

Program EVALPLOT
(Version 2017-1)

by

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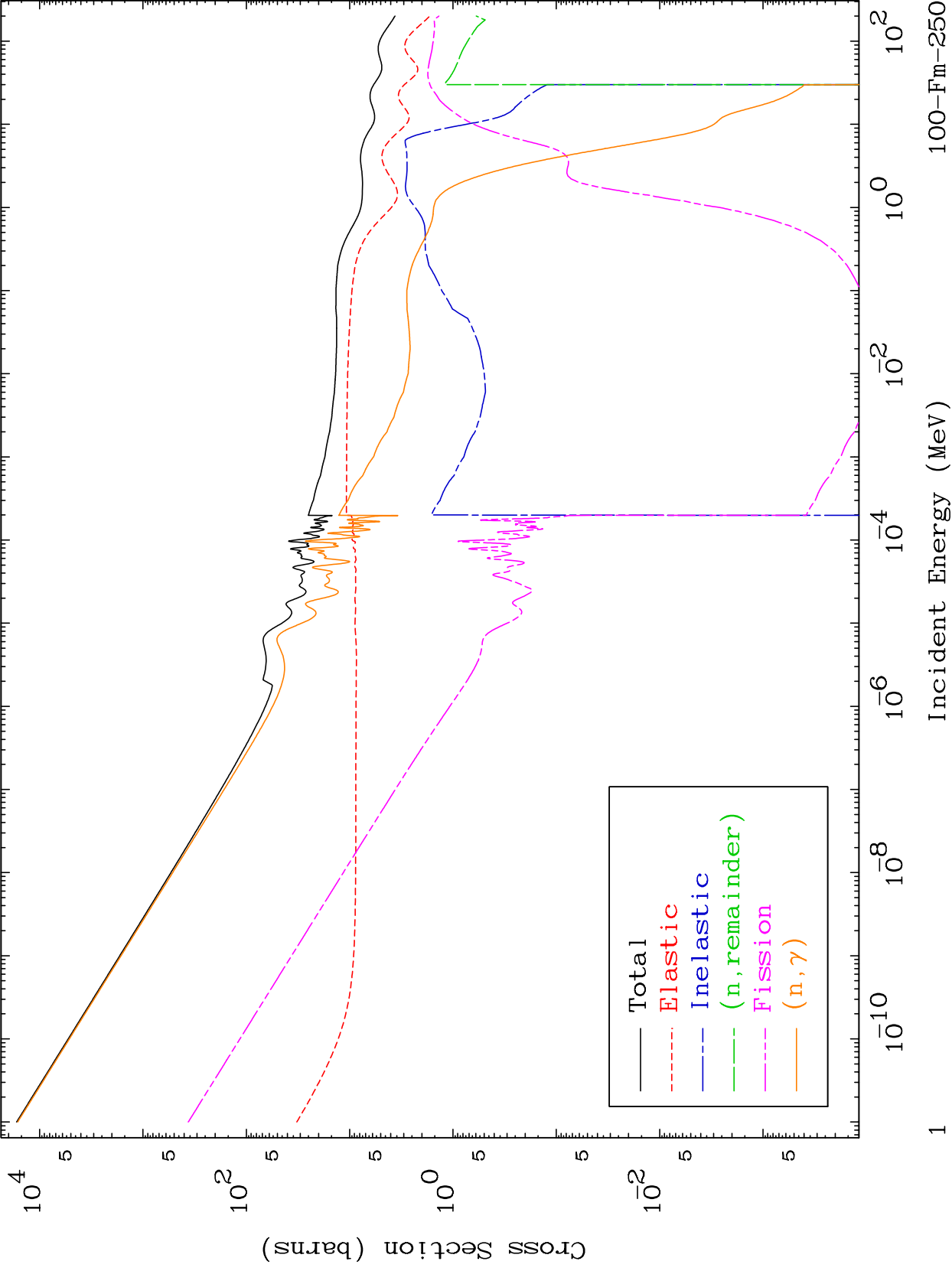
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9936

