

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net

Web: redcullen1.net/HOMEPAGE.NEW

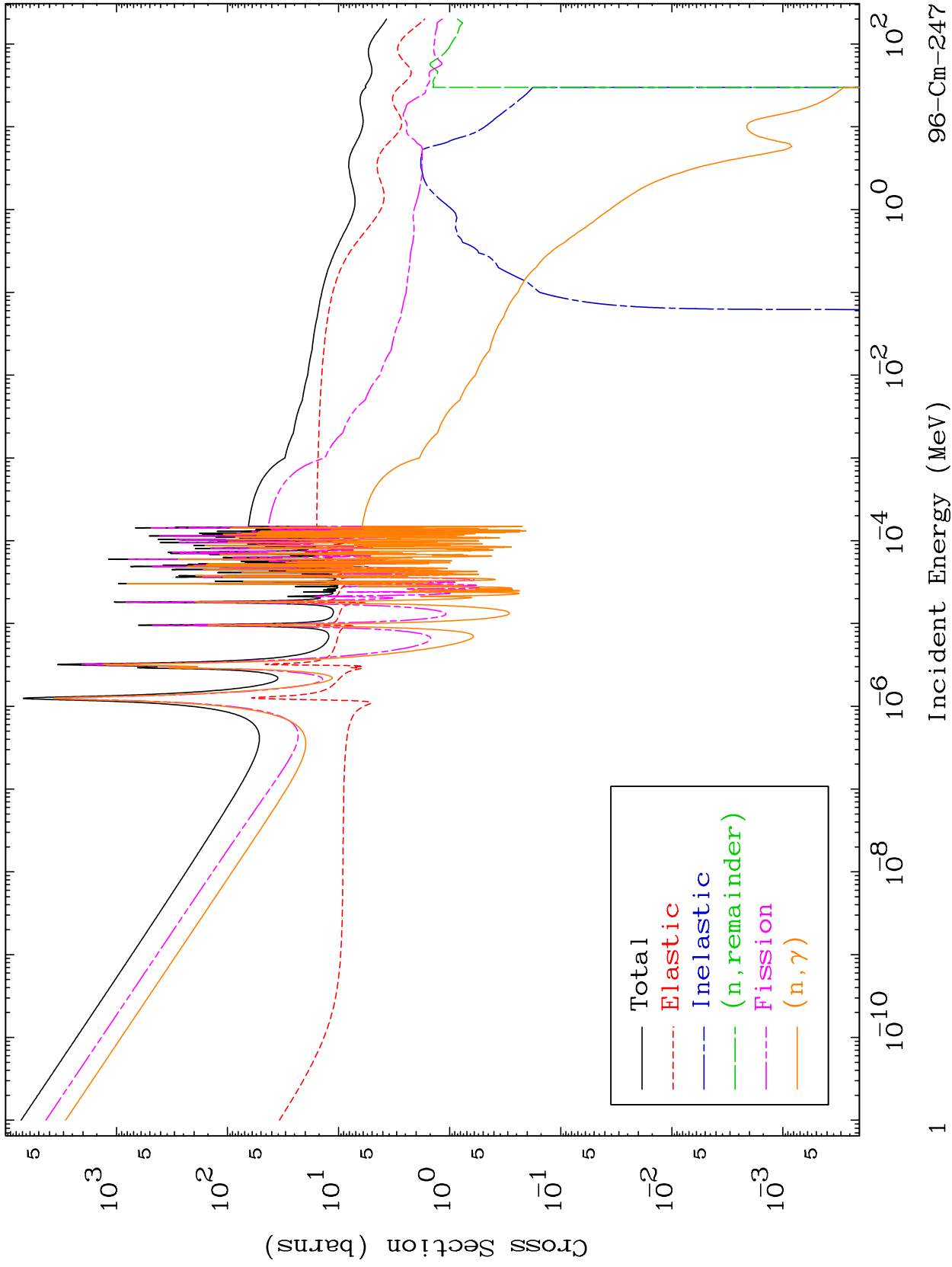
Press Mouse Button to Start

MAT 9646

Major

293 Kelvin Cross Sections

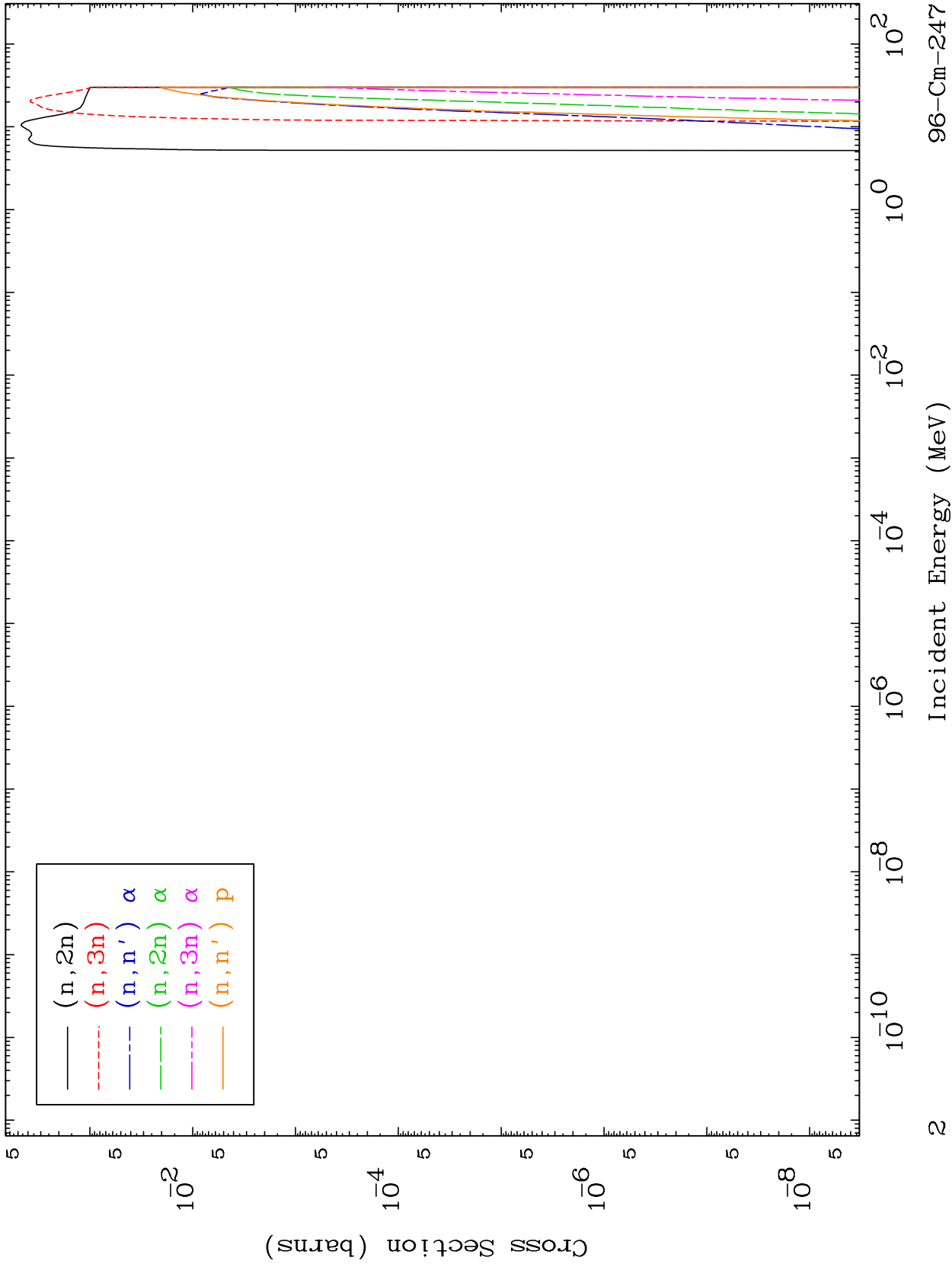
96-Cm-247



MAT 9646

Neutron Production
293 Kelvin Cross Sections

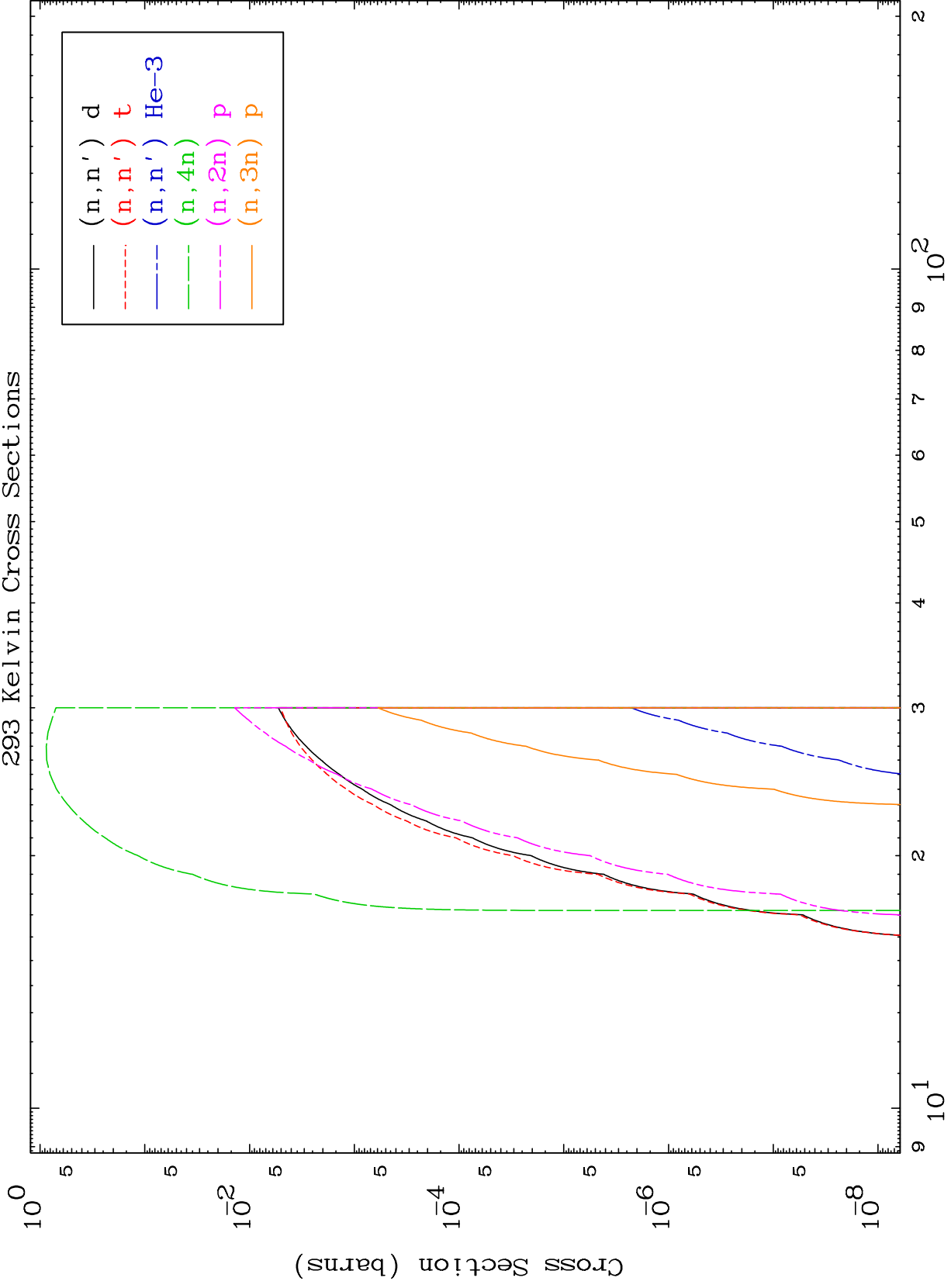
96-Cm-247



MAT 9646

Neutron Production
293 Kelvin Cross Sections

96-Cm-247



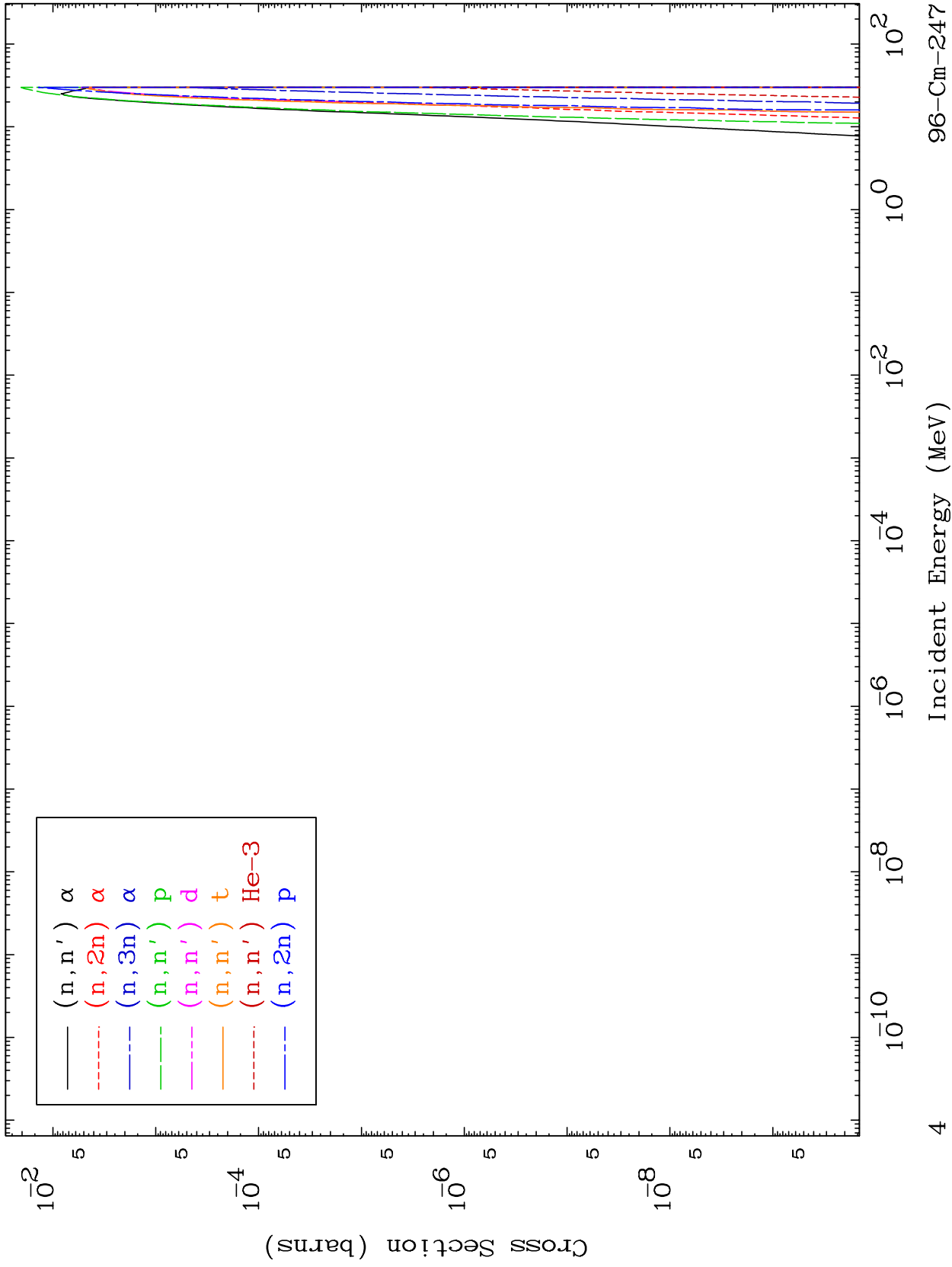
Incident Energy (MeV)

