

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

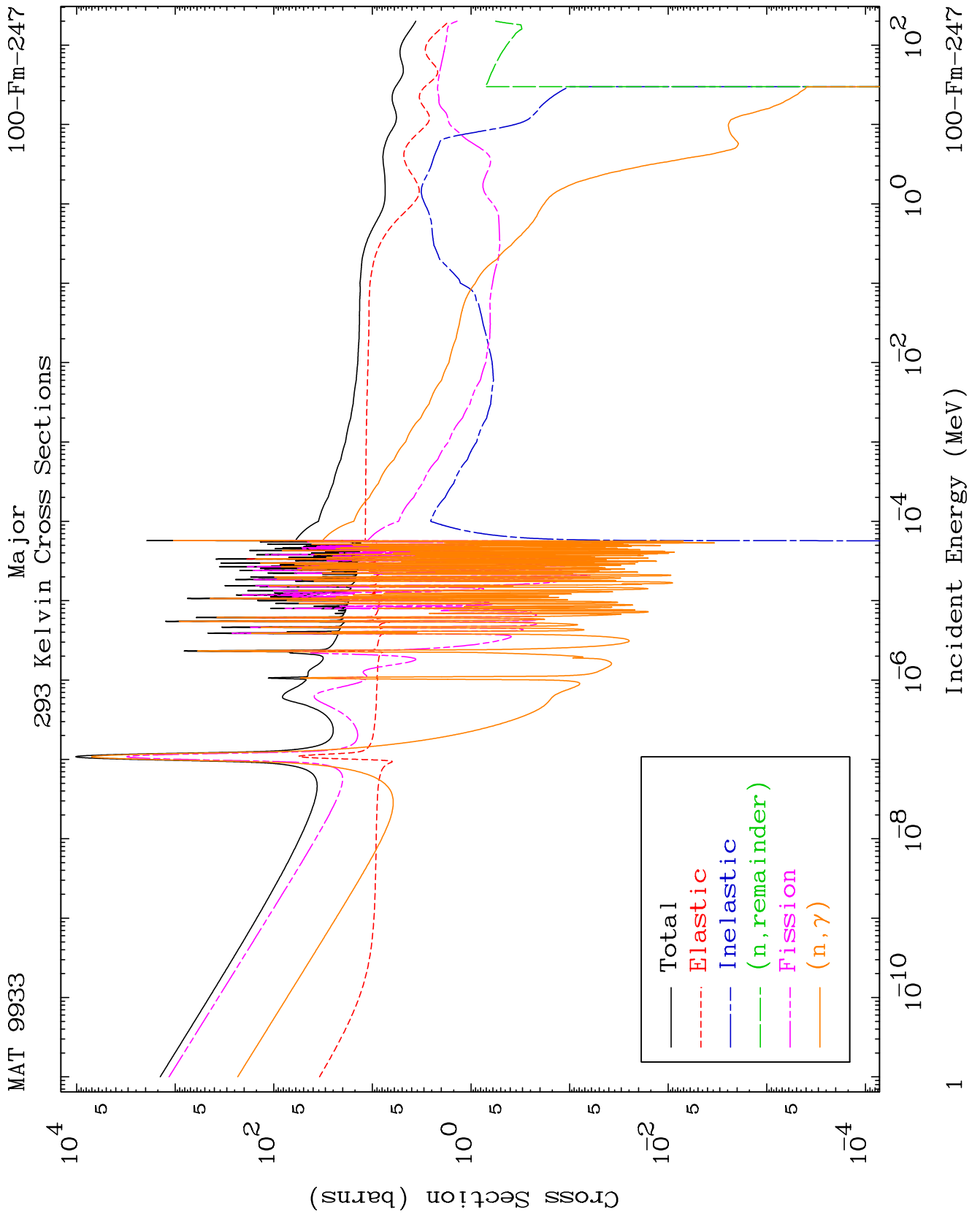
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

