

Program EVALPLOT  
(Version 2017-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

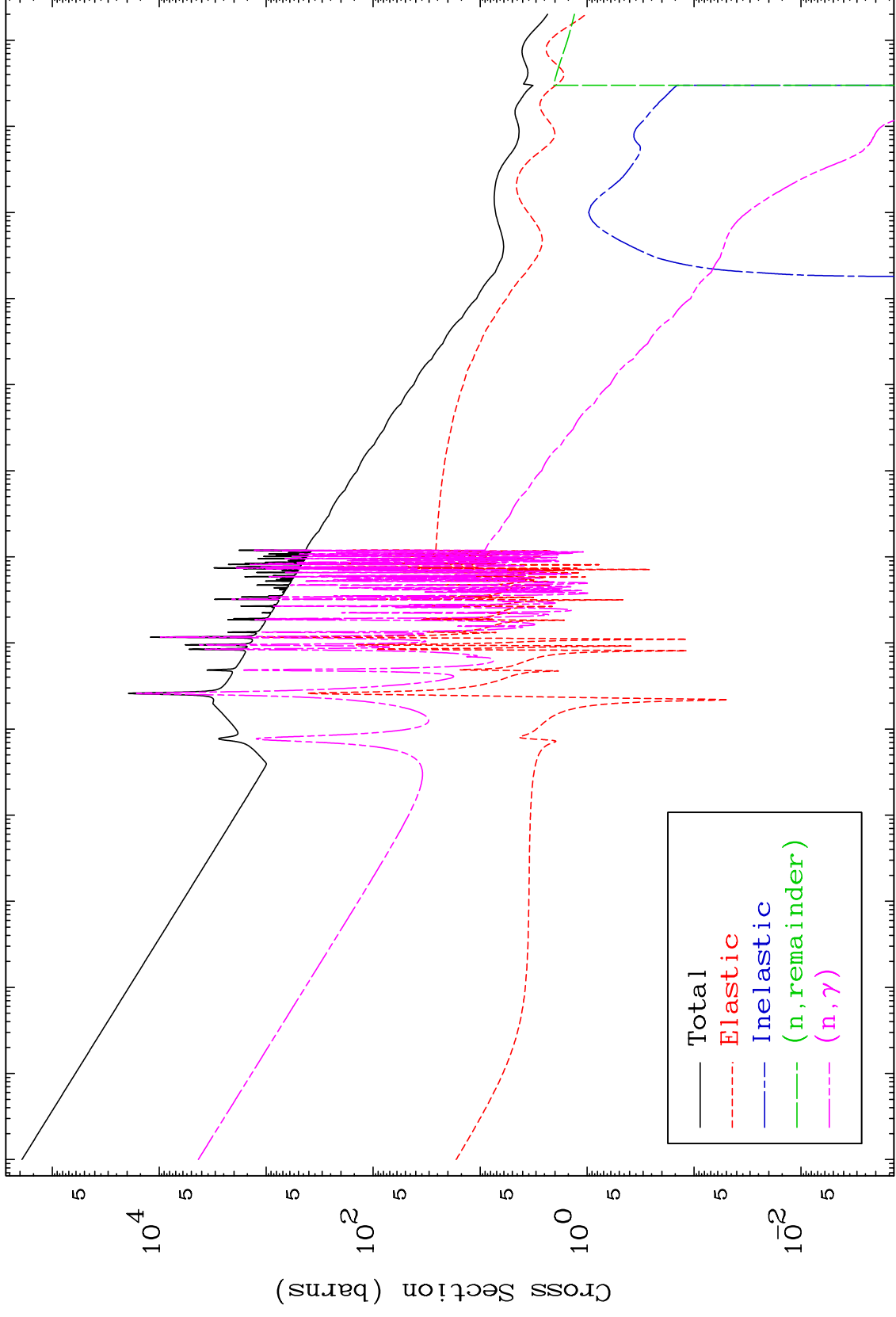
Press Mouse Button to Start

MAT 6777

Major

293 Kelvin Cross Sections

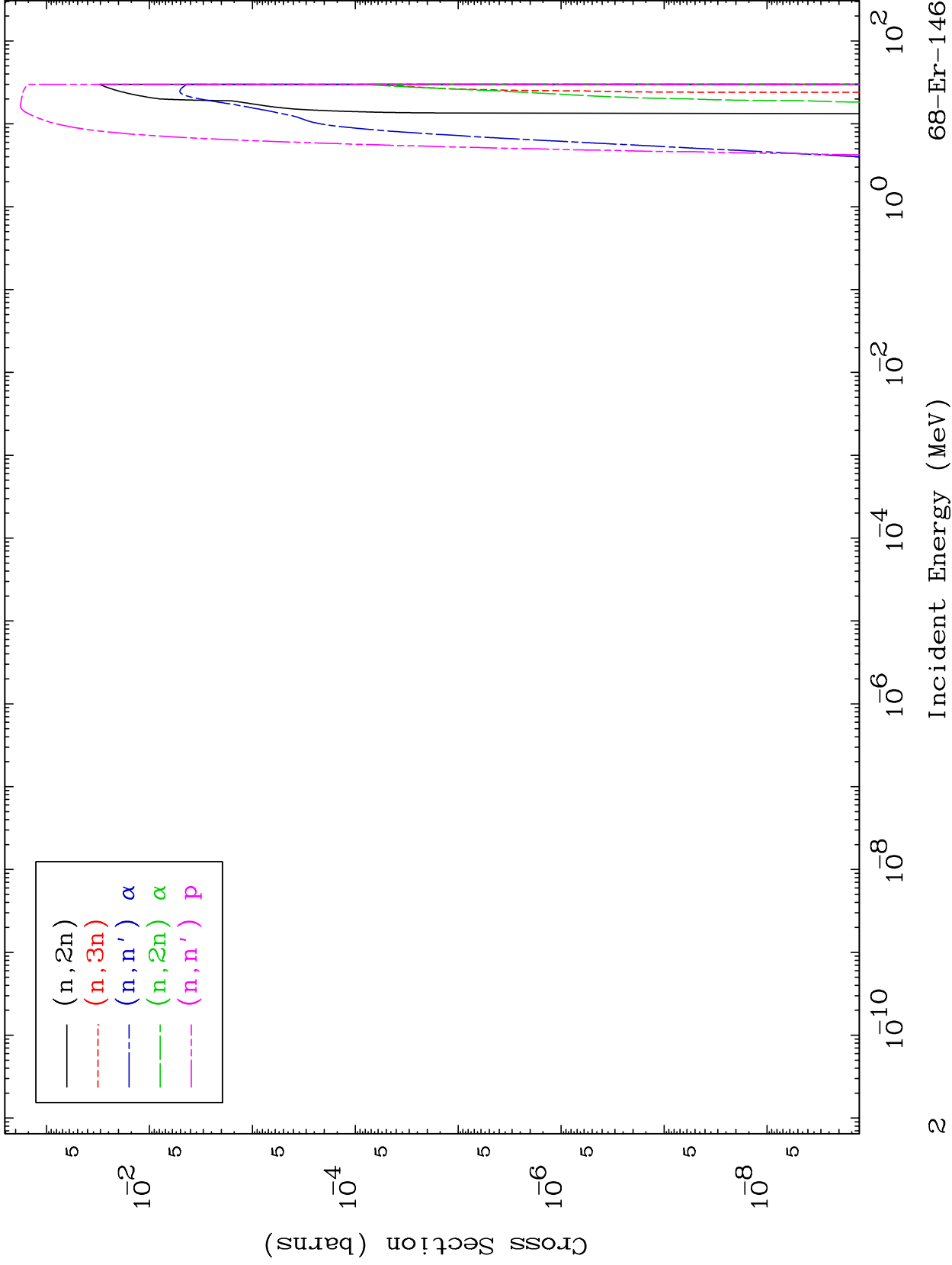
68-Er-146



MAT 6777

Neutron Production  
293 Kelvin Cross Sections

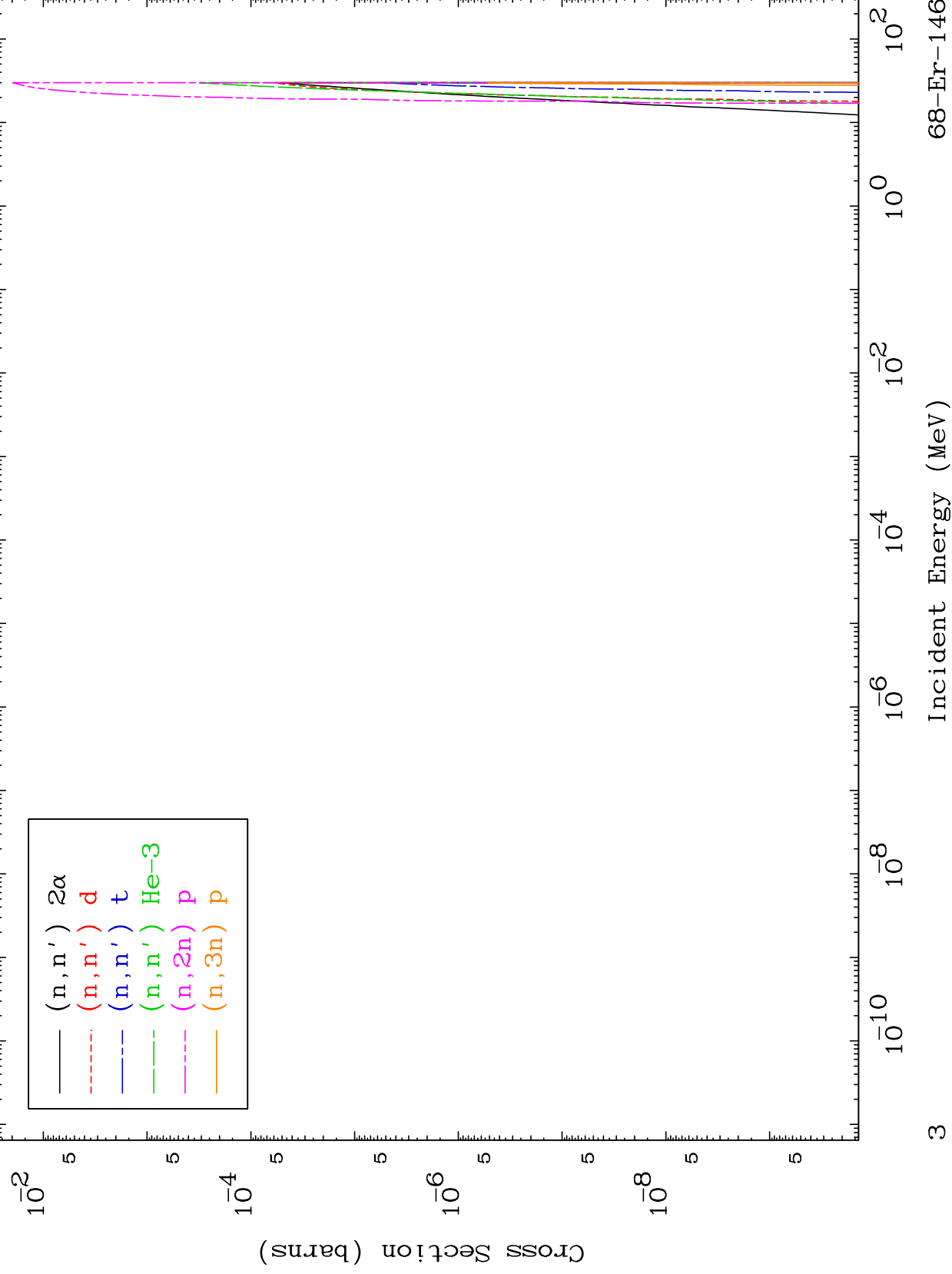
68-Er-146



MAT 6777

Neutron Production  
293 Kelvin Cross Sections

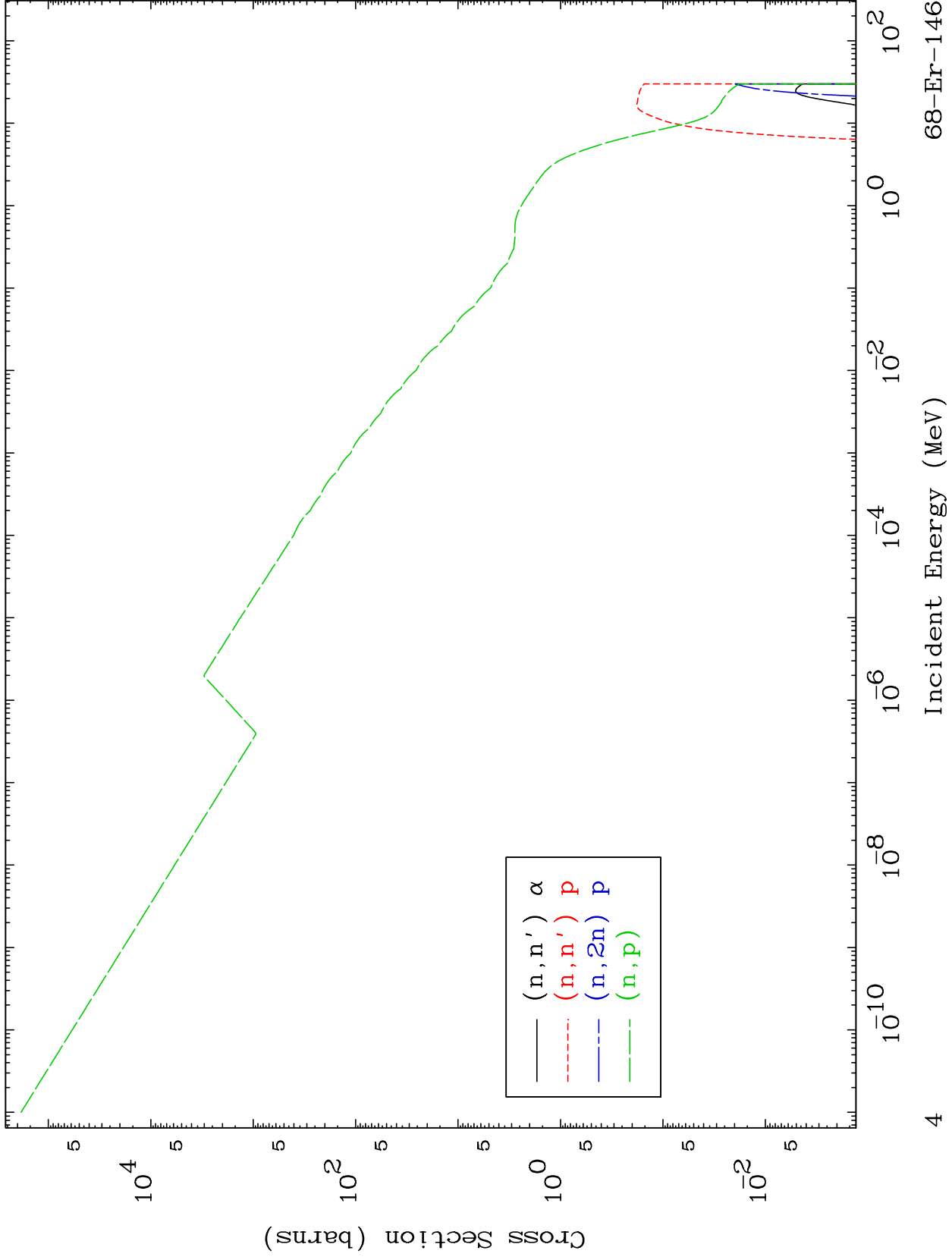
68-Er-146

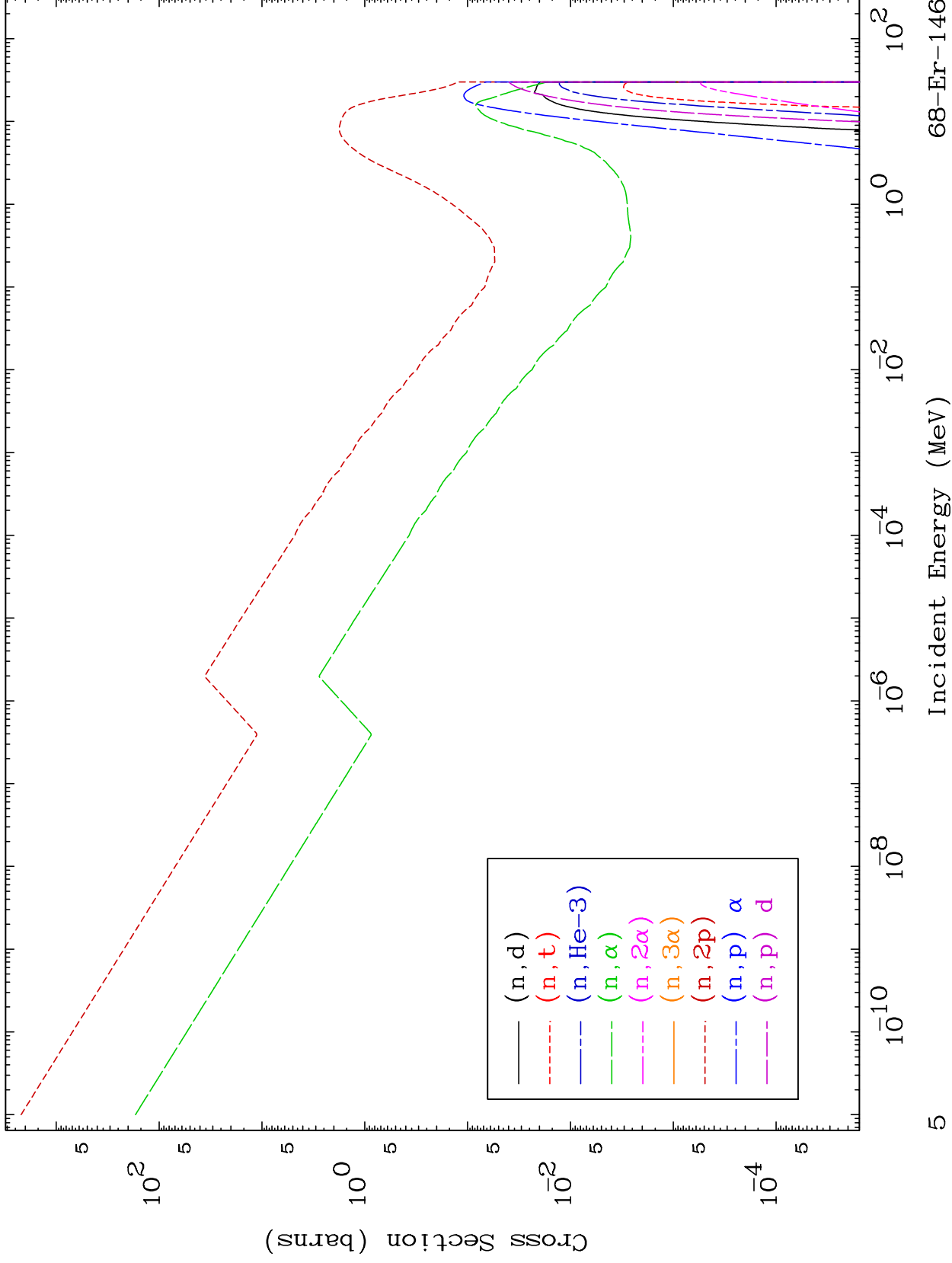


