

Program EVALPLOT
(Version 2017-1)

by

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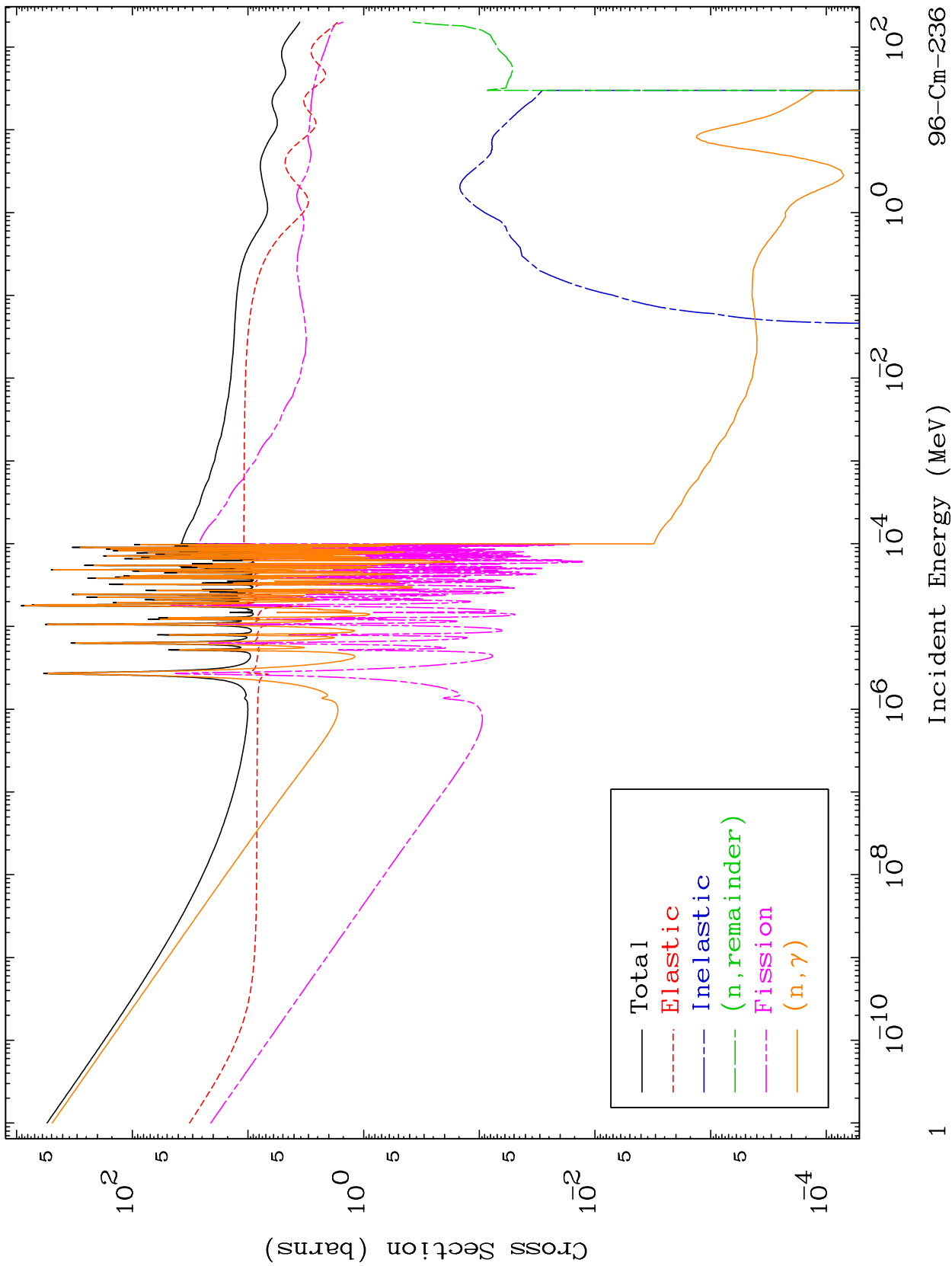
Press Mouse Button to Start