96-Cm-247

MAT 9646

Charged Particle
293 Kelvin Cross Sections

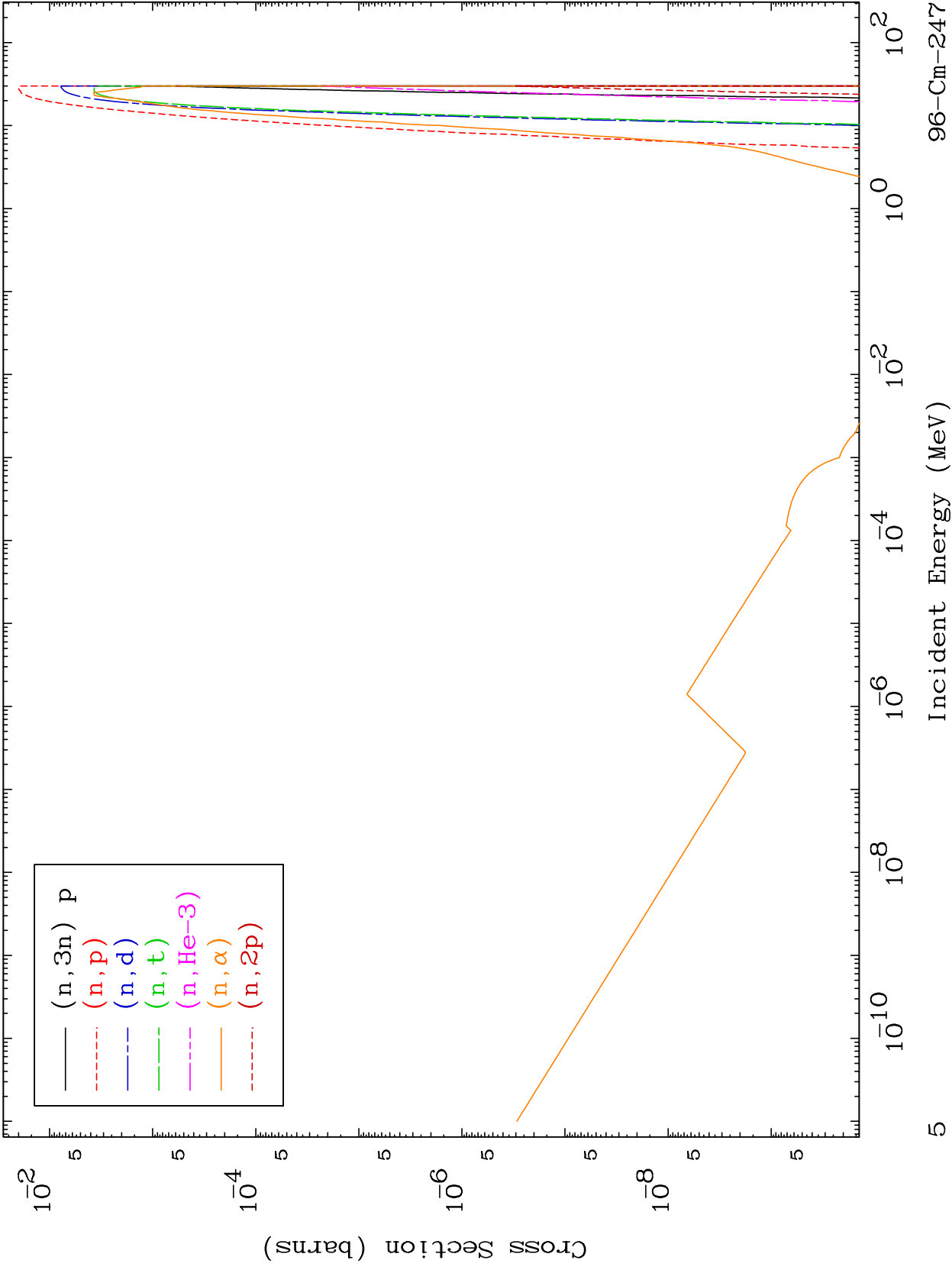
96-Cm-247



MAT 9646

Charged Particle
293 Kelvin Cross Sections

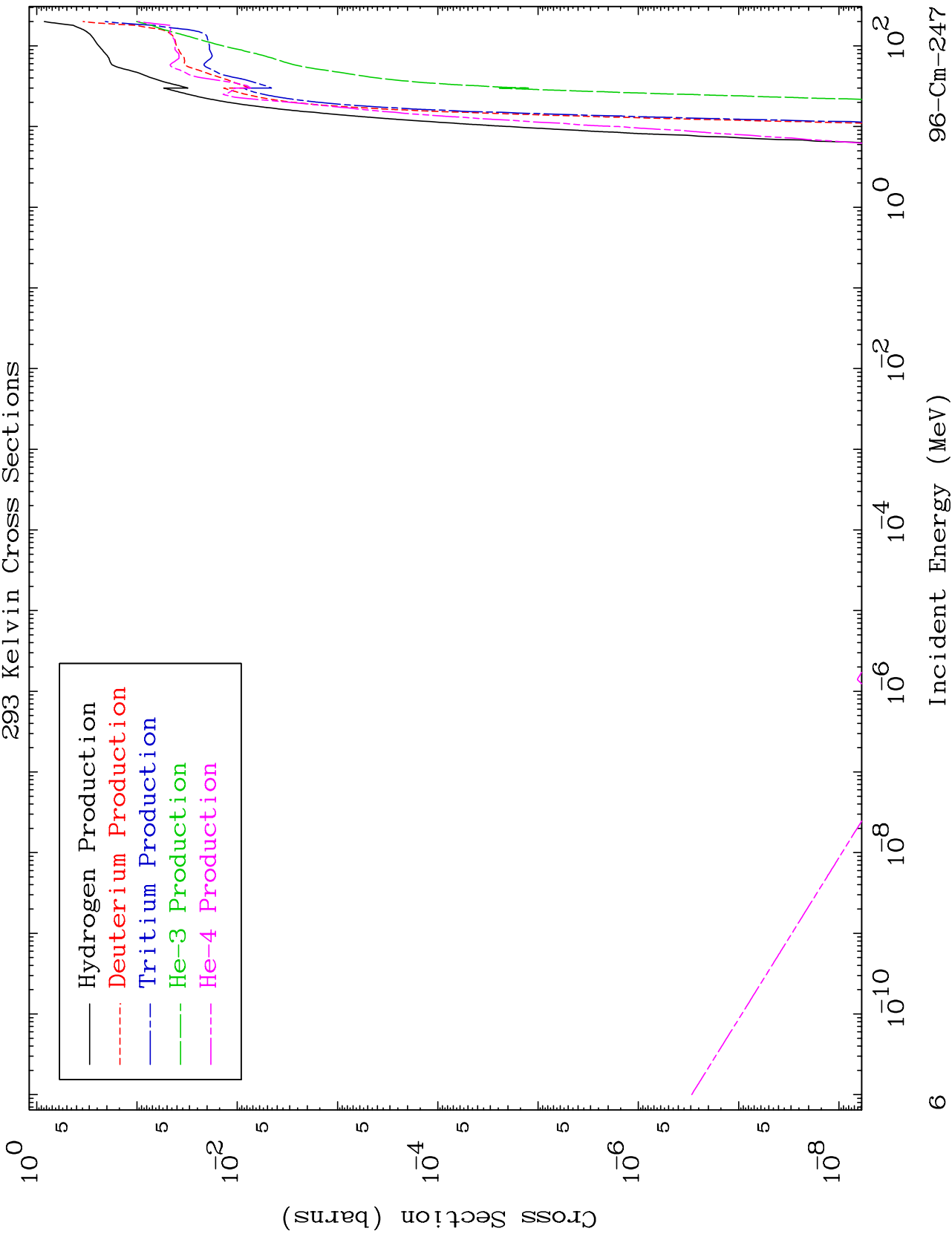
96-Cm-247



MAT 9646

Particle Production
293 Kelvin Cross Sections

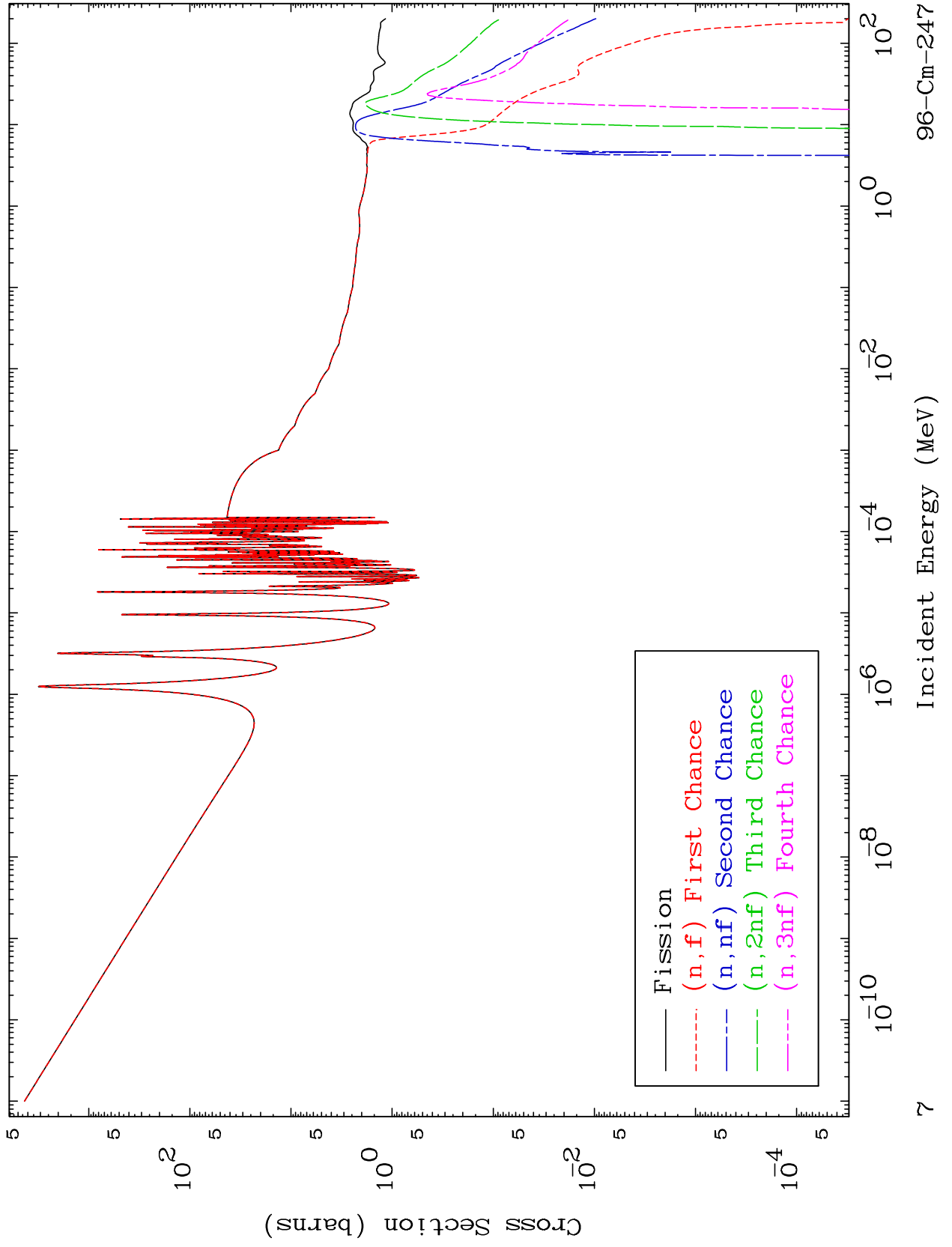
96-Cm-247



MAT 9646

Fission
293 Kelvin Cross Sections

96-Cm-247

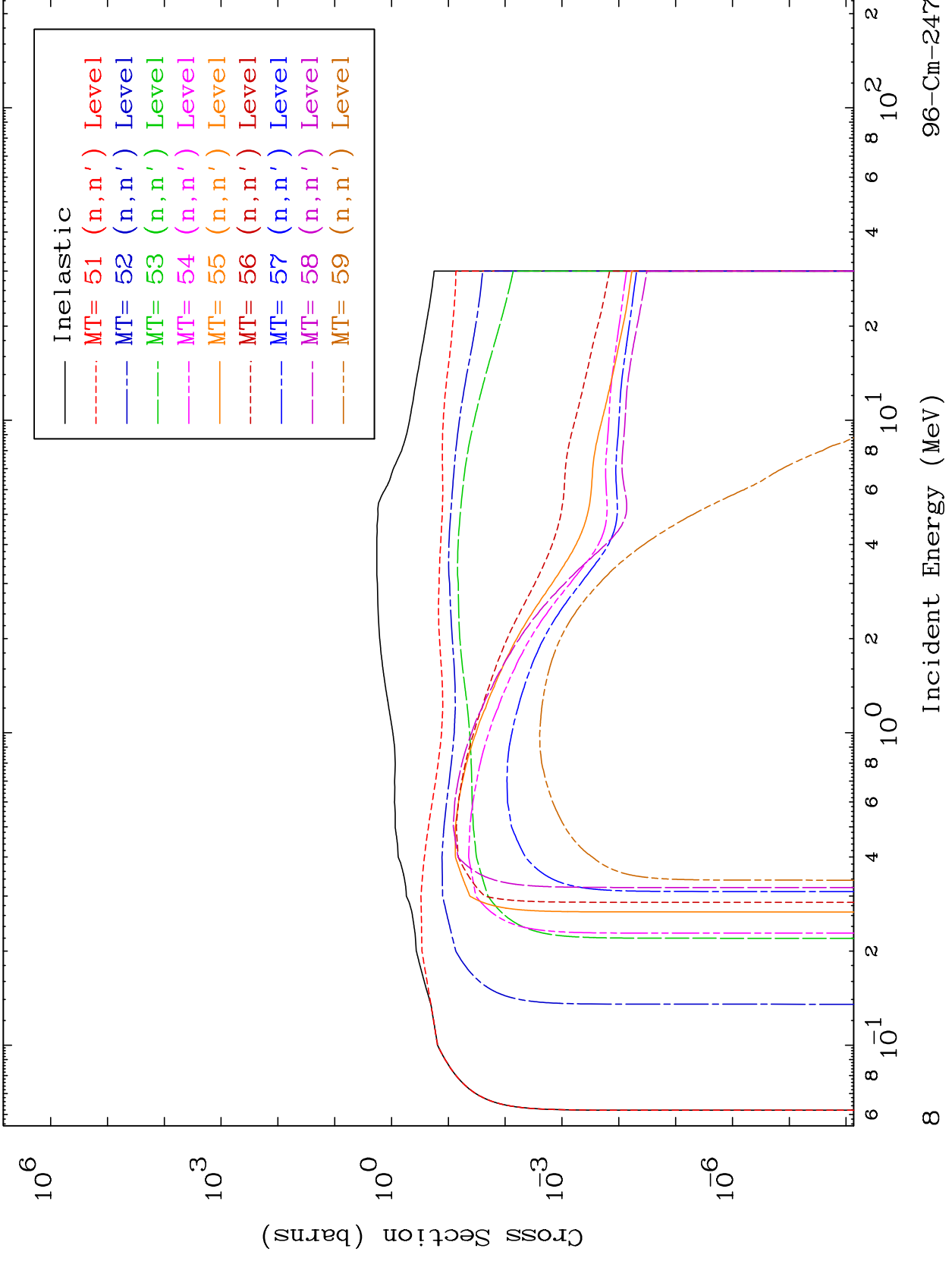


MAT 9646

(n,n') Level

96-Cm-247

293 Kelvin Cross Sections