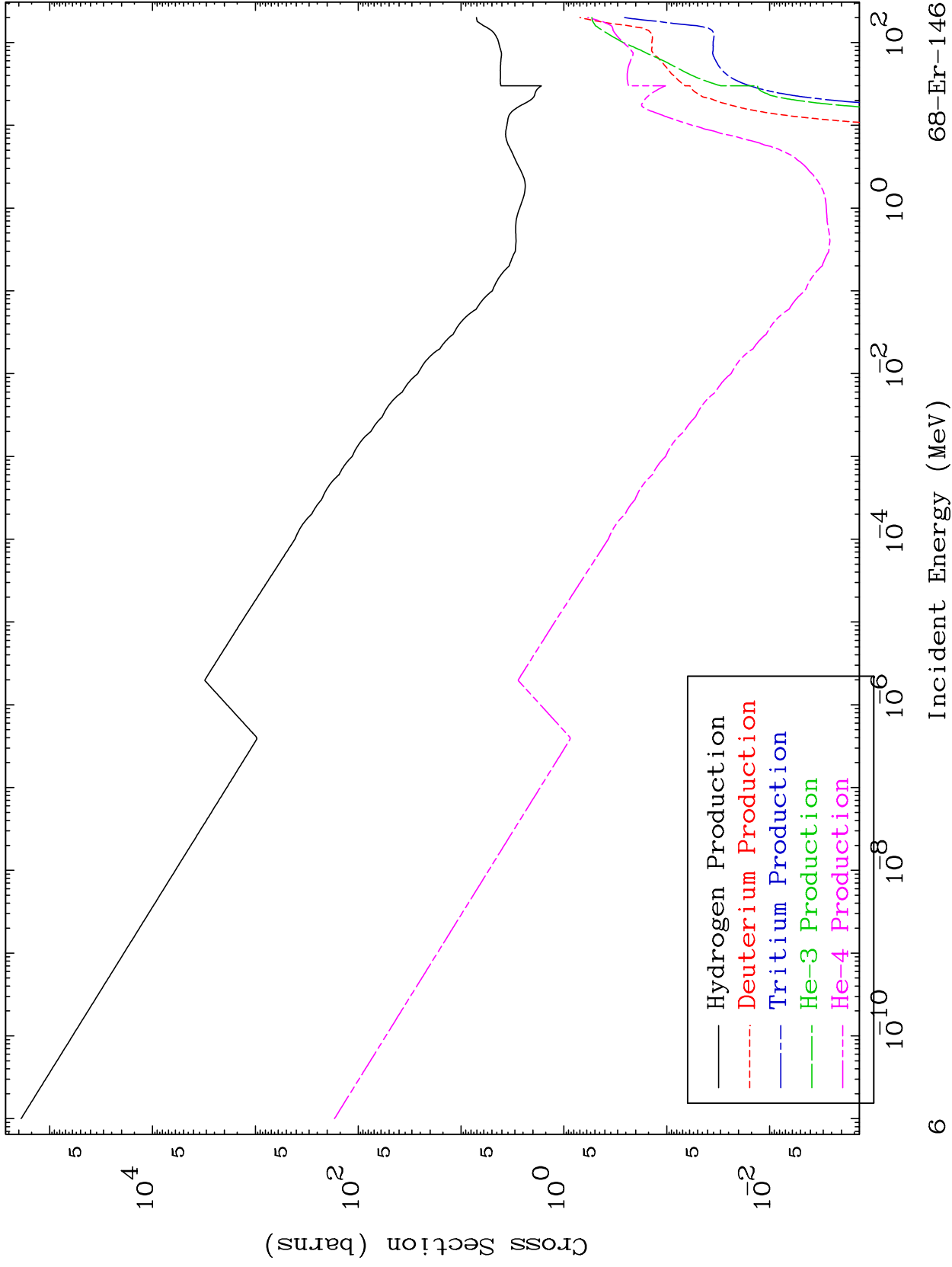
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Charged Particle  
293 Kelvin Cross Sections

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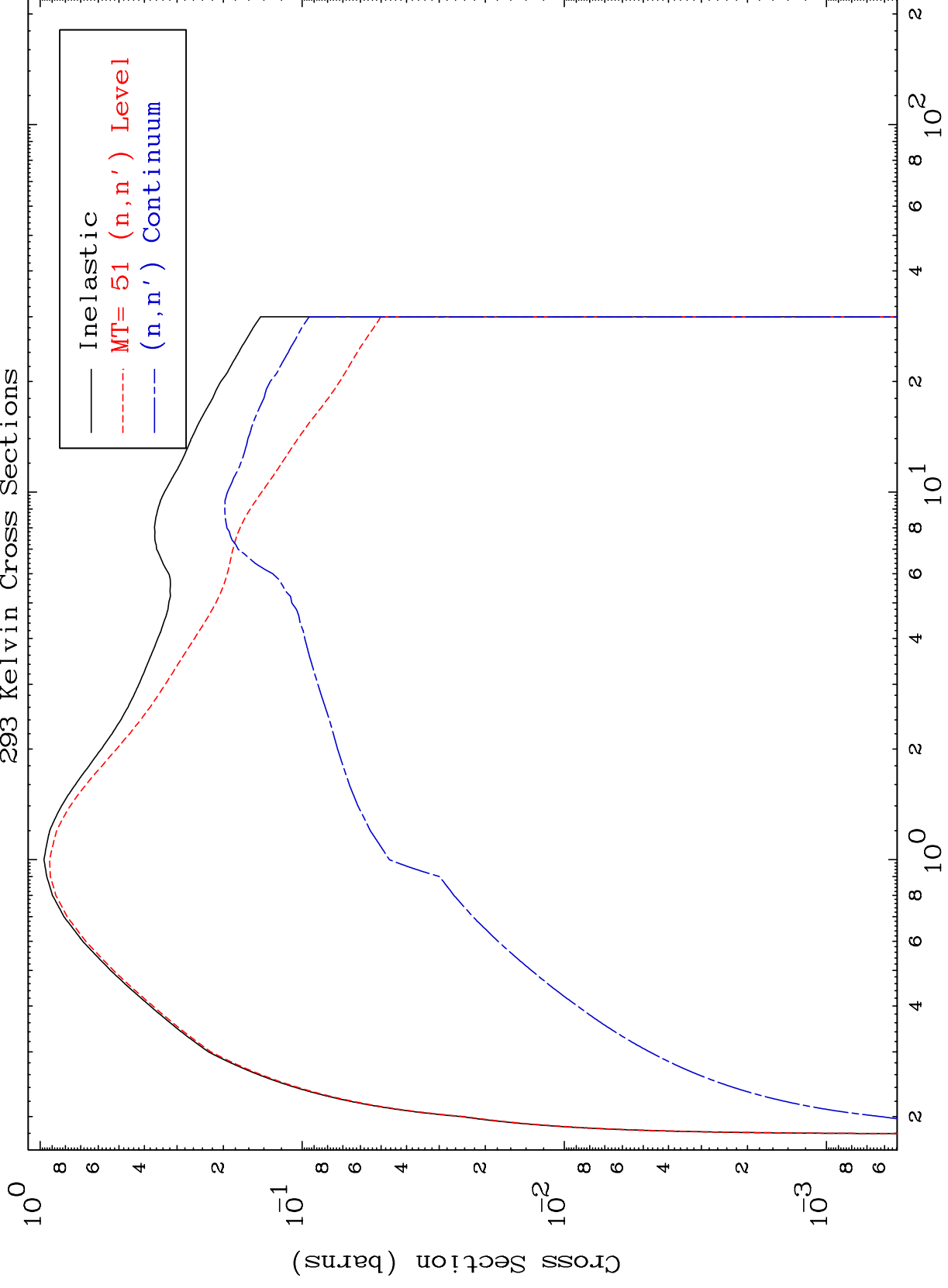


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(n,n') Level

68-Er-146

293 Kelvin Cross Sections

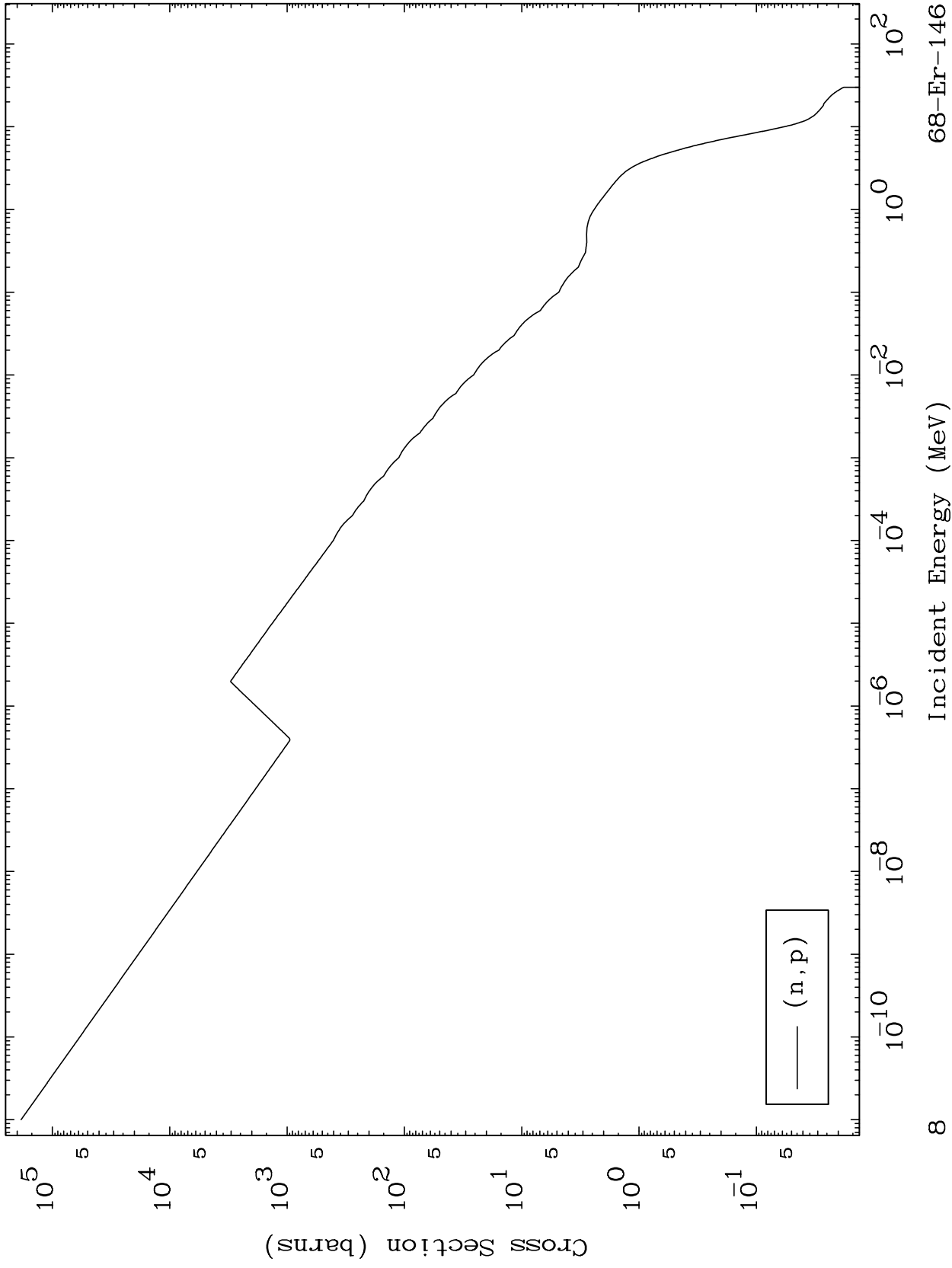




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(n,p) Levels  
293 Kelvin Cross Sections