Major
293 Kelvin Cross Sections

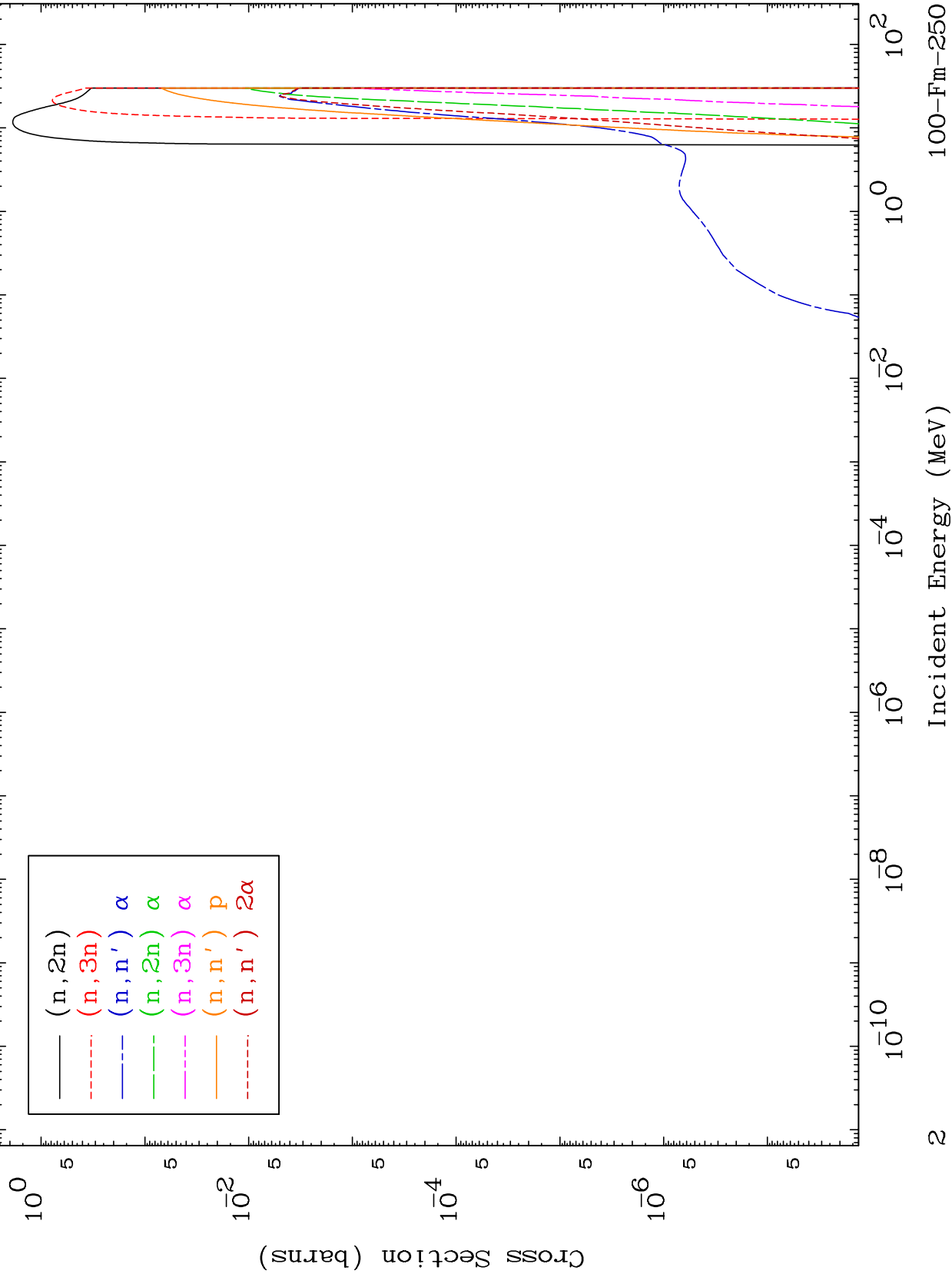
100-Fm-250



MAT 9936

Neutron Production
293 Kelvin Cross Sections

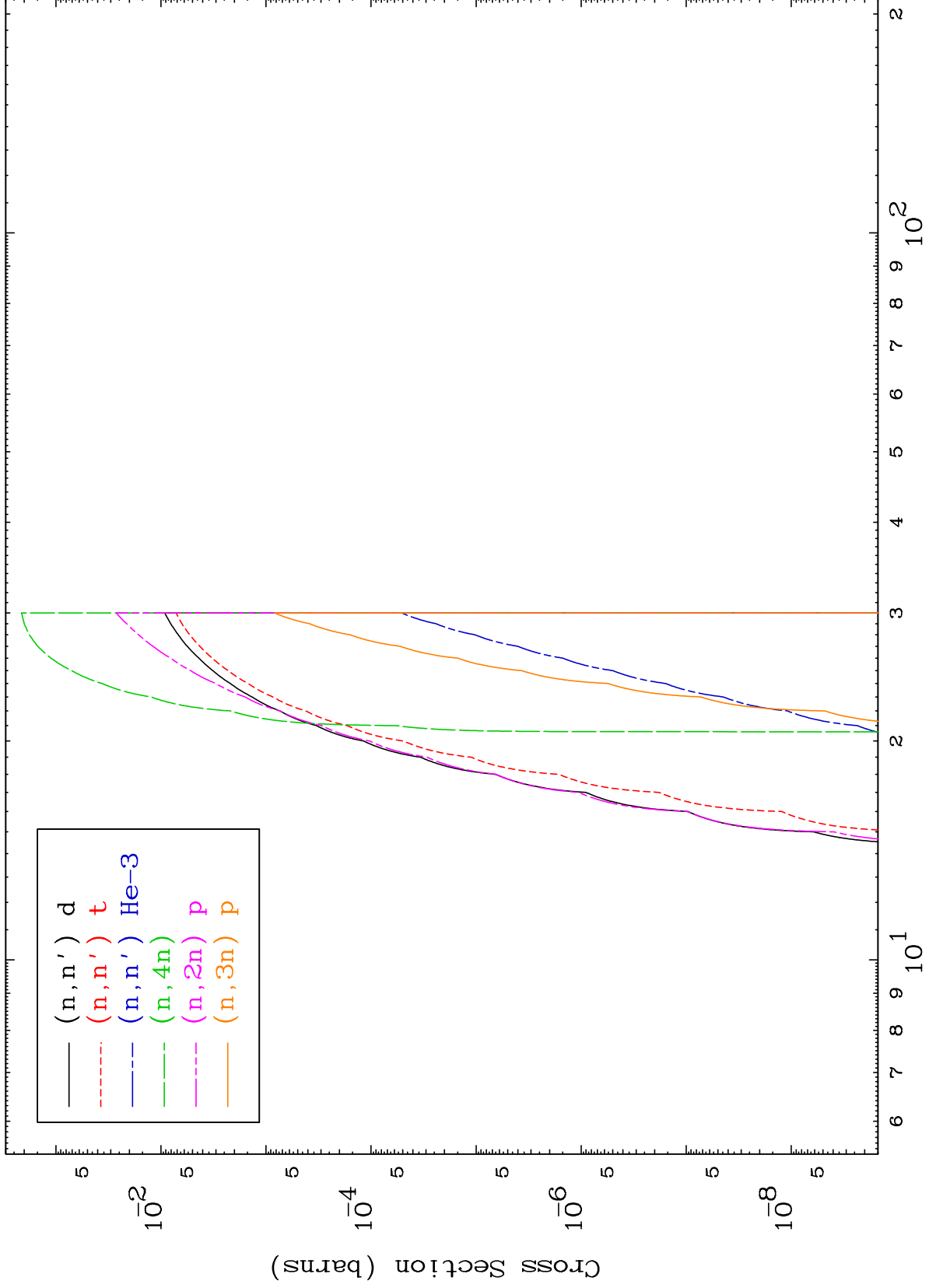
100-Fm-250



MAT 9936

Neutron Production
293 Kelvin Cross Sections

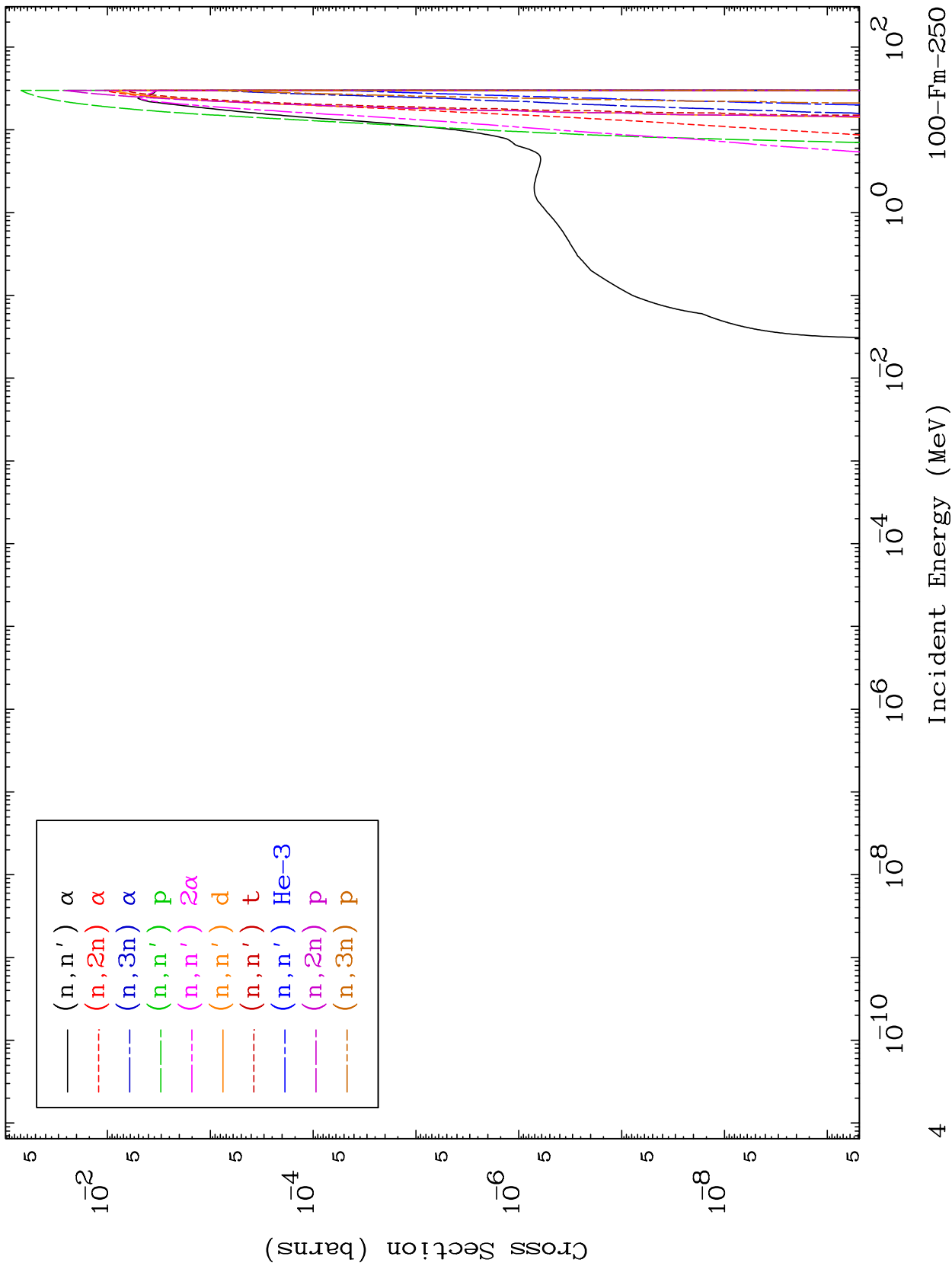
100-Fm-250



MAT 9936

Charged Particle
293 Kelvin Cross Sections

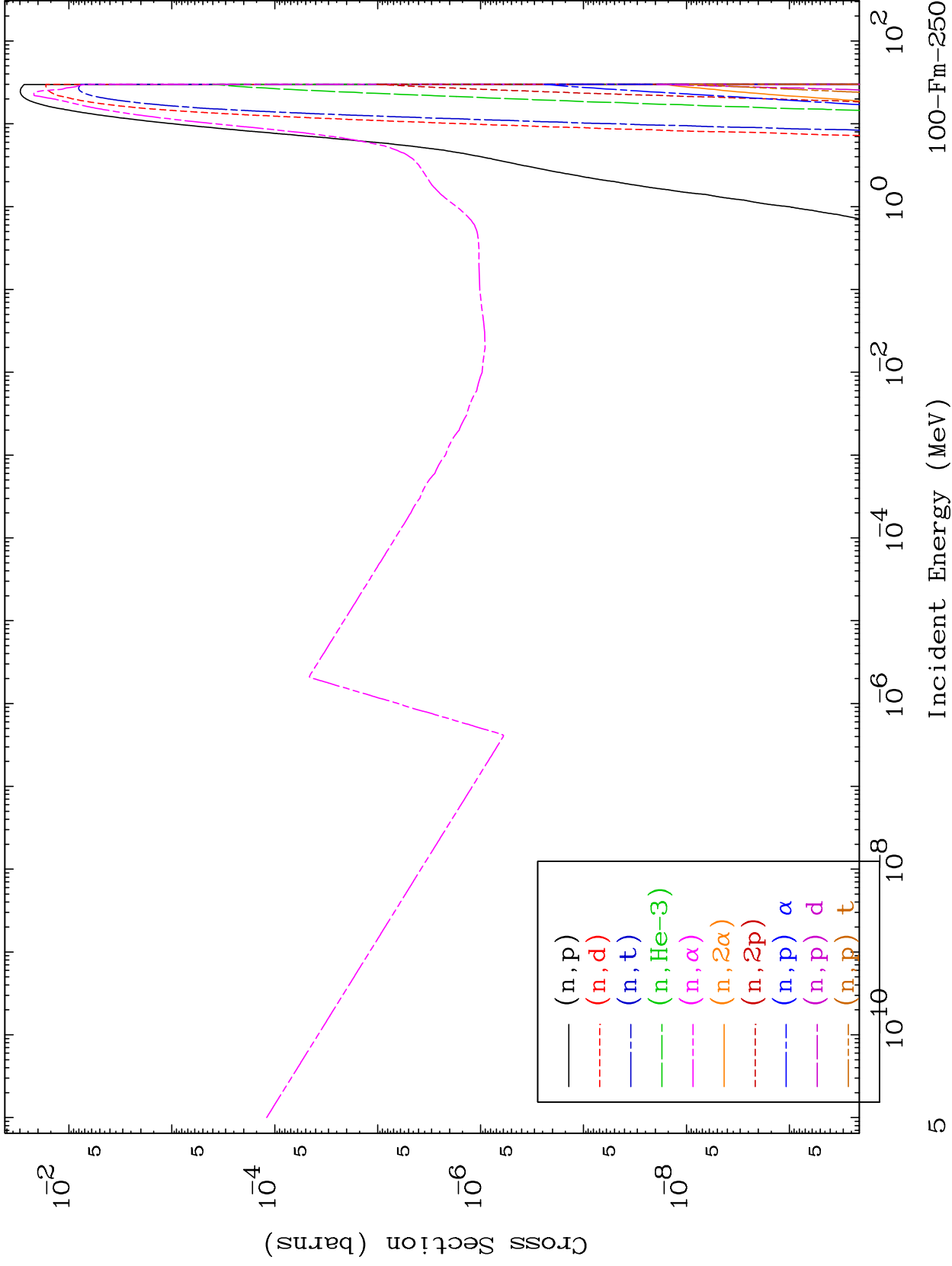
100-Fm-250



MAT 9936

Charged Particle
293 Kelvin Cross Sections