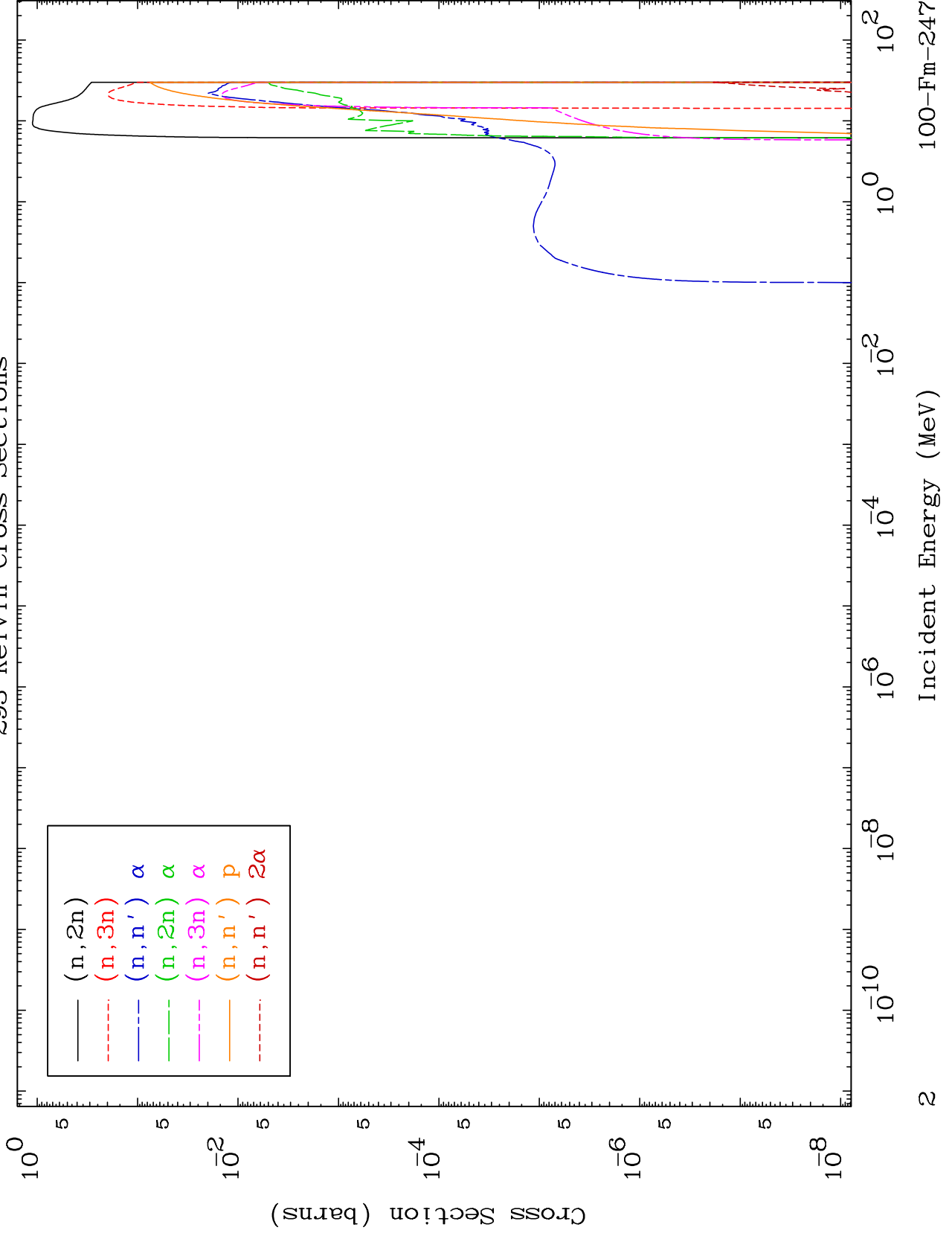
Press Mouse Button to Start



MAT 9933

Neutron Production
293 Kelvin Cross Sections

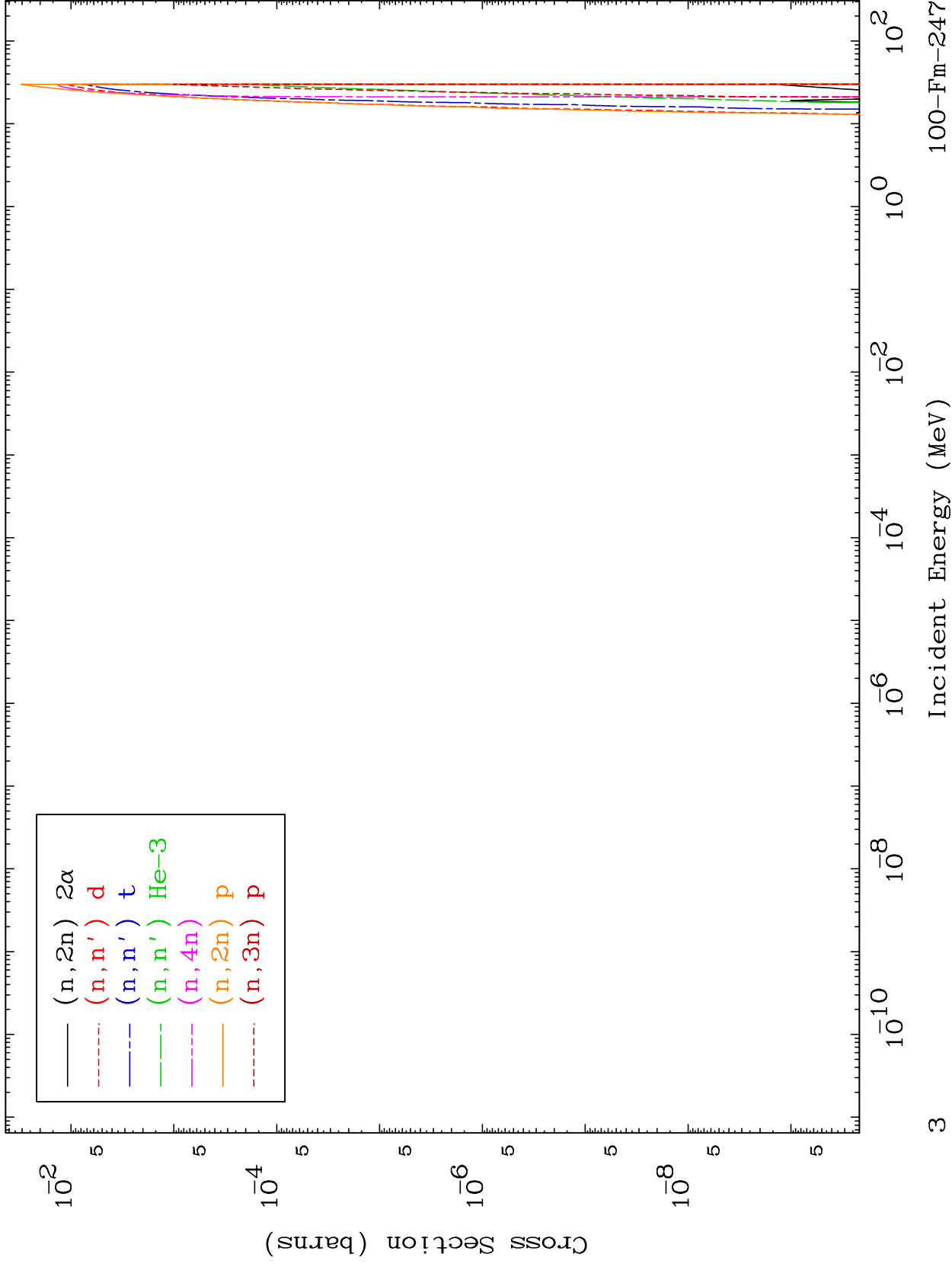
100-Fm-247



MAT 9933

Neutron Production
293 Kelvin Cross Sections

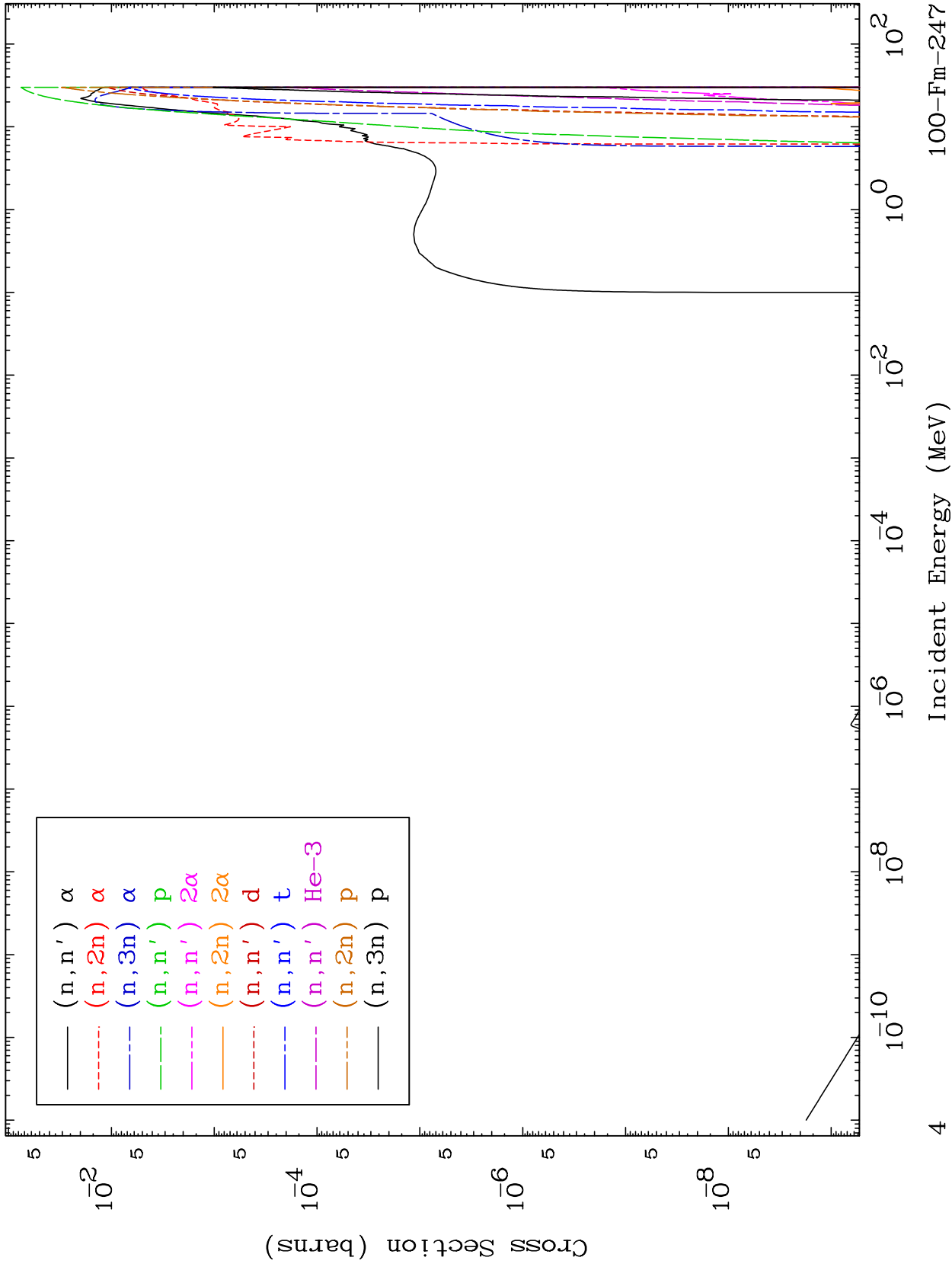
100-Fm-247



MAT 9933

Charged Particle
293 Kelvin Cross Sections

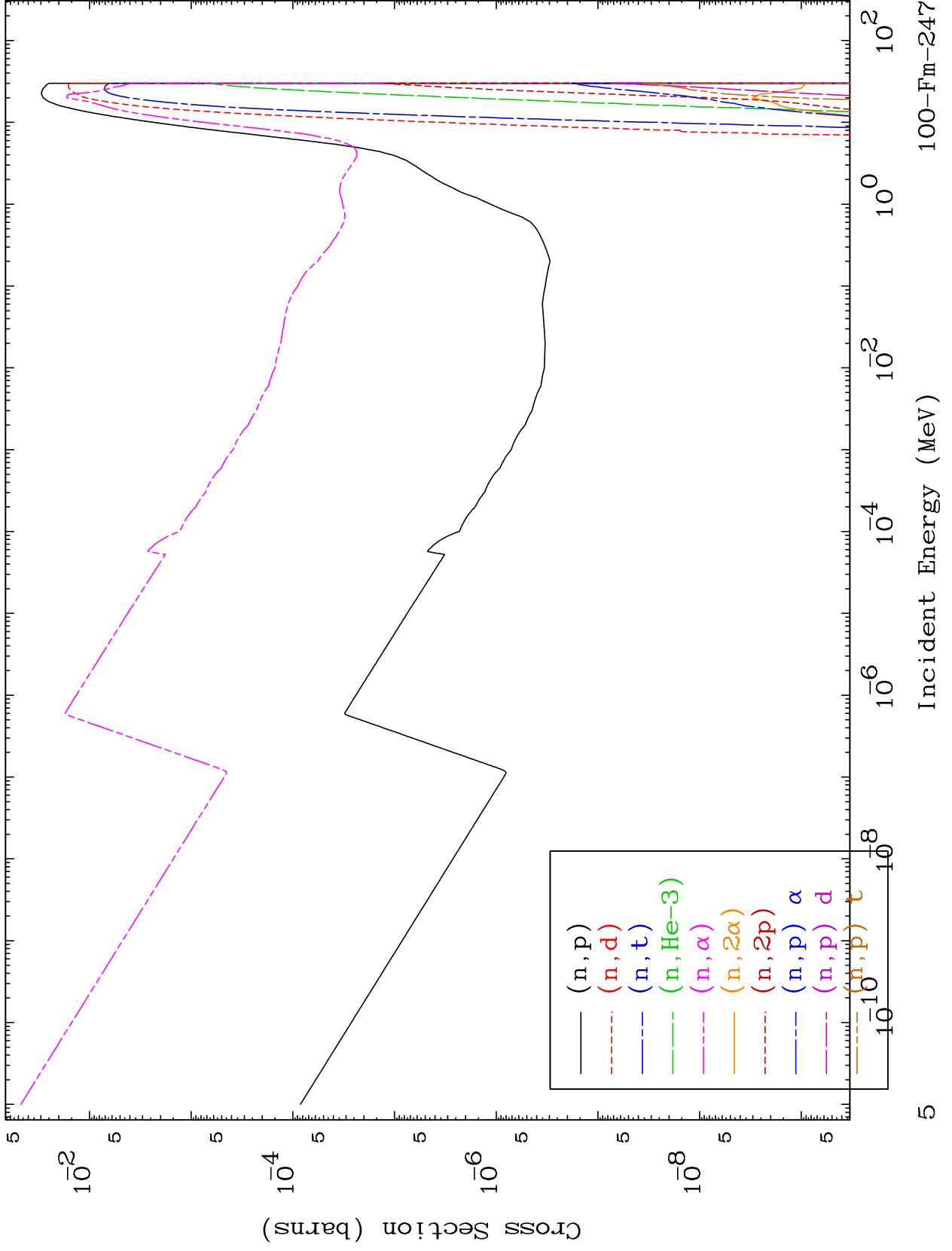
100-Fm-247



MAT 9933

Charged Particle
293 Kelvin Cross Sections

