

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net

Web: redcullen1.net/HOMEPAGE.NEW

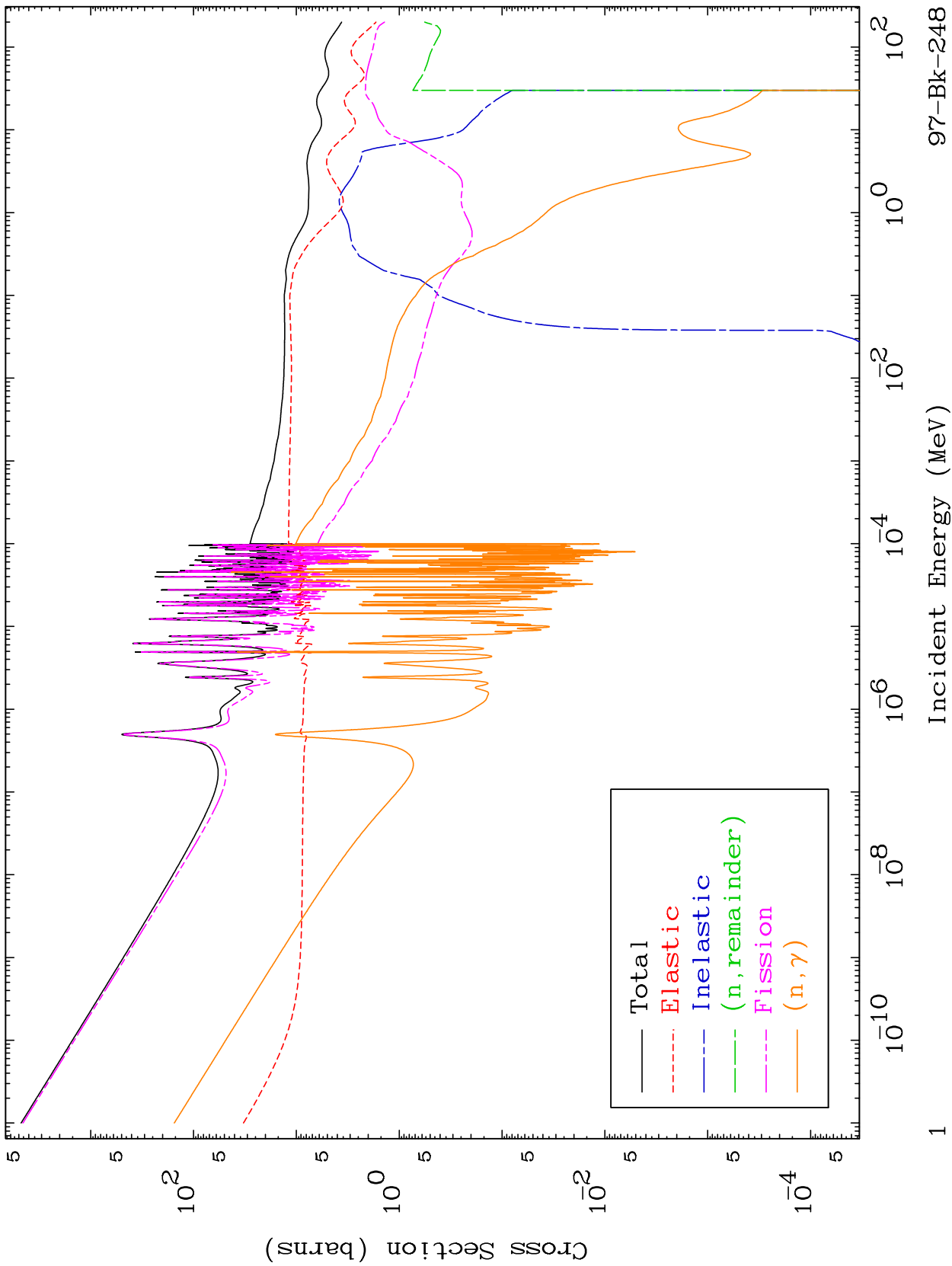
Press Mouse Button to Start

MAT 9750

Major

293 Kelvin Cross Sections

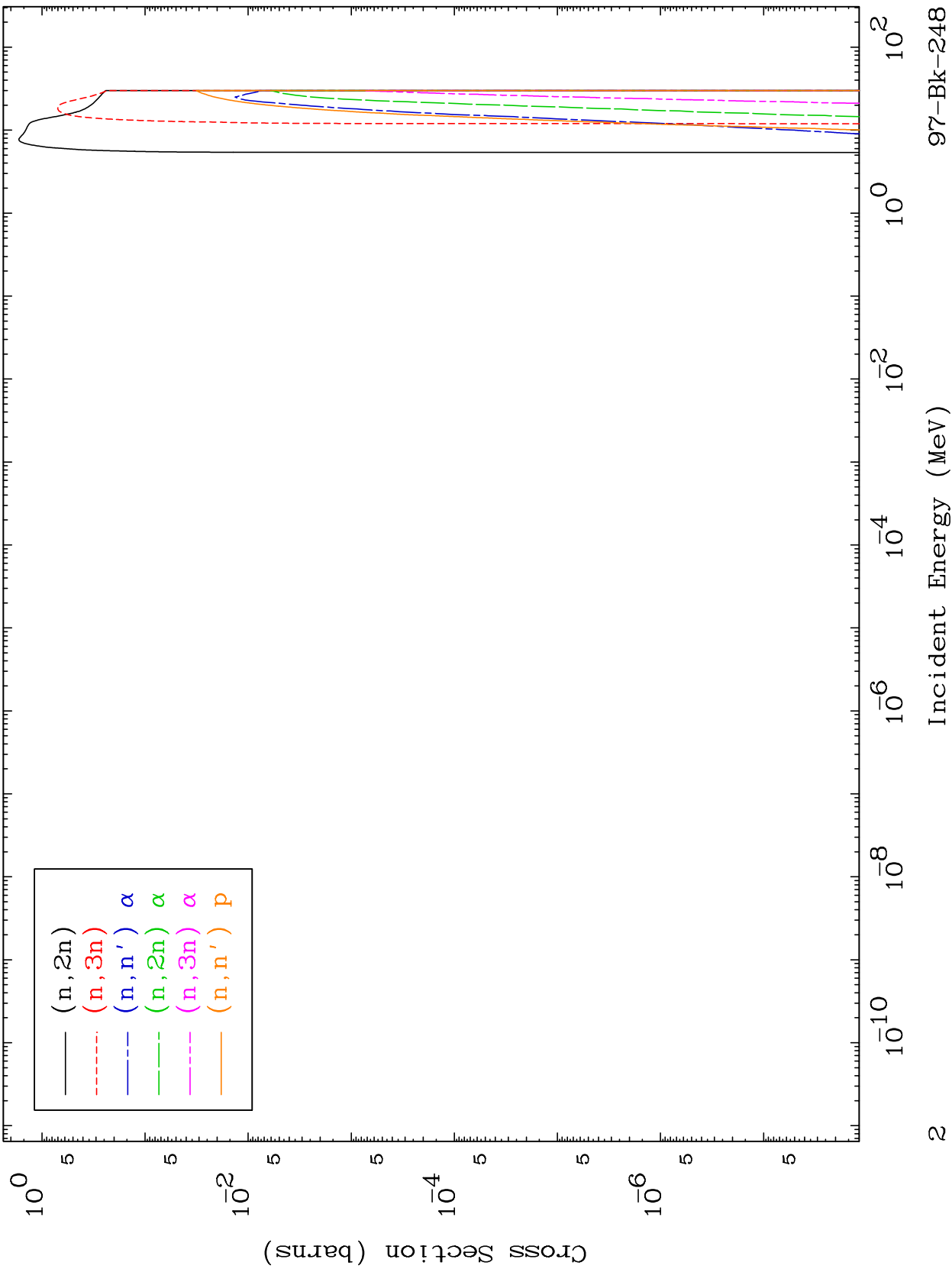
97-Bk-248

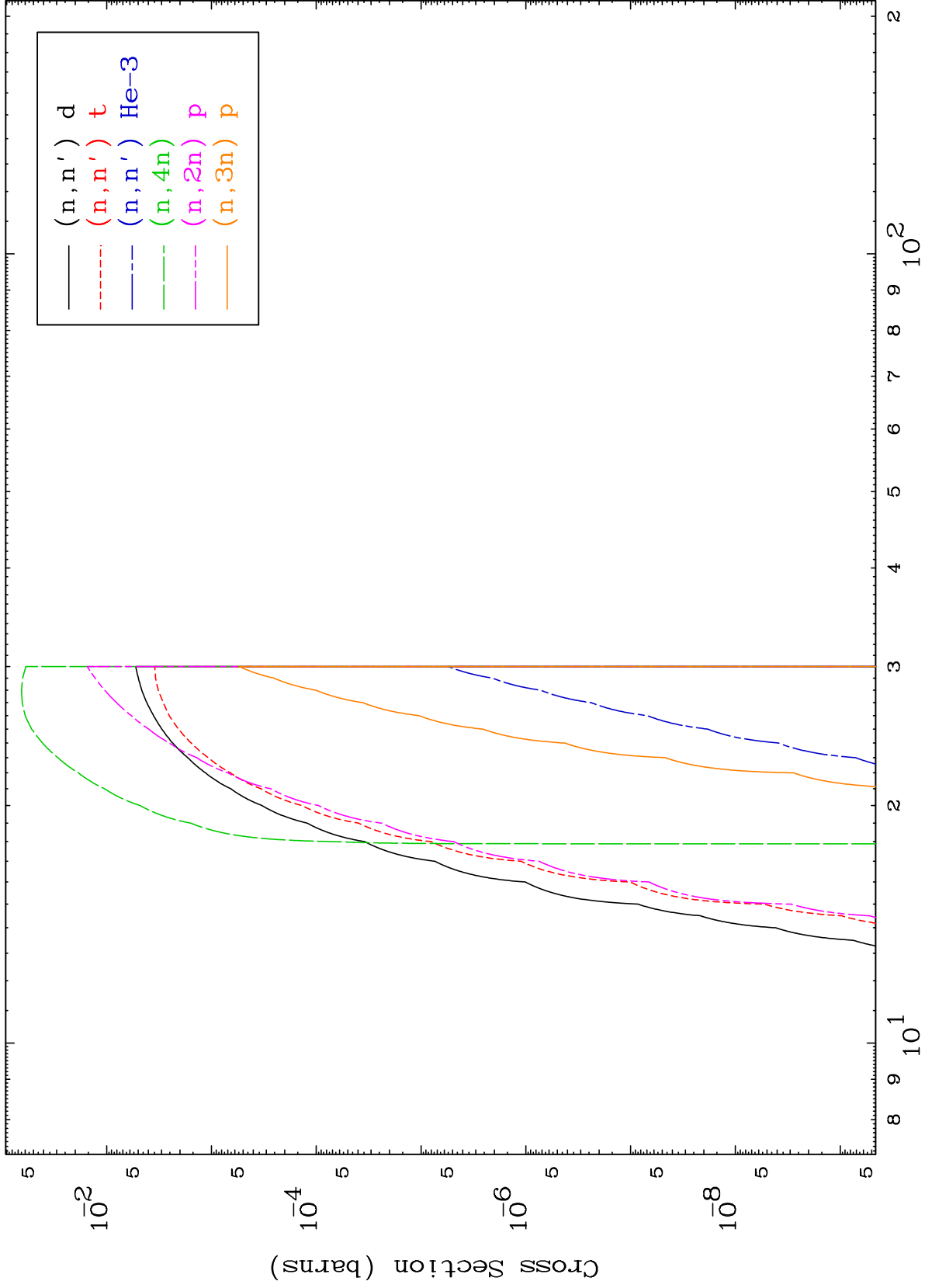