MAT 9613

Major

293 Kelvin Cross Sections

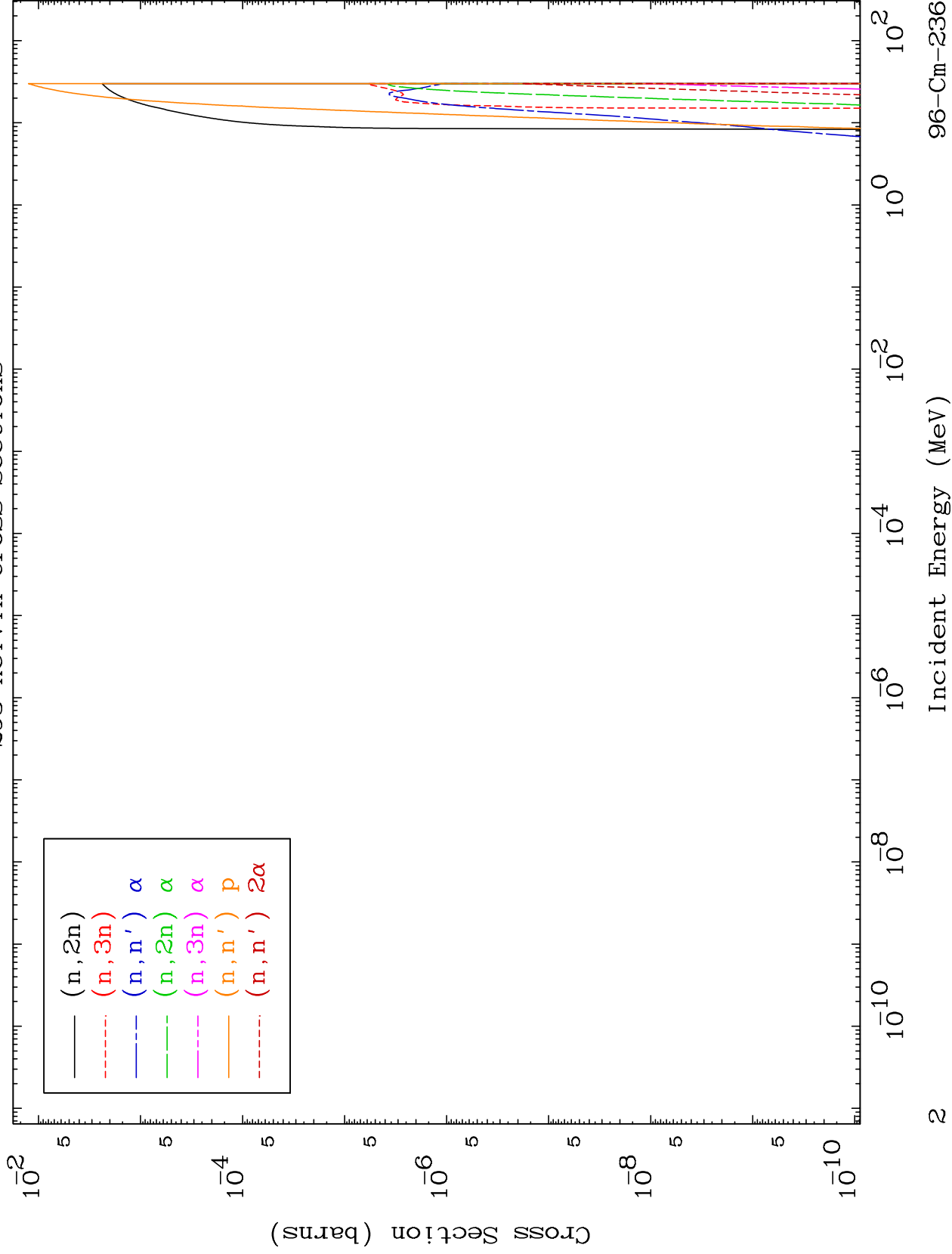
96-Cm-236

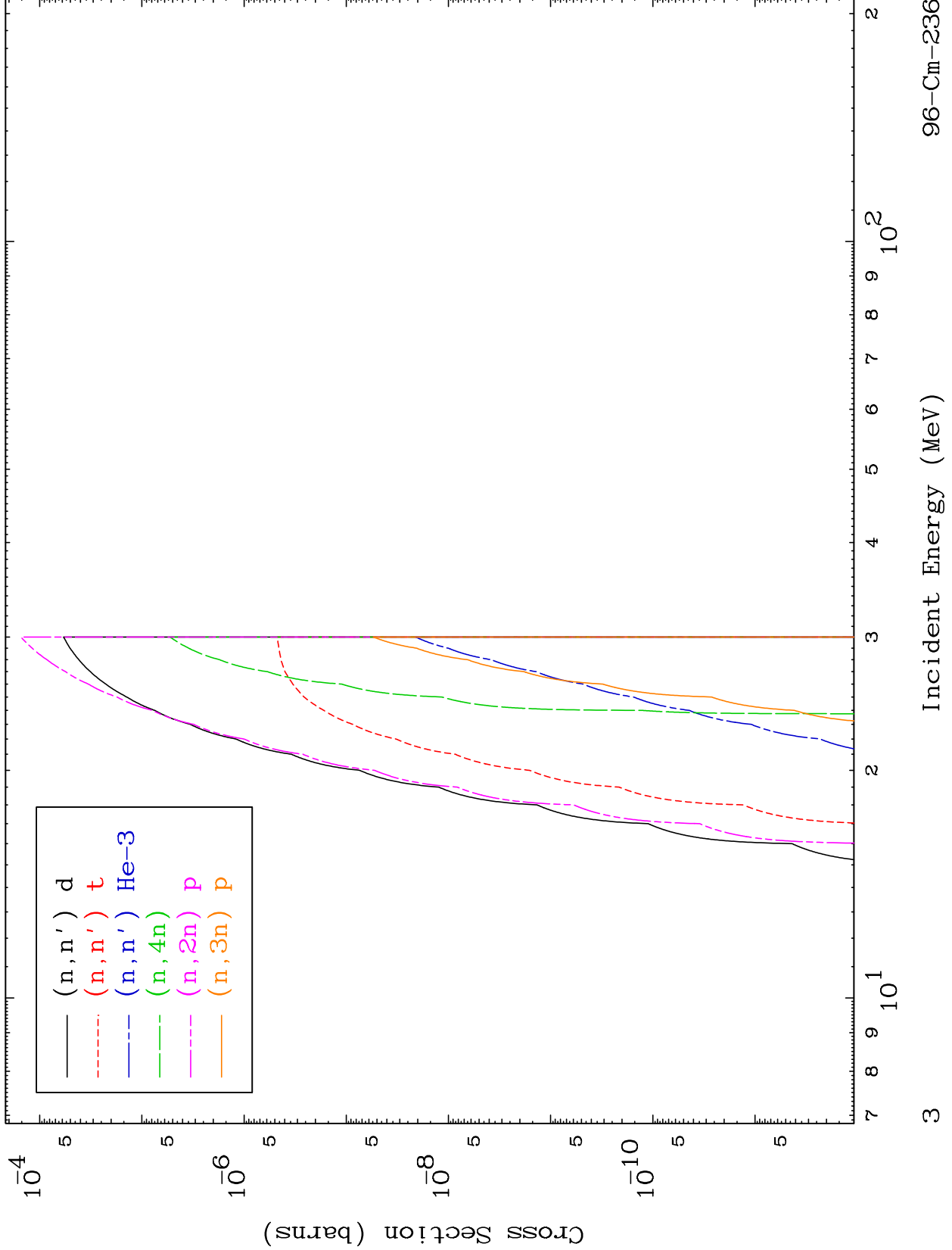


MAT 9613

Neutron Production
293 Kelvin Cross Sections

96-Cm-236

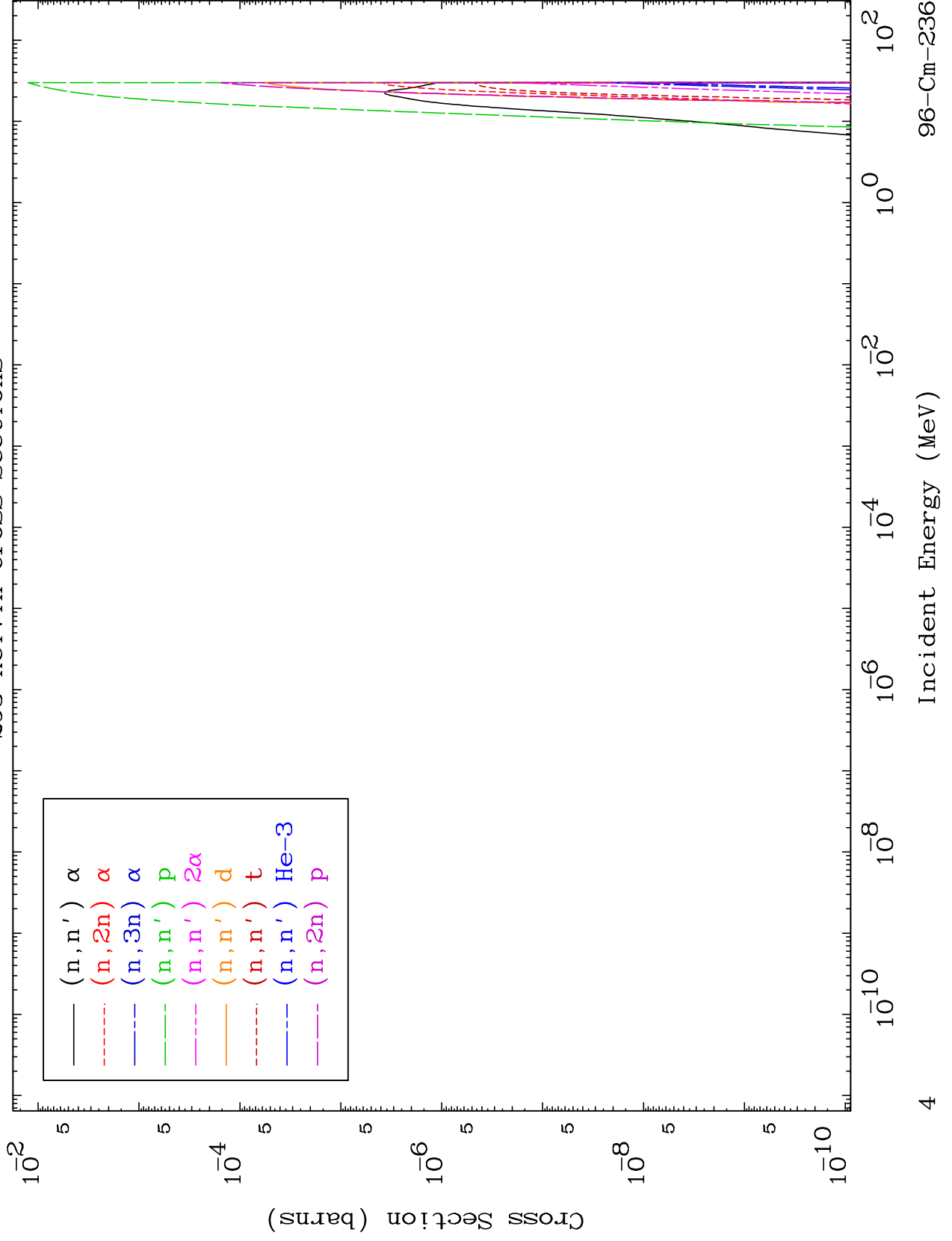




MAT 9613

Charged Particle
293 Kelvin Cross Sections

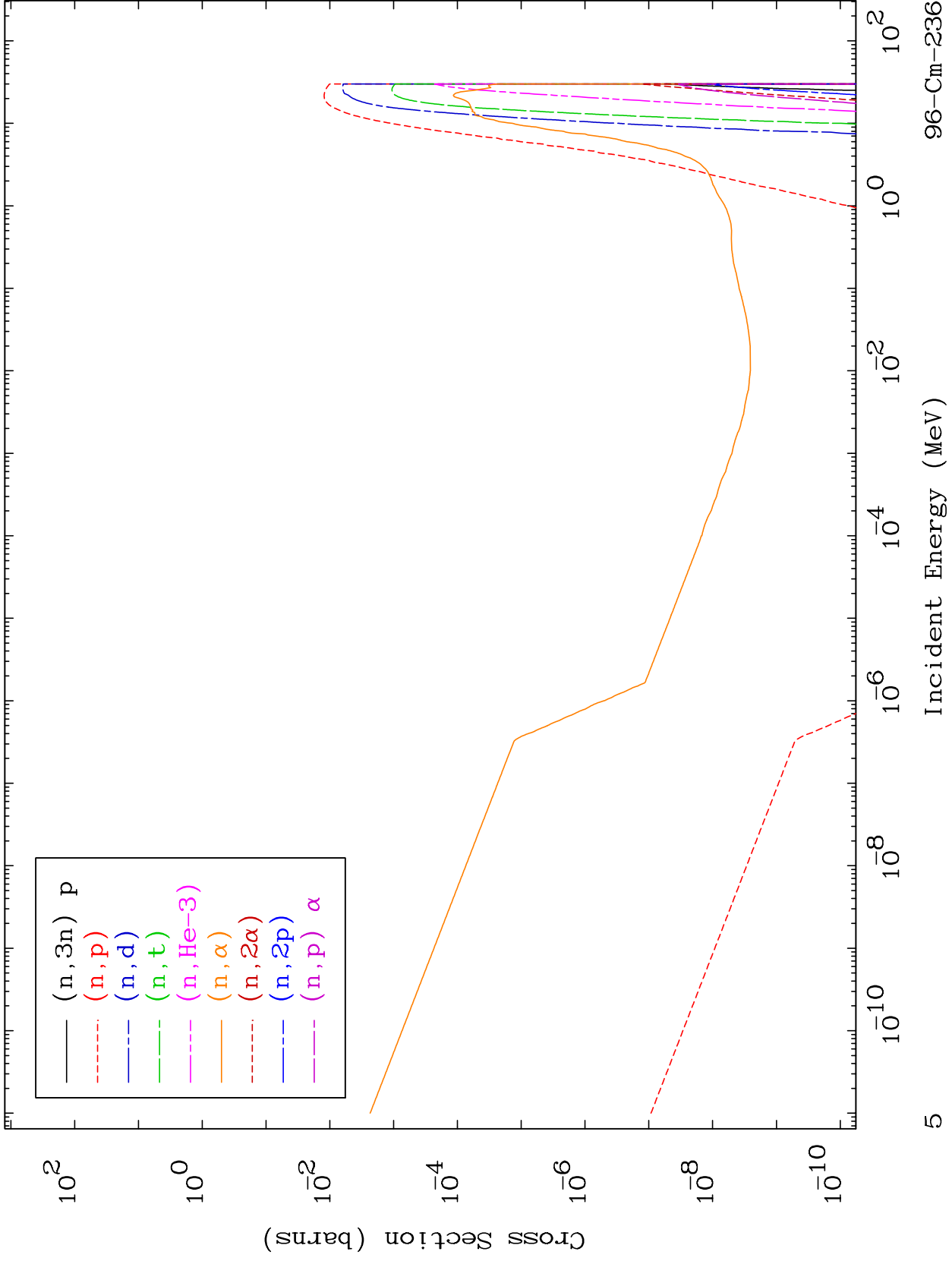
96-Cm-236



MAT 9613

Charged Particle
293 Kelvin Cross Sections