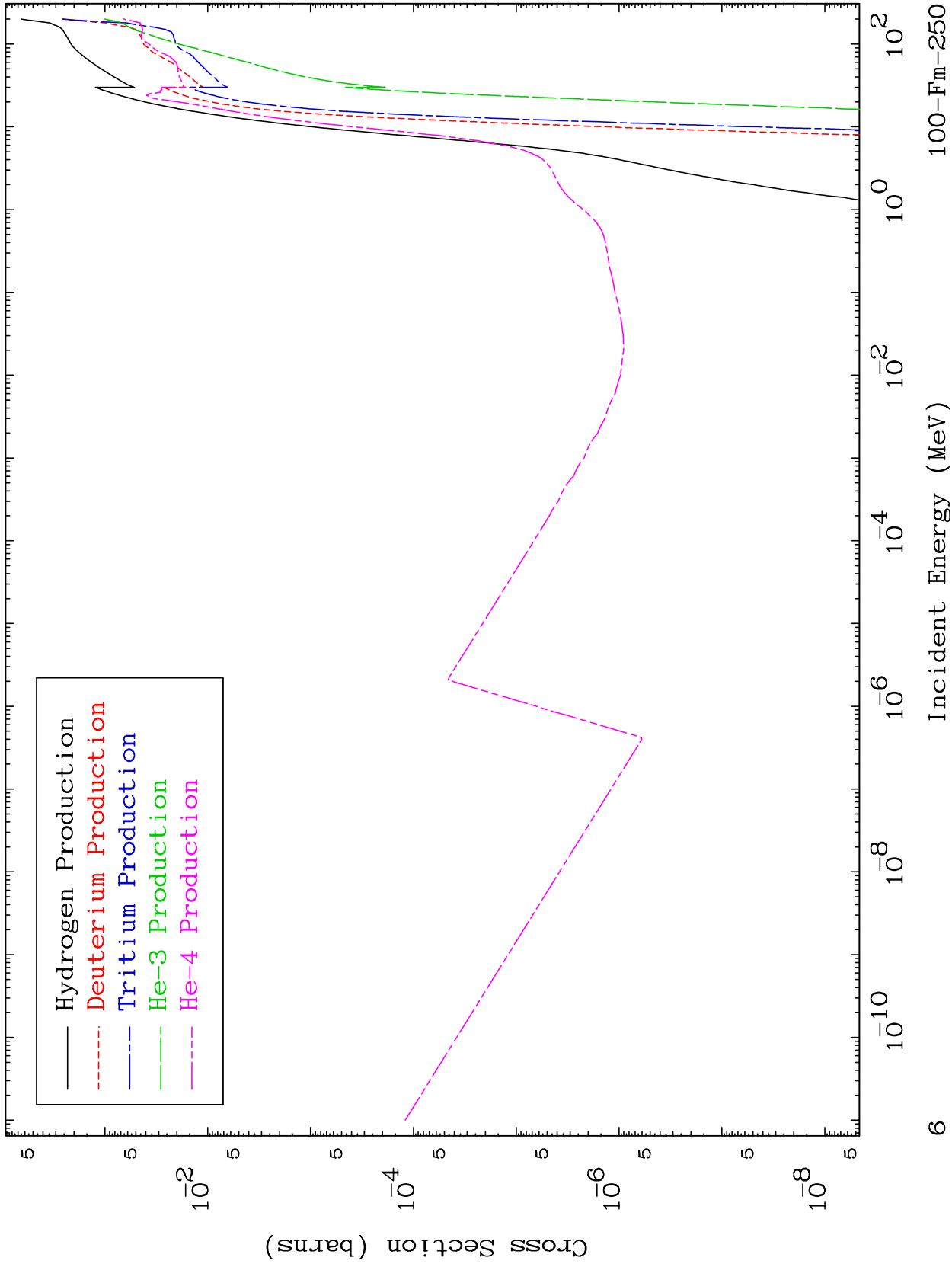
100-Fm-250



MAT 9936

Particle Production
293 Kelvin Cross Sections

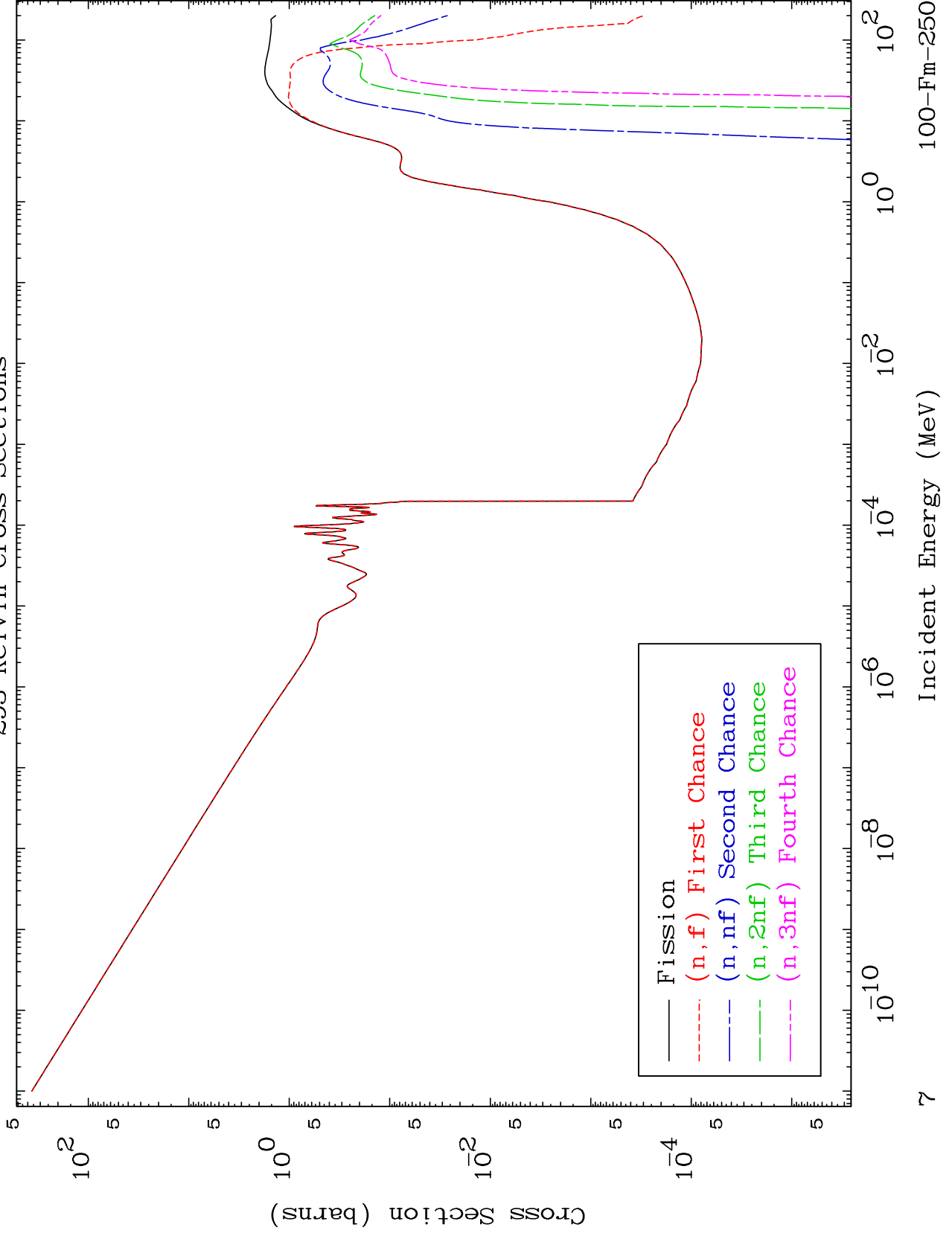
100-Fm-250



MAT 9936

Fission
293 Kelvin Cross Sections

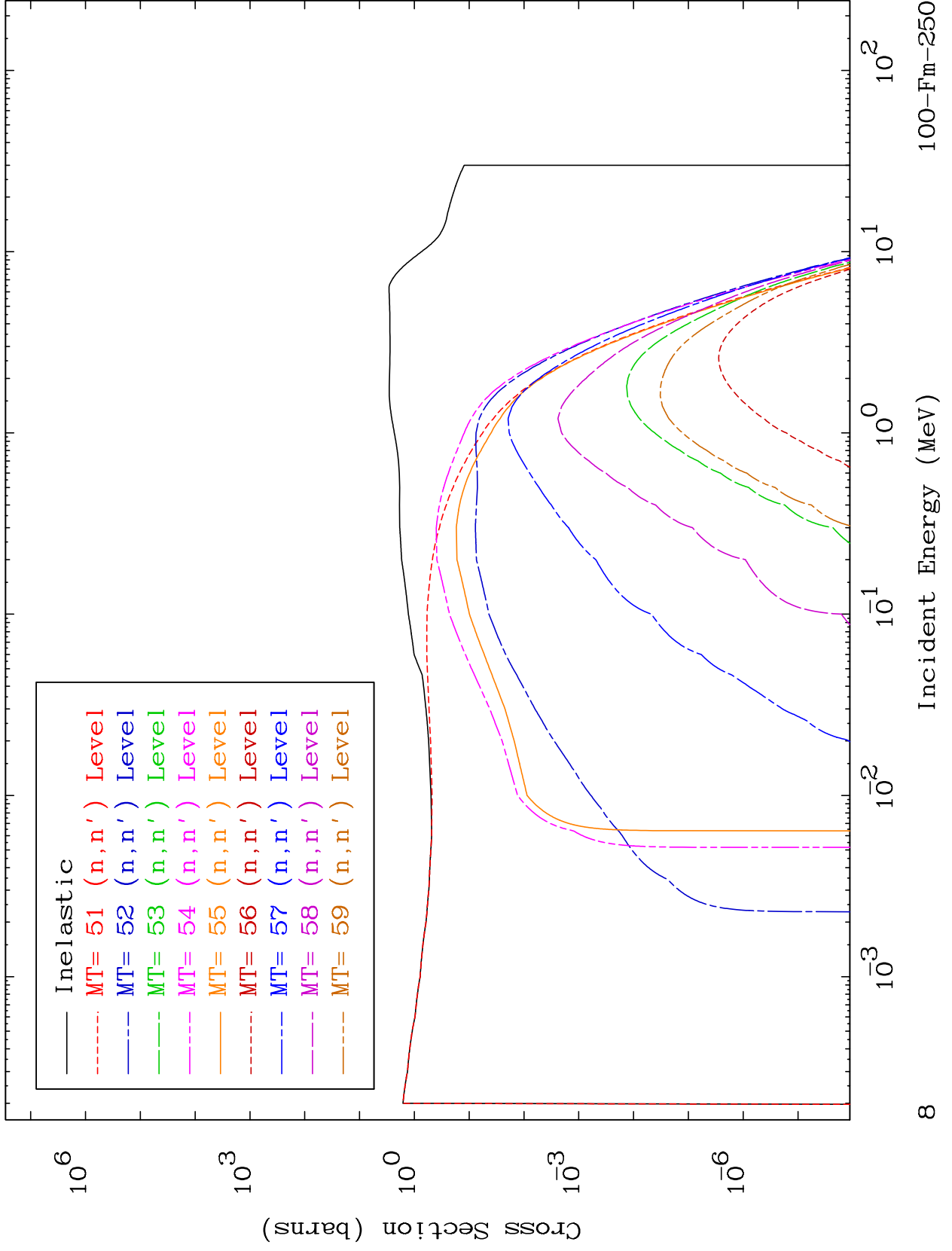
100-Fm-250



MAT 9936

(n,n') Level
293 Kelvin Cross Sections

100-Fm-250

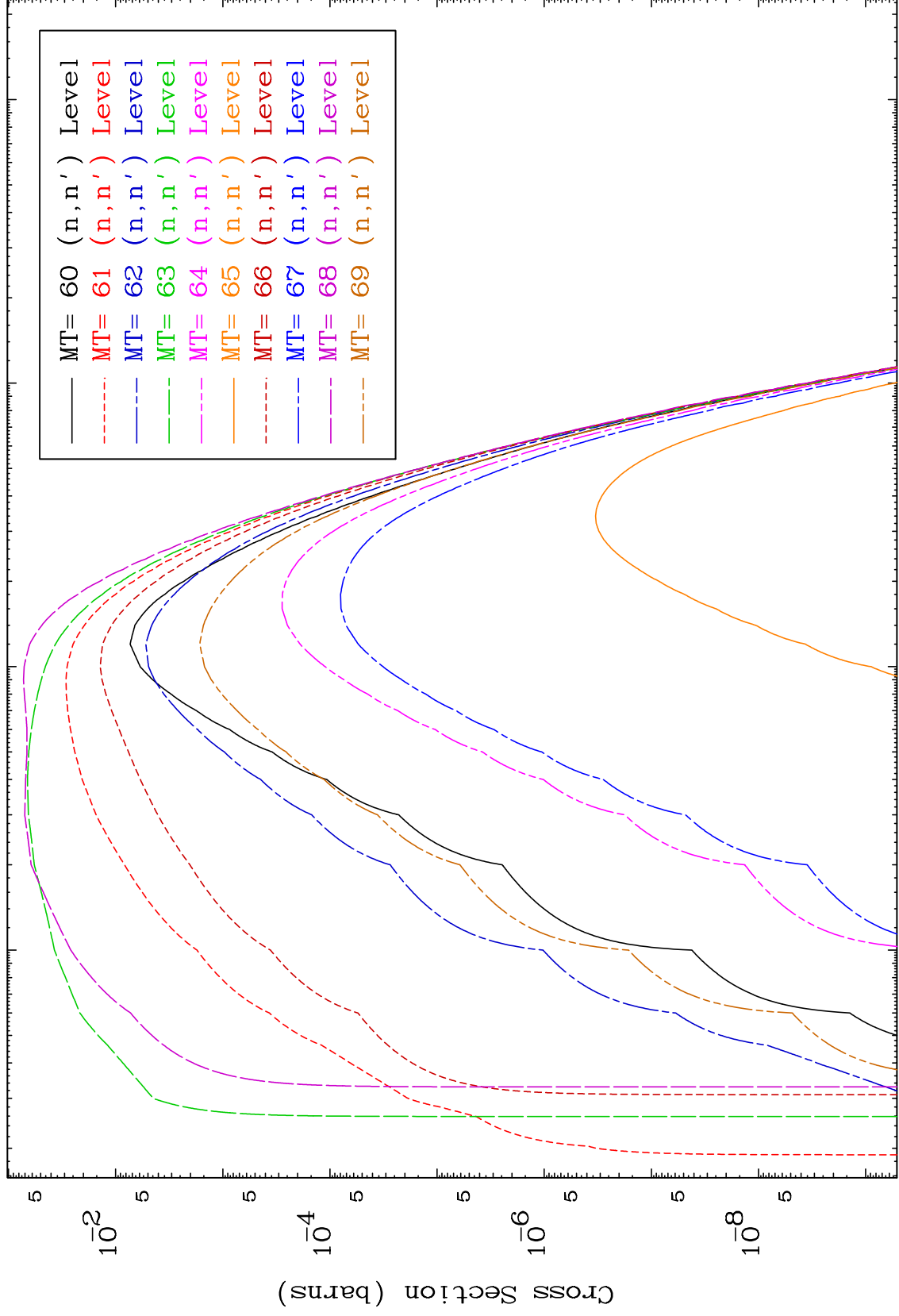


MAT 9936

(n,n') Level

100-Fm-250

293 Kelvin Cross Sections



9

Incident Energy (MeV)