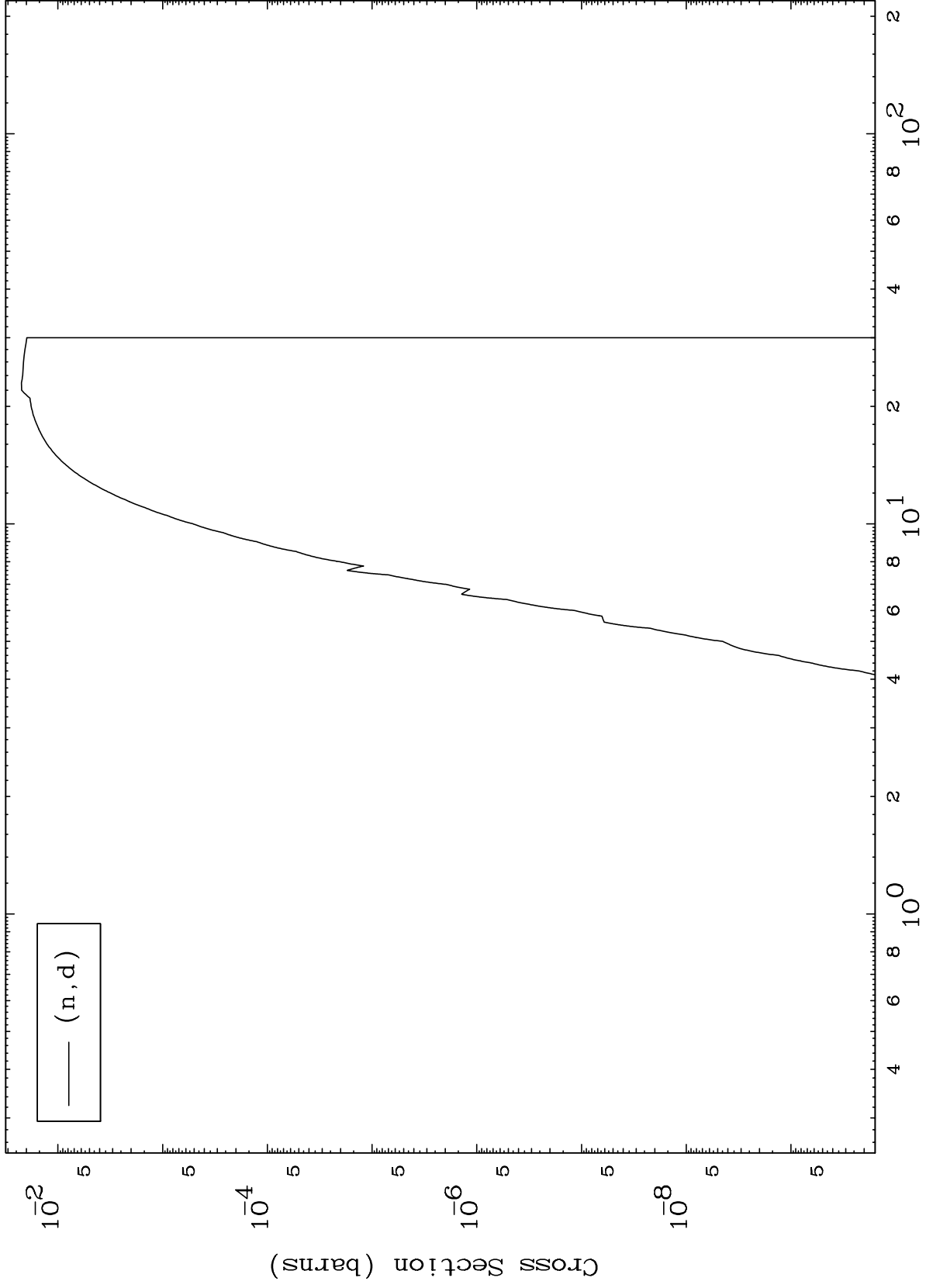
68-Er-146



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(n,d) Levels  
293 Kelvin Cross Sections

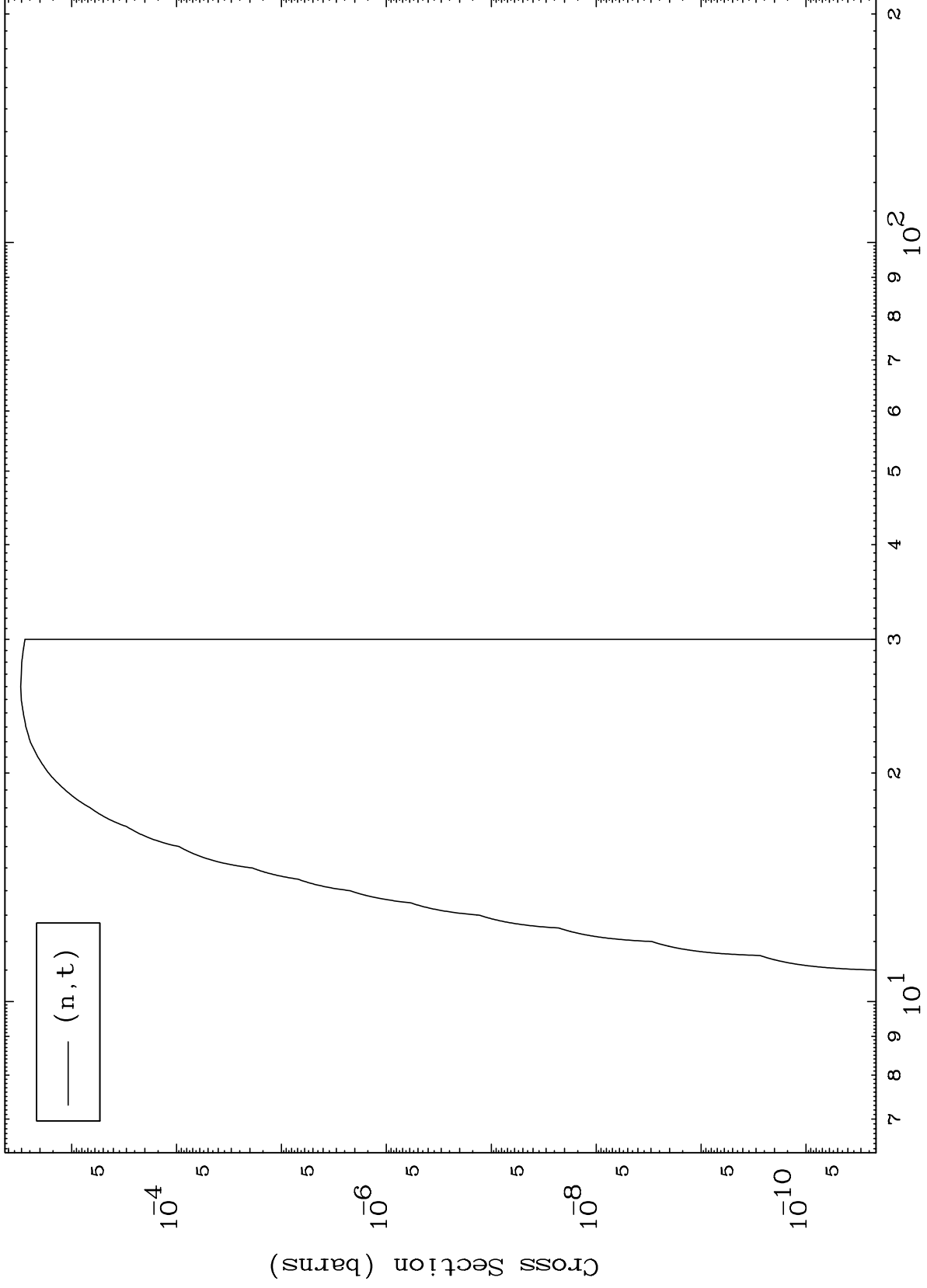
68-Er-146



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(n,t) Levels  
293 Kelvin Cross Sections

68-Er-146



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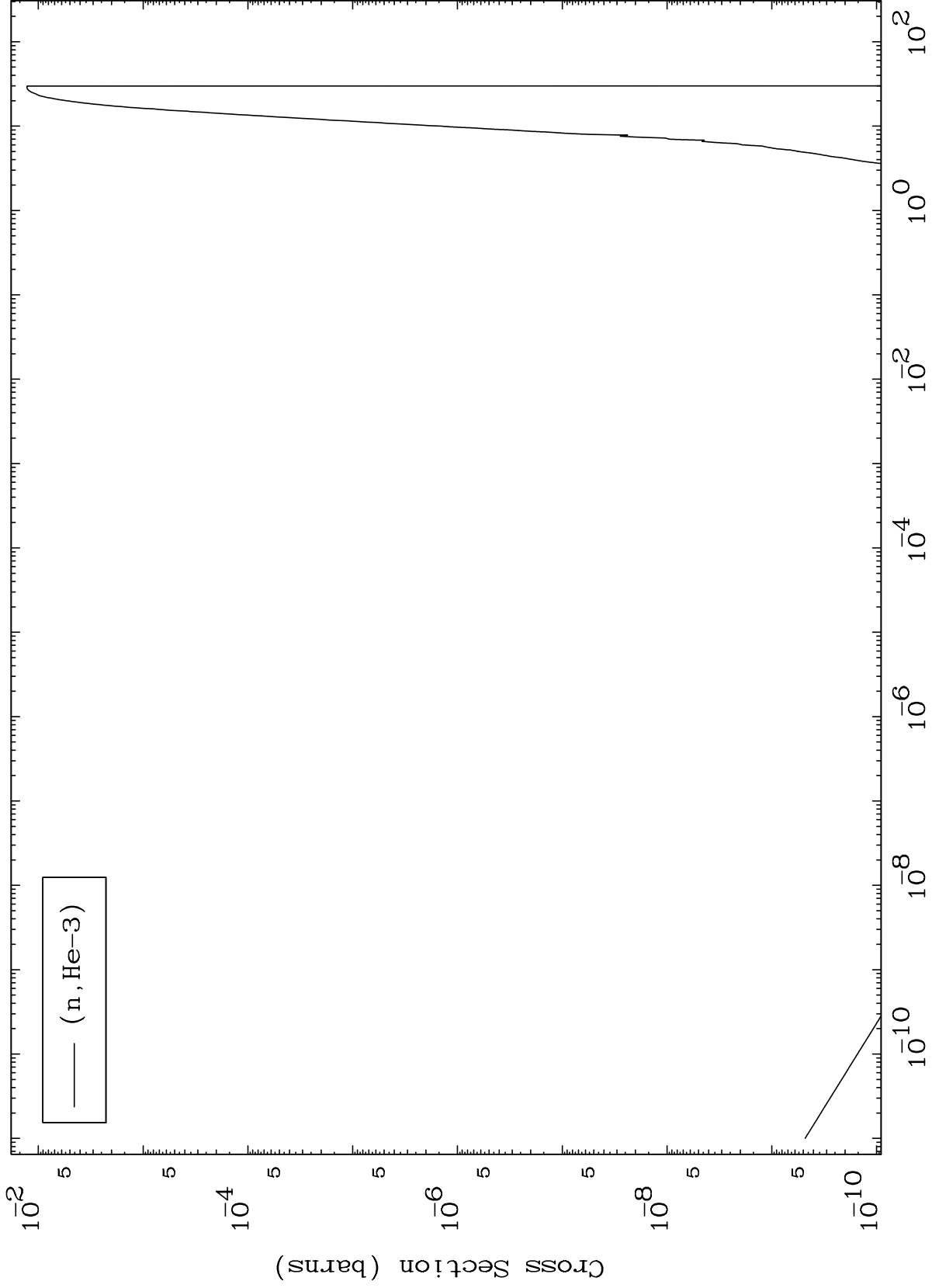
Incident Energy (MeV)

68-Er-146

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(n,He3) Levels  
293 Kelvin Cross Sections