MAT 9750

Neutron Production
293 Kelvin Cross Sections

97-Bk-248

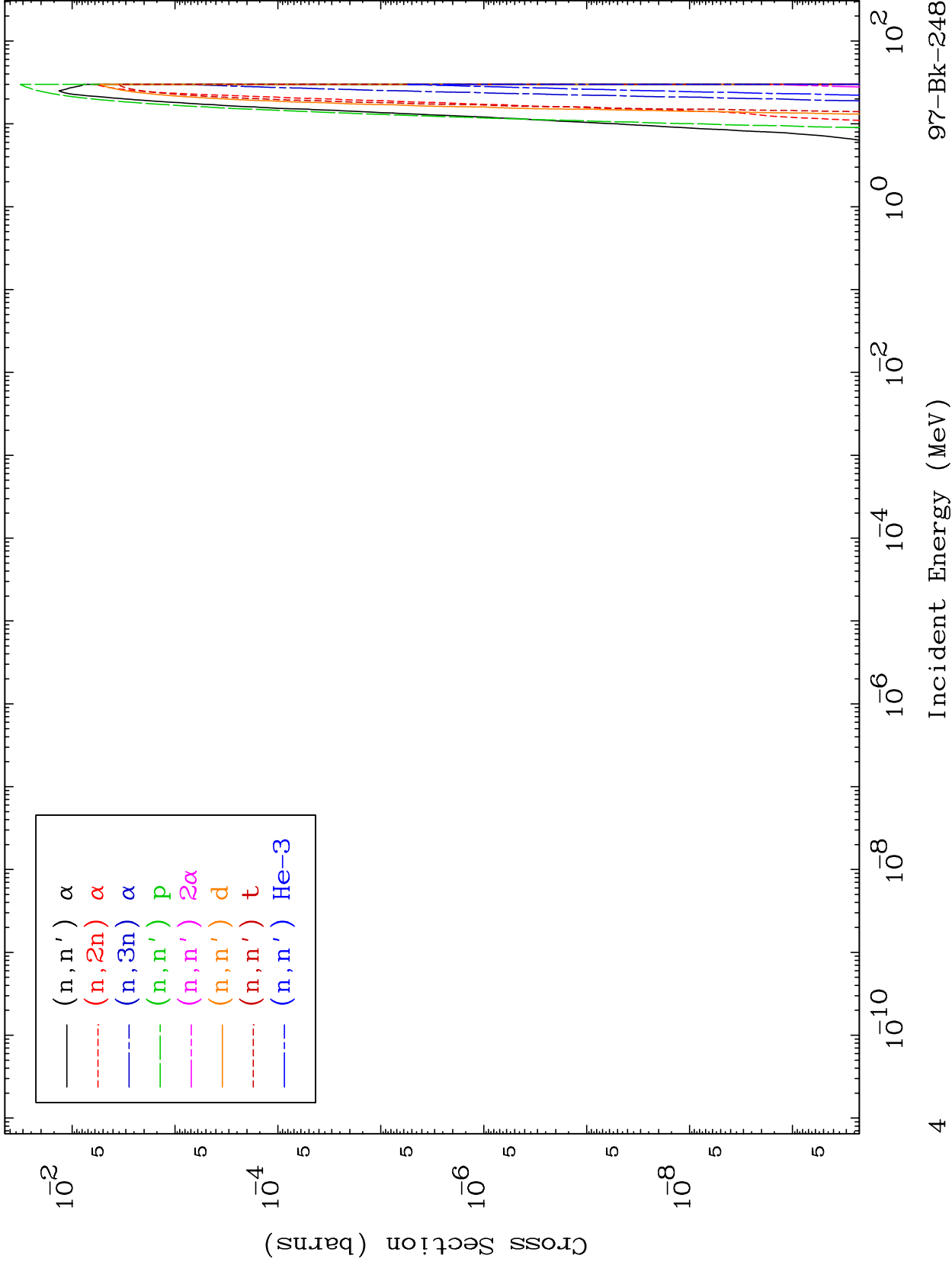




MAT 9750

Charged Particle
293 Kelvin Cross Sections

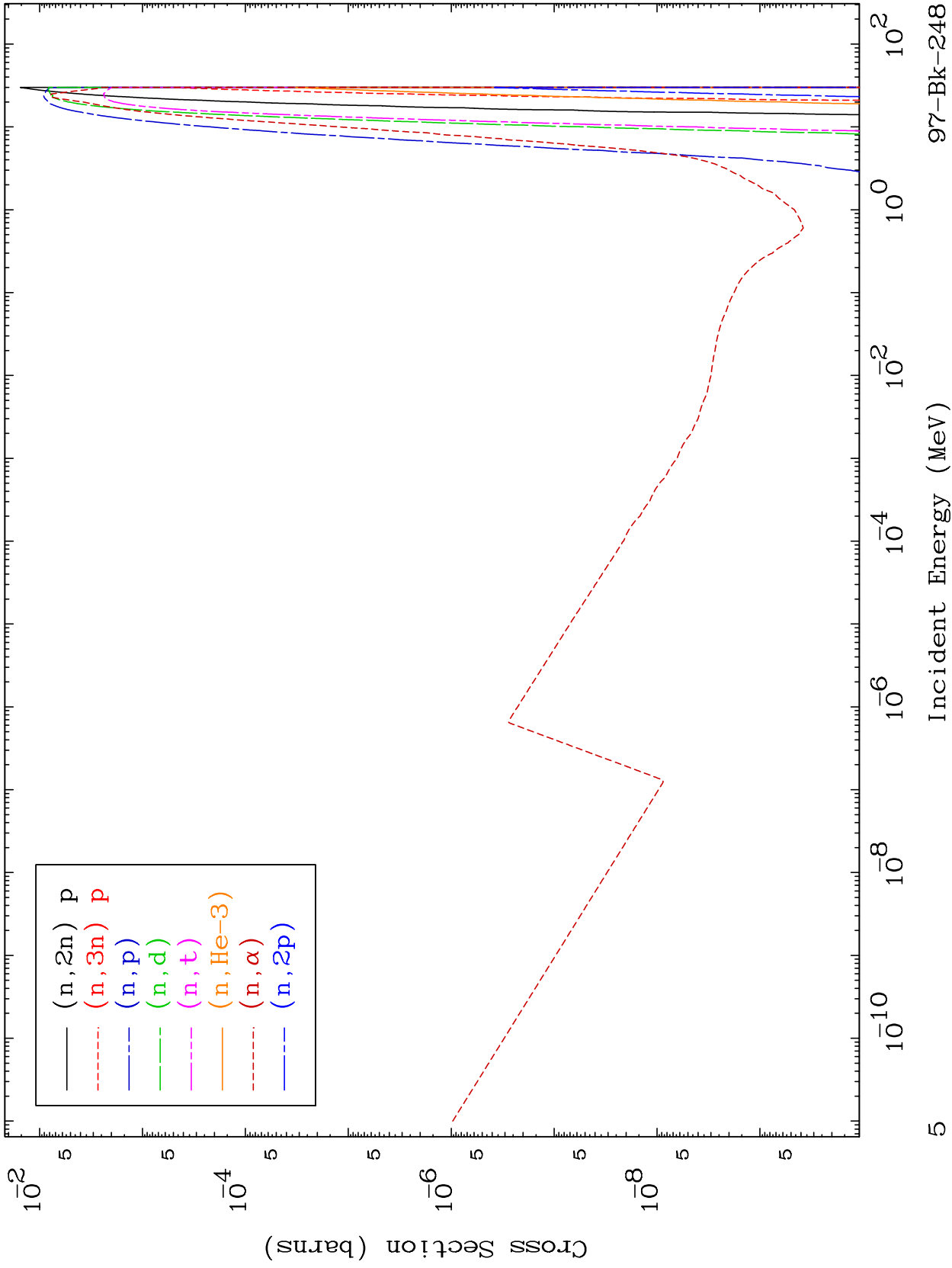
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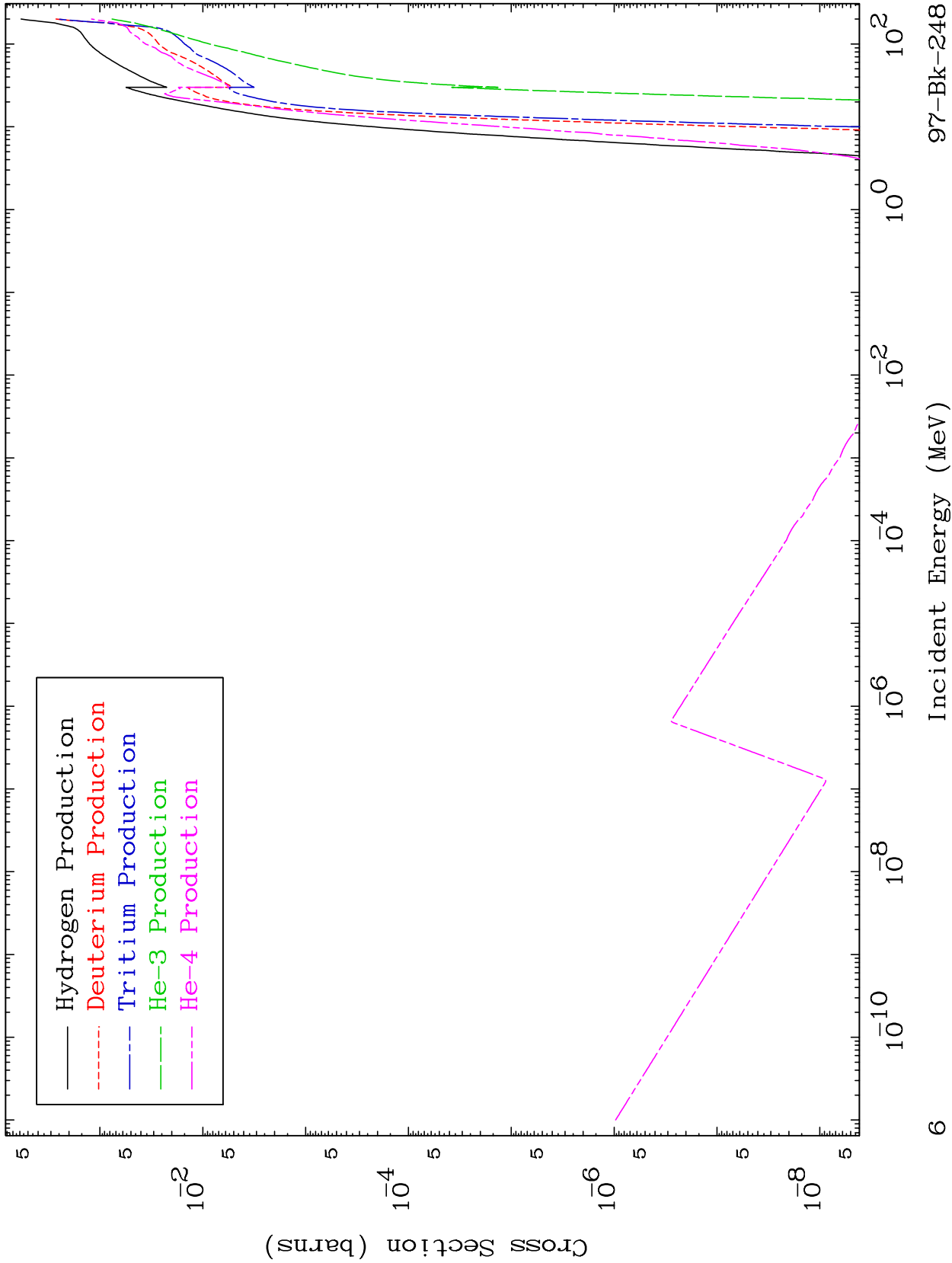