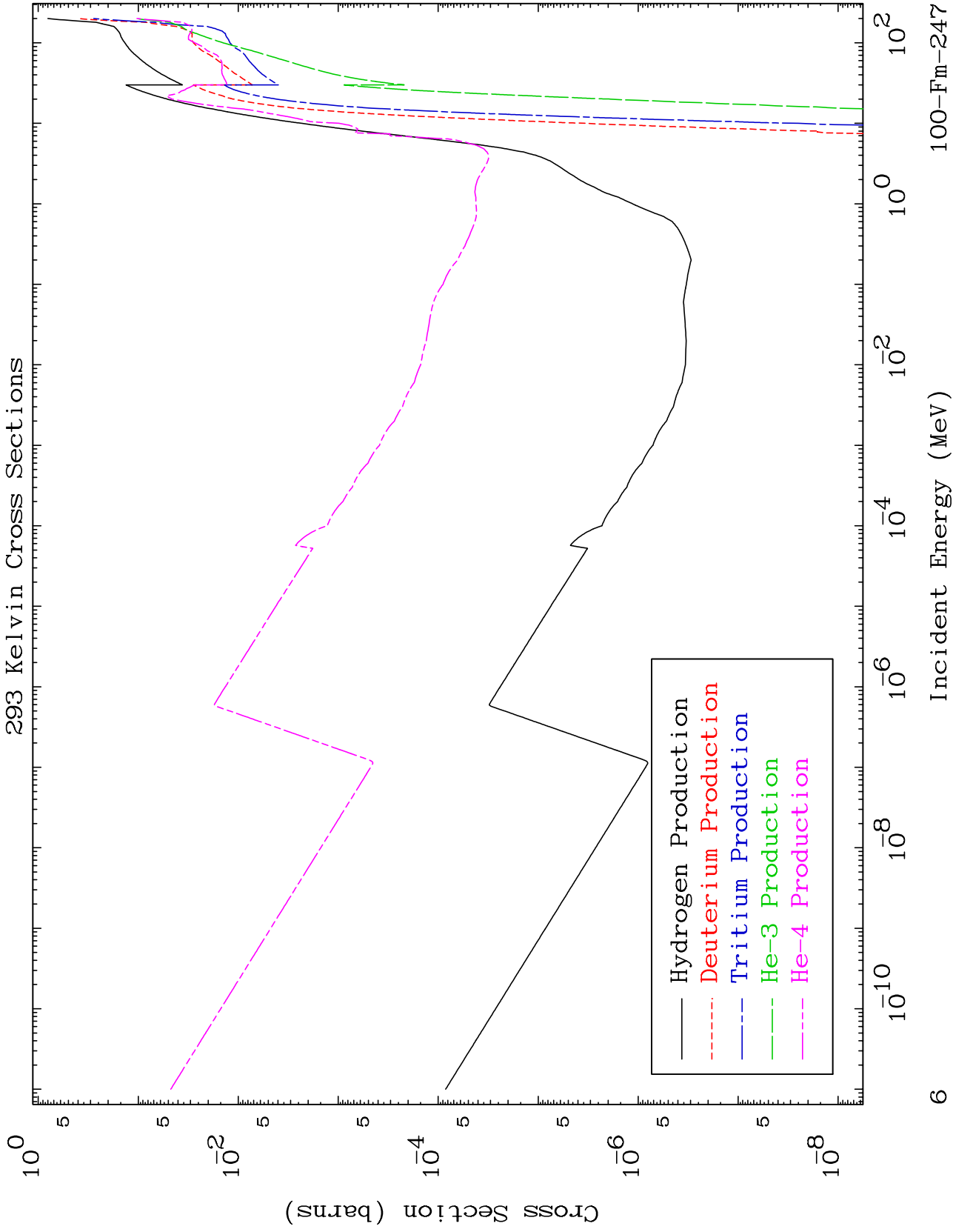
100-Fm-247



MAT 9933

Particle Production
293 Kelvin Cross Sections

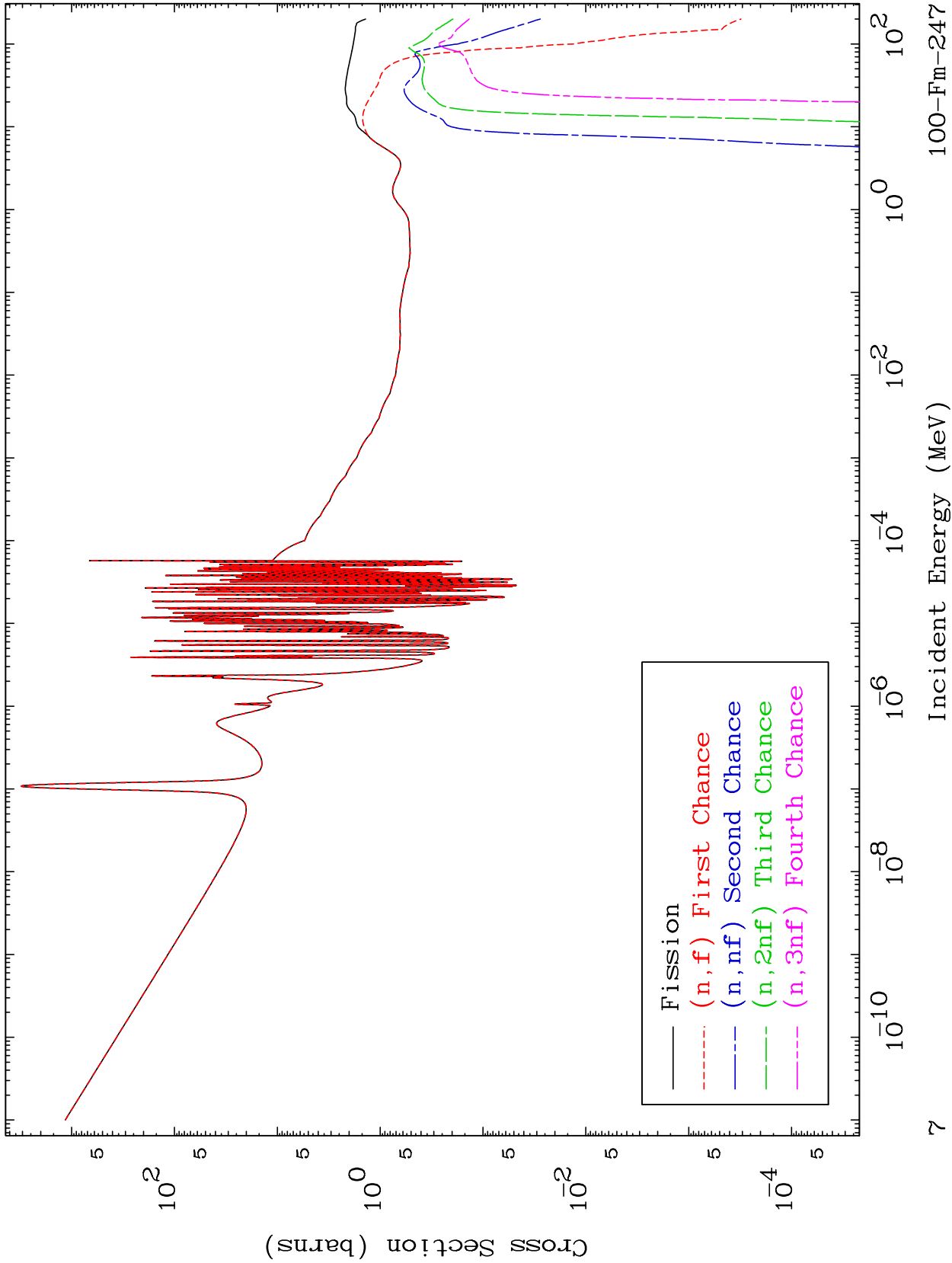
100-Fm-247



MAT 9933

Fission
293 Kelvin Cross Sections

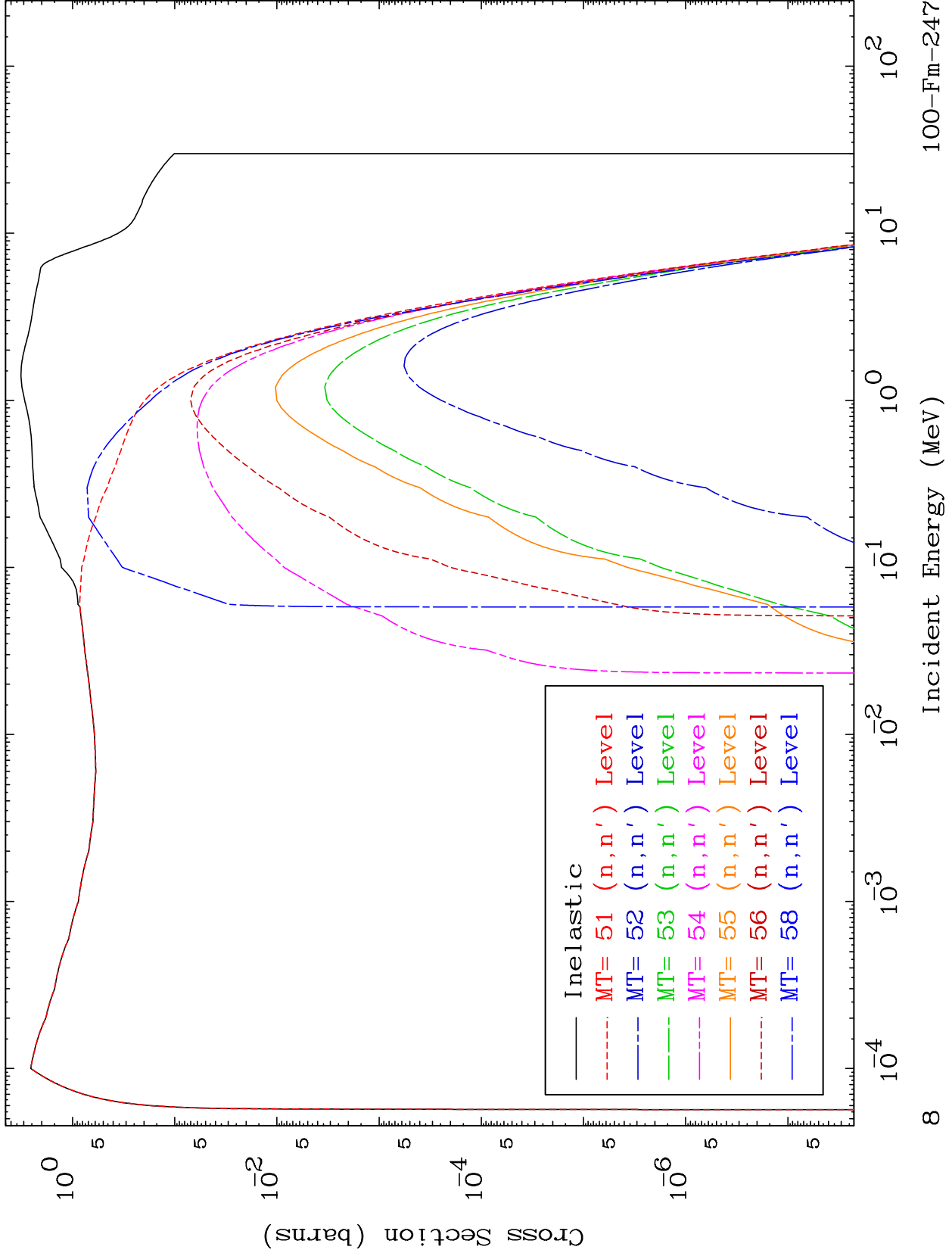
100-Fm-247



MAT 9933

(n,n') Level
293 Kelvin Cross Sections

100-Fm-247

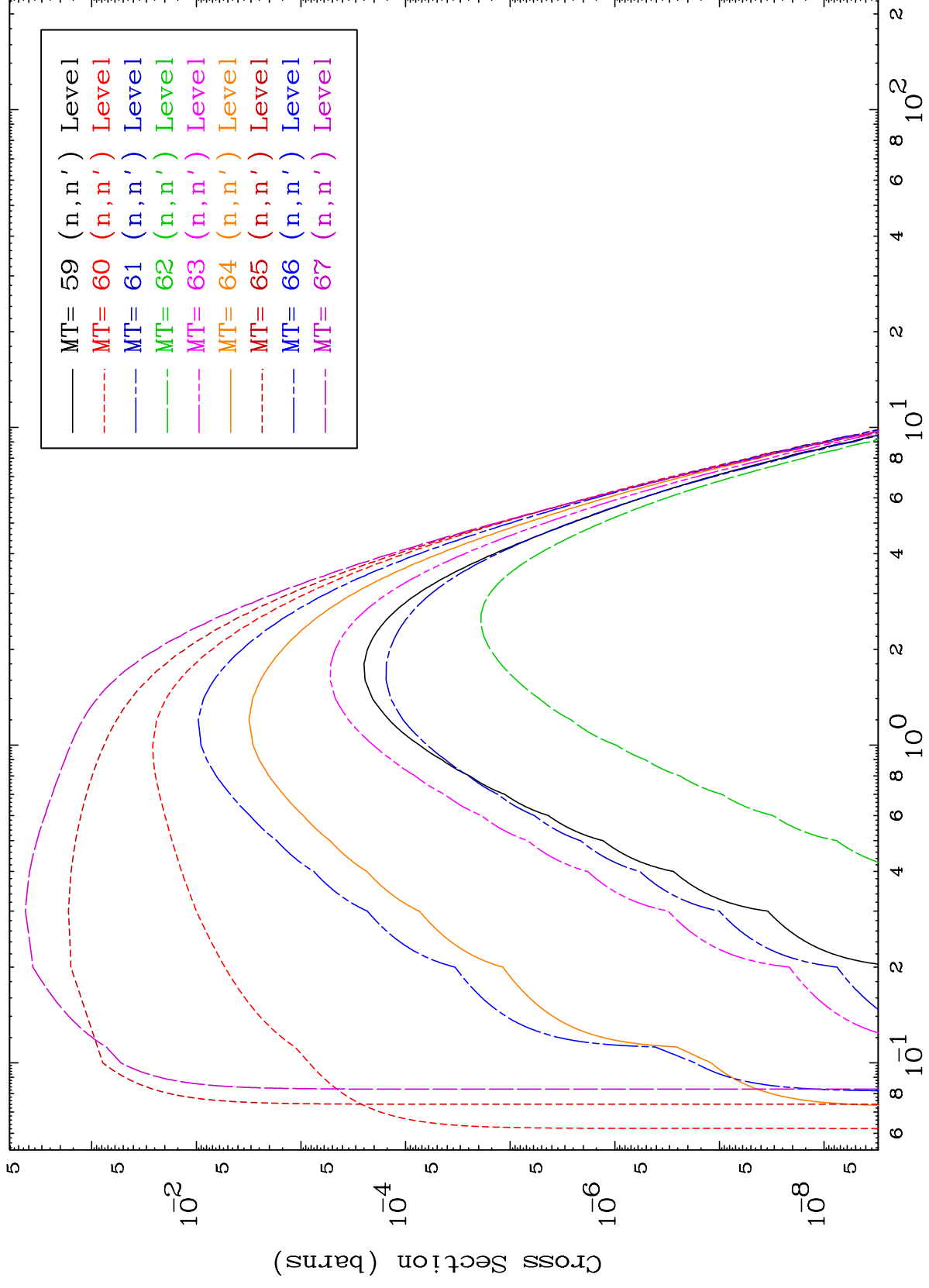


MAT 9933

(n,n') Level

100-Fm-247

293 Kelvin Cross Sections



9

Incident Energy (MeV)