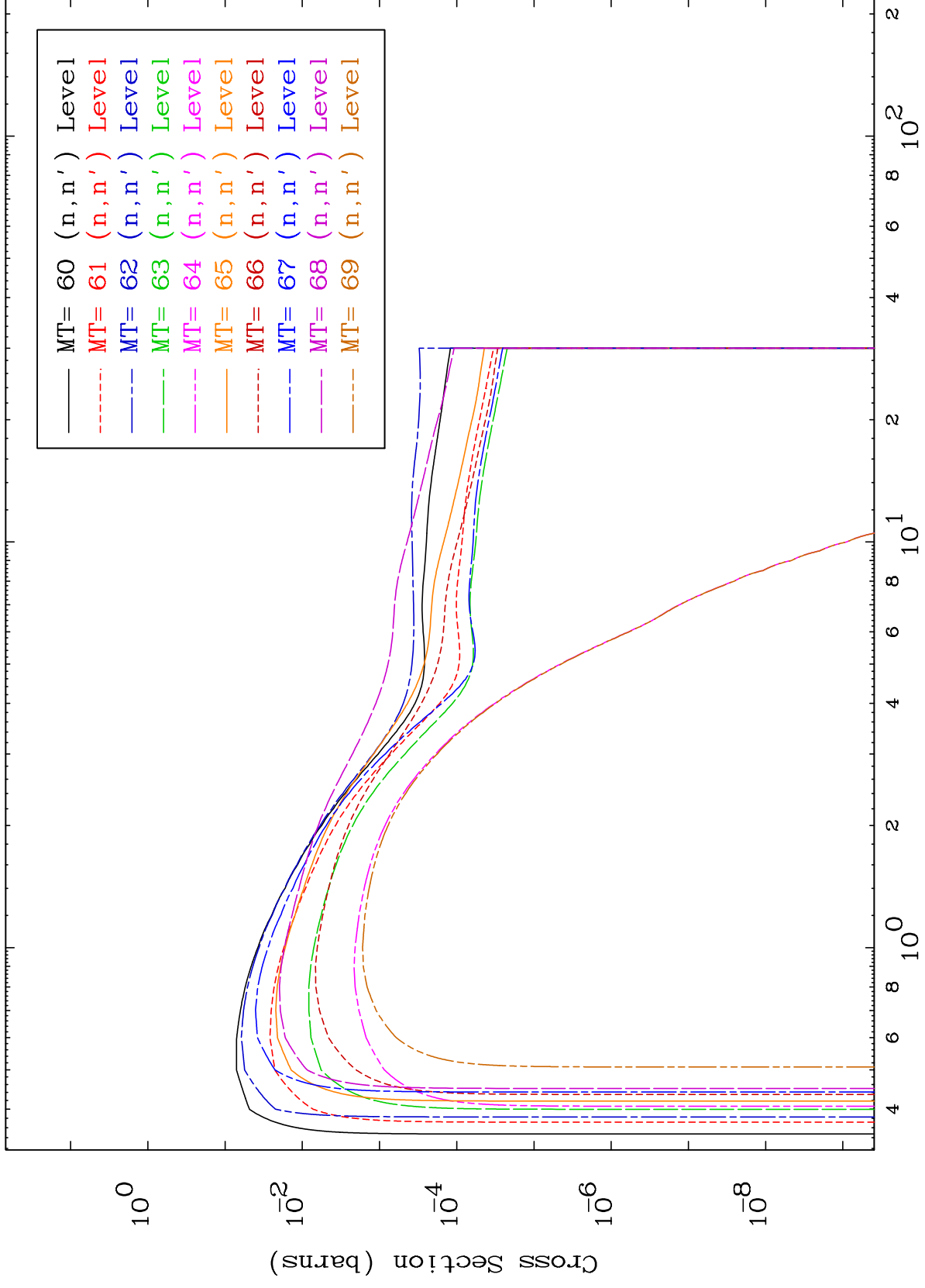


MAT 9646

(n,n') Level

96-Cm-247

293 Kelvin Cross Sections



9

Incident Energy (MeV)

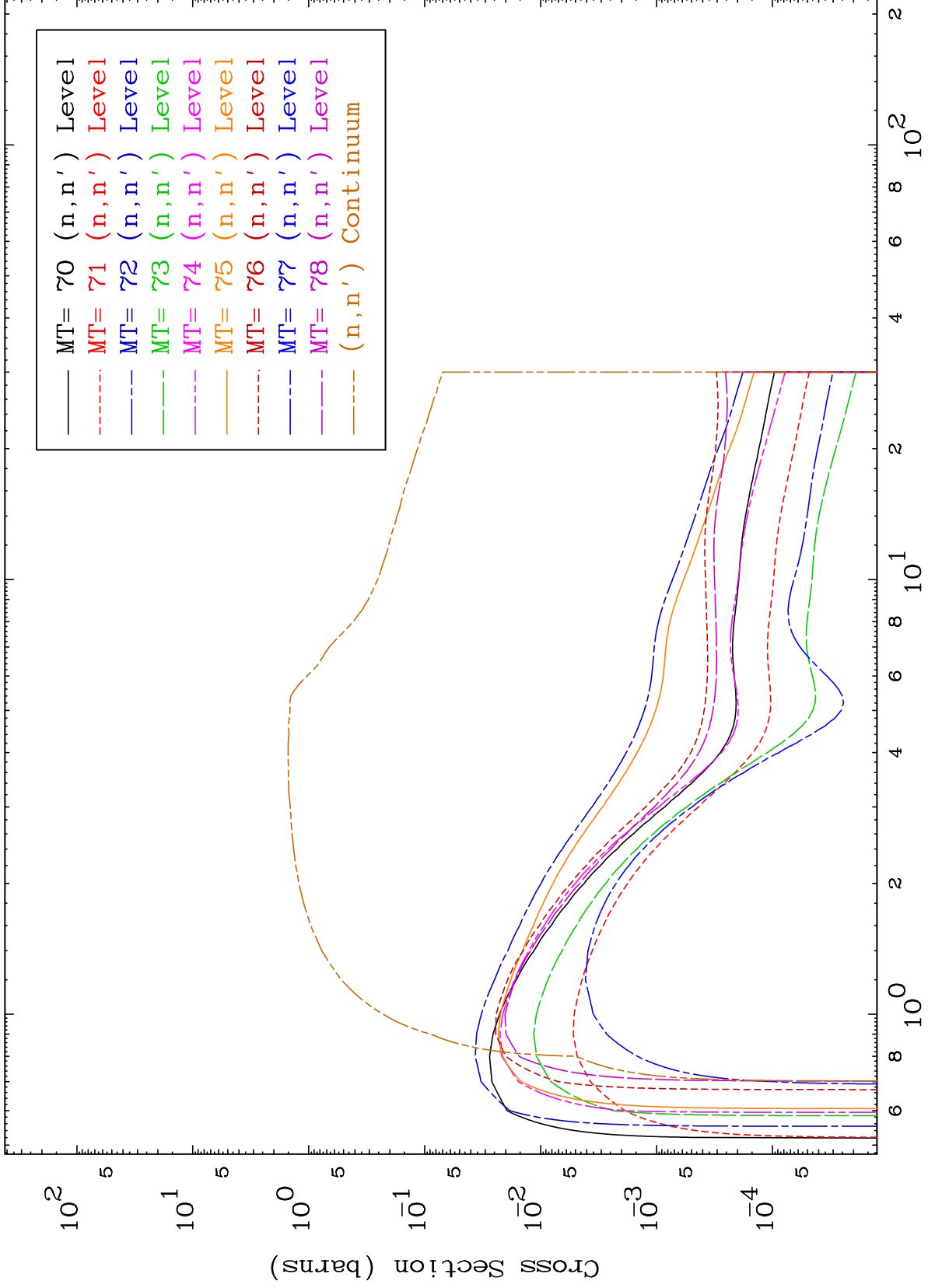
96-Cm-247

MAT 9646

(n,n') Level

96-Cm-247

293 Kelvin Cross Sections



10

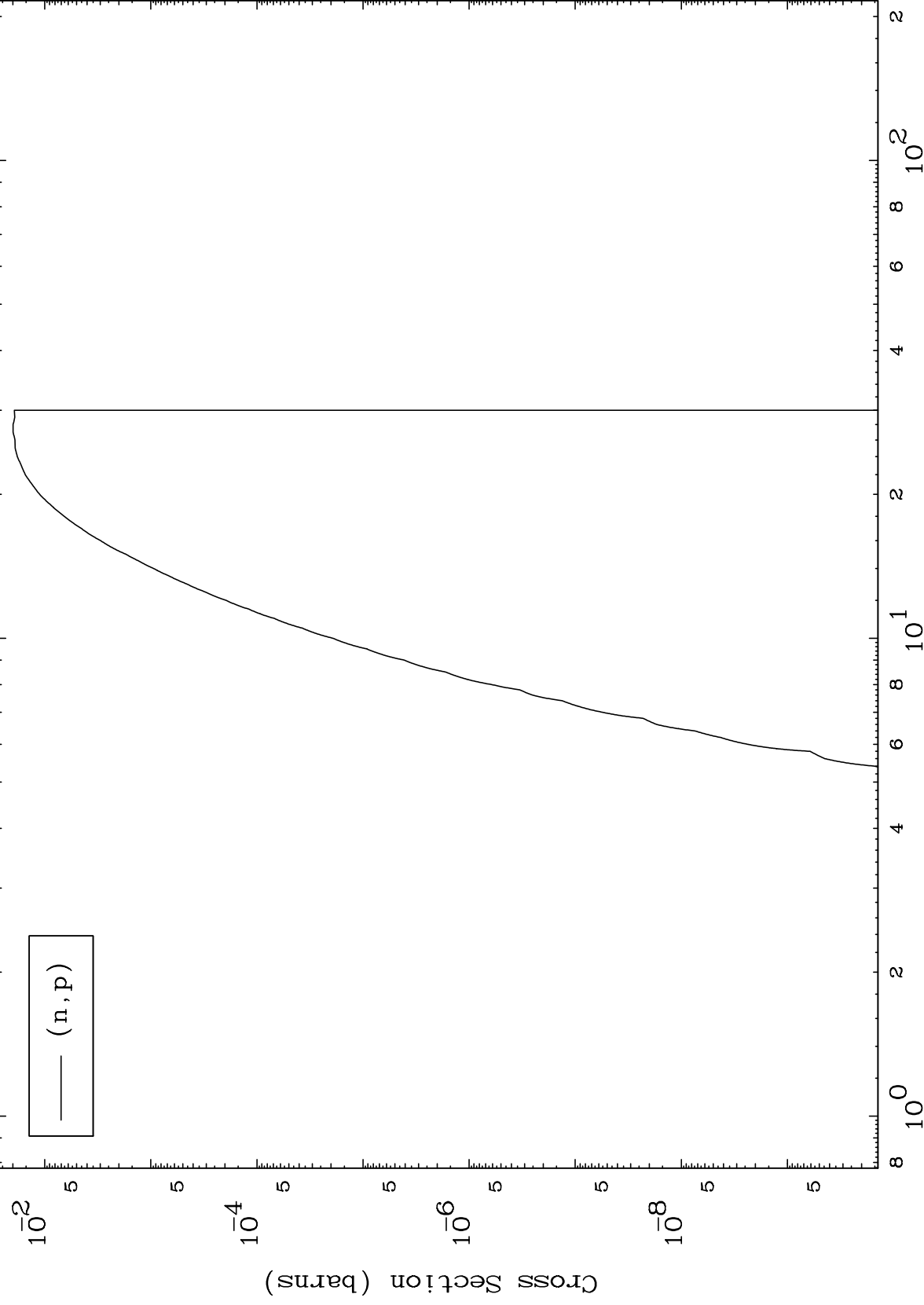
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,p) Levels
293 Kelvin Cross Sections

96-Cm-247



11

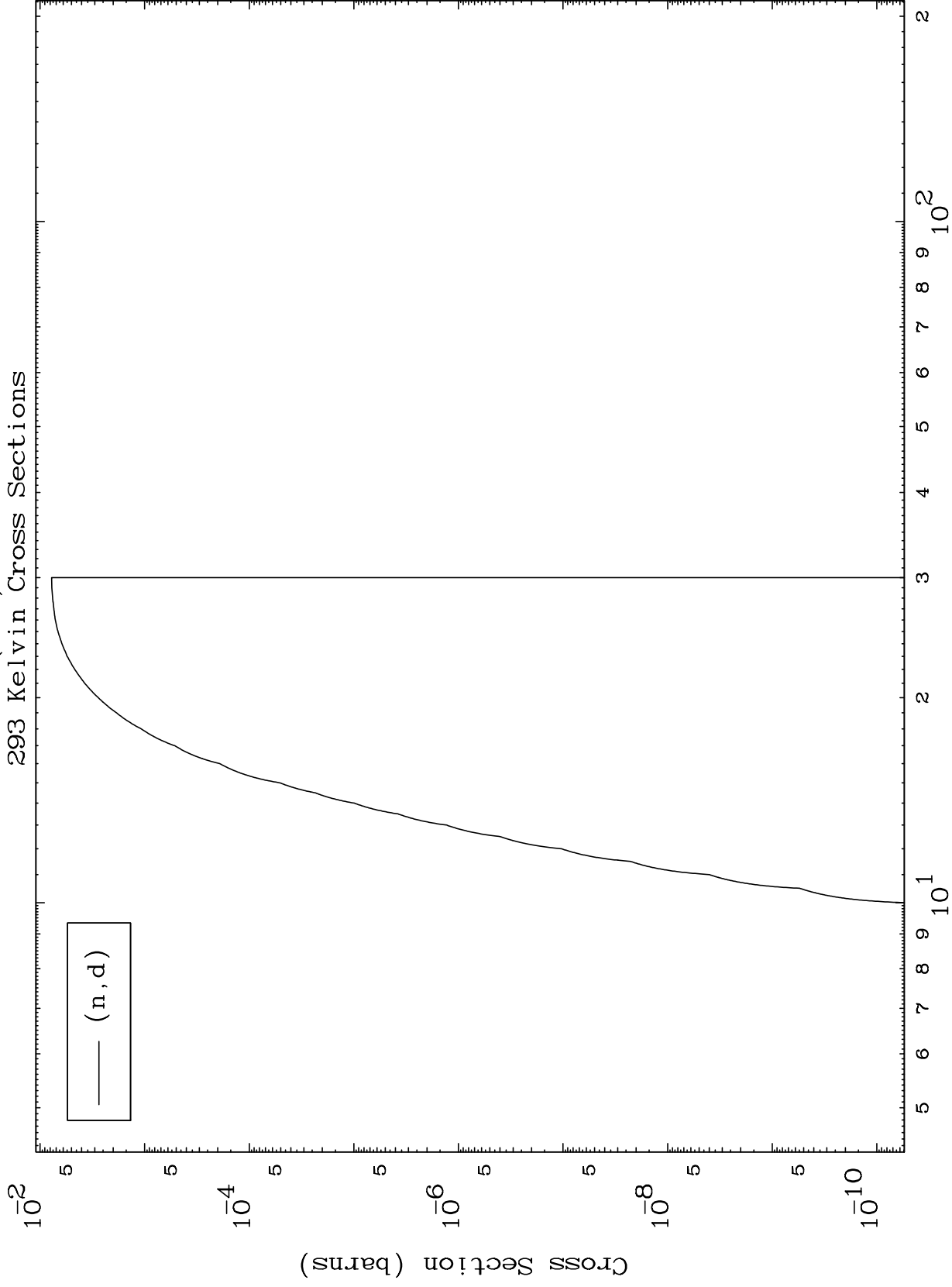
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,d) Levels
293 Kelvin Cross Sections