MAT 9750

Charged Particle
293 Kelvin Cross Sections

97-Bk-248

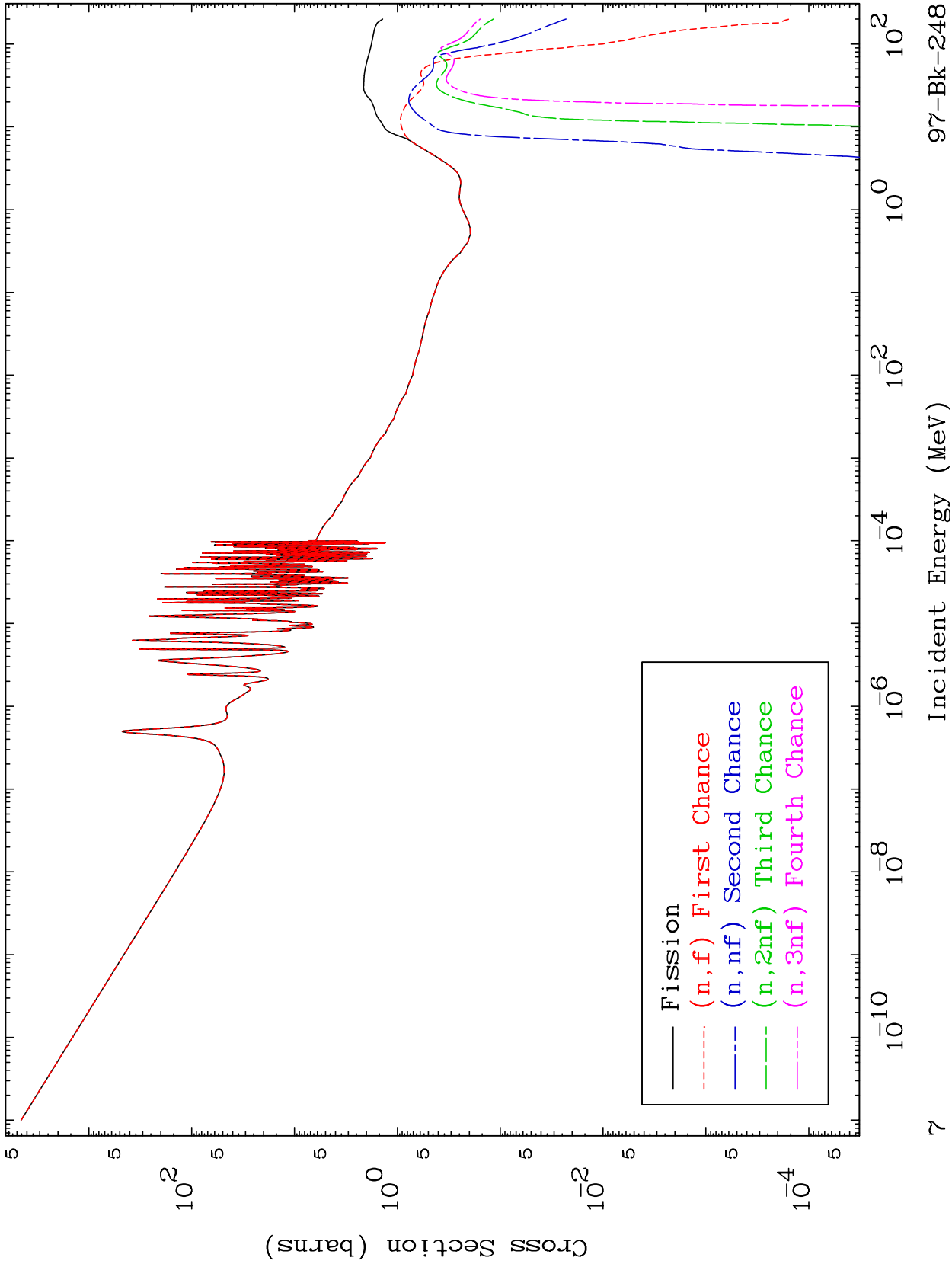




MAT 9750

Fission
293 Kelvin Cross Sections

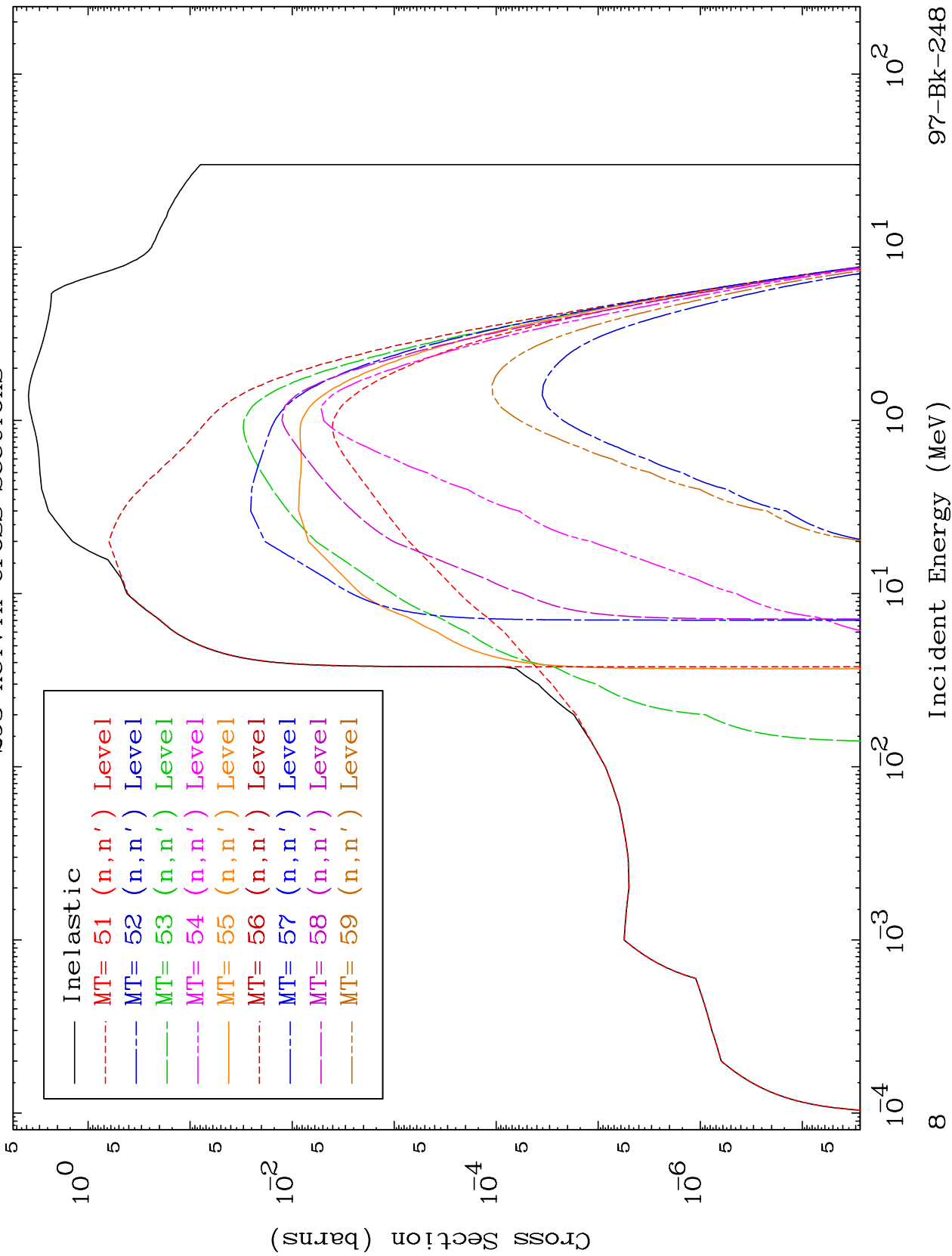
97-Bk-248



MAT 9750

293 Kelvin Cross Sections

97-Bk-248



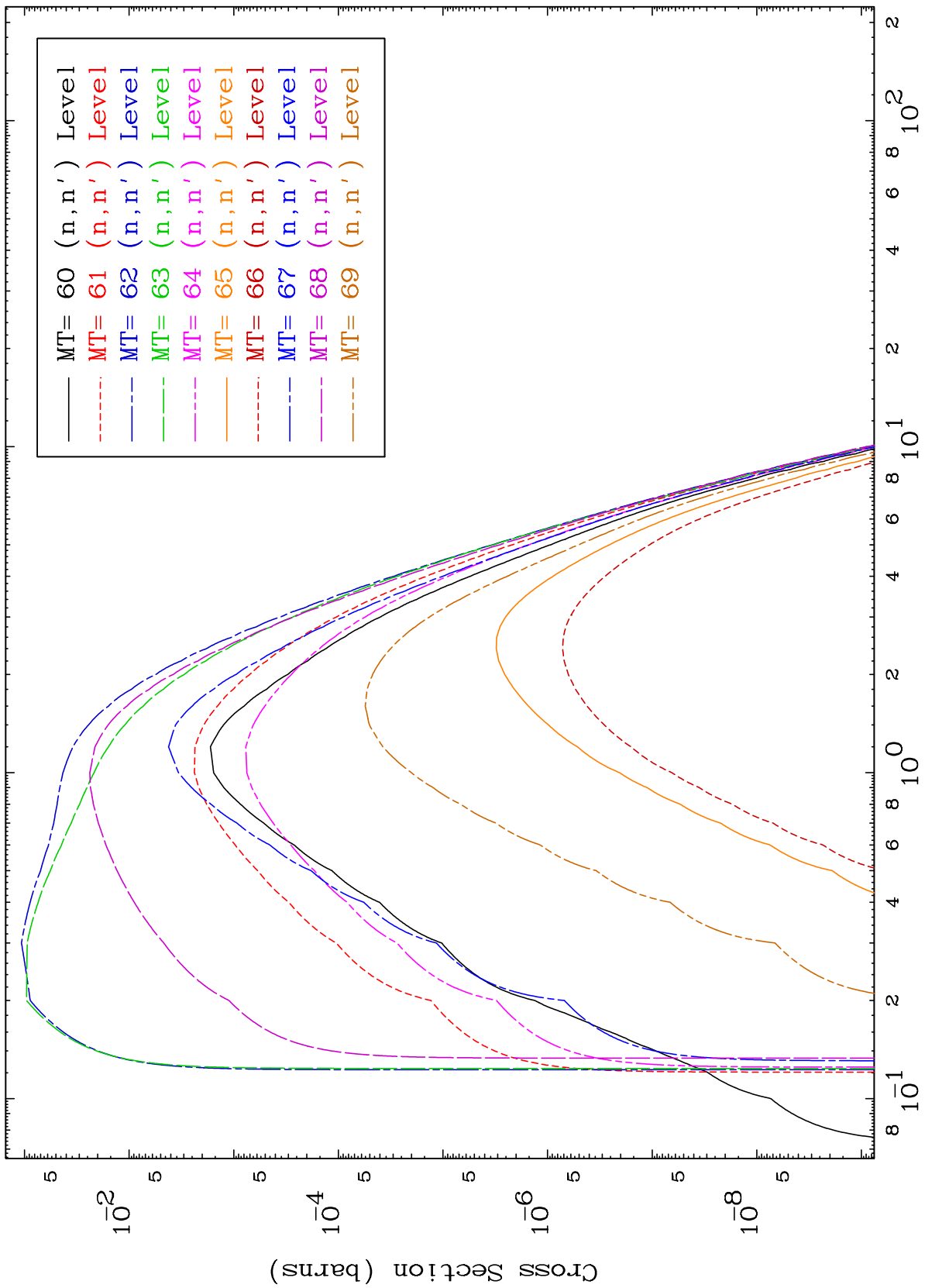
8

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(n,n') Level

97-Bk-248

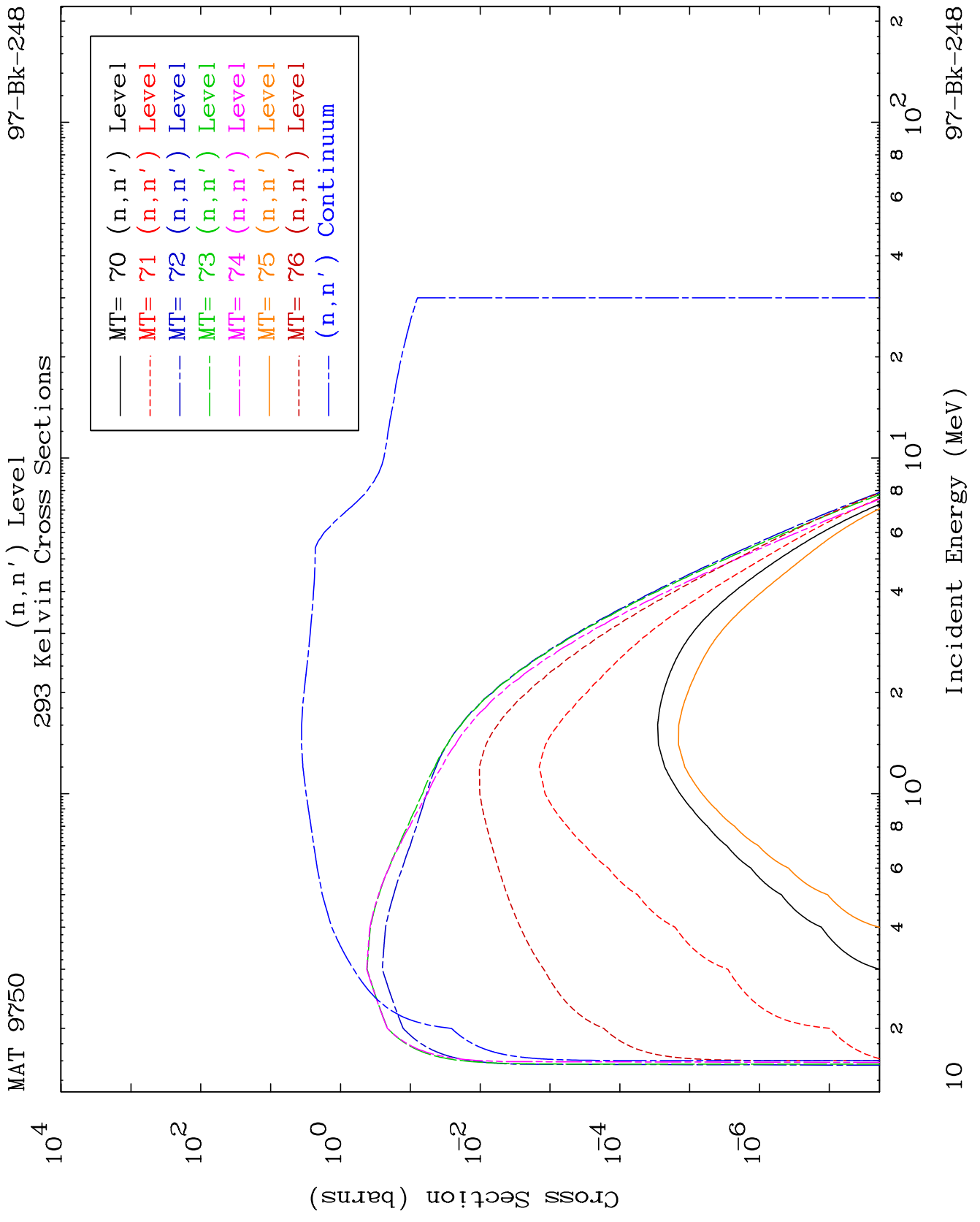
293 Kelvin Cross Sections



9

Incident Energy (MeV)

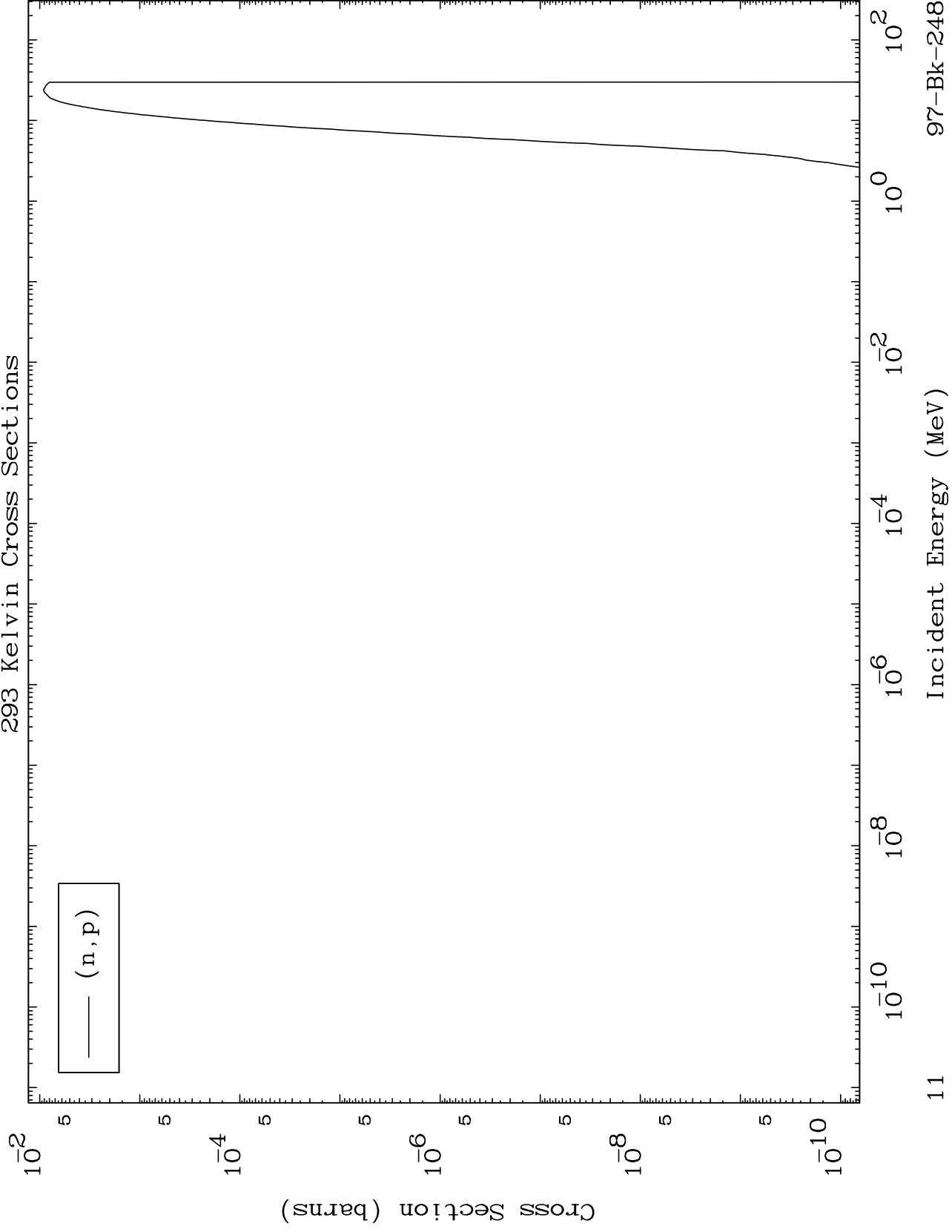
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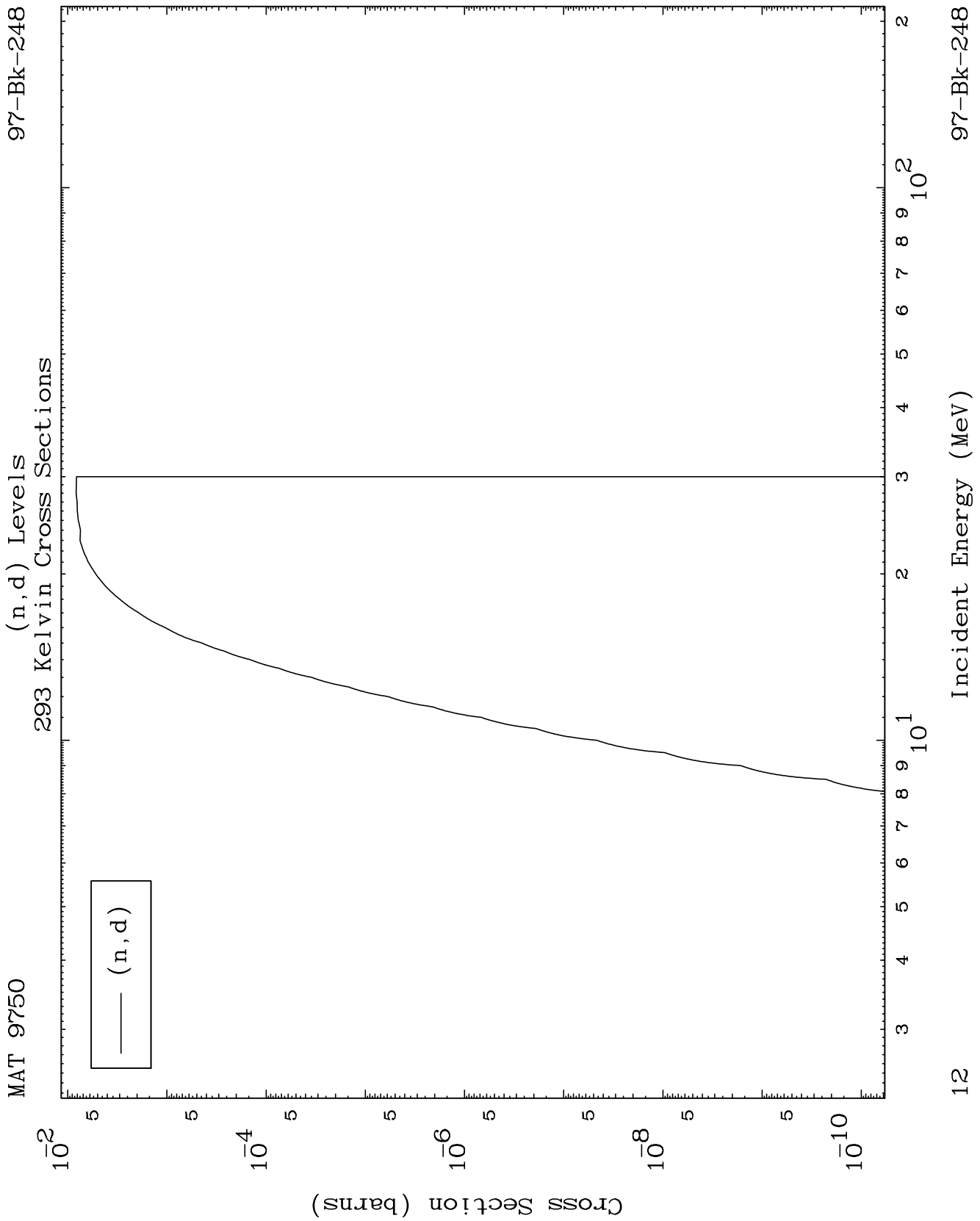