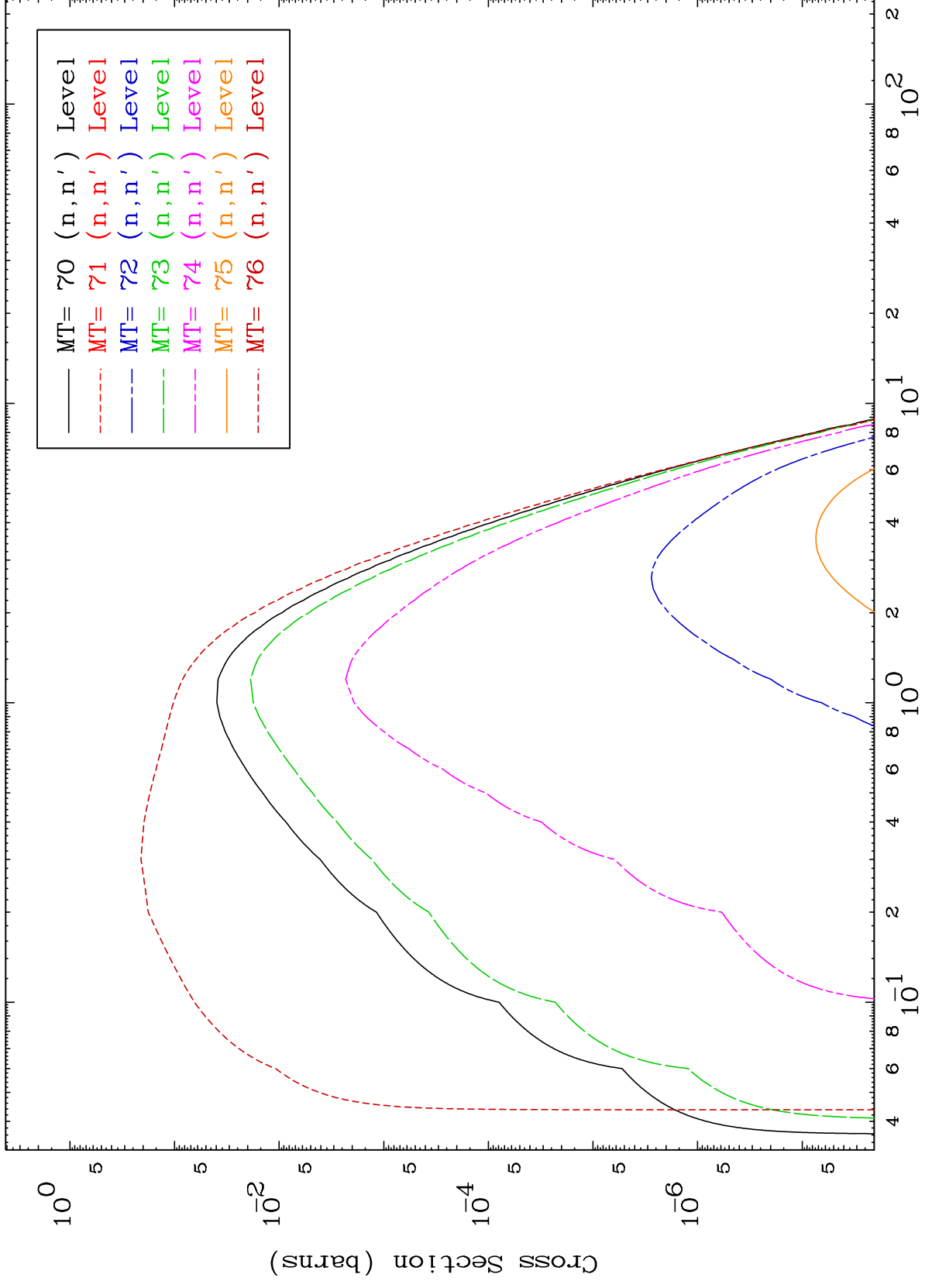
100-Fm-250

MAT 9936

(n,n') Level

100-Fm-250

293 Kelvin Cross Sections



10

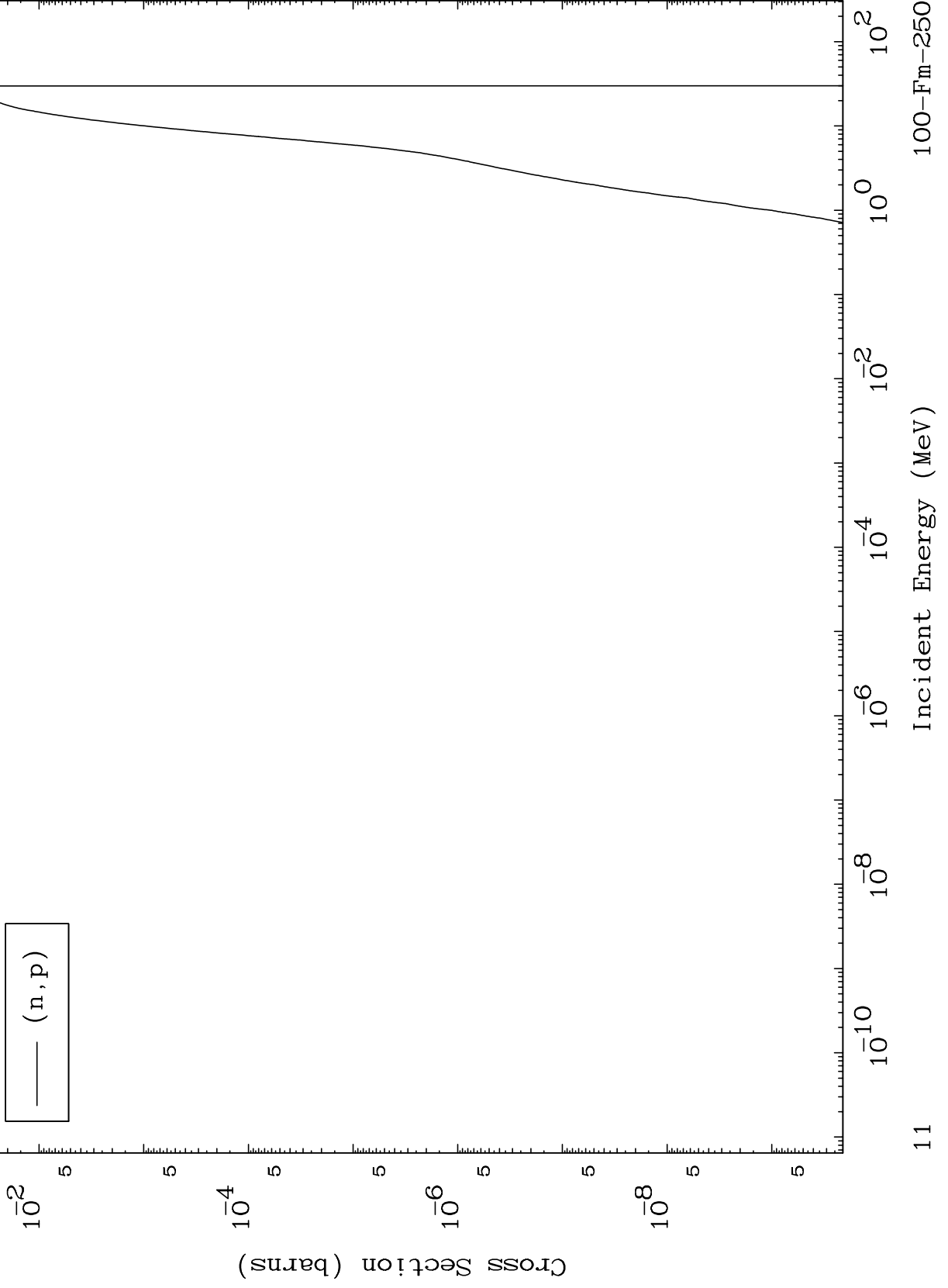
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,p) Levels
293 Kelvin Cross Sections

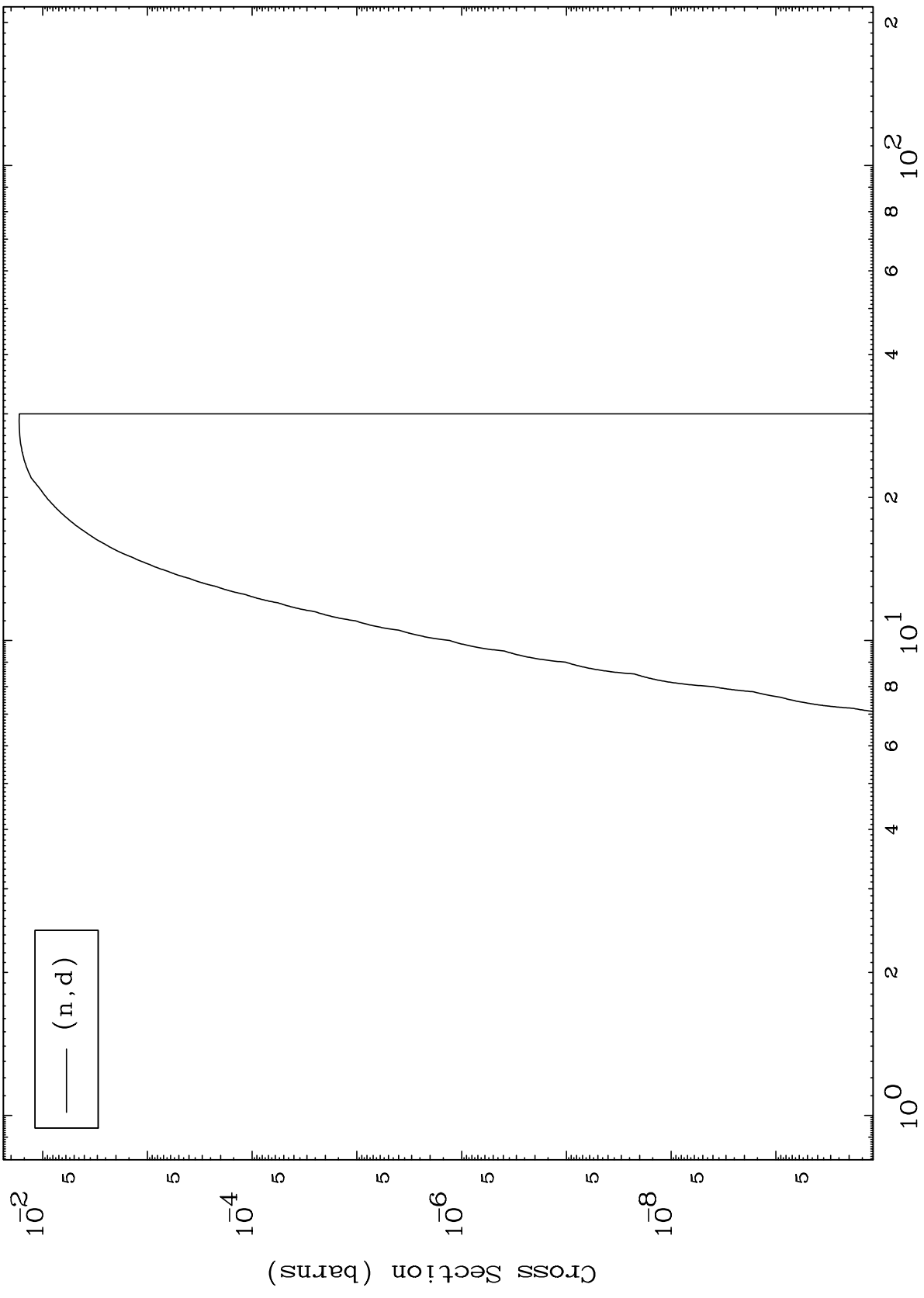
100-Fm-250



MAT 9936

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-250



Incident Energy (MeV)

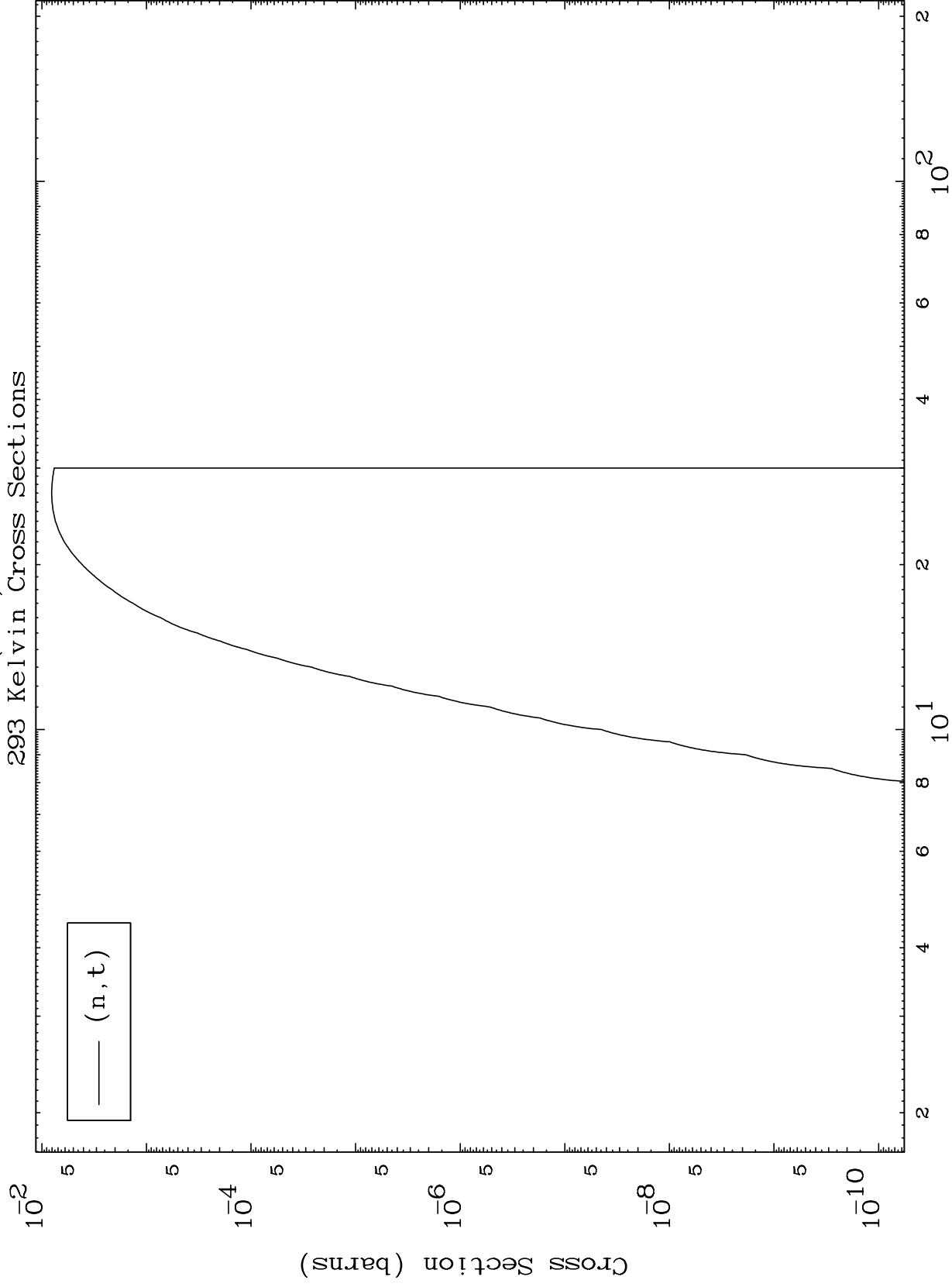
100-Fm-250

12

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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-250



13

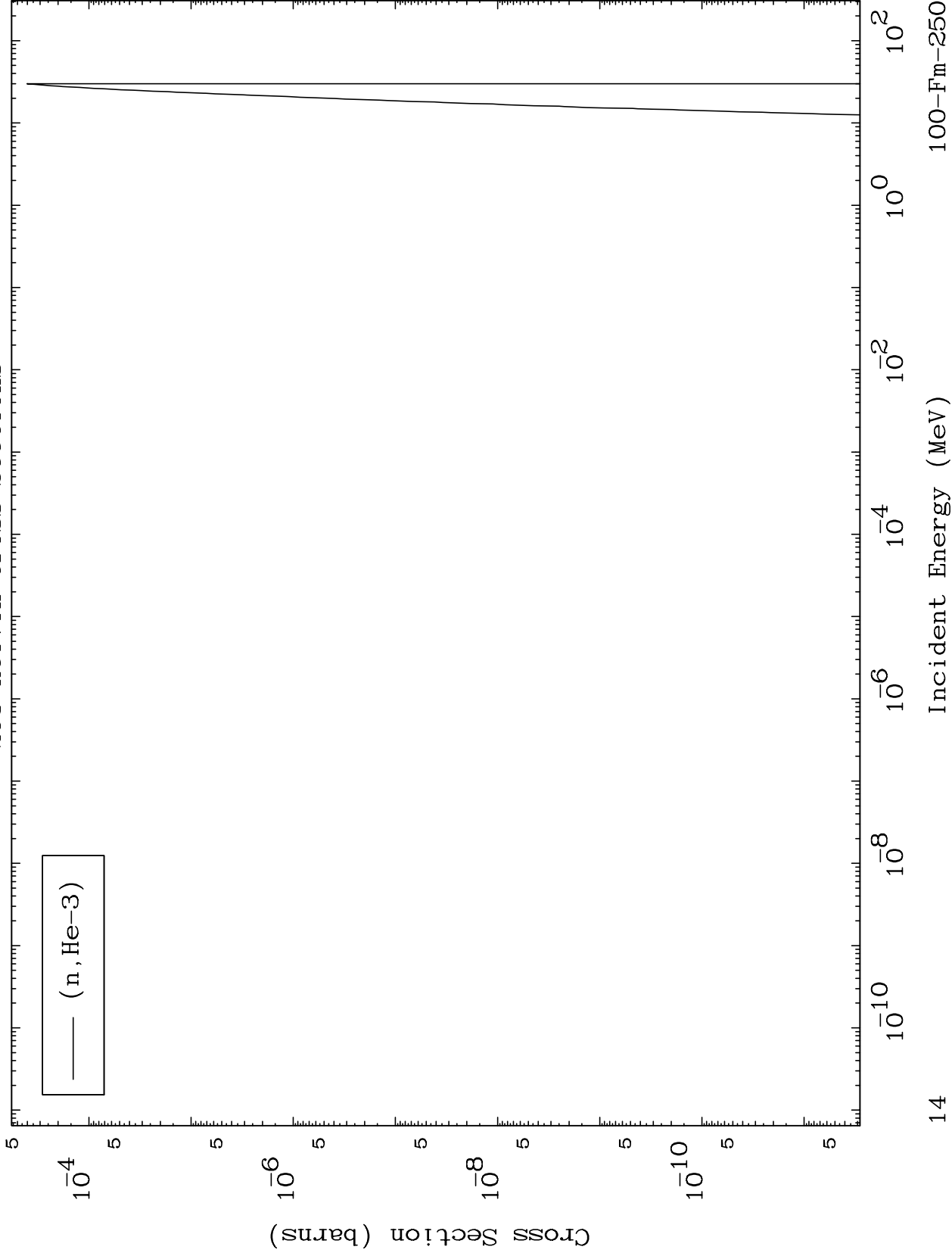
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,He3) Levels
293 Kelvin Cross Sections