96-Cm-247



12

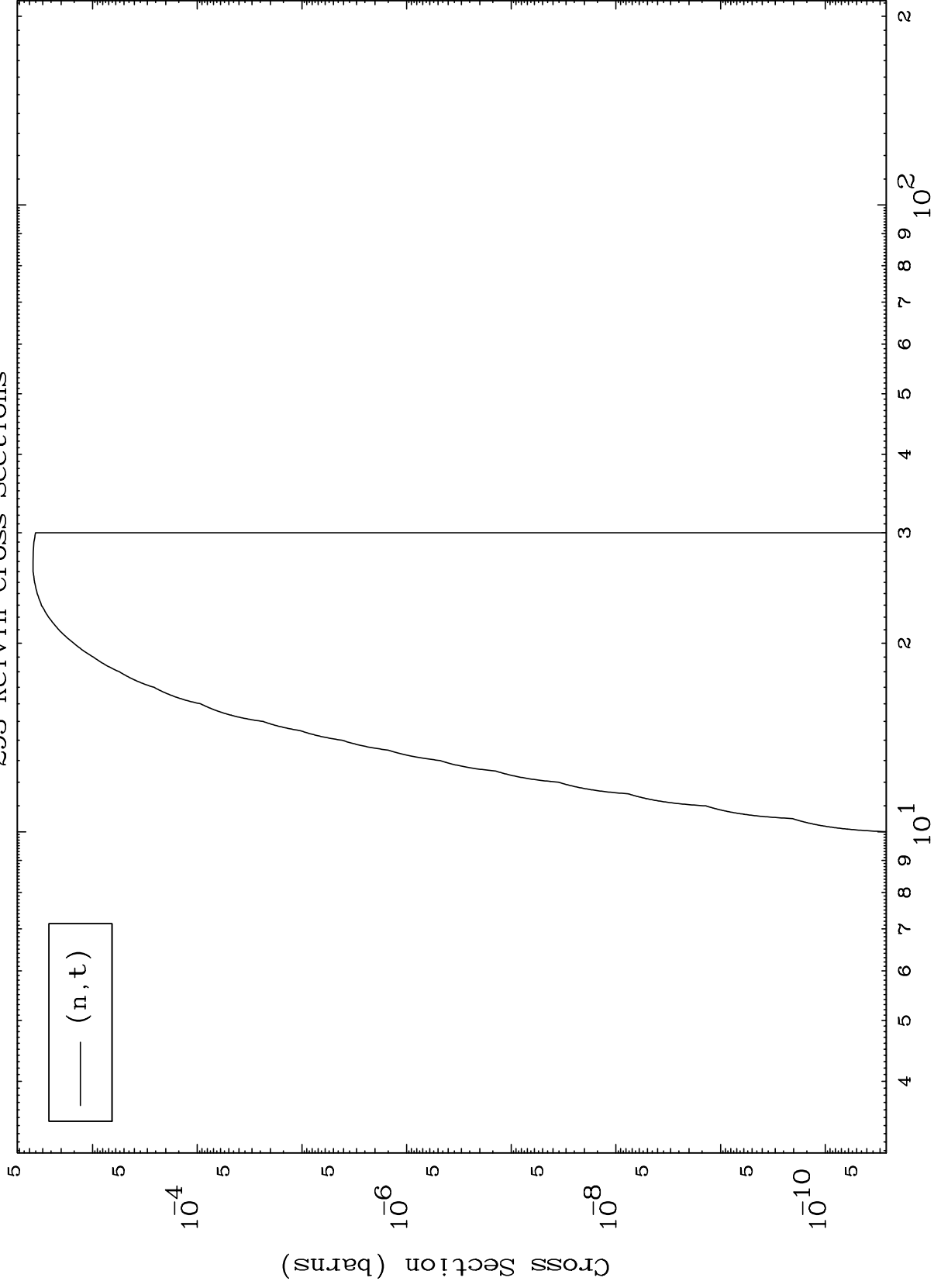
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,t) Levels
293 Kelvin Cross Sections

96-Cm-247



13

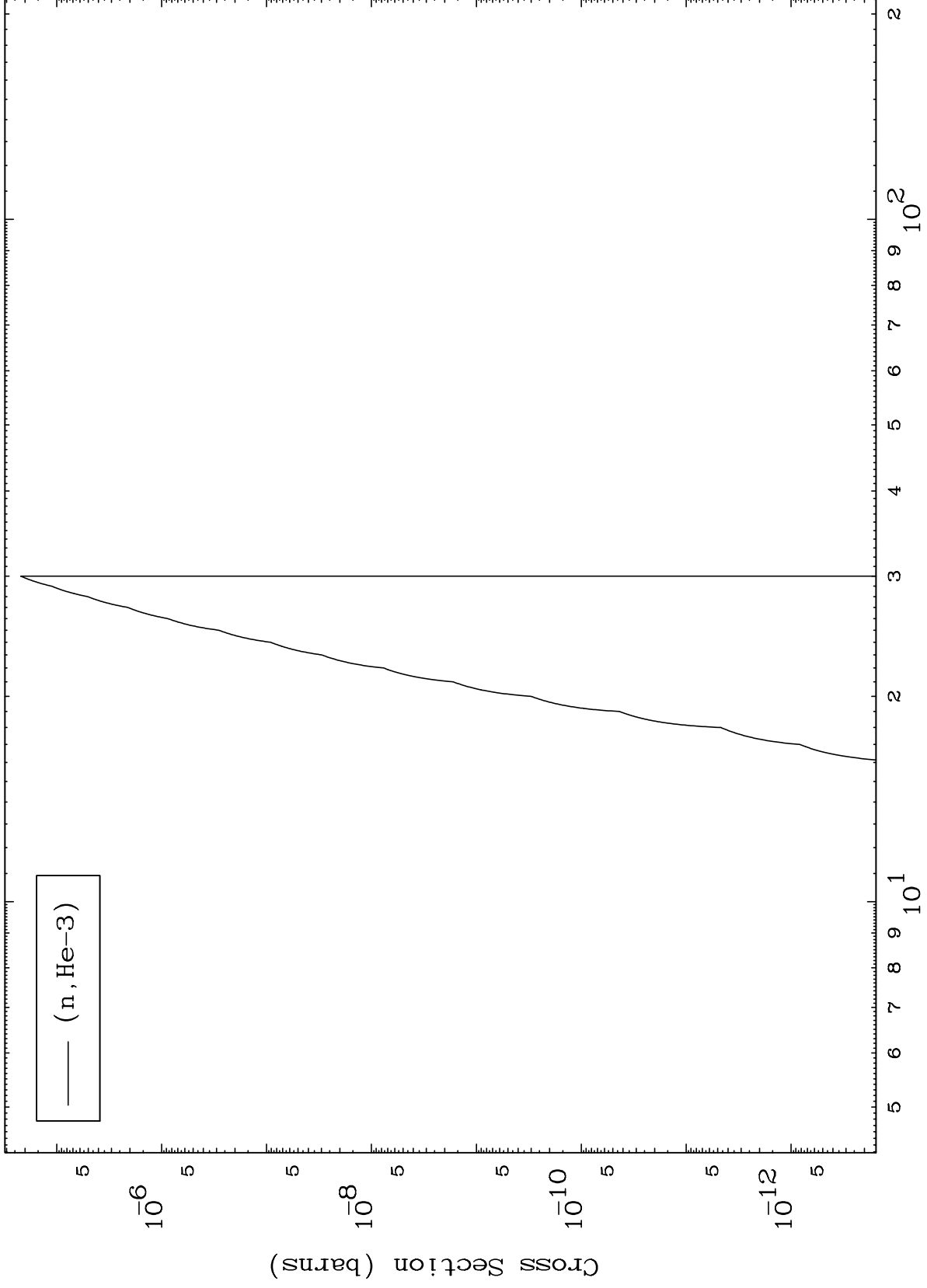
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,He3) Levels
293 Kelvin Cross Sections

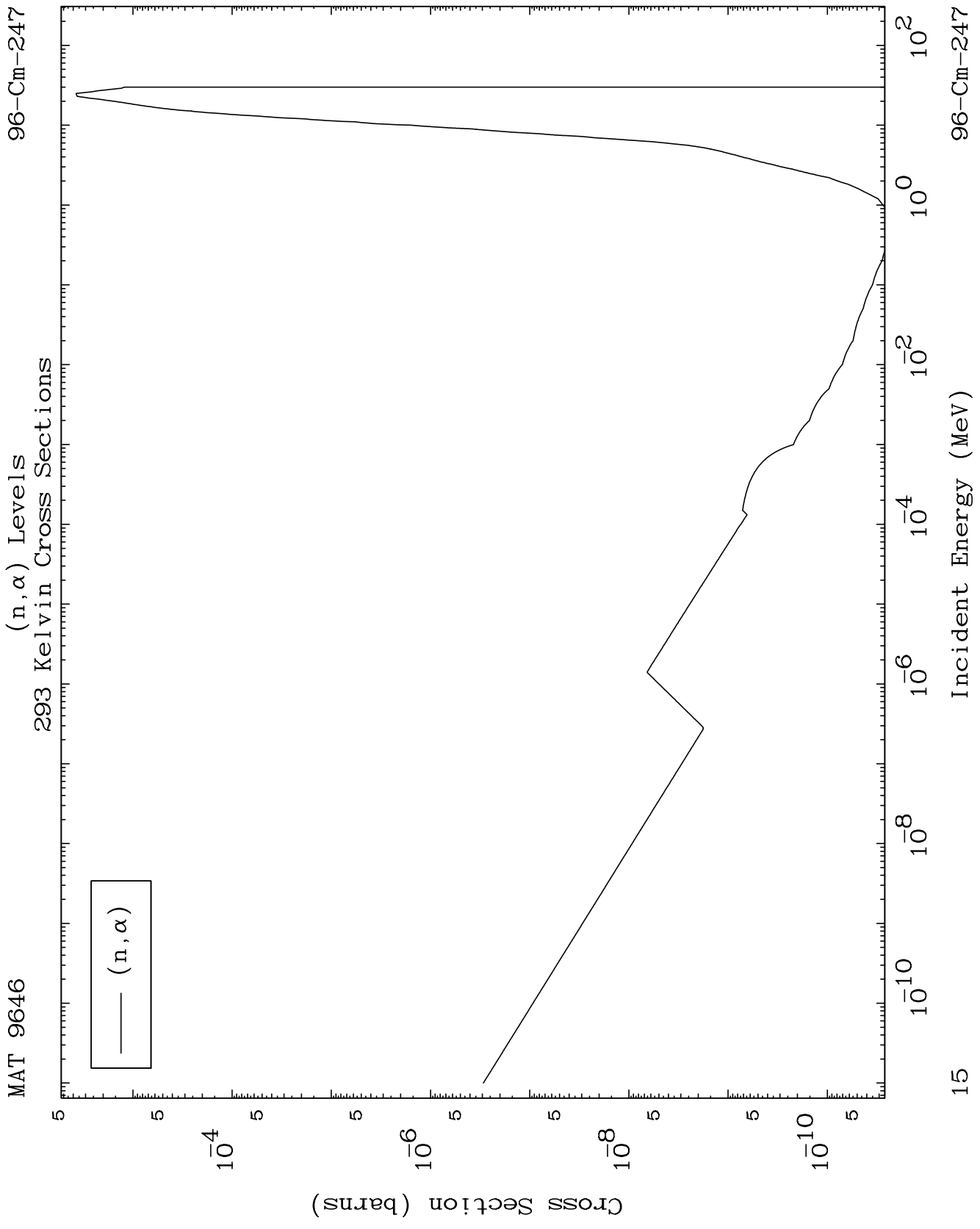
96-Cm-247



14

Incident Energy (MeV)