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(n,p) Levels
293 Kelvin Cross Sections

97-Bk-248

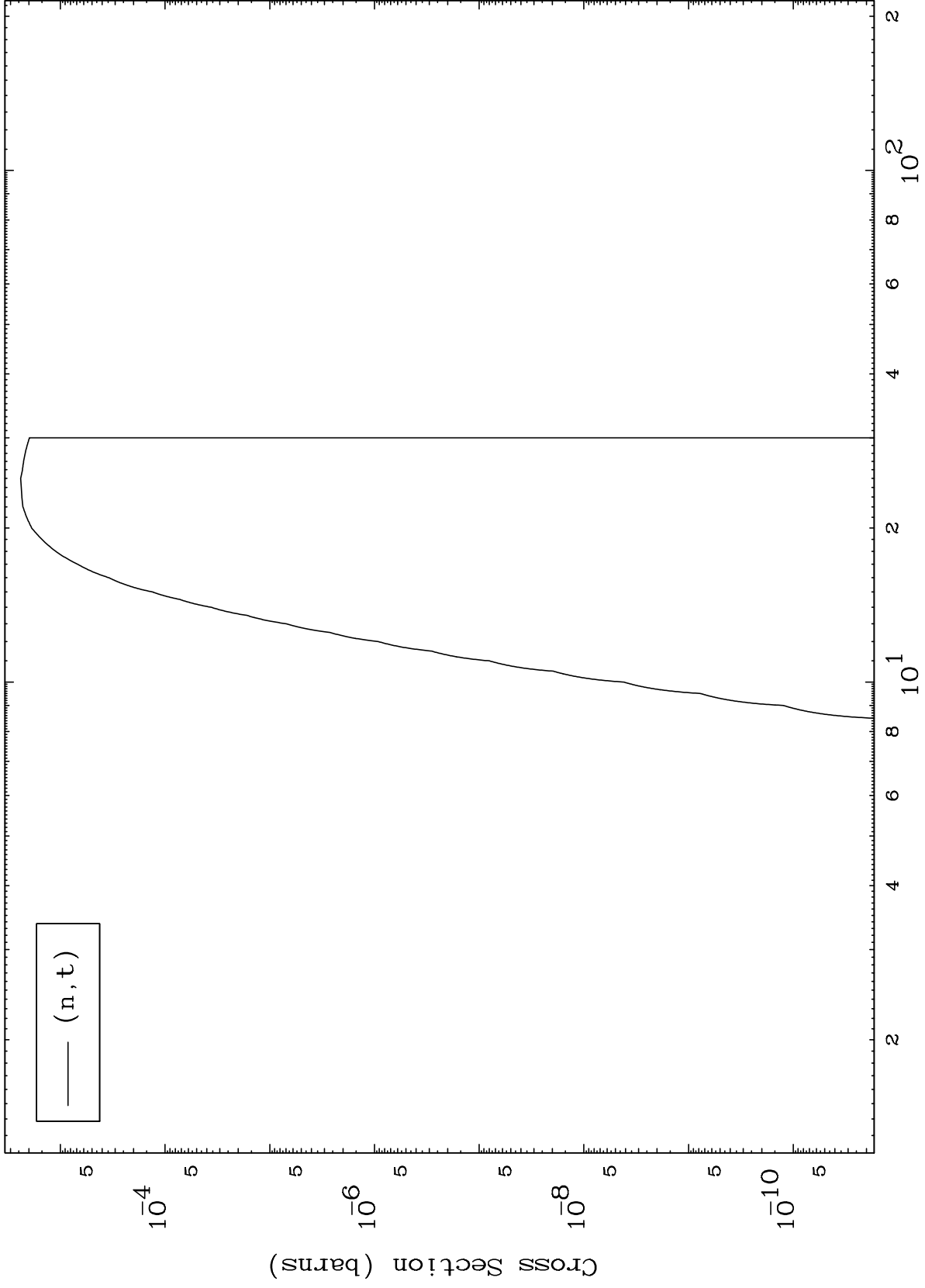




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(n,t) Levels
293 Kelvin Cross Sections

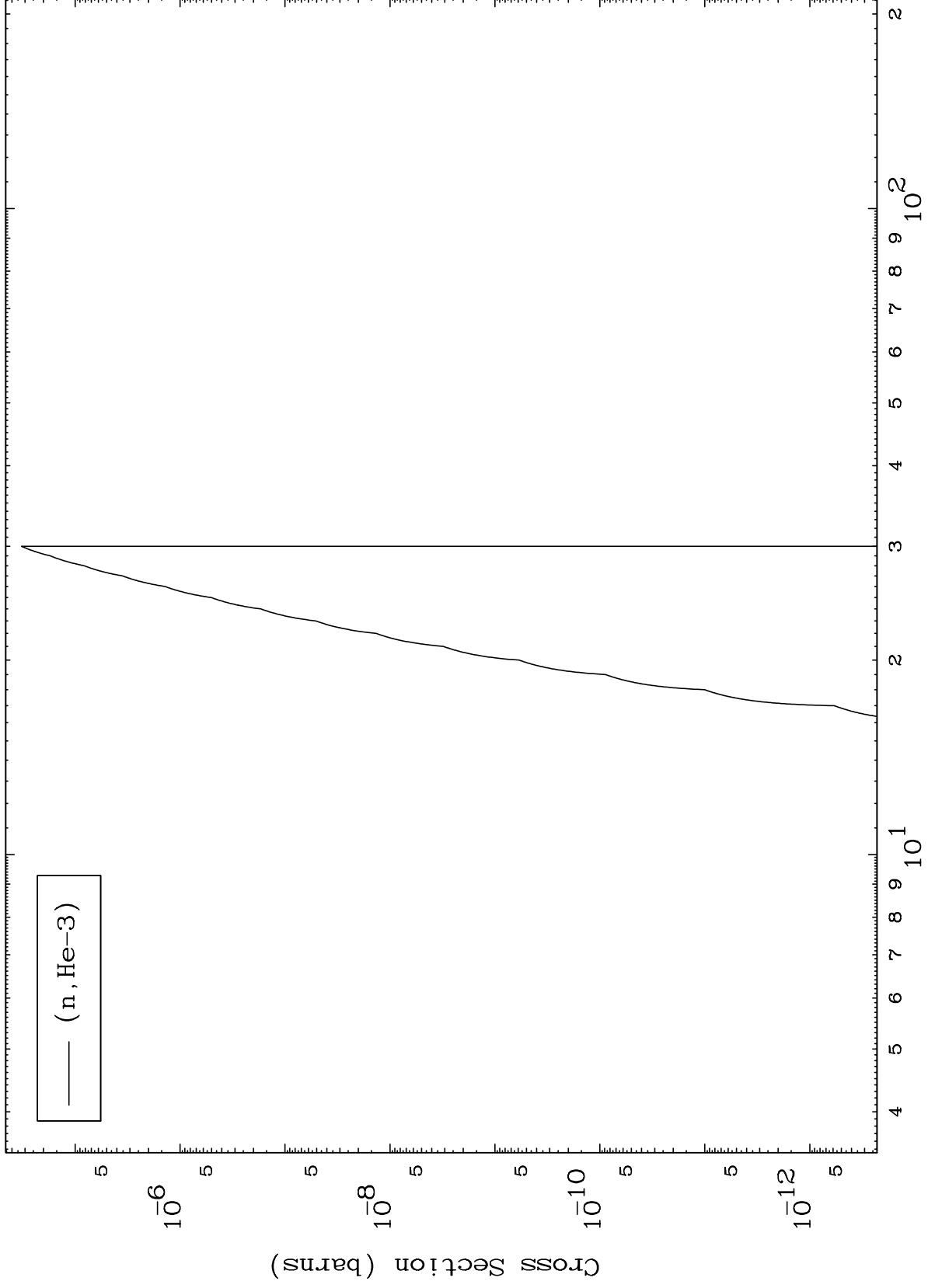
97-Bk-248



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(n,He3) Levels
293 Kelvin Cross Sections

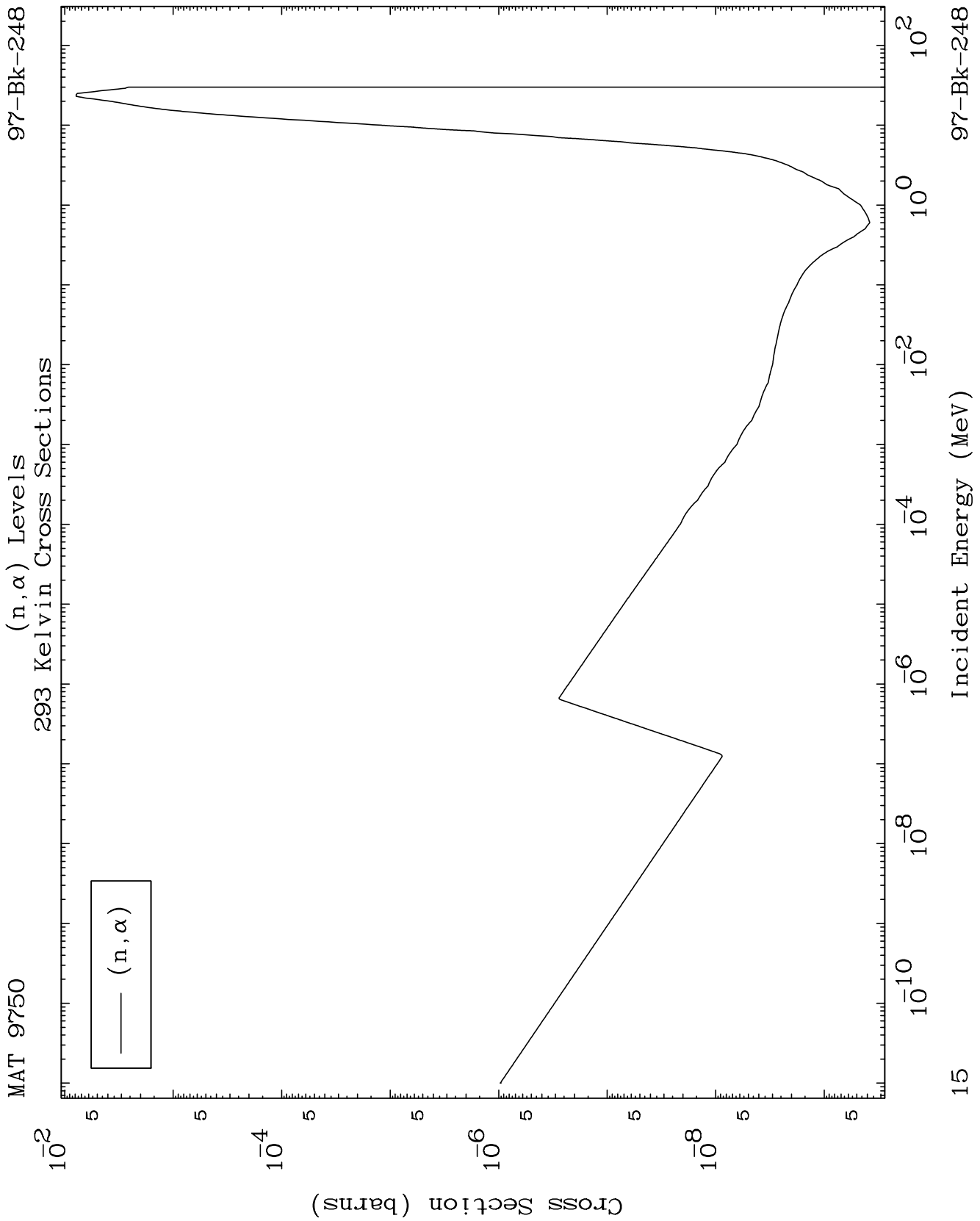
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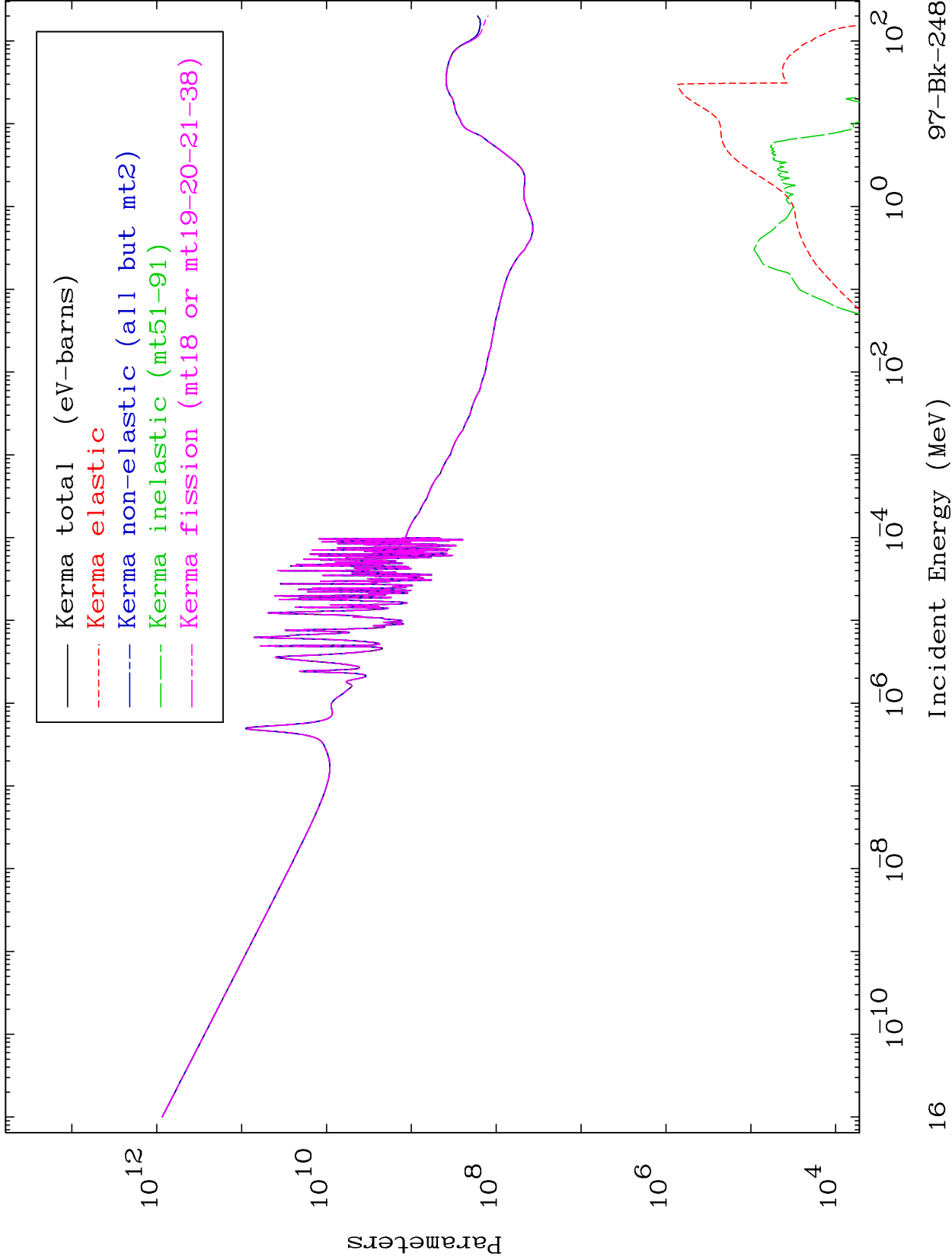