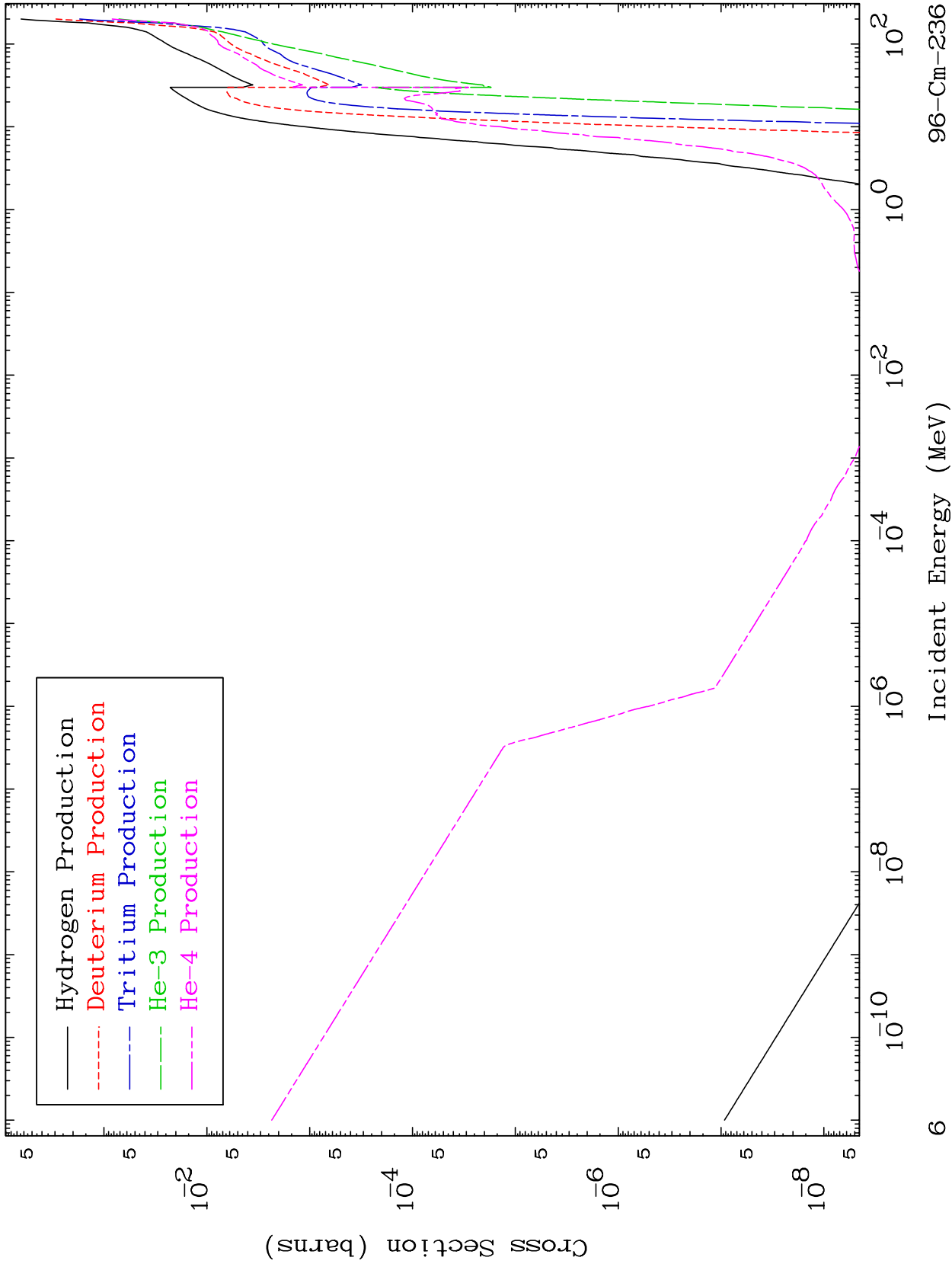
96-Cm-236



MAT 9613

Particle Production
293 Kelvin Cross Sections

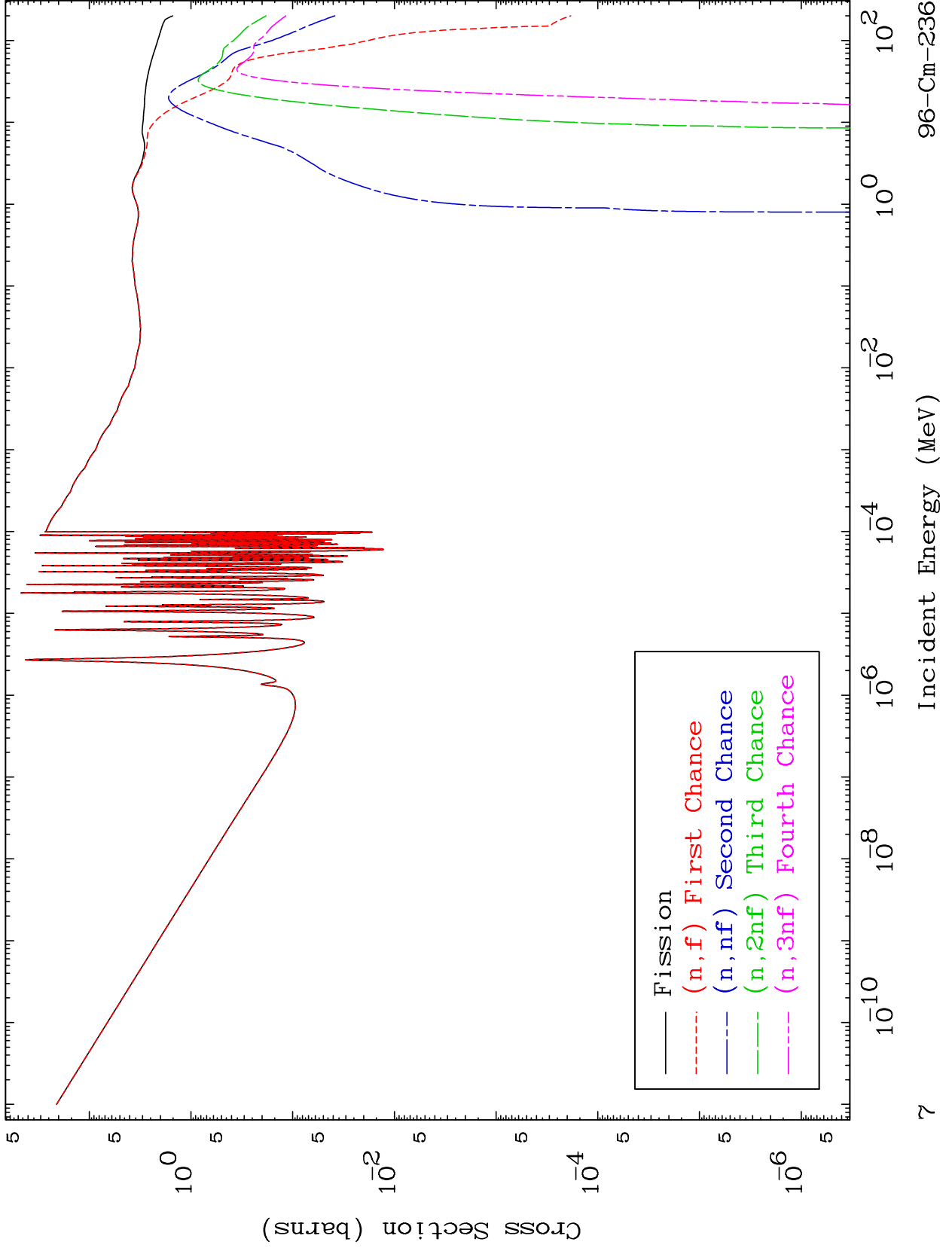
96-Cm-236



MAT 9613

Fission
293 Kelvin Cross Sections

96-Cm-236

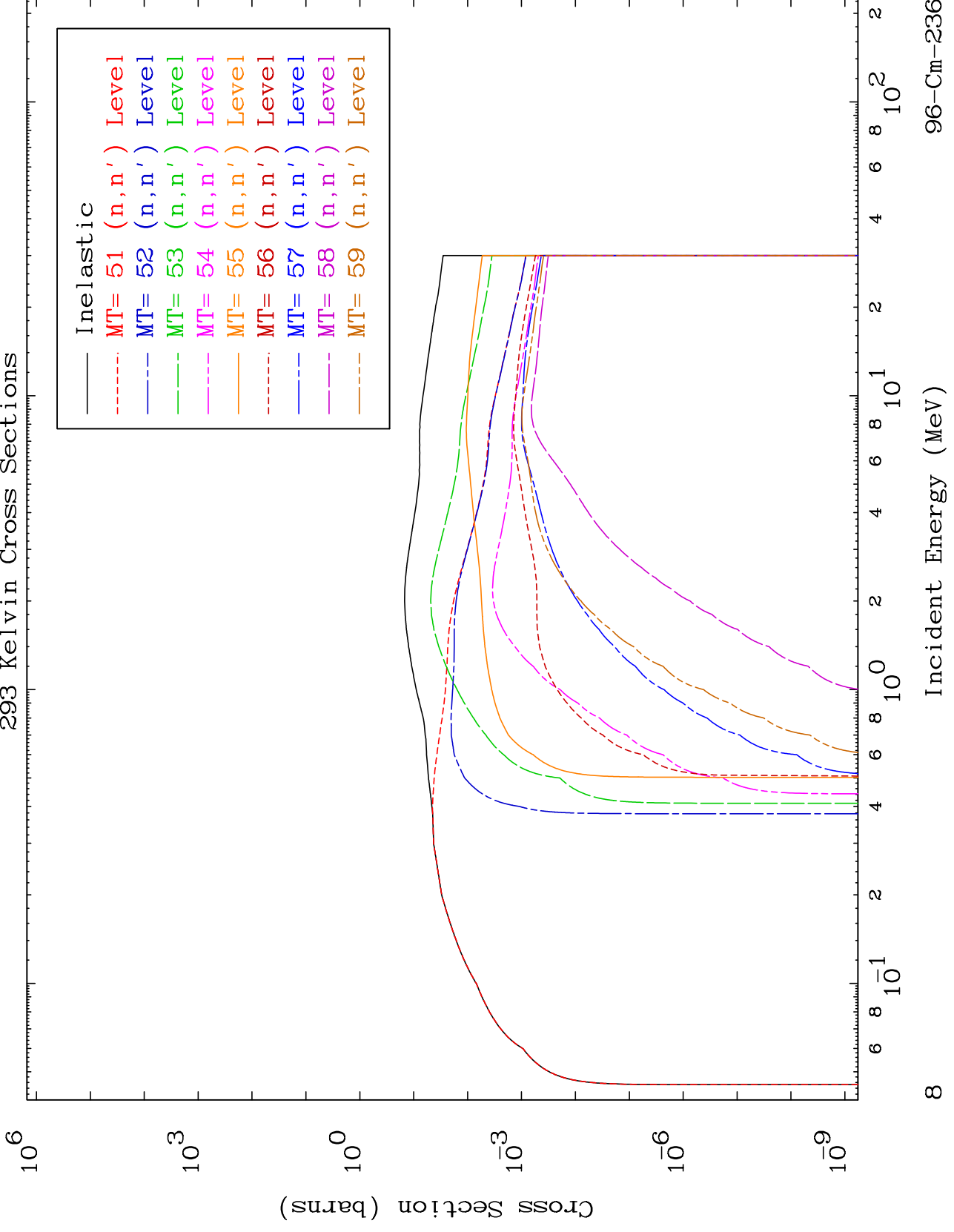


MAT 9613

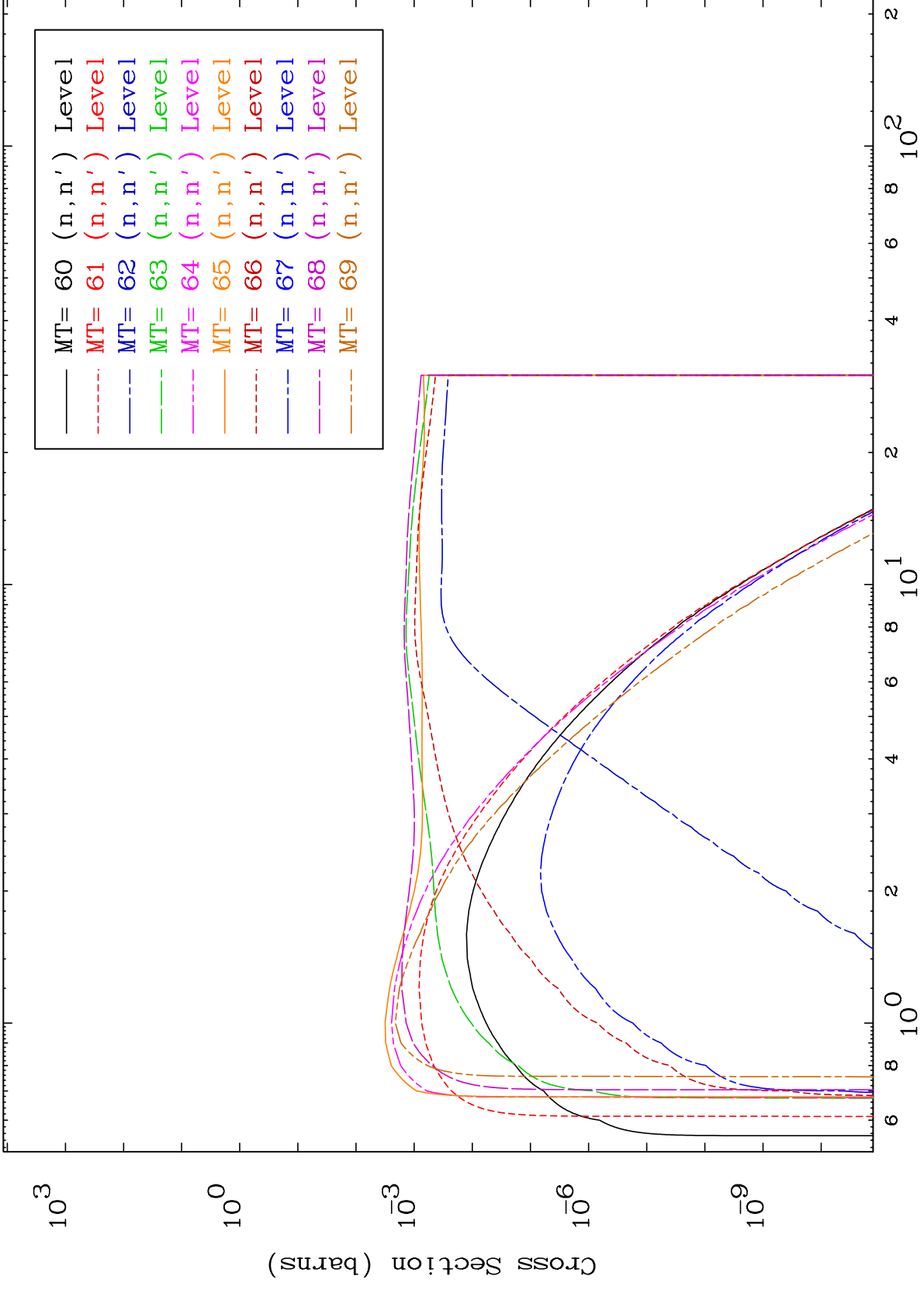
(n,n') Level

96-Cm-236

293 Kelvin Cross Sections



293 Kelvin Cross Sections

