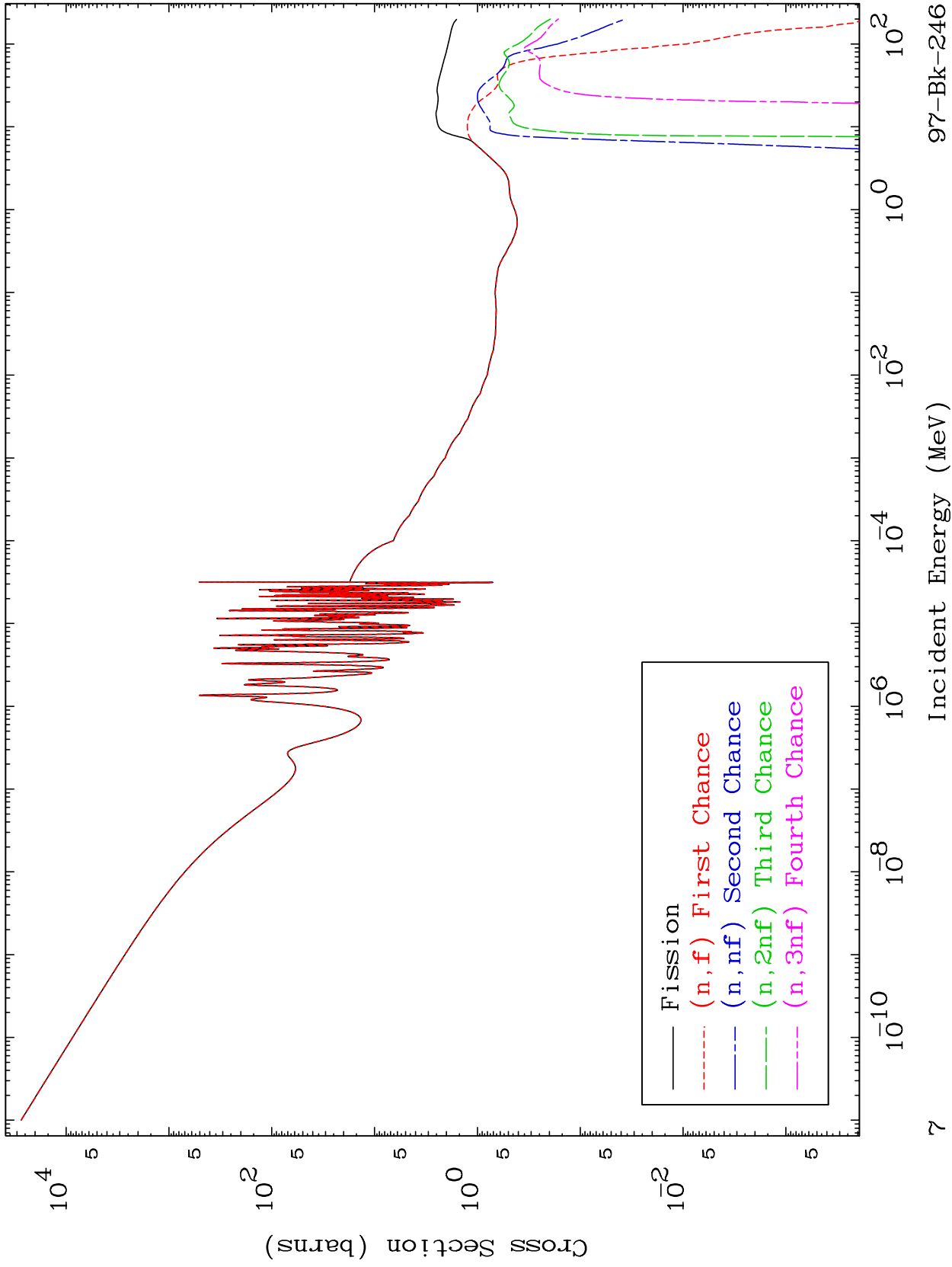
97-Bk-246



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Fission  
293 Kelvin Cross Sections

97-Bk-246

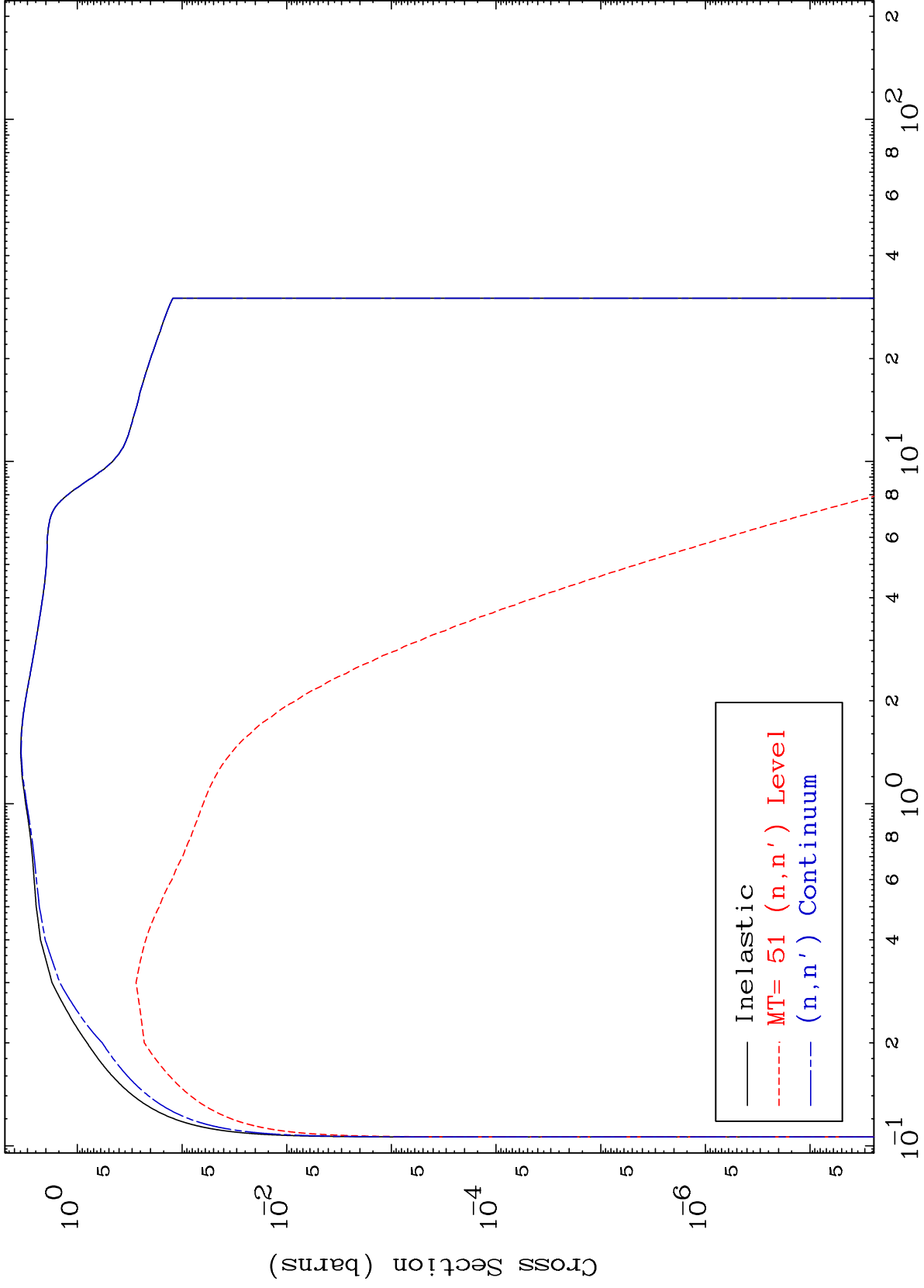




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(n,n') Level  
293 Kelvin Cross Sections

97-Bk-246



Incident Energy (MeV)

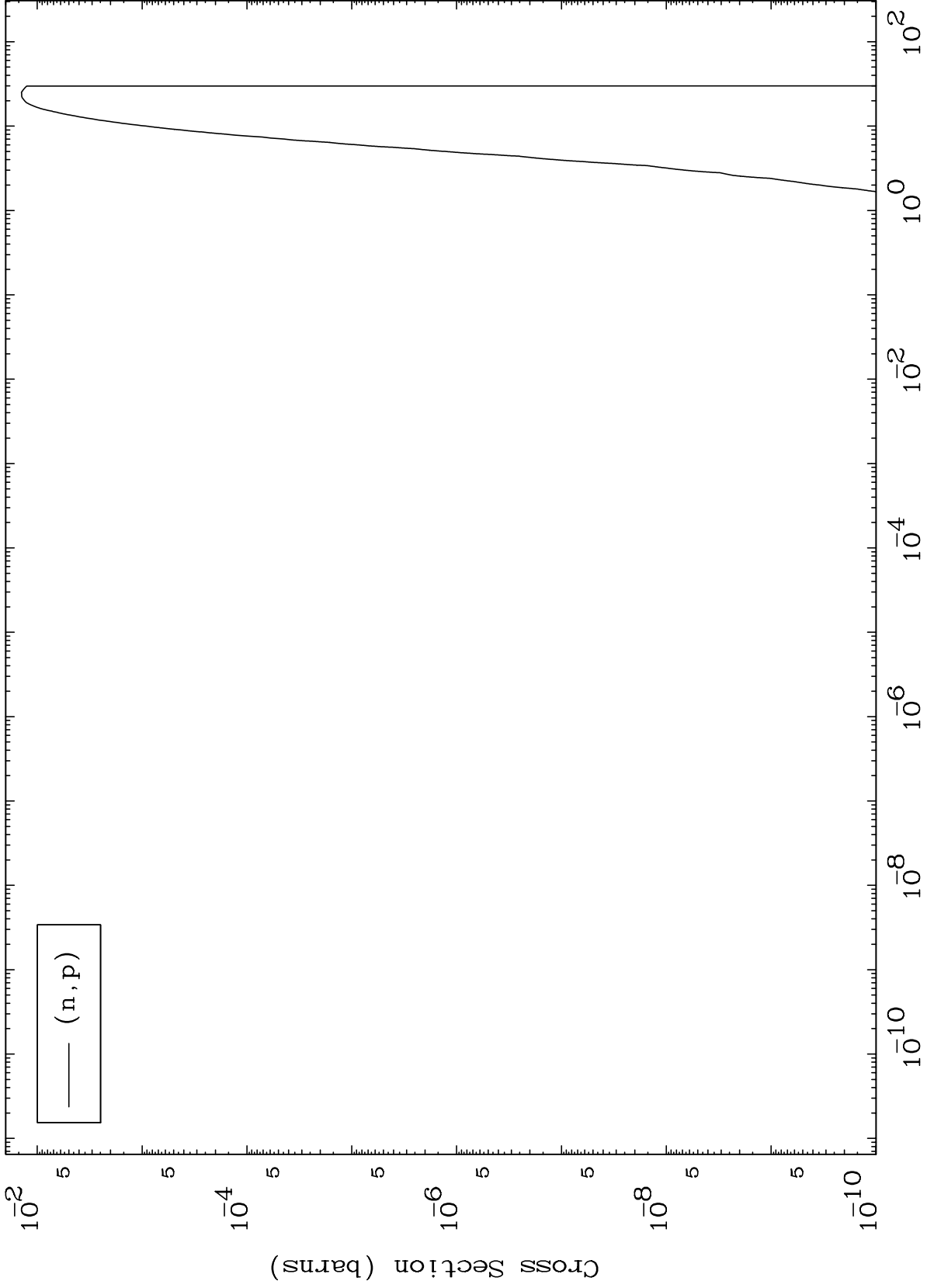
97-Bk-246

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(n,p) Levels  
293 Kelvin Cross Sections