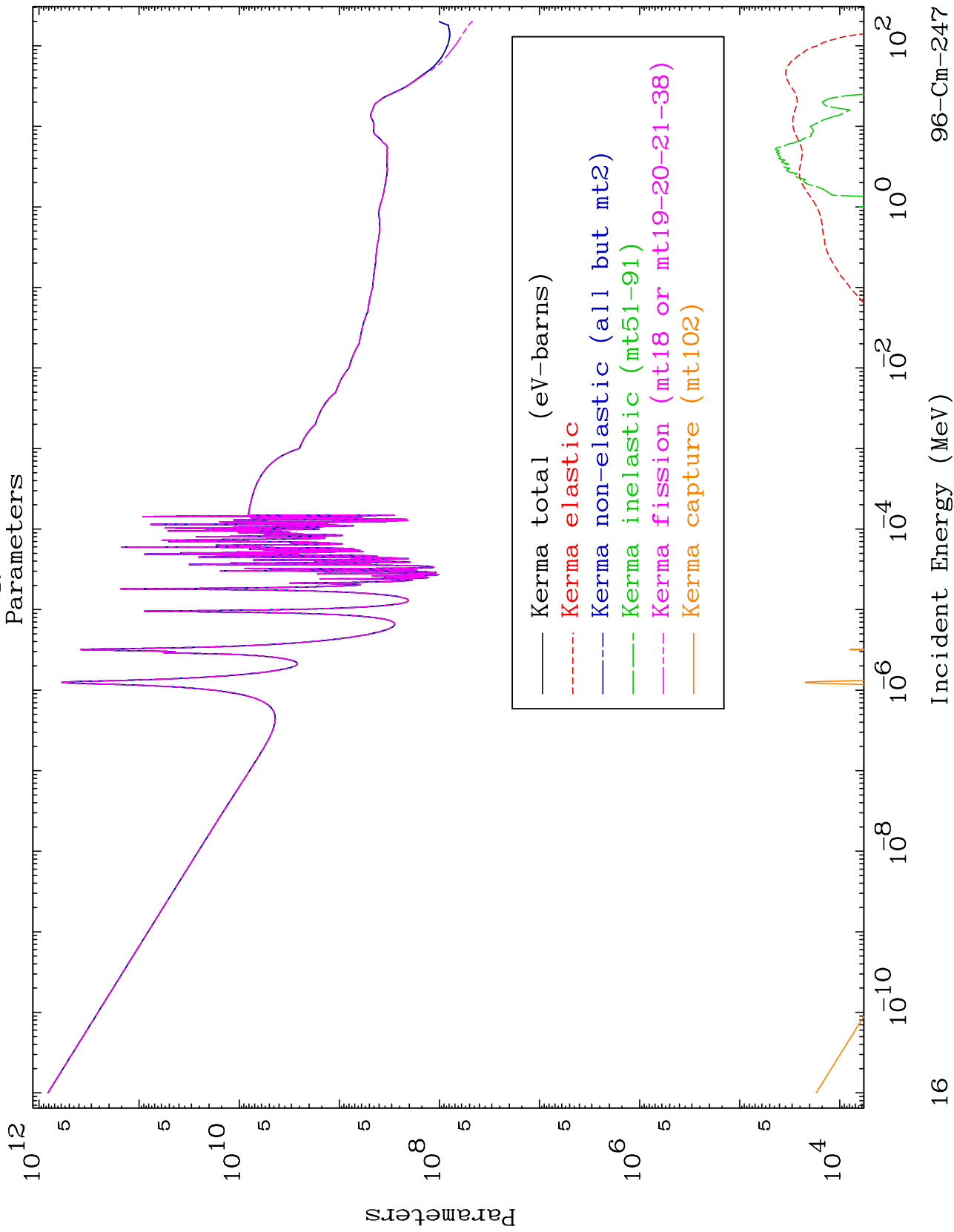
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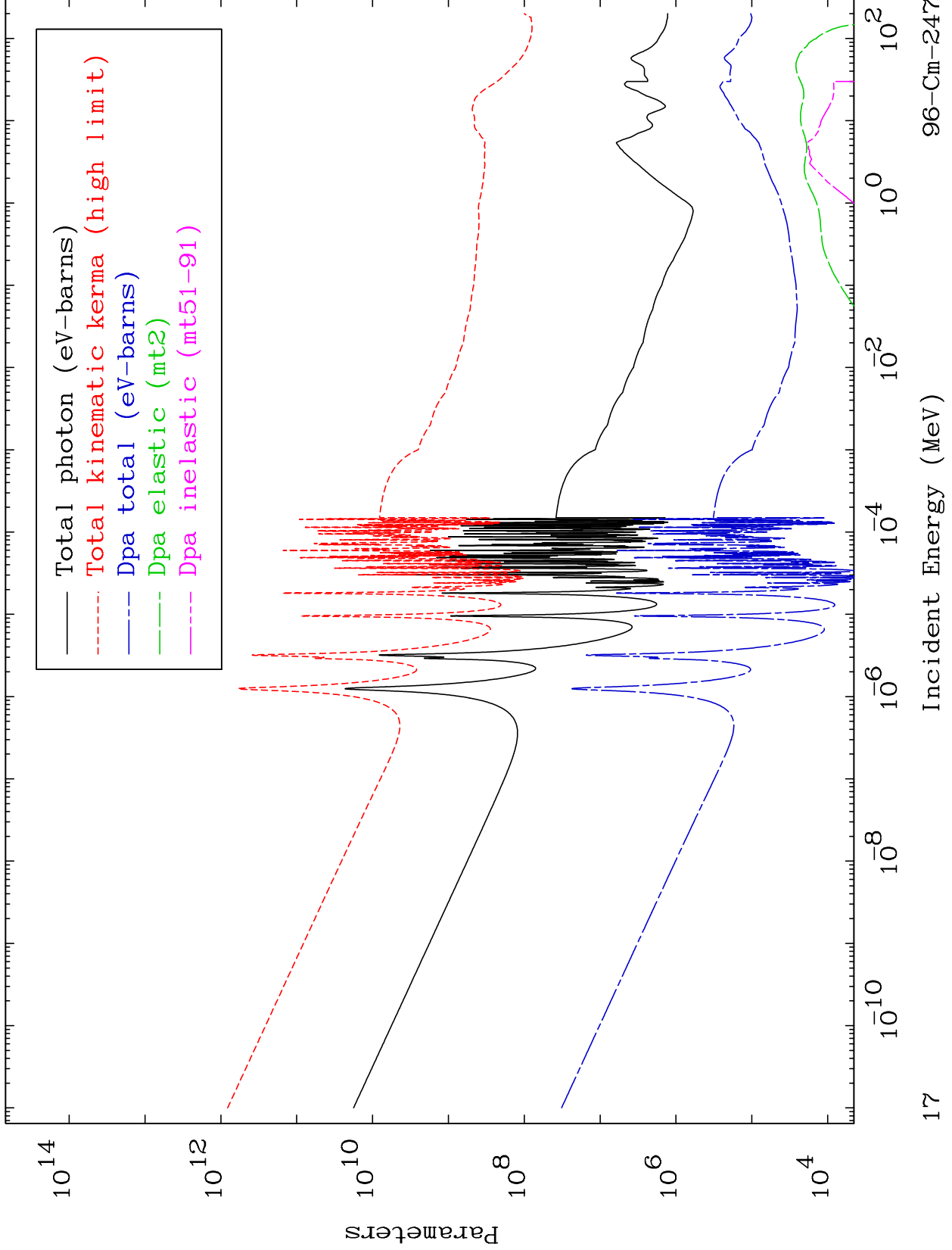


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Energy Release
Parameters

96-Cm-247

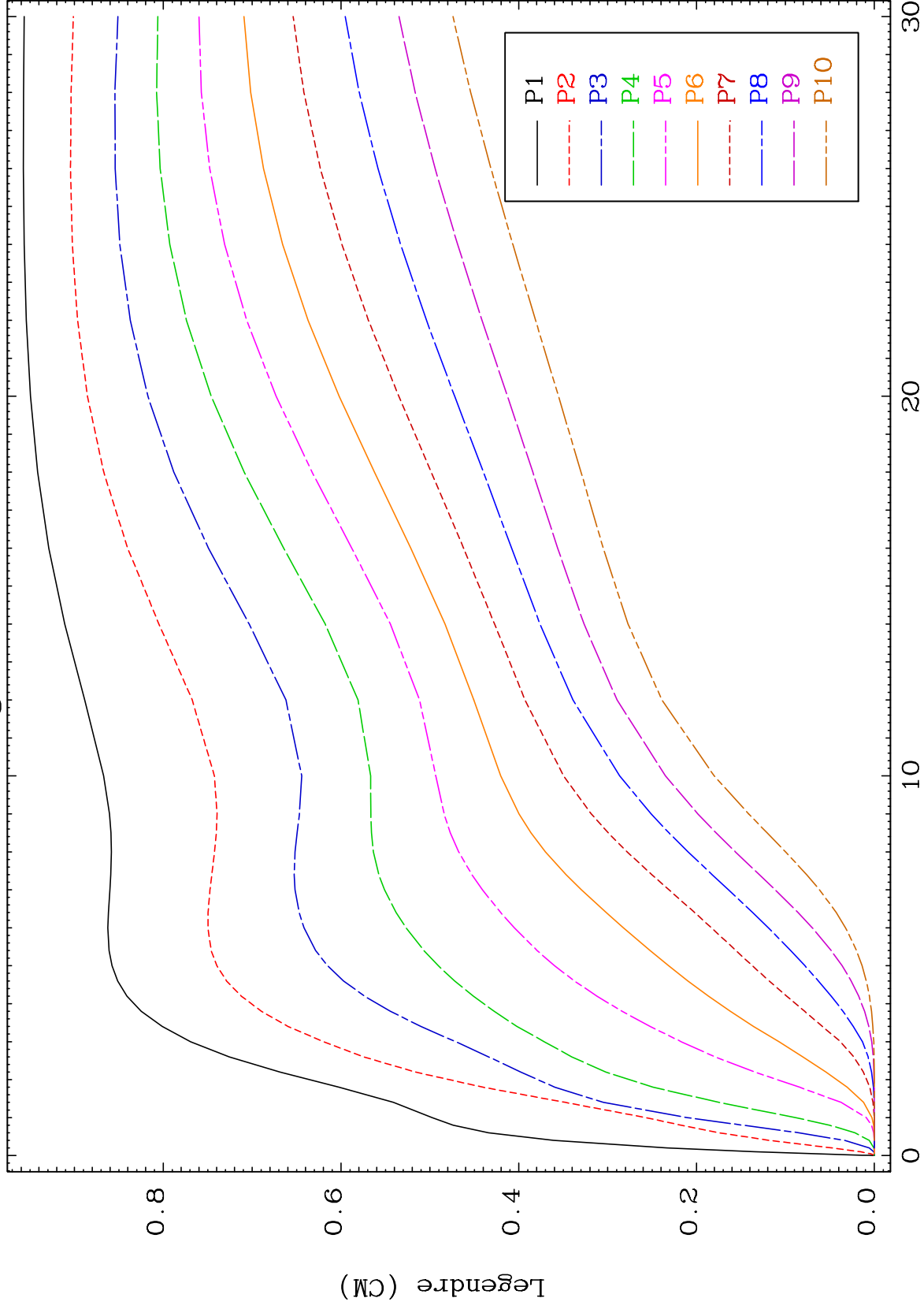




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96-Cm-247

Elastic Legendre Coefficients



18

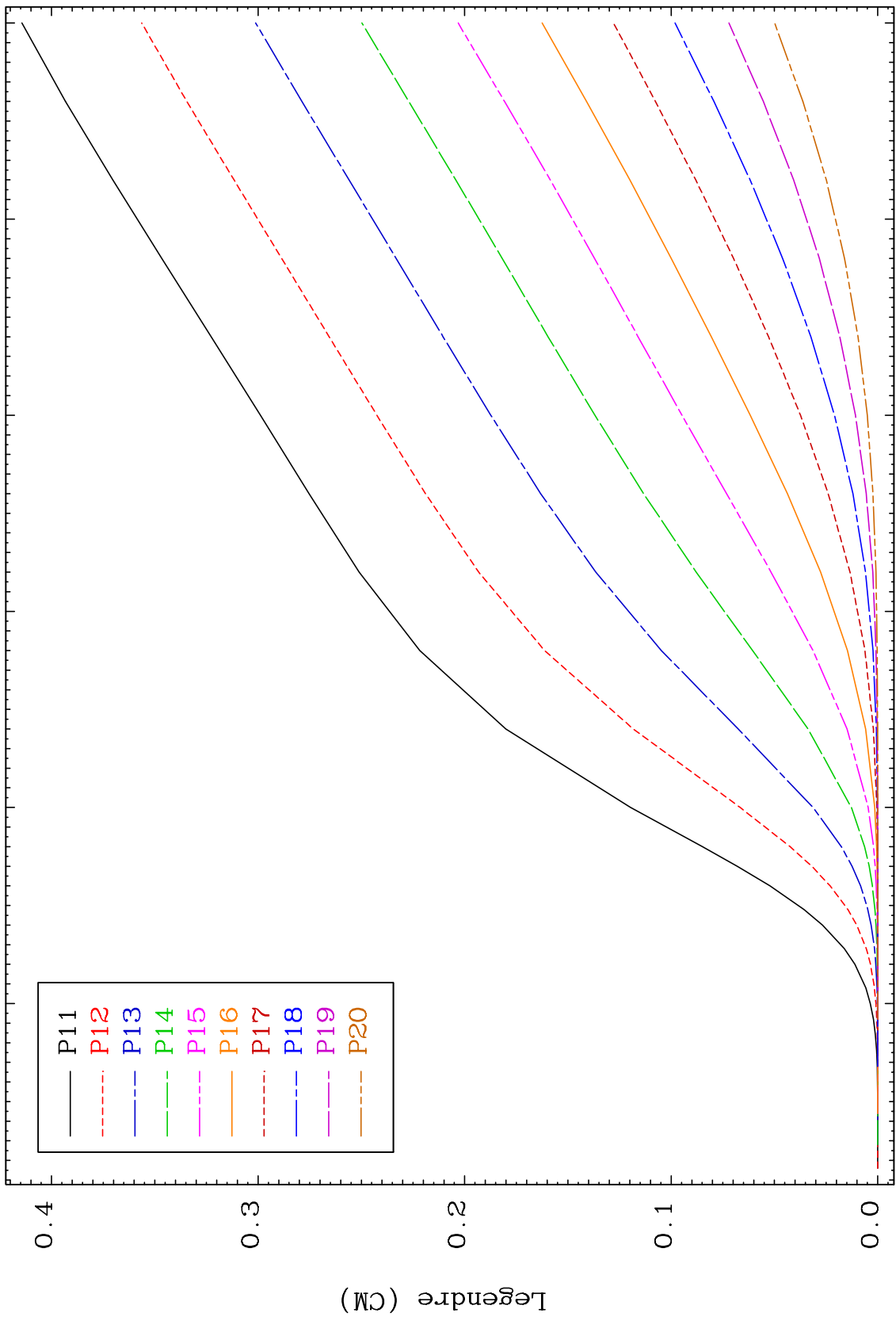
Incident Energy (MeV)

