

Program EVALPLOT  
(Version 2017-1)

by

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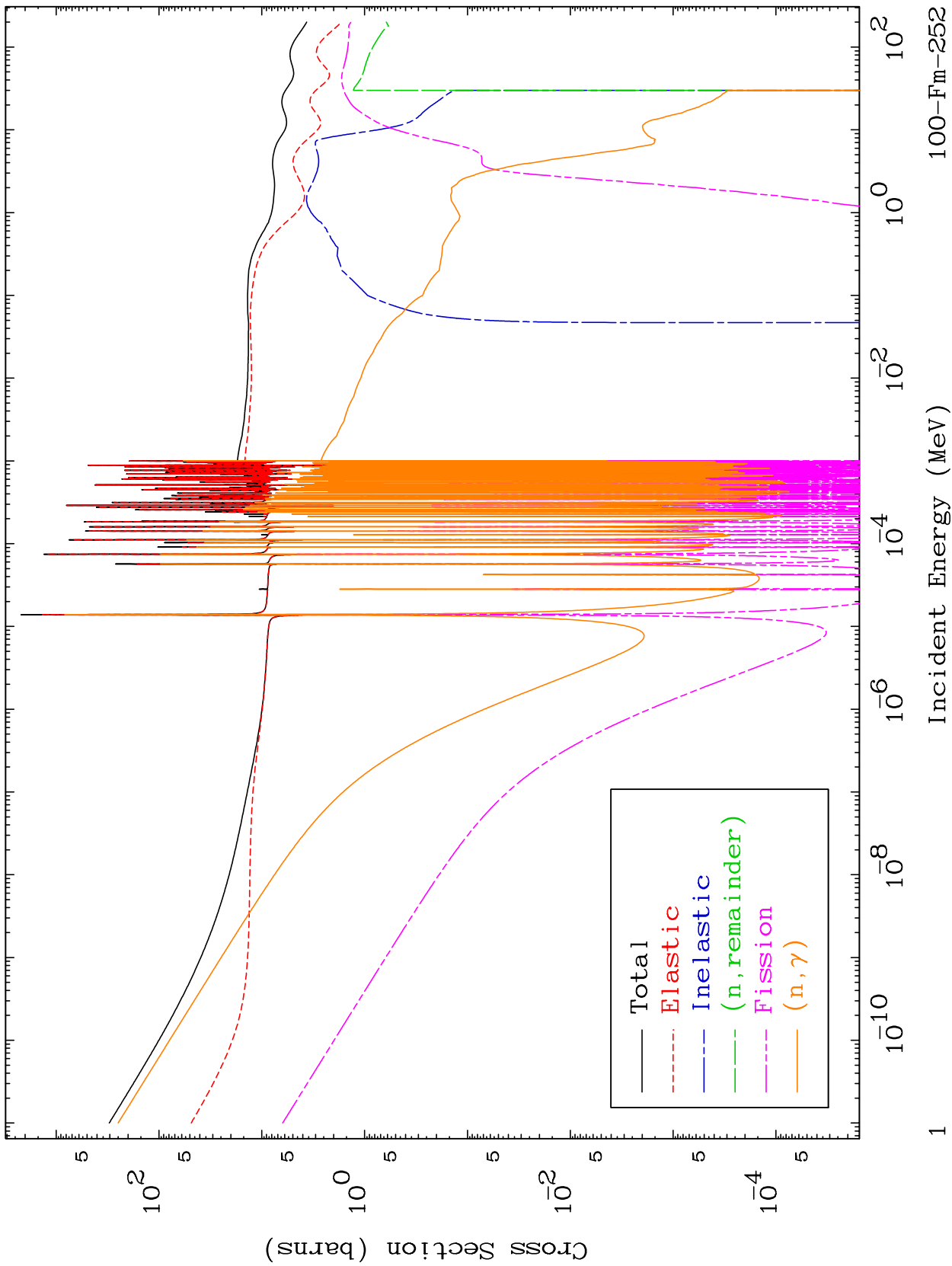
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9933