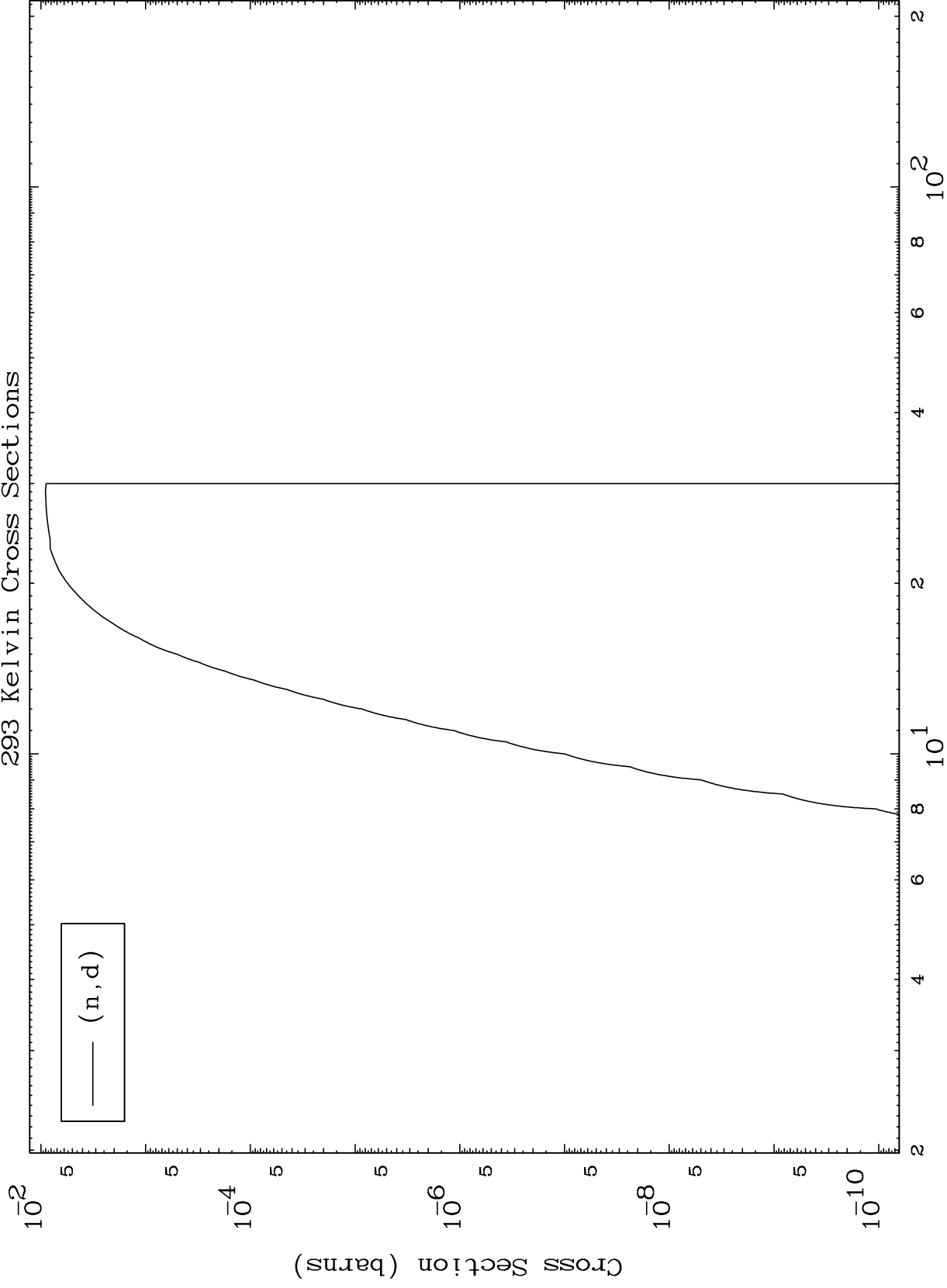
97-Bk-246



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-246



10

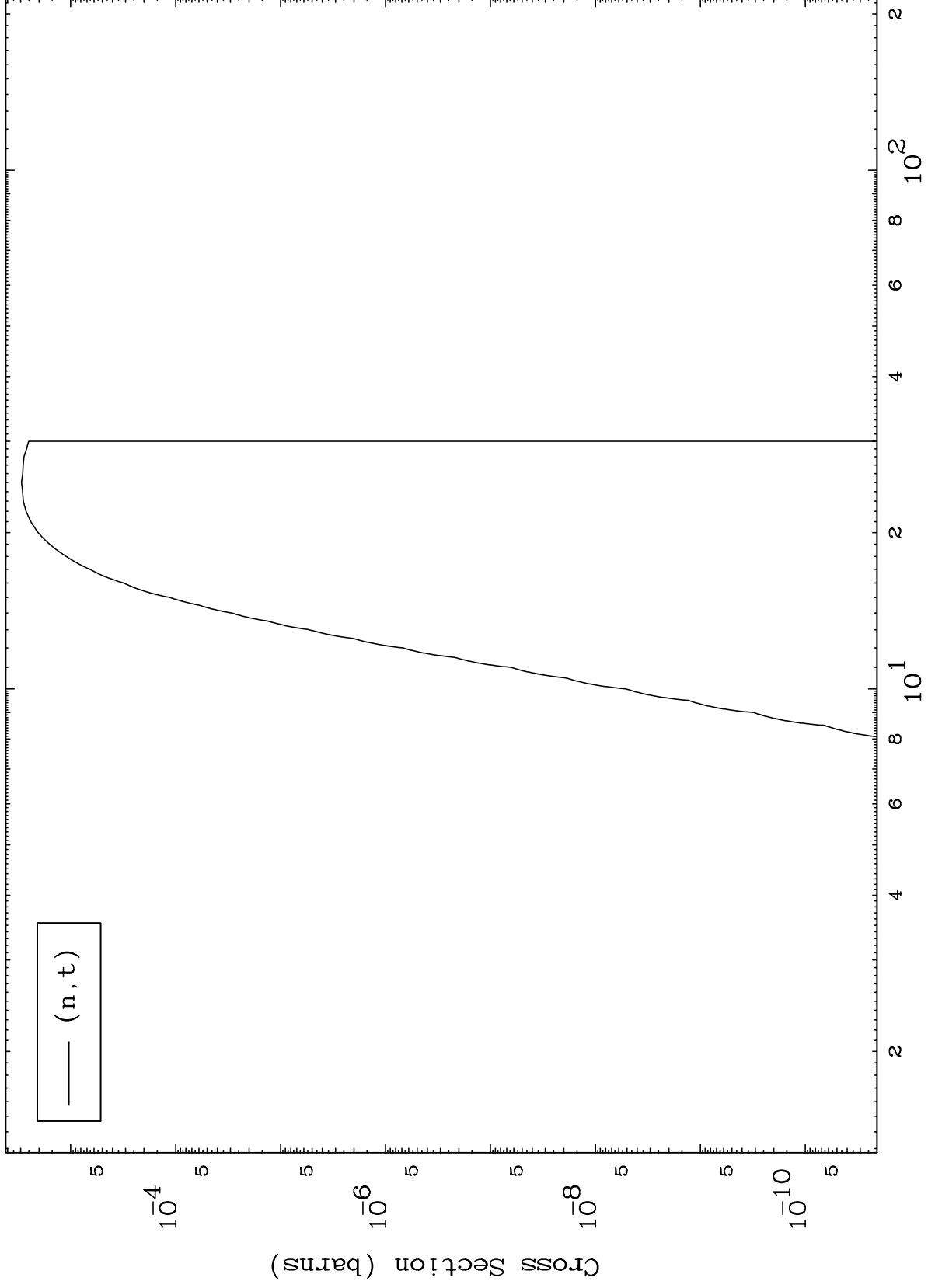
Incident Energy (MeV)

97-Bk-246

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(n,t) Levels  
293 Kelvin Cross Sections

