

Program EVALPLOT
(Version 2017-1)

by

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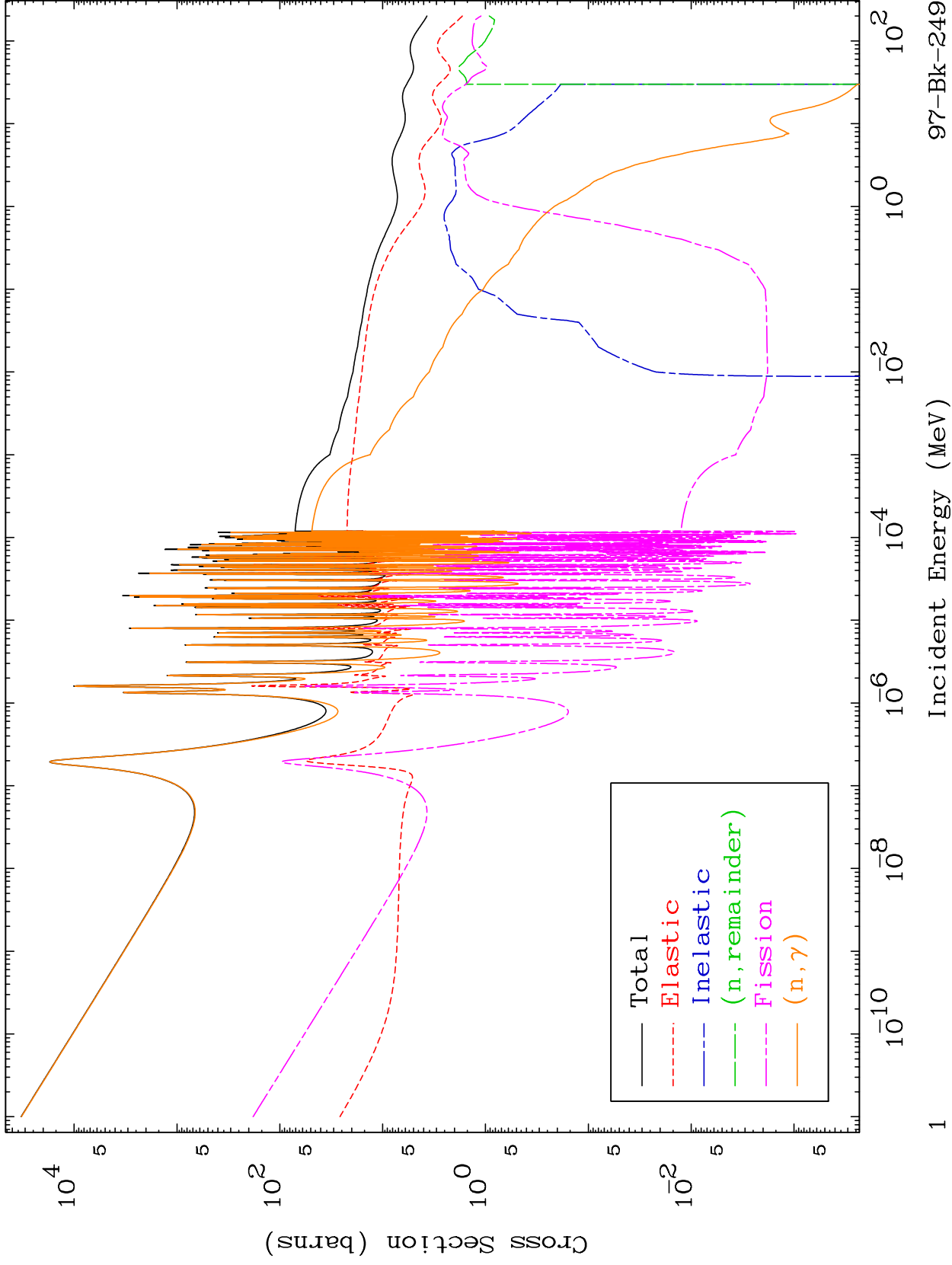
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9752

Major
293 Kelvin Cross Sections

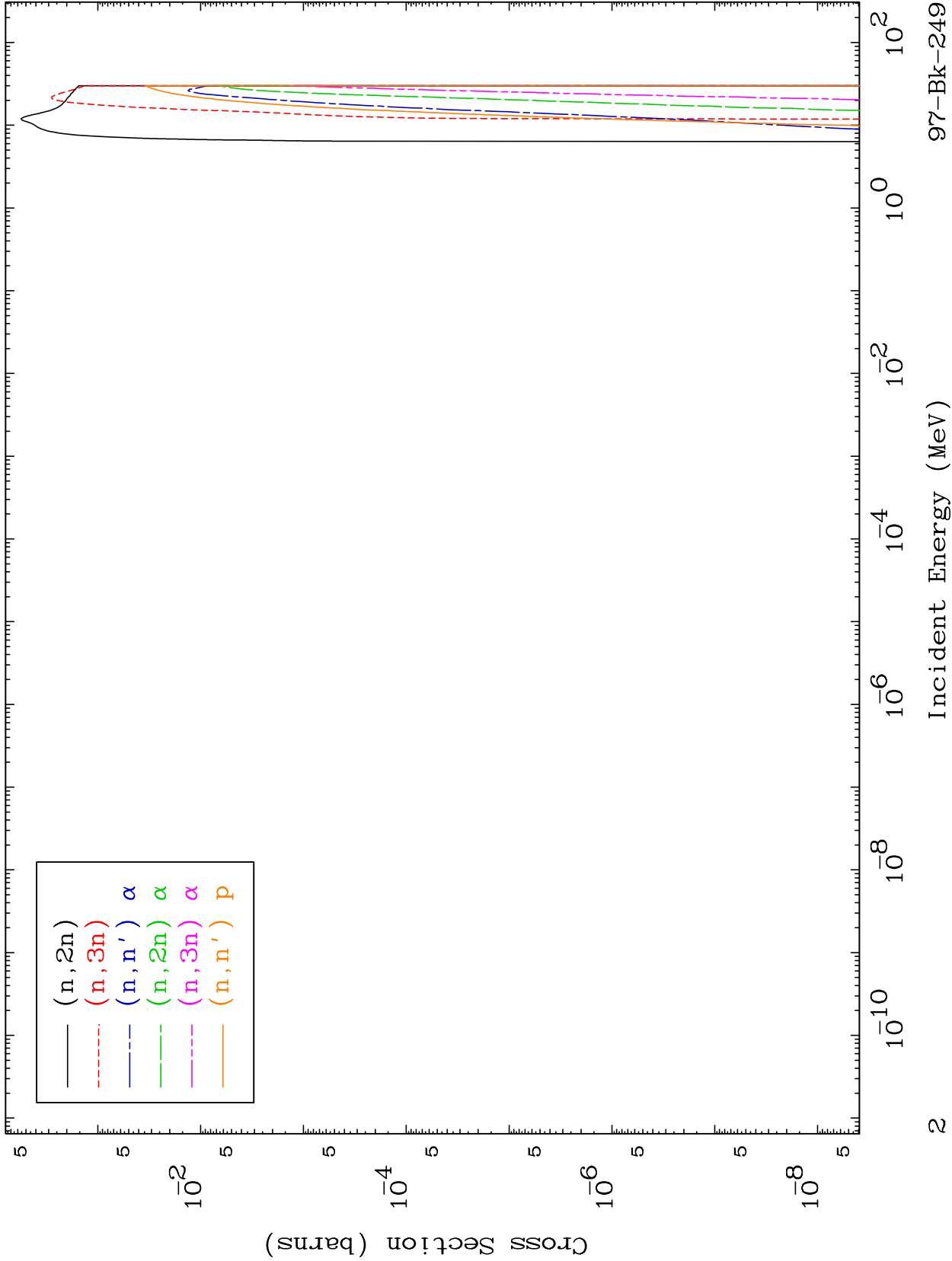
97-Bk-249



MAT 9752

Neutron Production
293 Kelvin Cross Sections

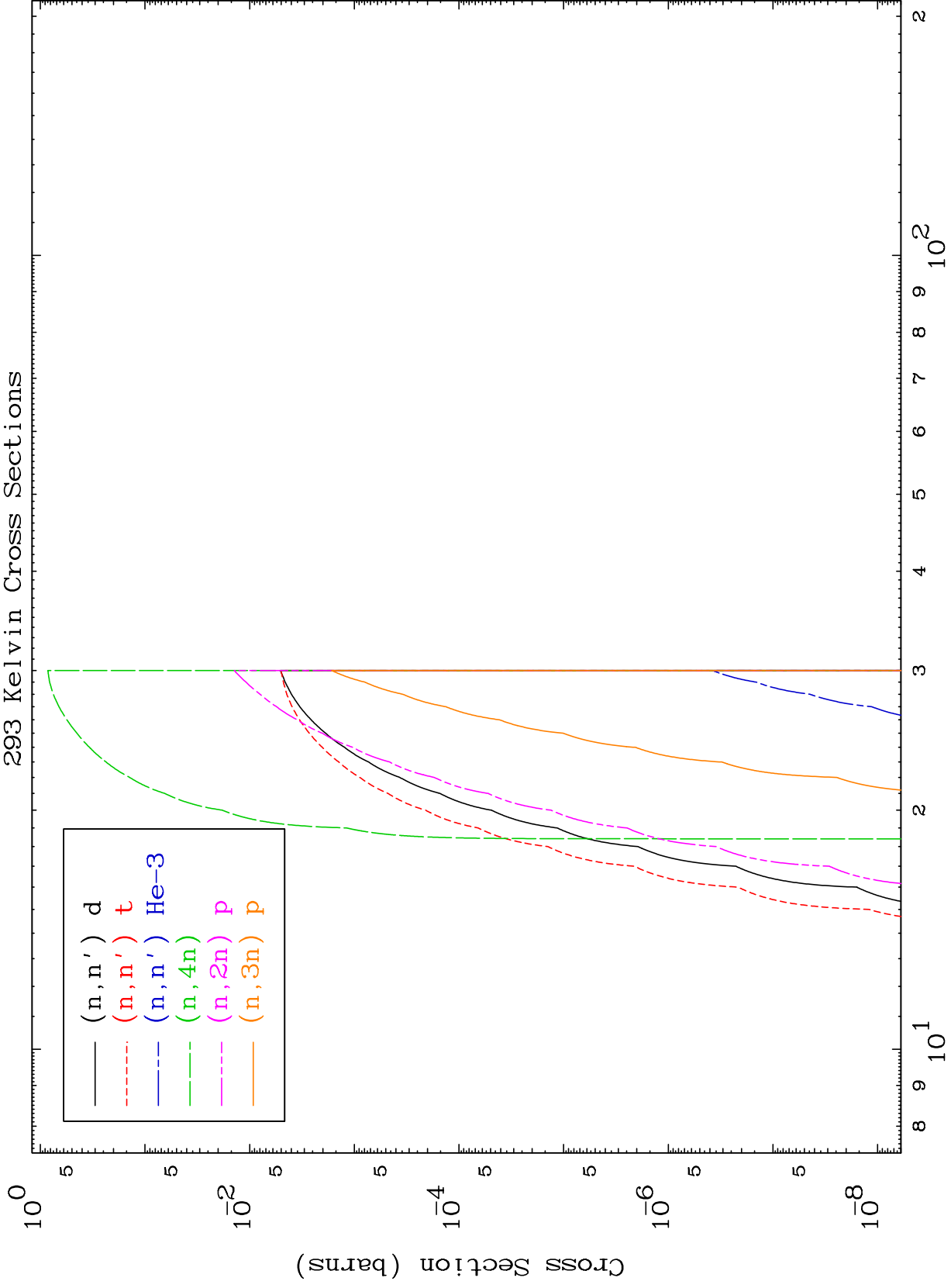
97-Bk-249



MAT 9752

Neutron Production
293 Kelvin Cross Sections

97-Bk-249



Incident Energy (MeV)