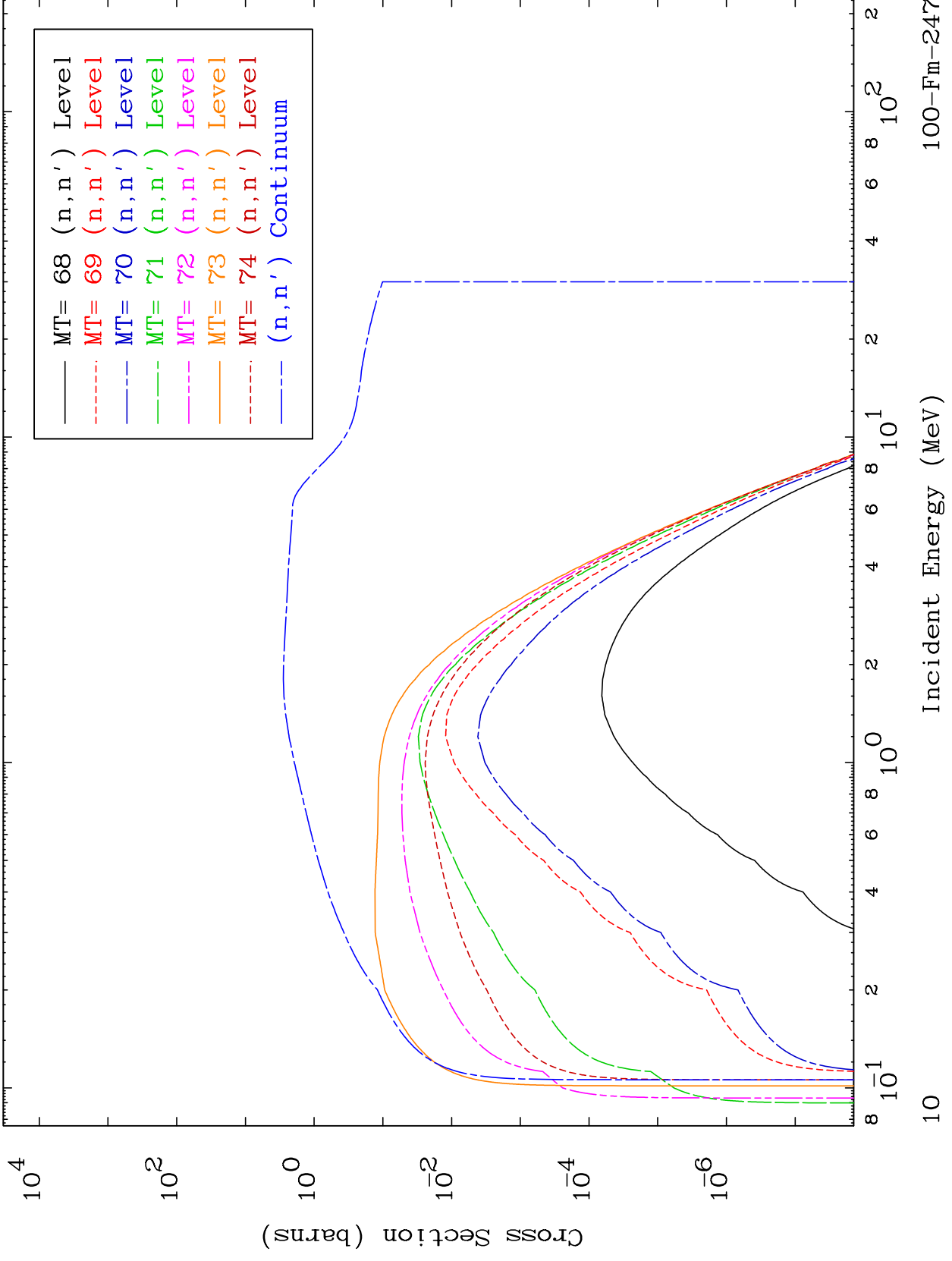
100-Fm-247

MAT 9933

(n,n') Level

100-Fm-247

293 Kelvin Cross Sections



100-Fm-247

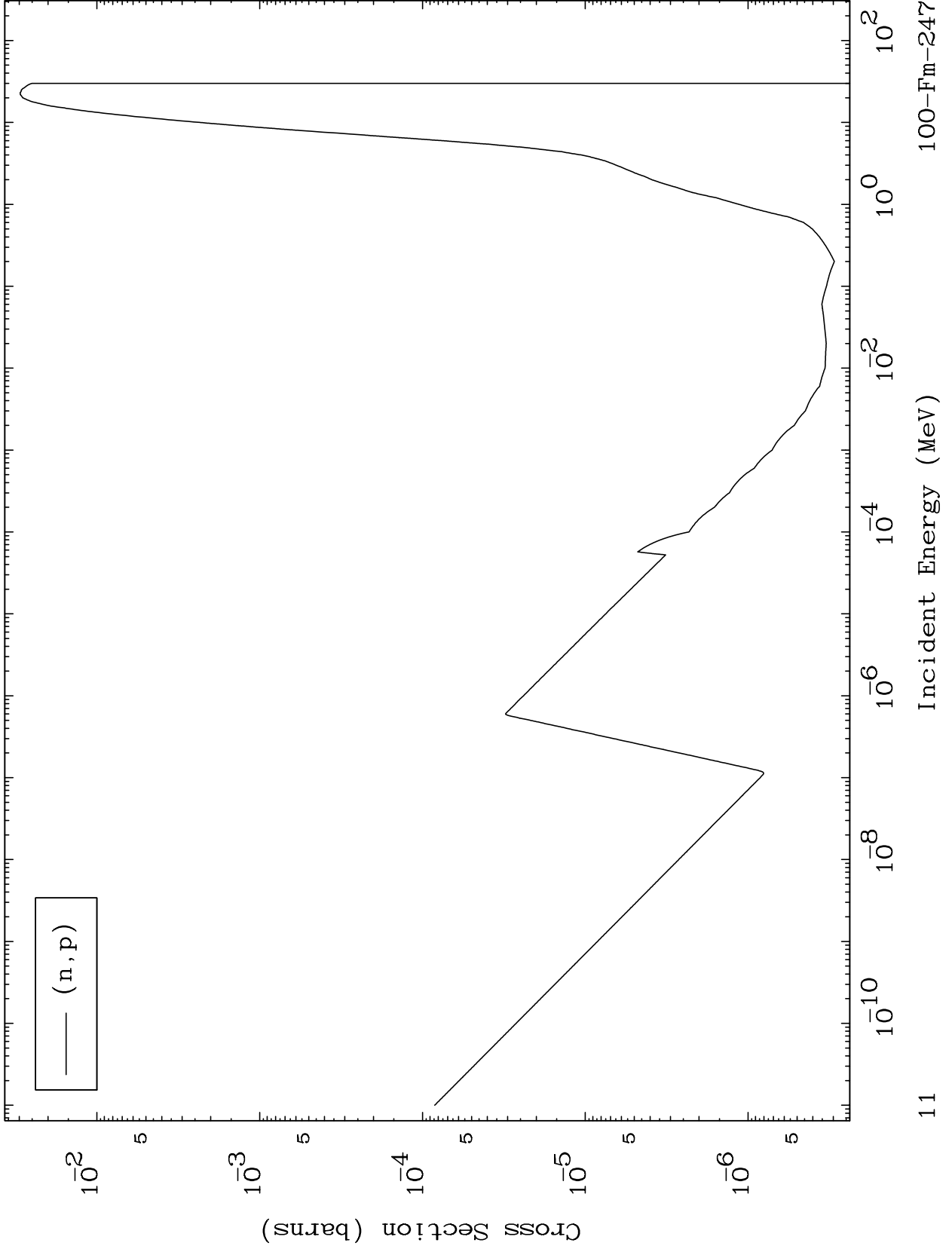
Incident Energy (MeV)

10

MAT 9933

(n,p) Levels
293 Kelvin Cross Sections

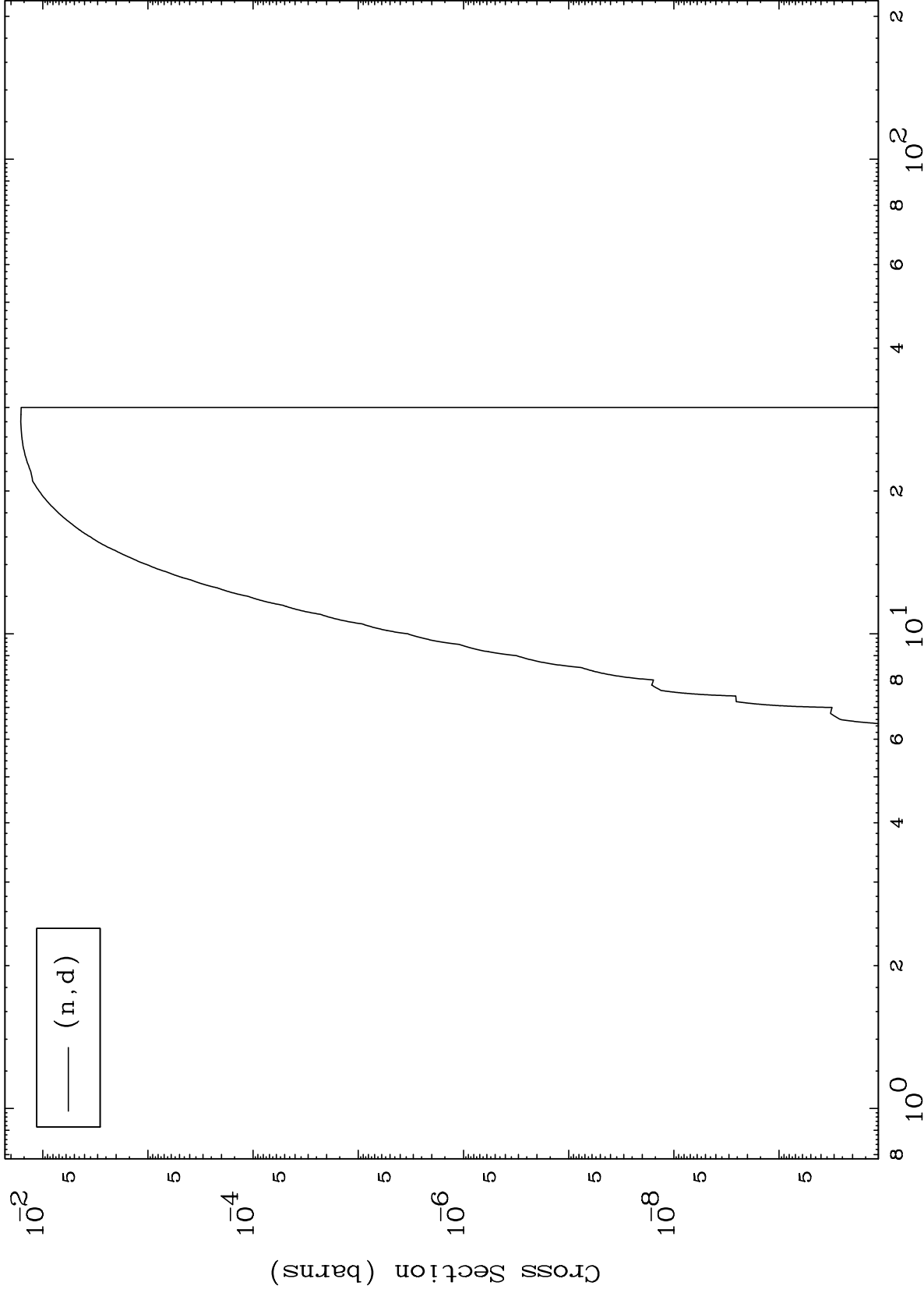
100-Fm-247



MAT 9933

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-247



12

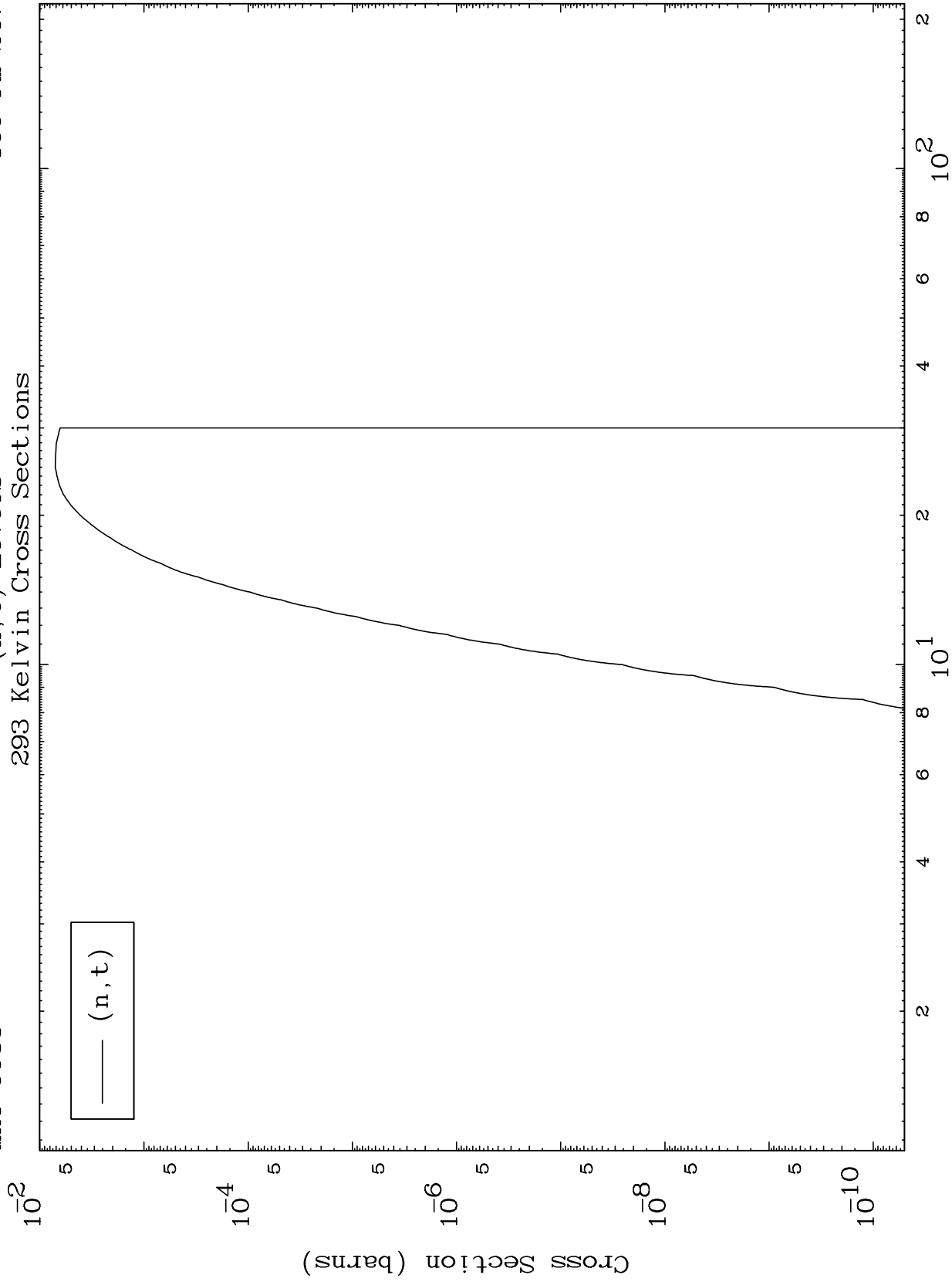
Incident Energy (MeV)