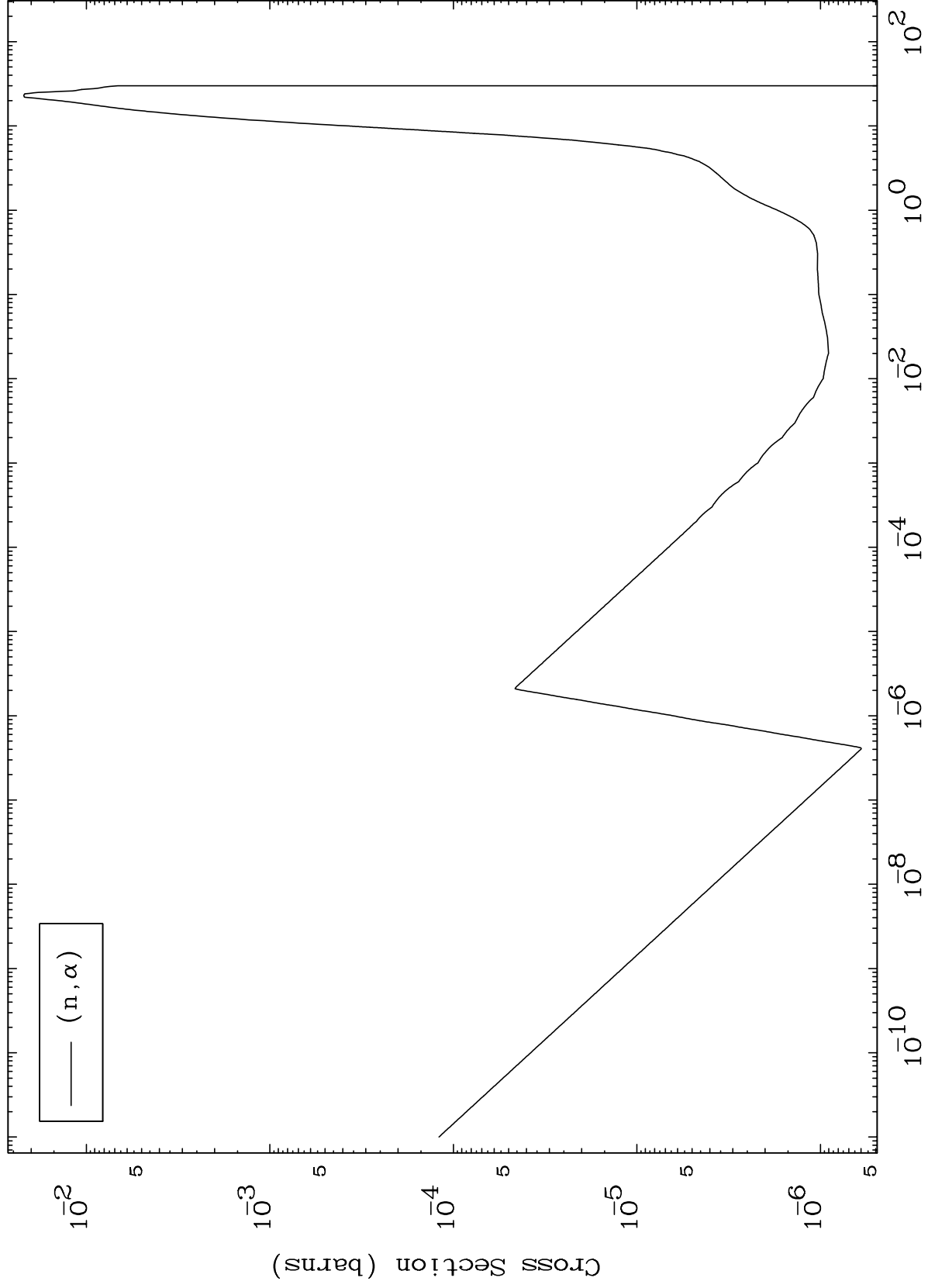
100-Fm-250



MAT 9936

(n, α) Levels
293 Kelvin Cross Sections

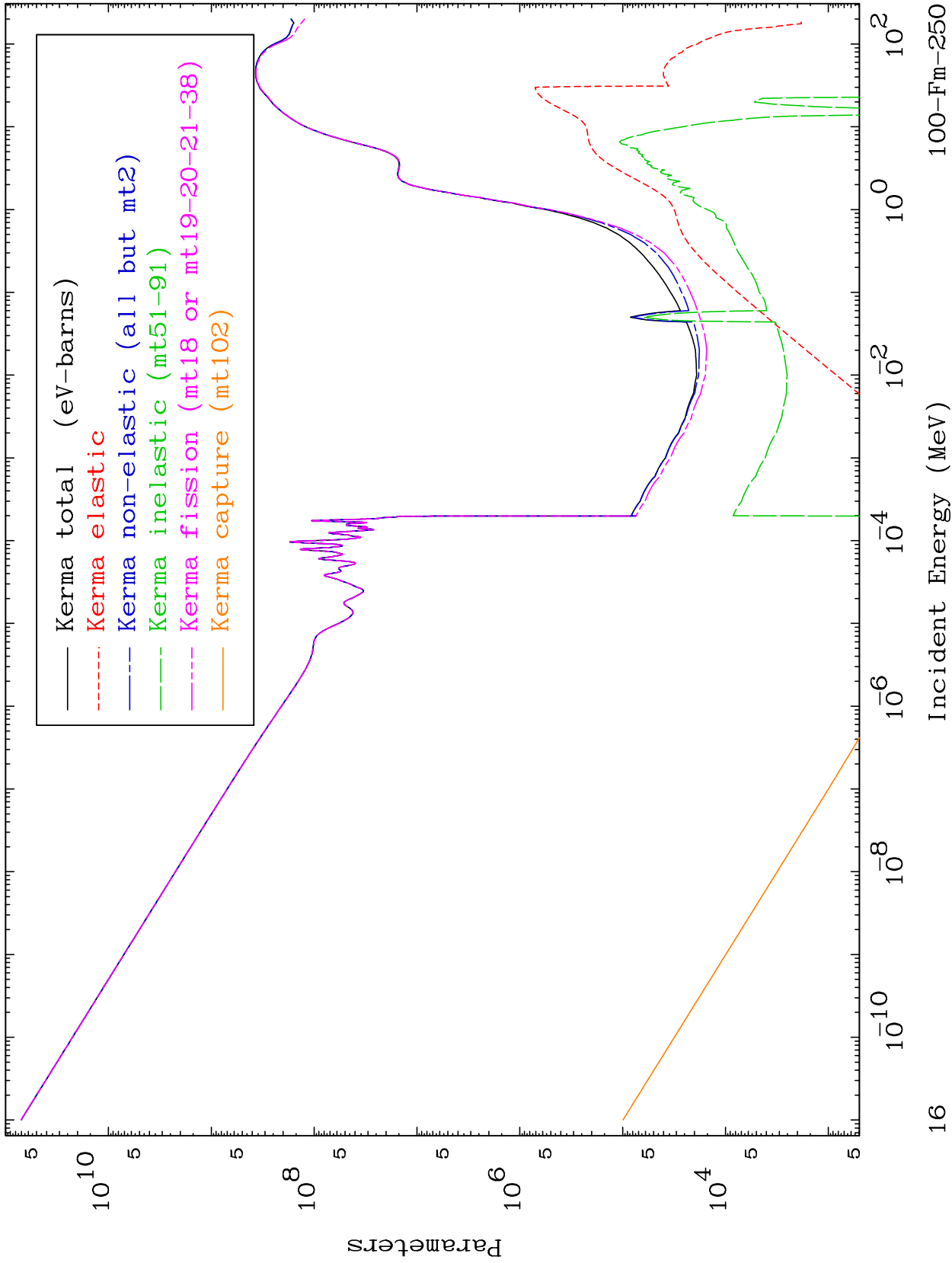
100-Fm-250

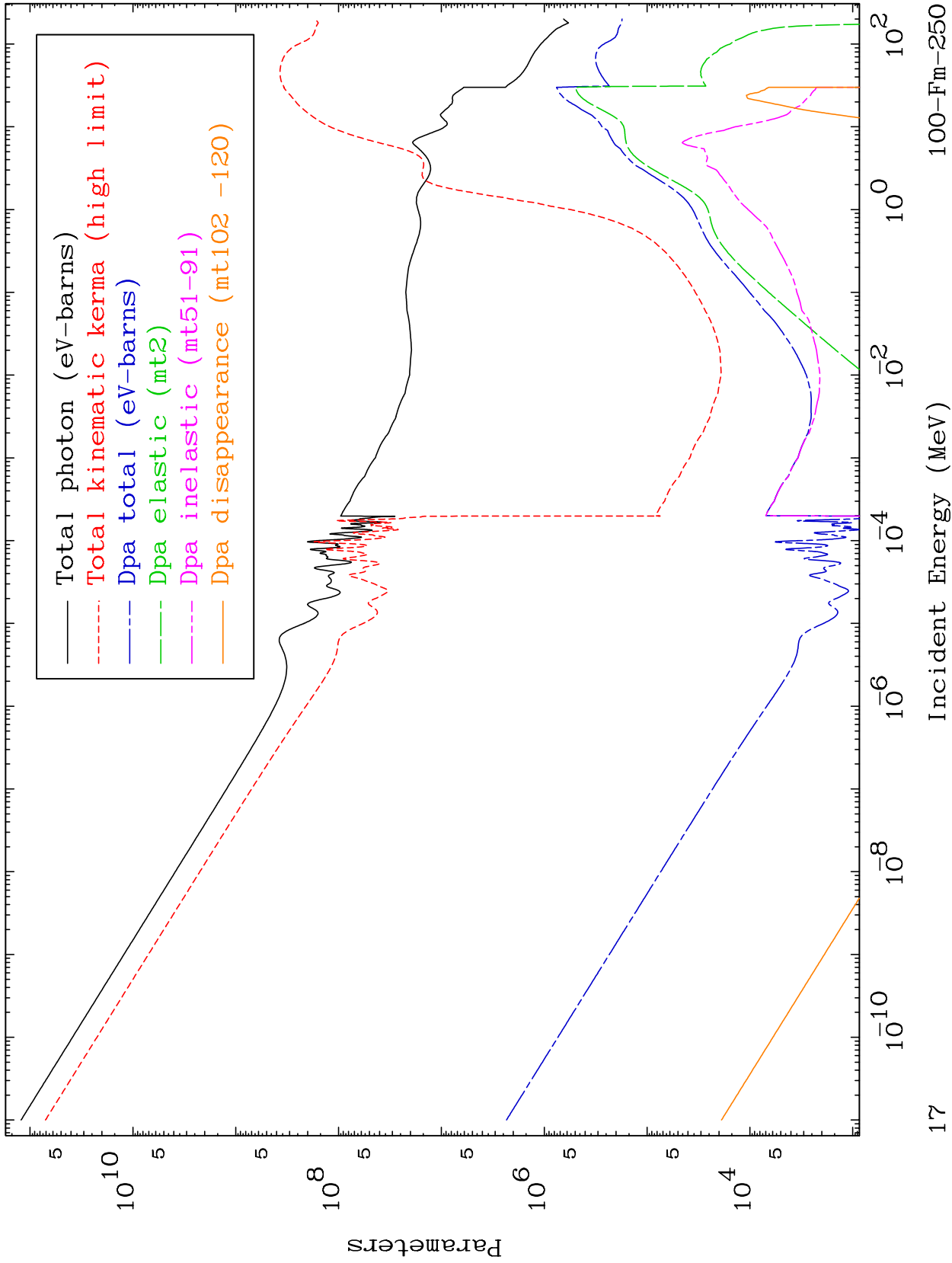


15

Incident Energy (MeV)