Major  
293 Kelvin Cross Sections

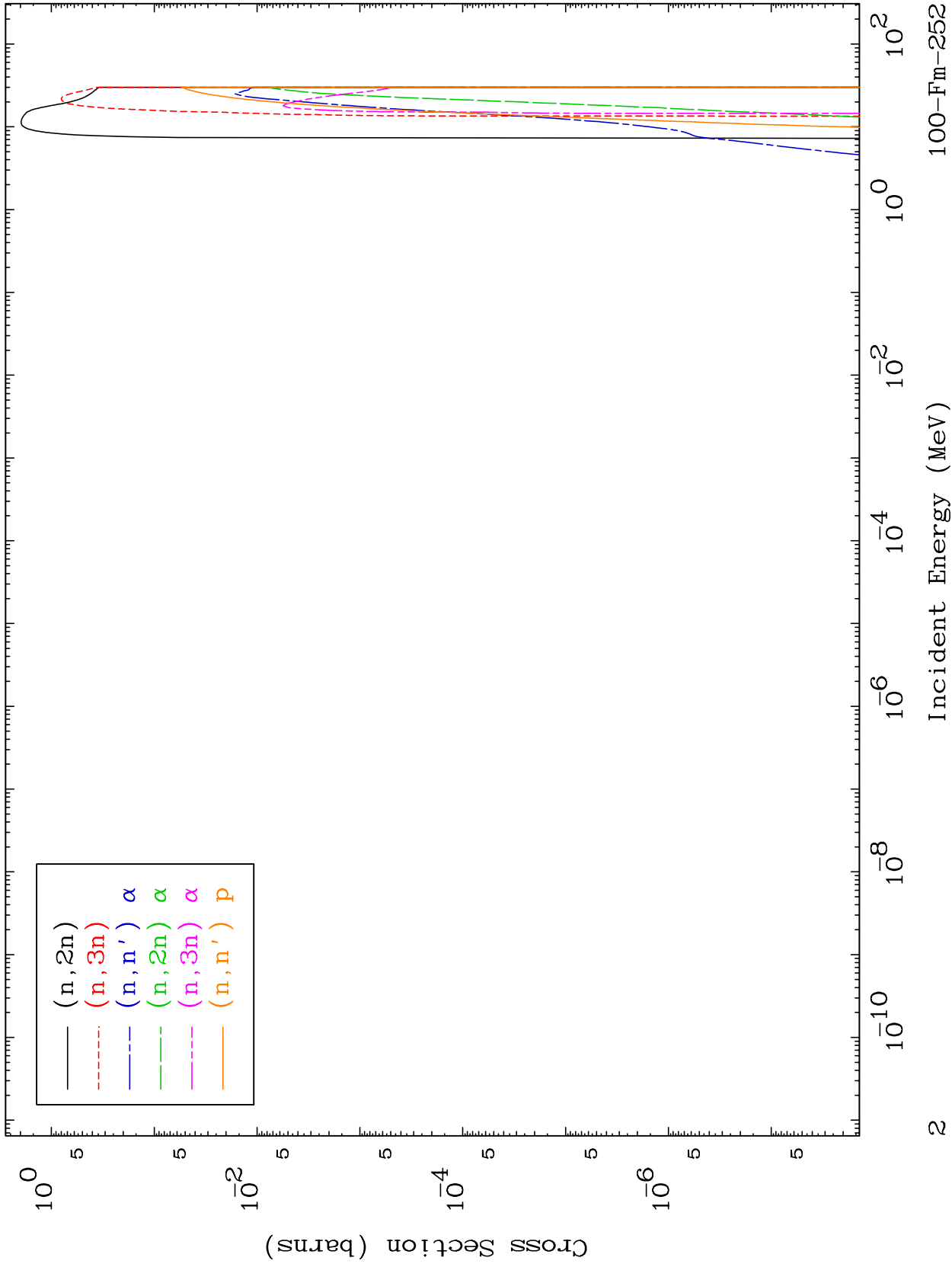
100-Fm-252



MAT 9933

Neutron Production  
293 Kelvin Cross Sections

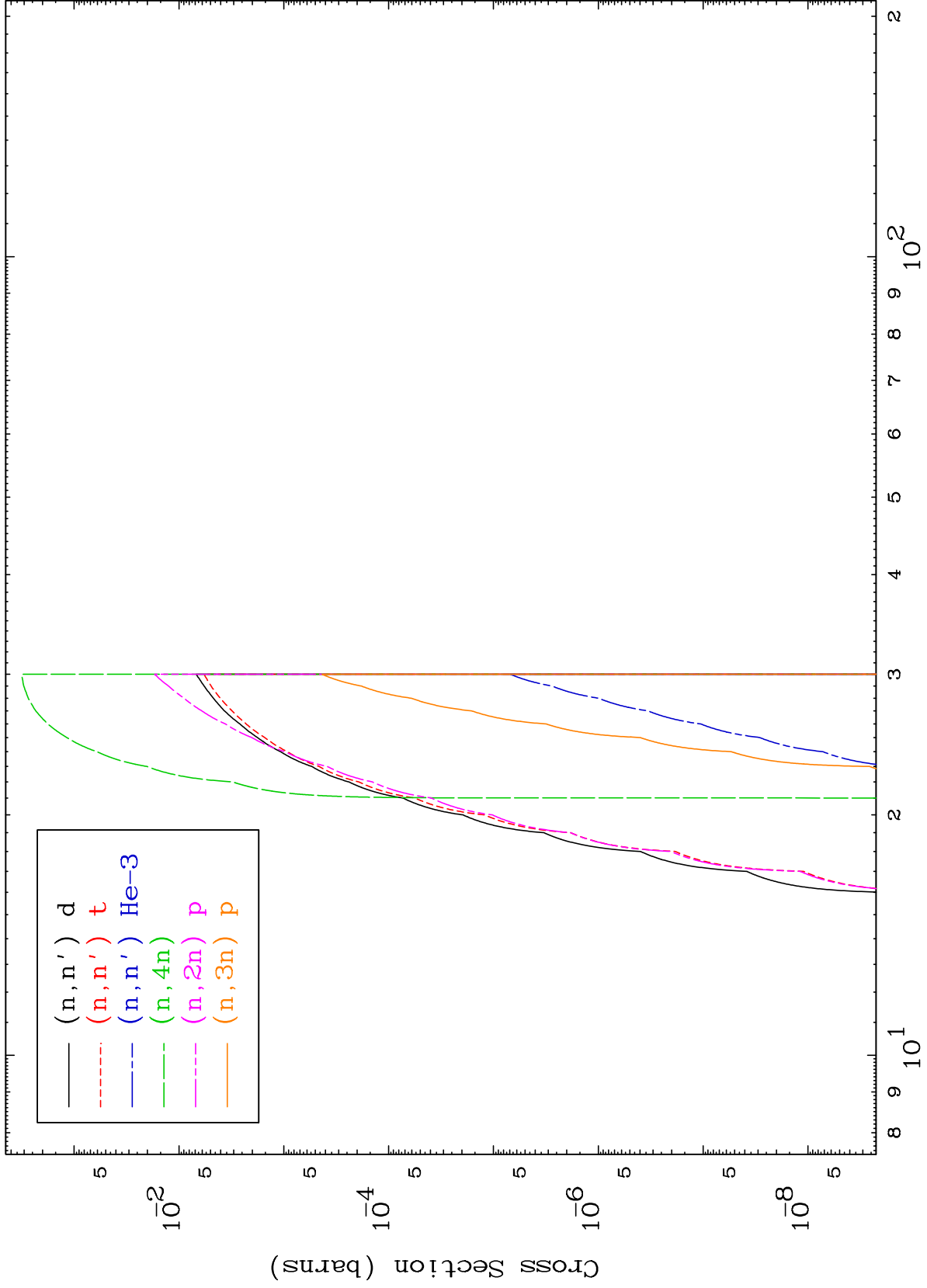
100-Fm-252



MAT 9933

Neutron Production  
293 Kelvin Cross Sections

100-Fm-252



3

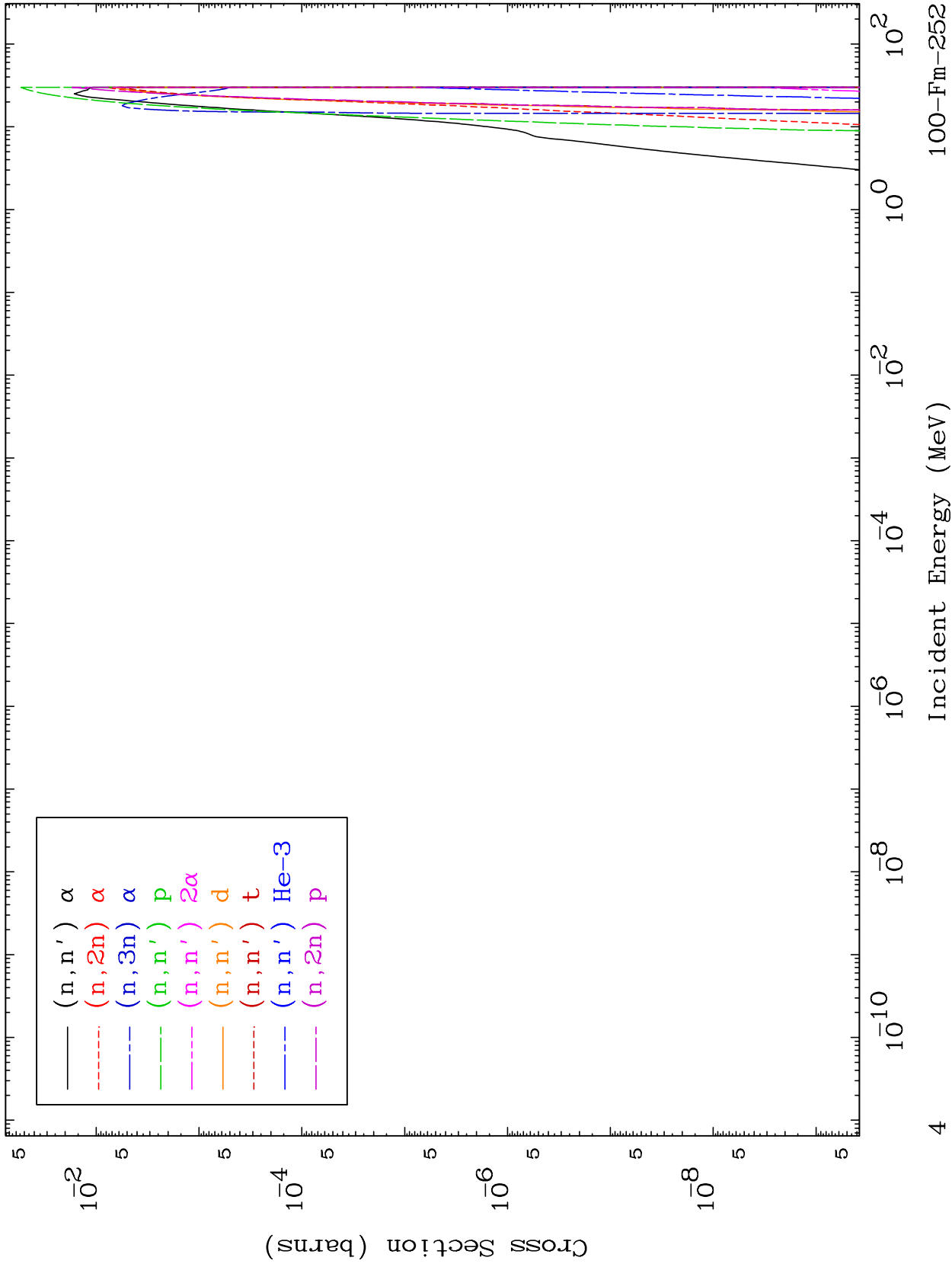
Incident Energy (MeV)