100-Fm-247

MAT 9933

(n,t) Levels
293 Kelvin Cross Sections

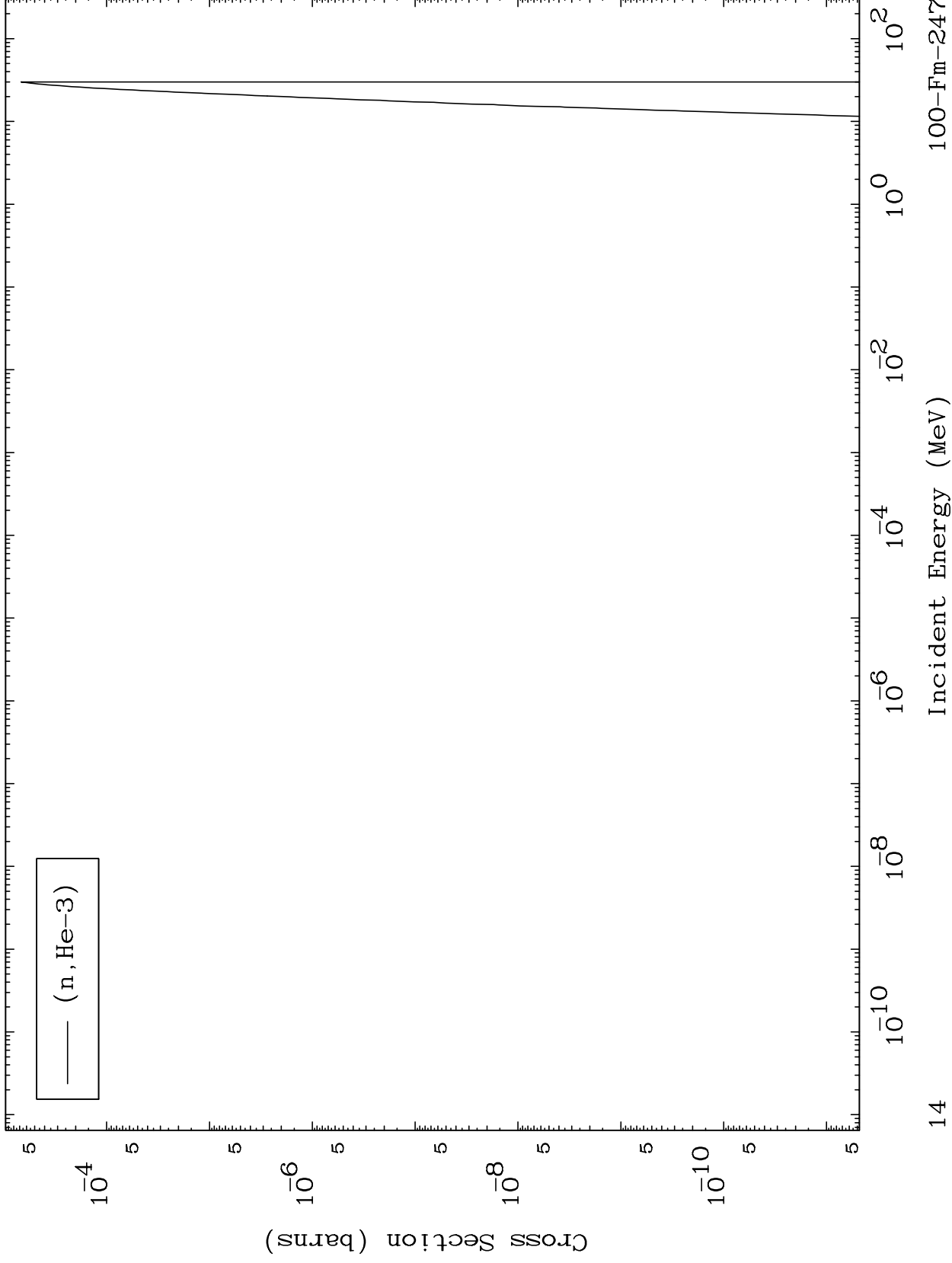
100-Fm-247



MAT 9933

(n,He3) Levels
293 Kelvin Cross Sections

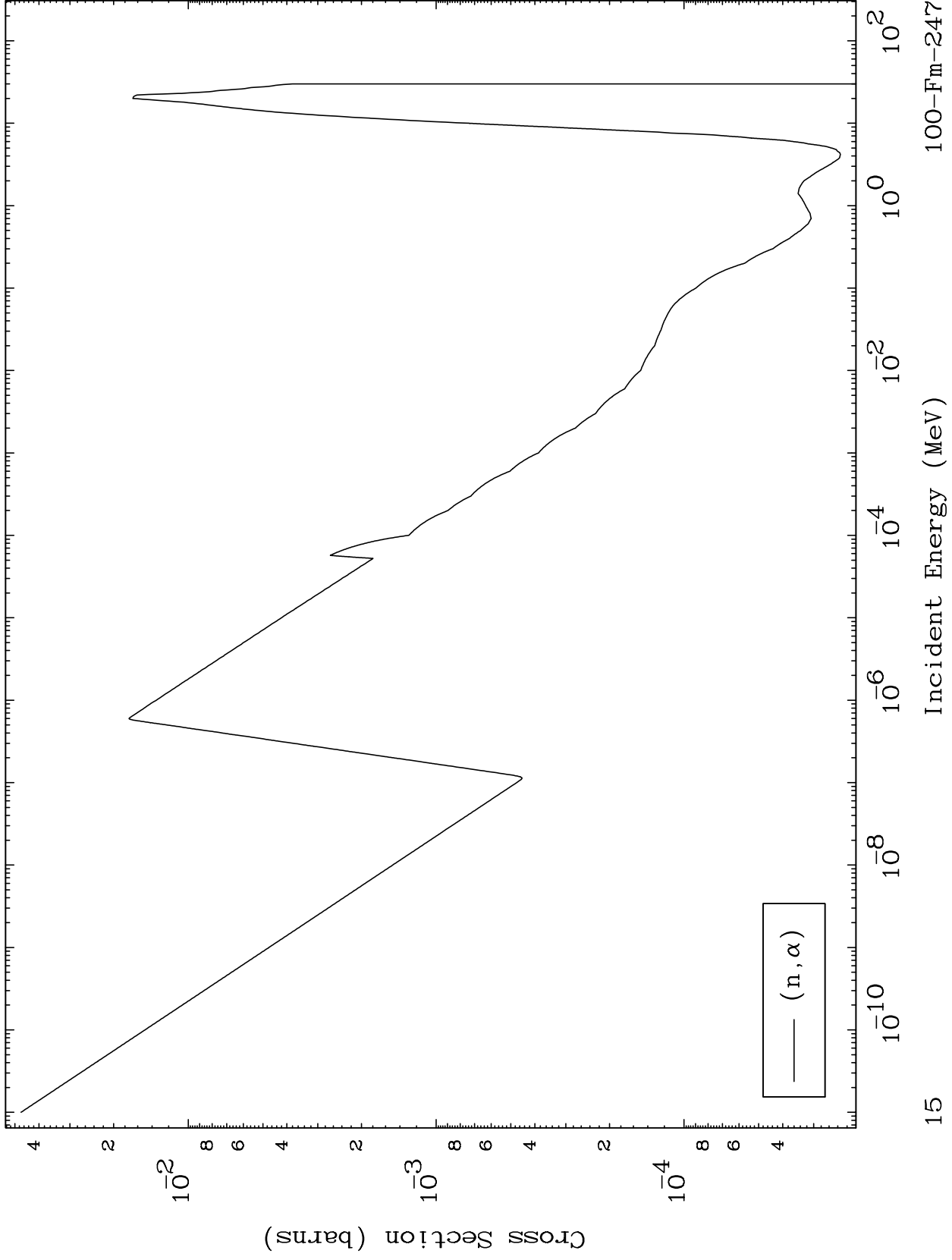
100-Fm-247

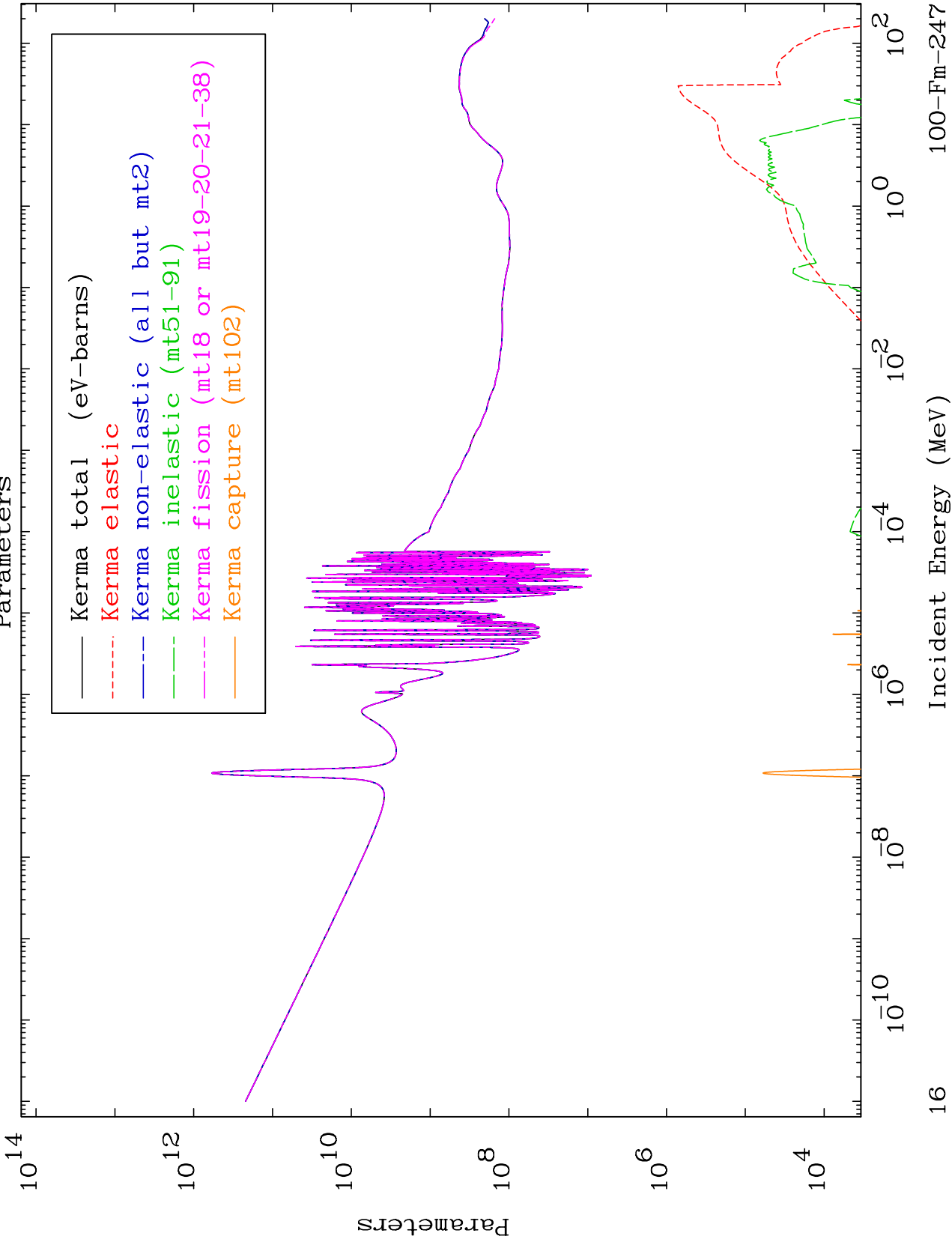


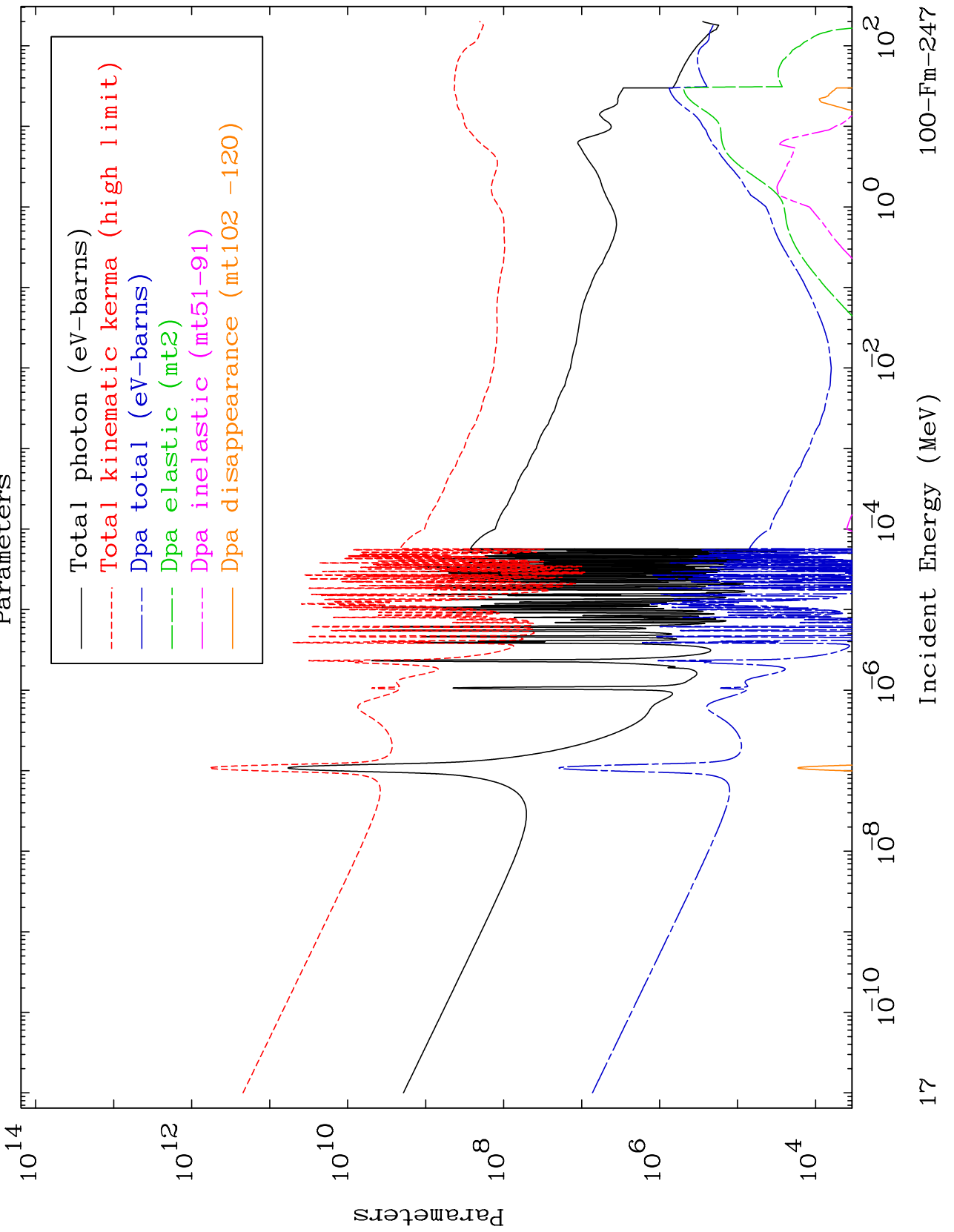
MAT 9933

(n,α) Levels