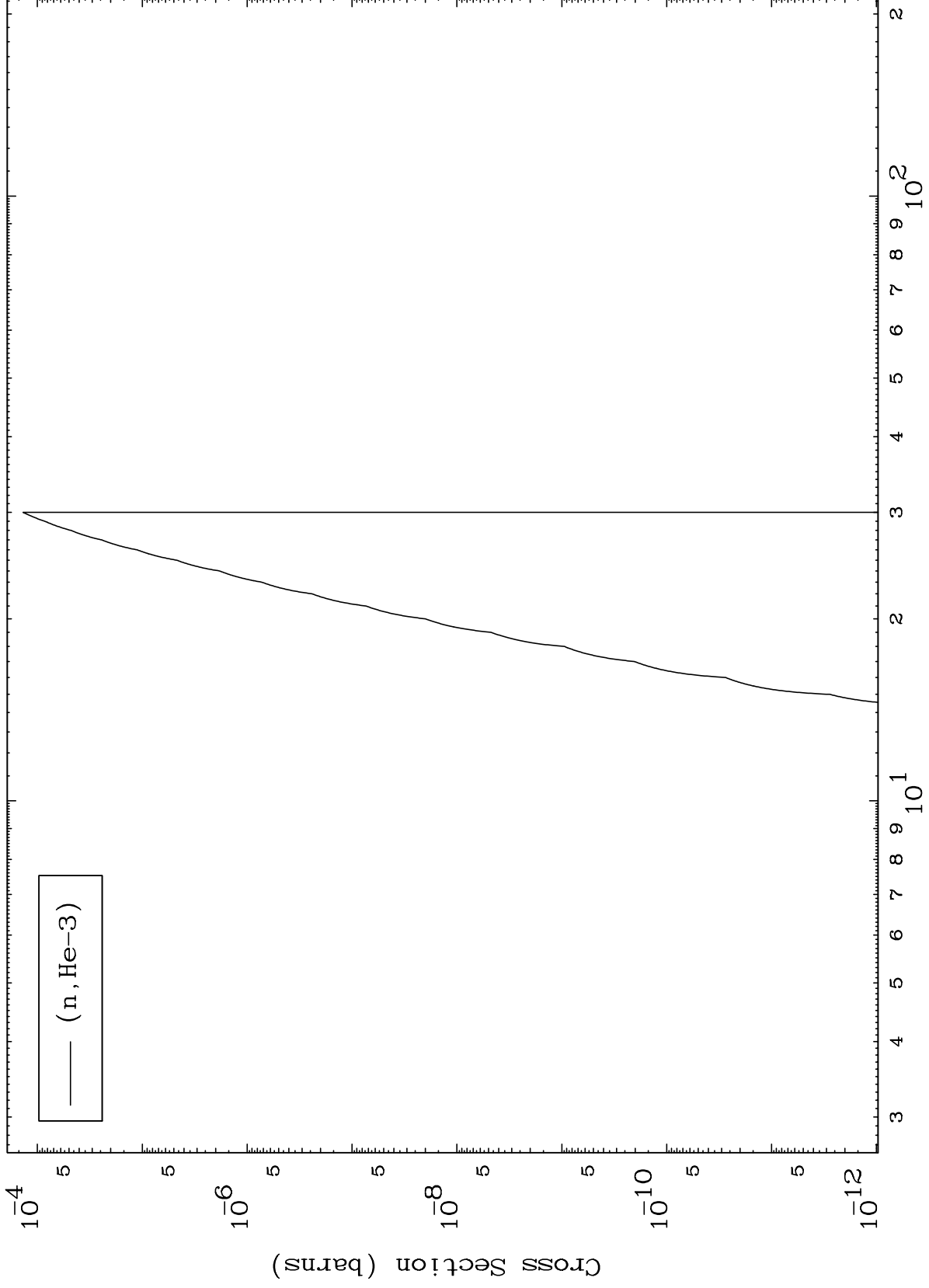
97-Bk-246



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(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-246



12

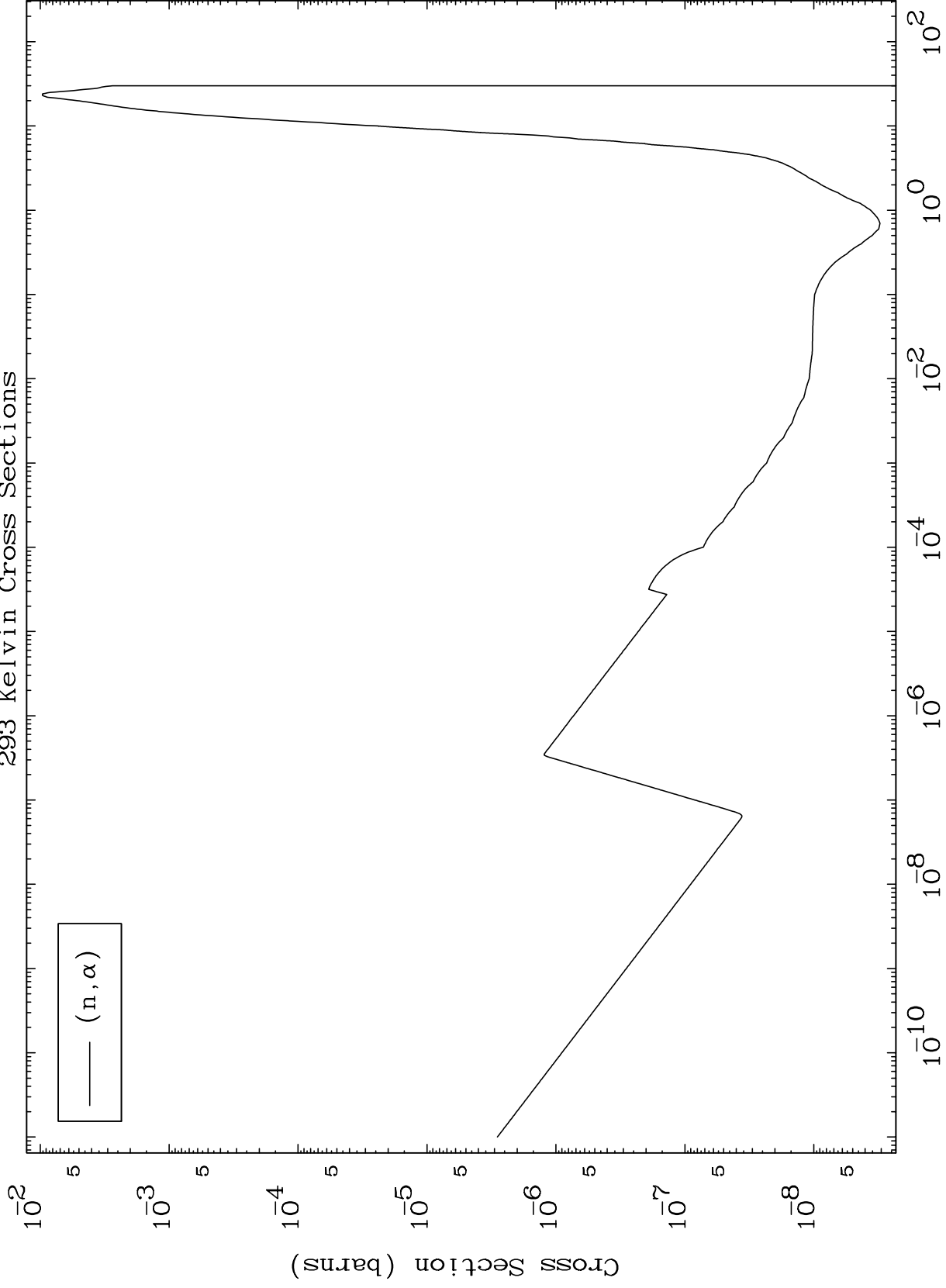
Incident Energy (MeV)