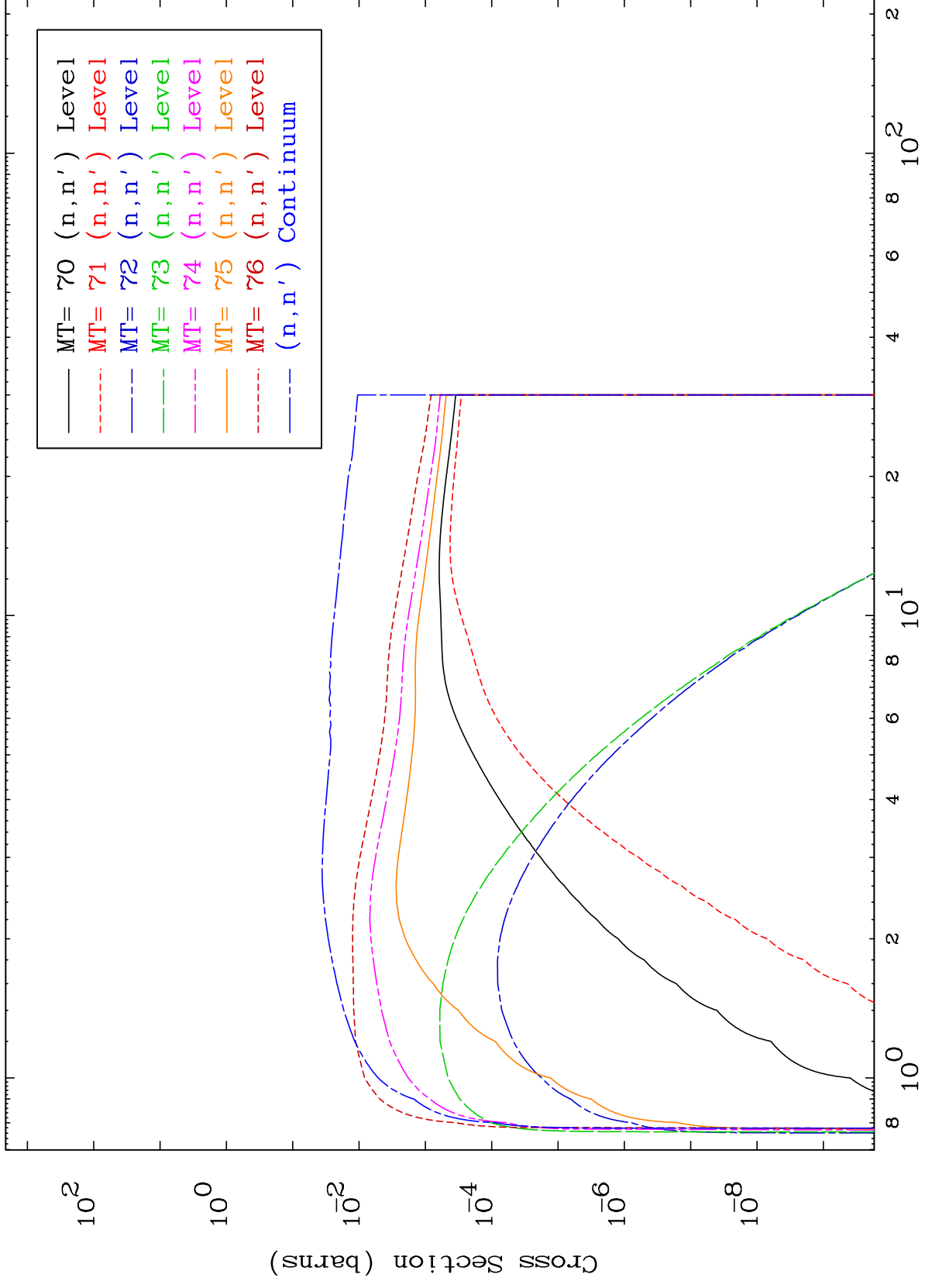


MAT 9613

(n,n') Level

293 Kelvin Cross Sections

96-Cm-236



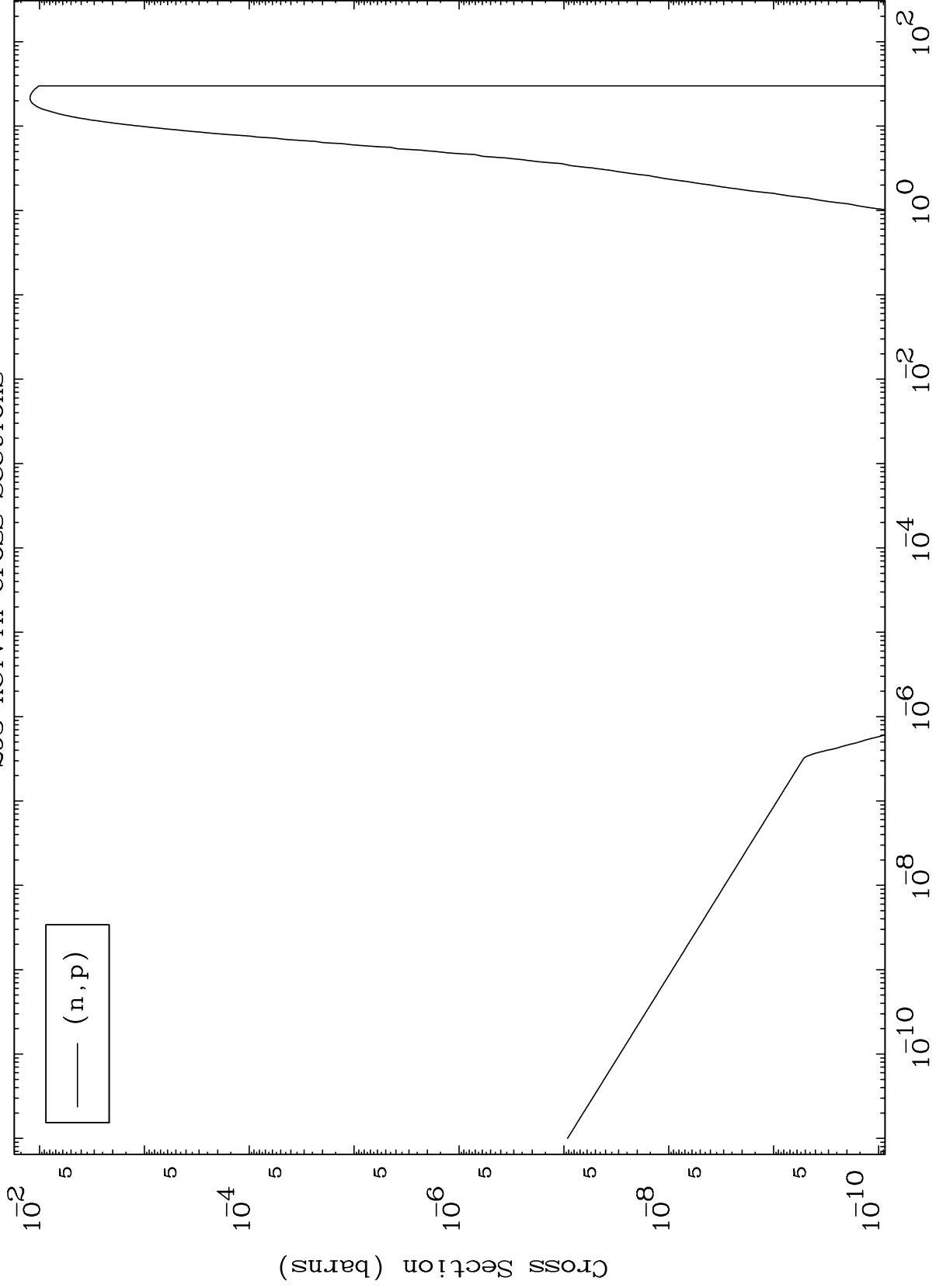
Incident Energy (MeV)

96-Cm-236

MAT 9613

(n,p) Levels
293 Kelvin Cross Sections

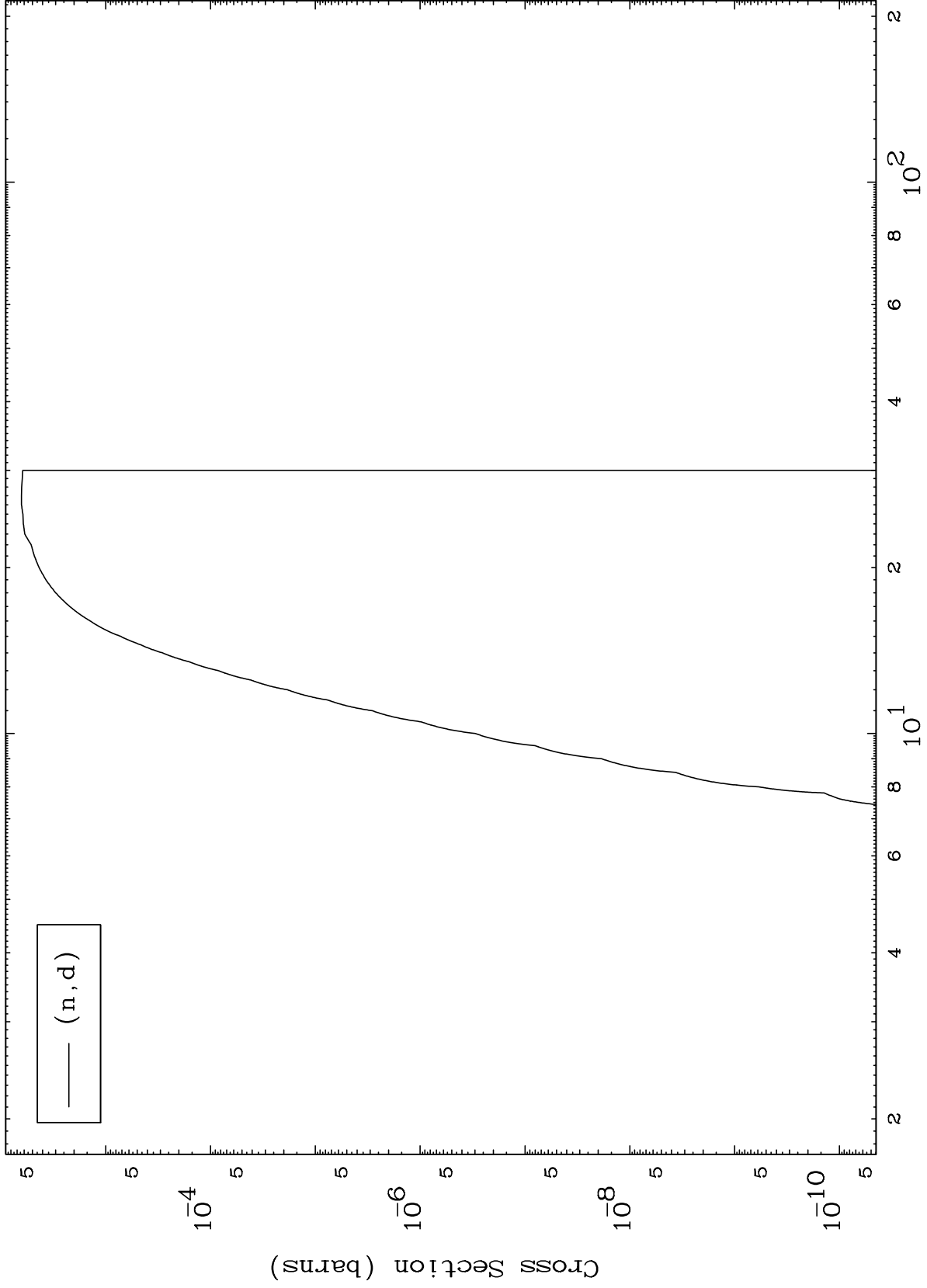
96-Cm-236



MAT 9613

(n,d) Levels
293 Kelvin Cross Sections

96-Cm-236



(n,d)

12

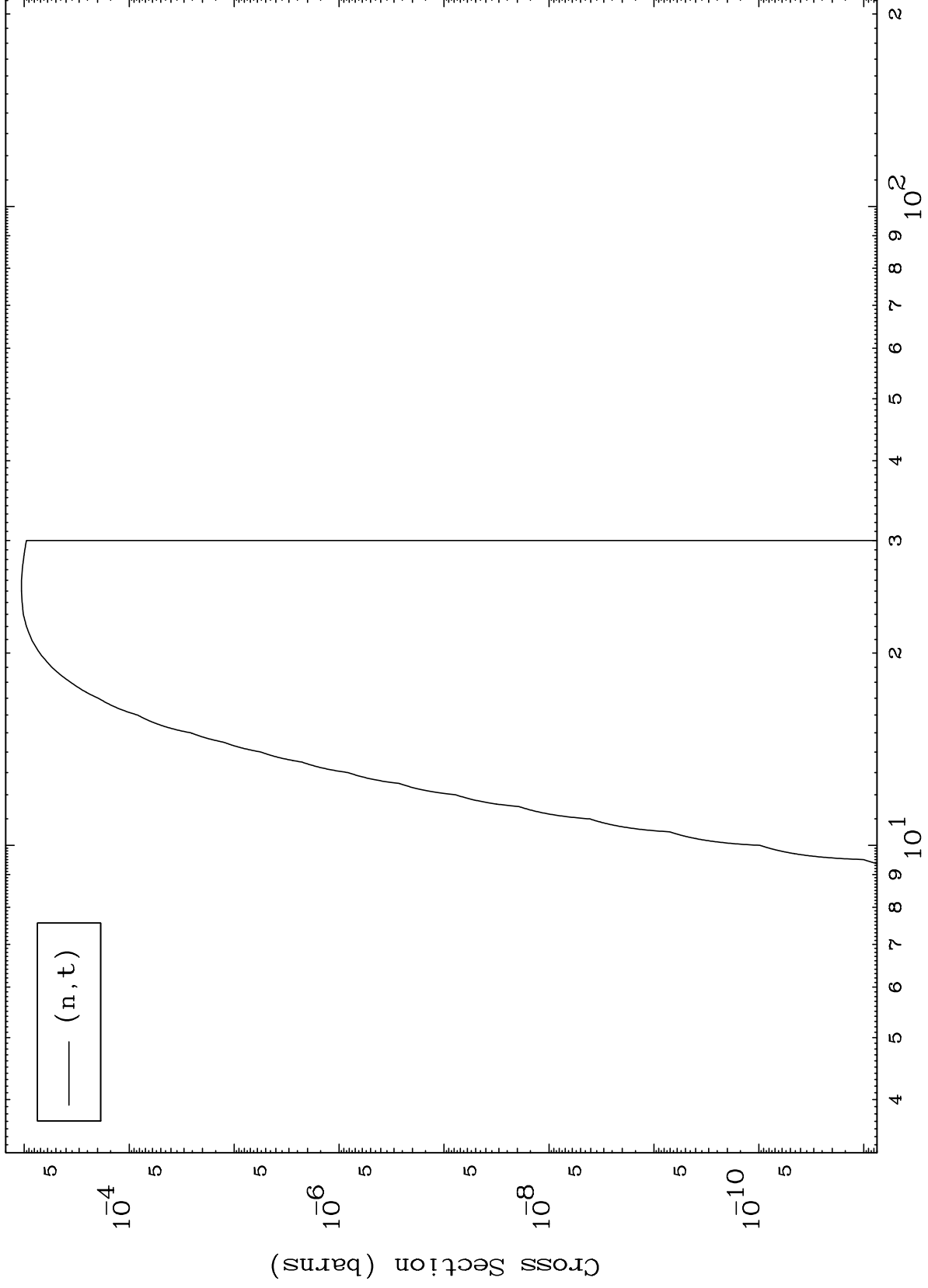
Incident Energy (MeV)