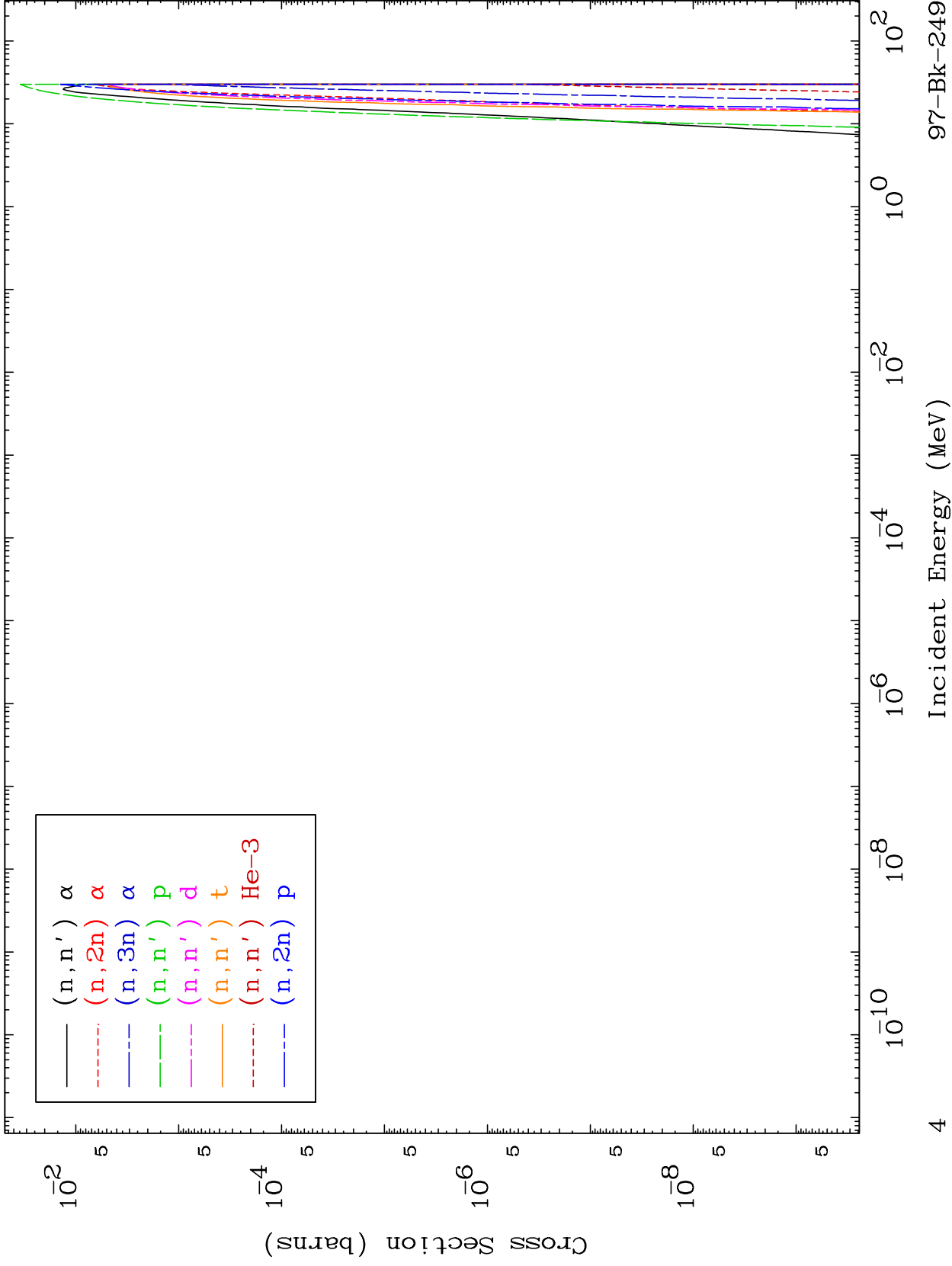
97-Bk-249

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MAT 9752

Charged Particle
293 Kelvin Cross Sections

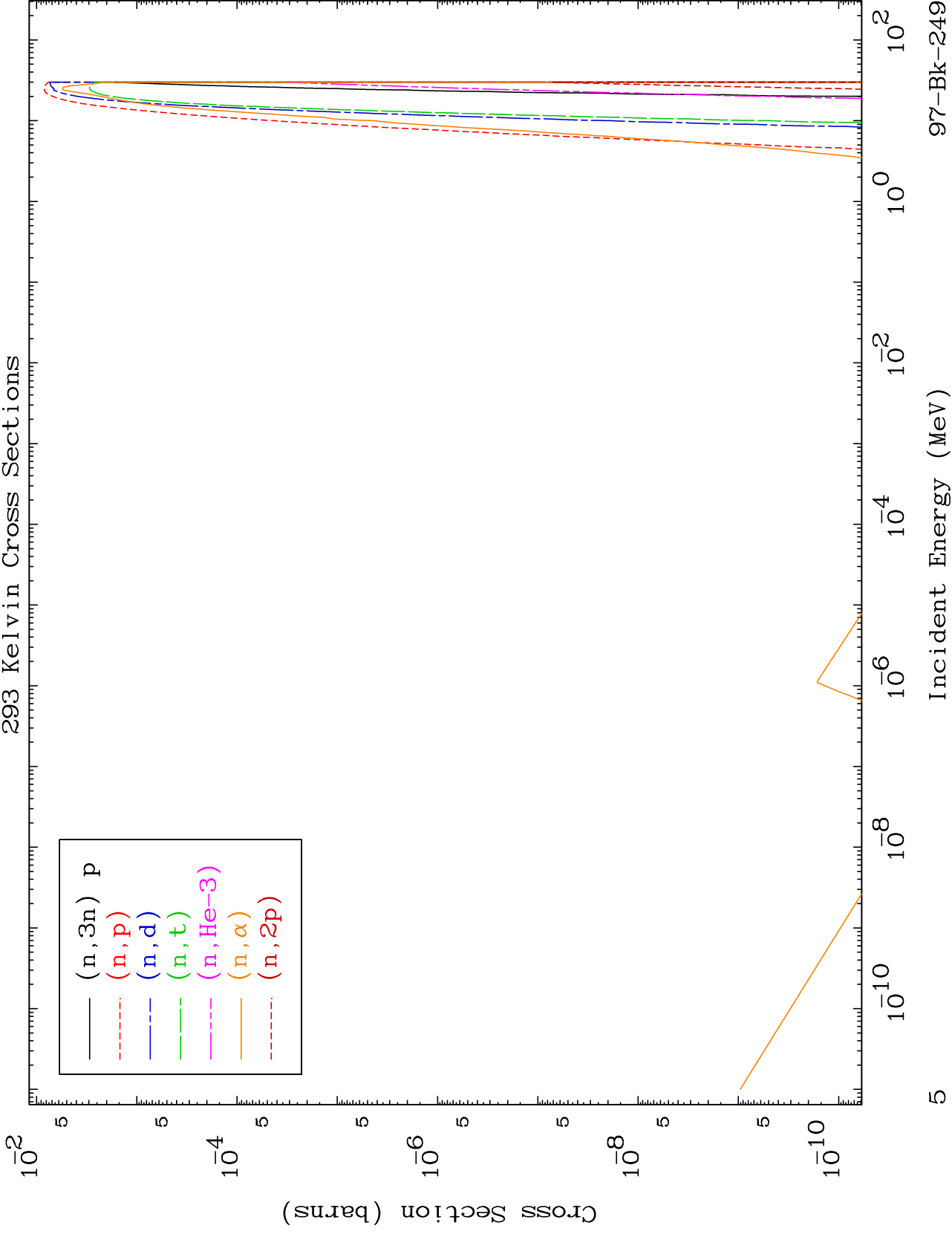
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MAT 9752

Charged Particle
293 Kelvin Cross Sections

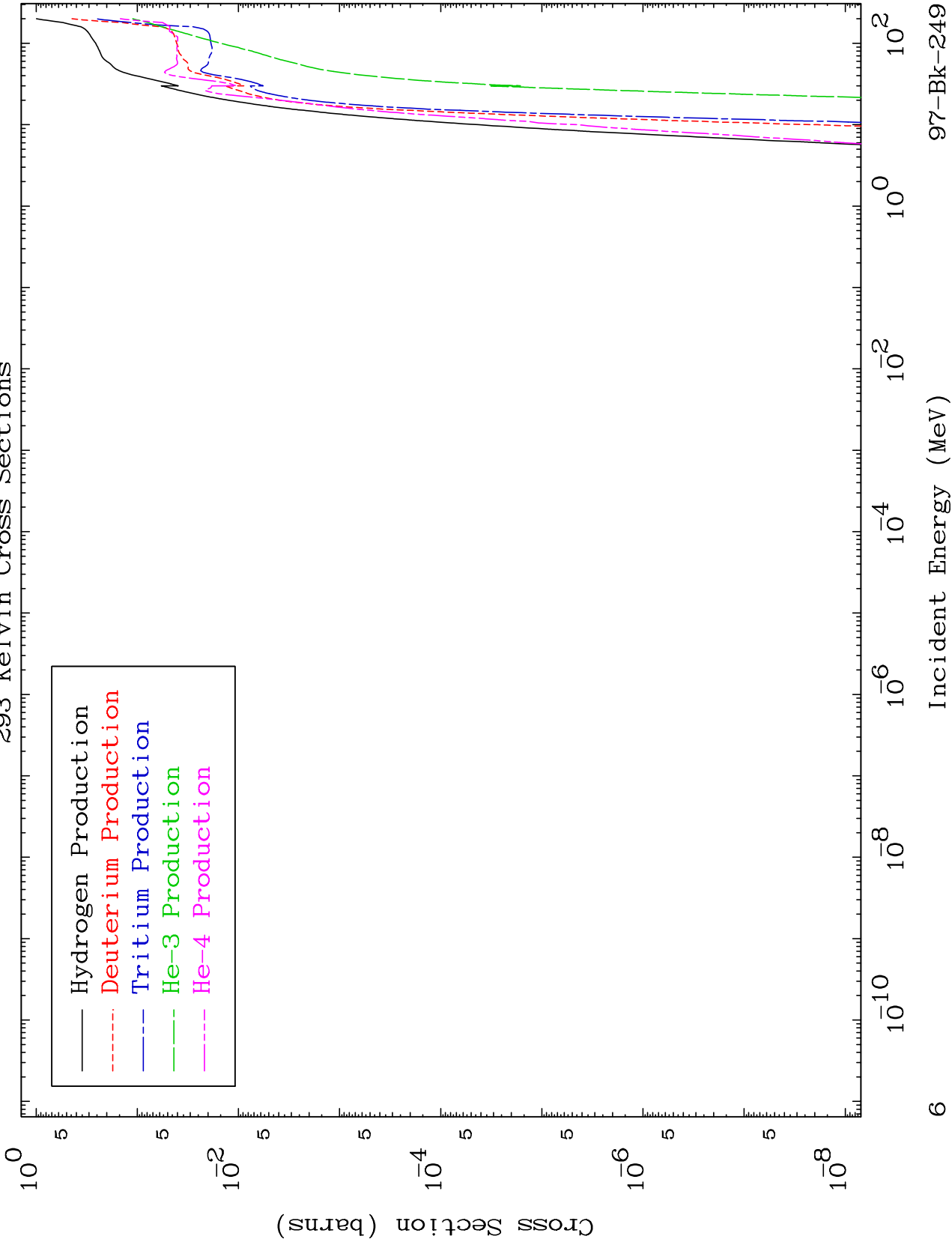
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MAT 9752