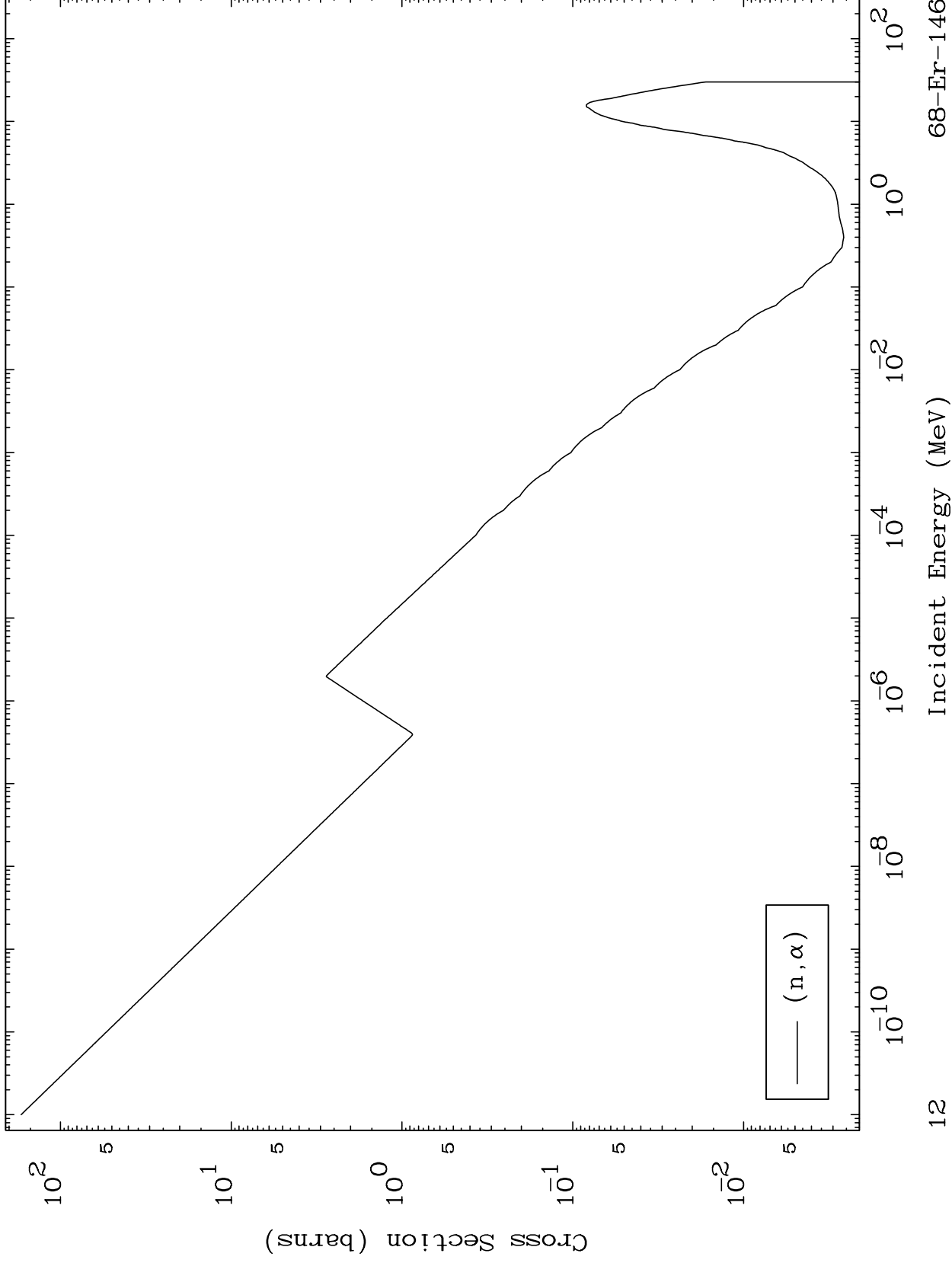
68-Er-146

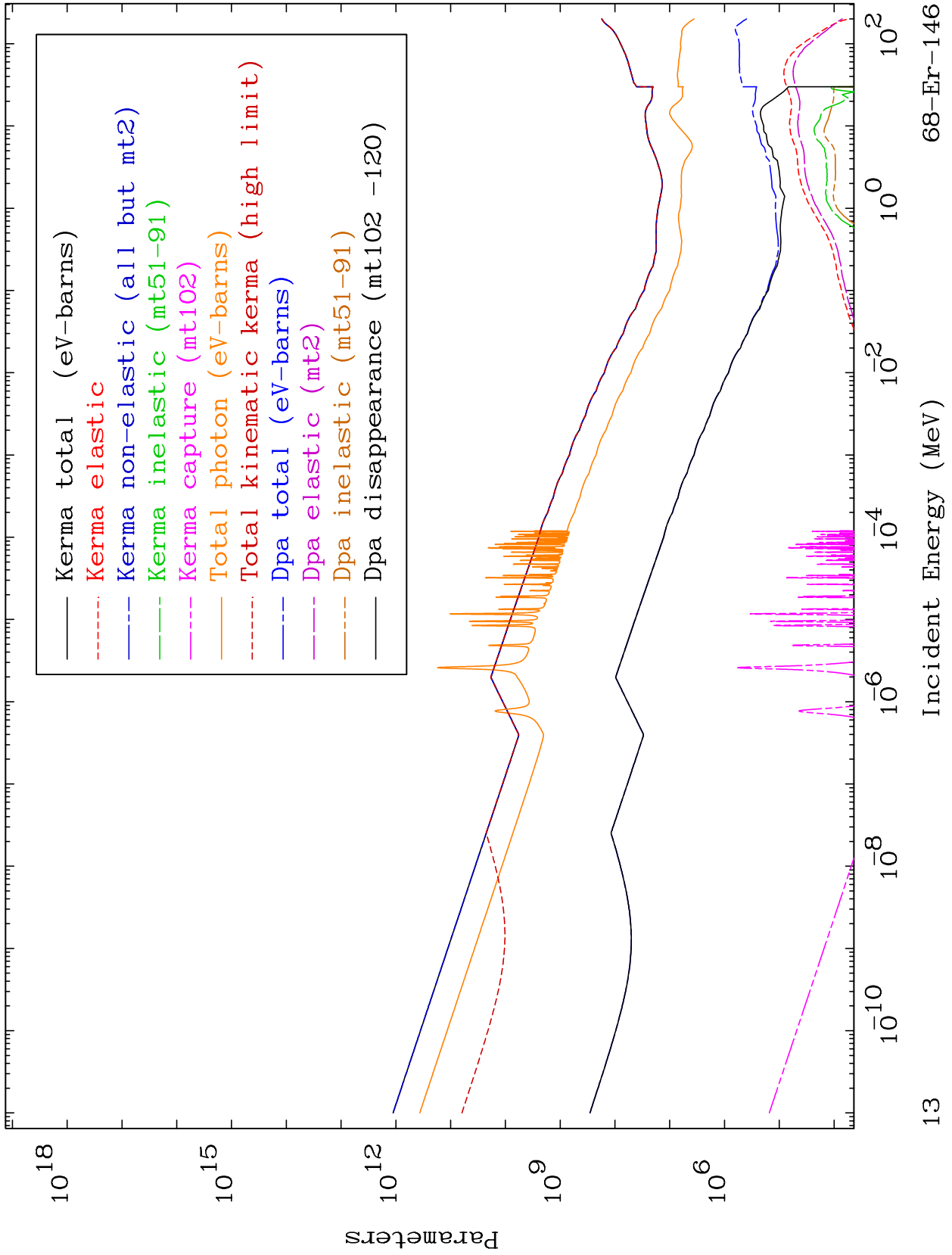


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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

68-Er-146

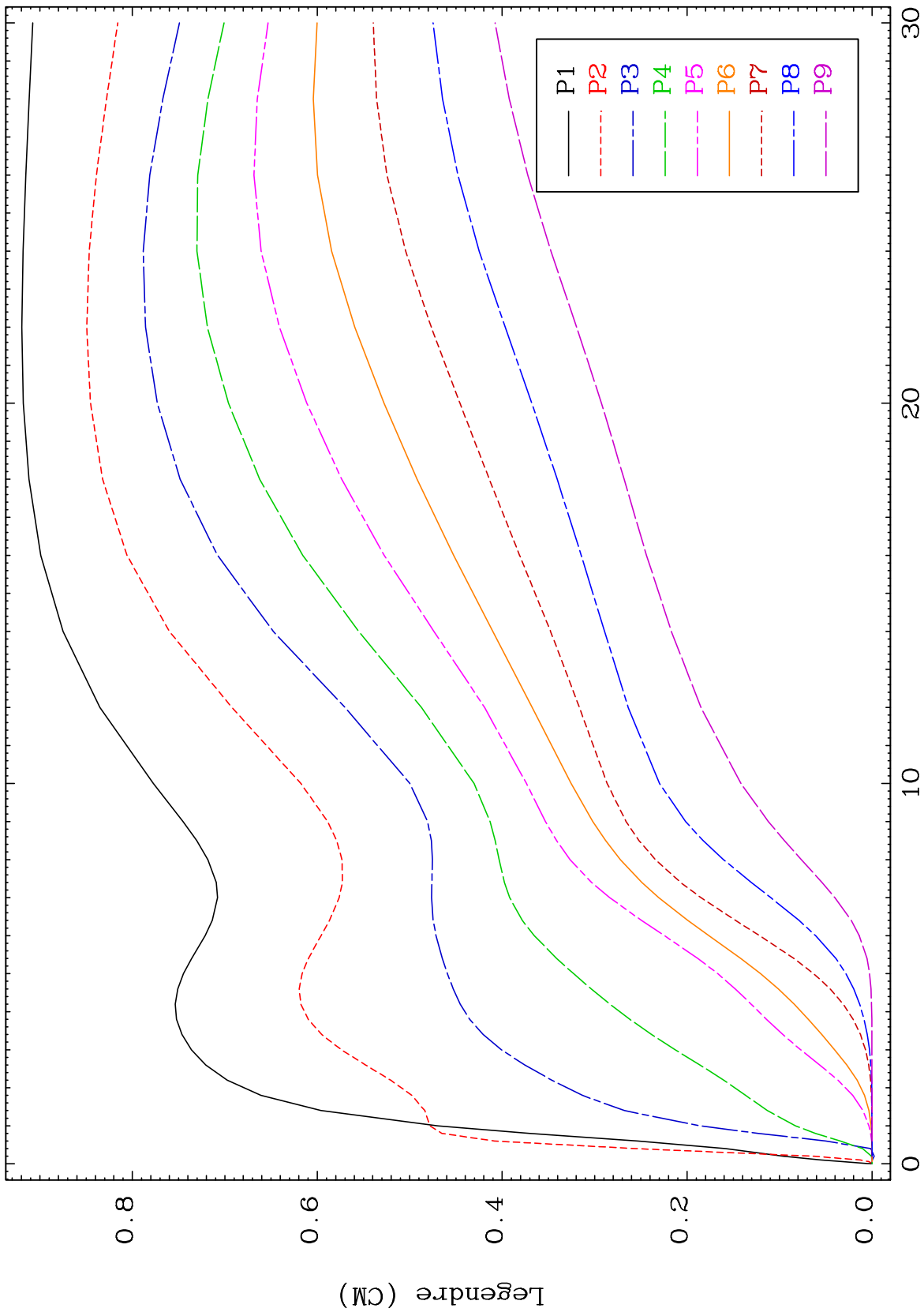




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Elastic Legendre Coefficients



68-Er-146

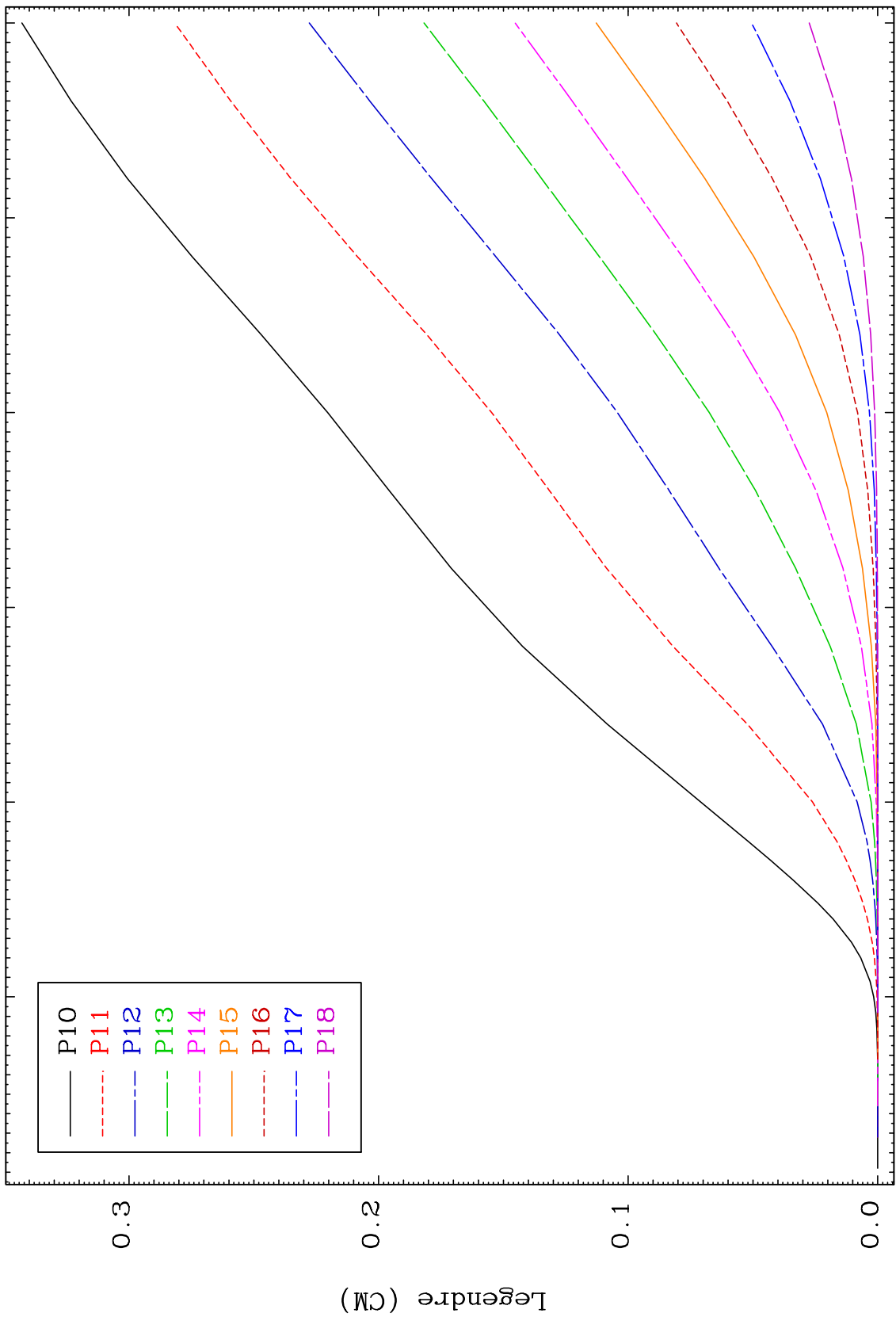
Incident Energy (MeV)

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Elastic Legendre Coefficients

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Incident Energy (MeV)