293 Kelvin Cross Sections

100-Fm-247



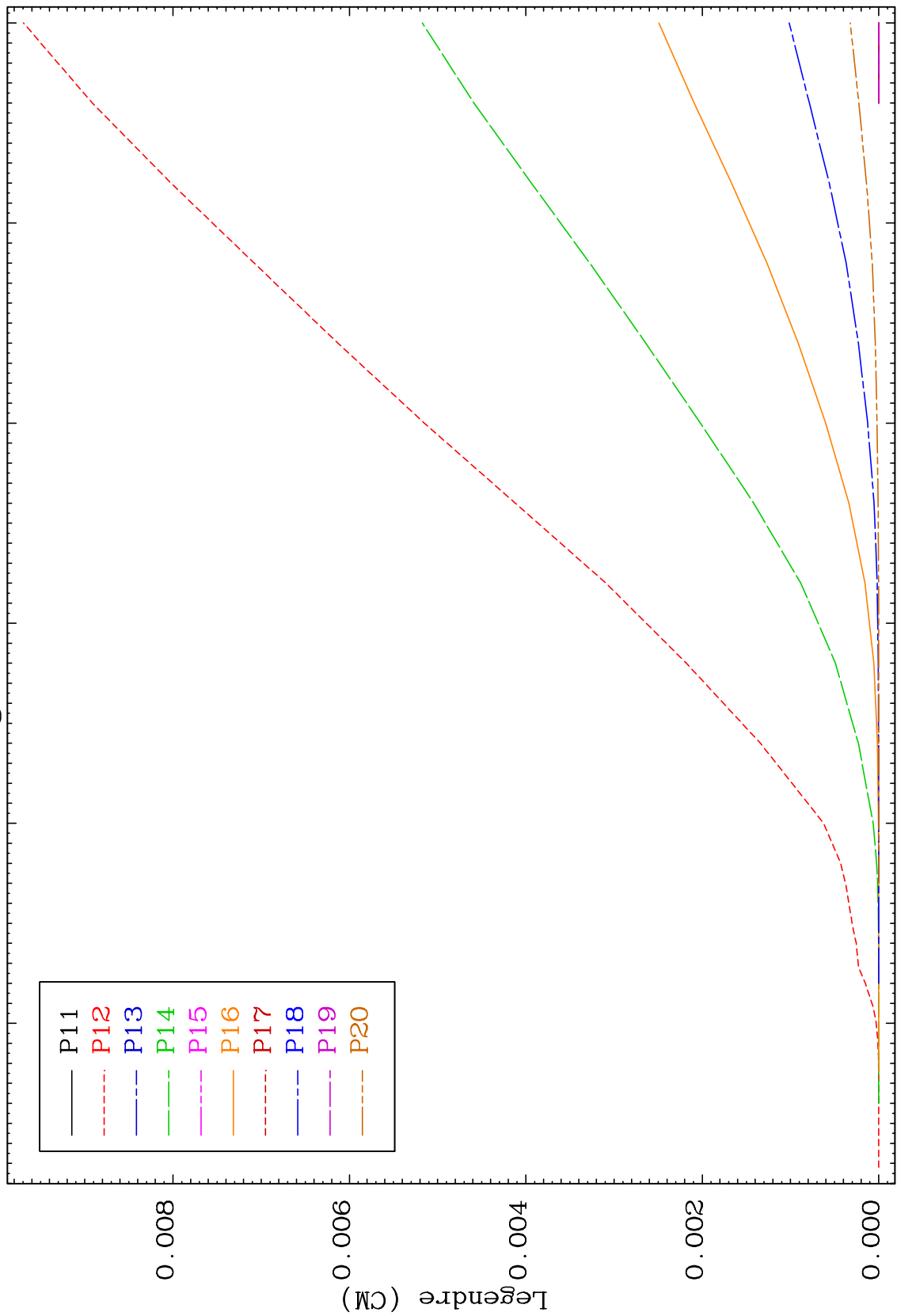




MAT 9933

Elastic Legendre Coefficients

100-Fm-247



19

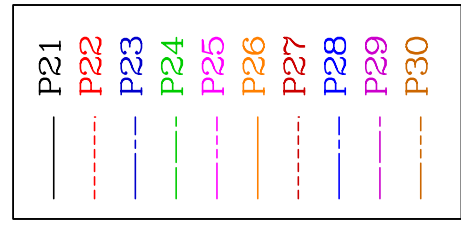
100-Fm-247

Incident Energy (MeV)

MAT 9933

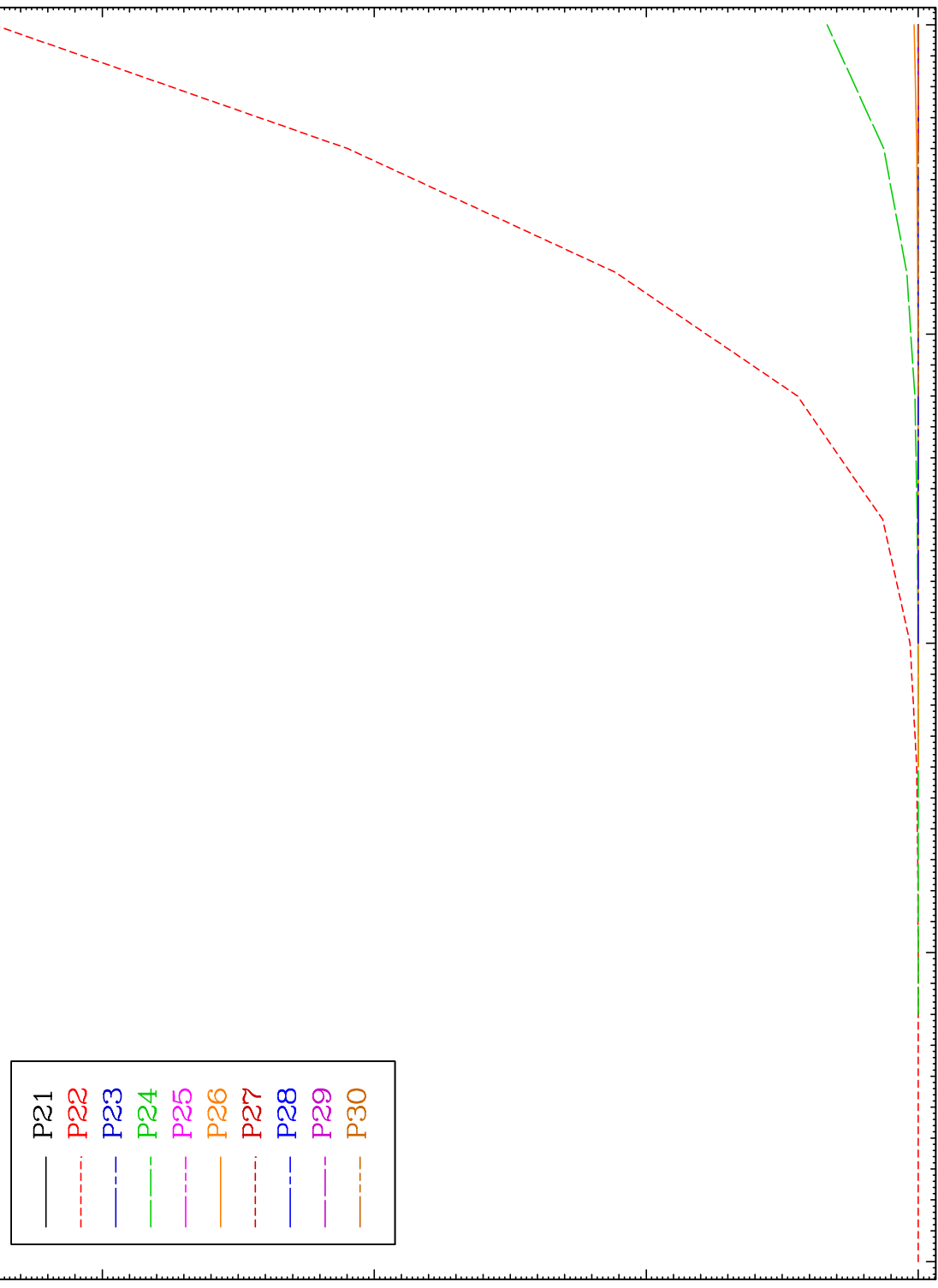
Elastic
Legendre Coefficients

100-Fm-247



$\times 10^{-5}$

Legendre (CM)