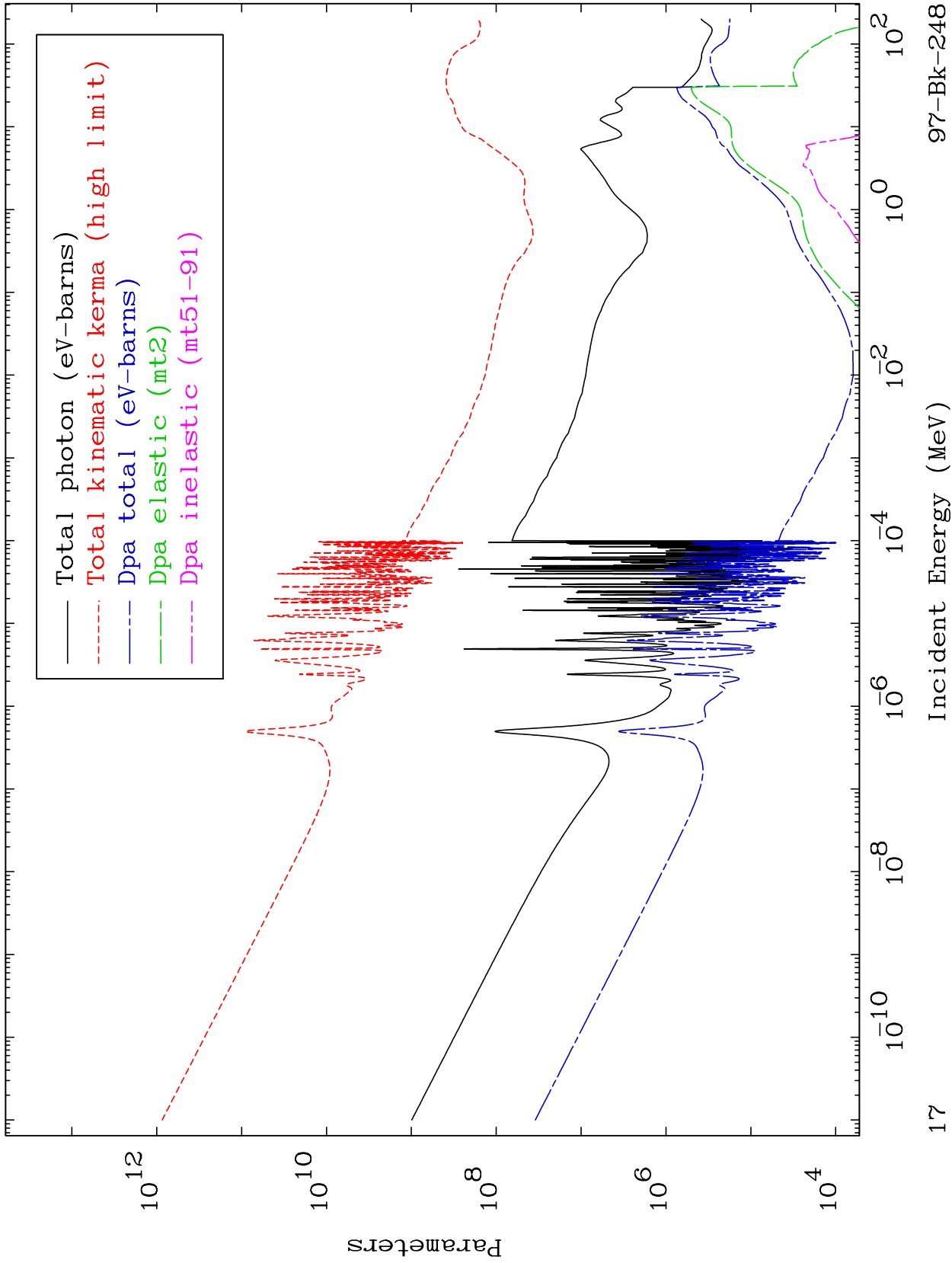
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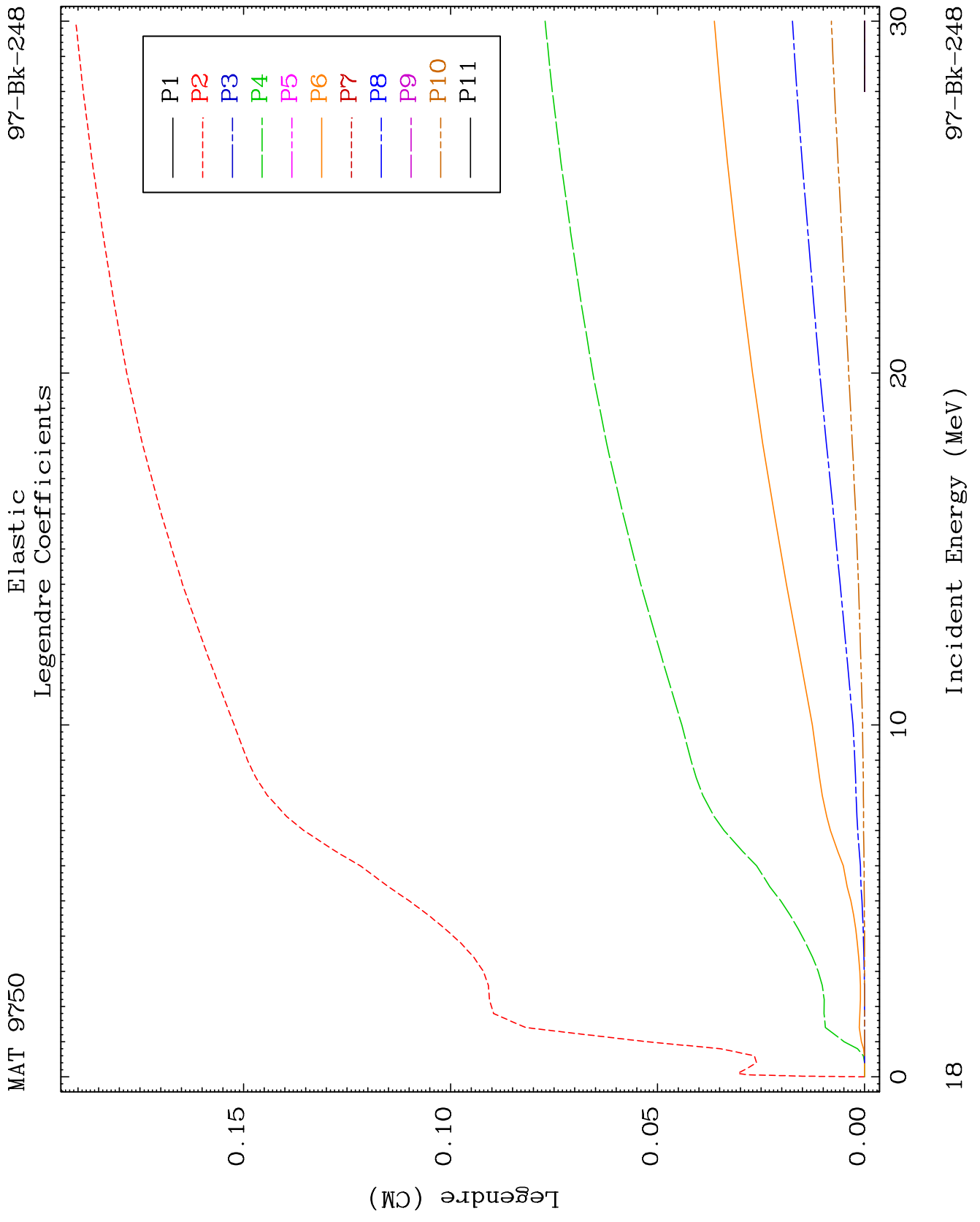
Incident Energy (MeV)