100-Fm-252

MAT 9933

Charged Particle  
293 Kelvin Cross Sections

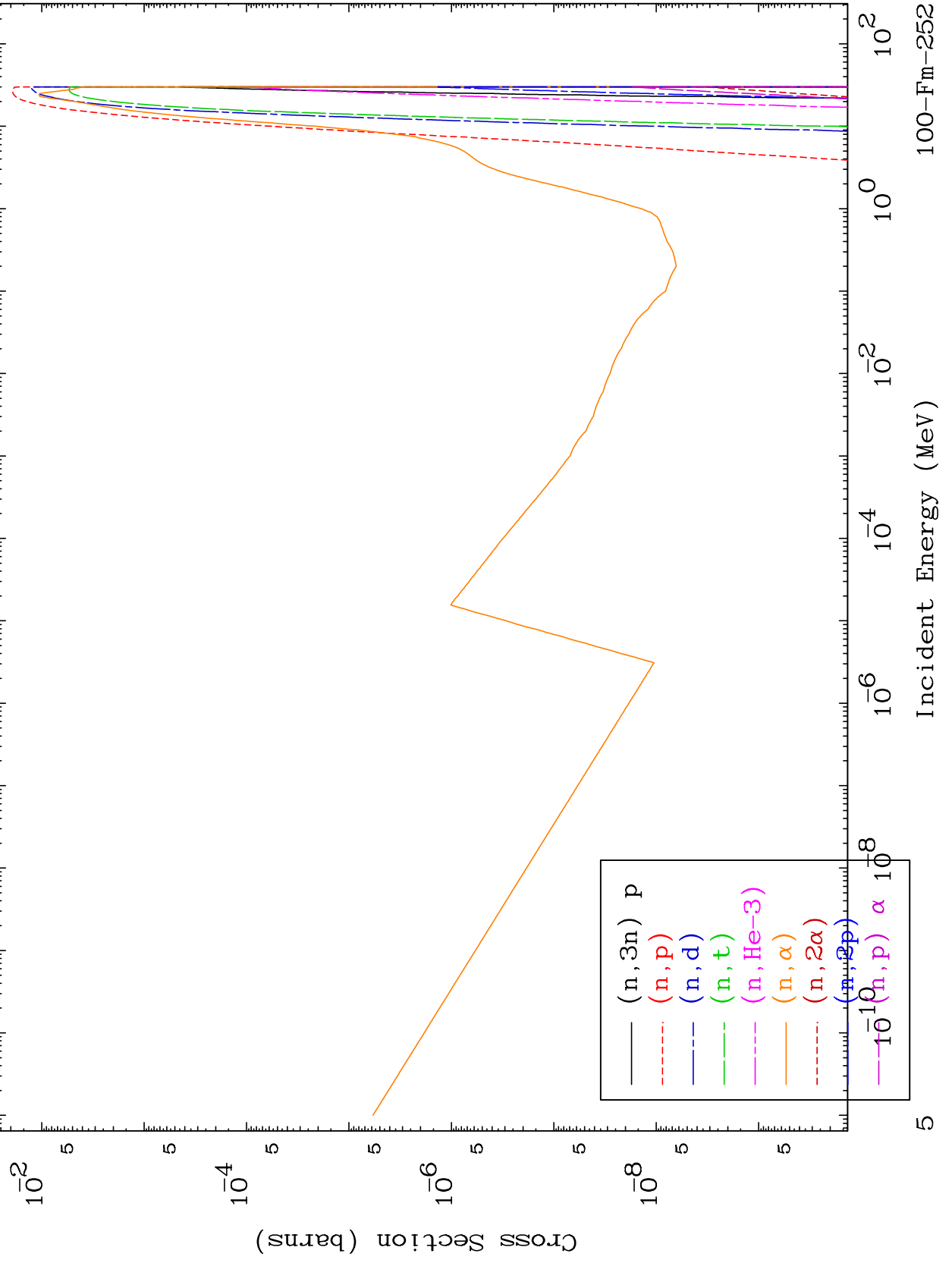
100-Fm-252



MAT 9933

Charged Particle  
293 Kelvin Cross Sections

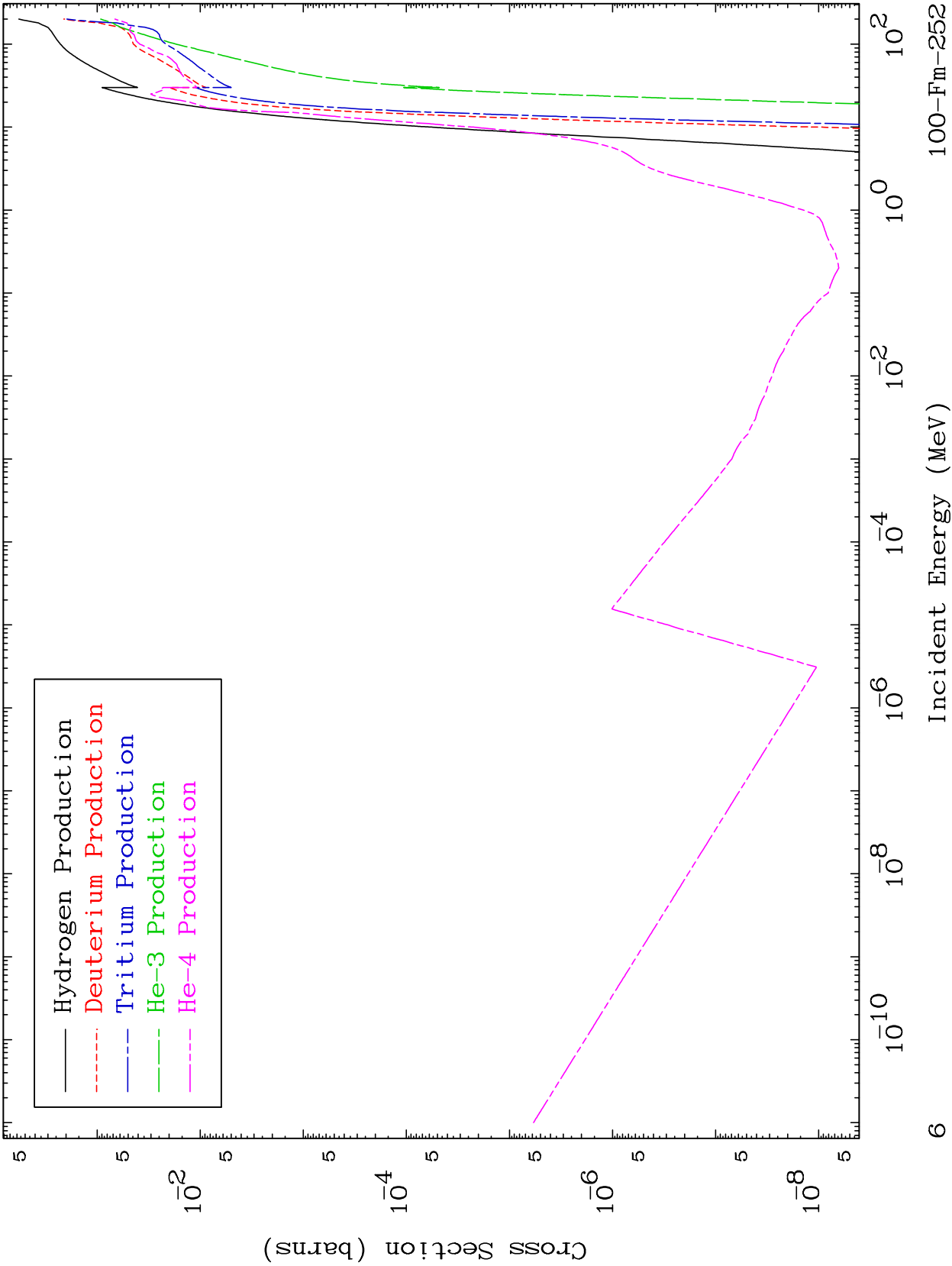
100-Fm-252



MAT 9933

Particle Production  
293 Kelvin Cross Sections