96-Cm-236

MAT 9613

(n,t) Levels
293 Kelvin Cross Sections

96-Cm-236



13

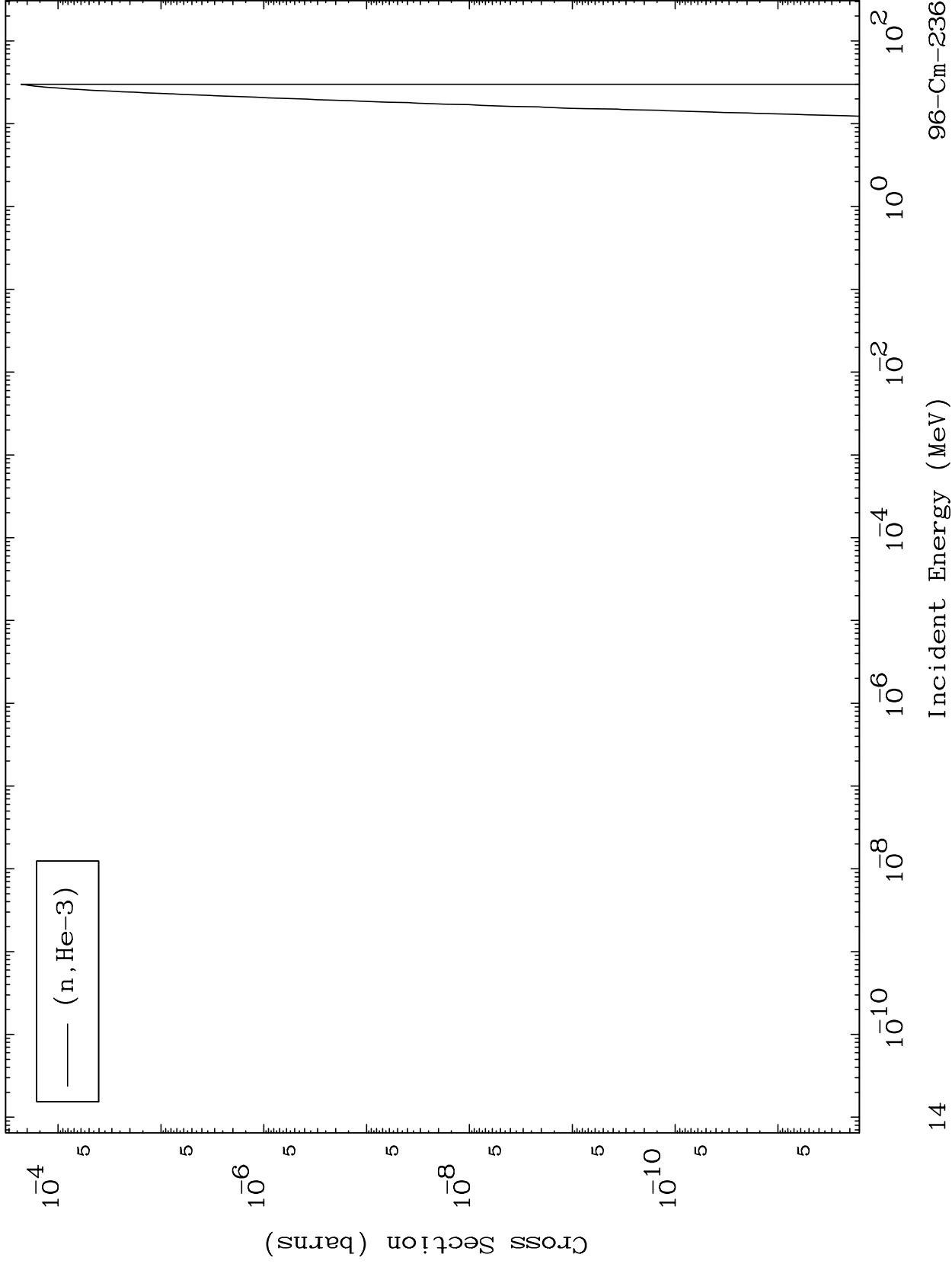
Incident Energy (MeV)

96-Cm-236

MAT 9613

(n,He3) Levels
293 Kelvin Cross Sections

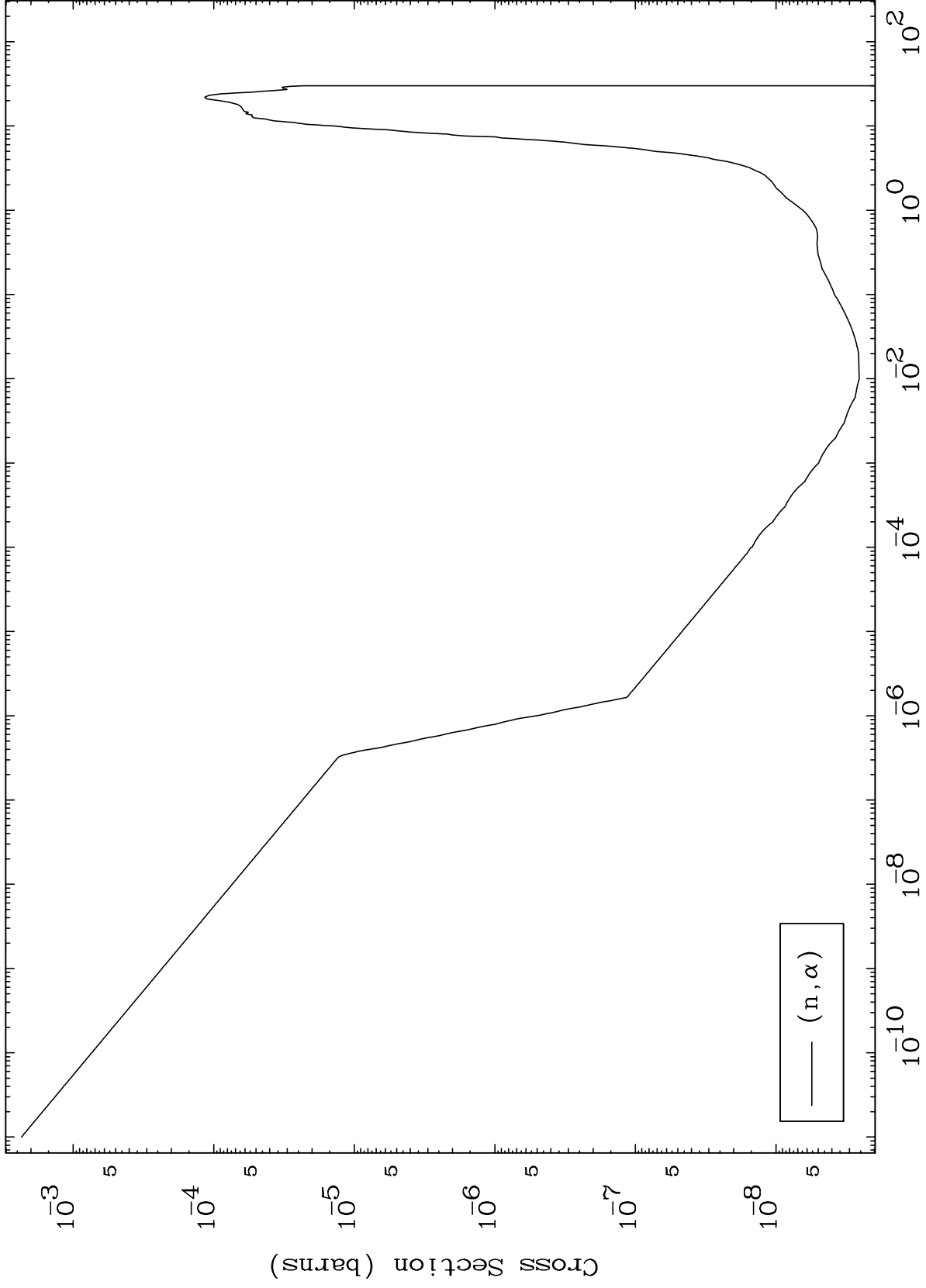
96-Cm-236



MAT 9613

(n,α) Levels
293 Kelvin Cross Sections

96-Cm-236



15

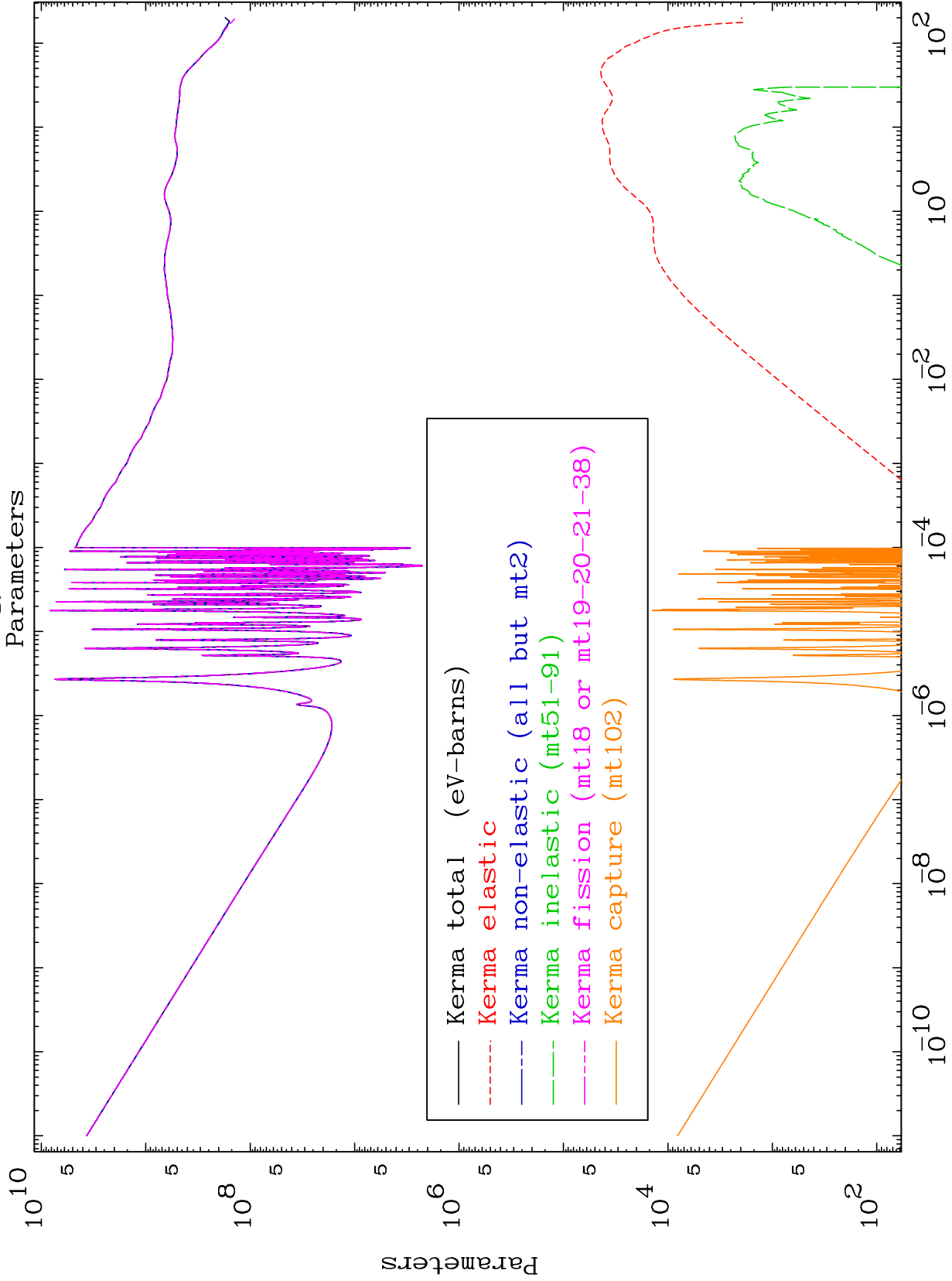
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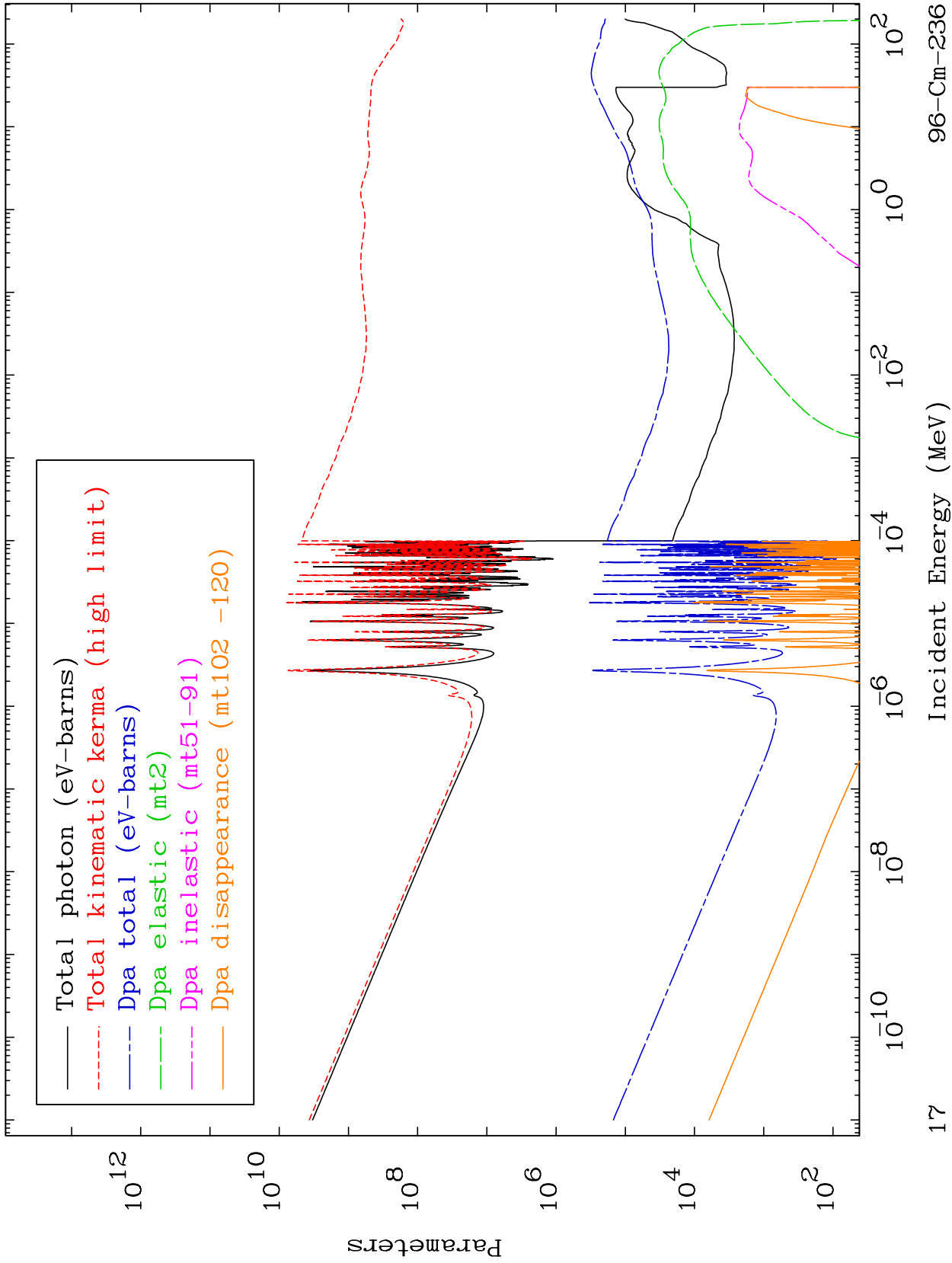
96-Cm-236