100-Fm-250

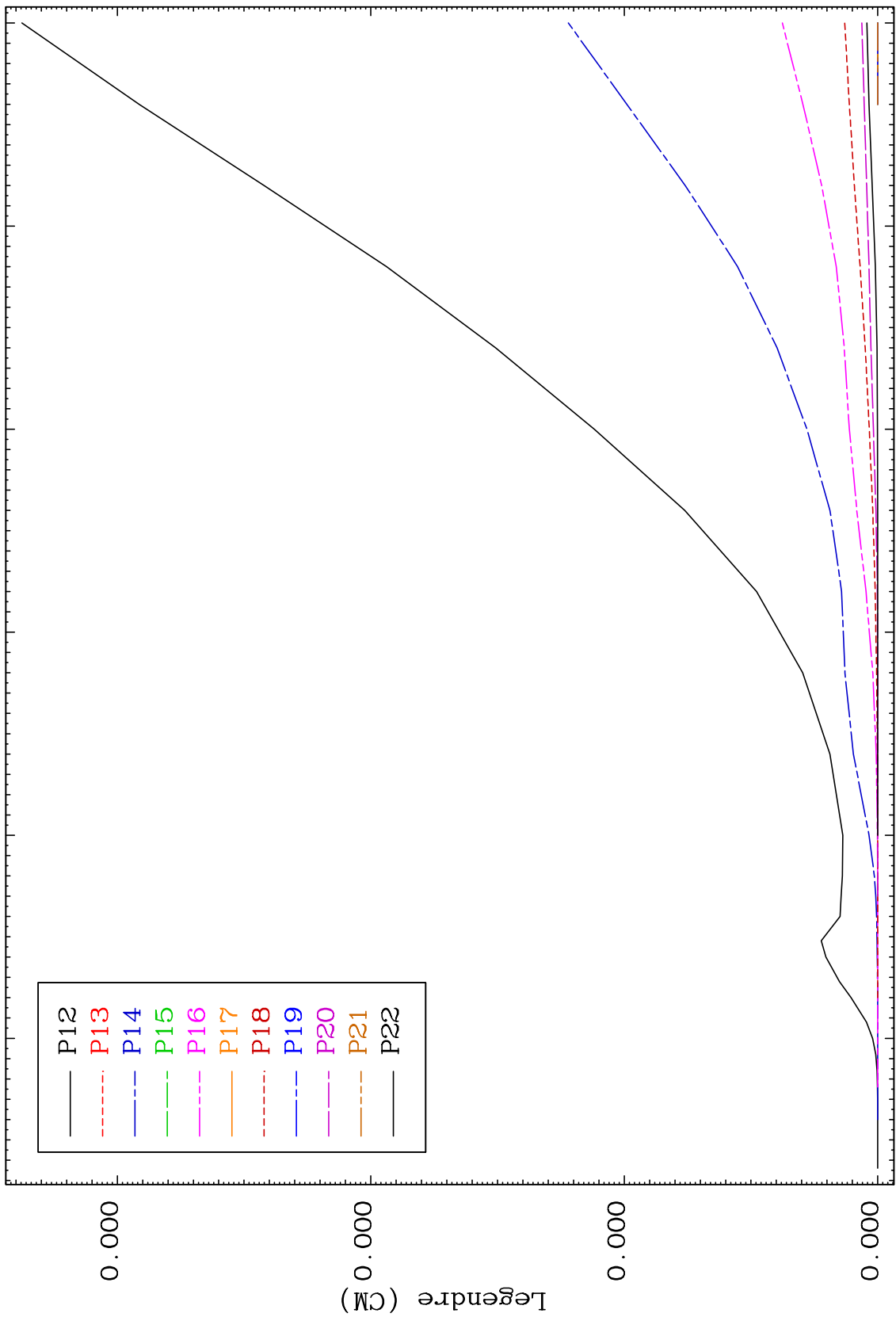




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Elastic Legendre Coefficients

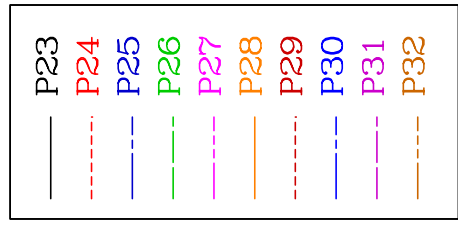
100-Fm-250



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Elastic
Legendre Coefficients

100-Fm-250



$\times 10^{-6}$

Legendre (CM)

6

4

2

0

15

20

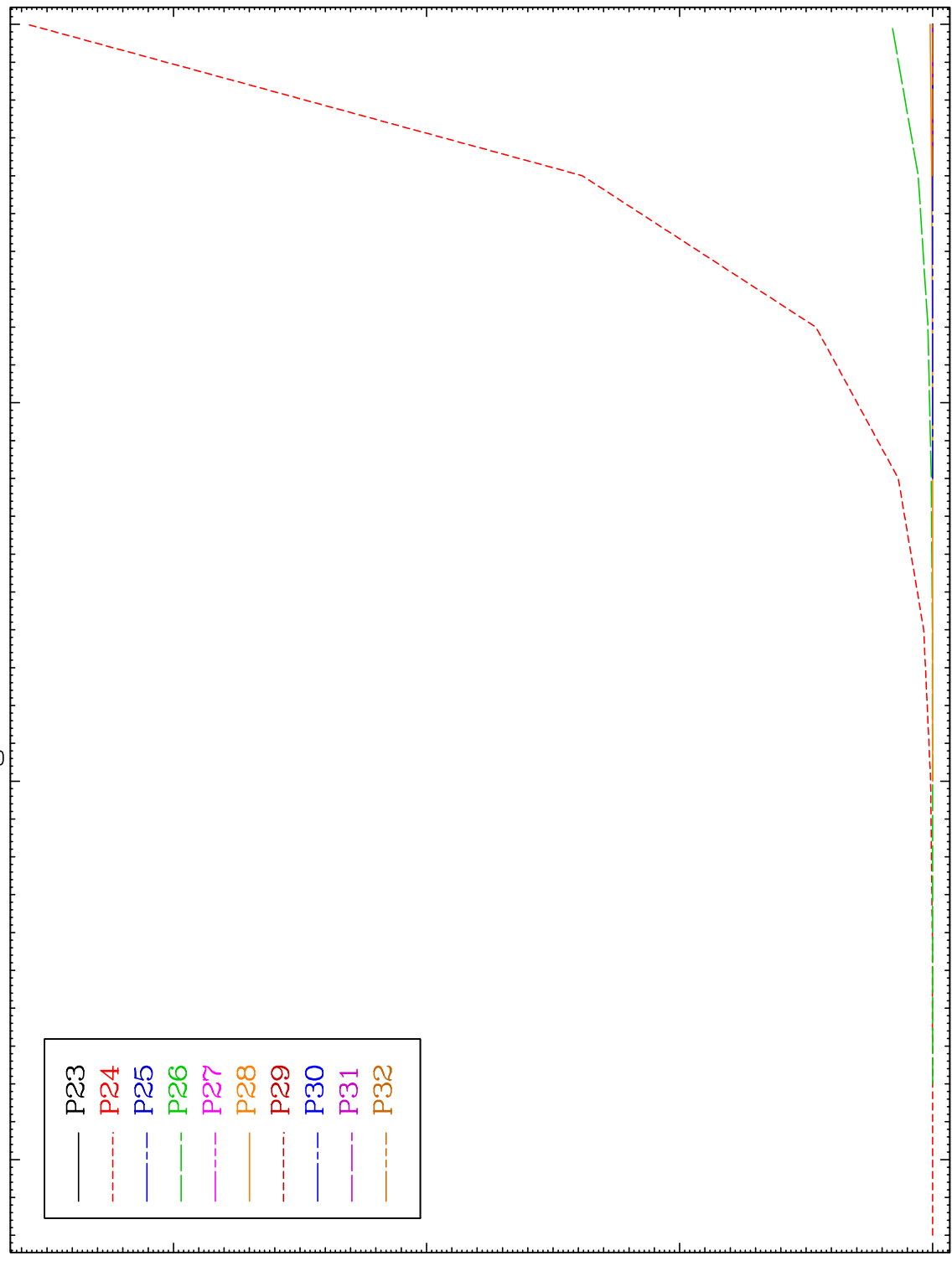
25

30

20

Incident Energy (MeV)