97-Bk-246

MAT 9743

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

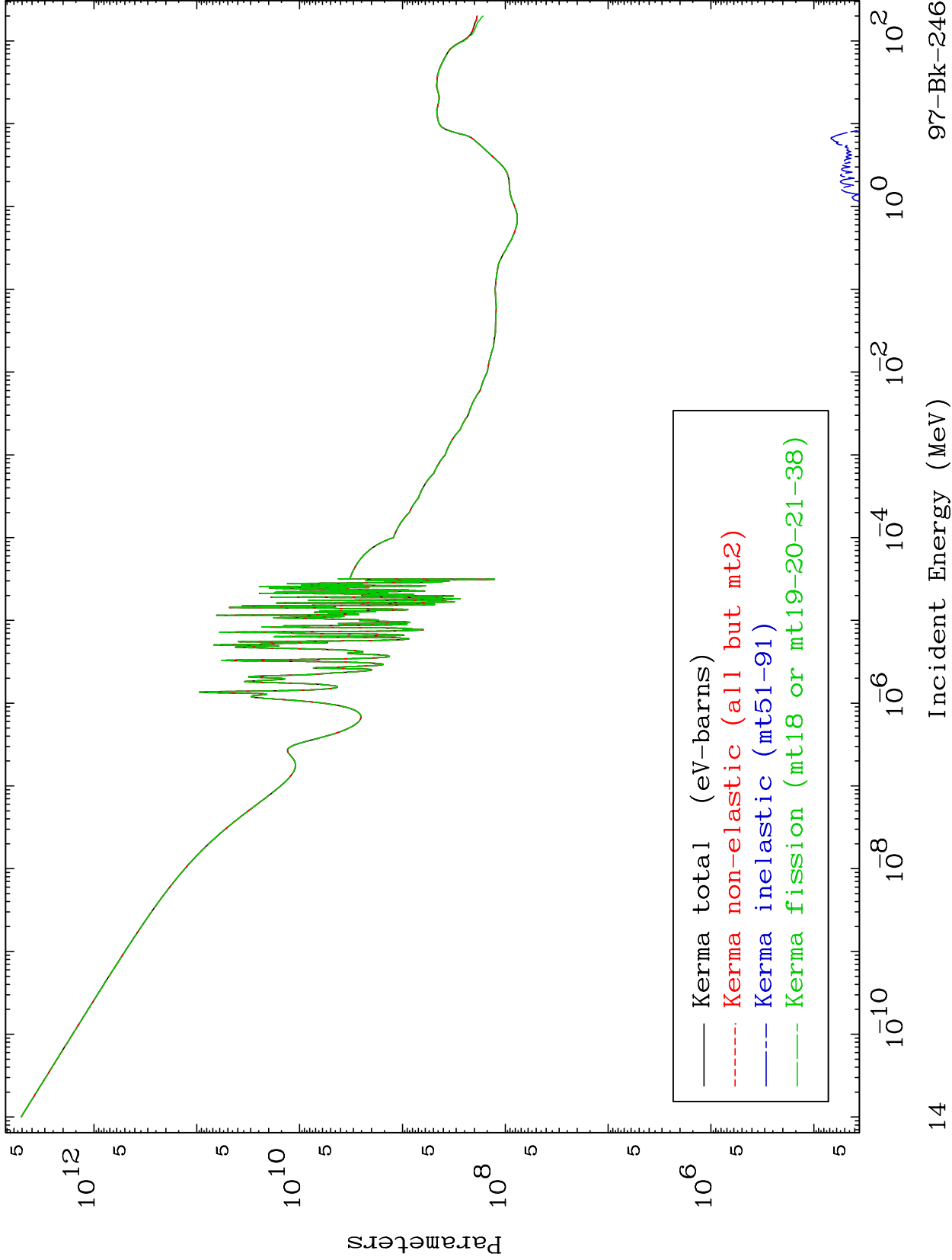
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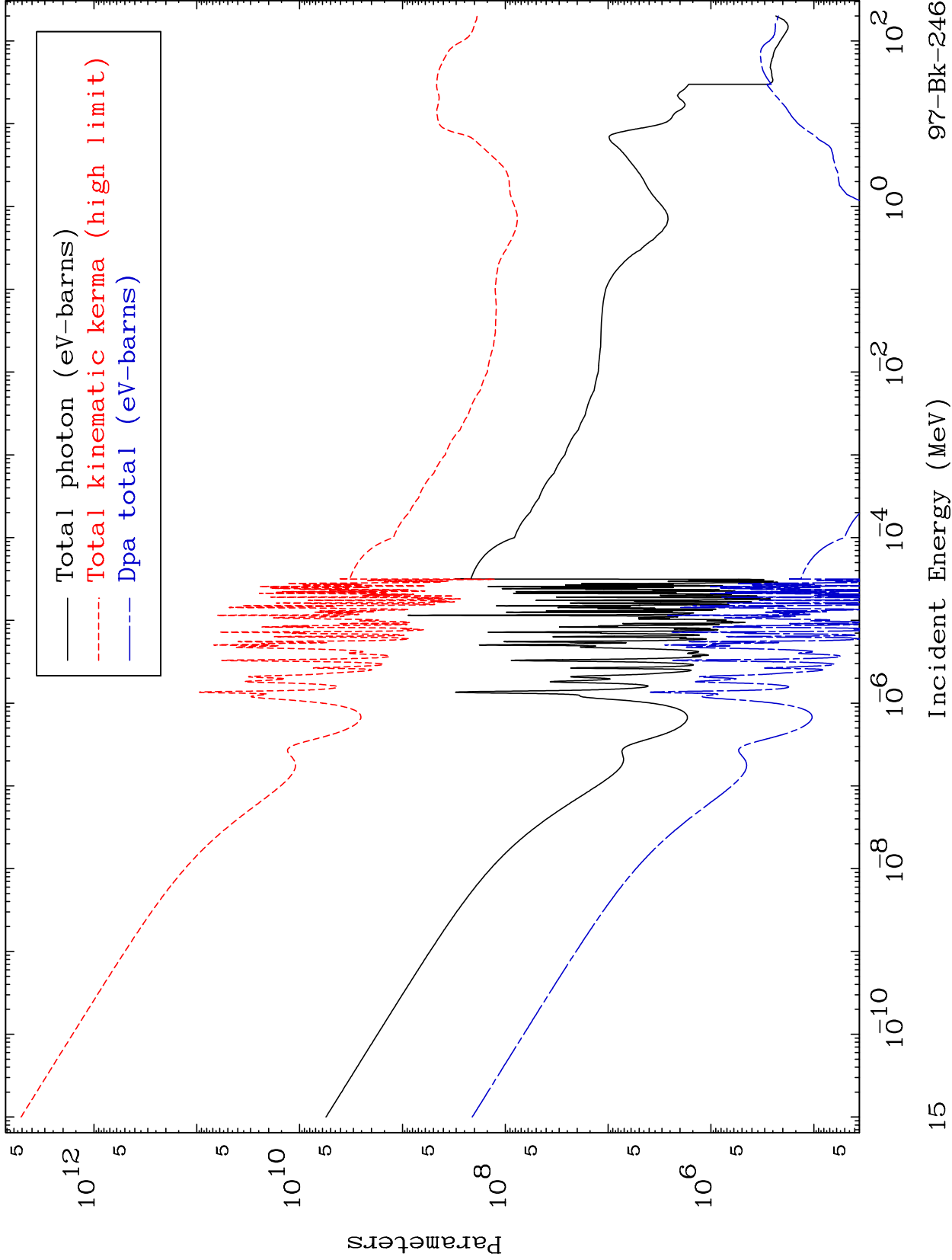


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Incident Energy (MeV)

97-Bk-246



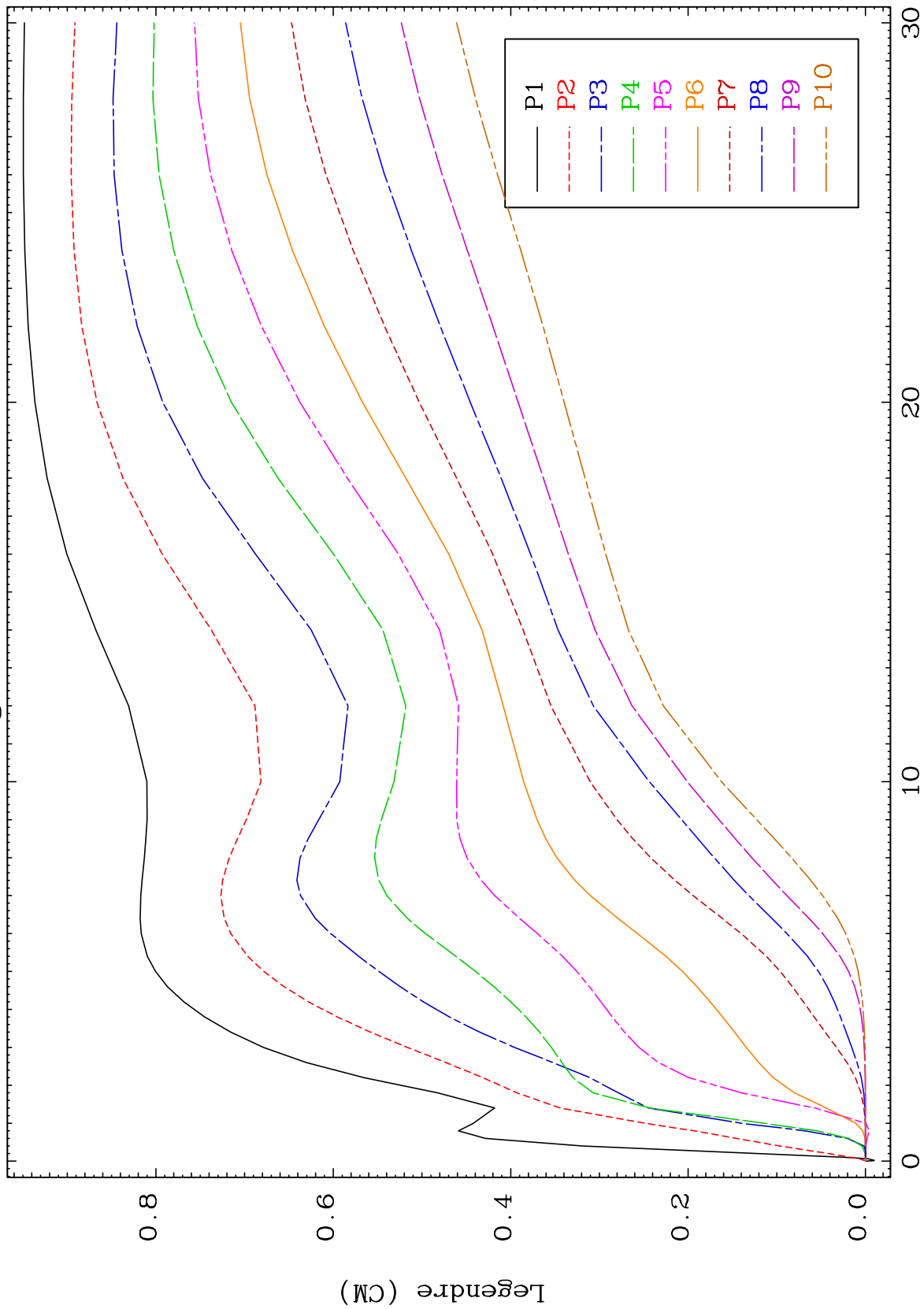




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Elastic Legendre Coefficients