Particle Production
293 Kelvin Cross Sections

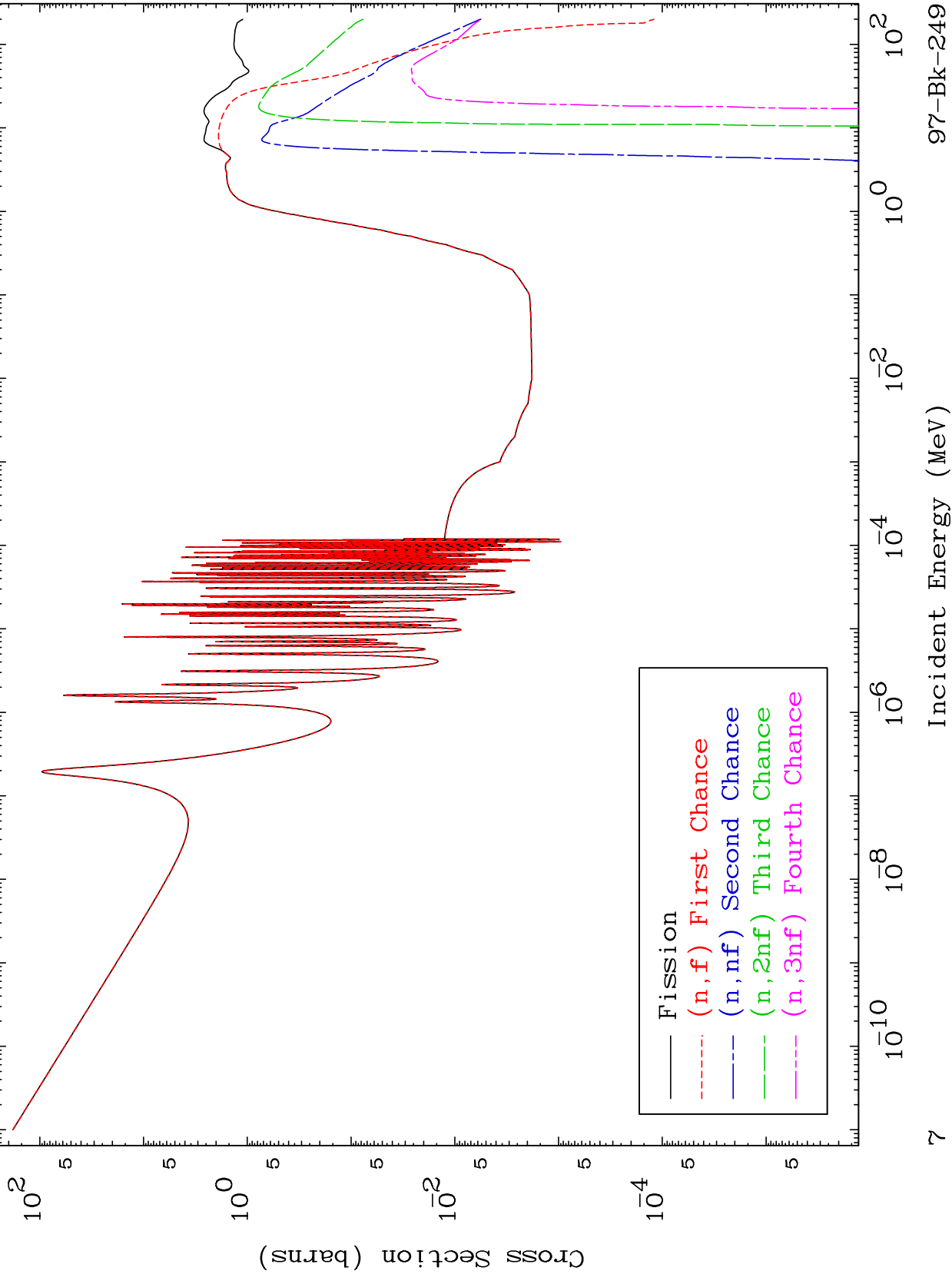
97-Bk-249



MAT 9752

Fission
293 Kelvin Cross Sections

97-Bk-249

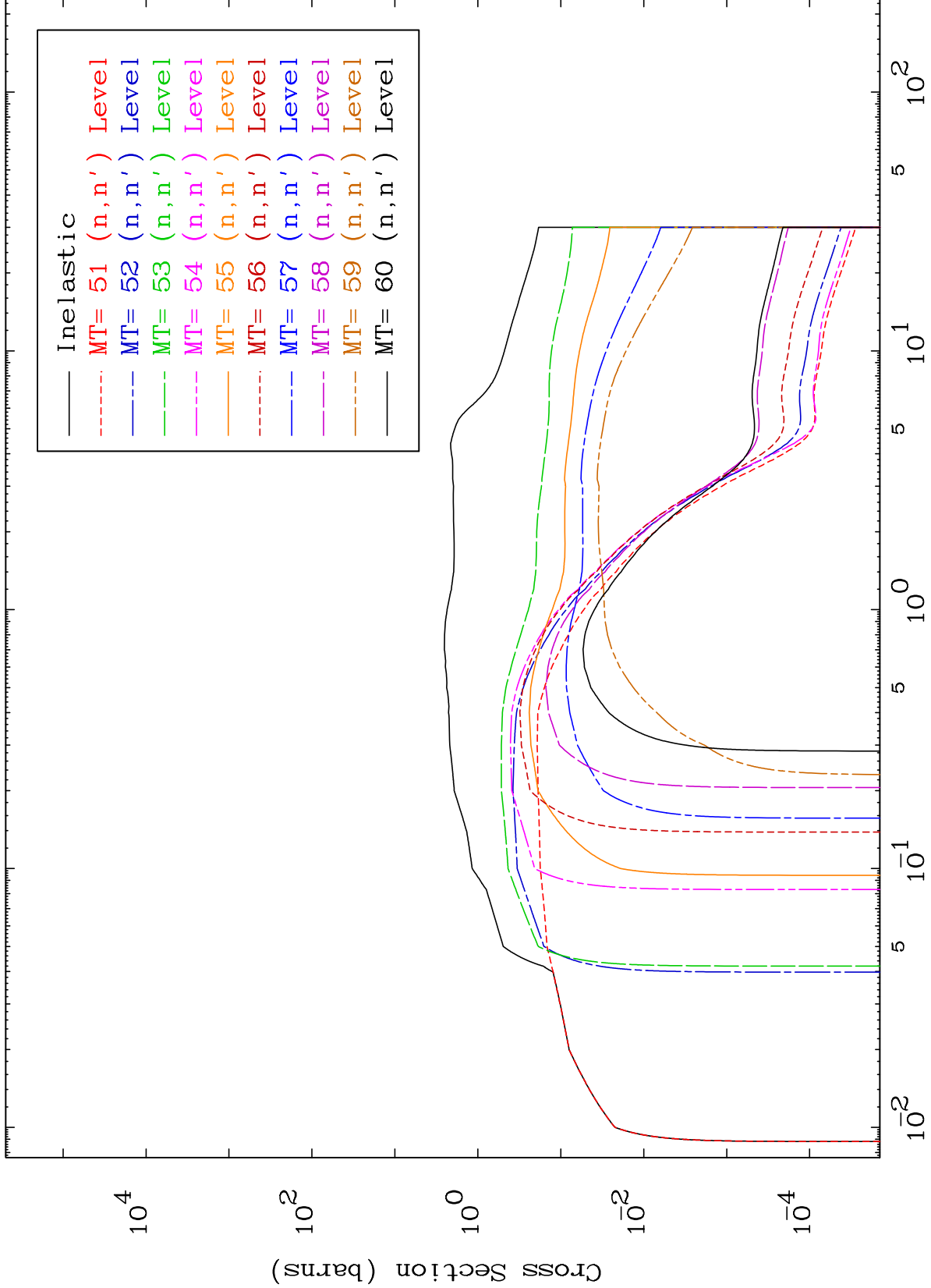


MAT 9752

(n,n') Level

97-Bk-249

293 Kelvin Cross Sections



Incident Energy (MeV)

97-Bk-249

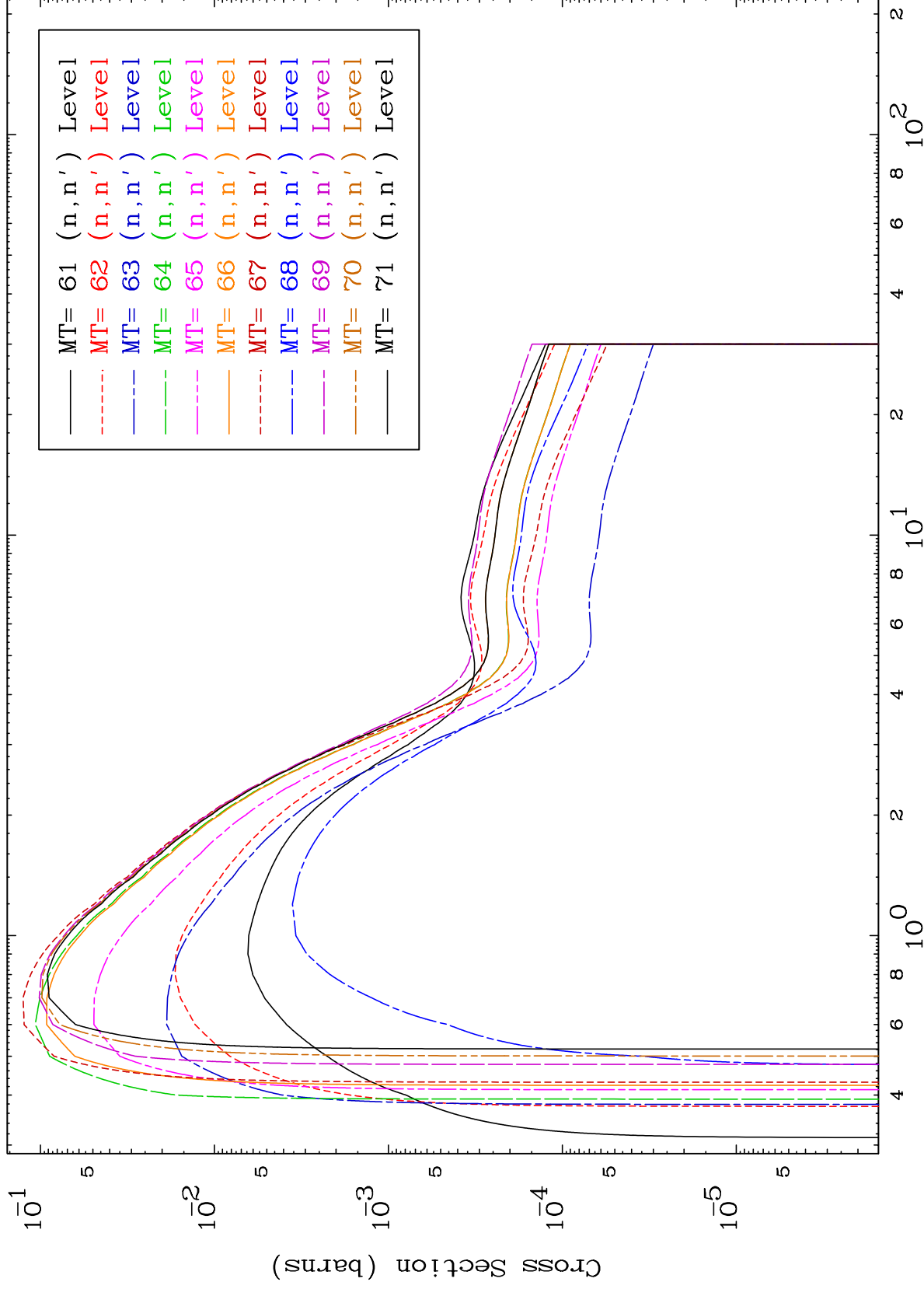
8

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(n,n') Level

293 Kelvin Cross Sections

97-Bk-249



9

Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,n') Level

97-Bk-249

293 Kelvin Cross Sections



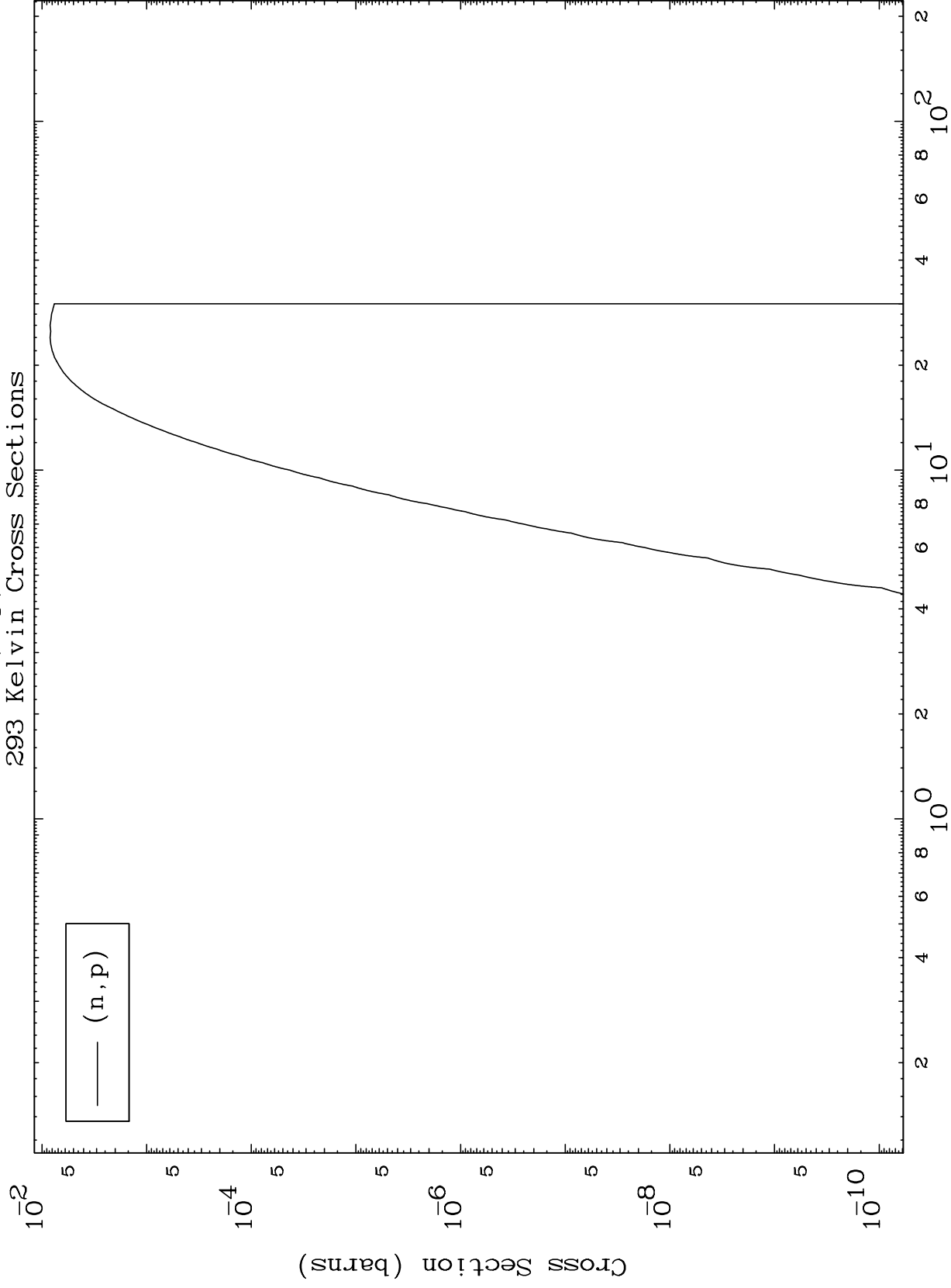
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,p) Levels
293 Kelvin Cross Sections