MAT 9613

Energy Release
Parameters

96-Cm-236

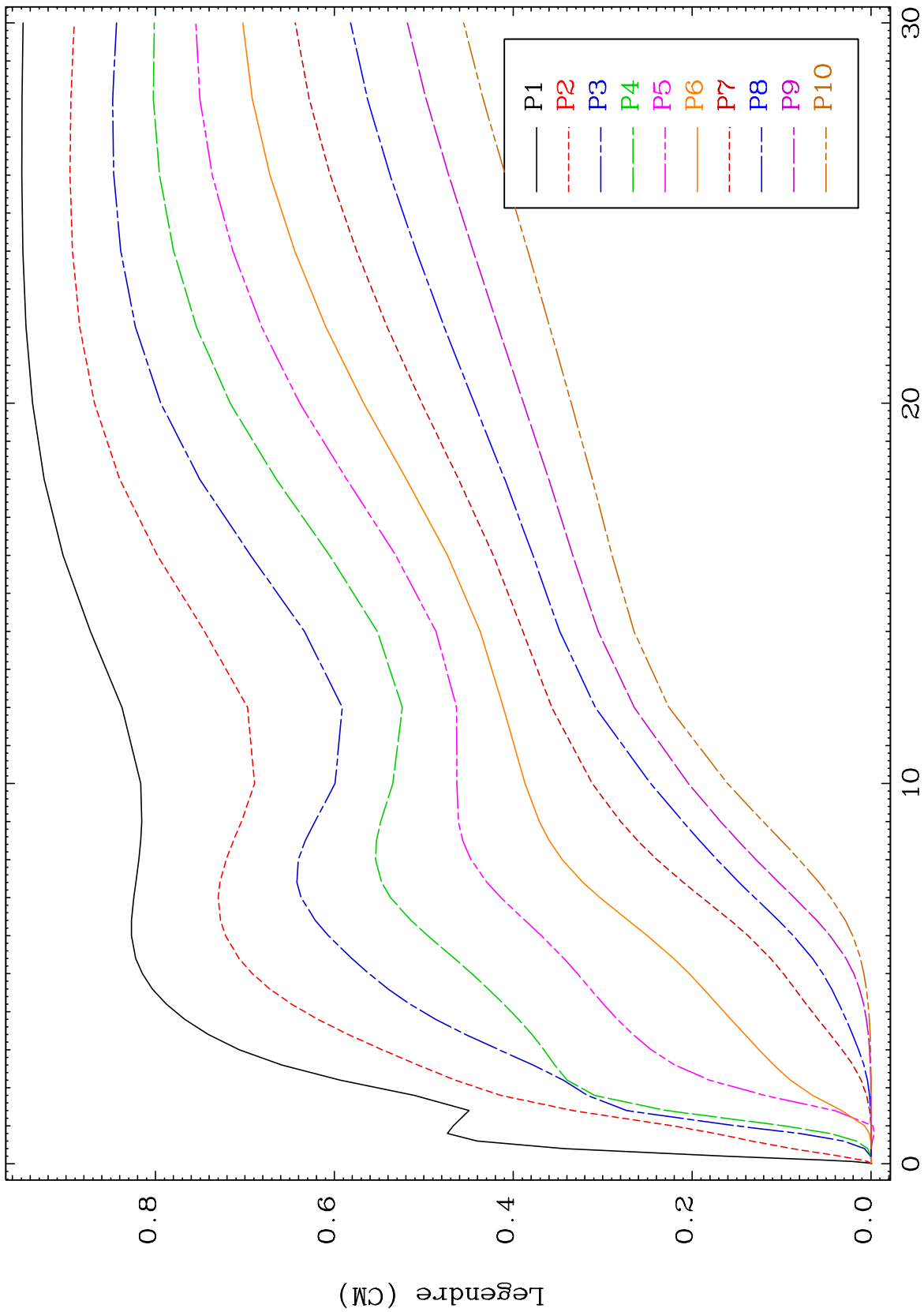




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96-Cm-236

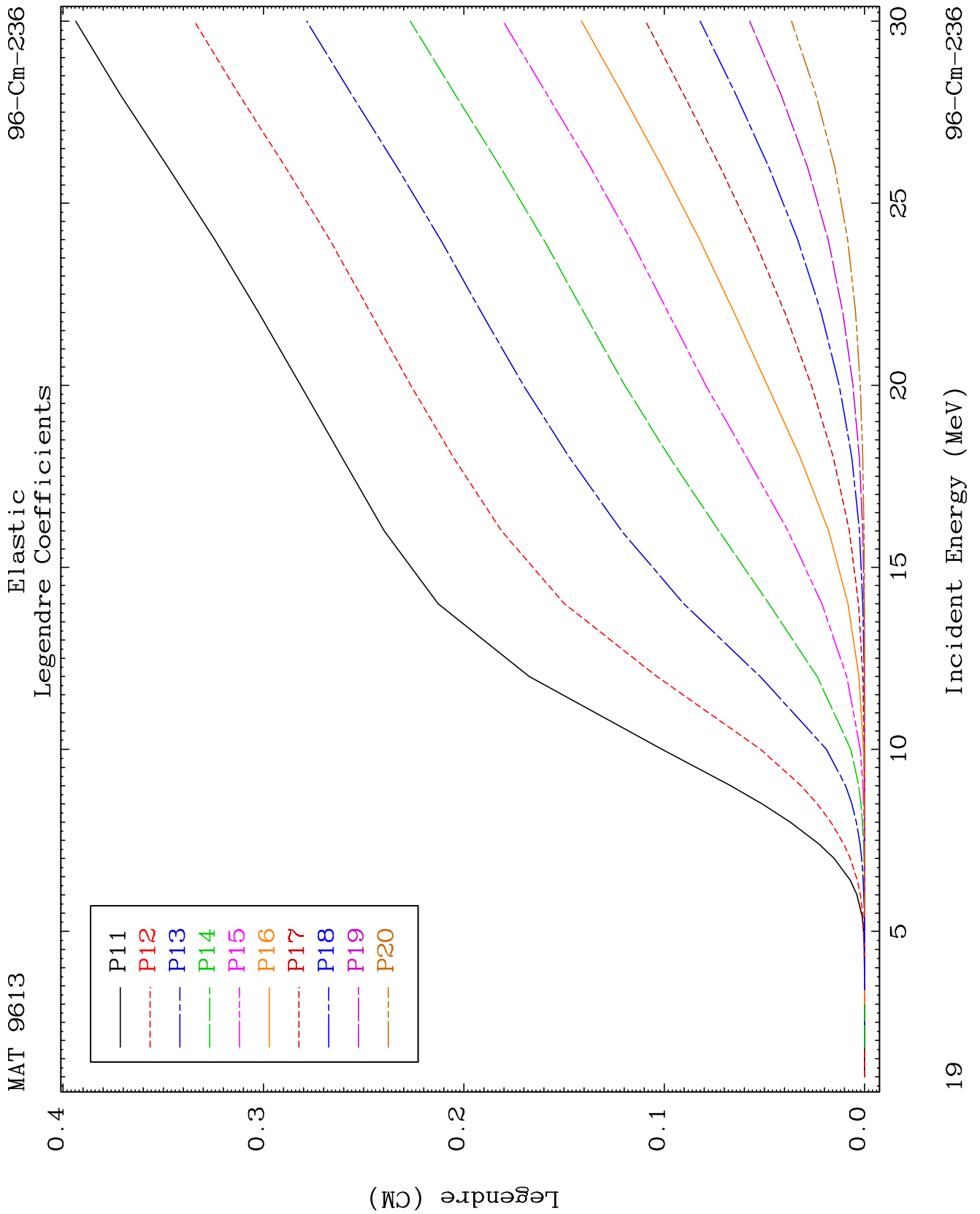
Elastic Legendre Coefficients



18

Incident Energy (MeV)