96-Cm-247

MAT 9646

Elastic Legendre Coefficients

96-Cm-247



19

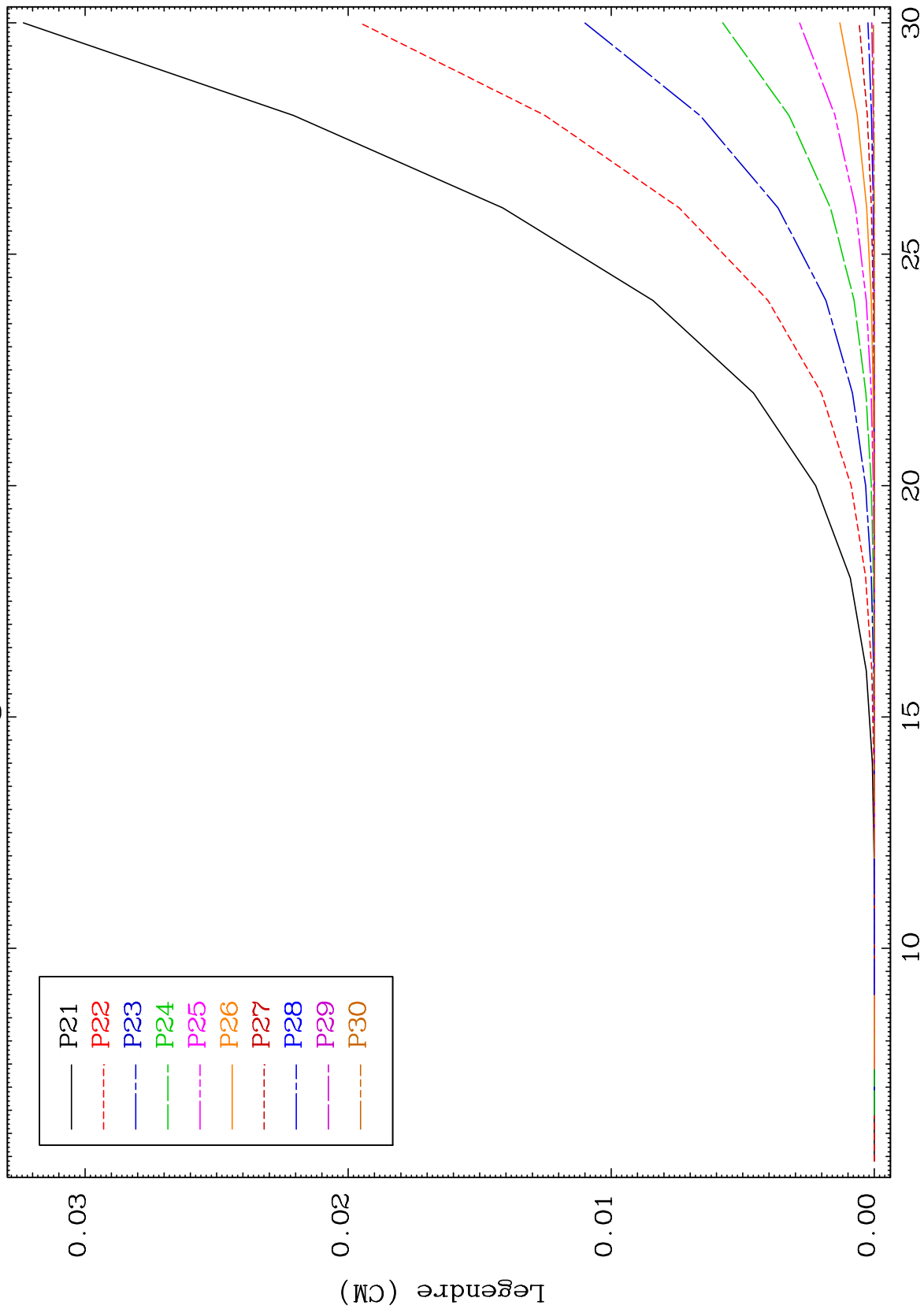
Incident Energy (MeV)

96-Cm-247

MAT 9646

Elastic Legendre Coefficients

96-Cm-247



96-Cm-247

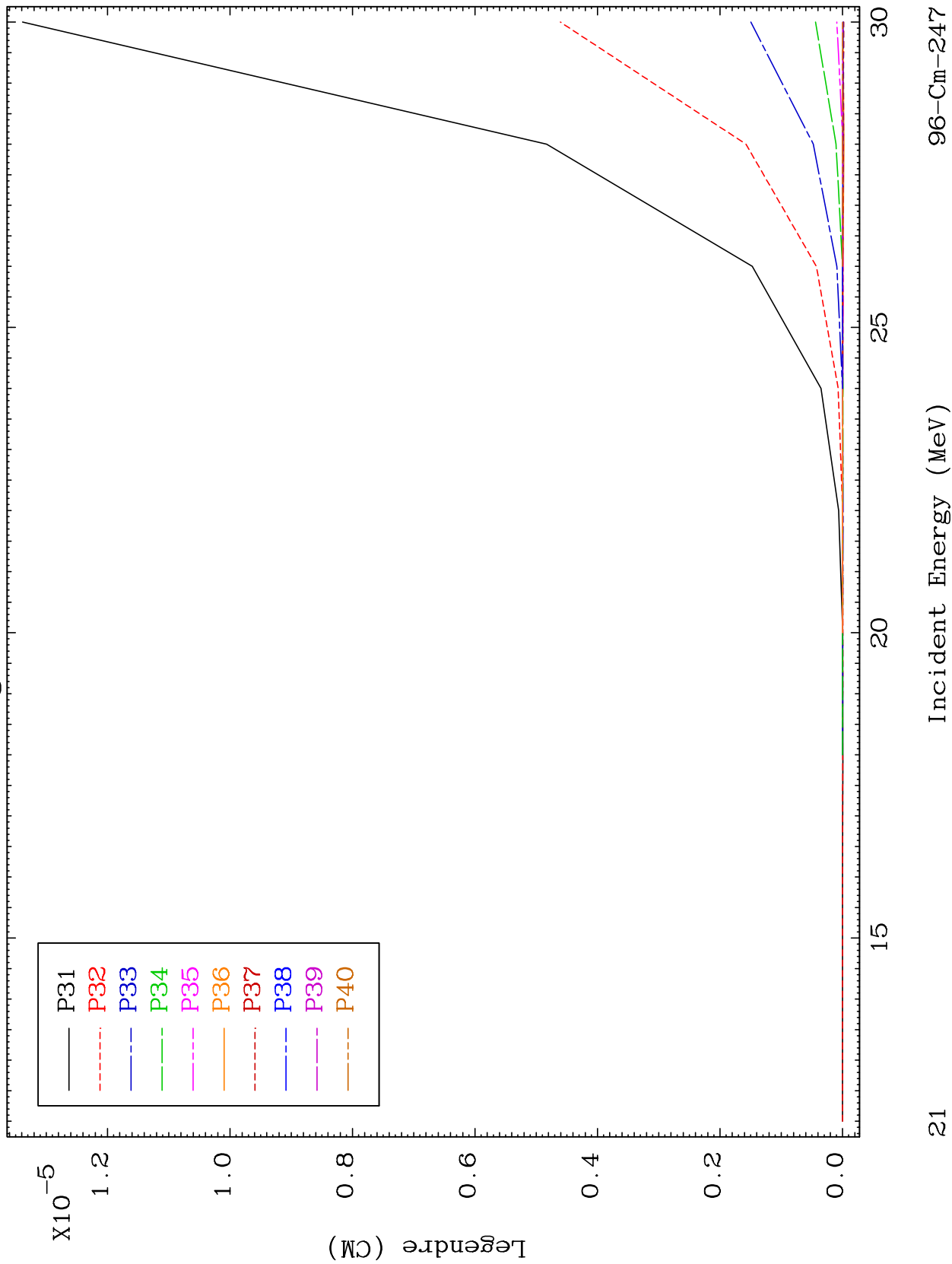
Incident Energy (MeV)