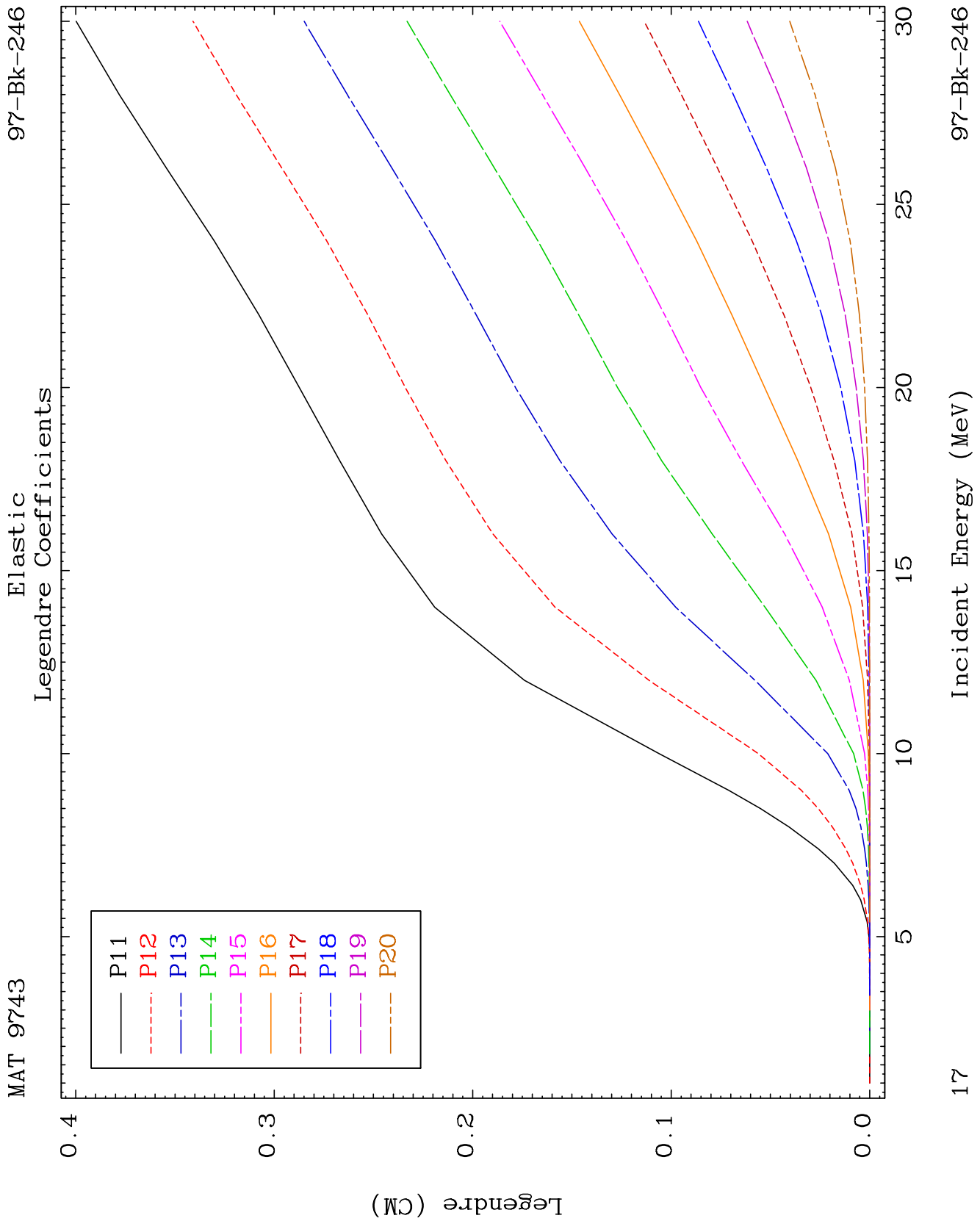
97-Bk-246



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Incident Energy (MeV)

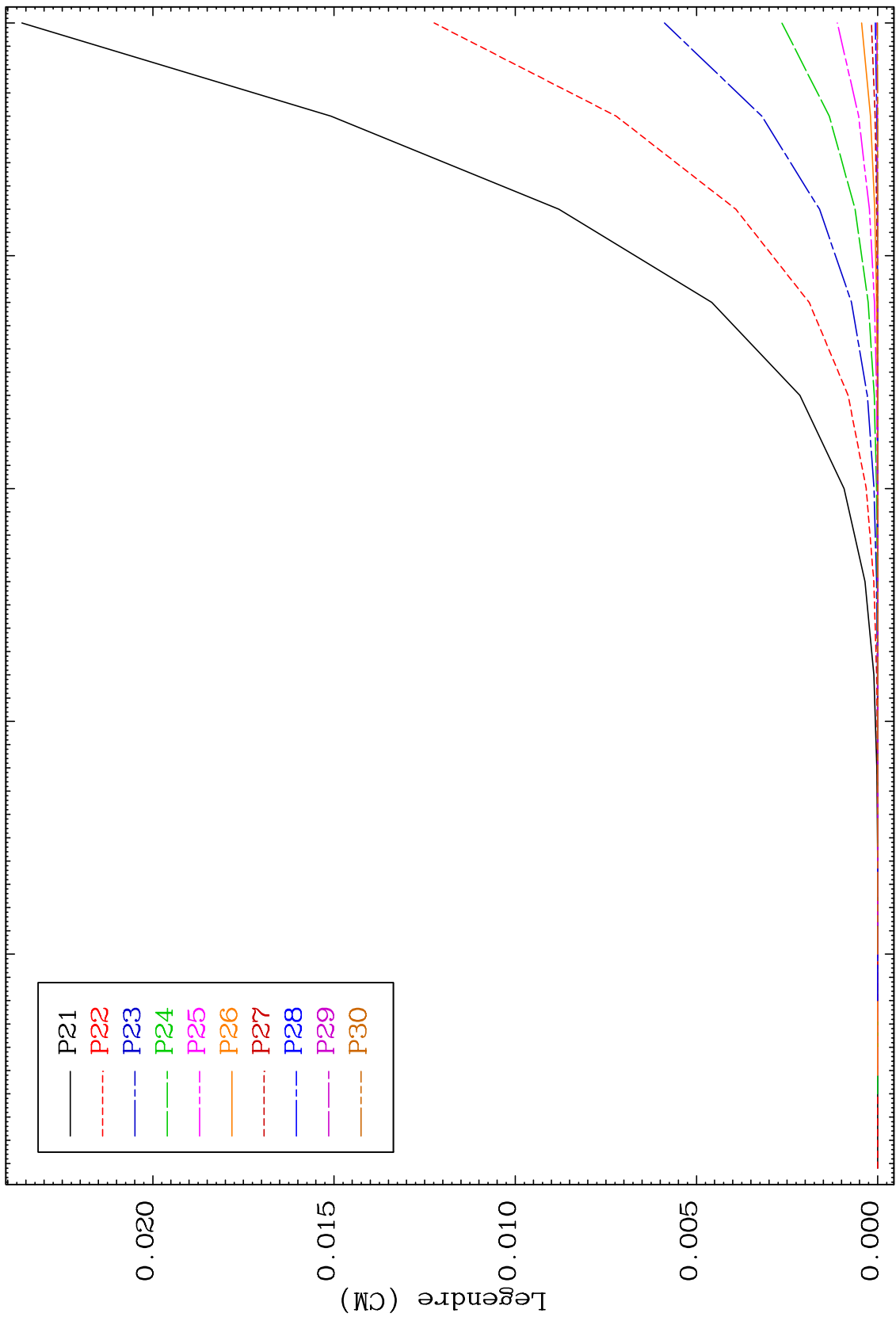
97-Bk-246



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### Elastic Legendre Coefficients

97-Bk-246



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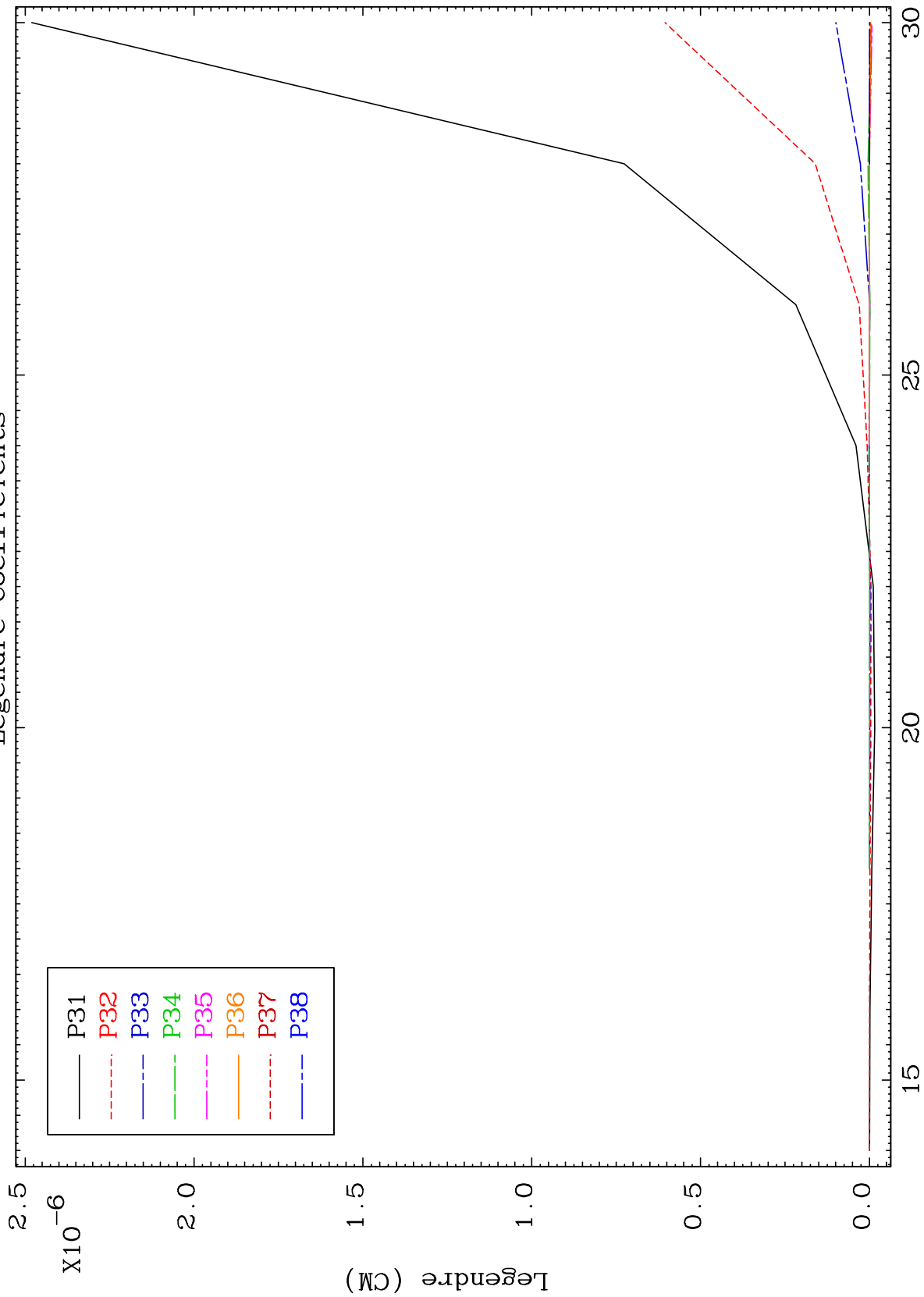
Incident Energy (MeV)

97-Bk-246

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Elastic Legendre Coefficients