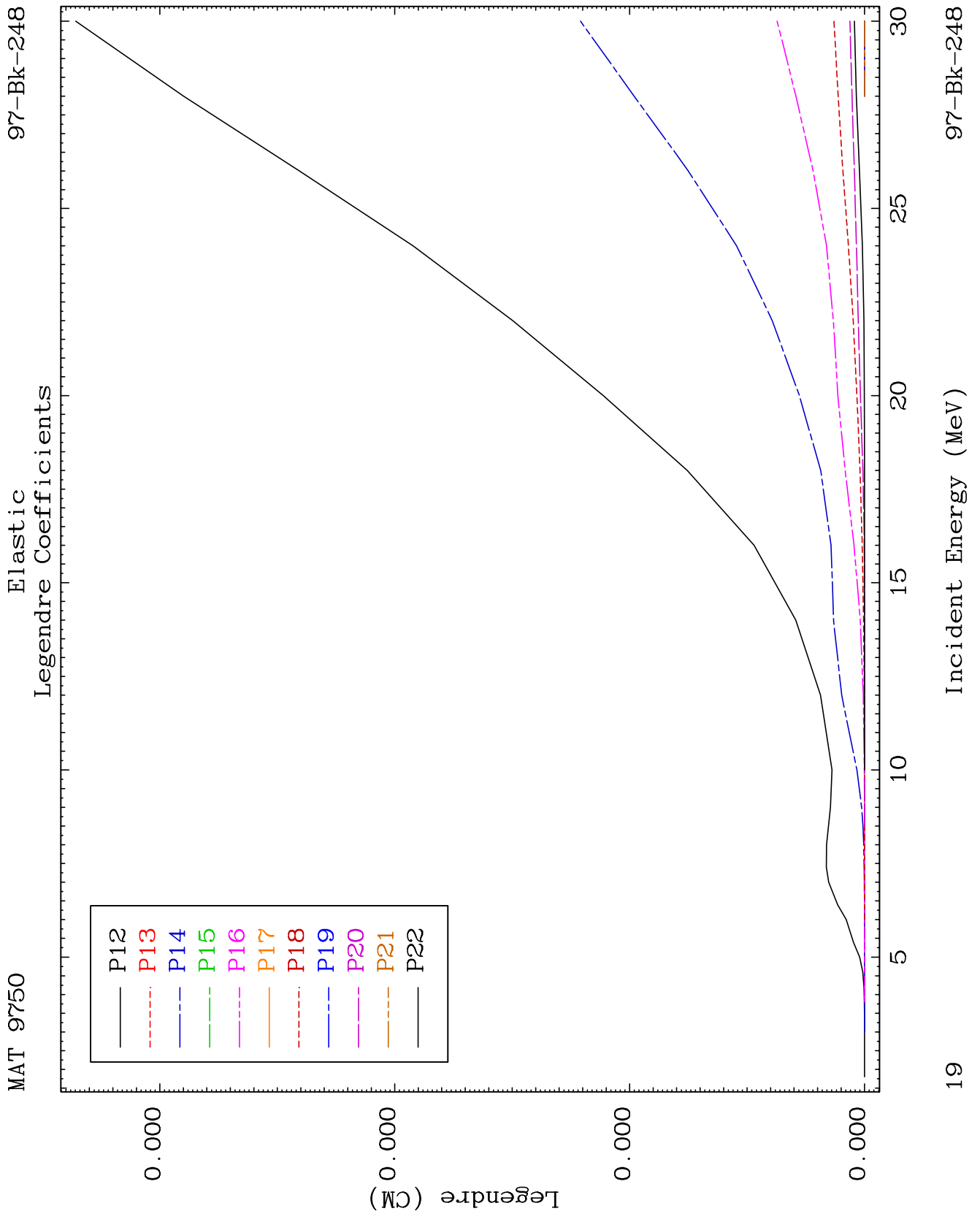
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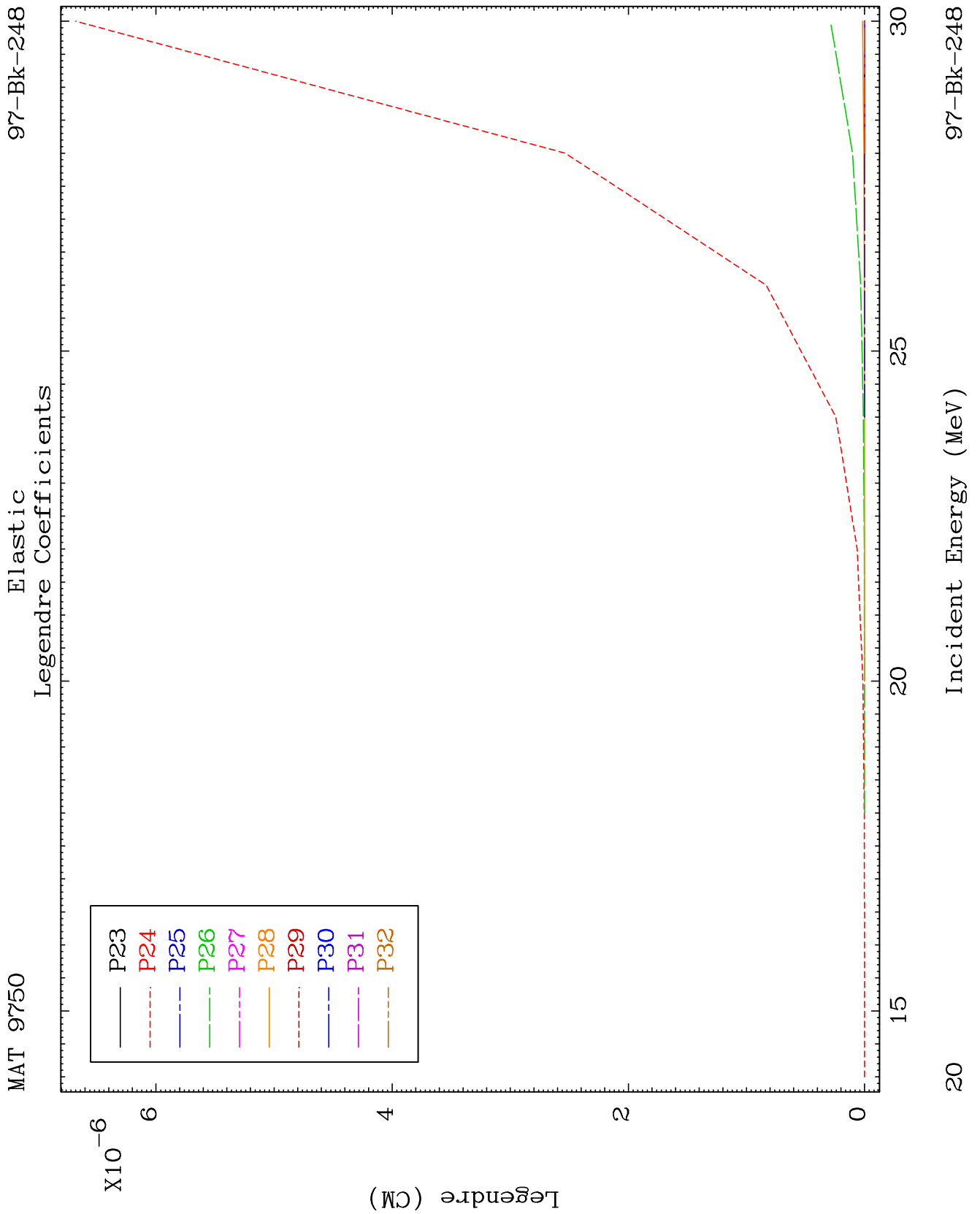