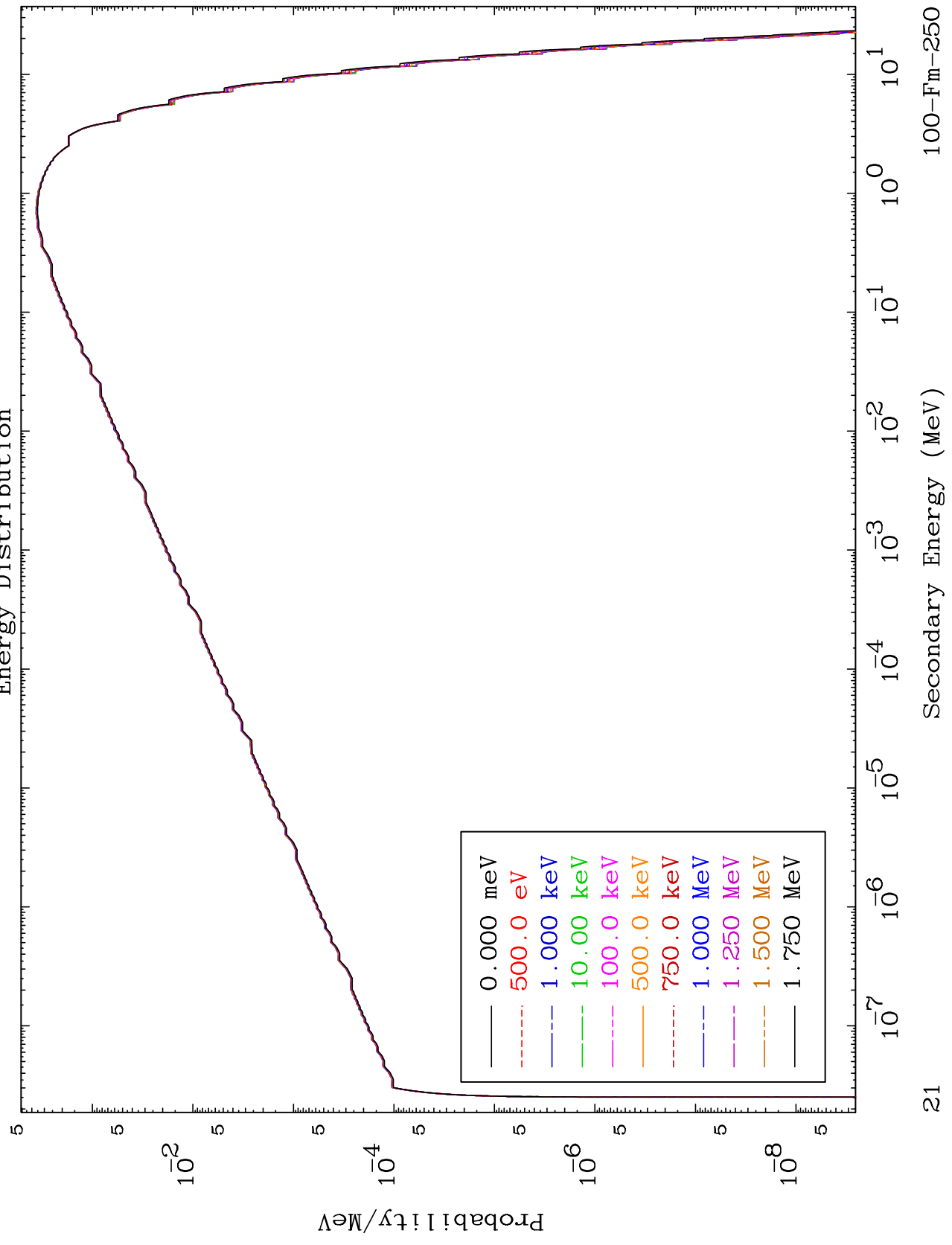
100-Fm-250



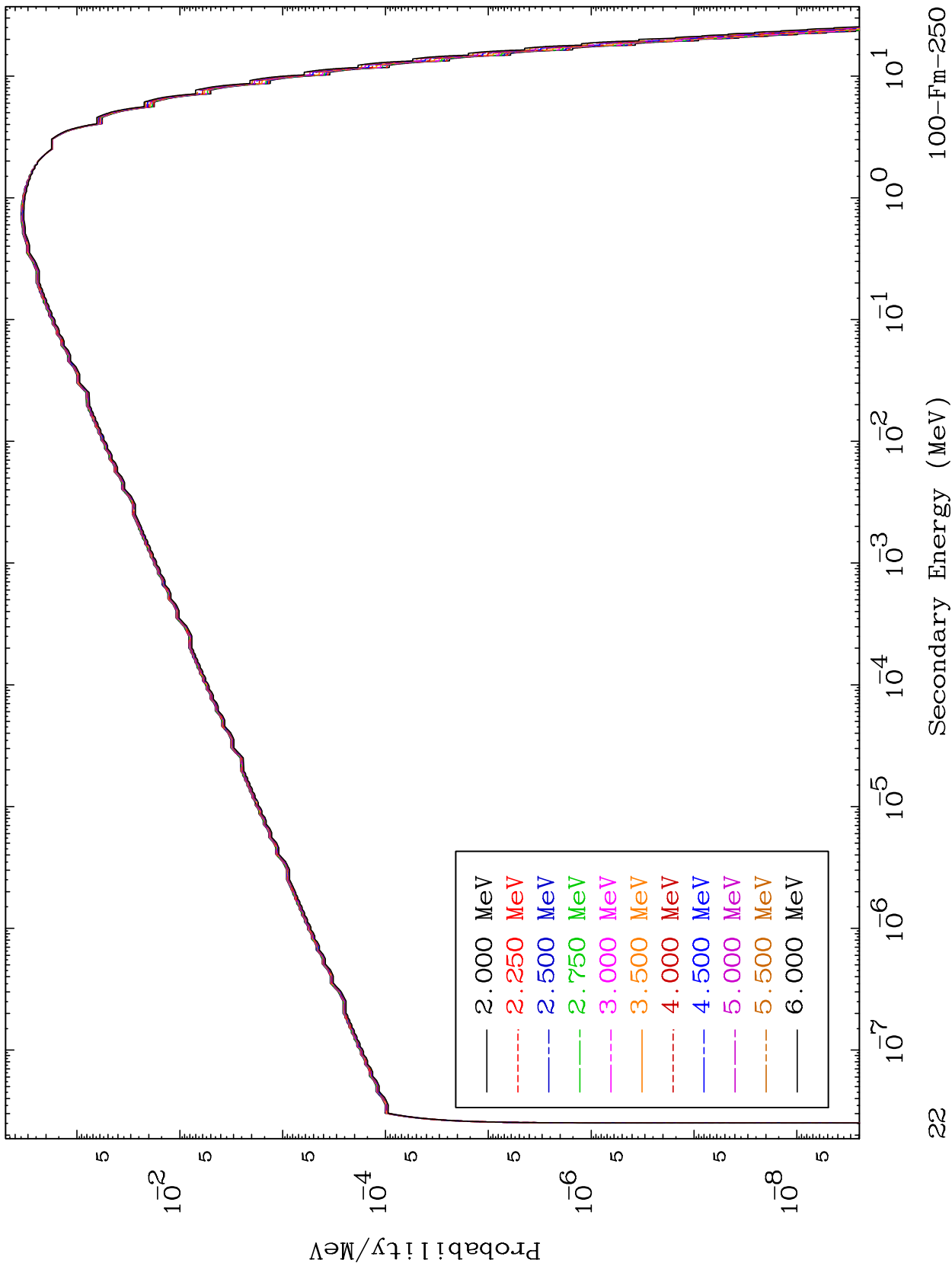
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Fission Energy Distribution

100-Fm-250



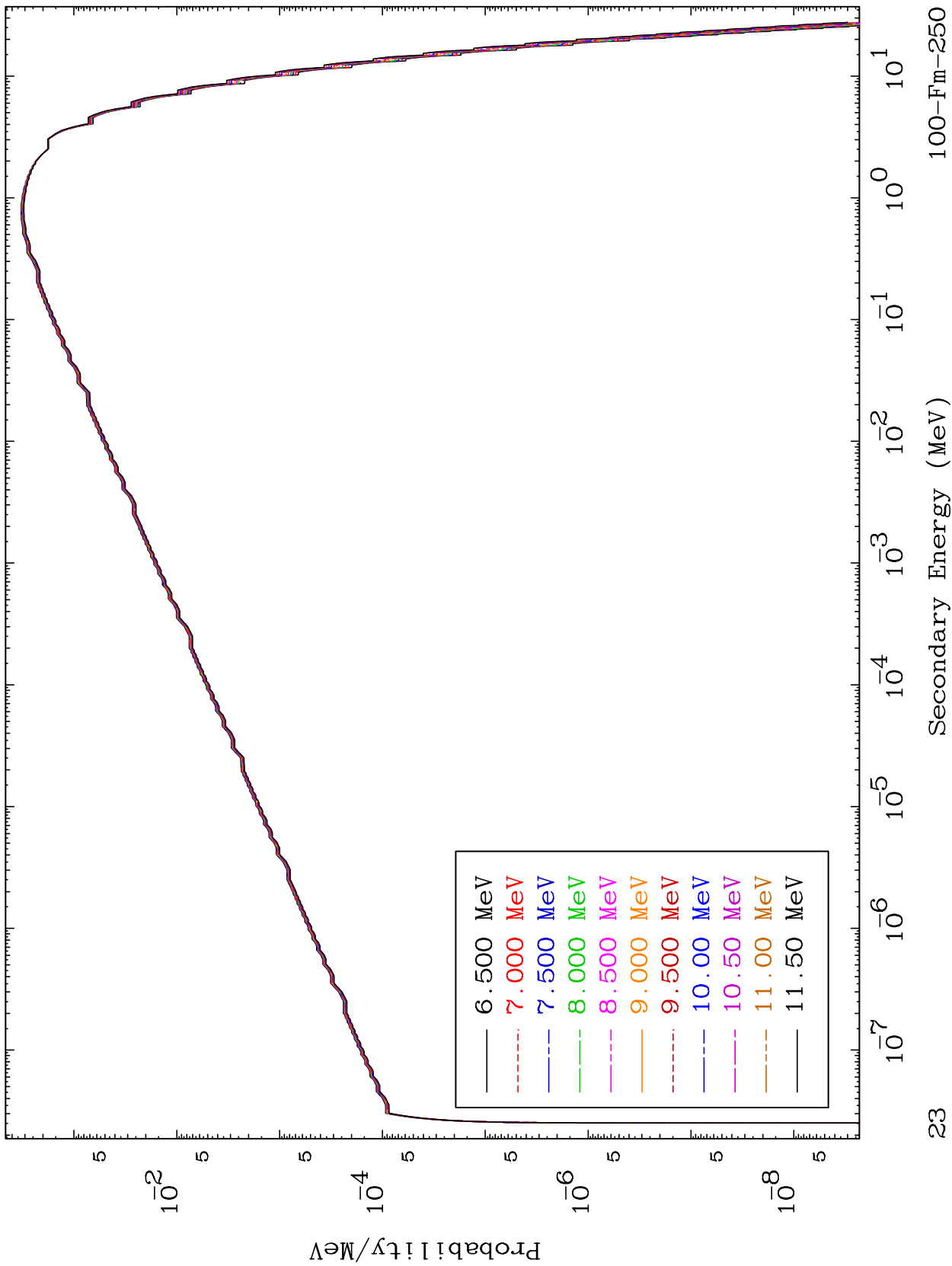
Fission Energy Distribution



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Fission Energy Distribution

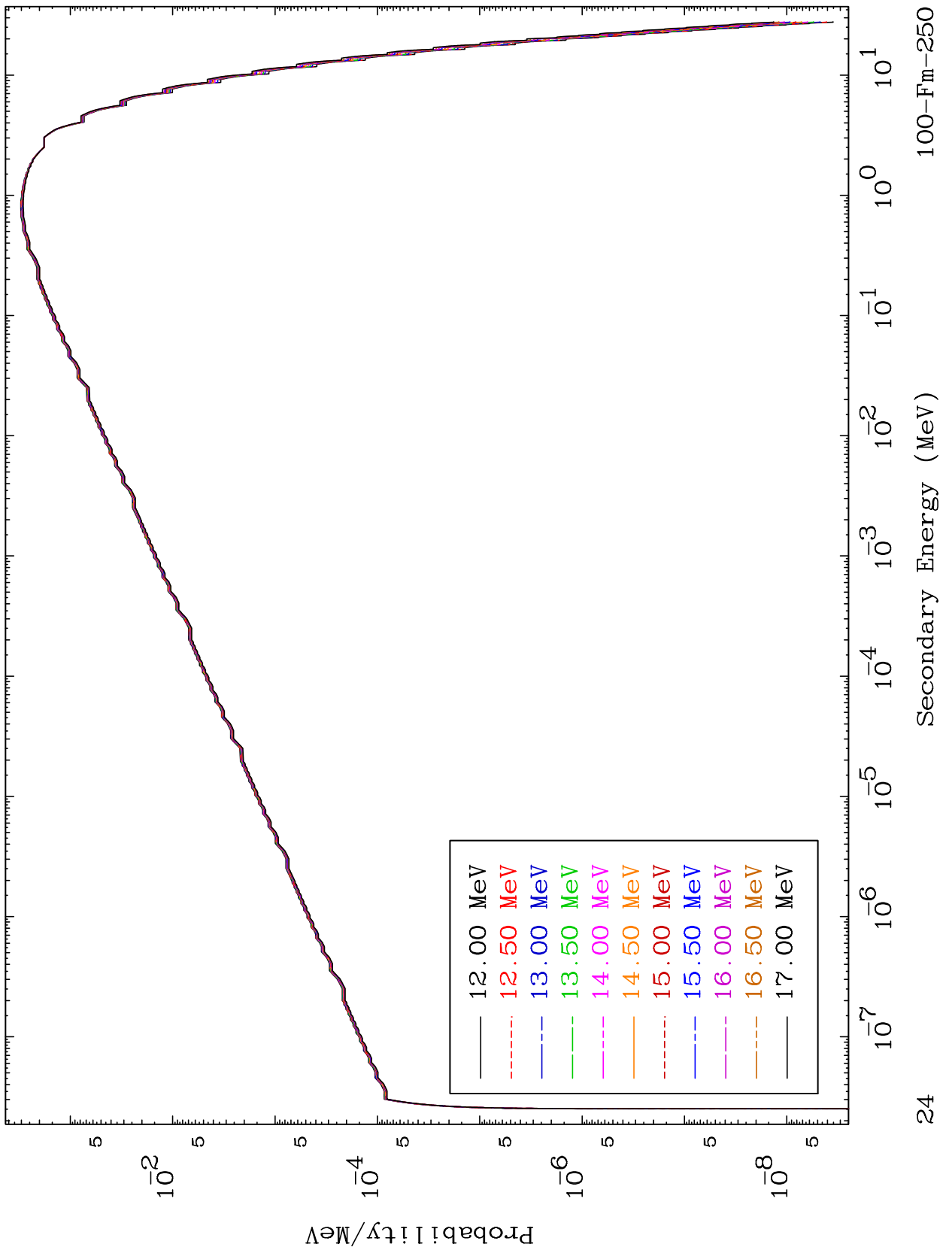
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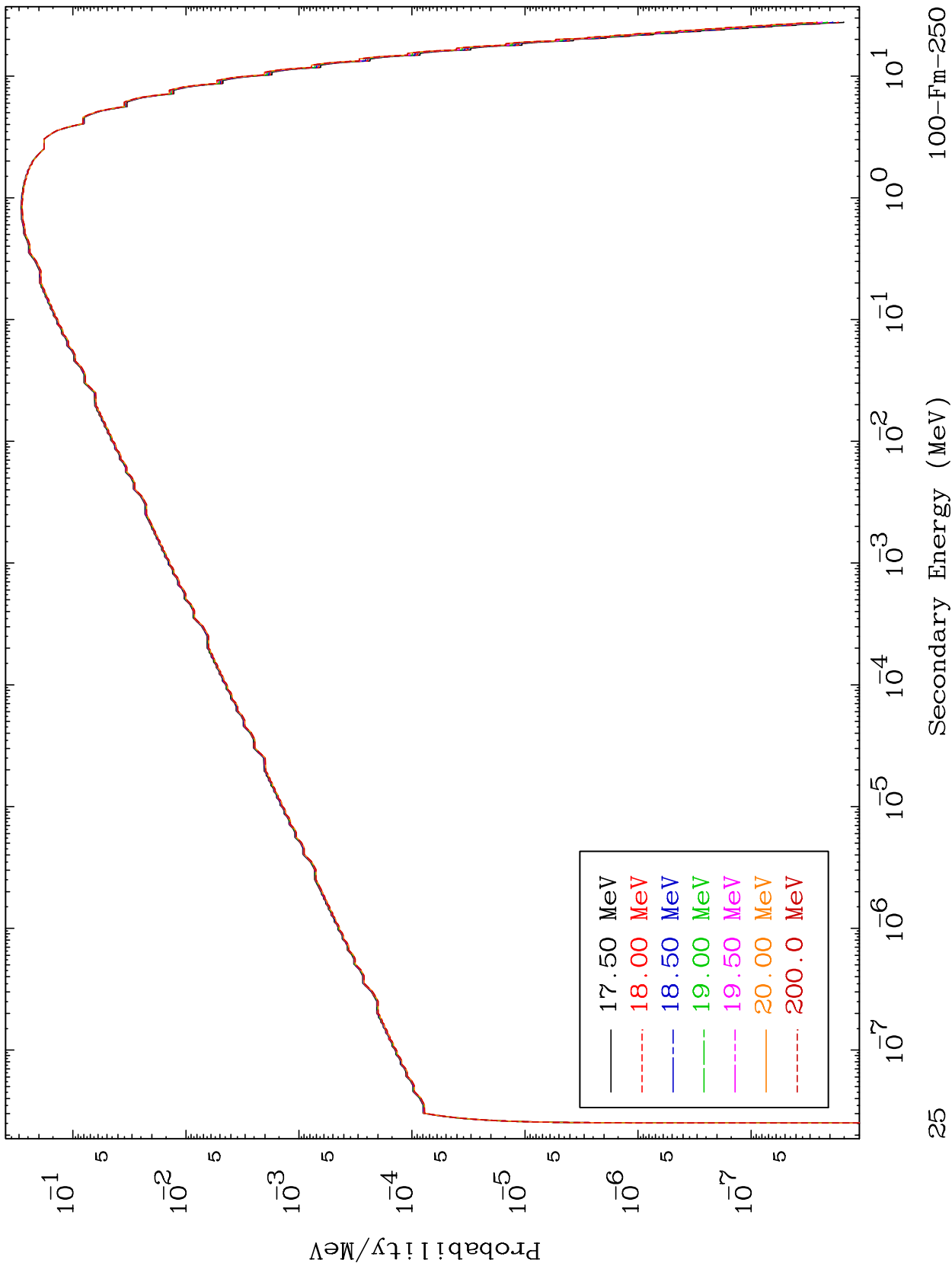
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Fission Energy Distribution