97-Bk-246



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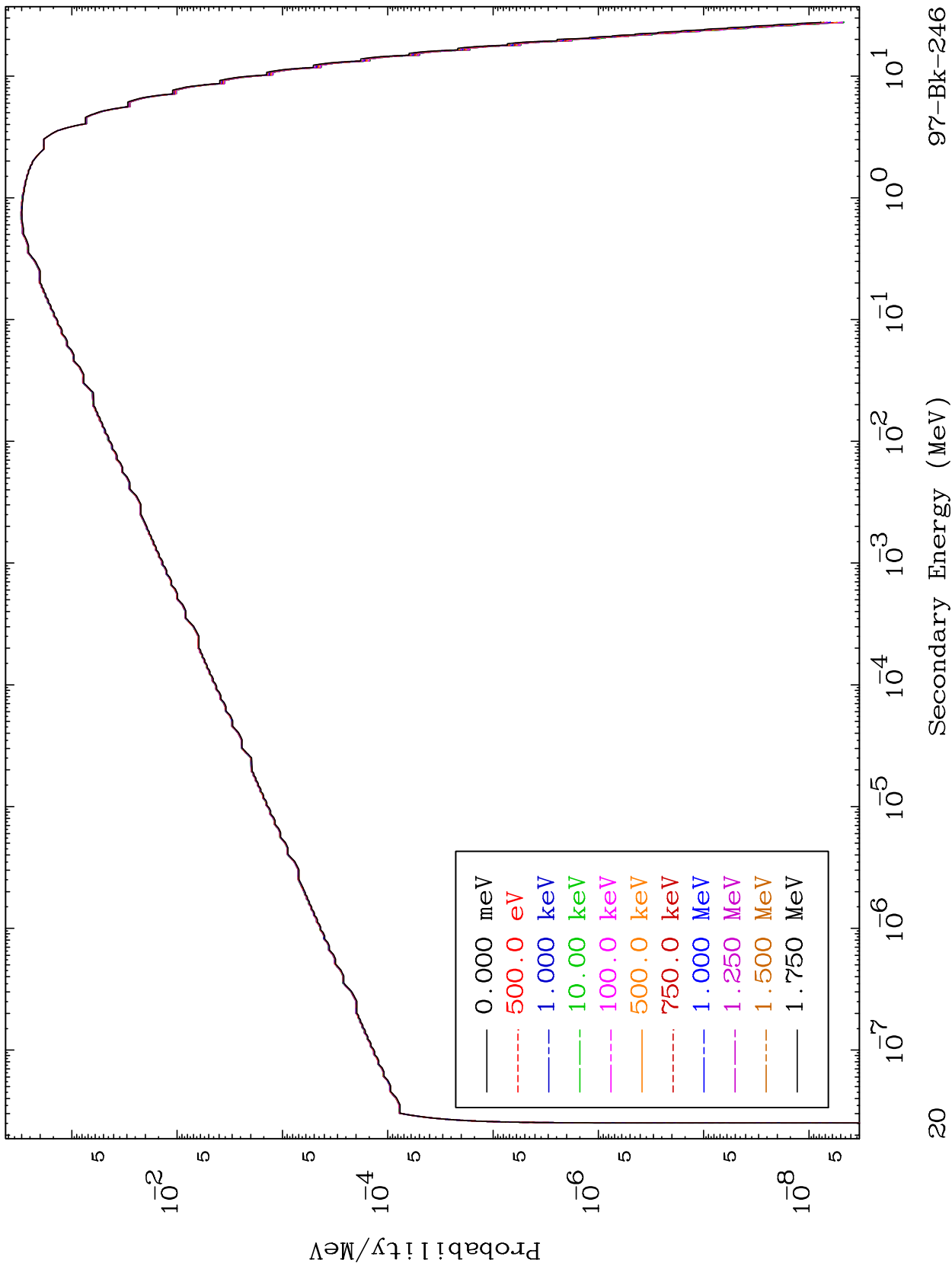
Incident Energy (MeV)

97-Bk-246

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### Fission Energy Distribution

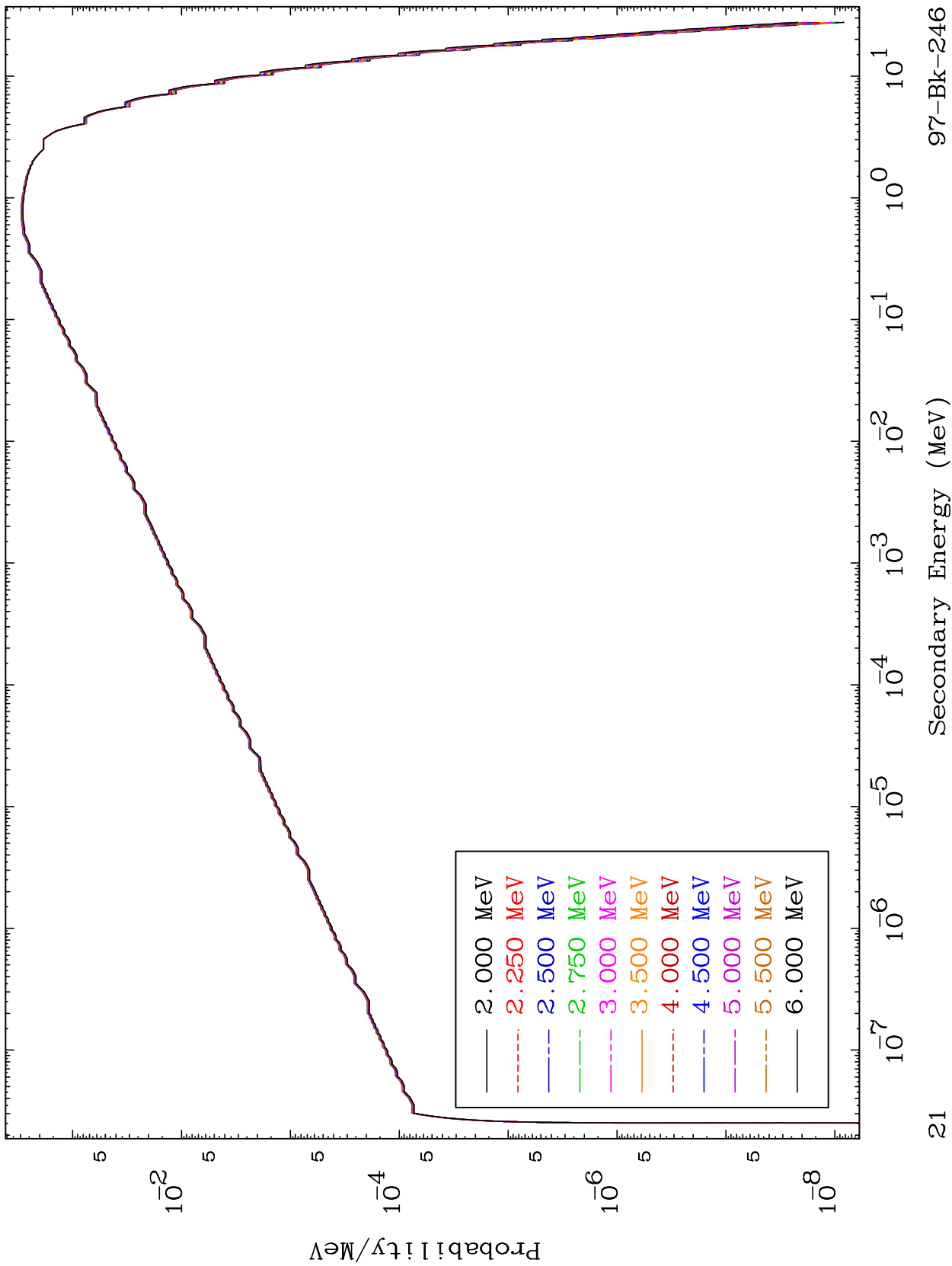
97-Bk-246



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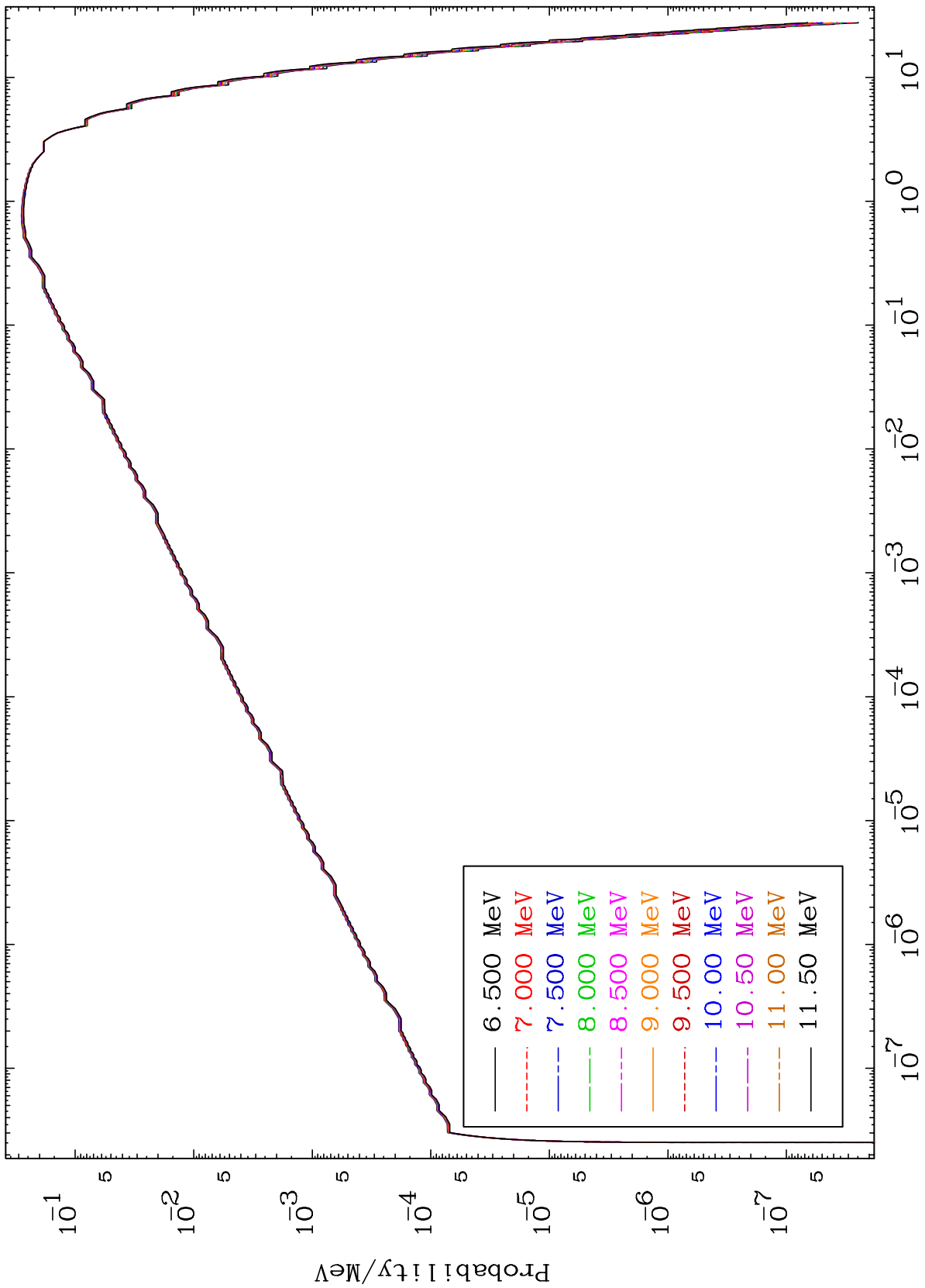
### Fission Energy Distribution