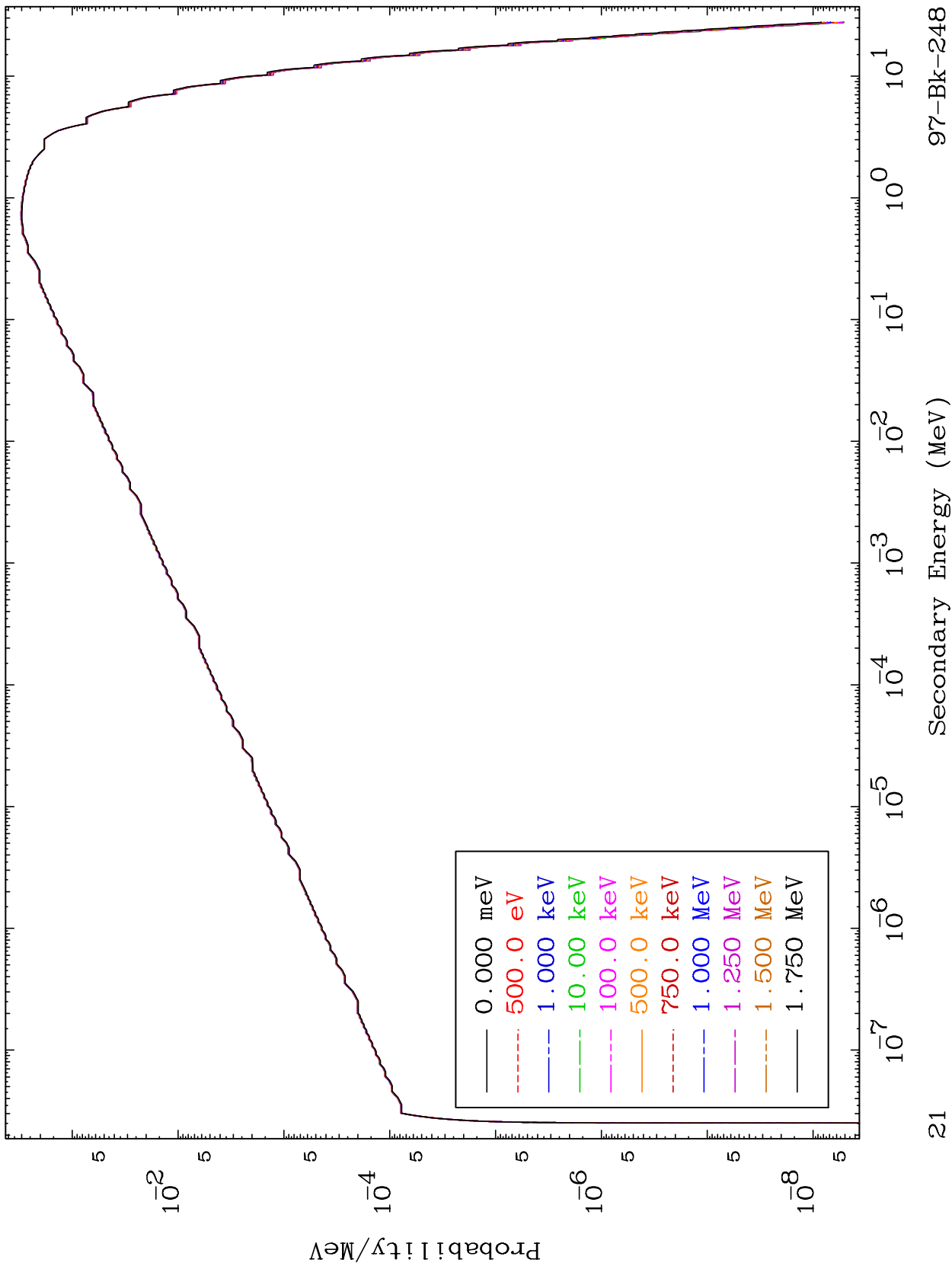




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Fission Energy Distribution

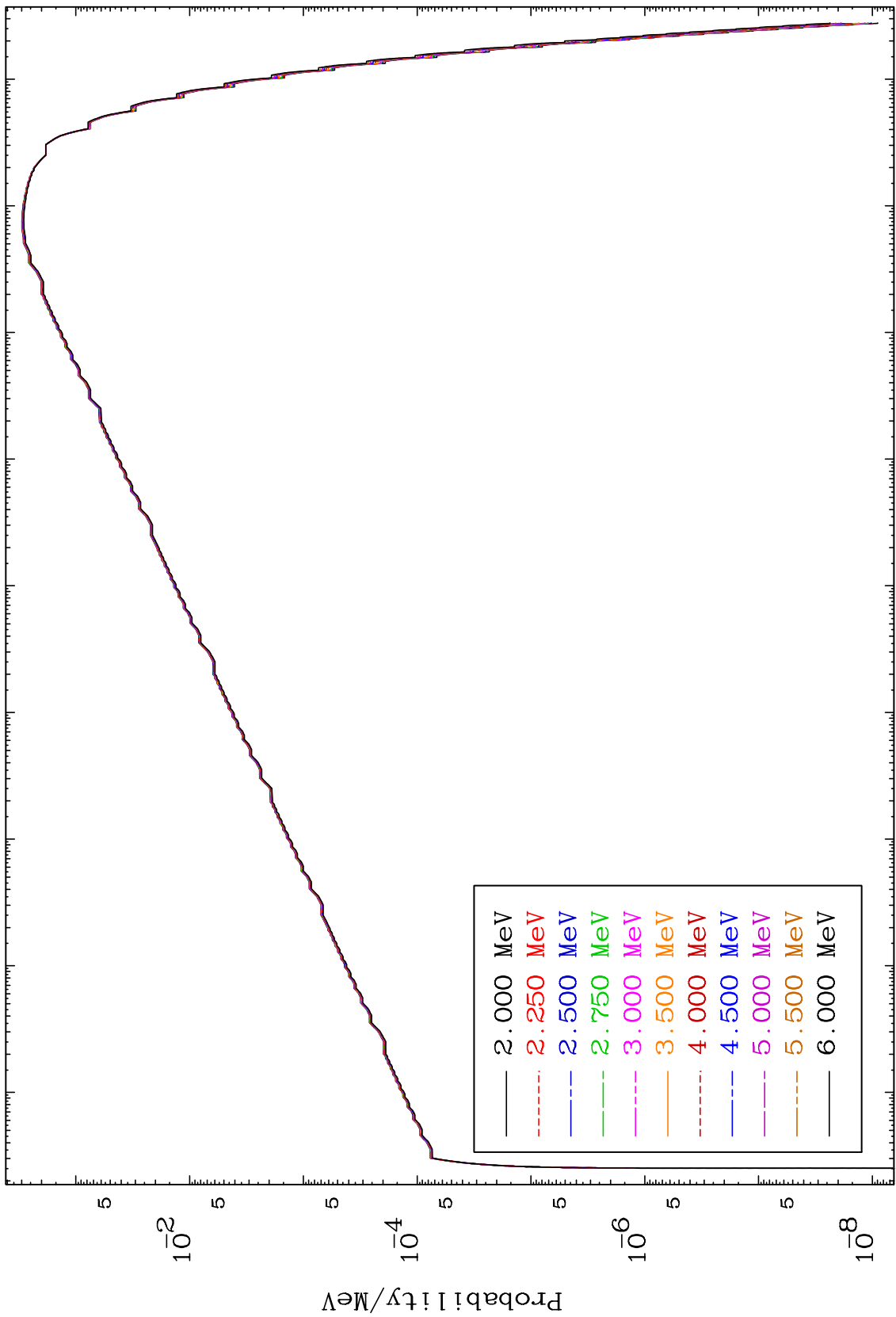
97-Bk-248



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Fission Energy Distribution

97-Bk-248

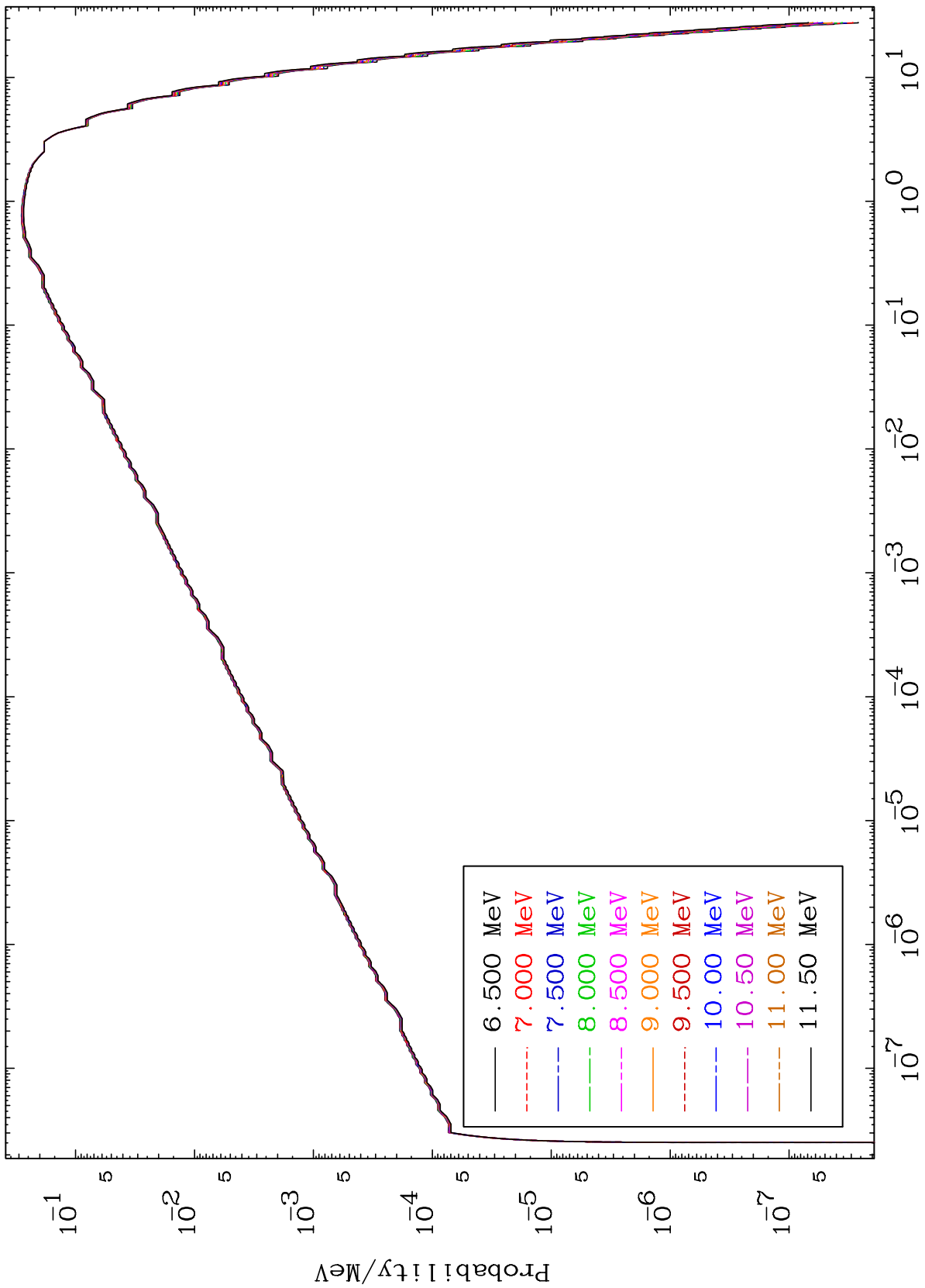


97-Bk-248

MAT 9750

Fission Energy Distribution

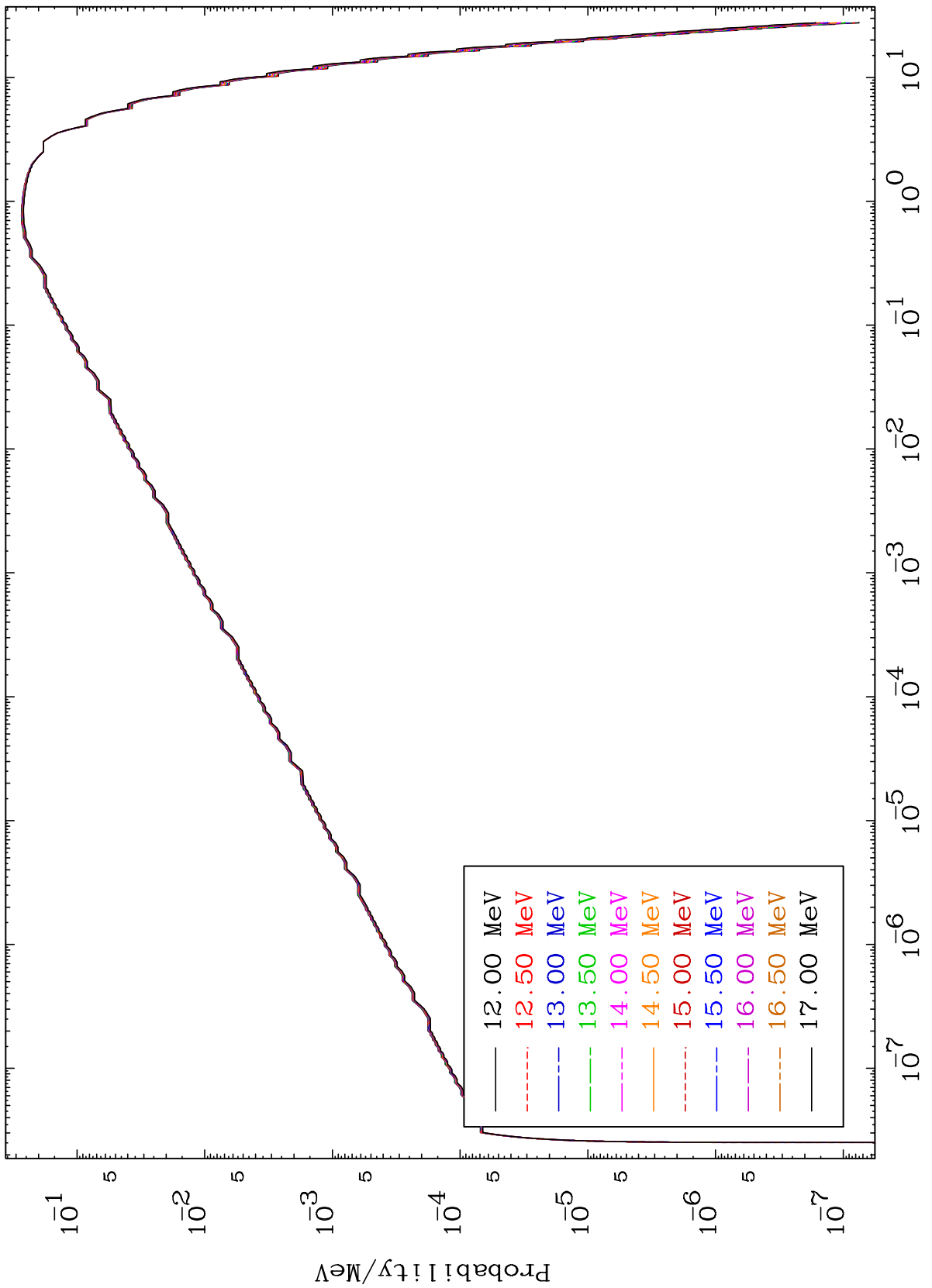
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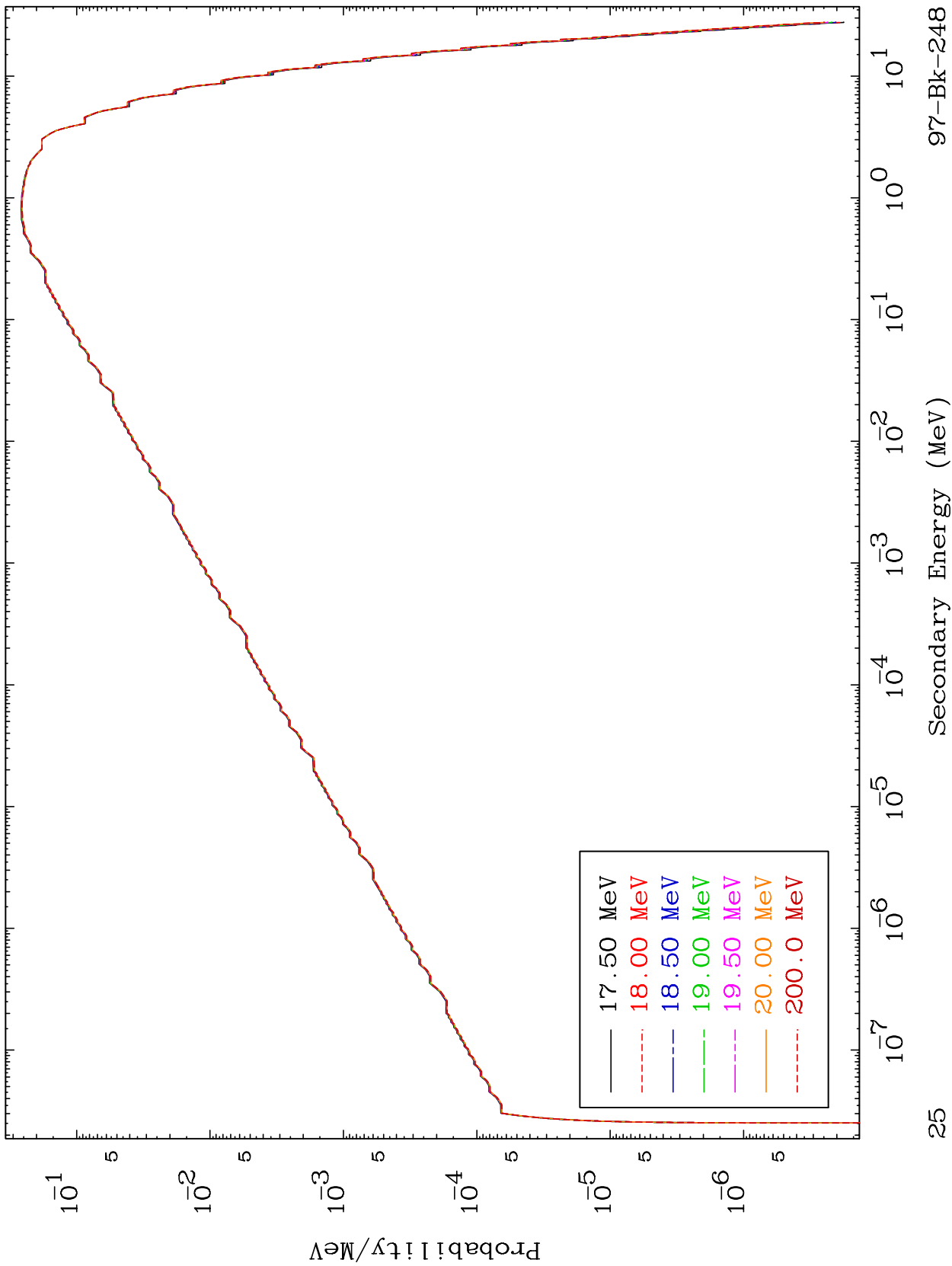
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Fission Energy Distribution

97-Bk-248



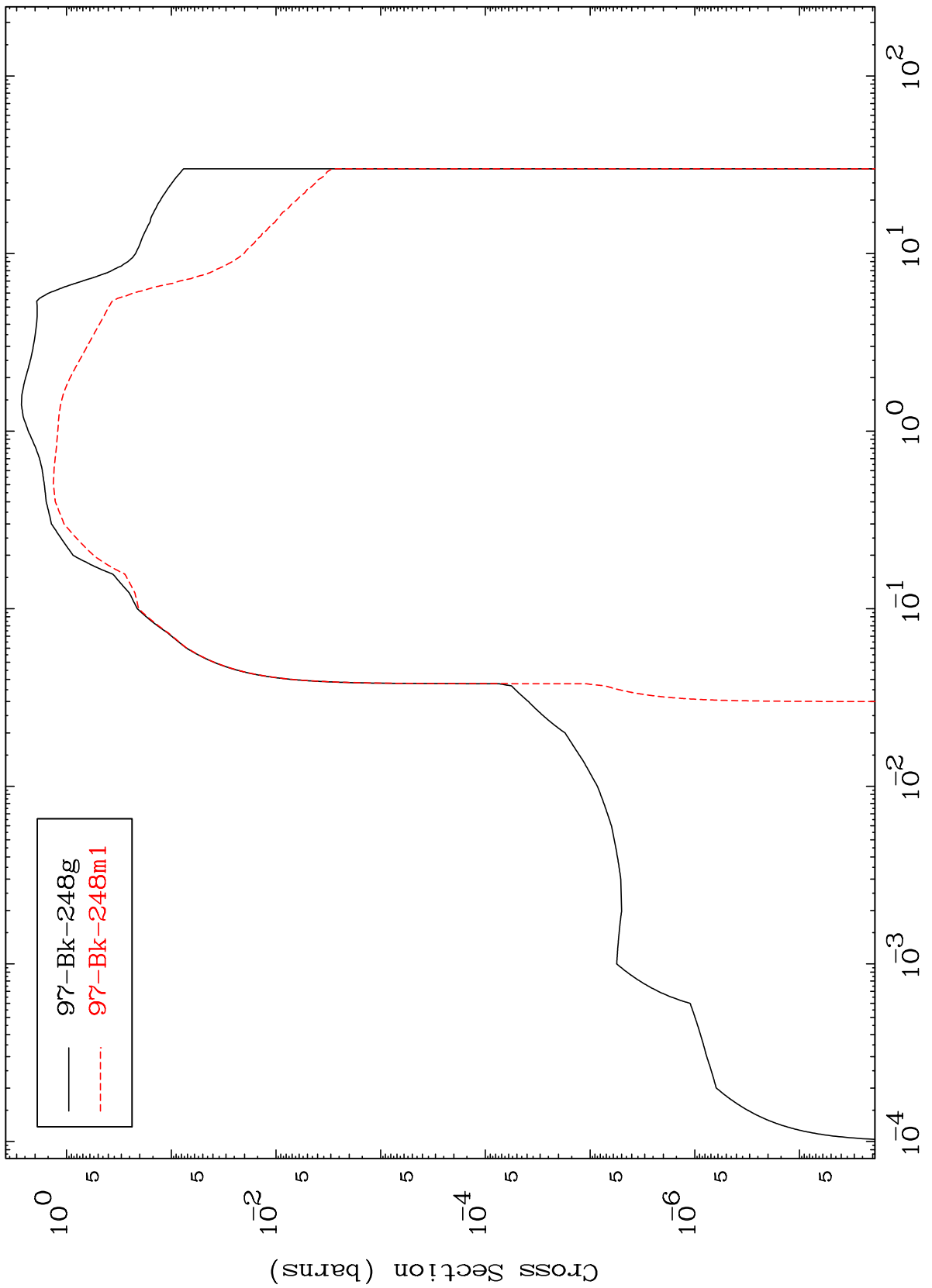
Fission
Energy Distribution



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97-Bk-248

Inelastic
Radionuclide Production Cross Section



97-Bk-248

Incident Energy (MeV)