20

MAT 9646

Elastic Legendre Coefficients

96-Cm-247



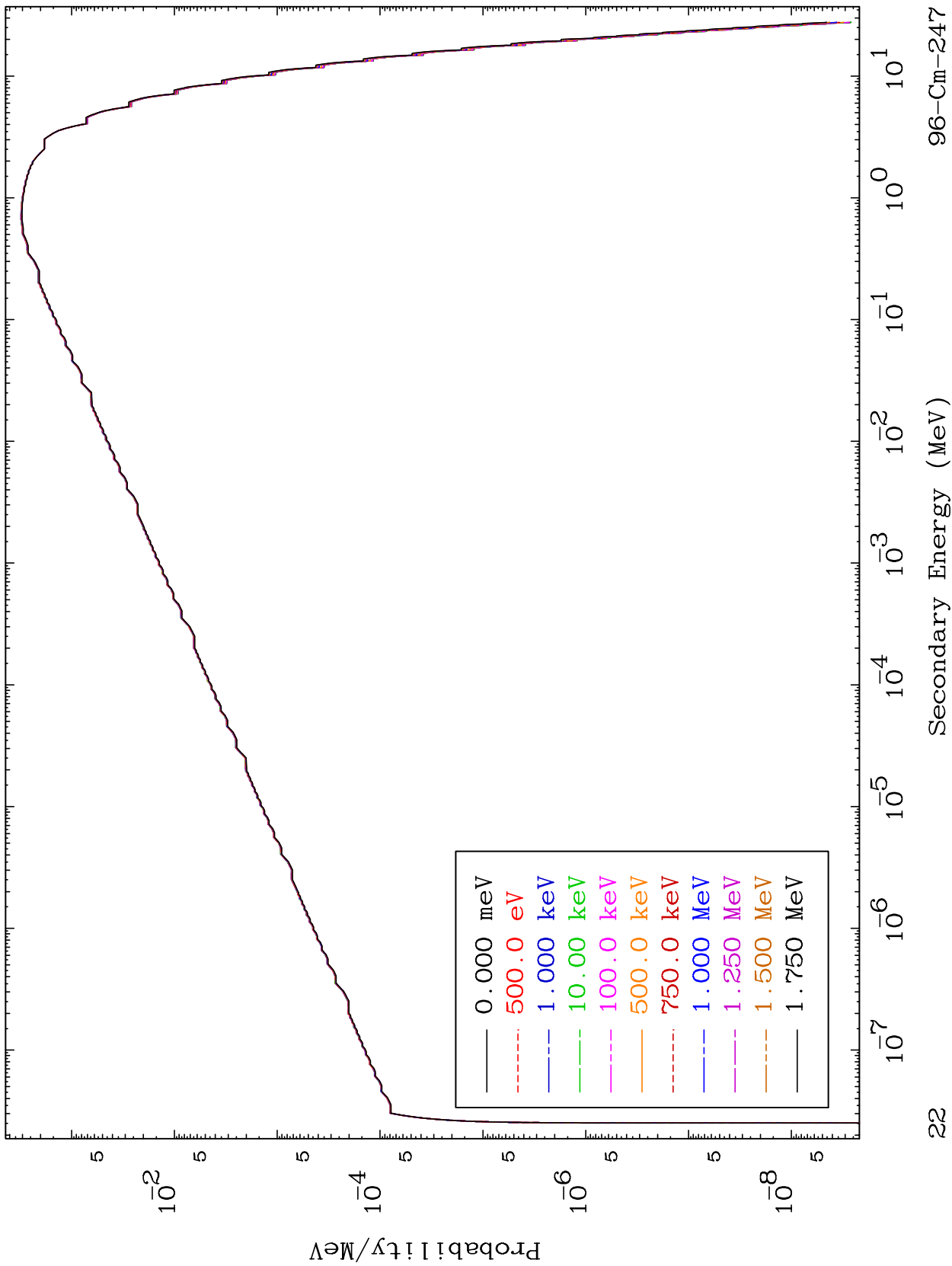
21

96-Cm-247

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Fission Energy Distribution

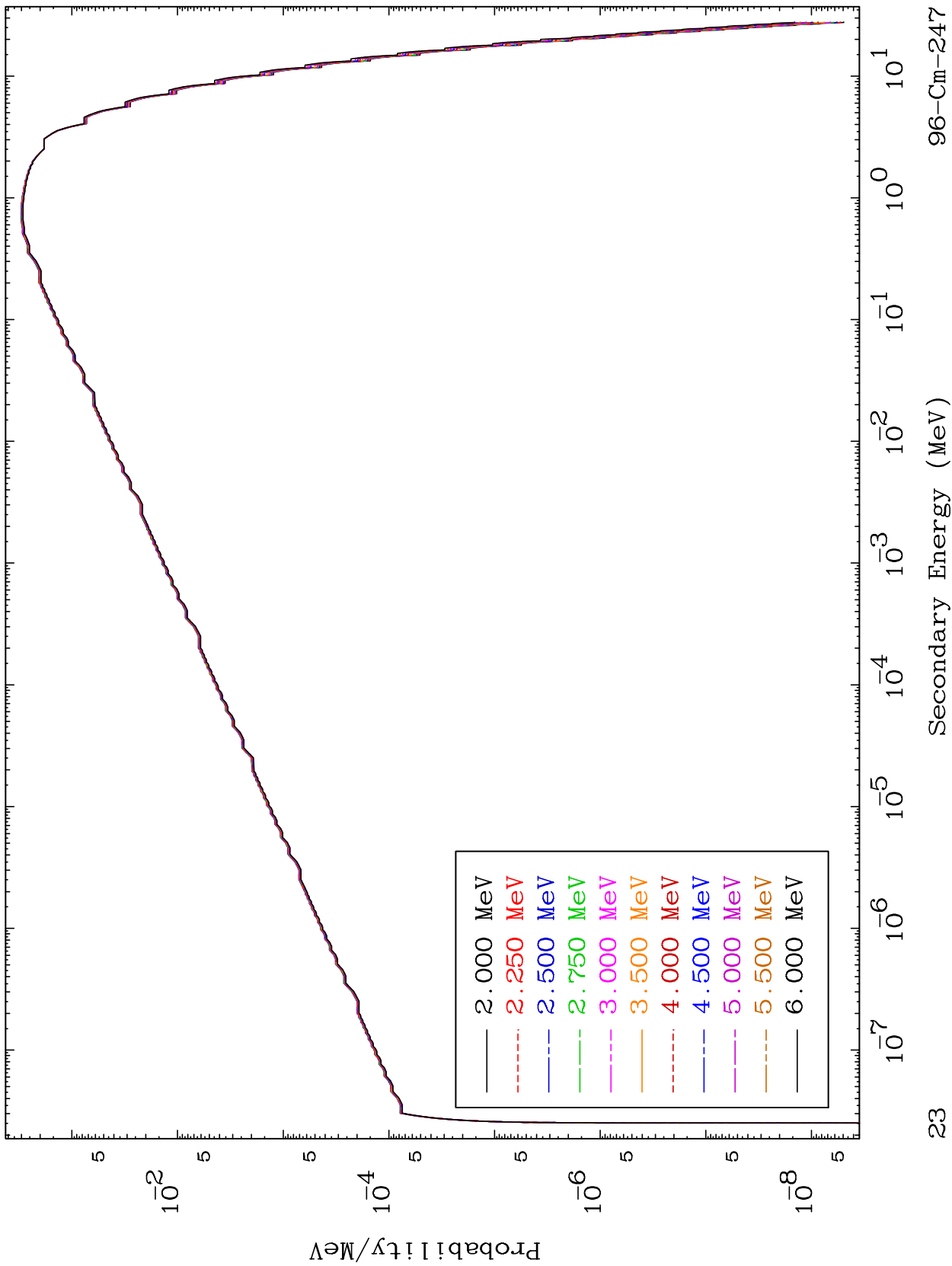
96-Cm-247



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Fission Energy Distribution

96-Cm-247

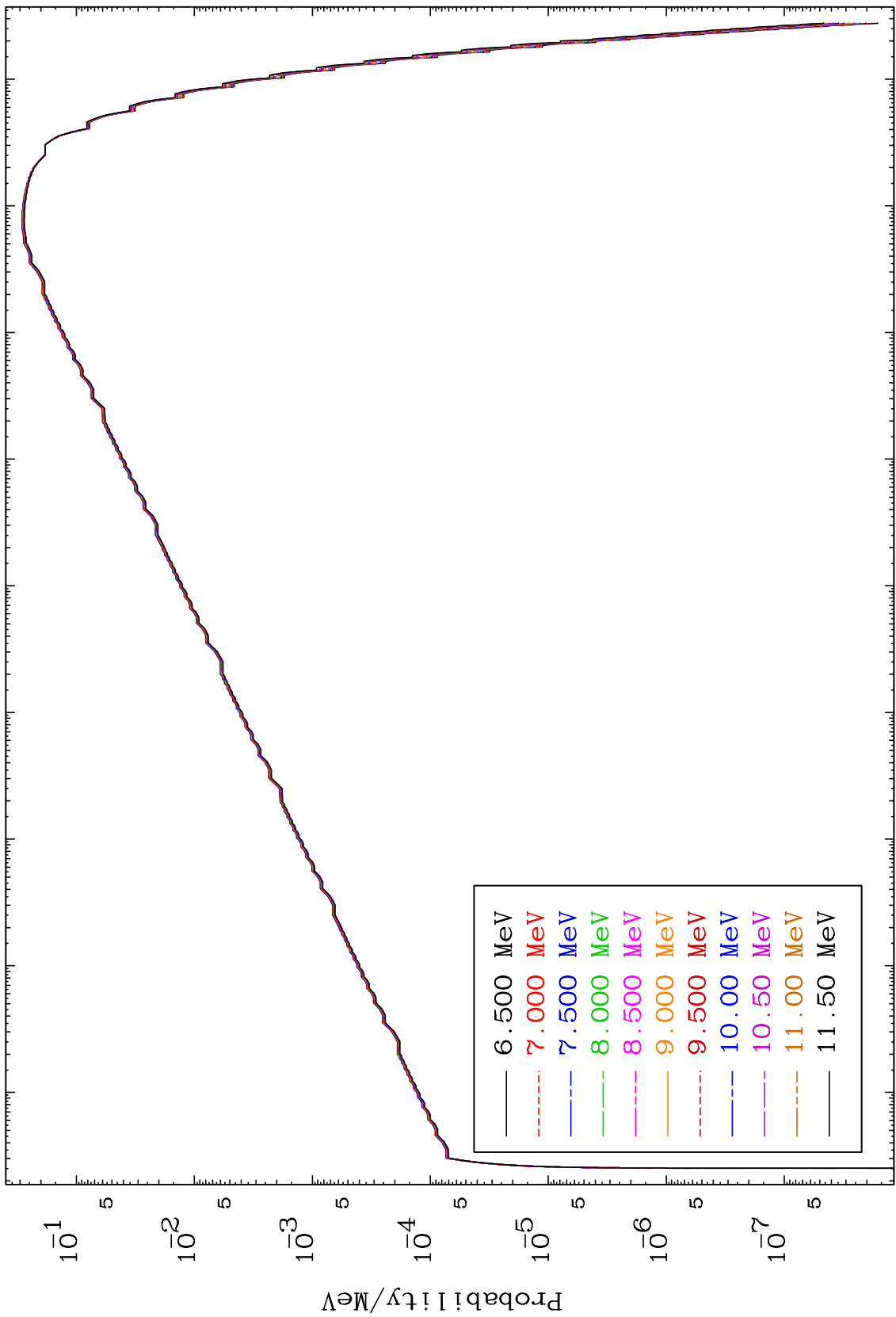


96-Cm-247

MAT 9646

Fission Energy Distribution

96-Cm-247

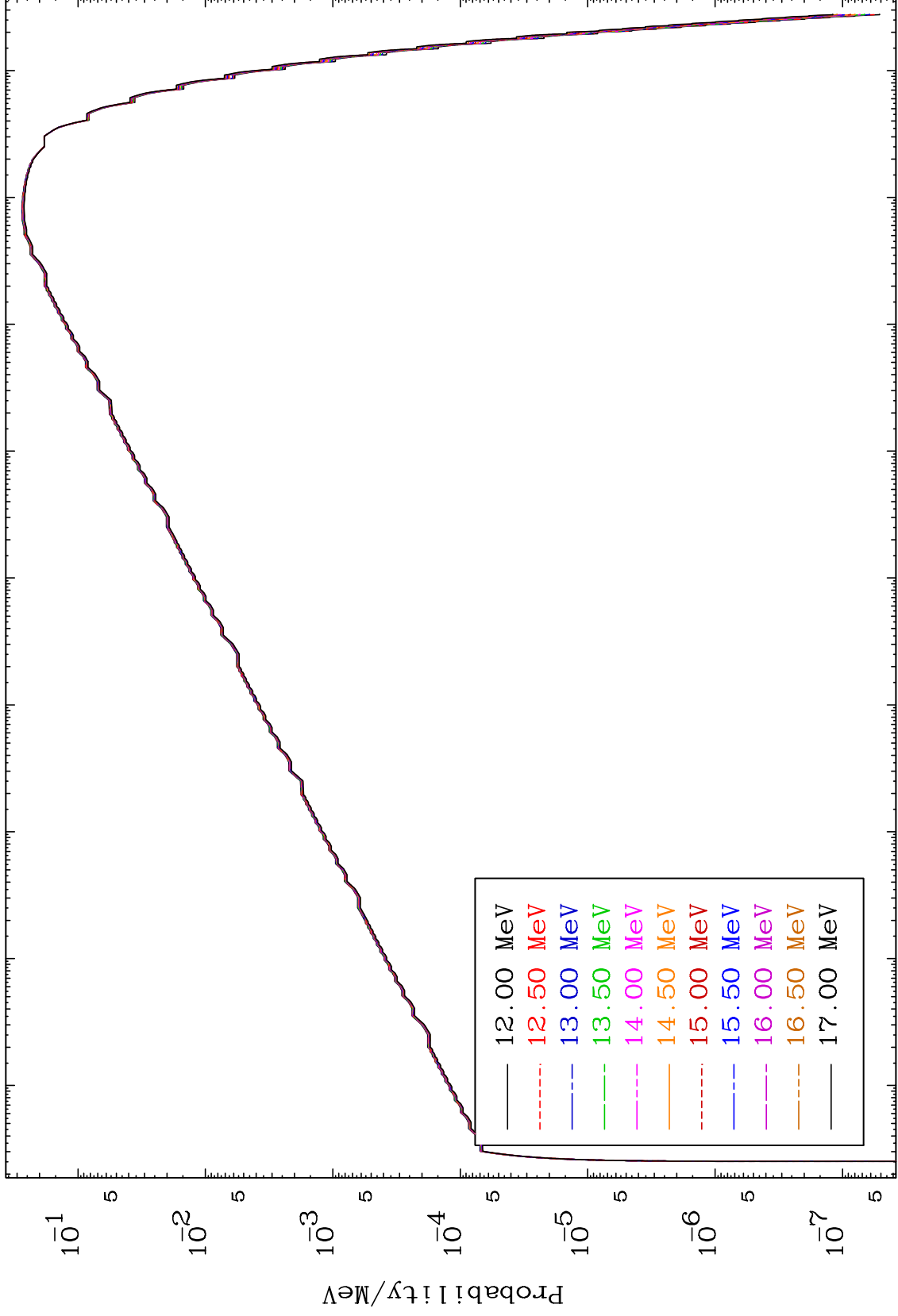


96-Cm-247

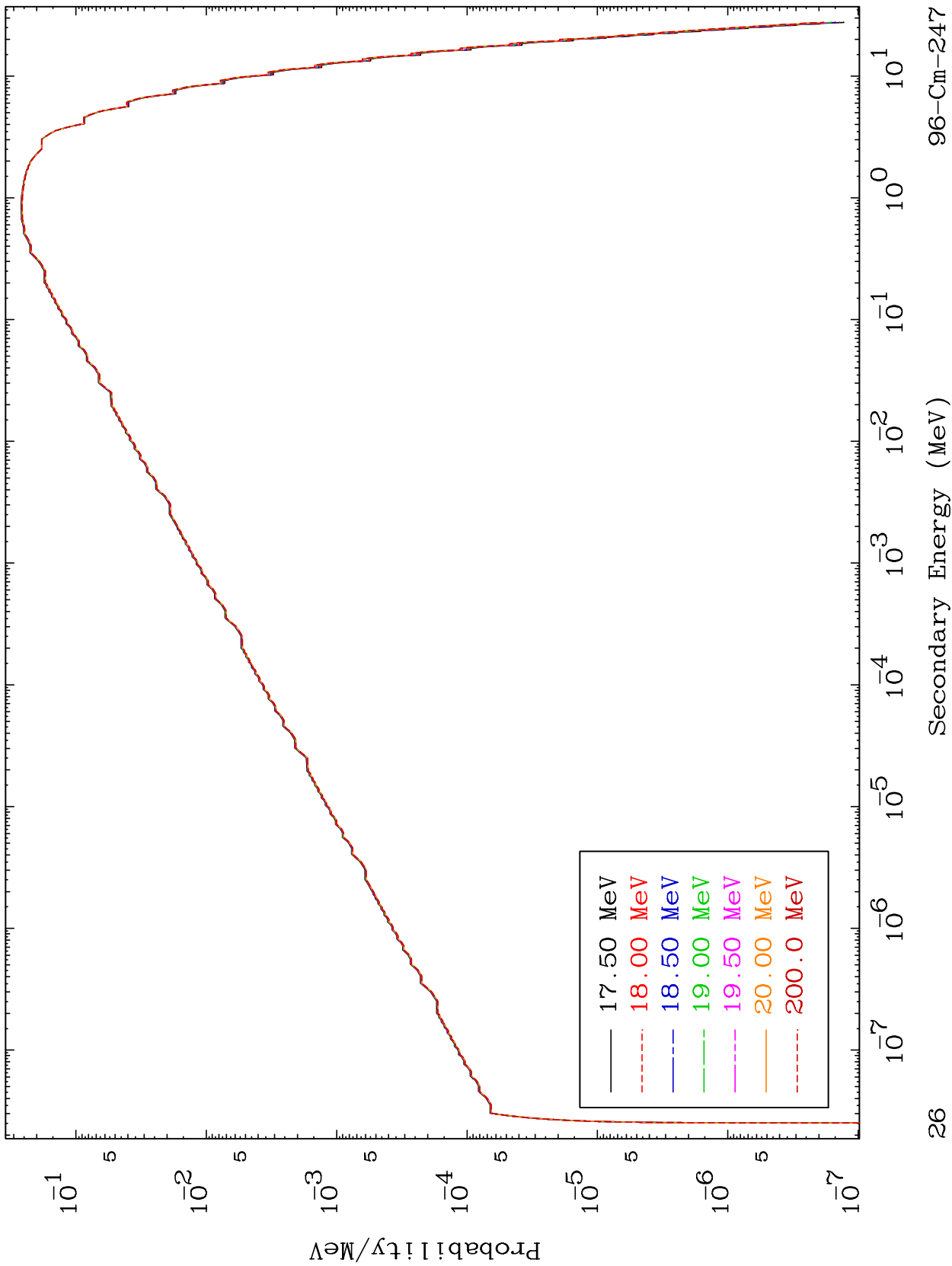
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Fission Energy Distribution

96-Cm-247



Fission
Energy Distribution

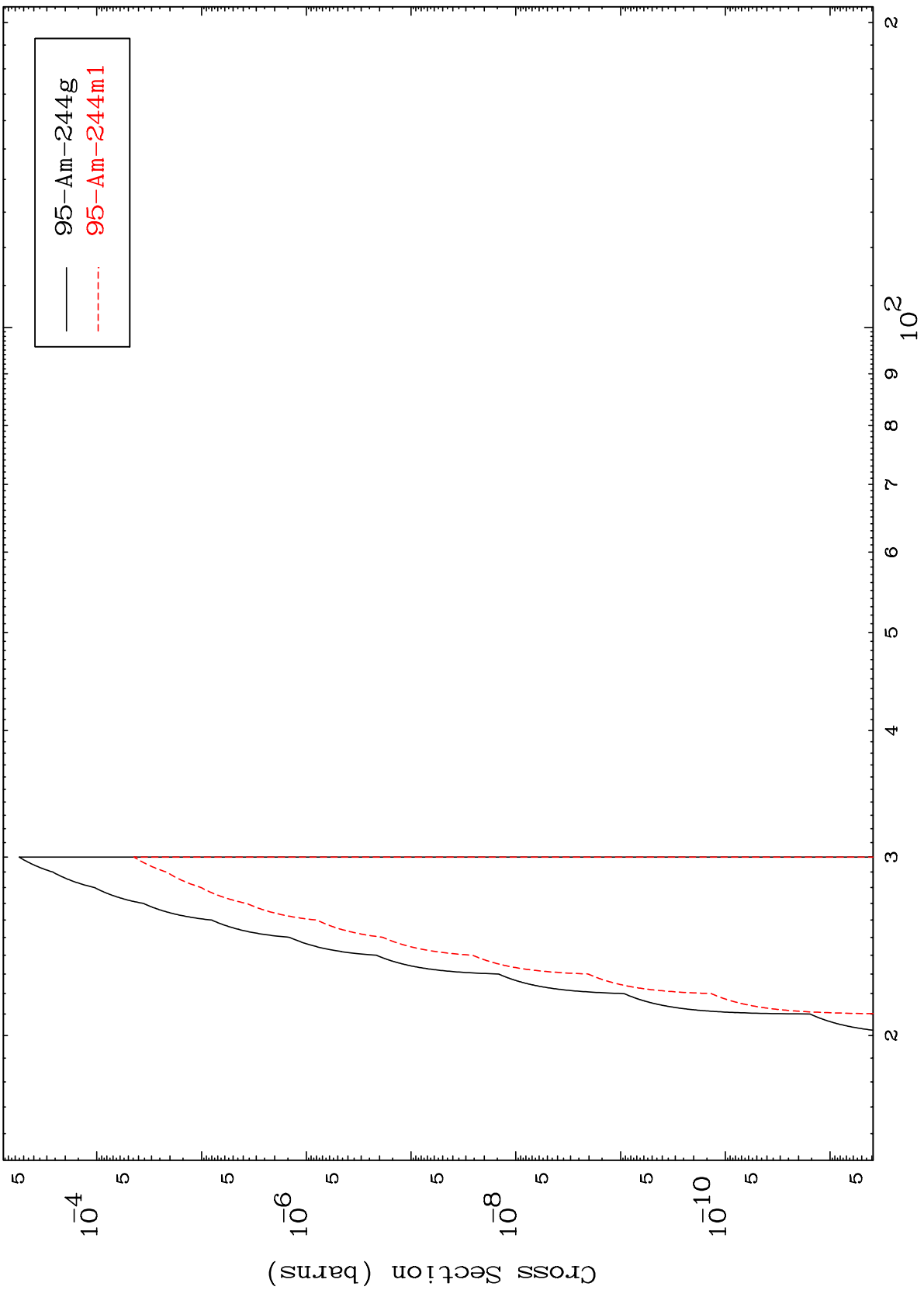


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(n,2n) d

96-Cm-247

Radionuclide Production Cross Section



27

Incident Energy (MeV)