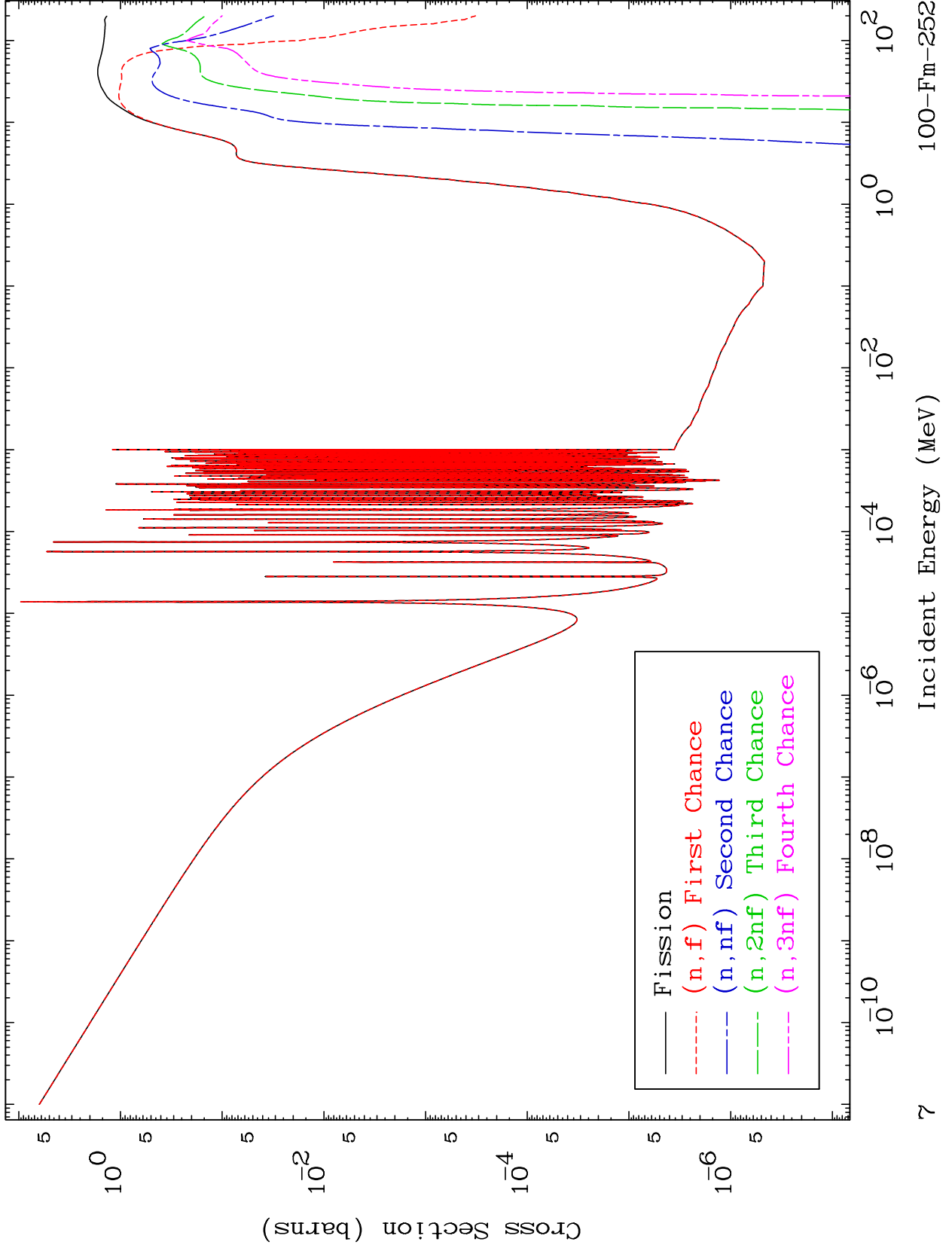
100-Fm-252



MAT 9933

Fission  
293 Kelvin Cross Sections

100-Fm-252



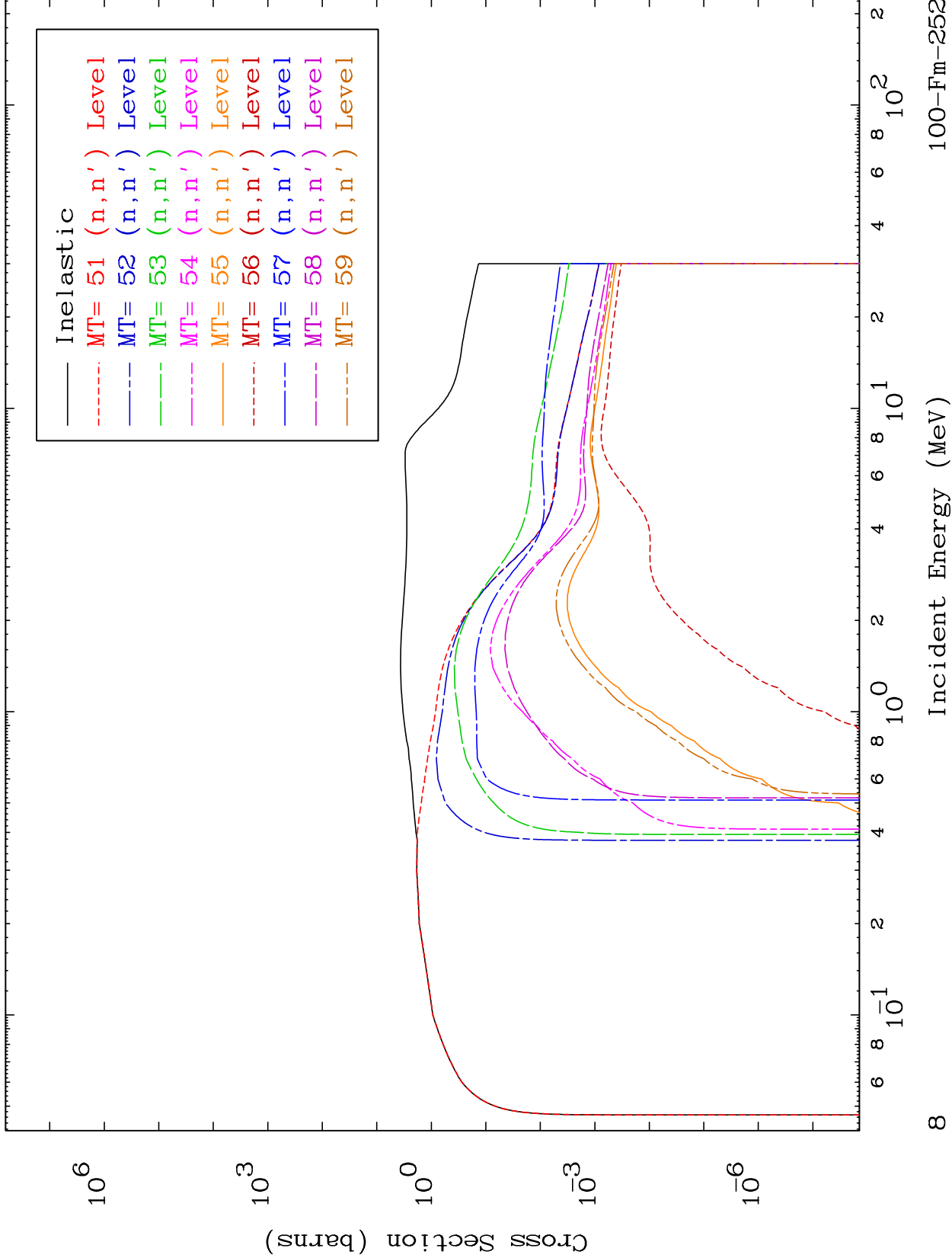


MAT 9933

(n,n') Level

100-Fm-252

293 Kelvin Cross Sections

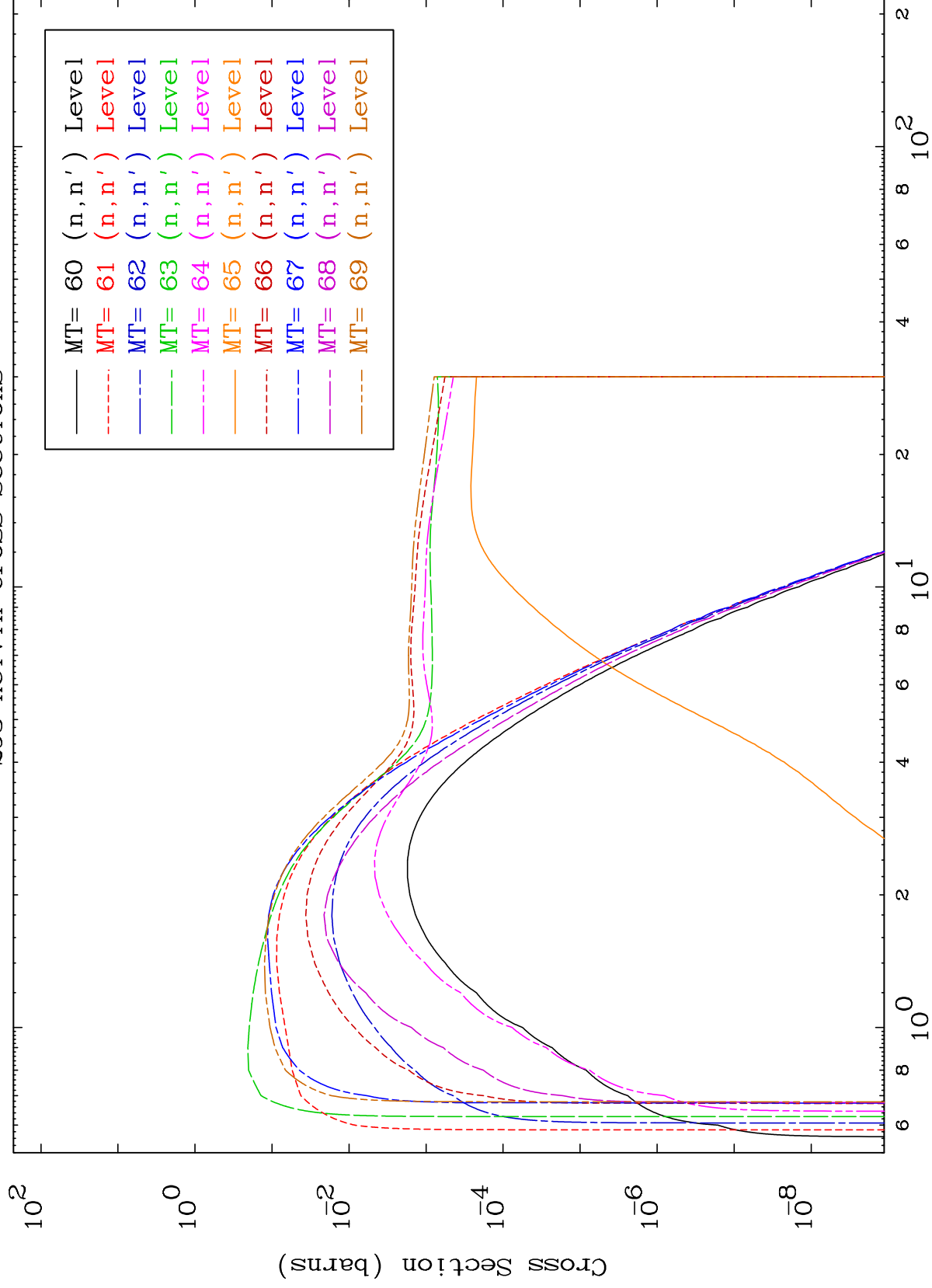


MAT 9933

(n,n') Level

100-Fm-252