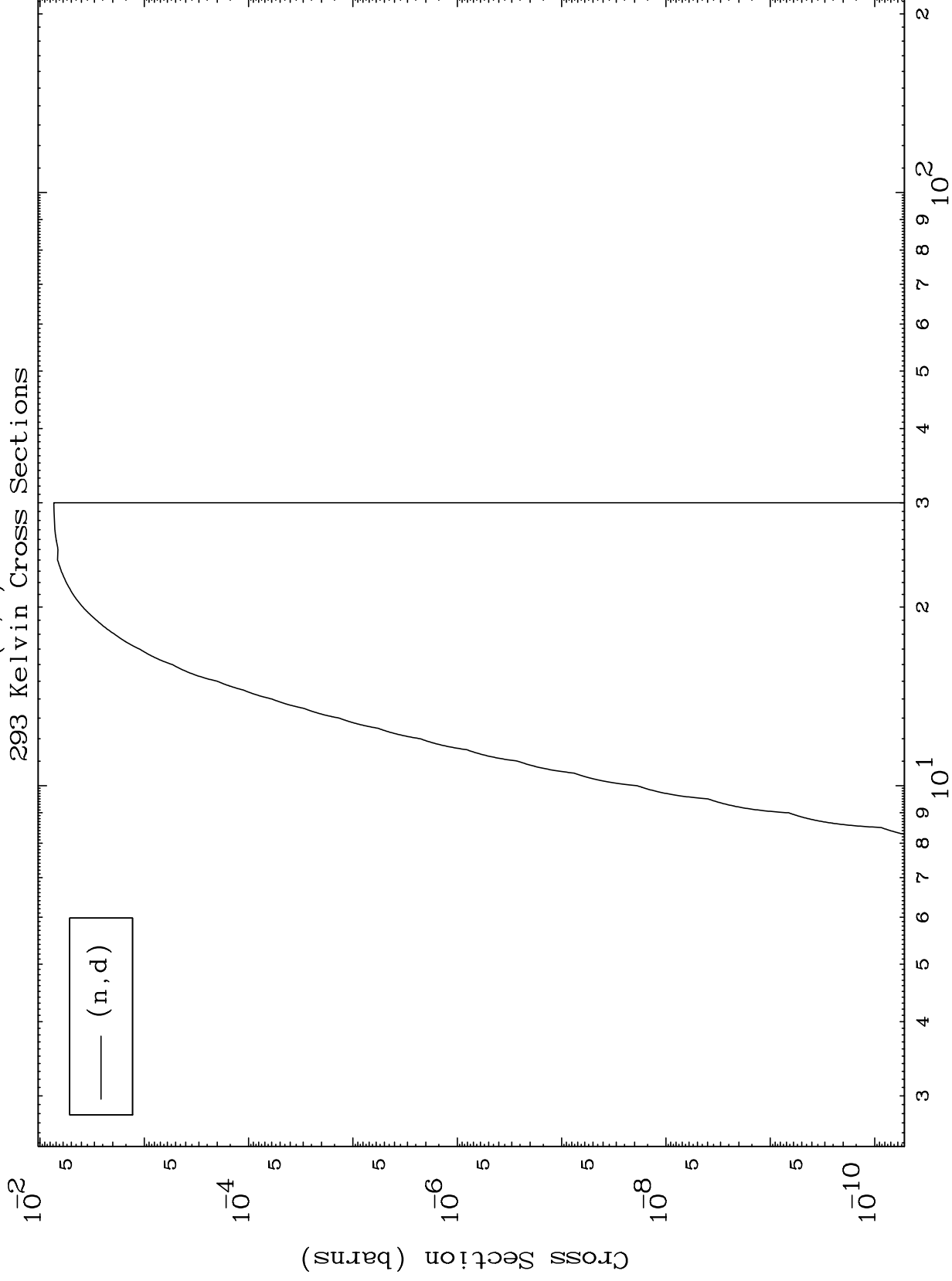
97-Bk-249



MAT 9752

(n,d) Levels
293 Kelvin Cross Sections

97-Bk-249



12

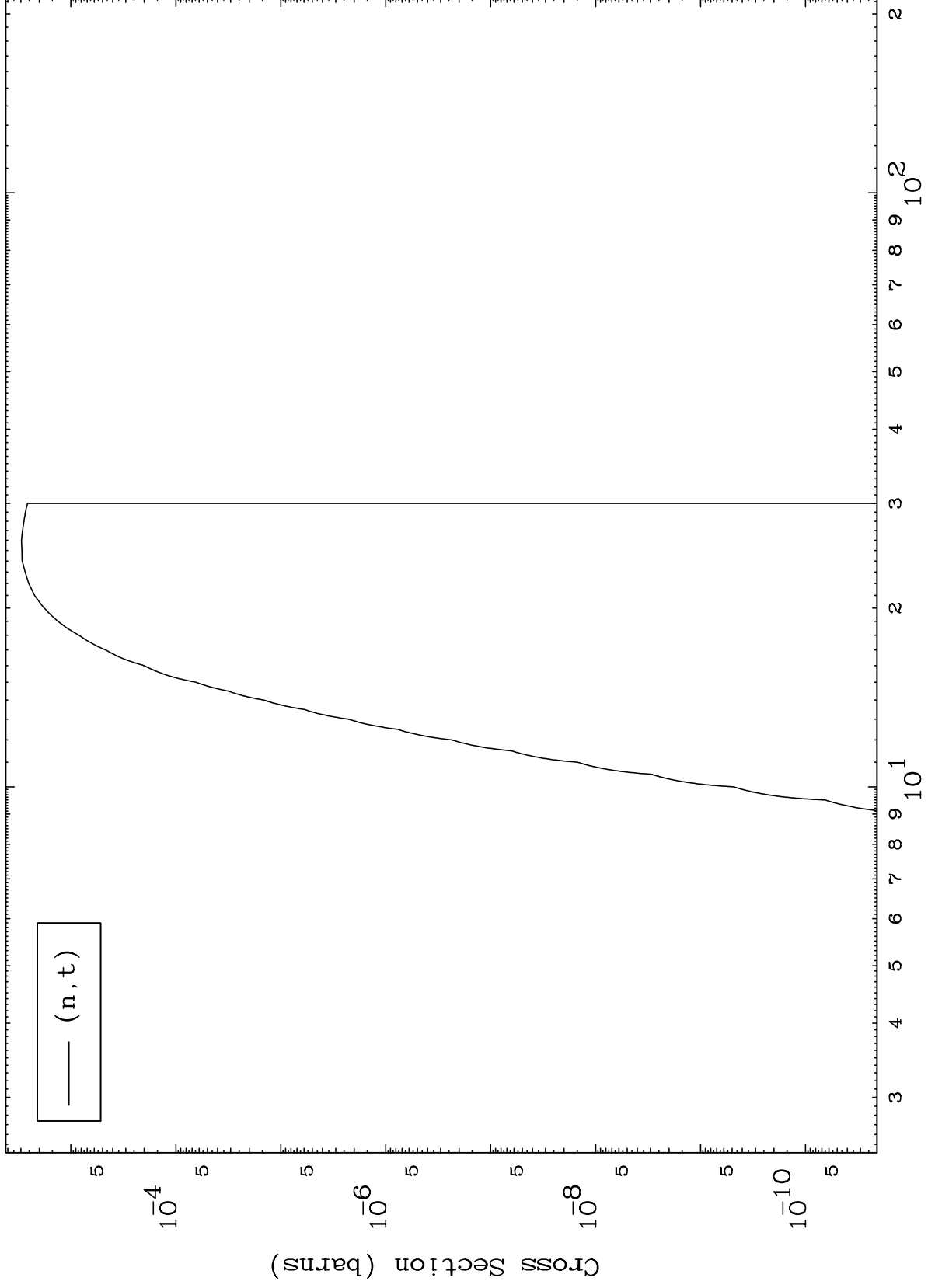
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-249



13

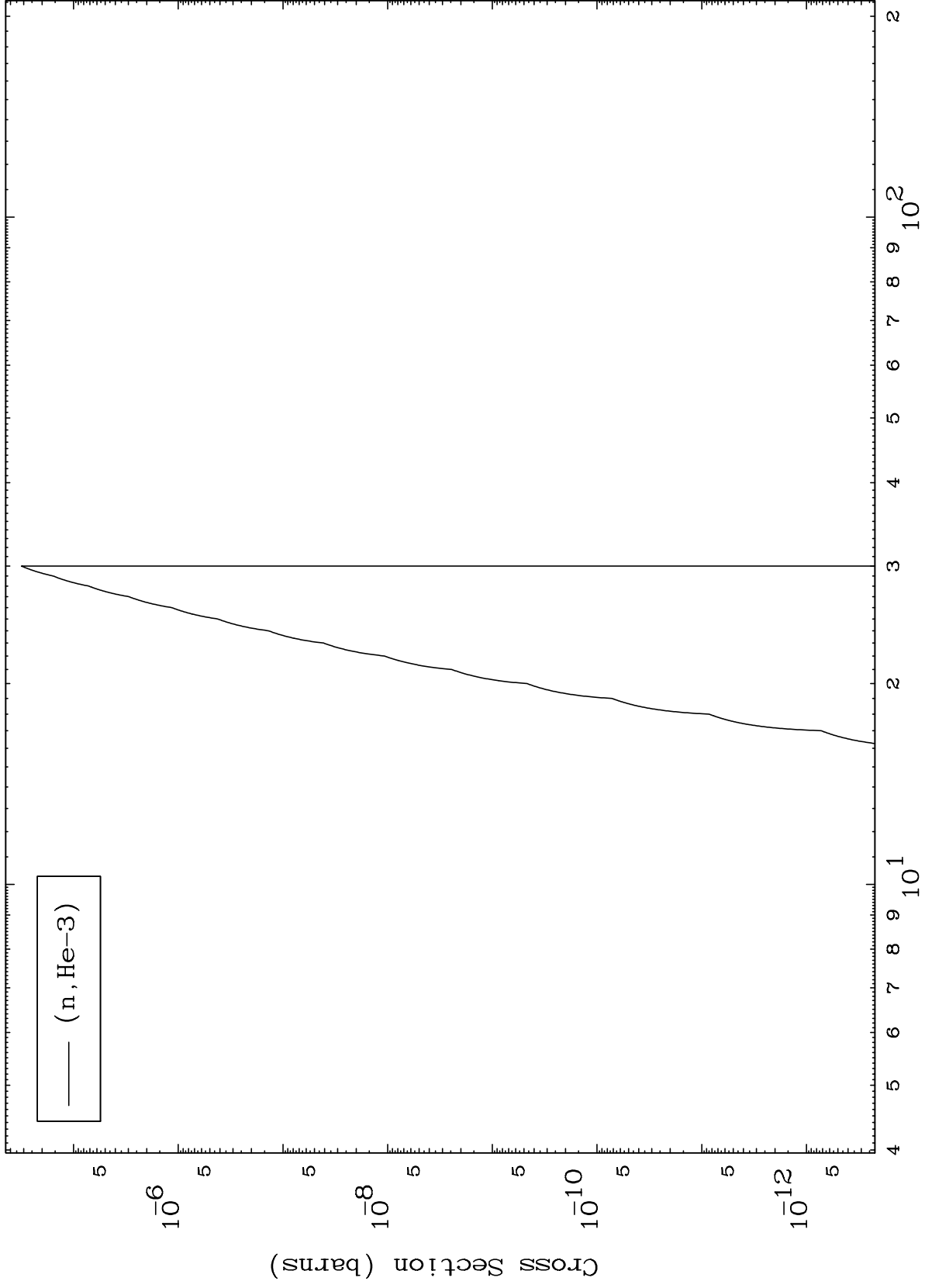
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-249



14

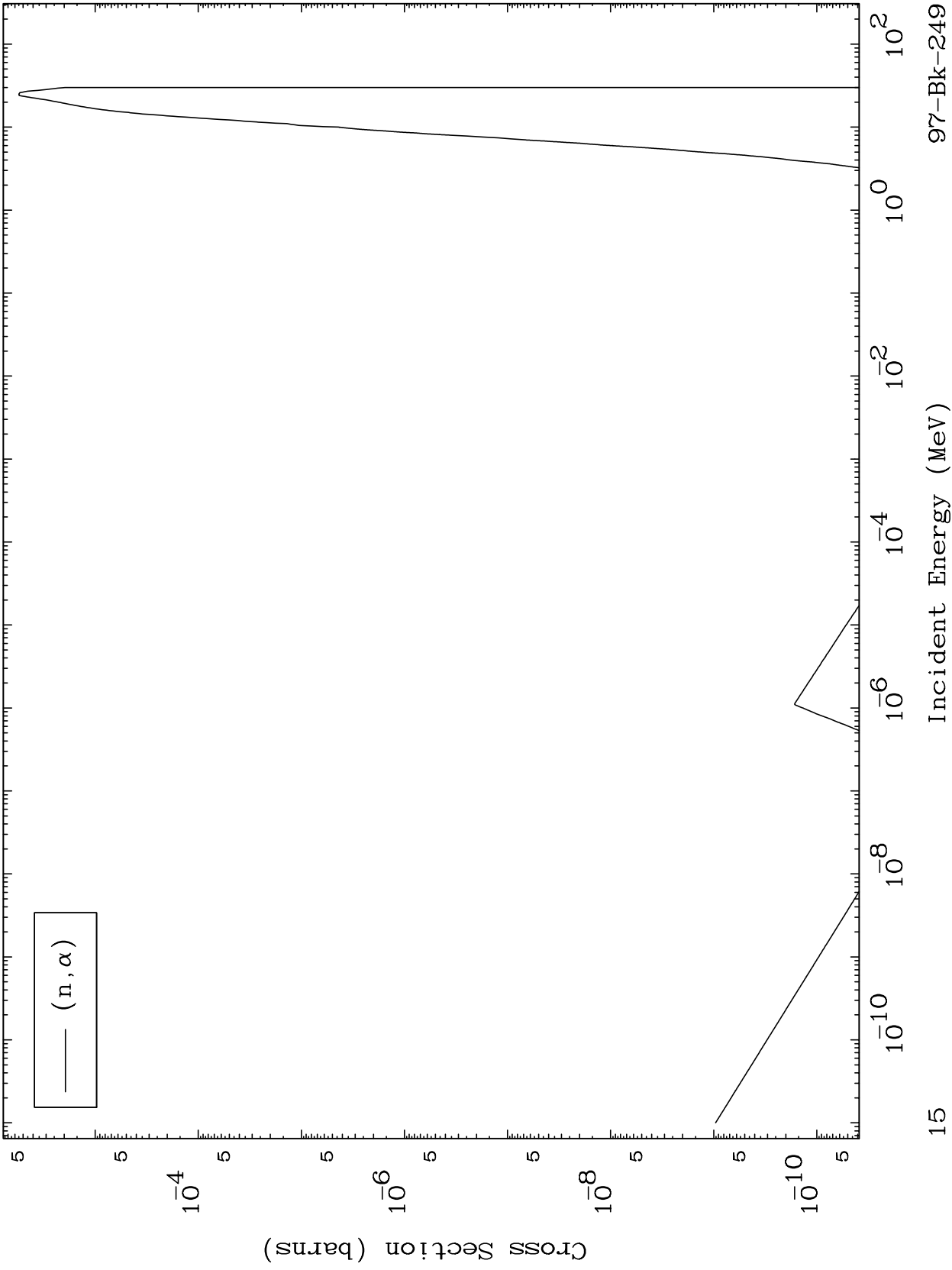
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n, α) Levels
293 Kelvin Cross Sections