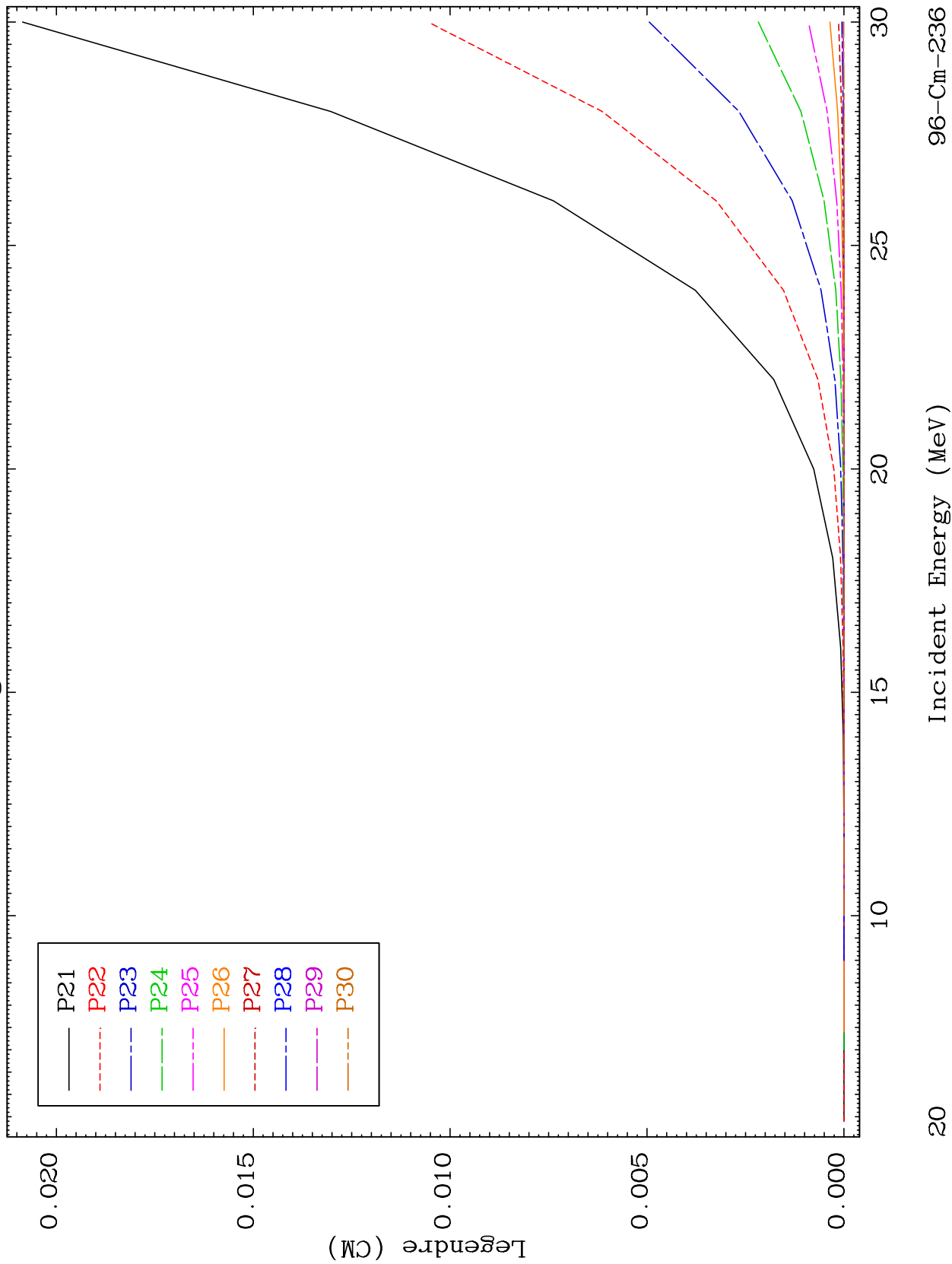
96-Cm-236



MAT 9613

Elastic Legendre Coefficients

96-Cm-236



20

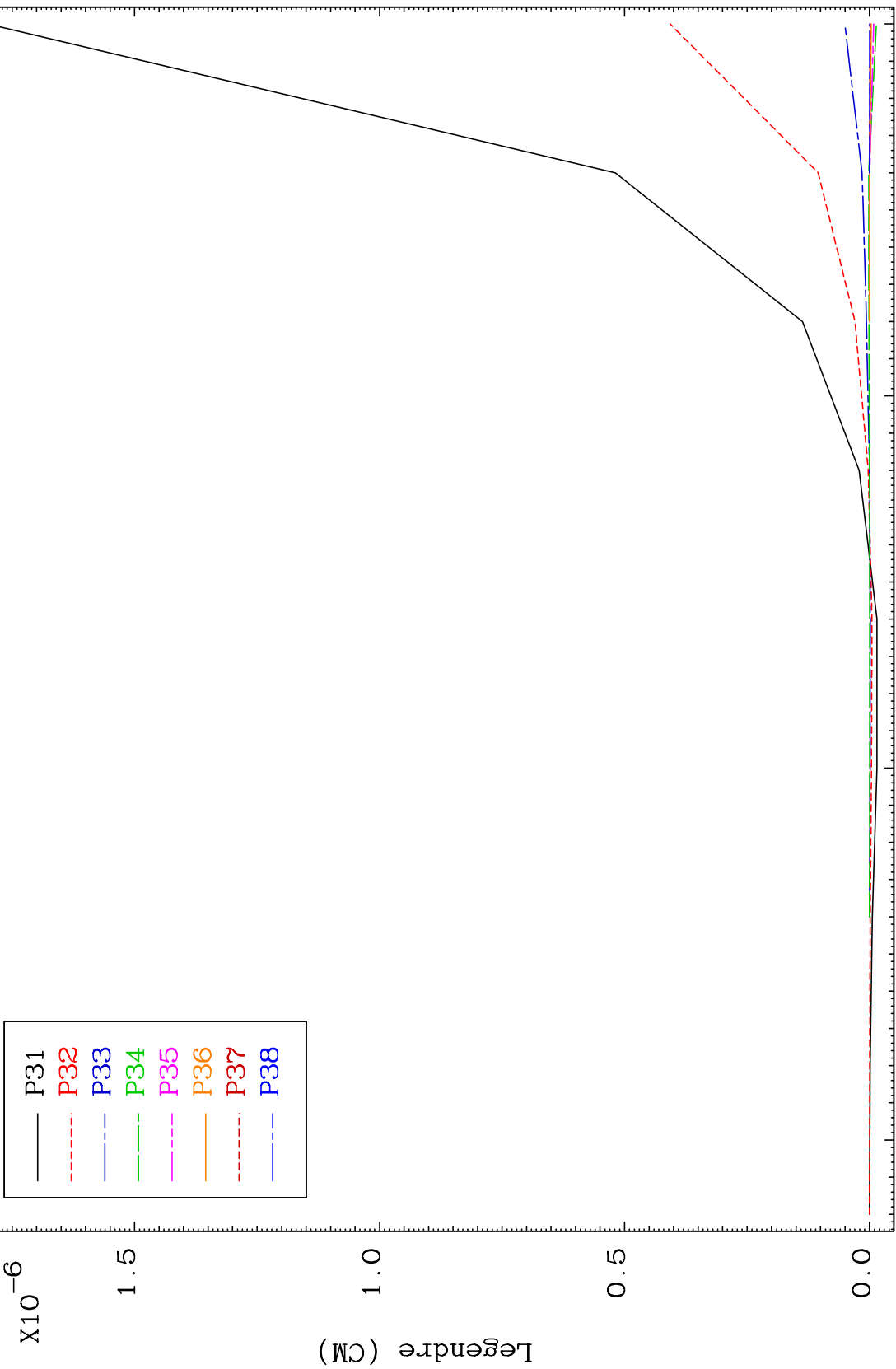
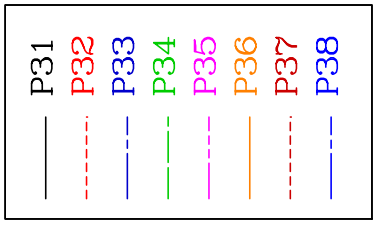
Incident Energy (MeV)

96-Cm-236

MAT 9613

Elastic Legendre Coefficients

96-Cm-236



21

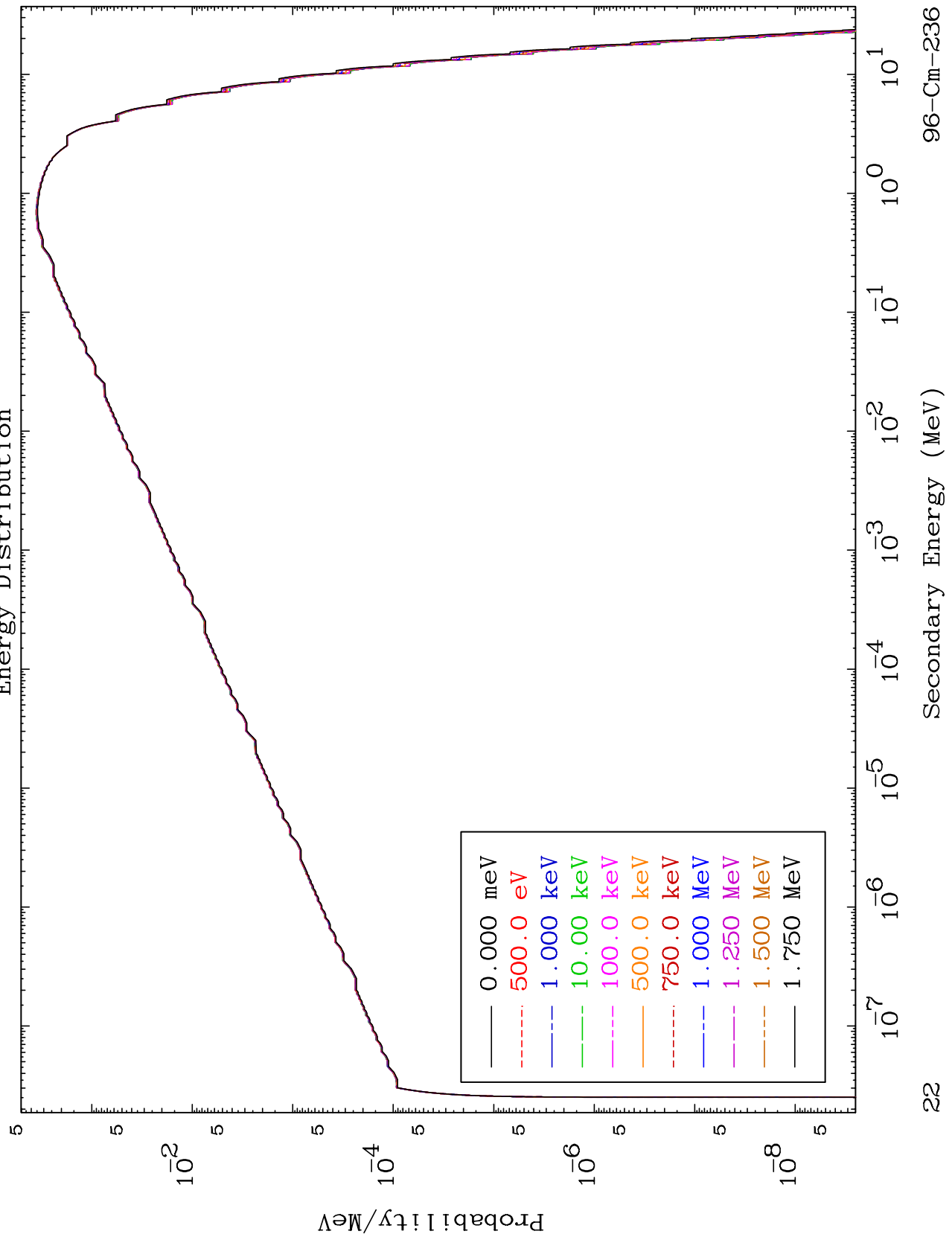
Incident Energy (MeV)