293 Kelvin Cross Sections



9

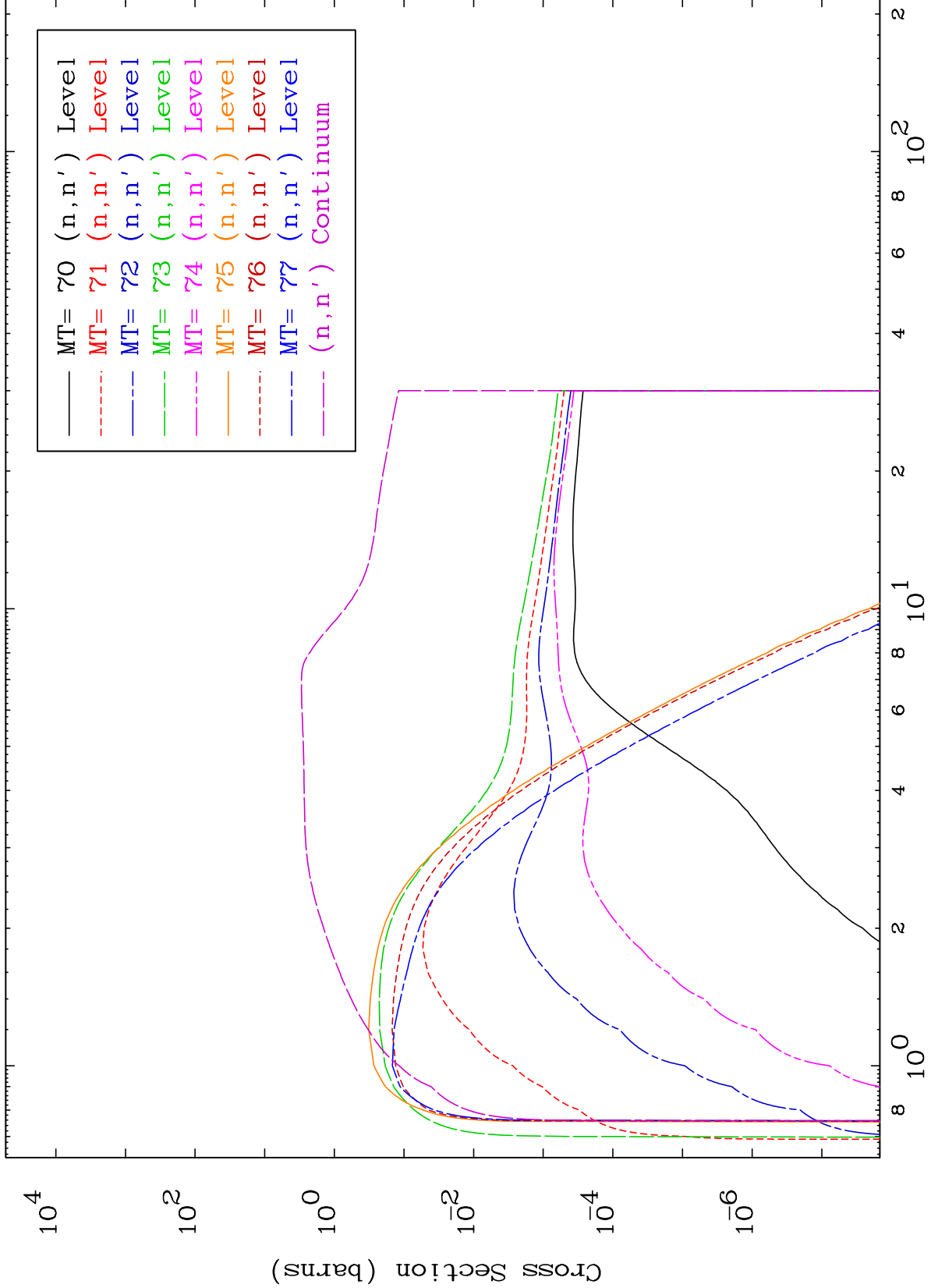
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,n') Level  
293 Kelvin Cross Sections

100-Fm-252



10

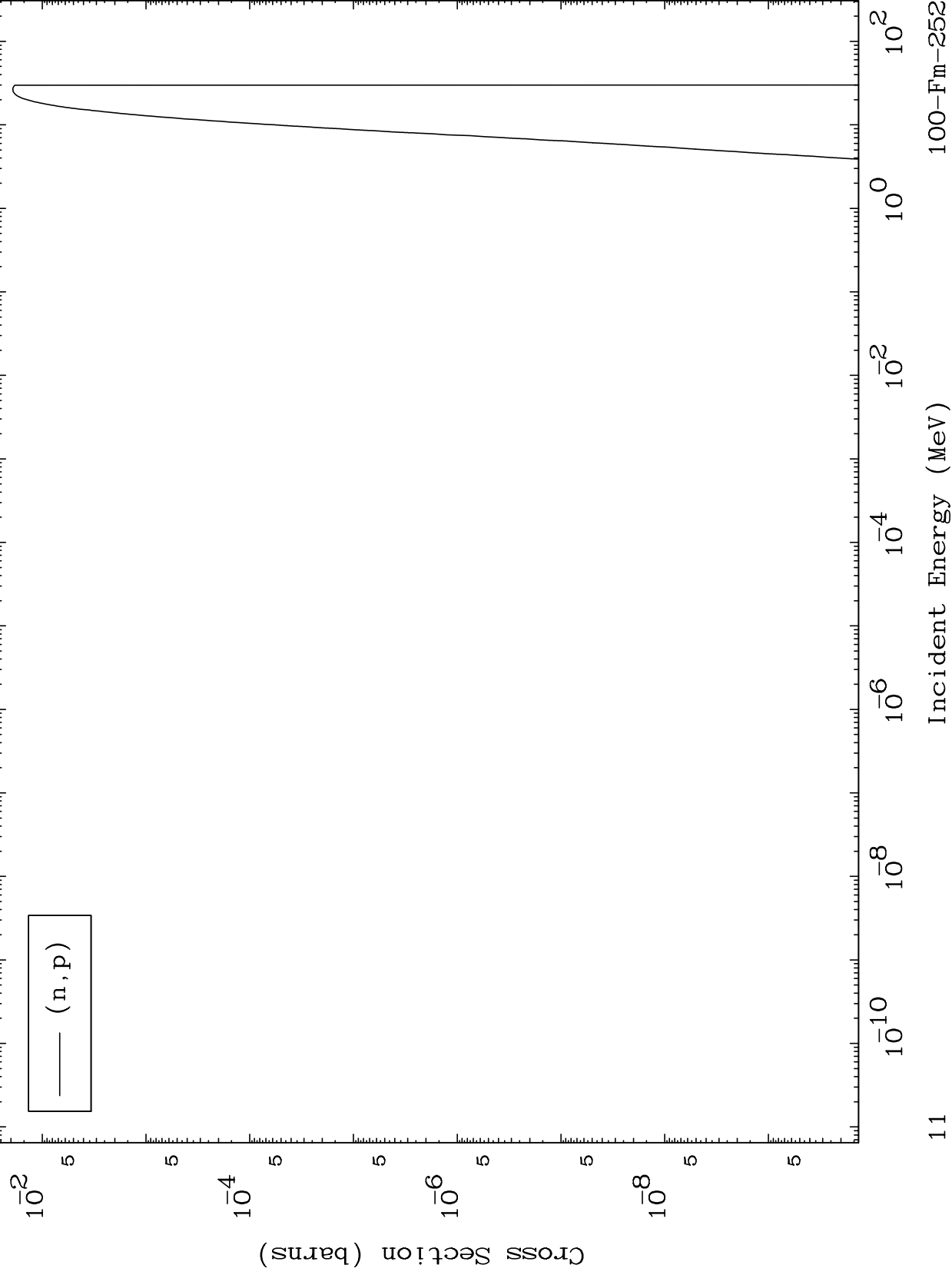
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,p) Levels  
293 Kelvin Cross Sections