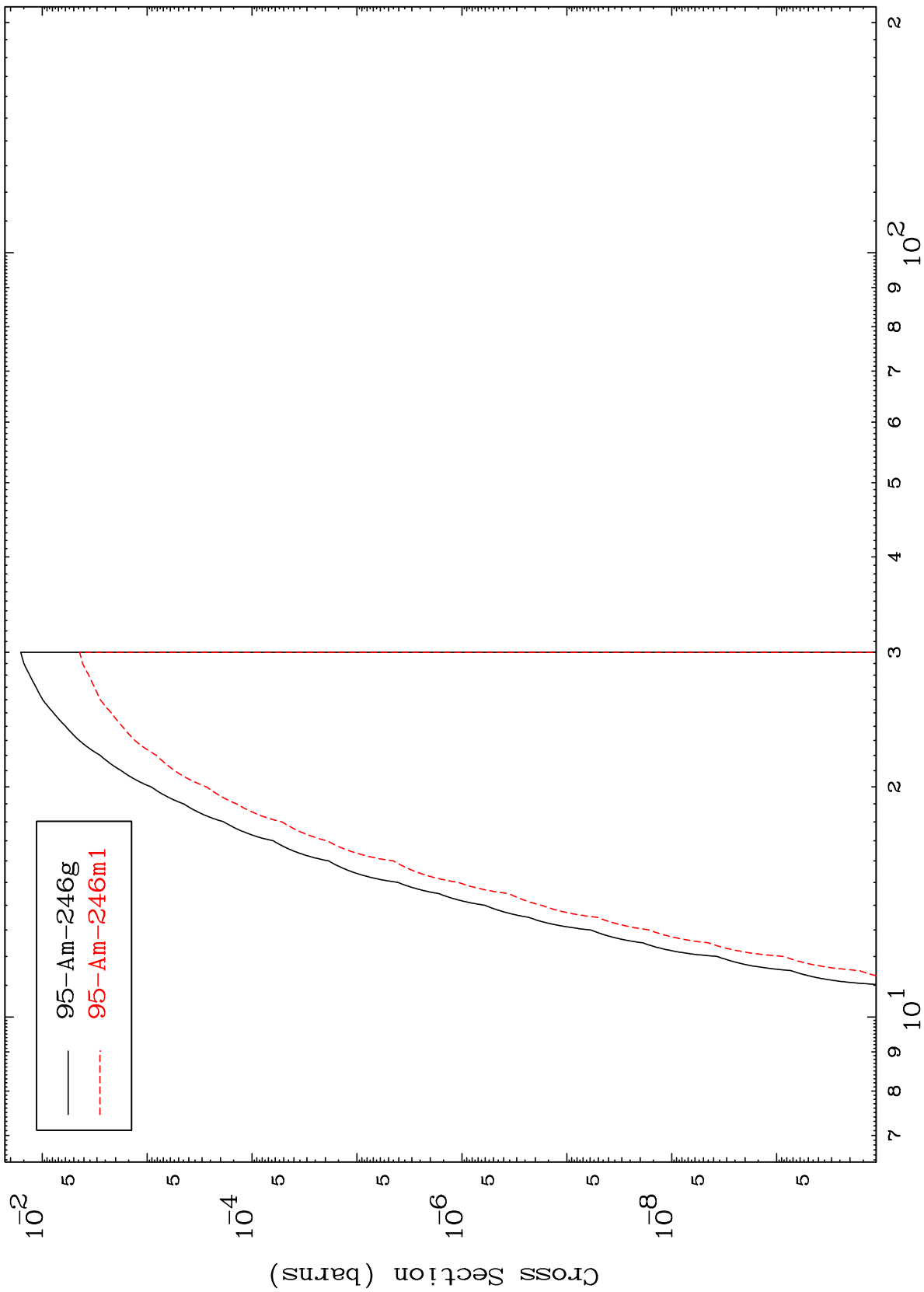
96-Cm-247

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(n,n') p

96-Cm-247

Radionuclide Production Cross Section



95-Am-246g
95-Am-246m1

28

Incident Energy (MeV)

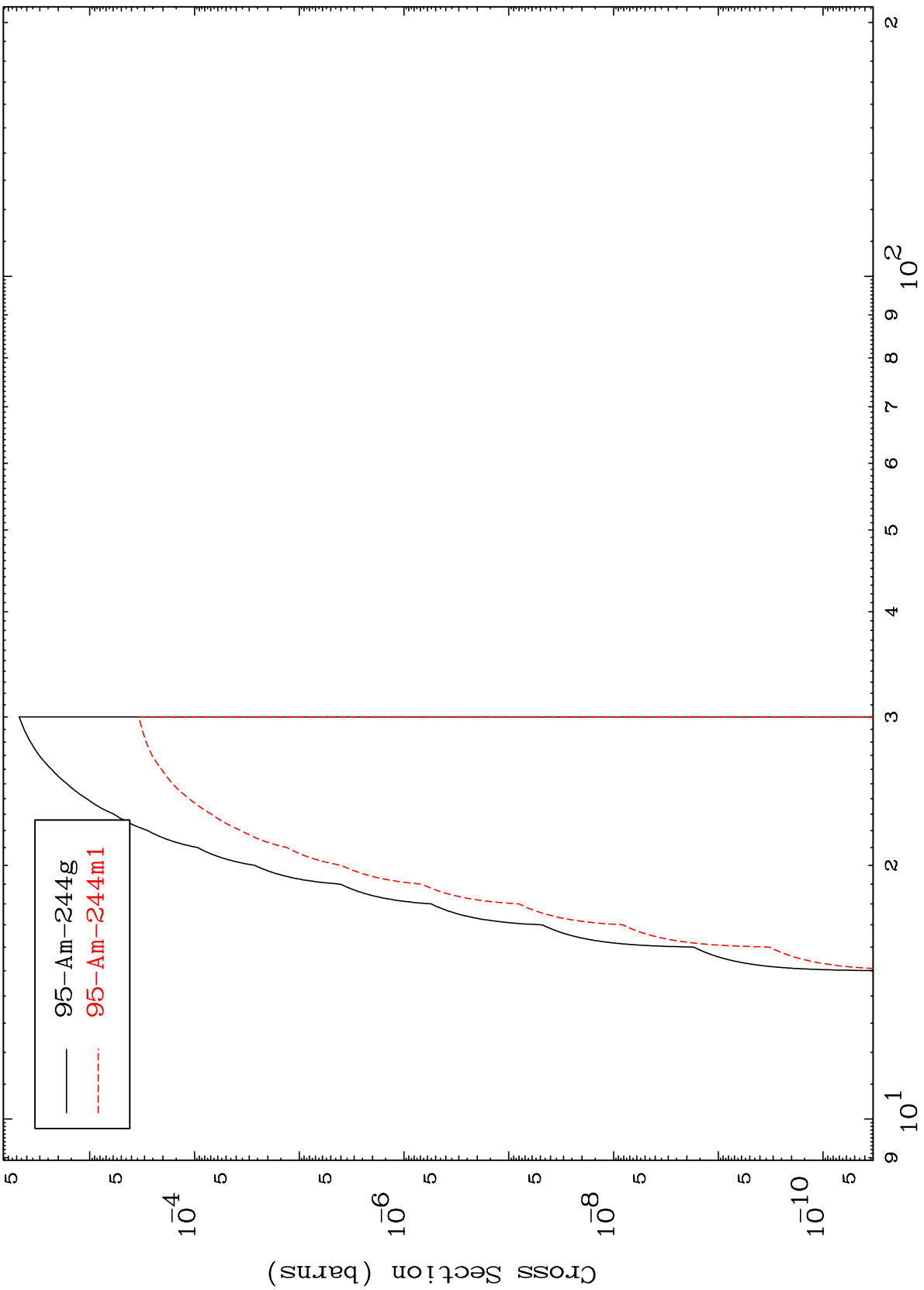
96-Cm-247

MAT 9646

(n,n') t

96-Cm-247

Radionuclide Production Cross Section



29

Incident Energy (MeV)

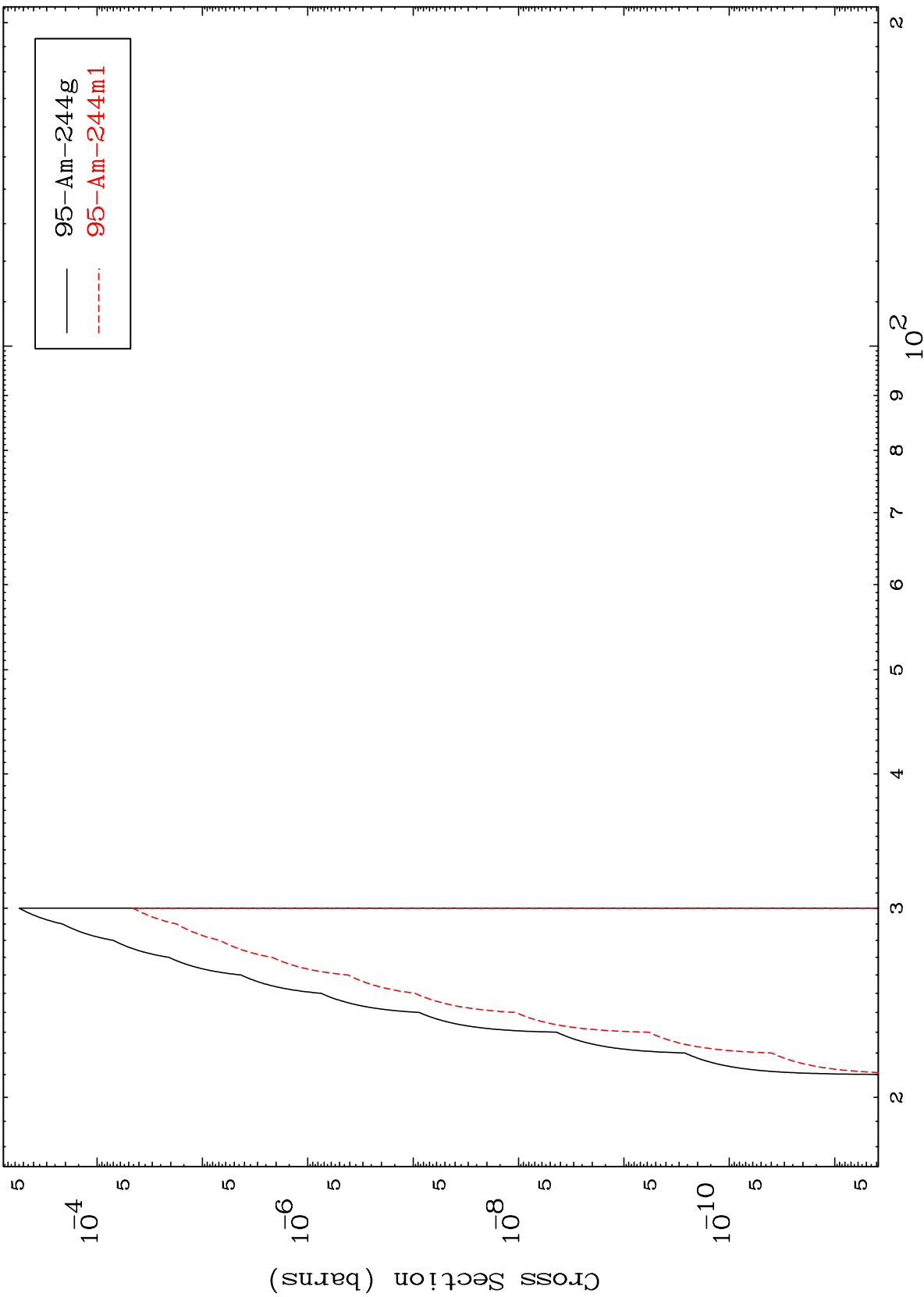
96-Cm-247

MAT 9646

(n,3n) p

96-Cm-247

Radionuclide Production Cross Section



95-Am-244g
95-Am-244m1

30

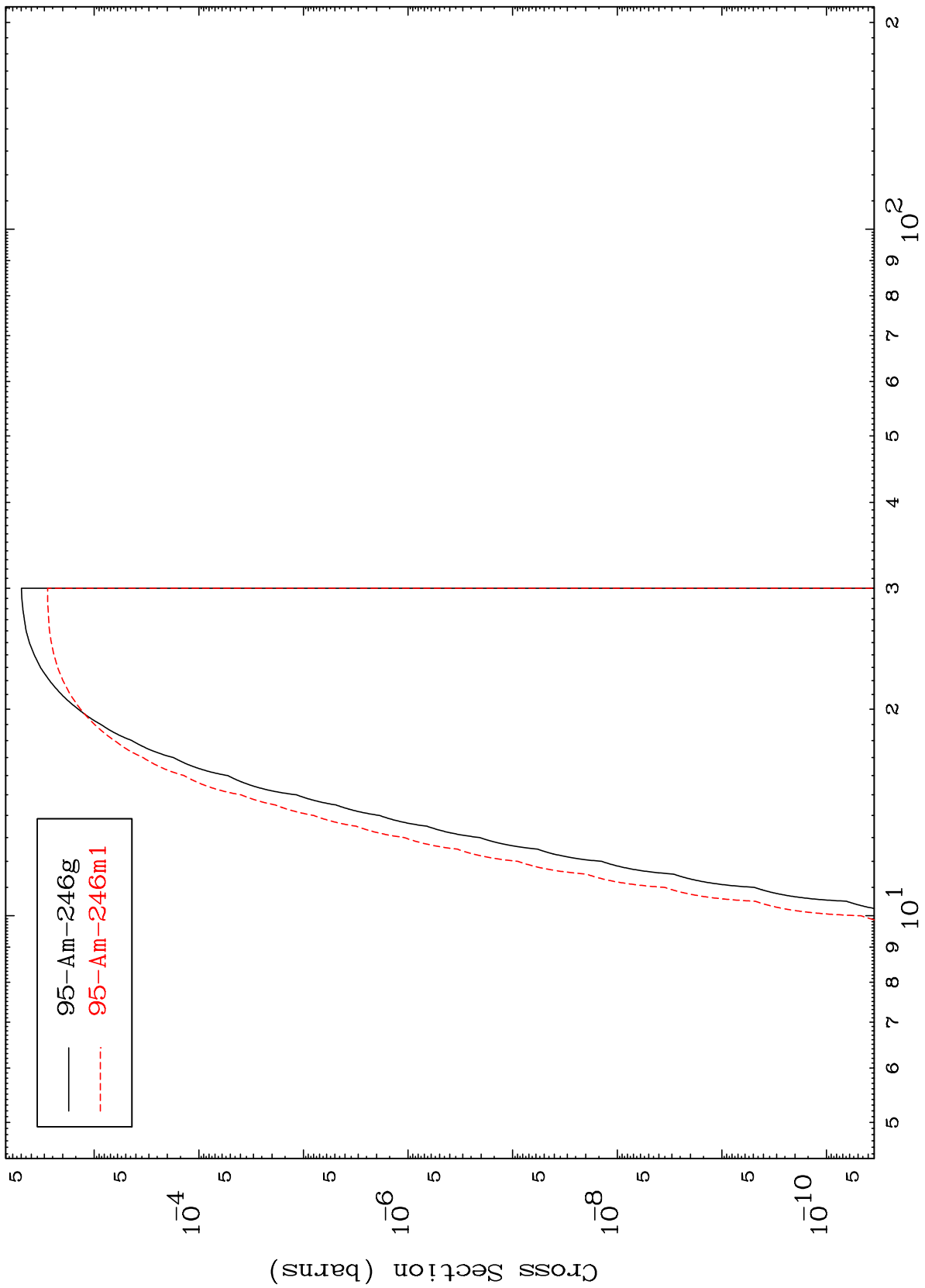
Incident Energy (MeV)

96-Cm-247

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96-Cm-247

(n,d)
Radionuclide Production Cross Section



31

Incident Energy (MeV)

96-Cm-247