96-Cm-236

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Fission Energy Distribution

96-Cm-236

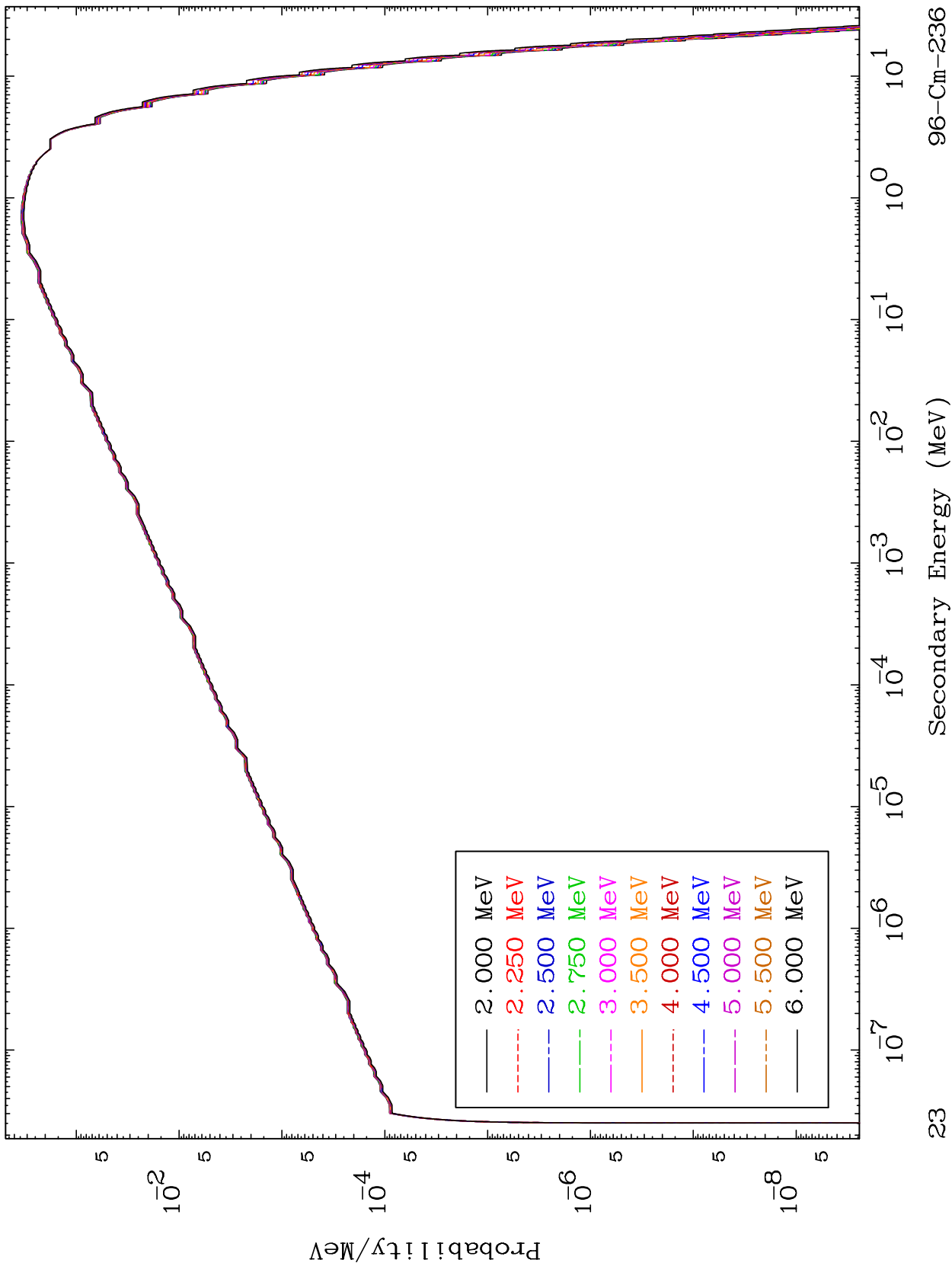


96-Cm-236

MAT 9613

Fission Energy Distribution

96-Cm-236

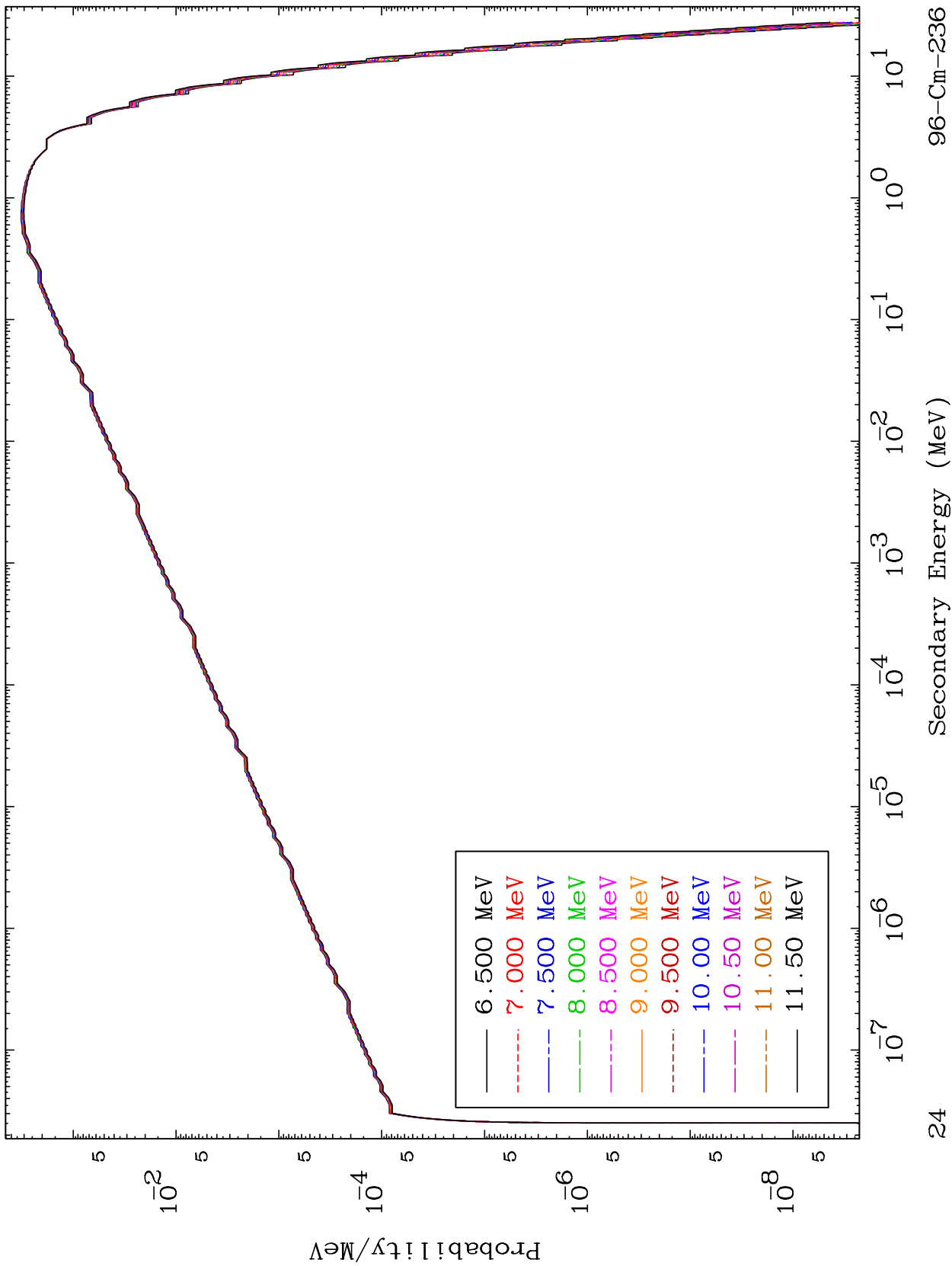


96-Cm-236

MAT 9613

Fission Energy Distribution

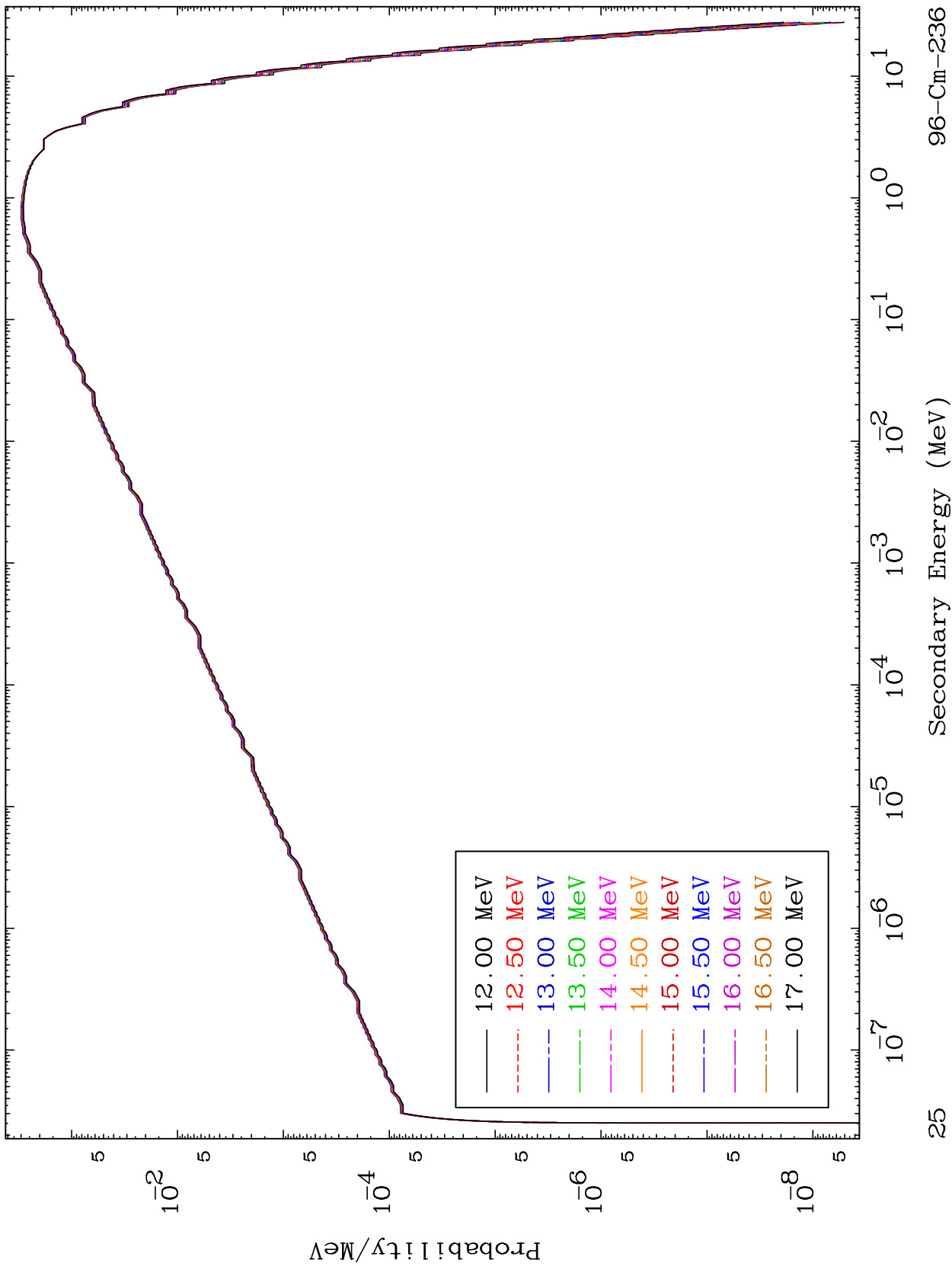
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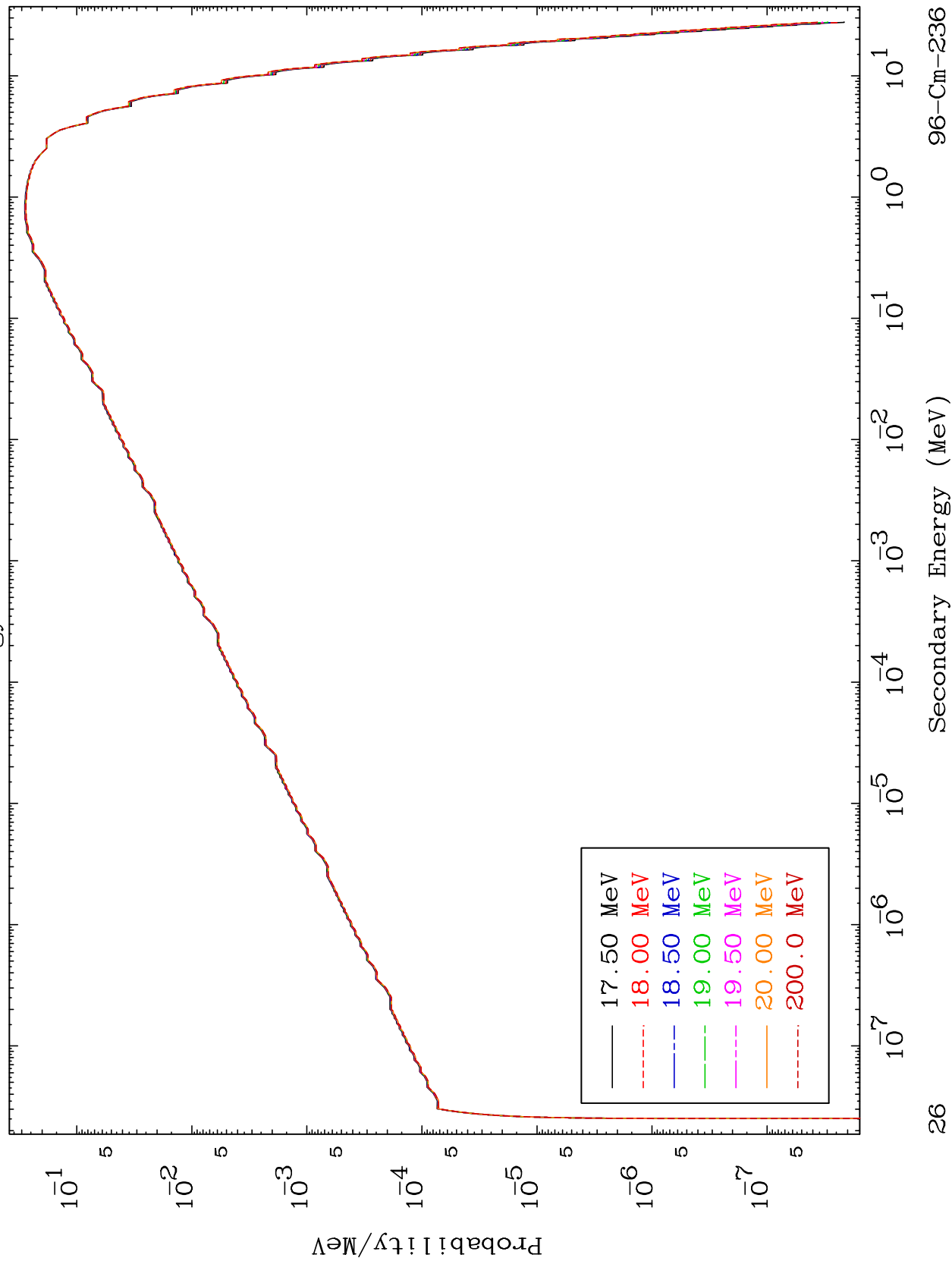
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Fission Energy Distribution

96-Cm-236



Fission
Energy Distribution



MAT 9613

96-Cm-236

(n,p)
Radionuclide Production Cross Section