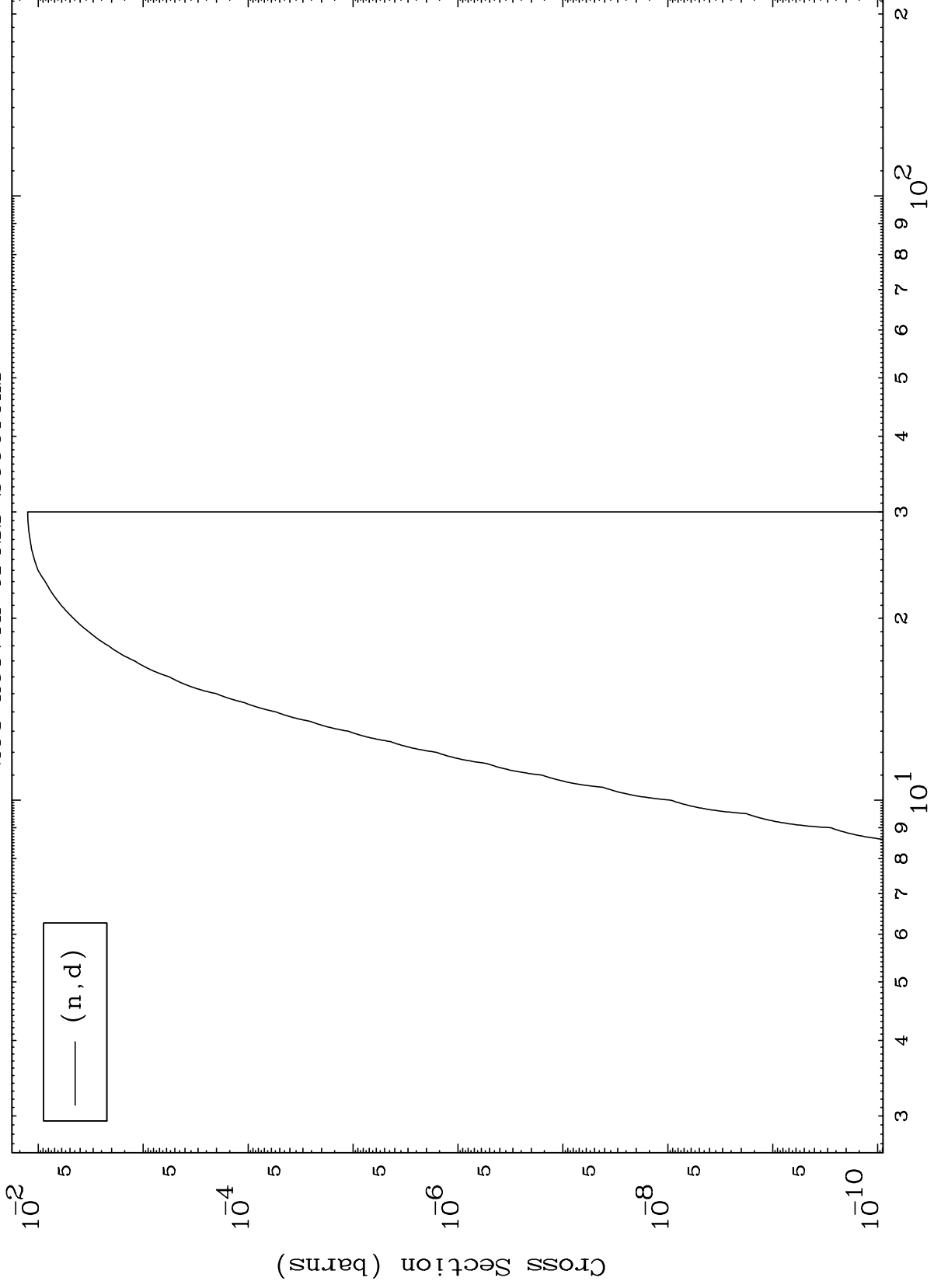
100-Fm-252



MAT 9933

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-252



12

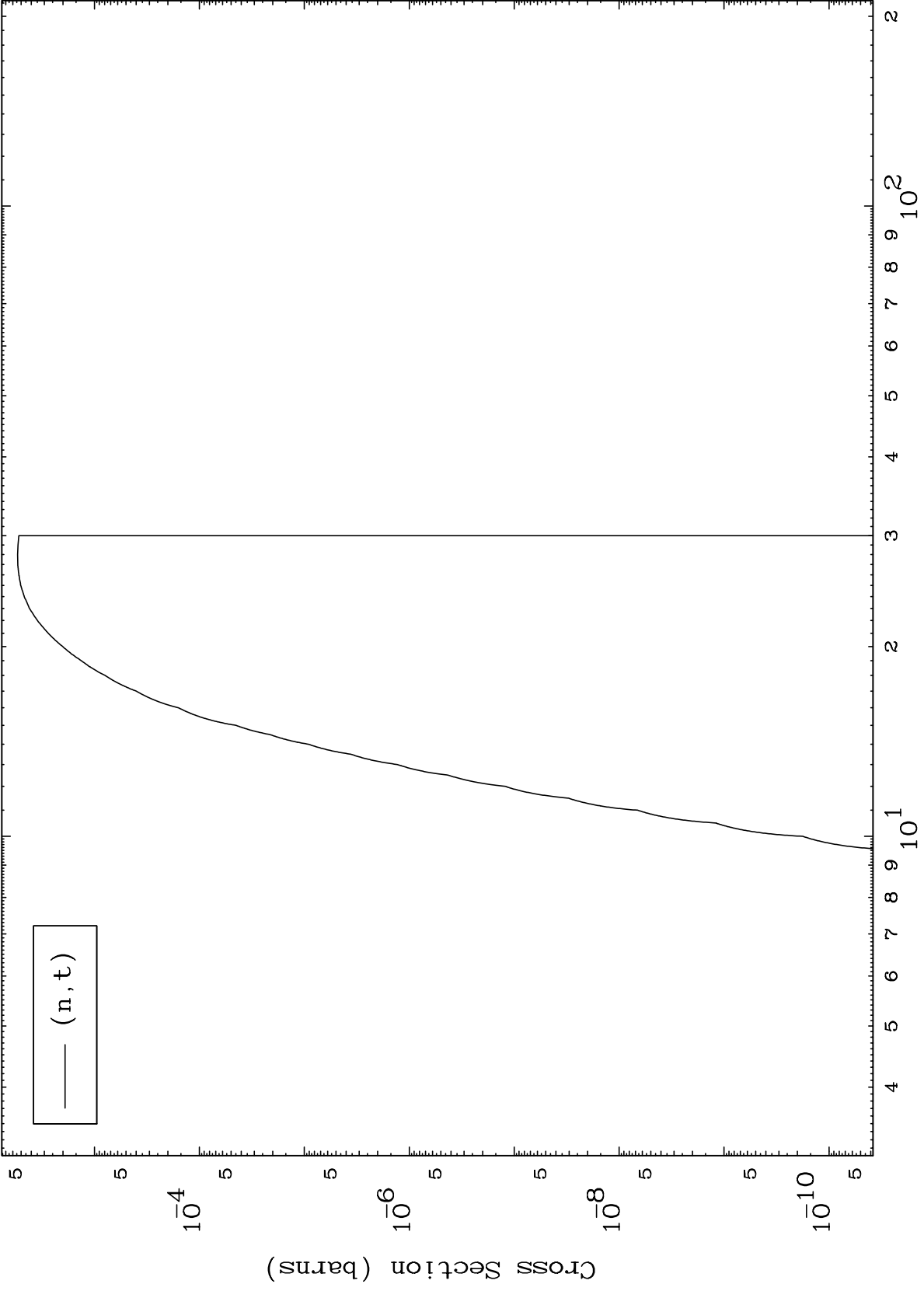
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-252



13

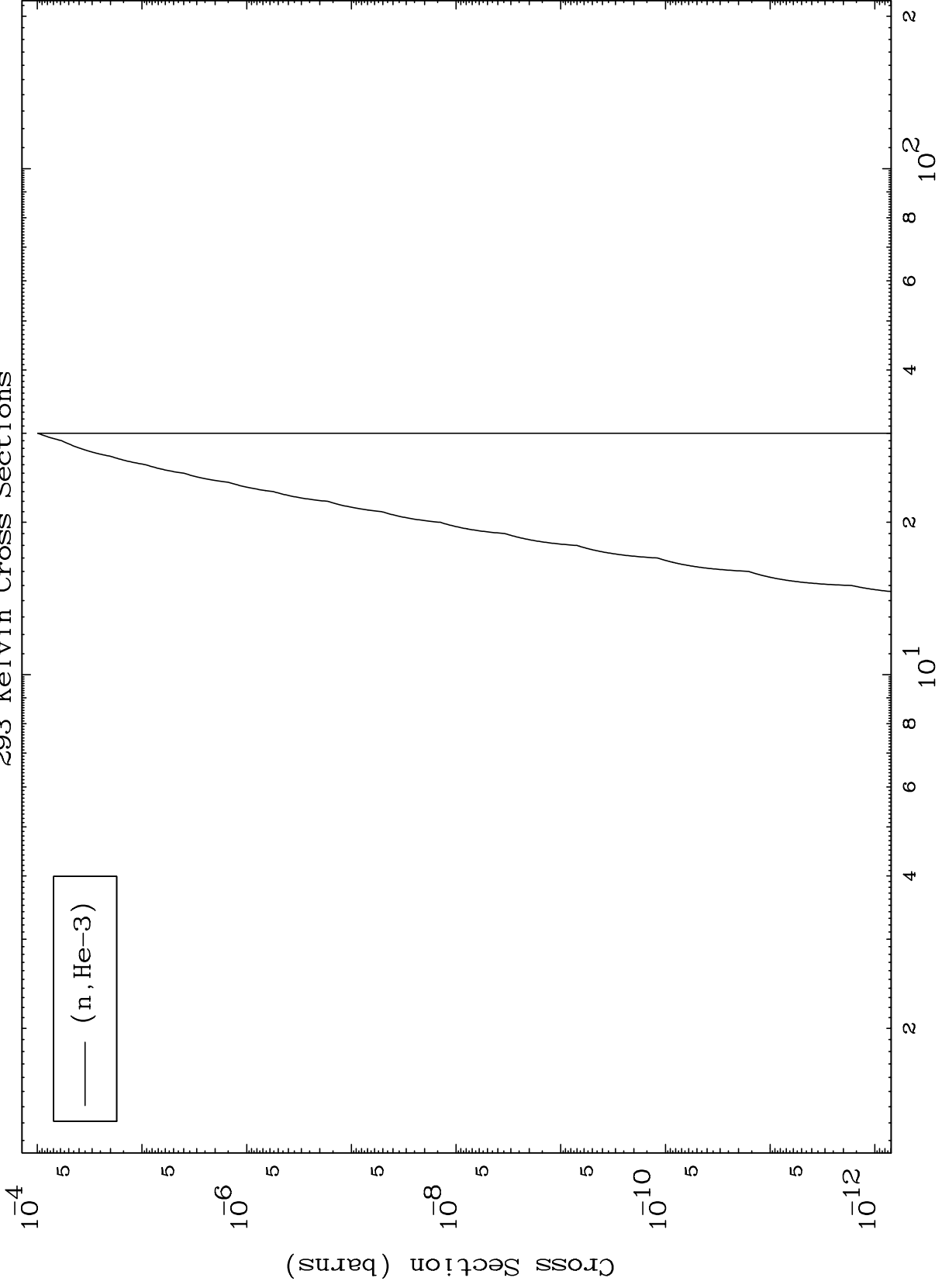
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,He3) Levels  
293 Kelvin Cross Sections