100-Fm-250



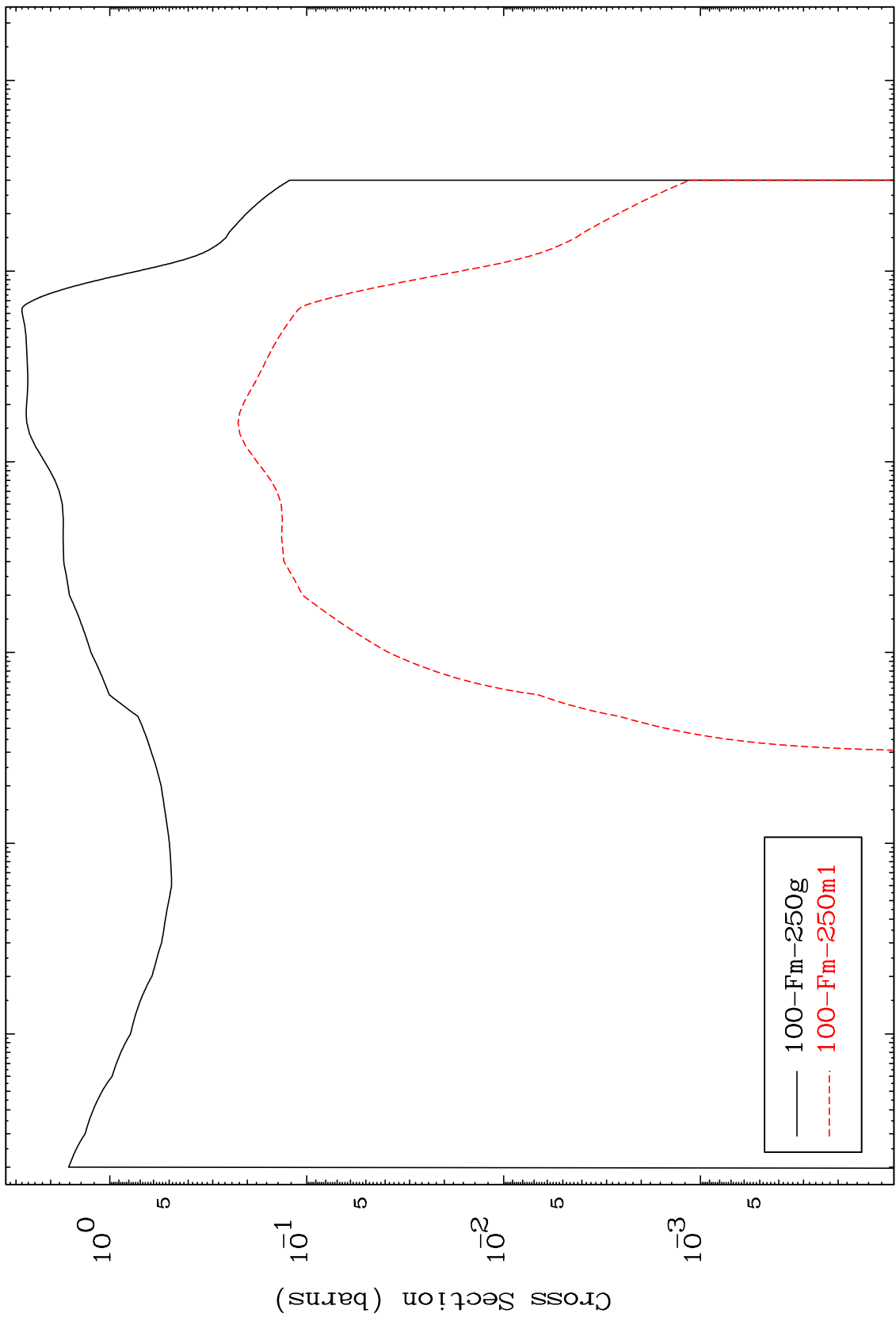
Fission
Energy Distribution



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100-Fm-250

Inelastic
Radionuclide Production Cross Section

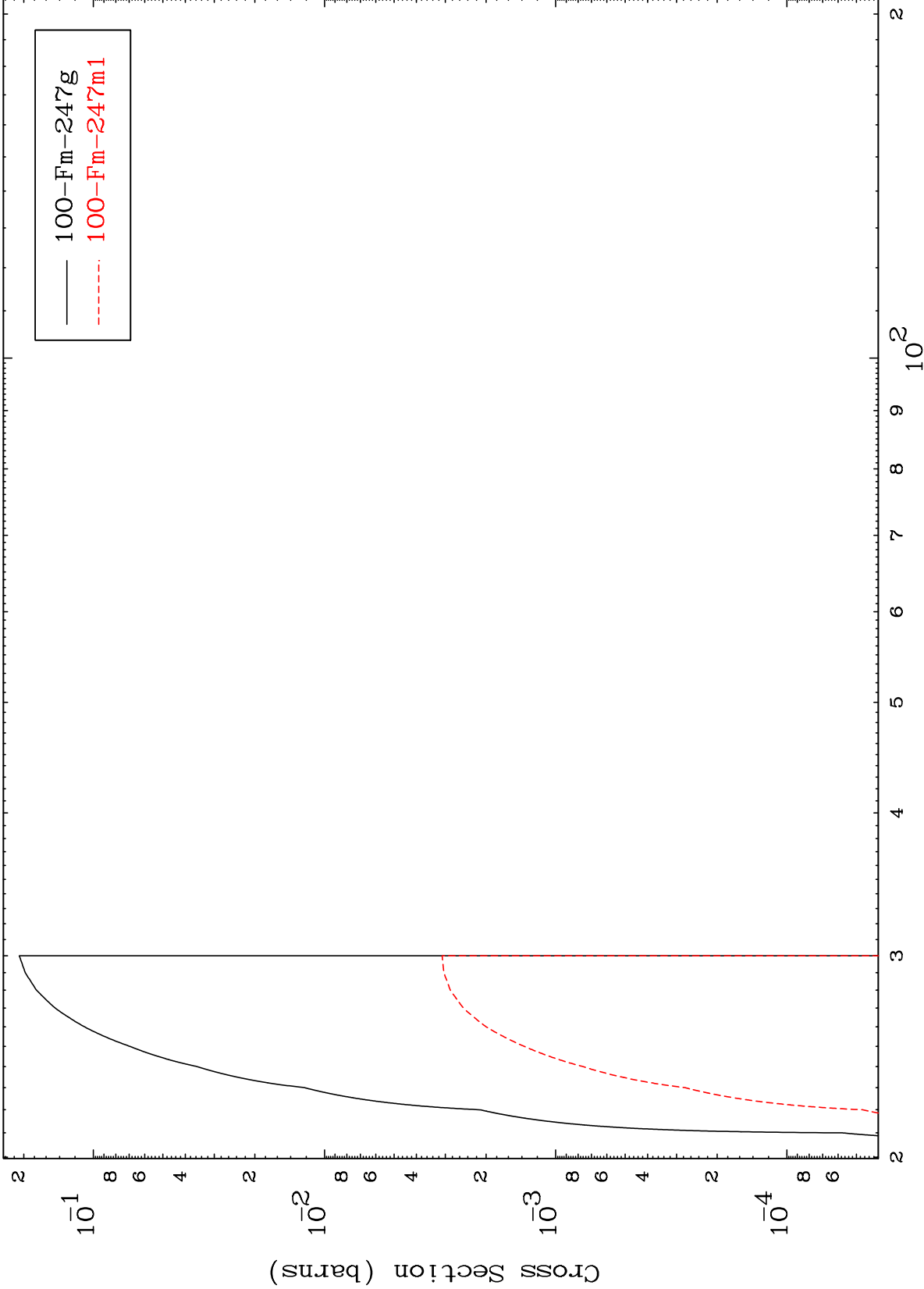


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(n,4n)

100-Fm-250

Radionuclide Production Cross Section



100-Fm-247g
100-Fm-247m1

27

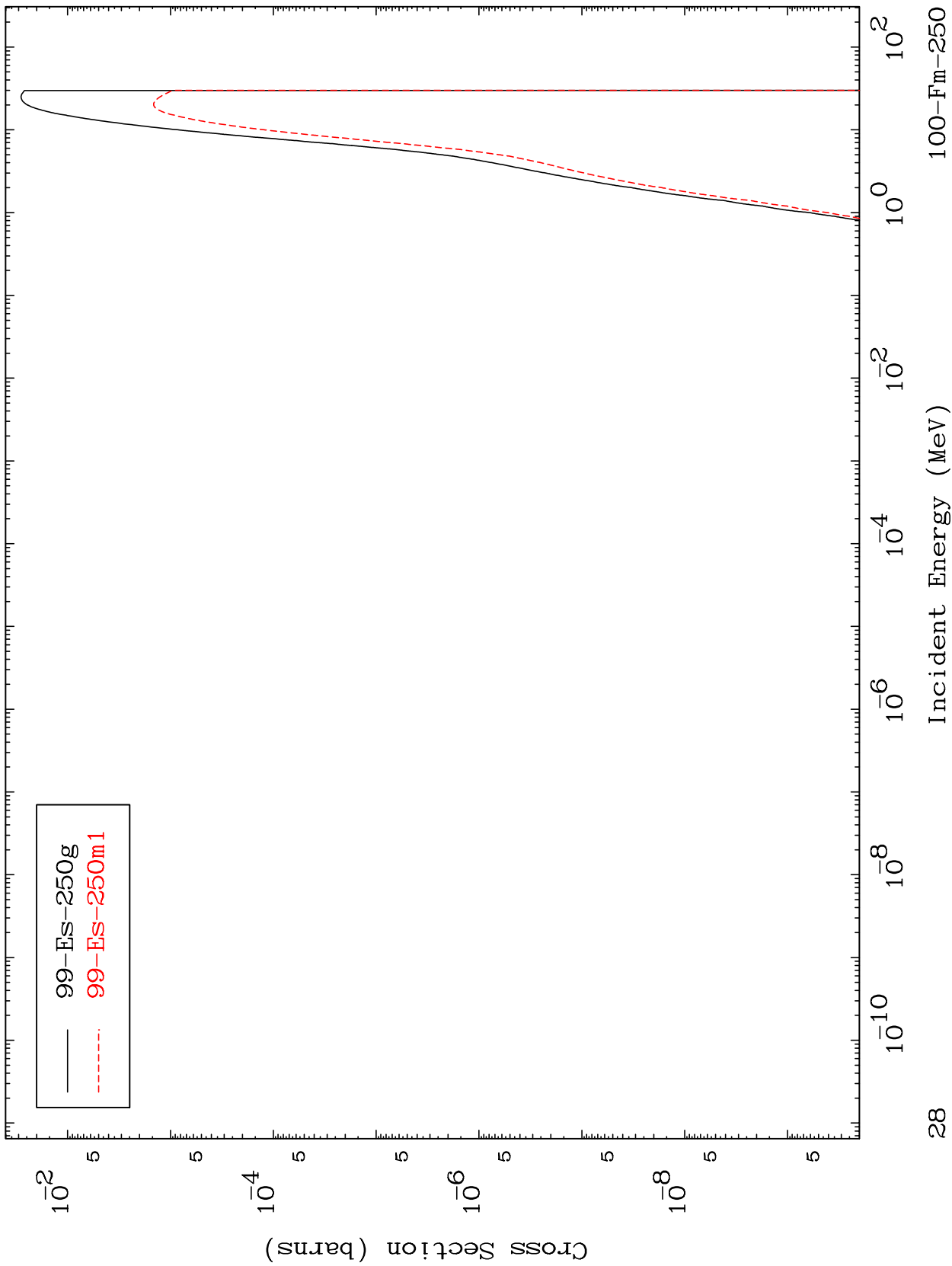
Incident Energy (MeV)

100-Fm-250

MAT 9936

100-Fm-250

(n,p)
Radionuclide Production Cross Section



— 99-Es-250g
- - - 99-Es-250m1