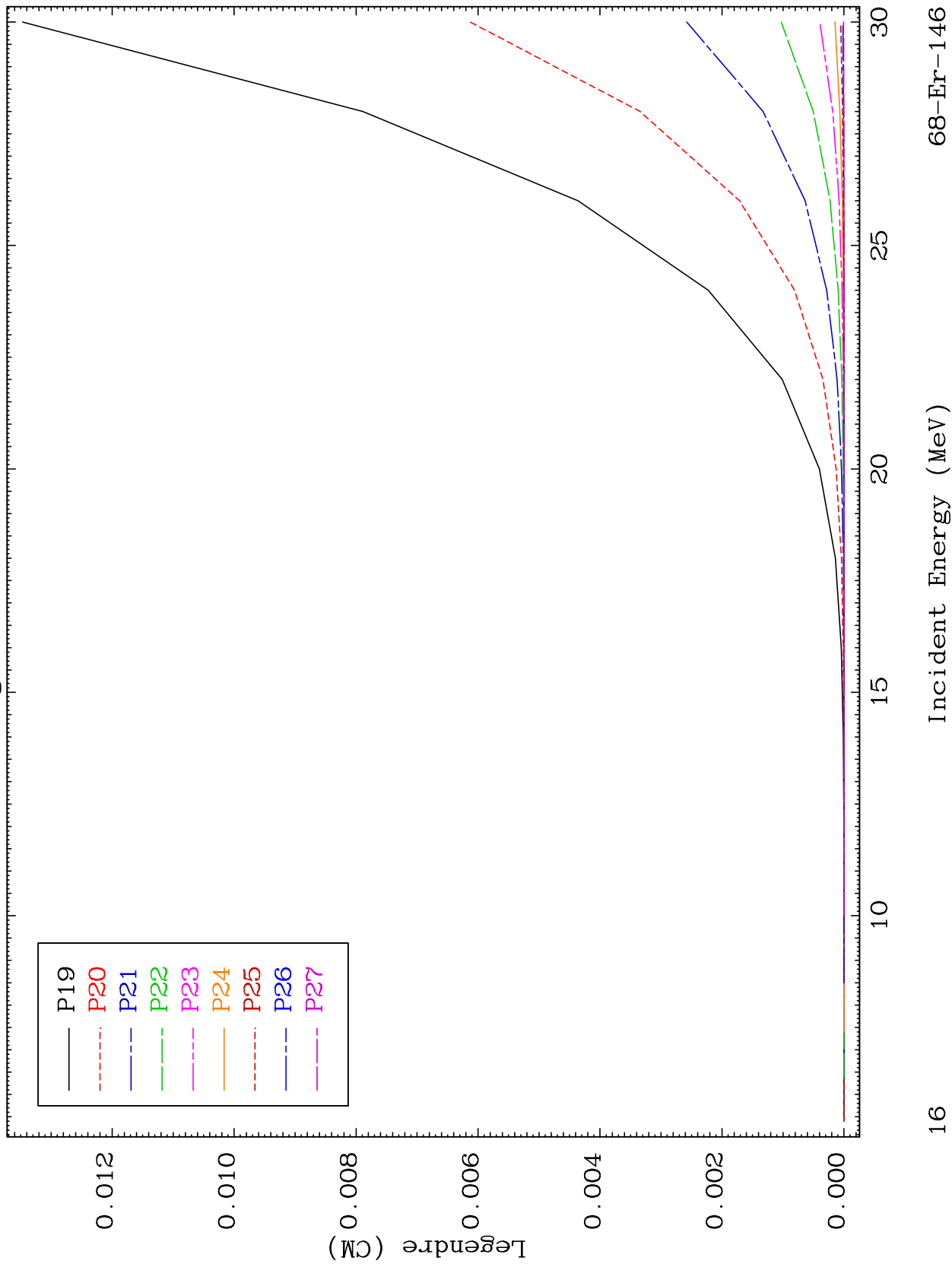
68-Er-146



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### Elastic Legendre Coefficients

68-Er-146



16

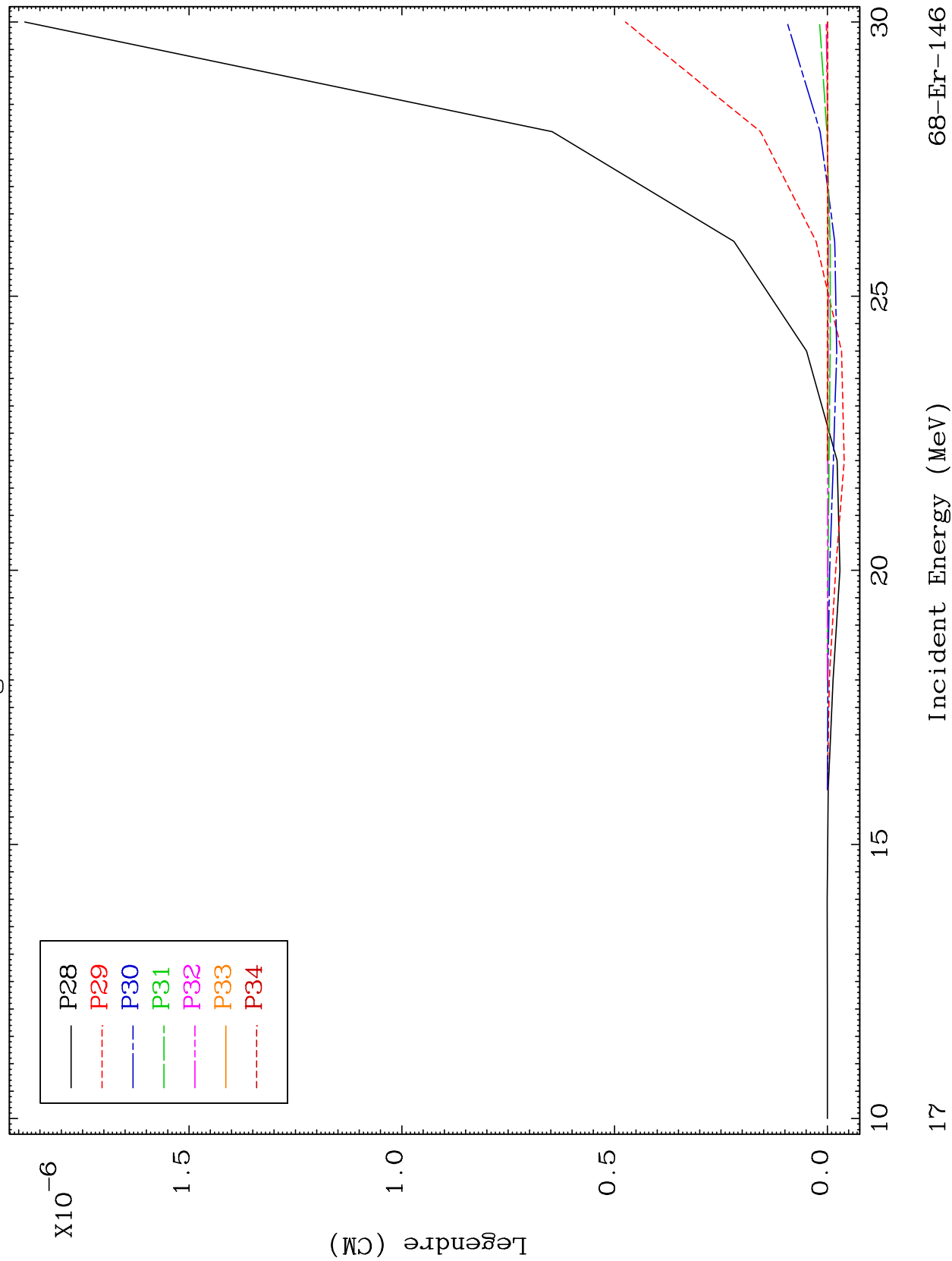
Incident Energy (MeV)

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Elastic  
Legendre Coefficients

68-Er-146



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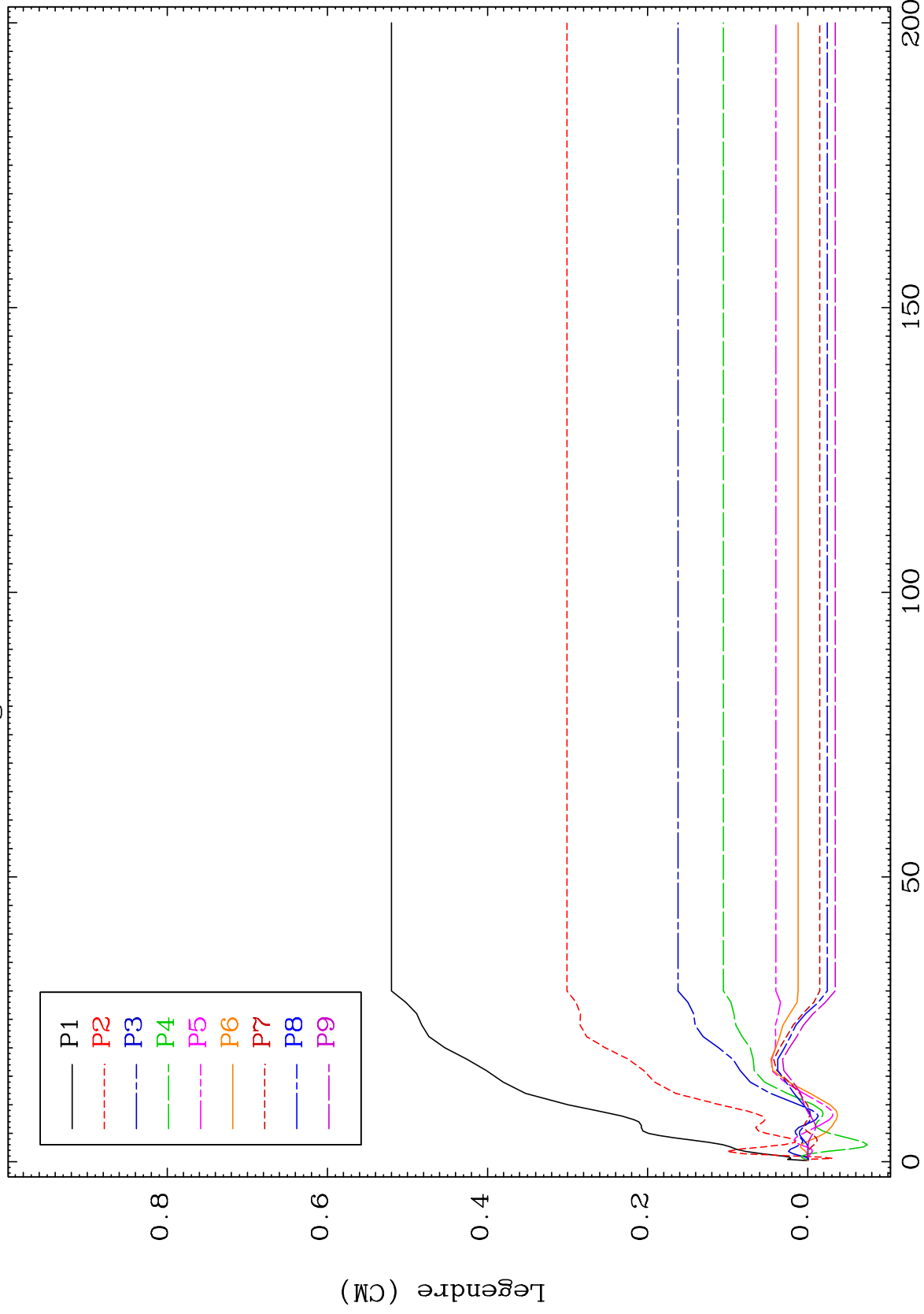
Incident Energy (MeV)

68-Er-146

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MT= 51 (n,n') Level  
Legendre Coefficients

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68-Er-146

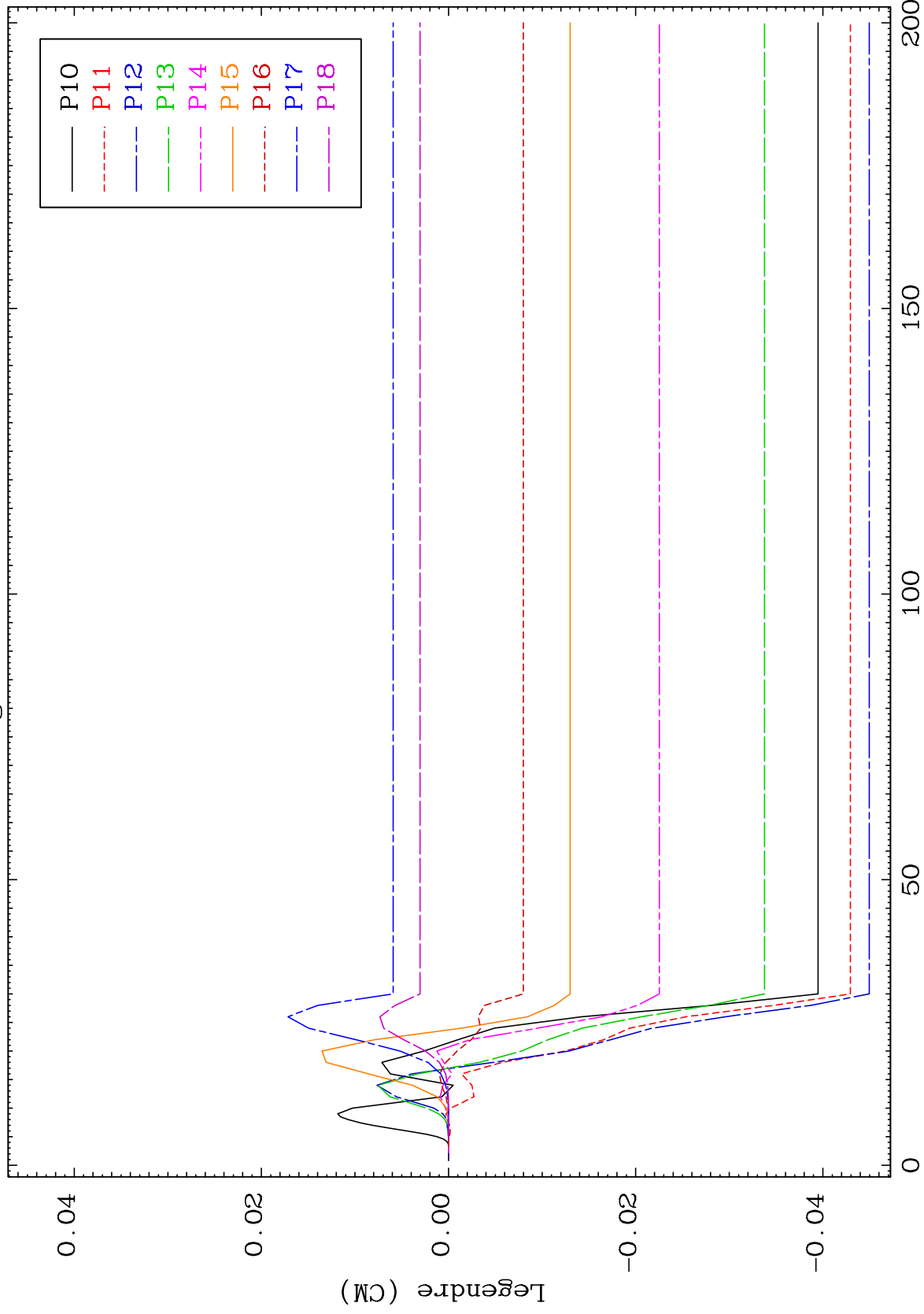
Incident Energy (MeV)