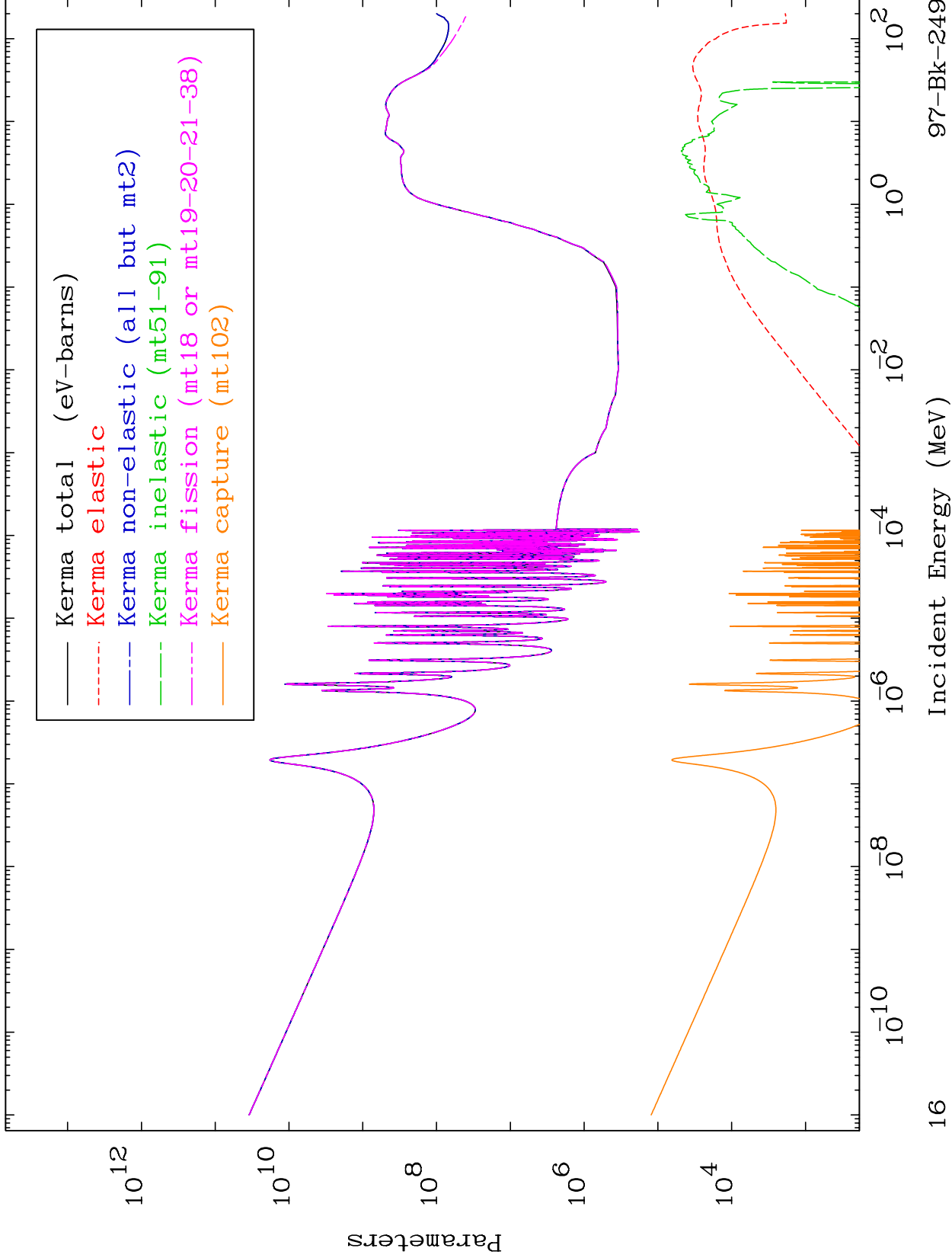
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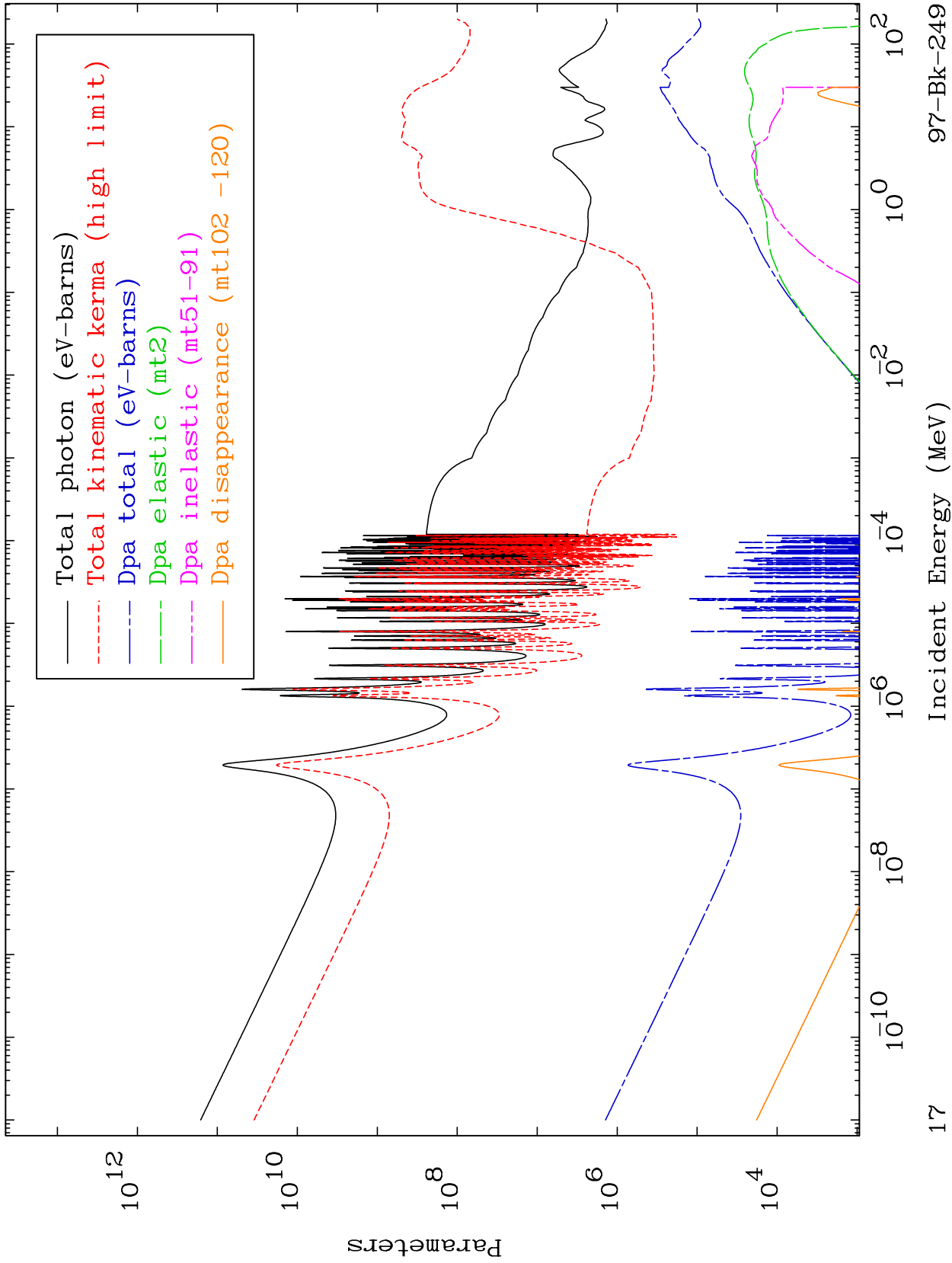


15

Incident Energy (MeV)

97-Bk-249

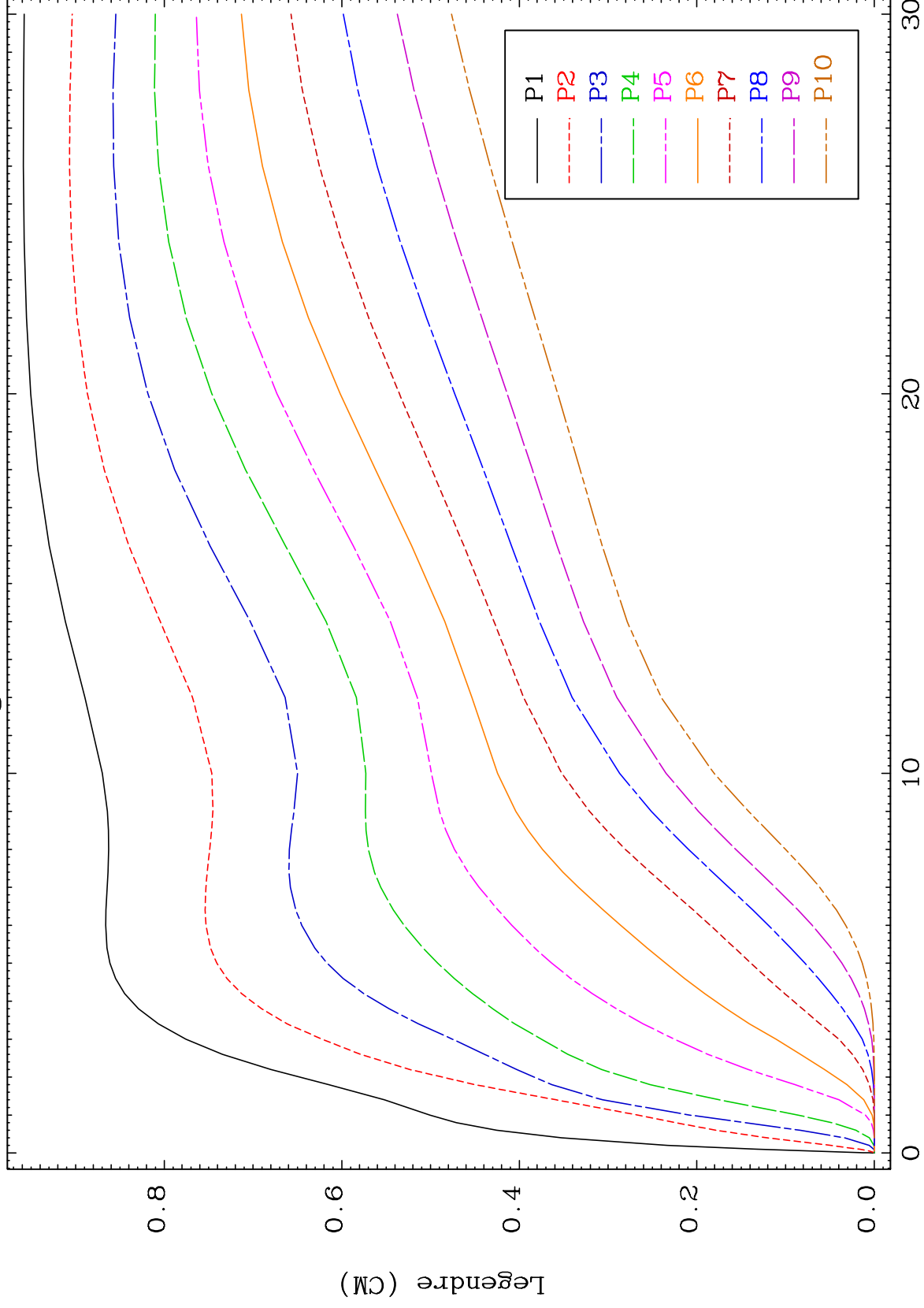




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Elastic Legendre Coefficients

97-Bk-249



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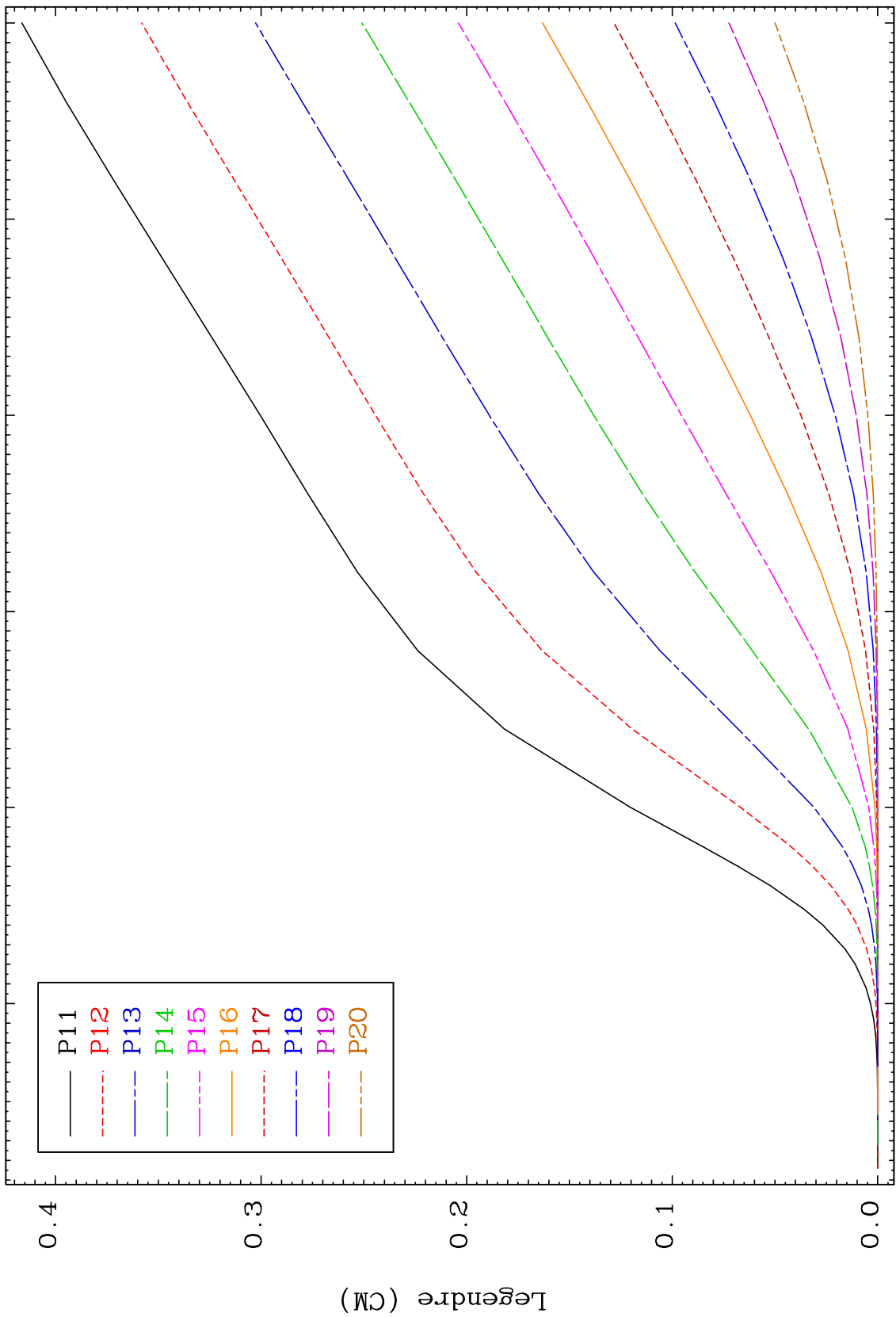
Incident Energy (MeV)