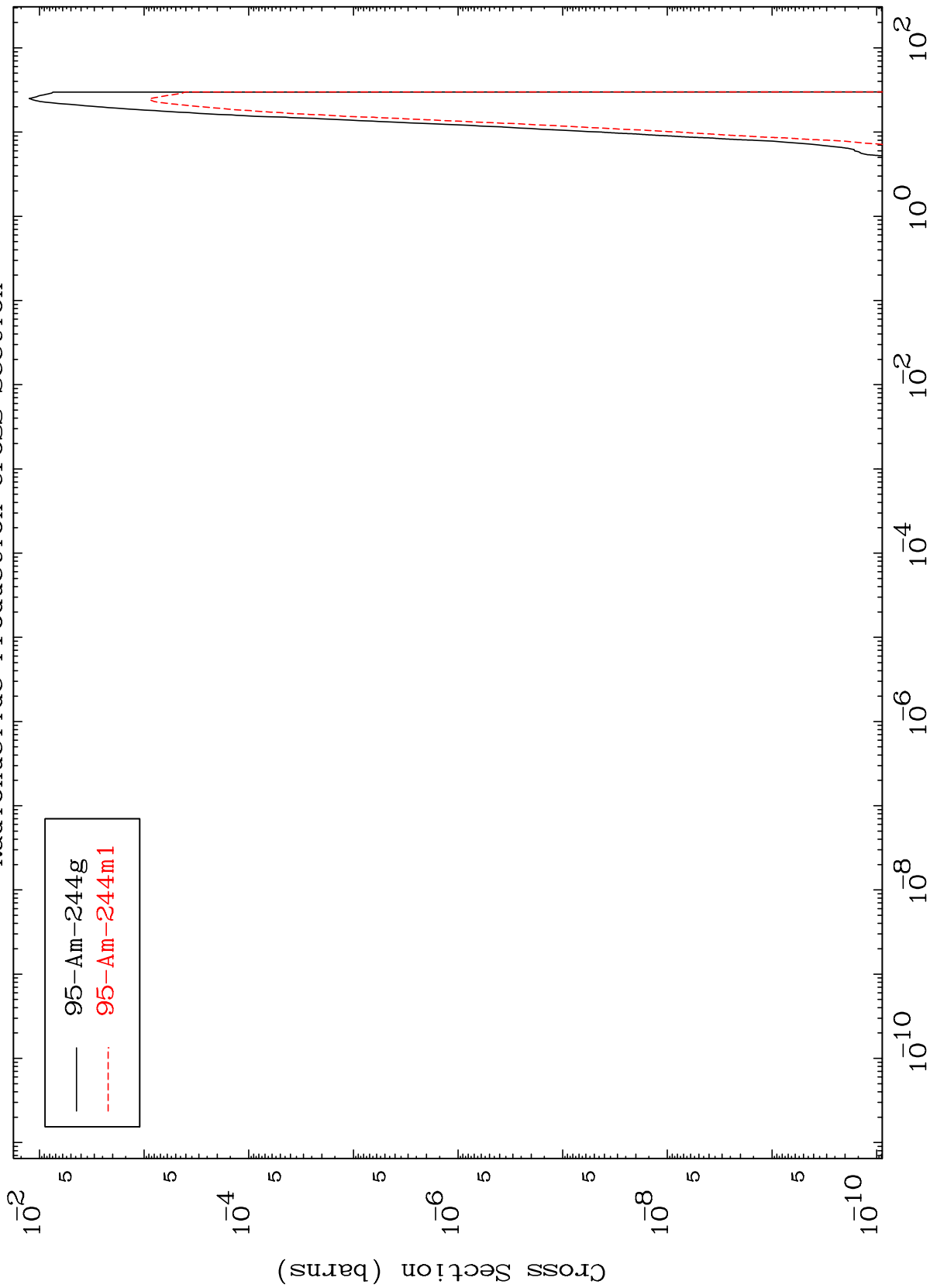
26

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$(n, n') \alpha$

97-Bk-248

Radionuclide Production Cross Section

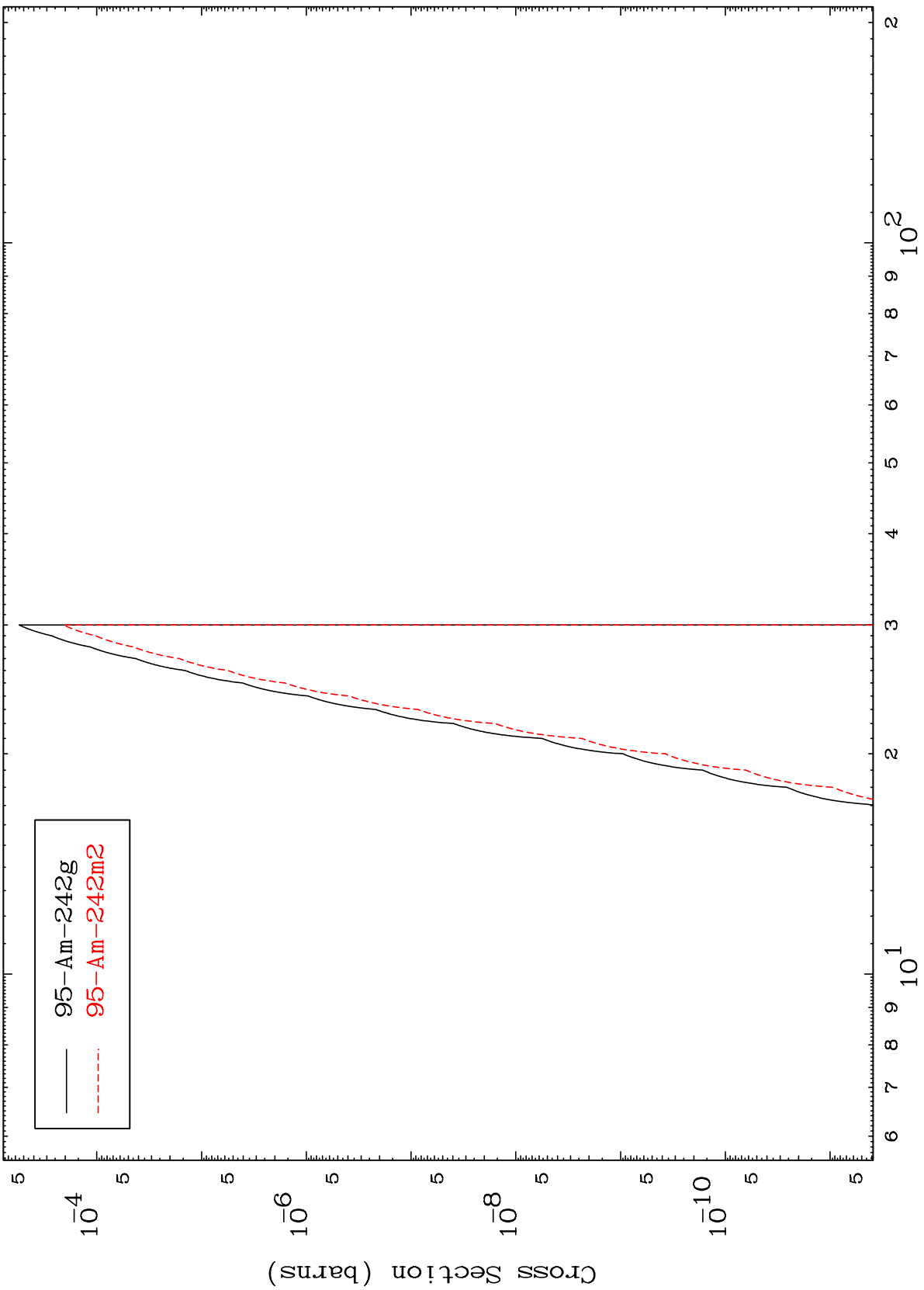


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(n,3n) α

97-Bk-248

Radionuclide Production Cross Section



95-Am-242g
95-Am-242m2

28

Incident Energy (MeV)

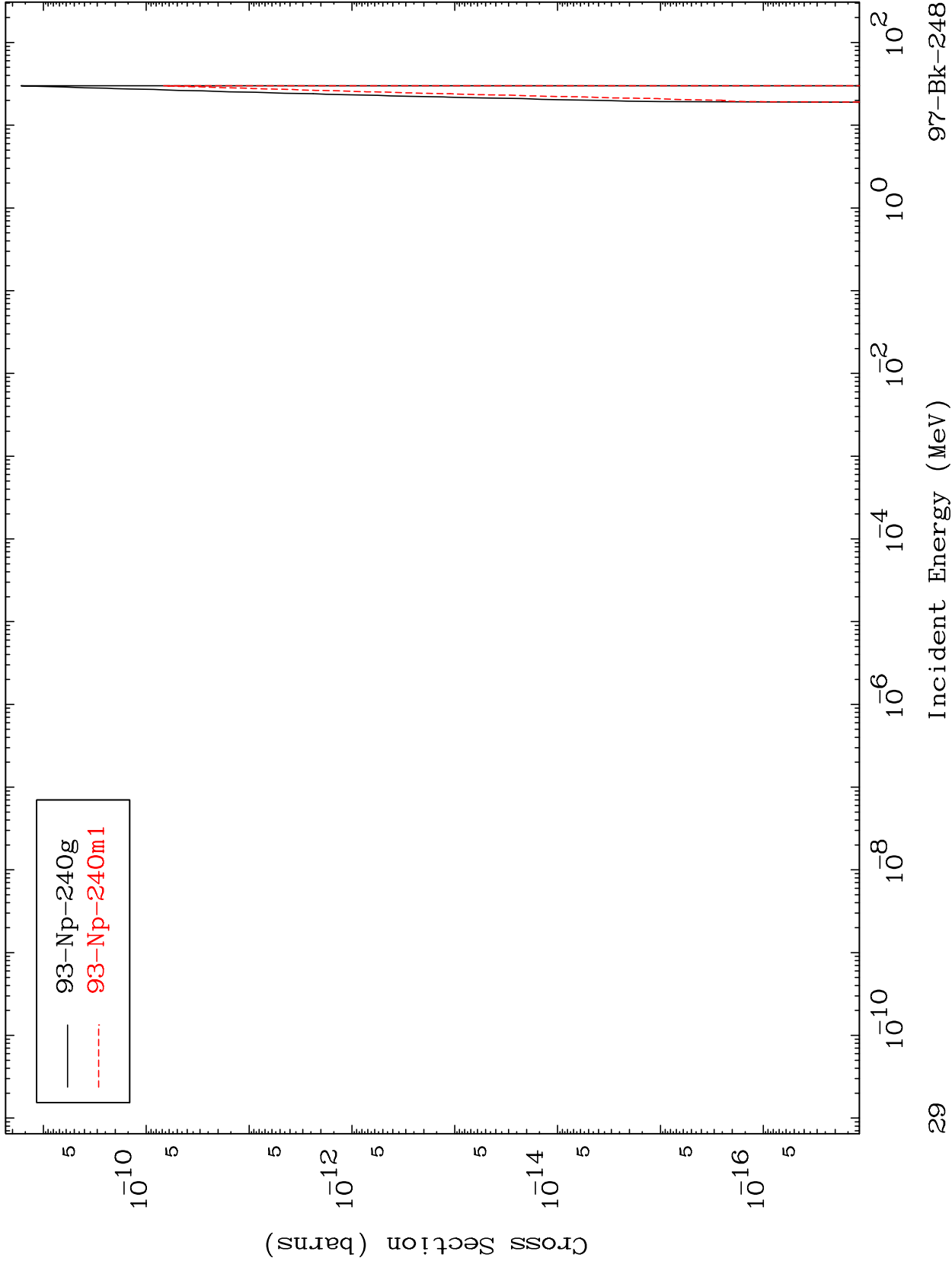
97-Bk-248

MAT 9750

(n,n') 2 α

97-Bk-248

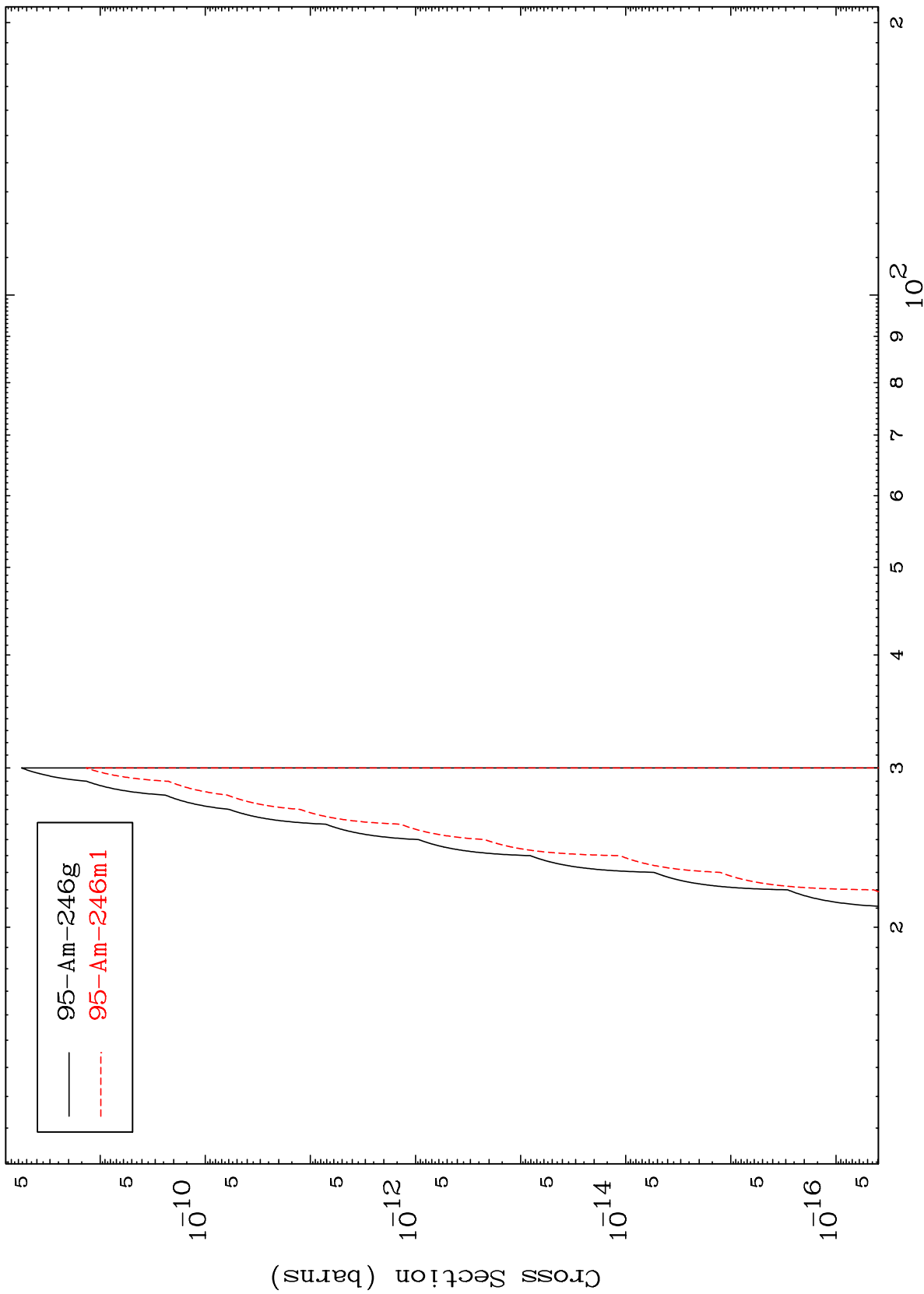
Radionuclide Production Cross Section



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97-Bk-248

(n,2n) p
Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-248

30

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(n,He-3)

97-Bk-248

Radionuclide Production Cross Section