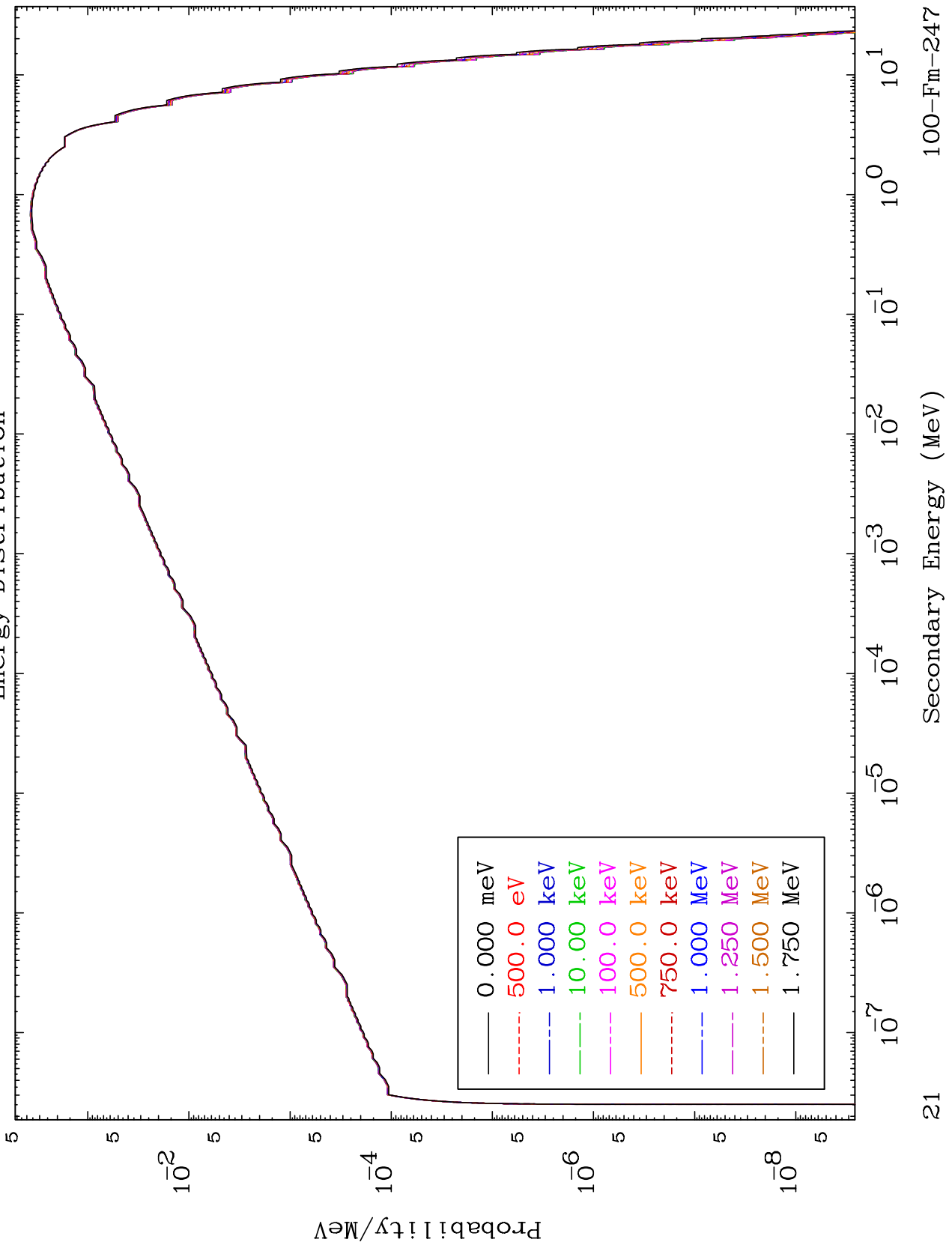
Incident Energy (MeV)