97-Bk-249

MAT 9752

Elastic Legendre Coefficients

97-Bk-249



19

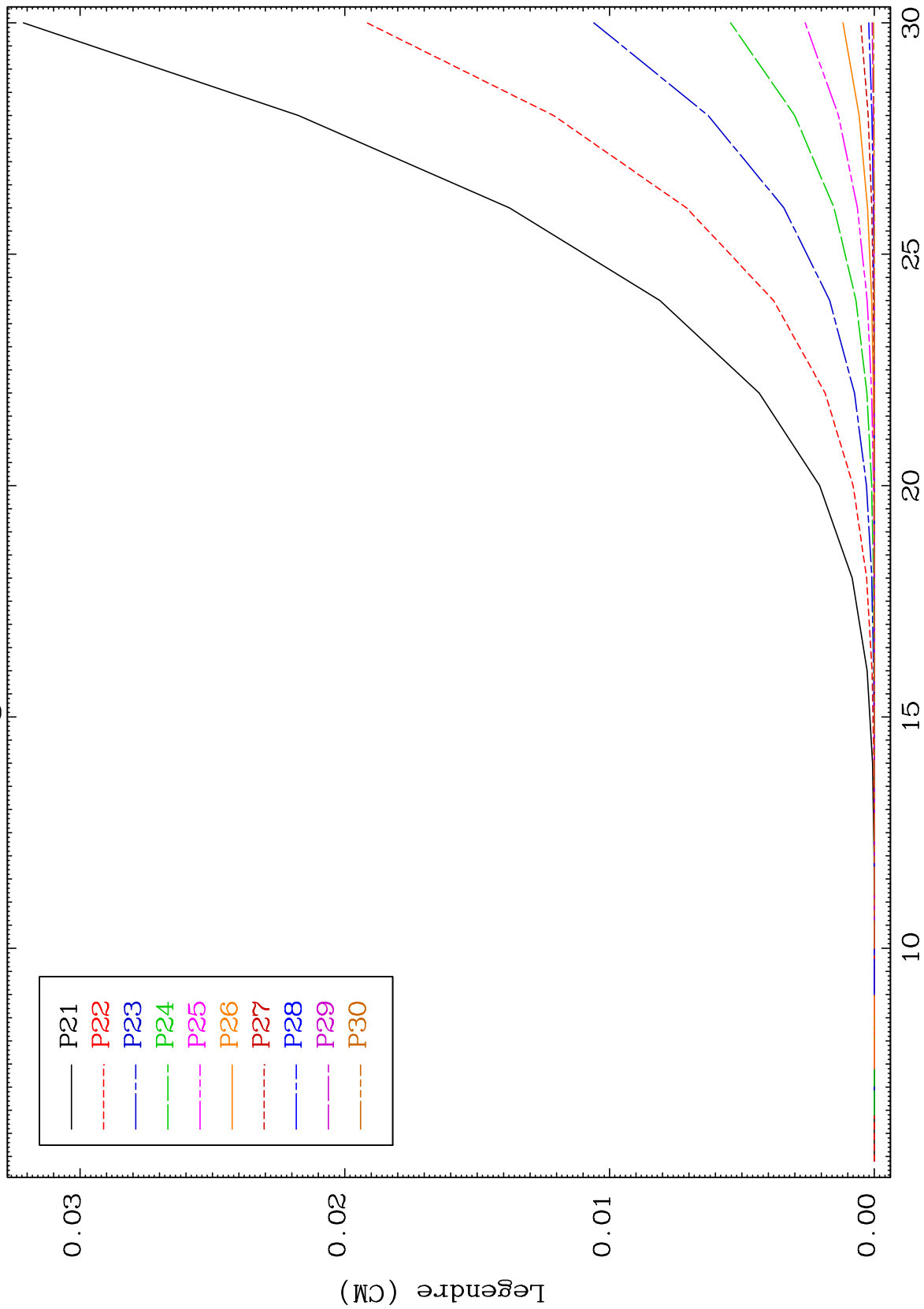
Incident Energy (MeV)

97-Bk-249

MAT 9752

Elastic
Legendre Coefficients

97-Bk-249



20

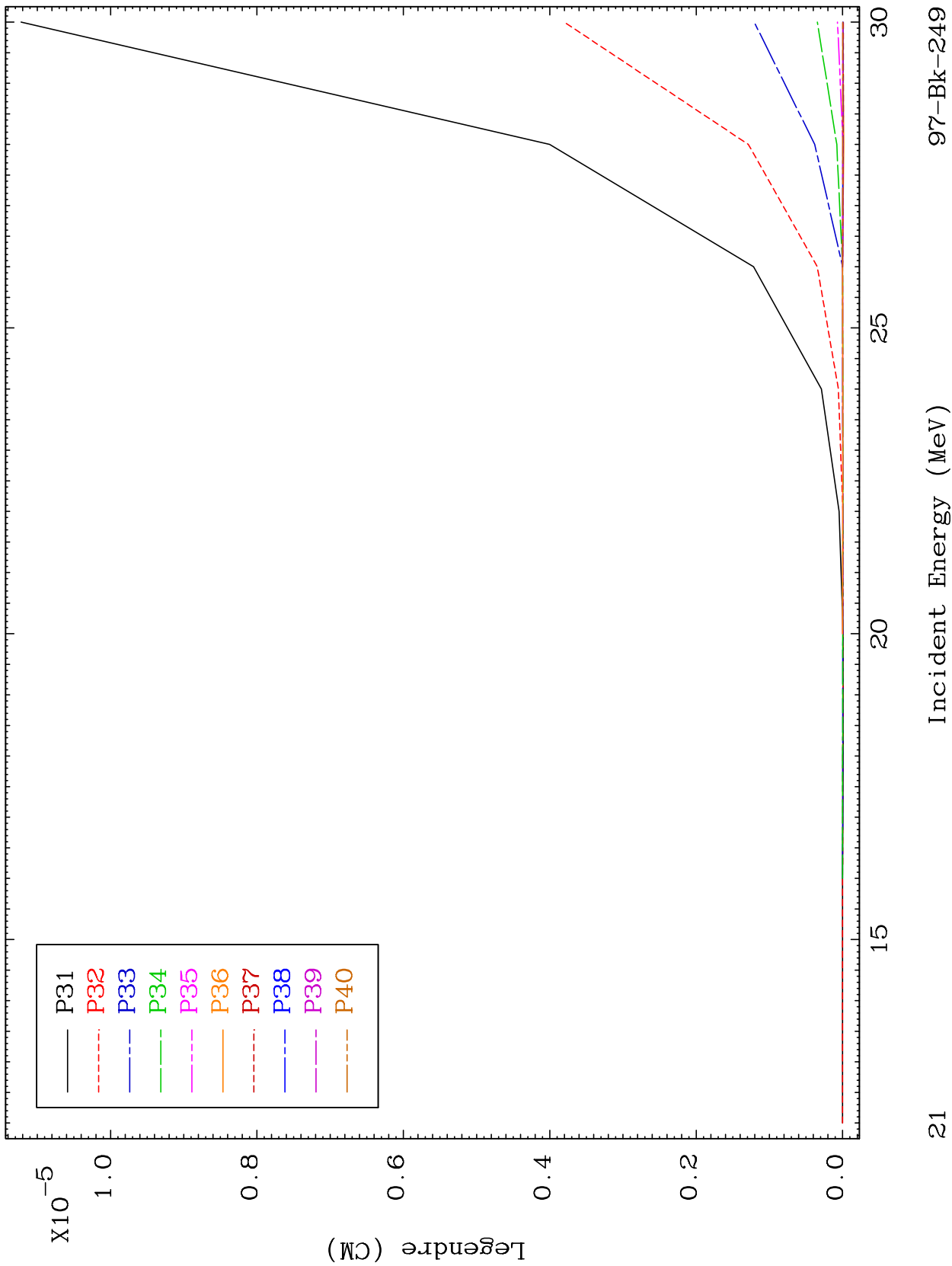
Incident Energy (MeV)

97-Bk-249

MAT 9752

Elastic Legendre Coefficients

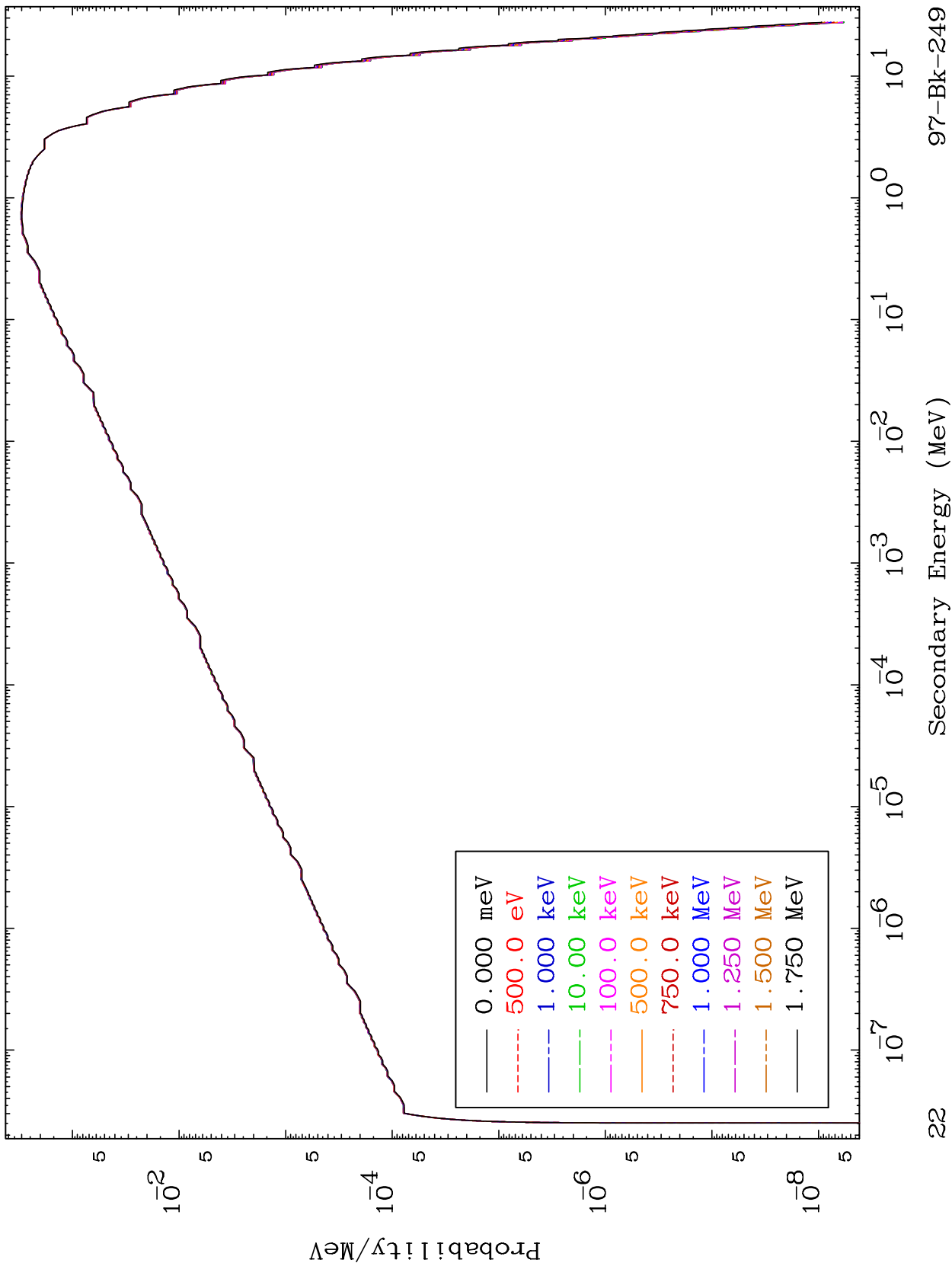
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97-Bk-249