18

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MT= 51 (n,n') Level  
Legendre Coefficients

68-Er-146



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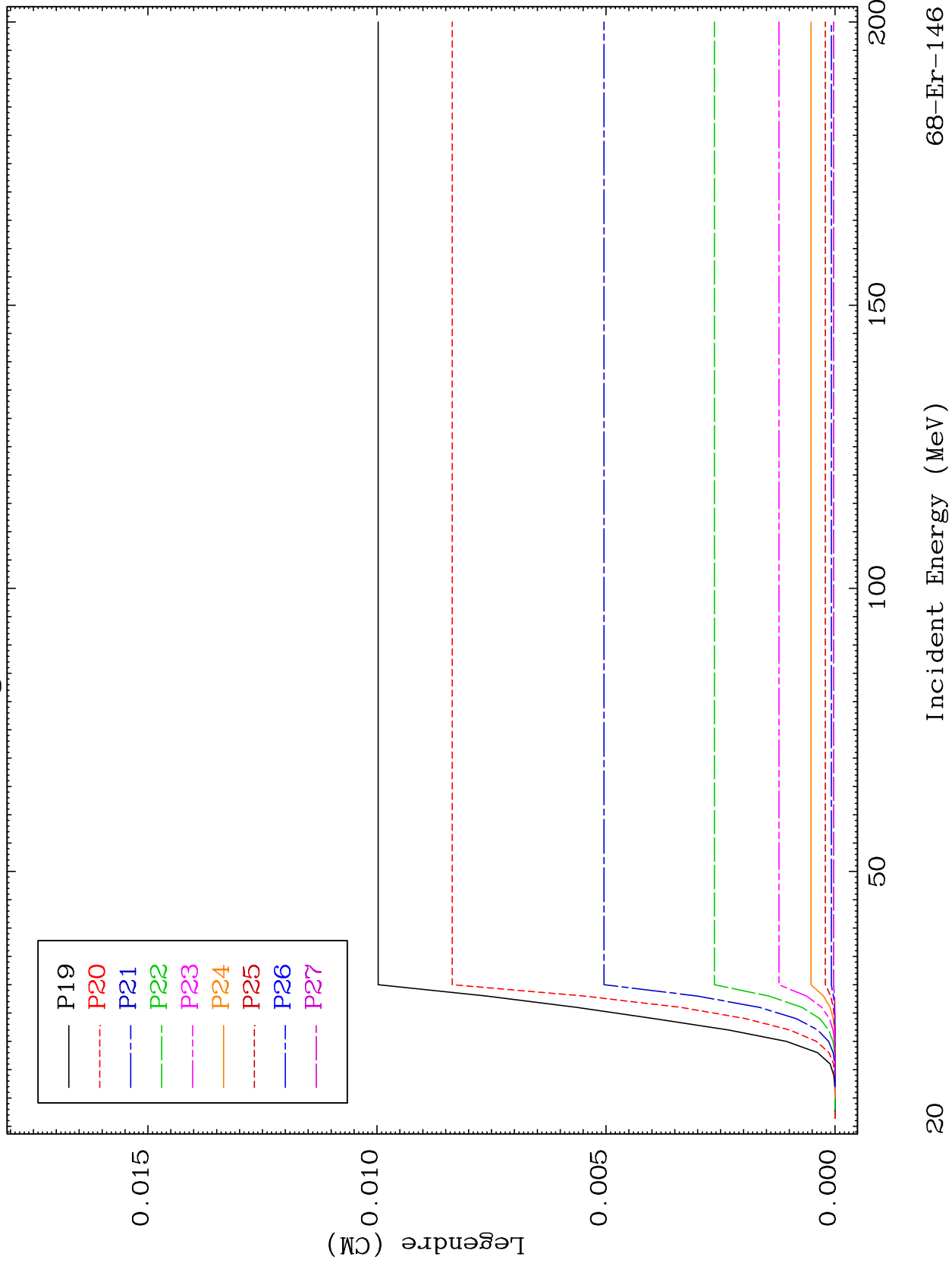
Incident Energy (MeV)

68-Er-146

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MT= 51 (n,n') Level  
Legendre Coefficients

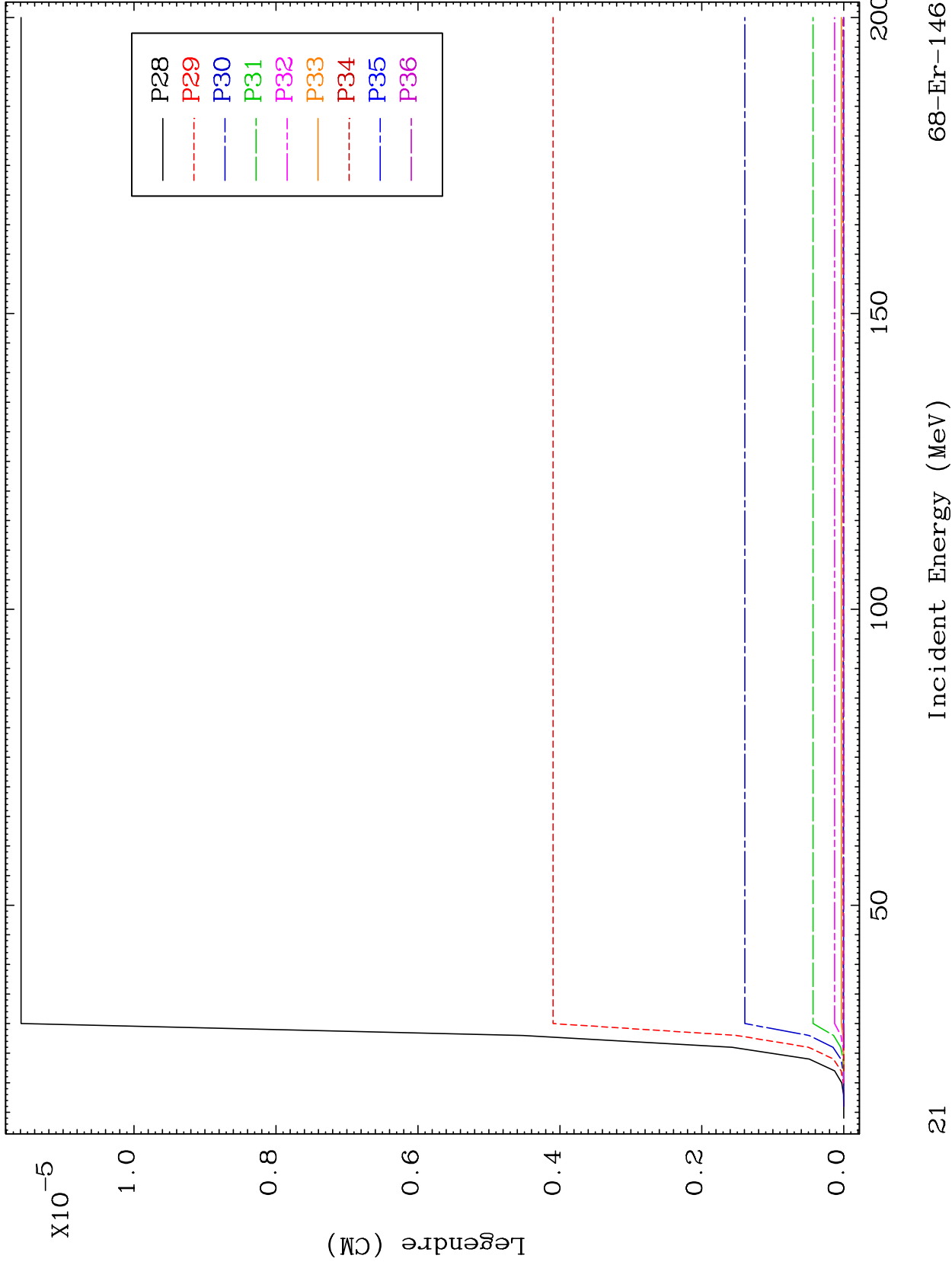
68-Er-146



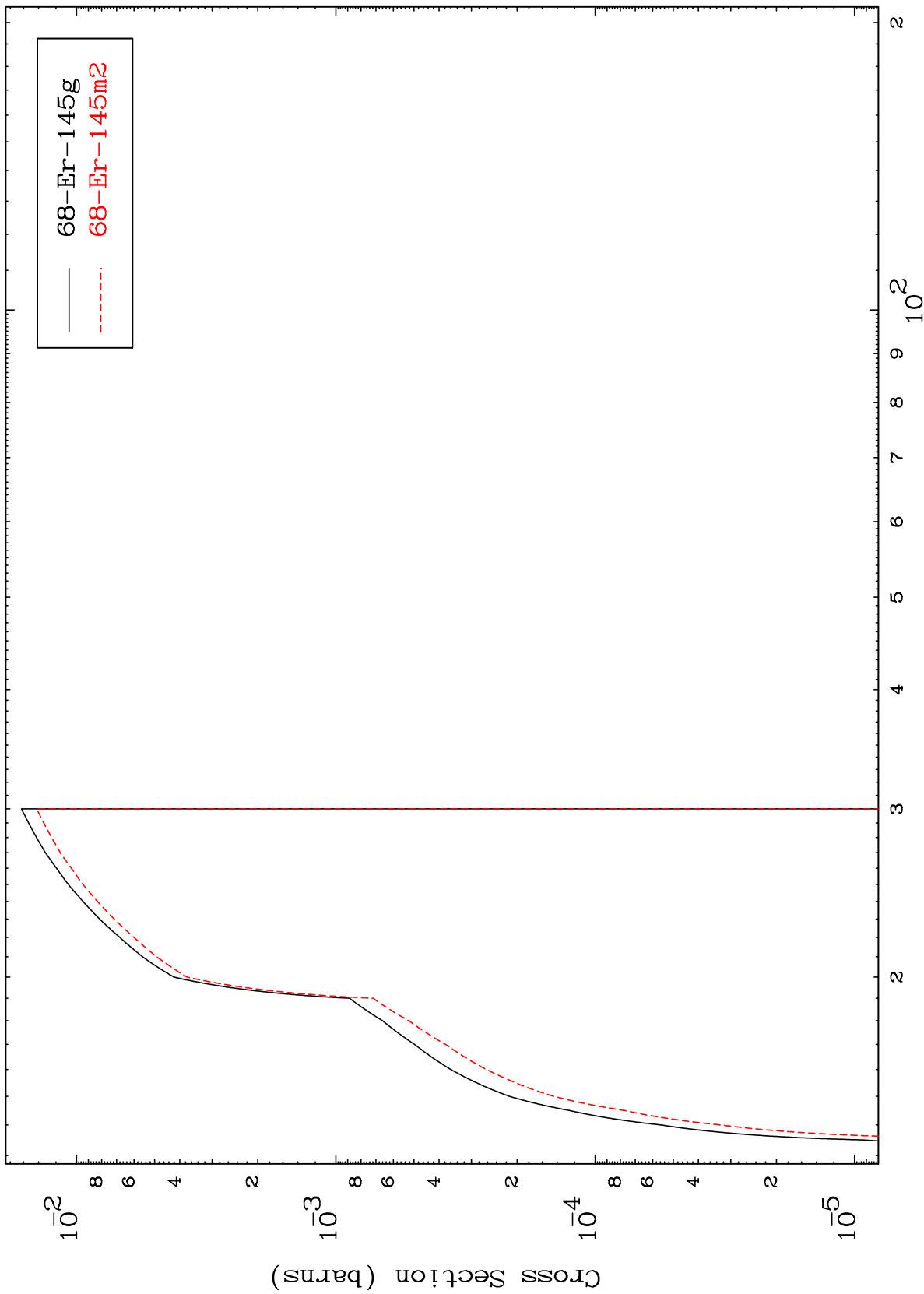
68-Er-146

Incident Energy (MeV)

20



(n,2n)  
Radionuclide Production Cross Section

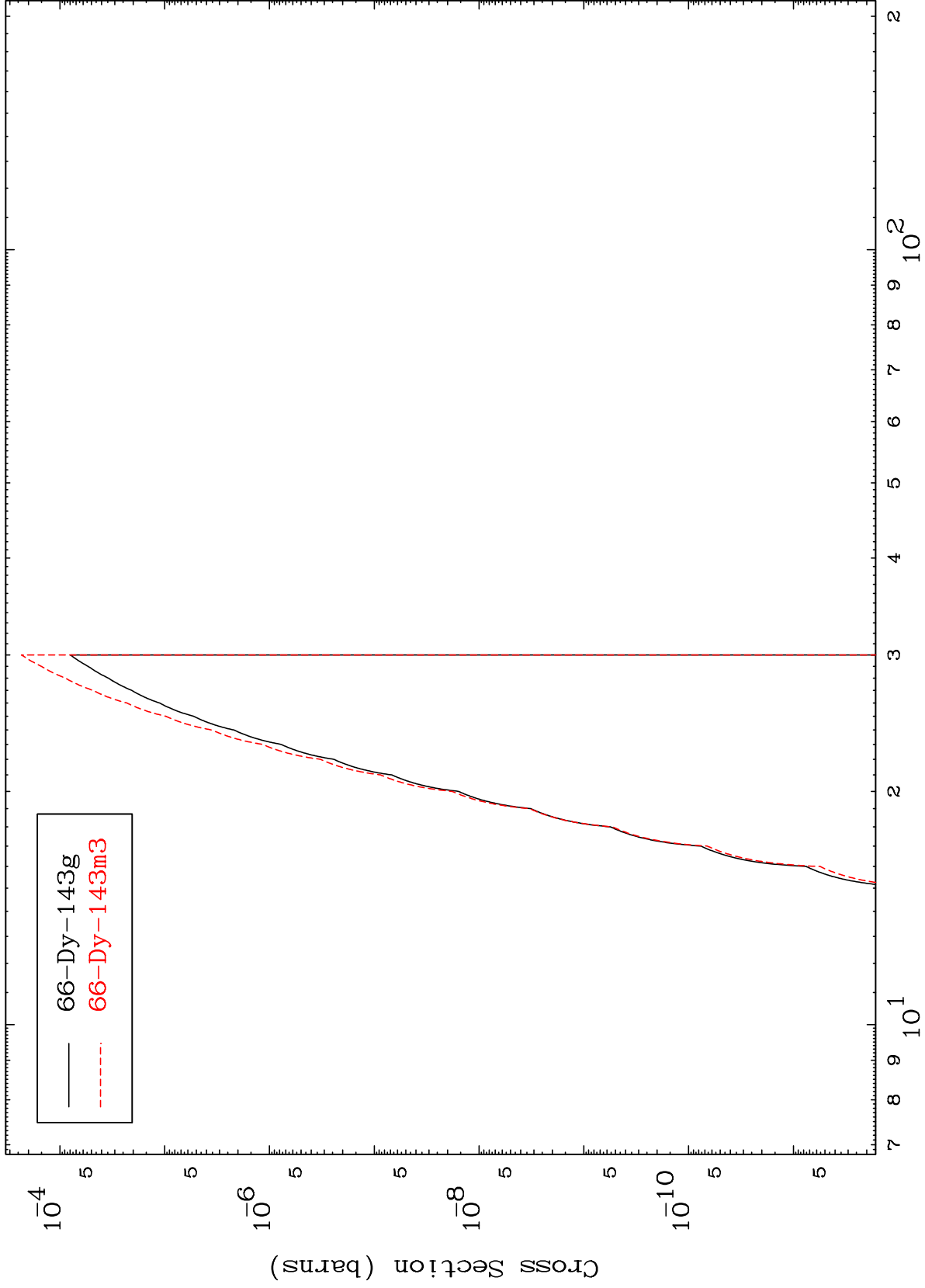


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(n,n') He-3

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Radionuclide Production Cross Section