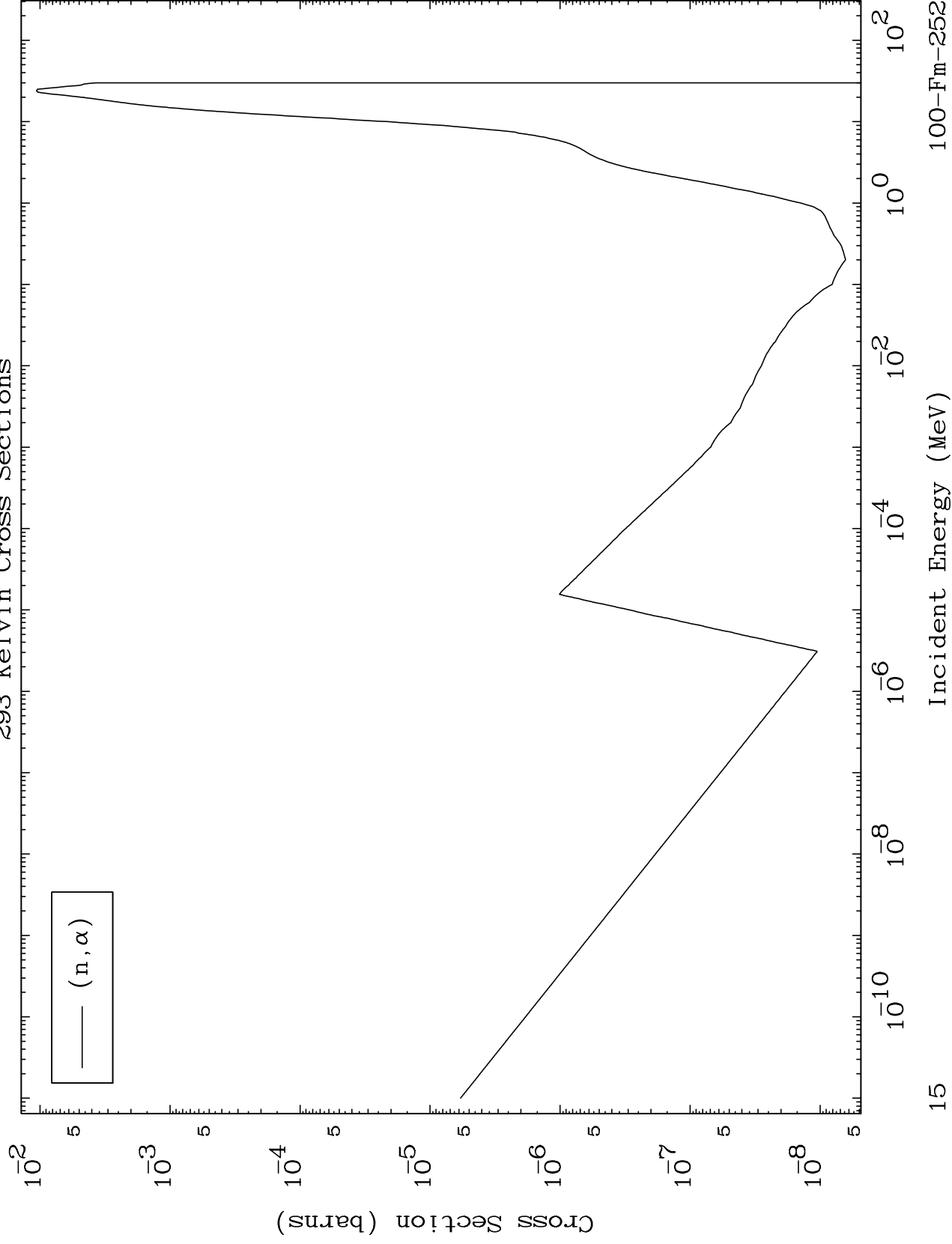
100-Fm-252



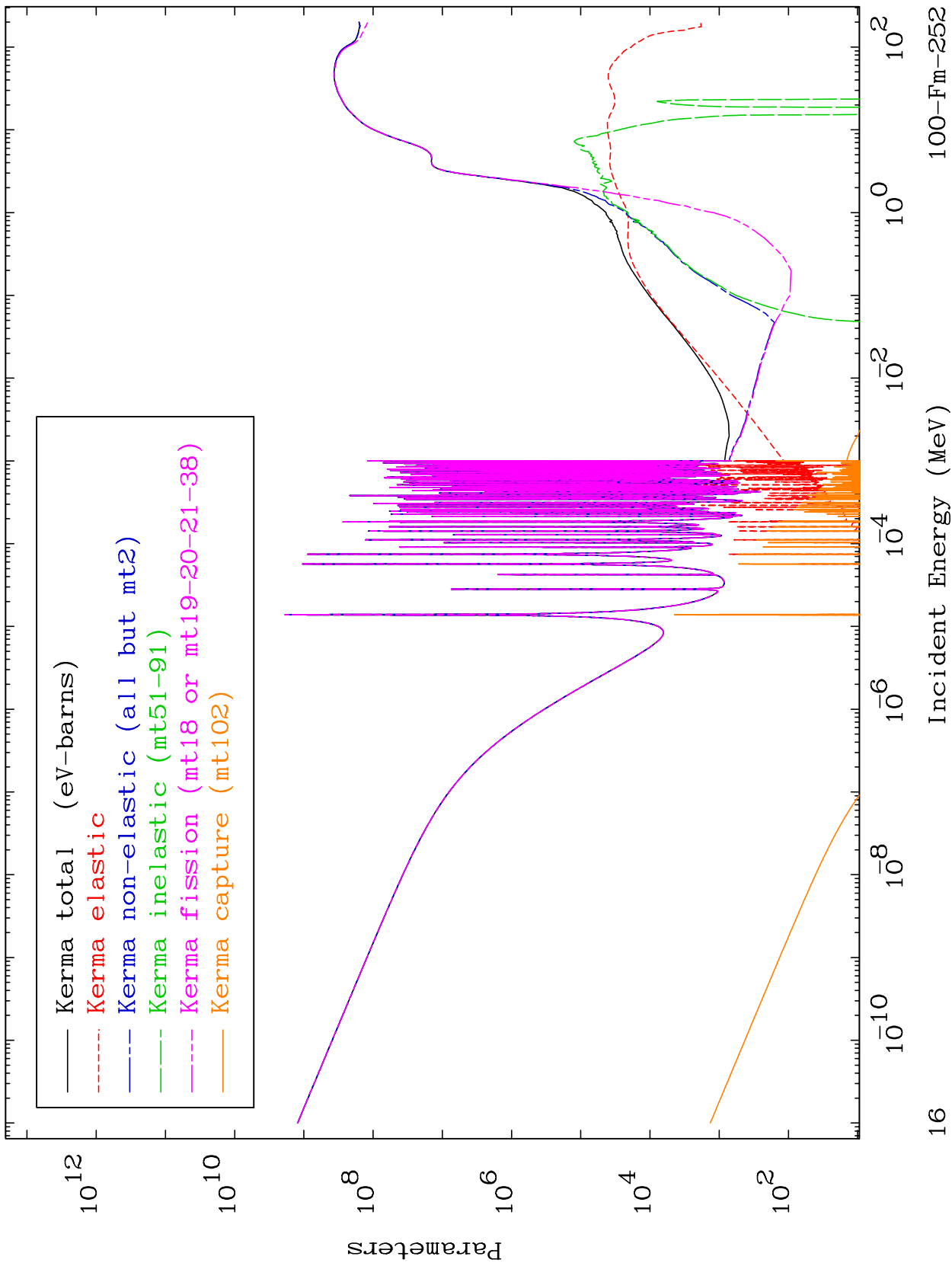
MAT 9933

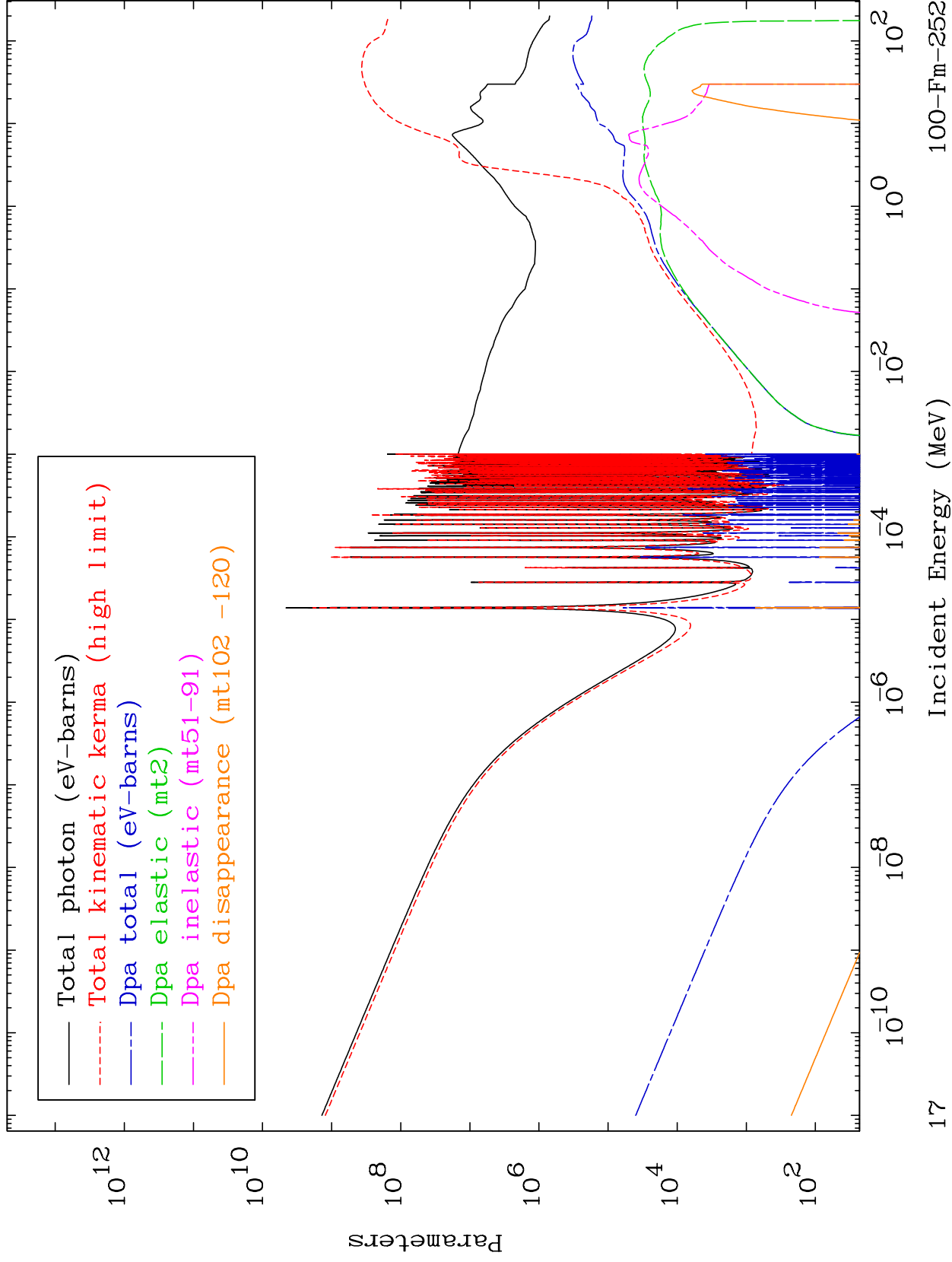
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

100-Fm-252