97-Bk-246



97-Bk-246

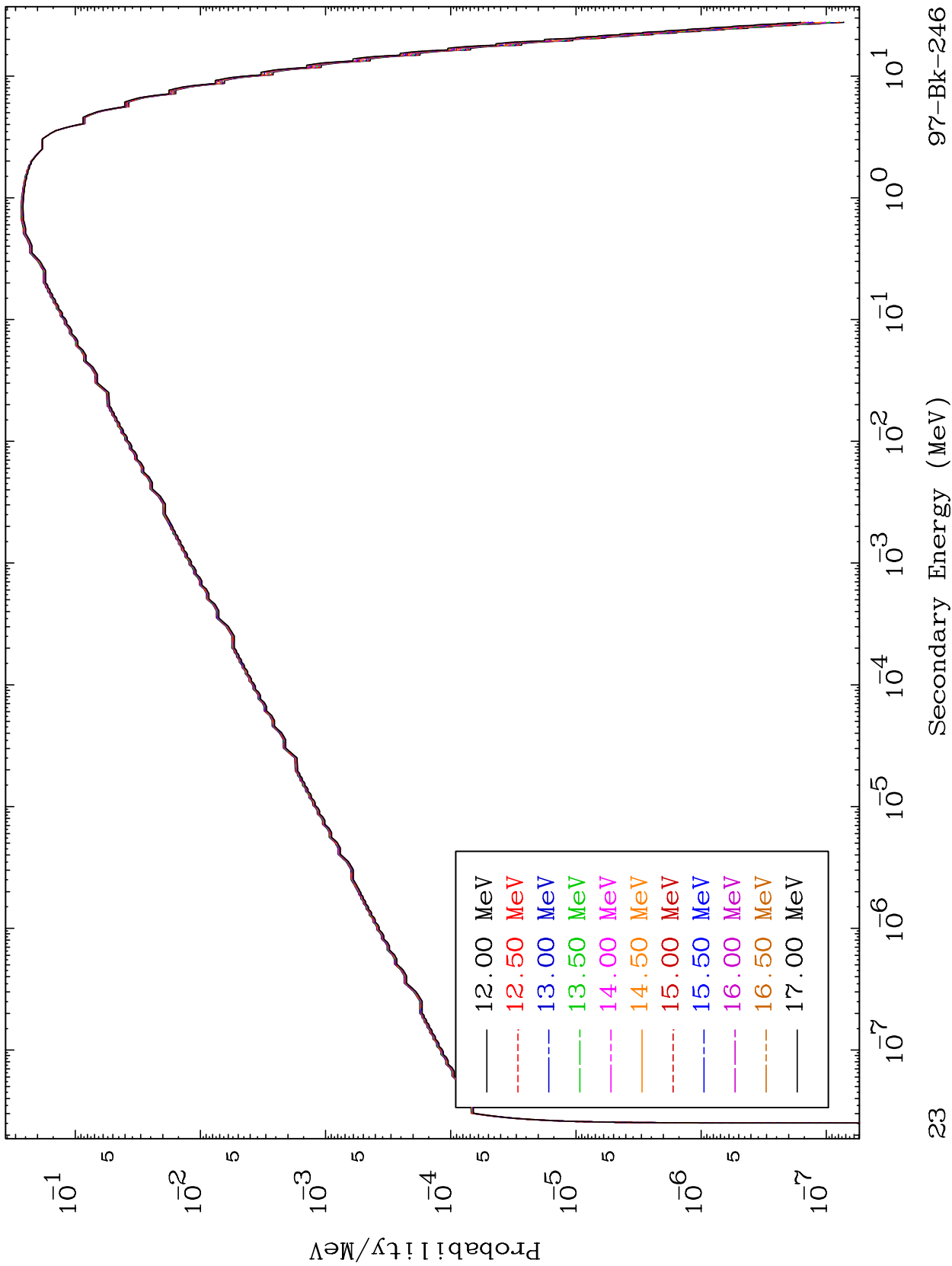
### Fission Energy Distribution



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### Fission Energy Distribution

97-Bk-246

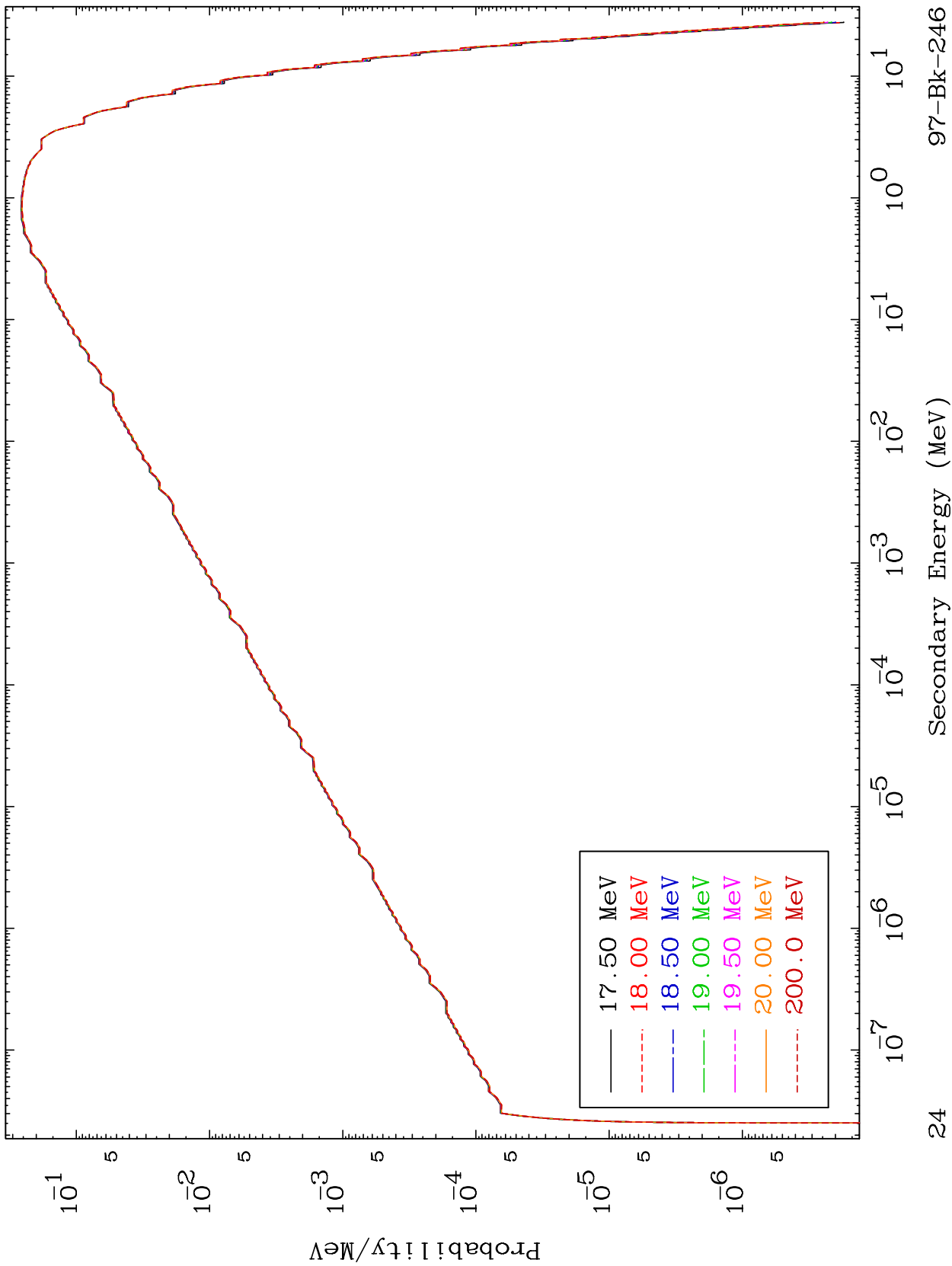


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Secondary Energy (MeV)