100-Fm-247

MAT 9933

Fission Energy Distribution

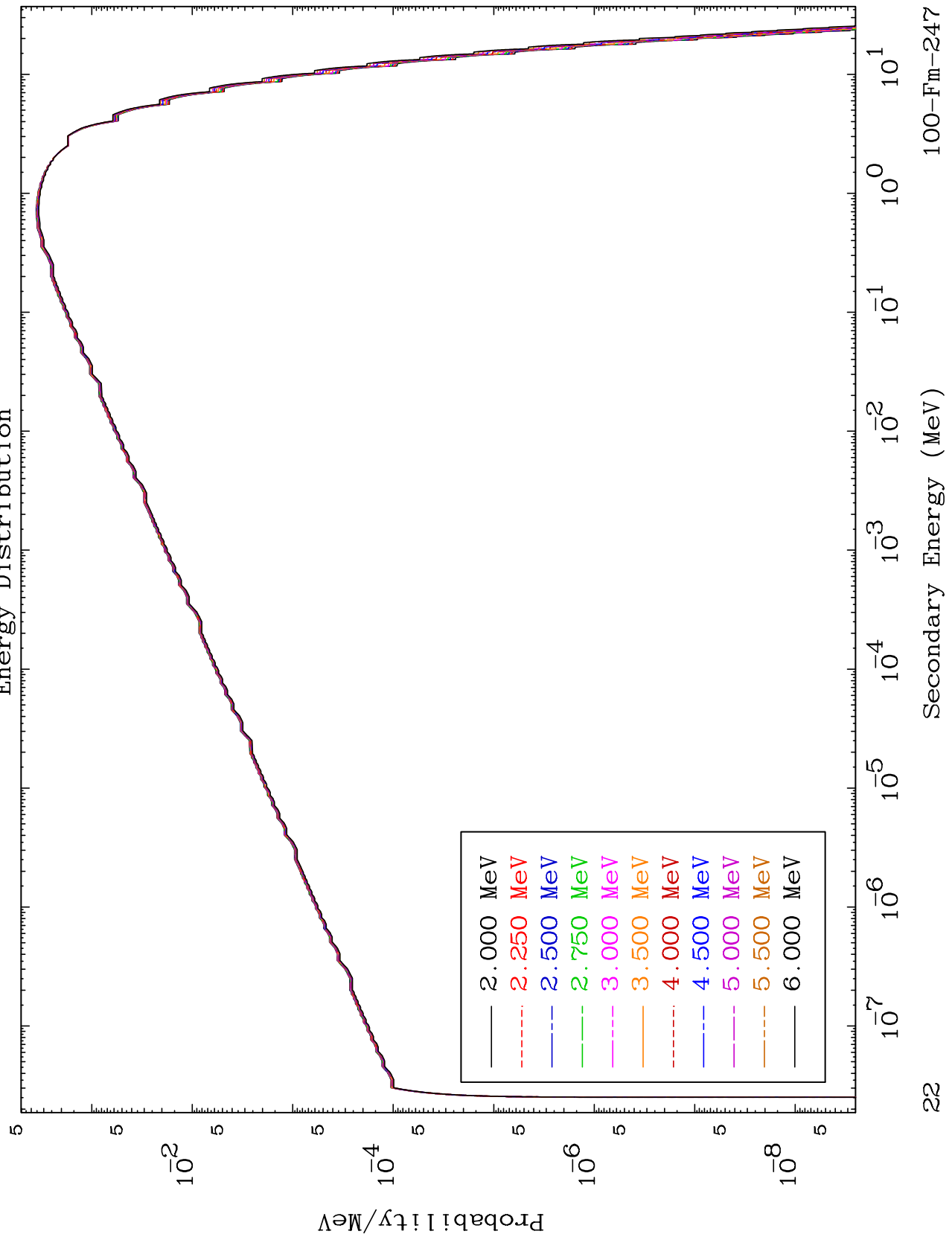
100-Fm-247



MAT 9933

Fission Energy Distribution

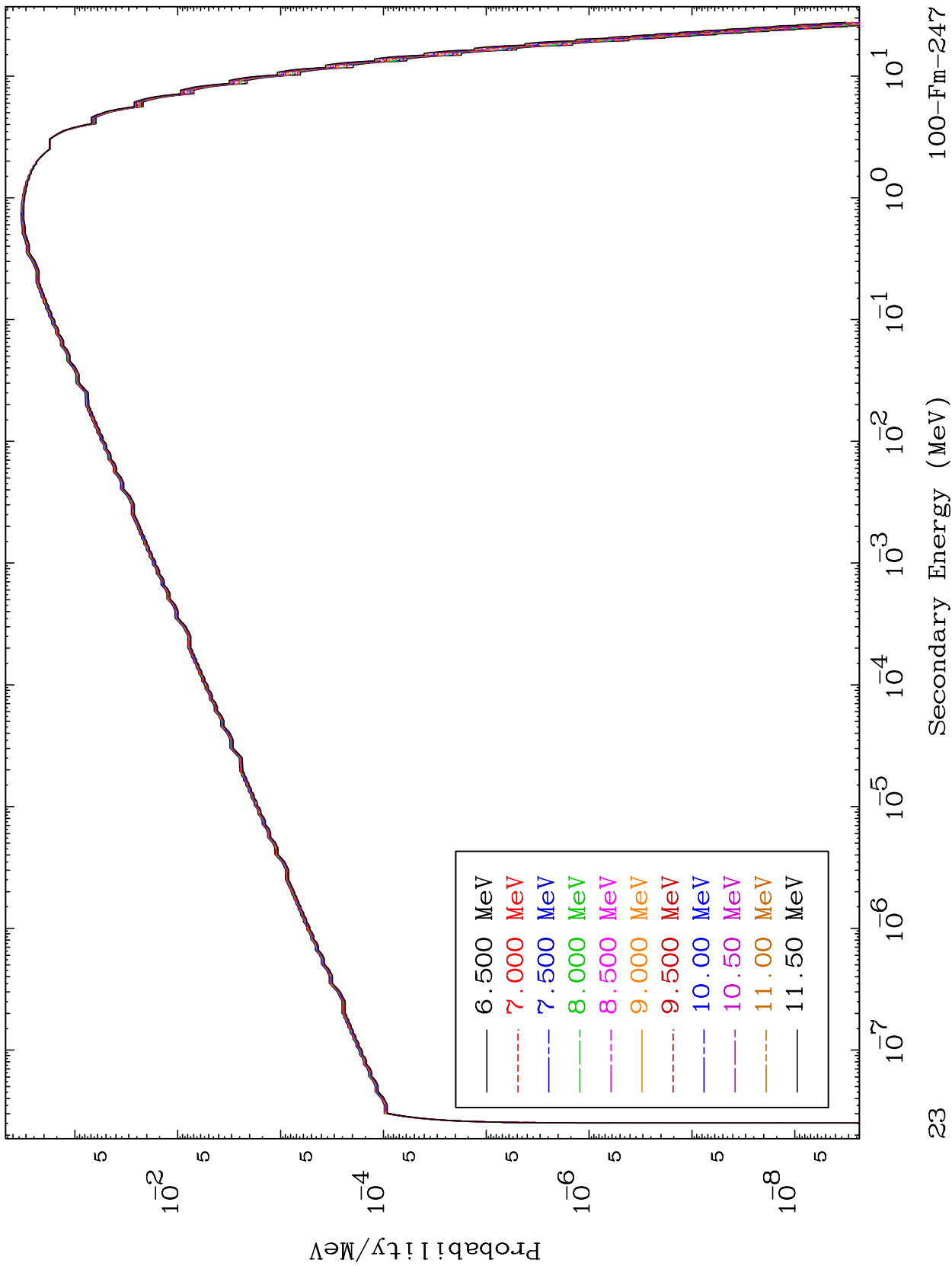
100-Fm-247



MAT 9933

Fission Energy Distribution

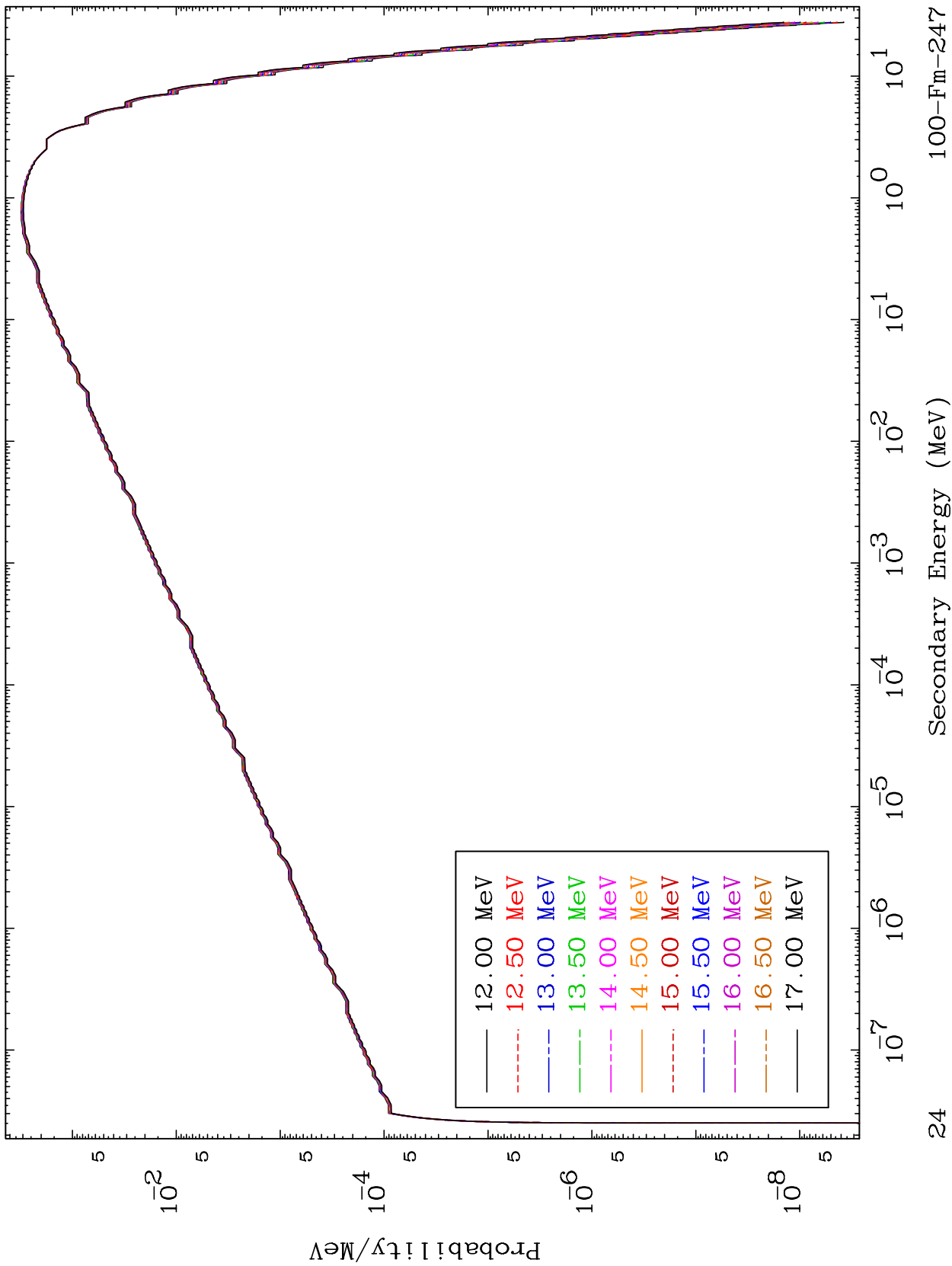
100-Fm-247



MAT 9933

Fission Energy Distribution

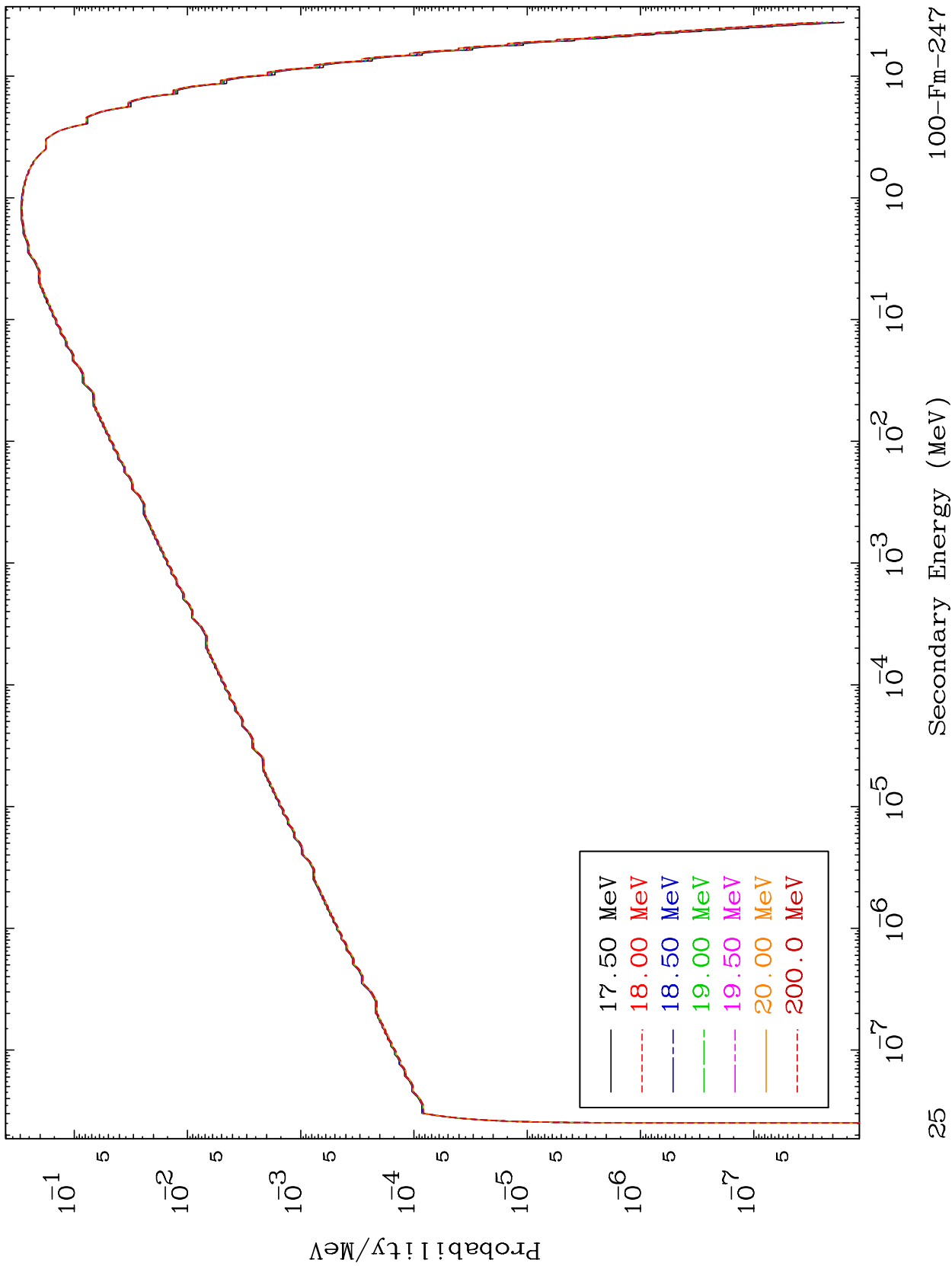
100-Fm-247



MAT 9933

Fission Energy Distribution

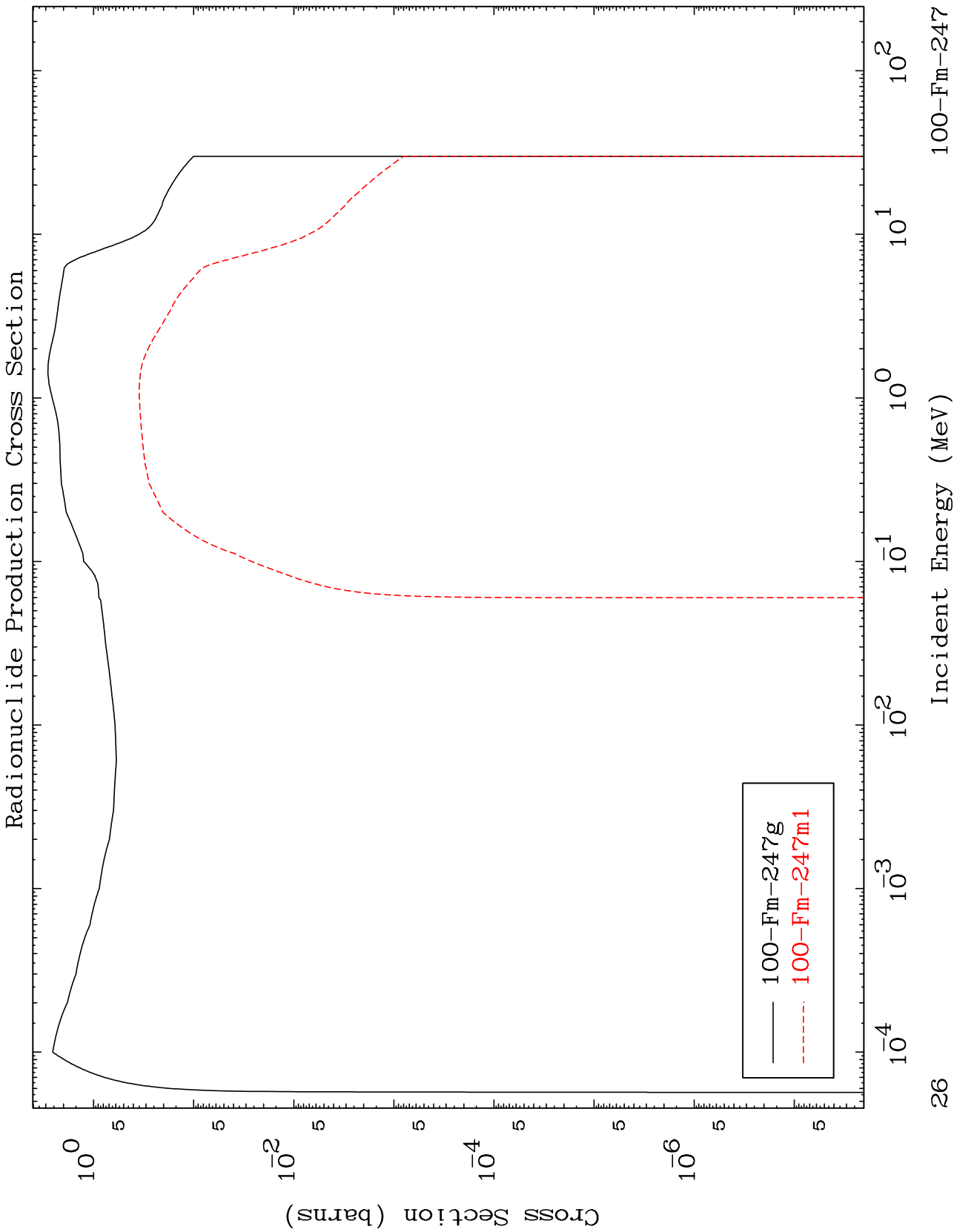
100-Fm-247



MAT 9933

100-Fm-247

Inelastic
Radionuclide Production Cross Section



26