23

Incident Energy (MeV)

68-Er-146

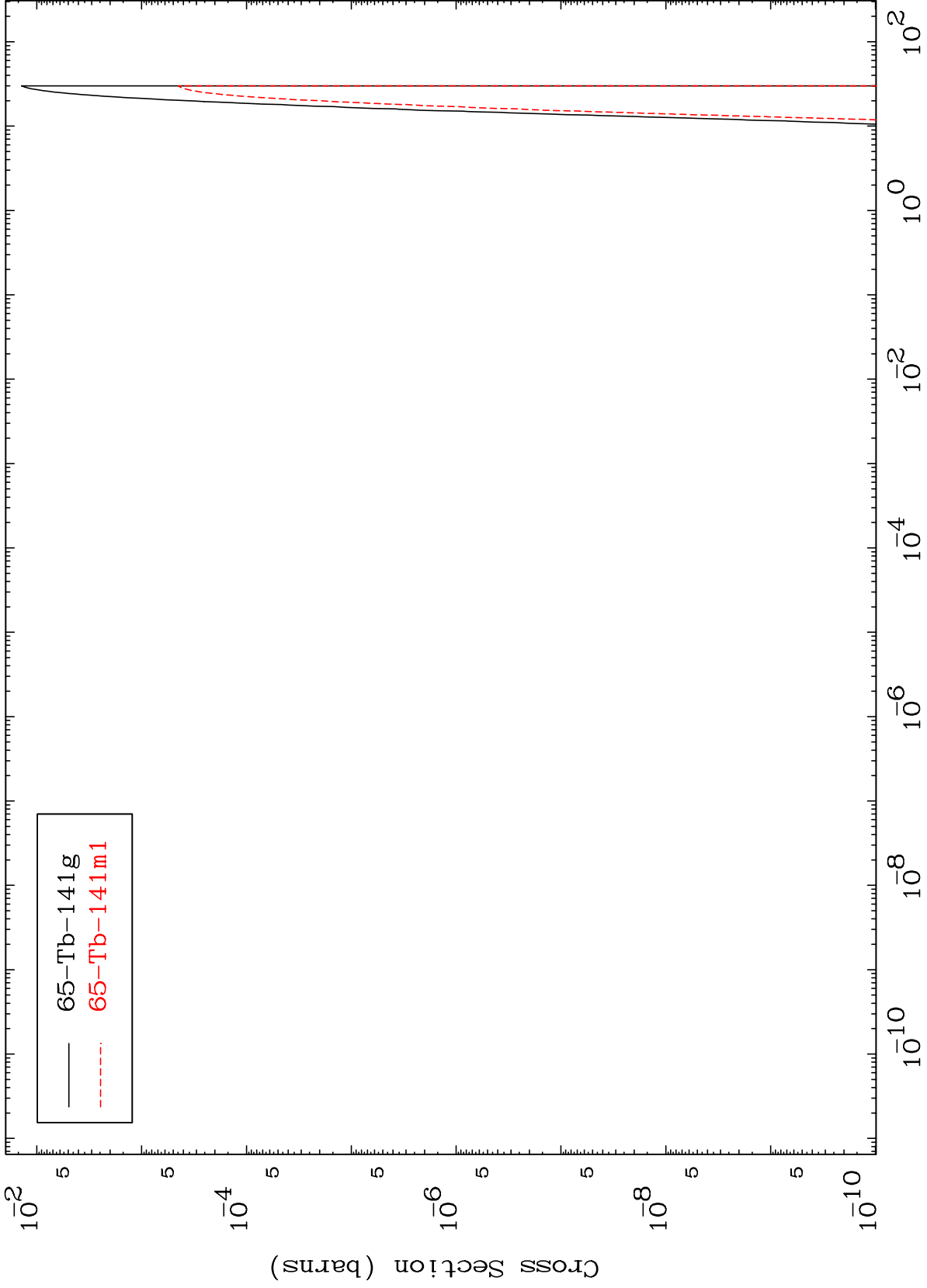


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(n,n') p  $\alpha$

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Radionuclide Production Cross Section

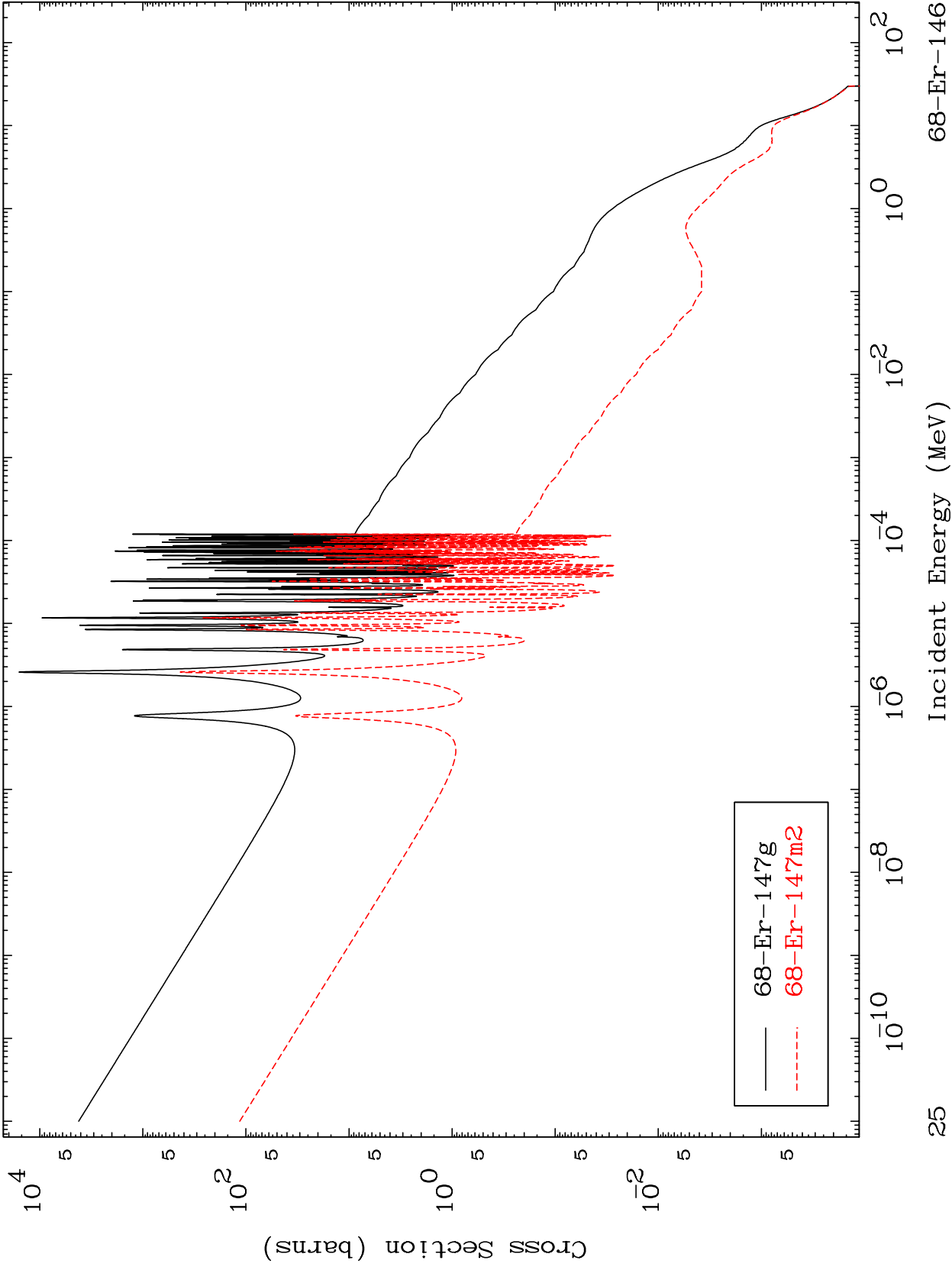


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Incident Energy (MeV)

68-Er-146

$(n, \gamma)$   
Radionuclide Production Cross Section

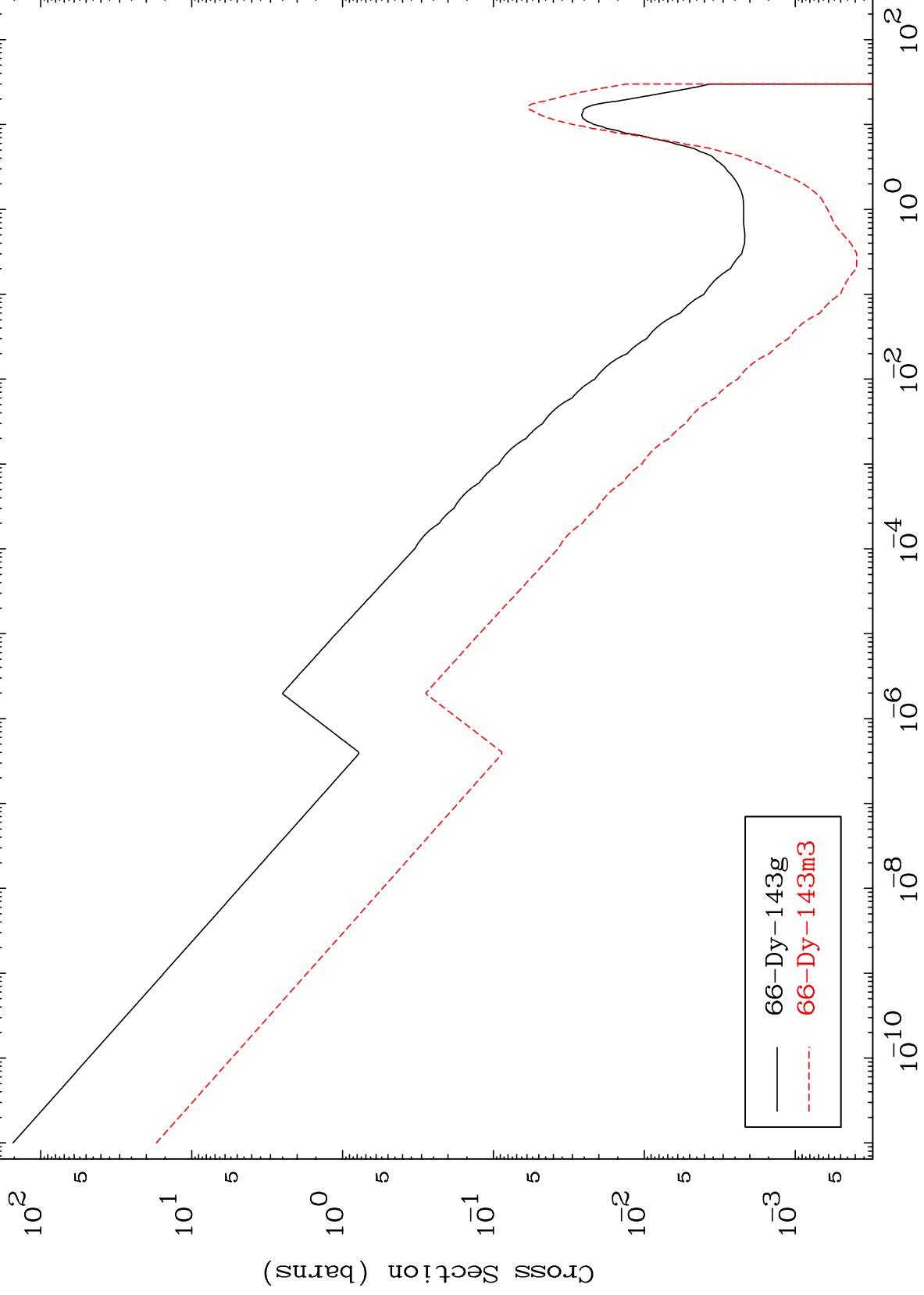


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(n,  $\alpha$ )

68-Er-146

Radionuclide Production Cross Section



26

Incident Energy (MeV)