Fission  
Energy Distribution

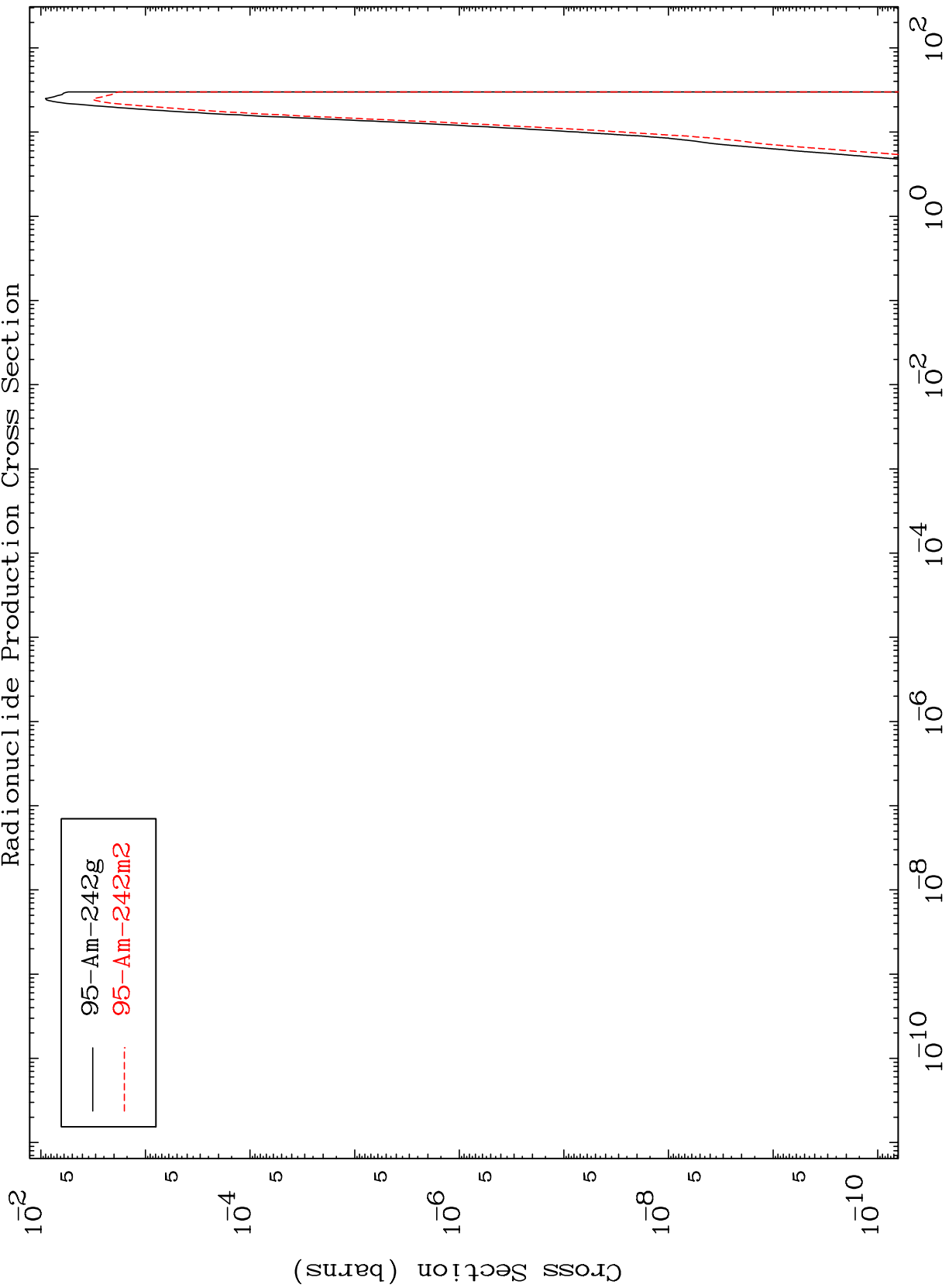


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(n,n')  $\alpha$

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Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-246

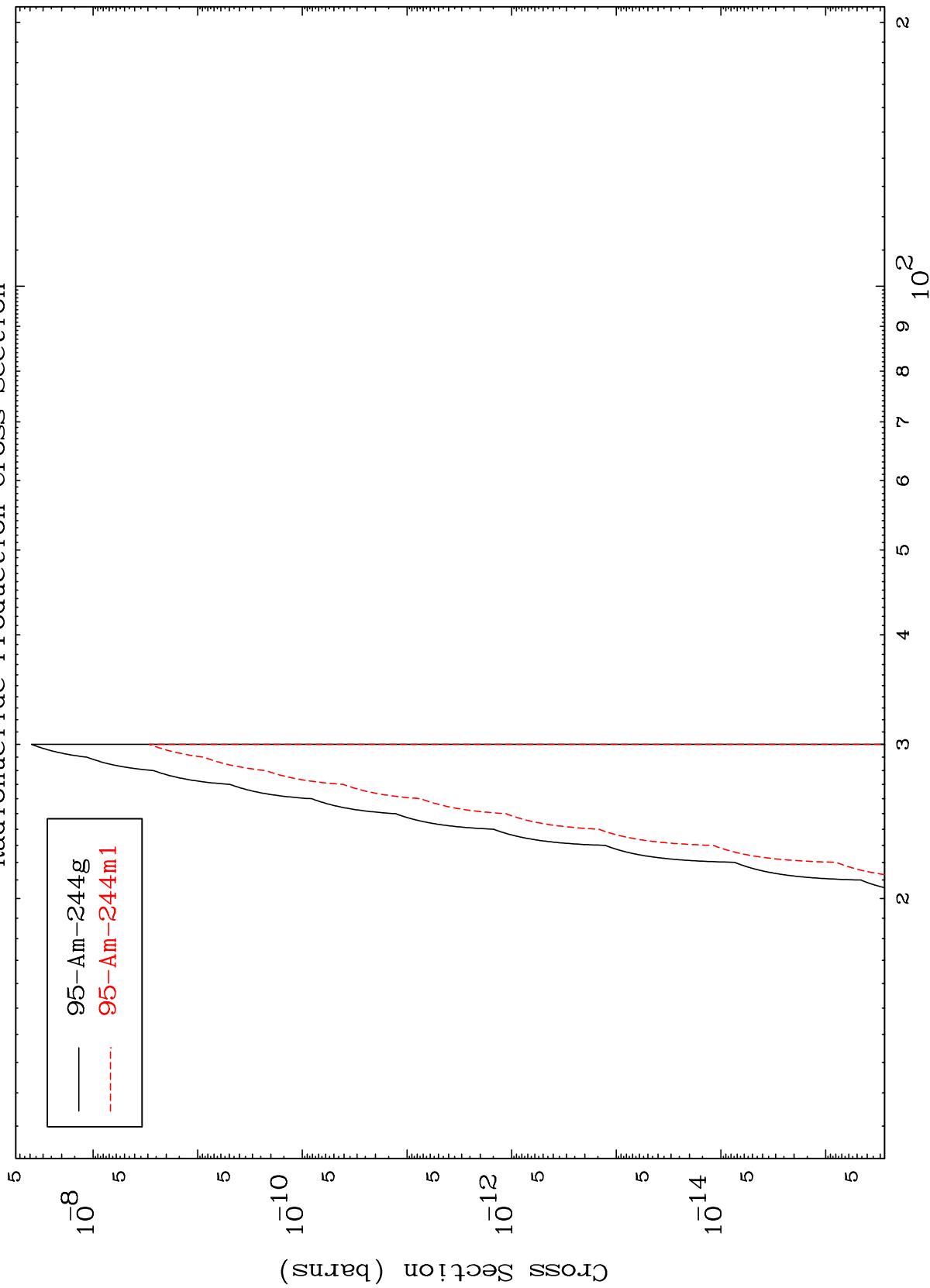
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(n,2n) p

97-Bk-246

Radionuclide Production Cross Section



95-Am-244g  
95-Am-244m1

Incident Energy (MeV)

97-Bk-246

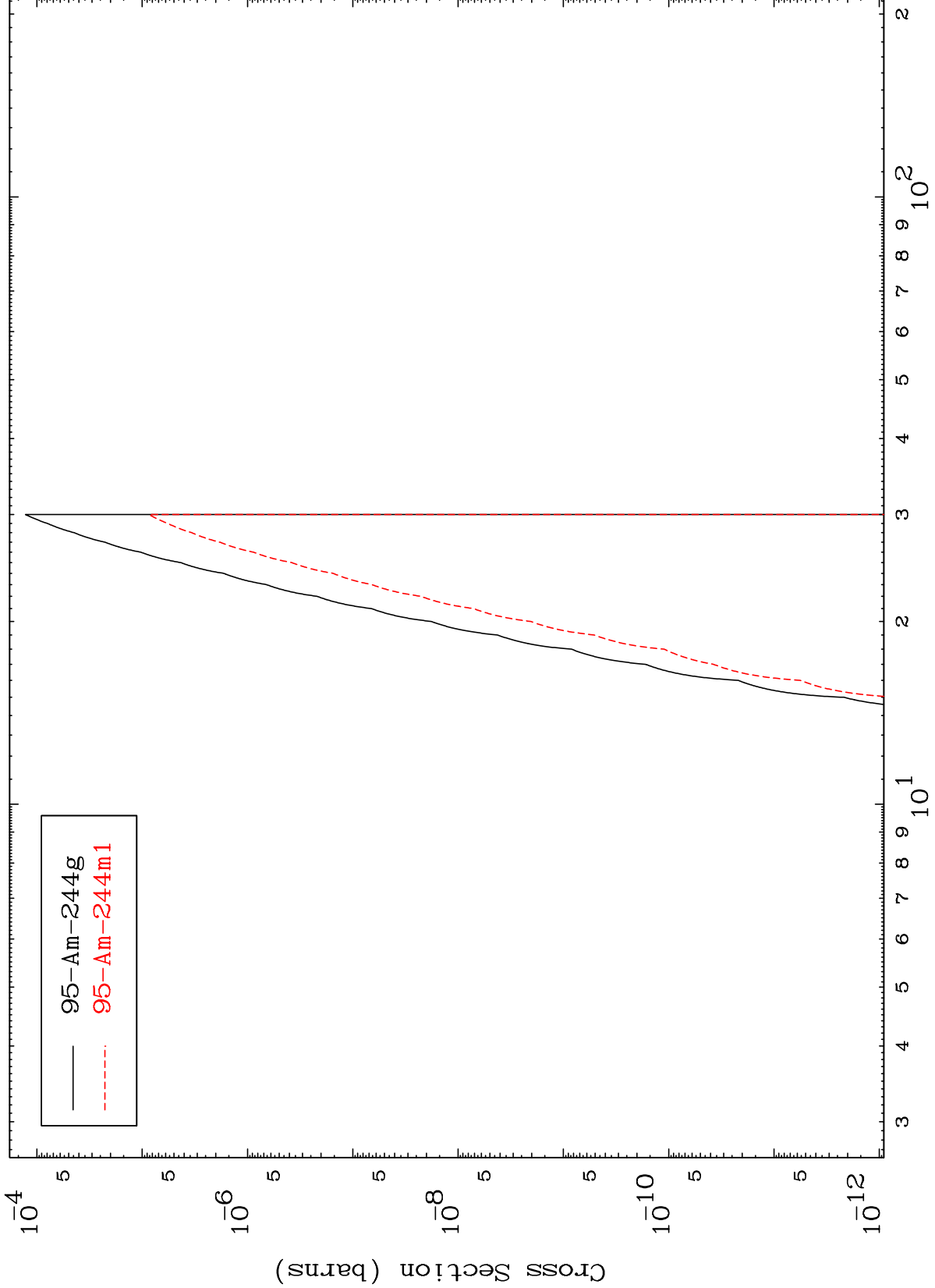
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(n,He-3)

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Radionuclide Production Cross Section



— 95-Am-244g  
- - - 95-Am-244m1