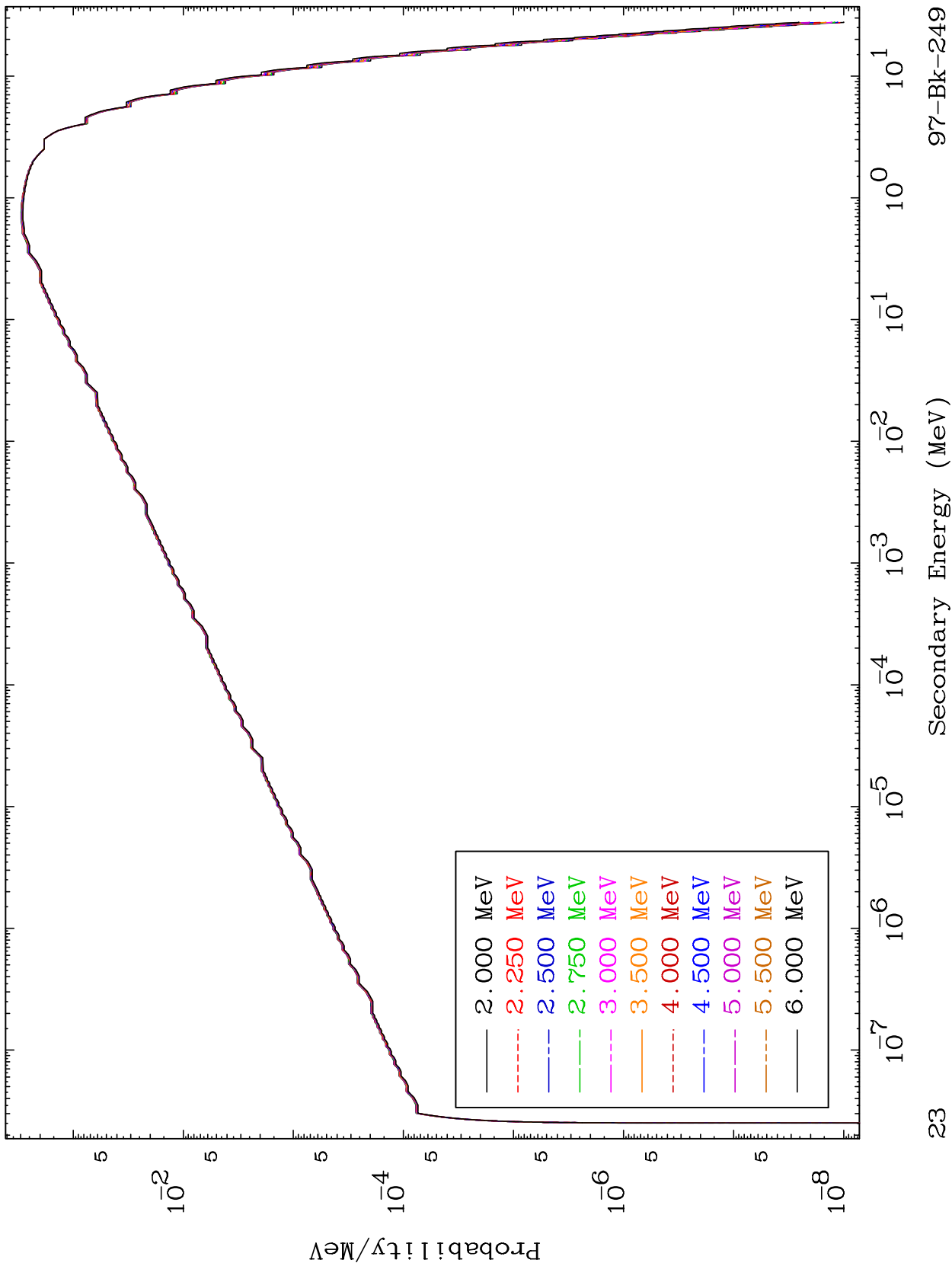
Fission Energy Distribution



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Fission Energy Distribution

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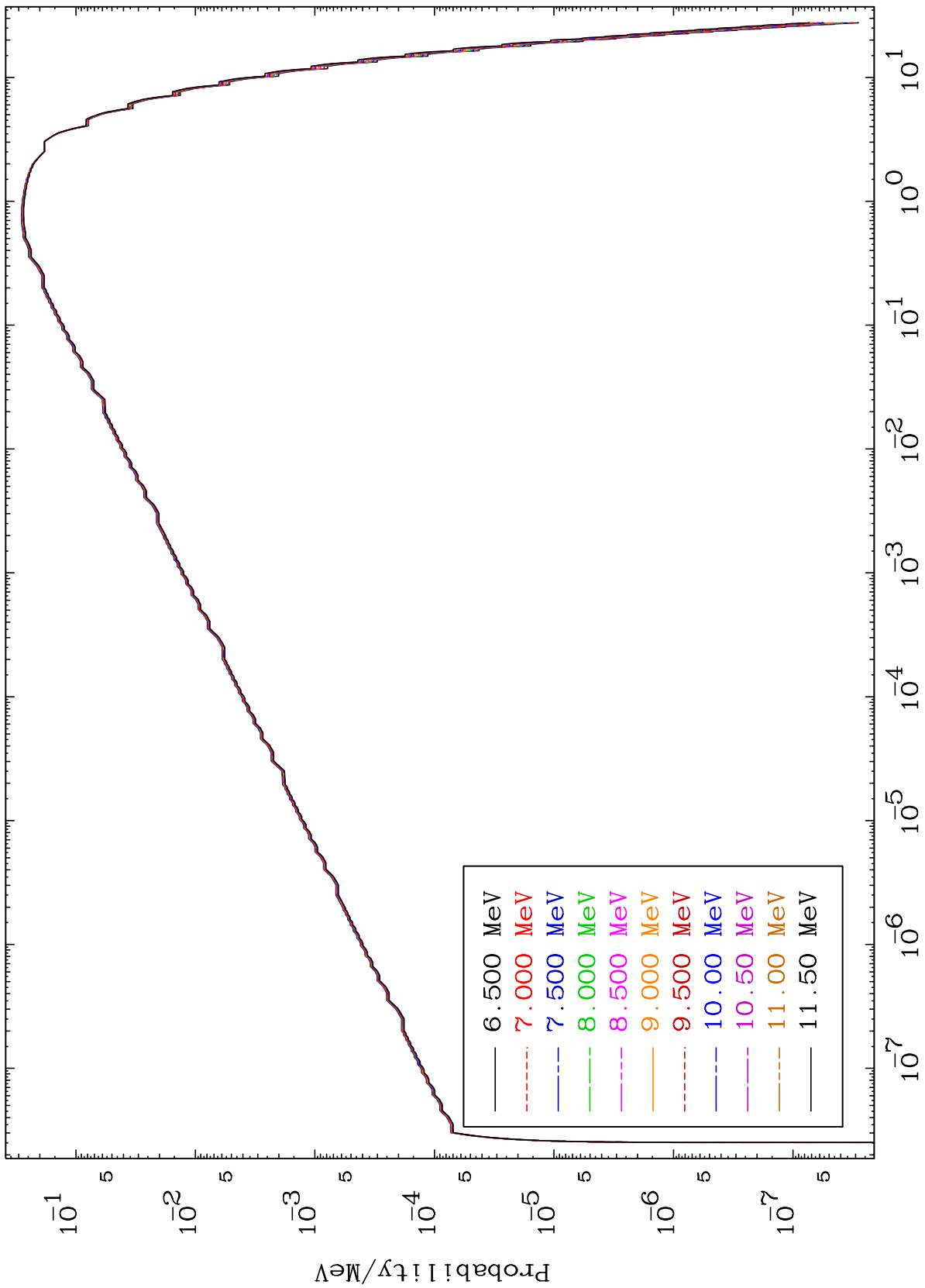


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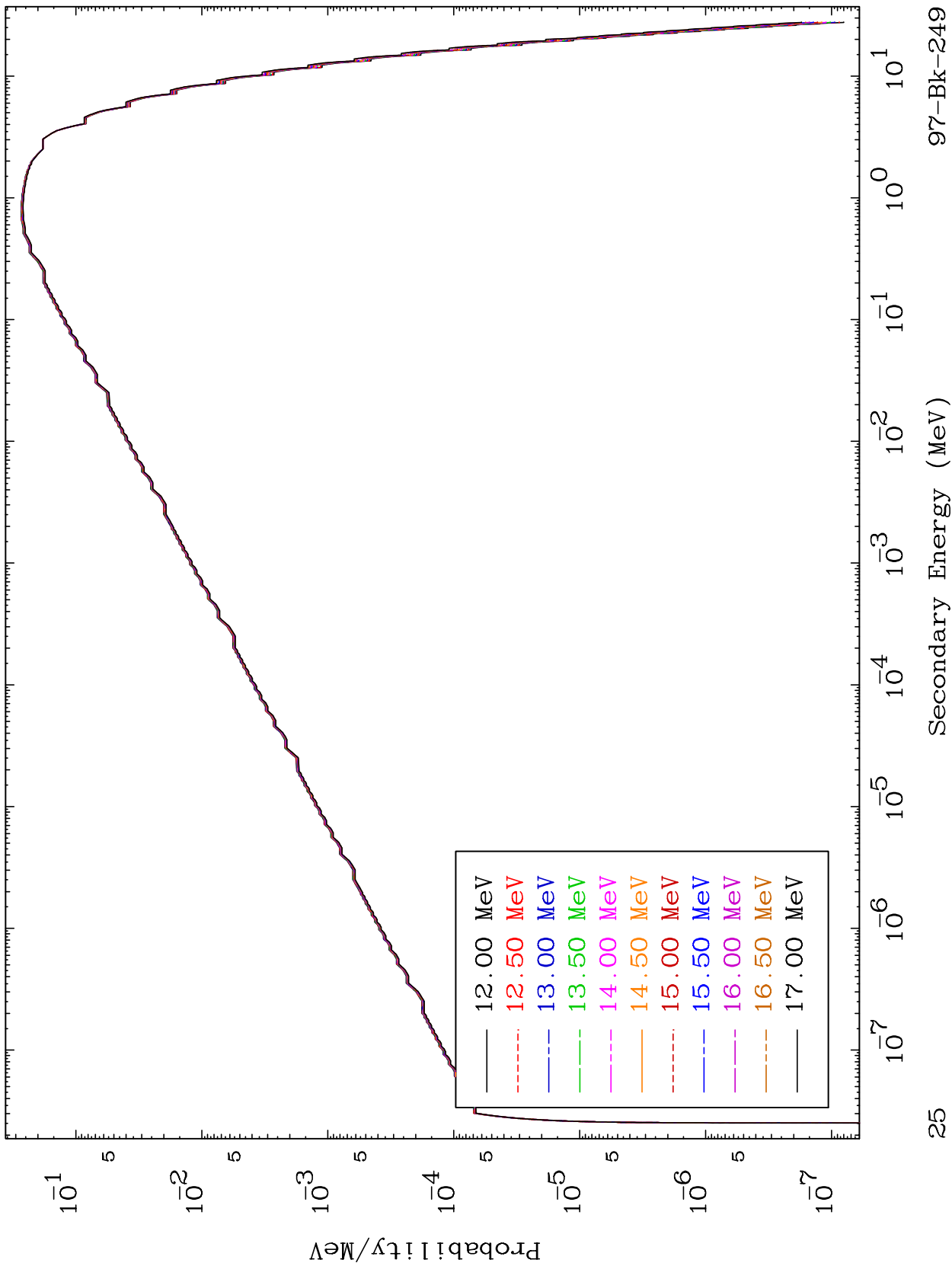
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Fission Energy Distribution

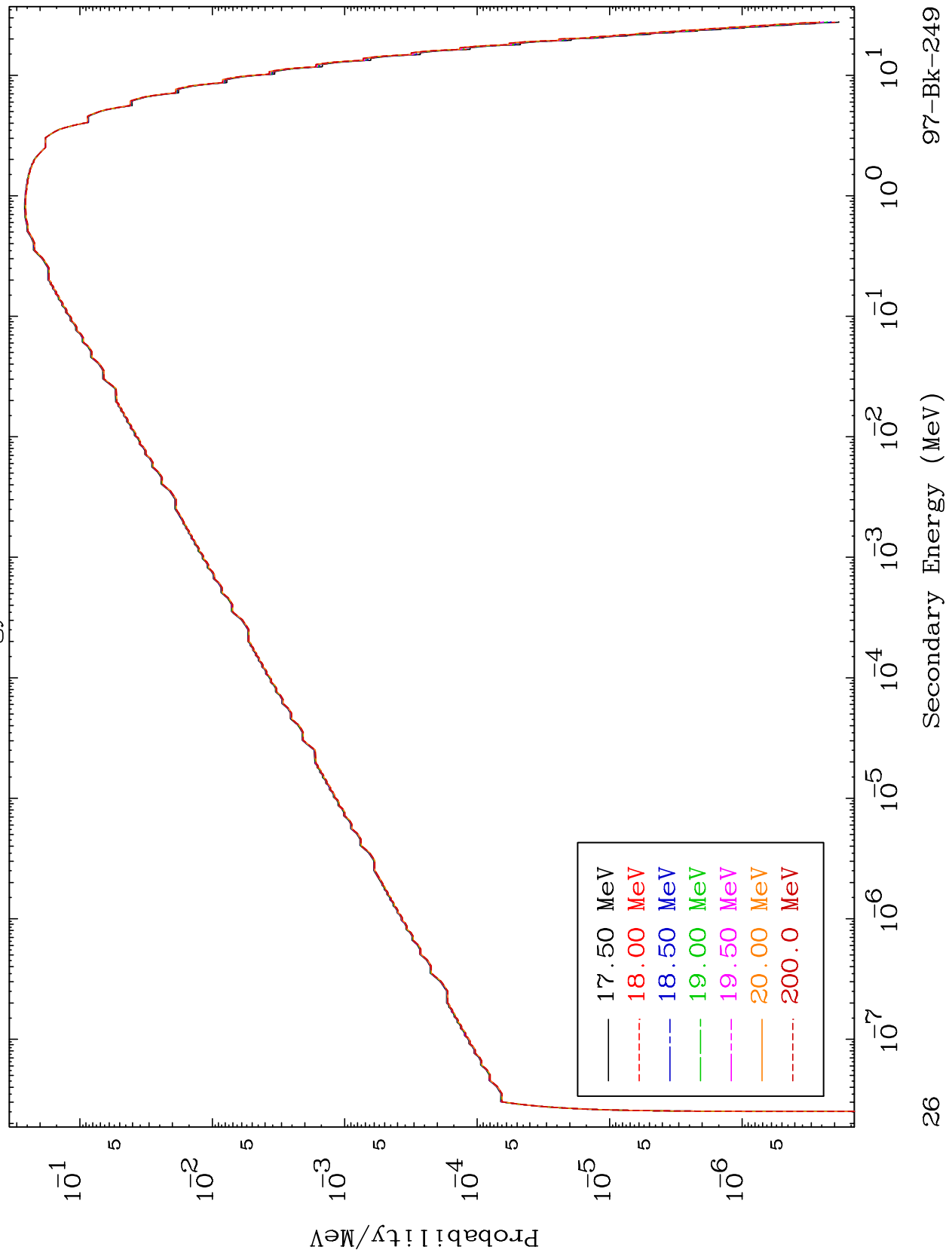
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Fission Energy Distribution