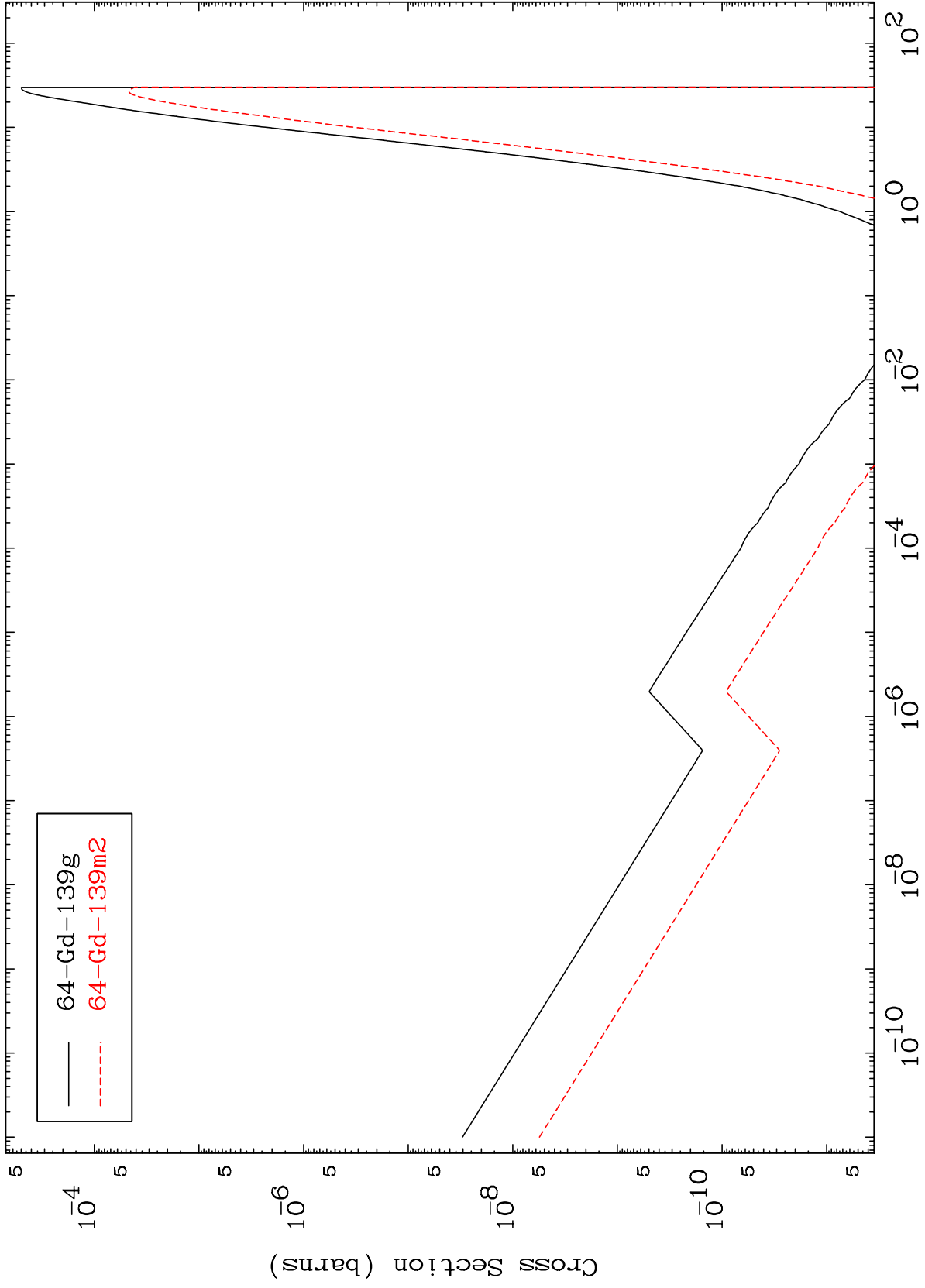
68-Er-146

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(n,2α)

68-Er-146

Radionuclide Production Cross Section



64-Gd-139g  
64-Gd-139m2

Incident Energy (MeV)

68-Er-146

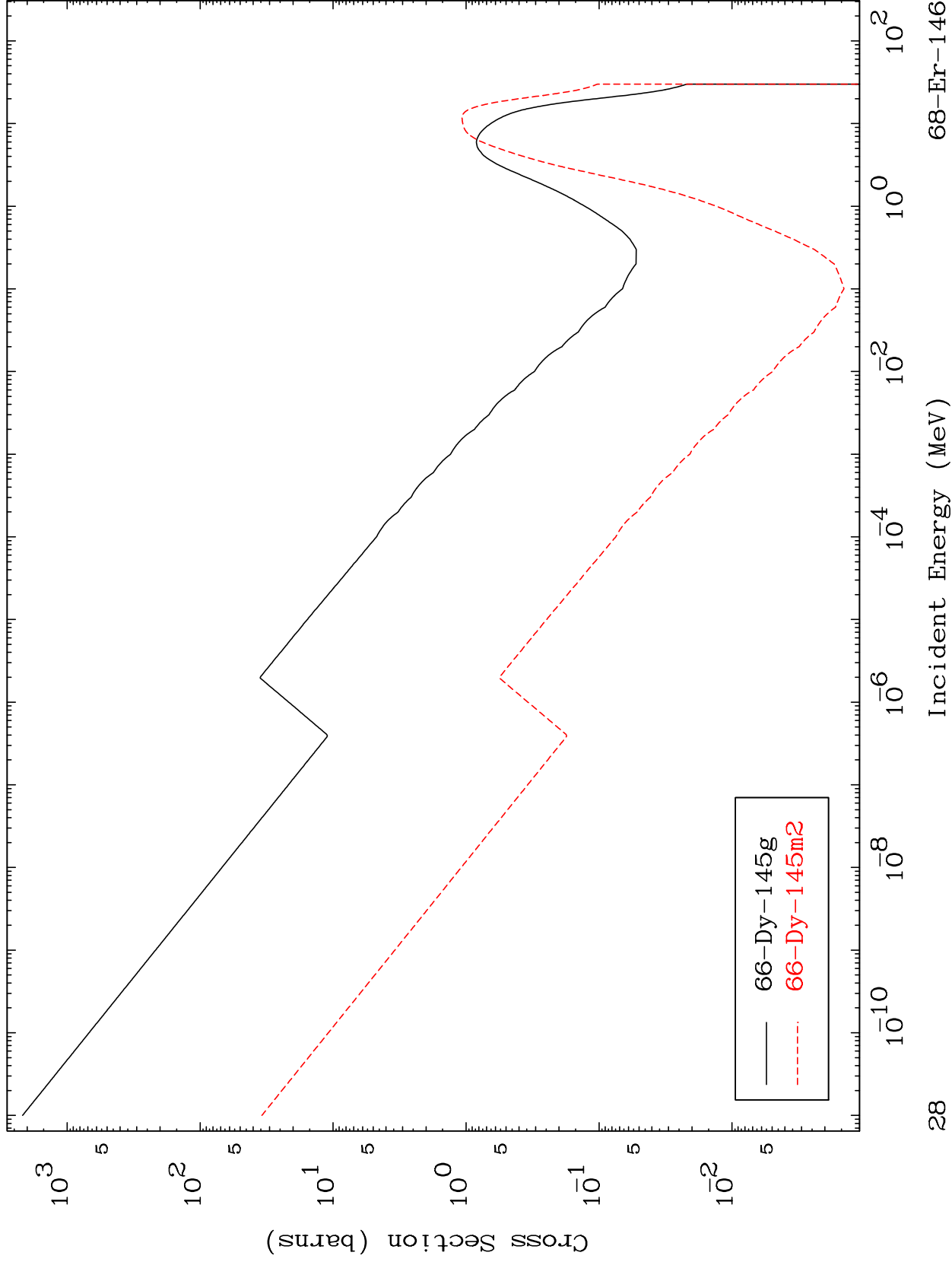
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(n,2p)

68-Er-146

Radionuclide Production Cross Section

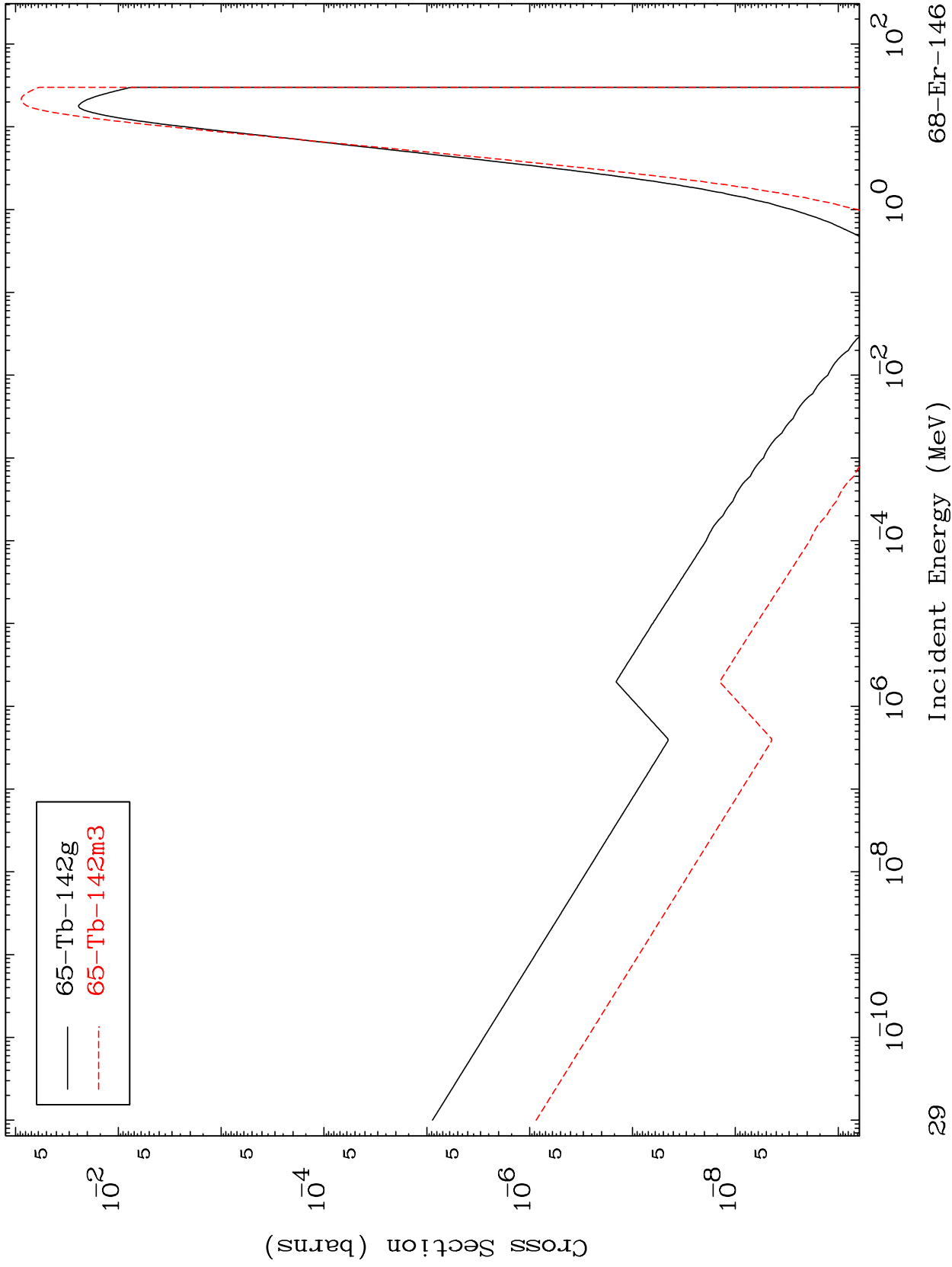


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(n,p)  $\alpha$

68-Er-146

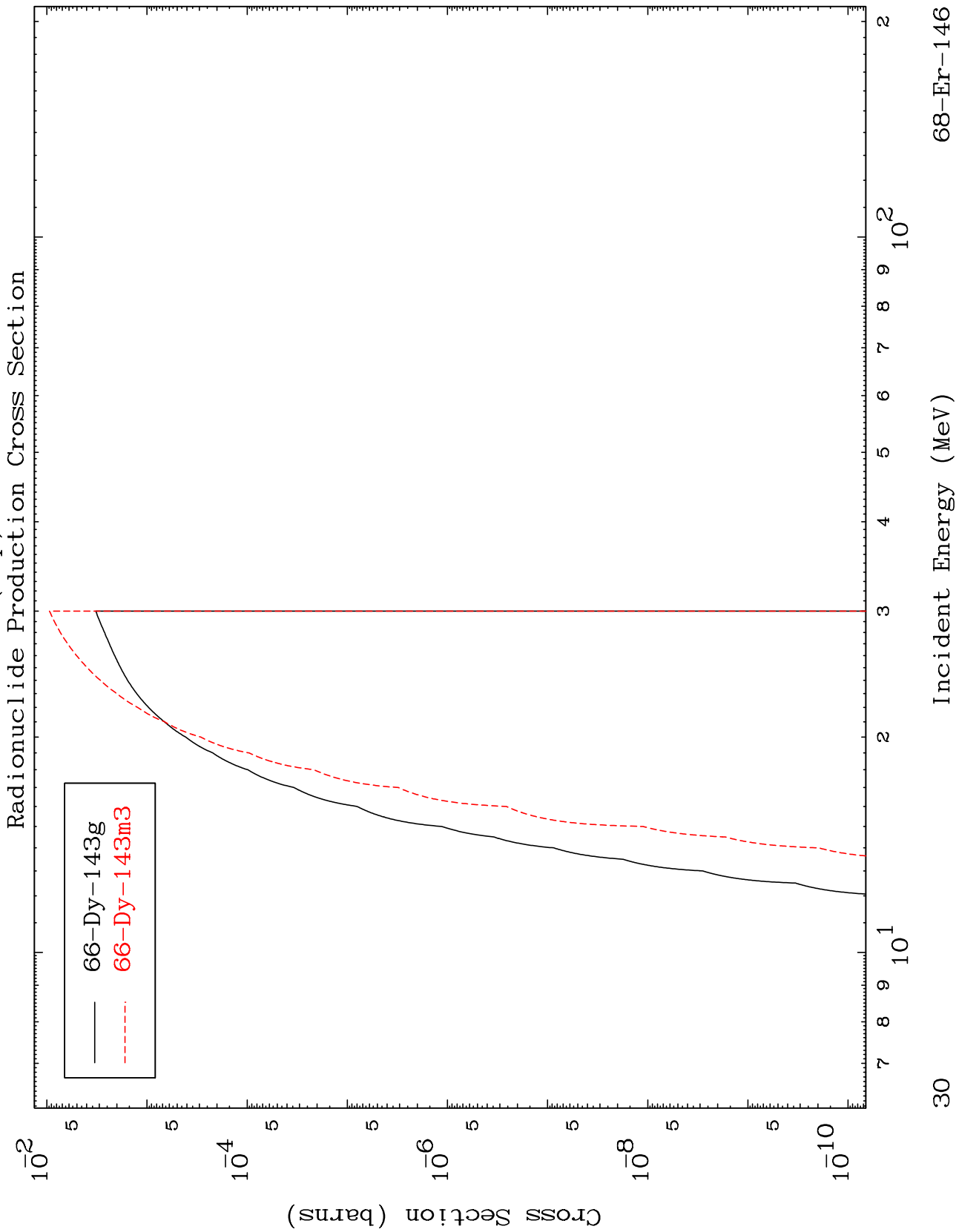
Radionuclide Production Cross Section



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(n,p) t

68-Er-146



MAT 6777

(n,d)  $\alpha$

68-Er-146

Radionuclide Production Cross Section