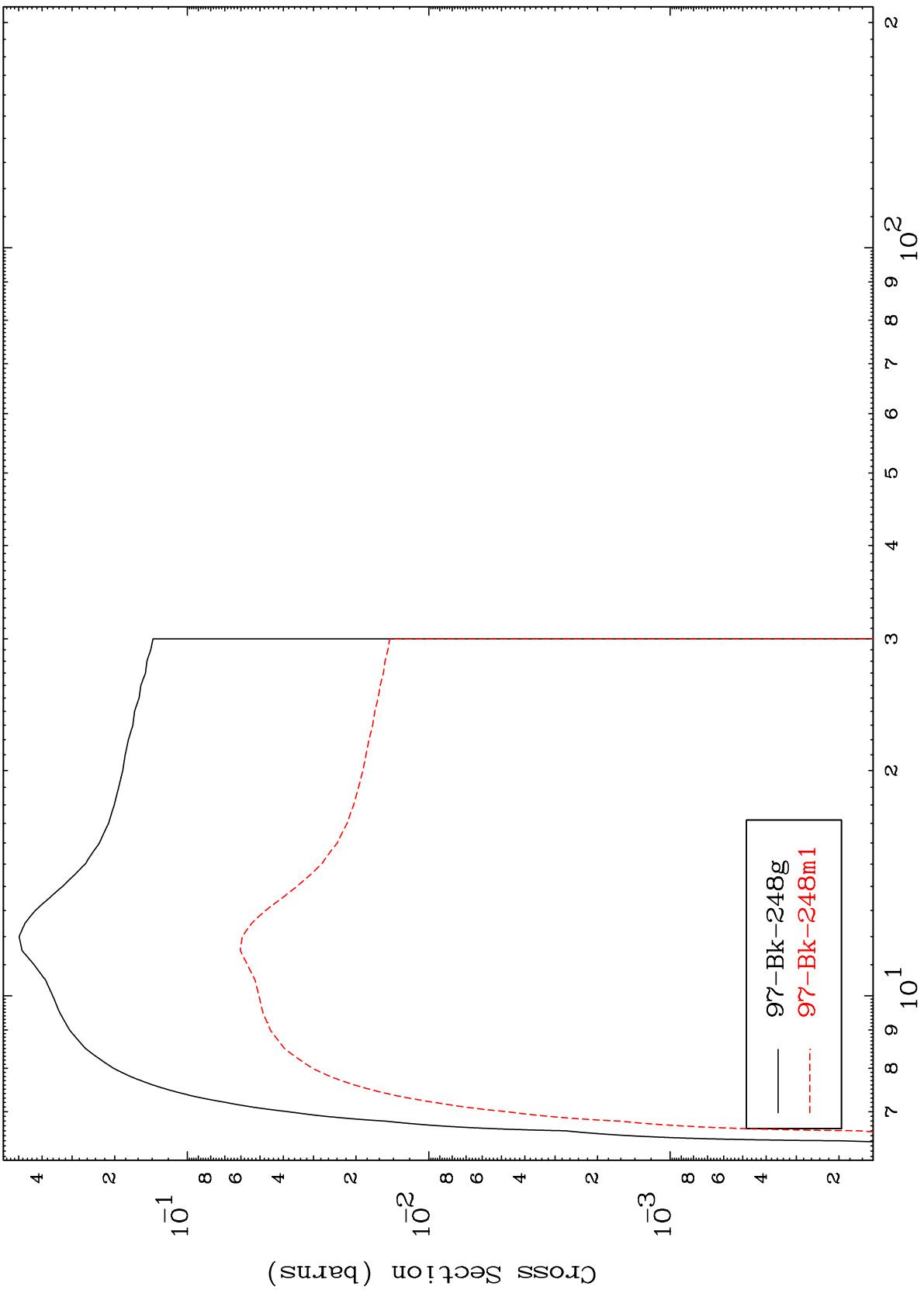
Fission
Energy Distribution



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Radionuclide Production Cross Section (n,2n)



27

Incident Energy (MeV)

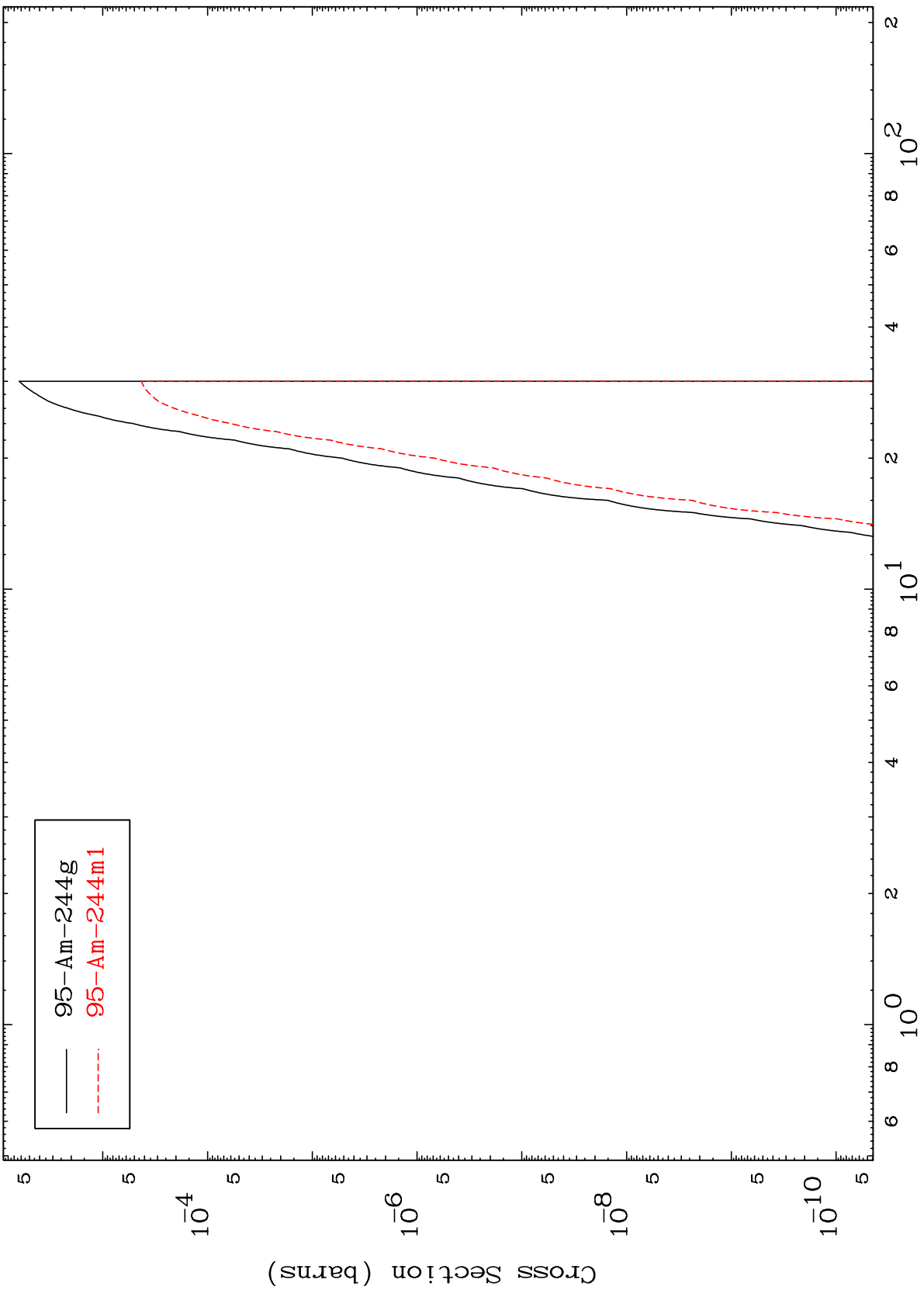
97-Bk-249

MAT 9752

(n,2n) α

97-Bk-249

Radionuclide Production Cross Section



28

Incident Energy (MeV)

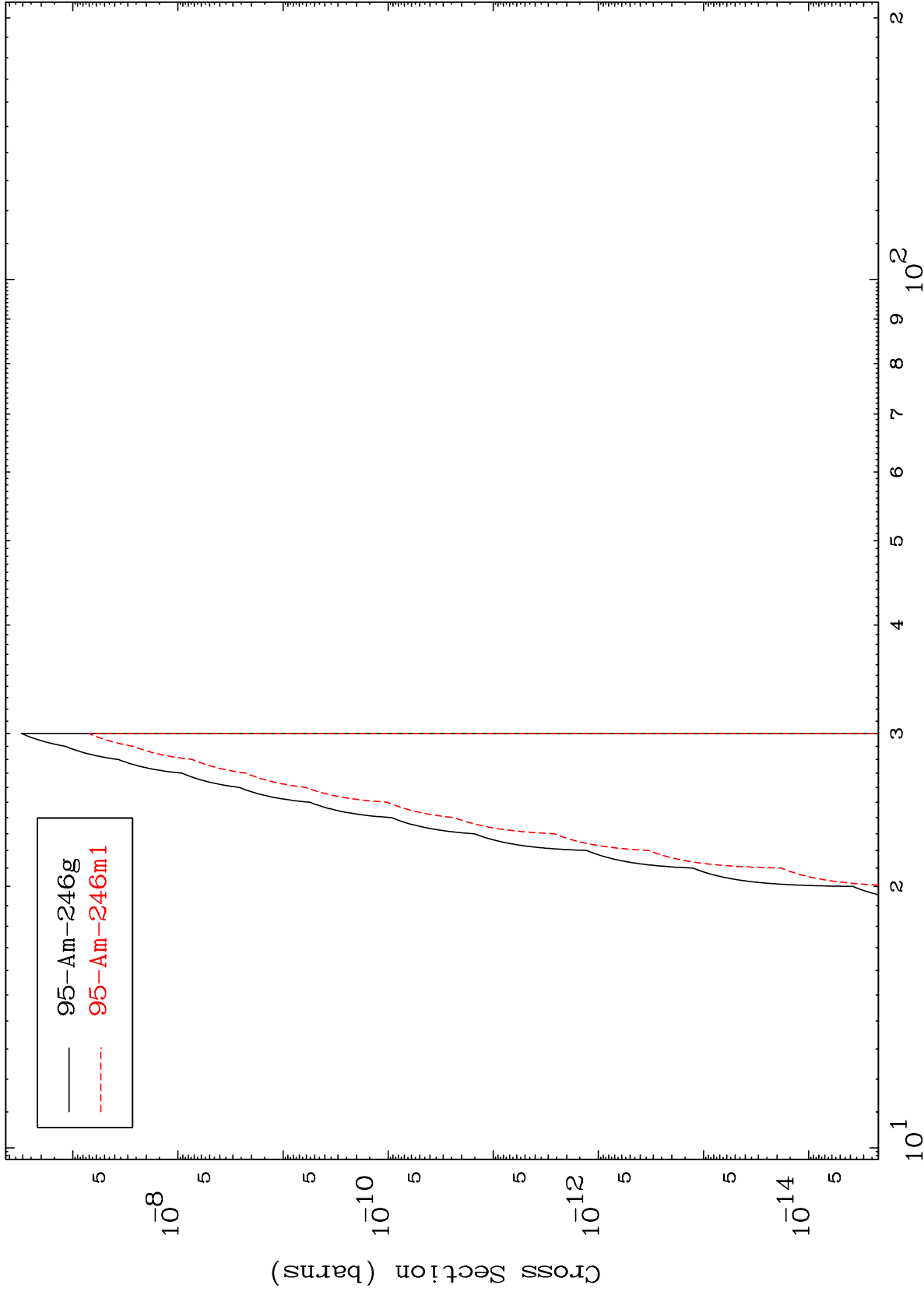
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(n,n') He-3

97-Bk-249

Radionuclide Production Cross Section



29

Incident Energy (MeV)

97-Bk-249

MAT 9752

(n, α)

97-Bk-249

Radionuclide Production Cross Section