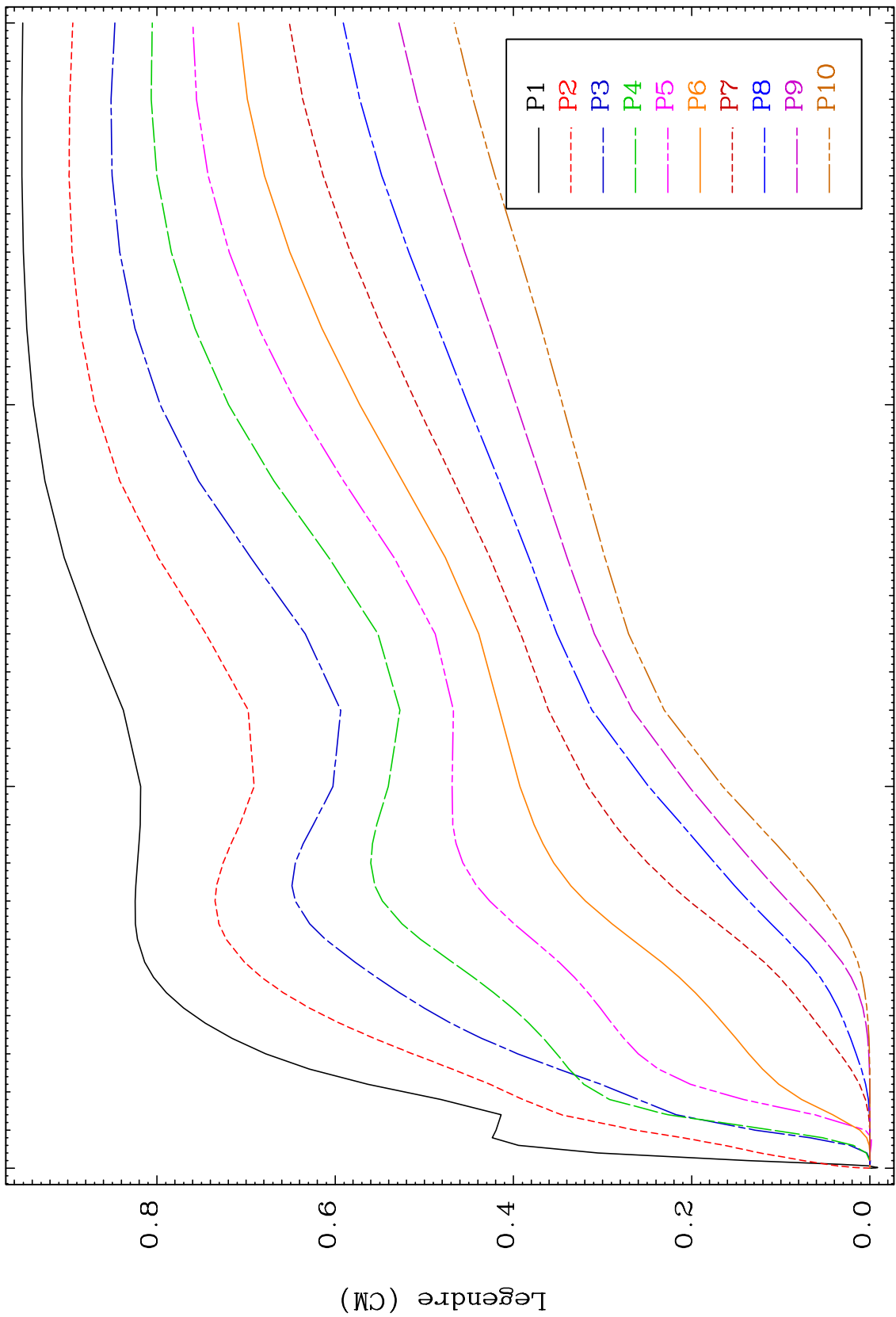




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Elastic Legendre Coefficients

100-Fm-252



18

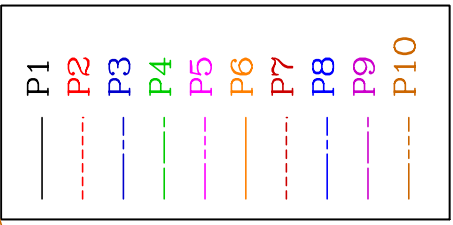
Incident Energy (MeV)

100-Fm-252

30

10

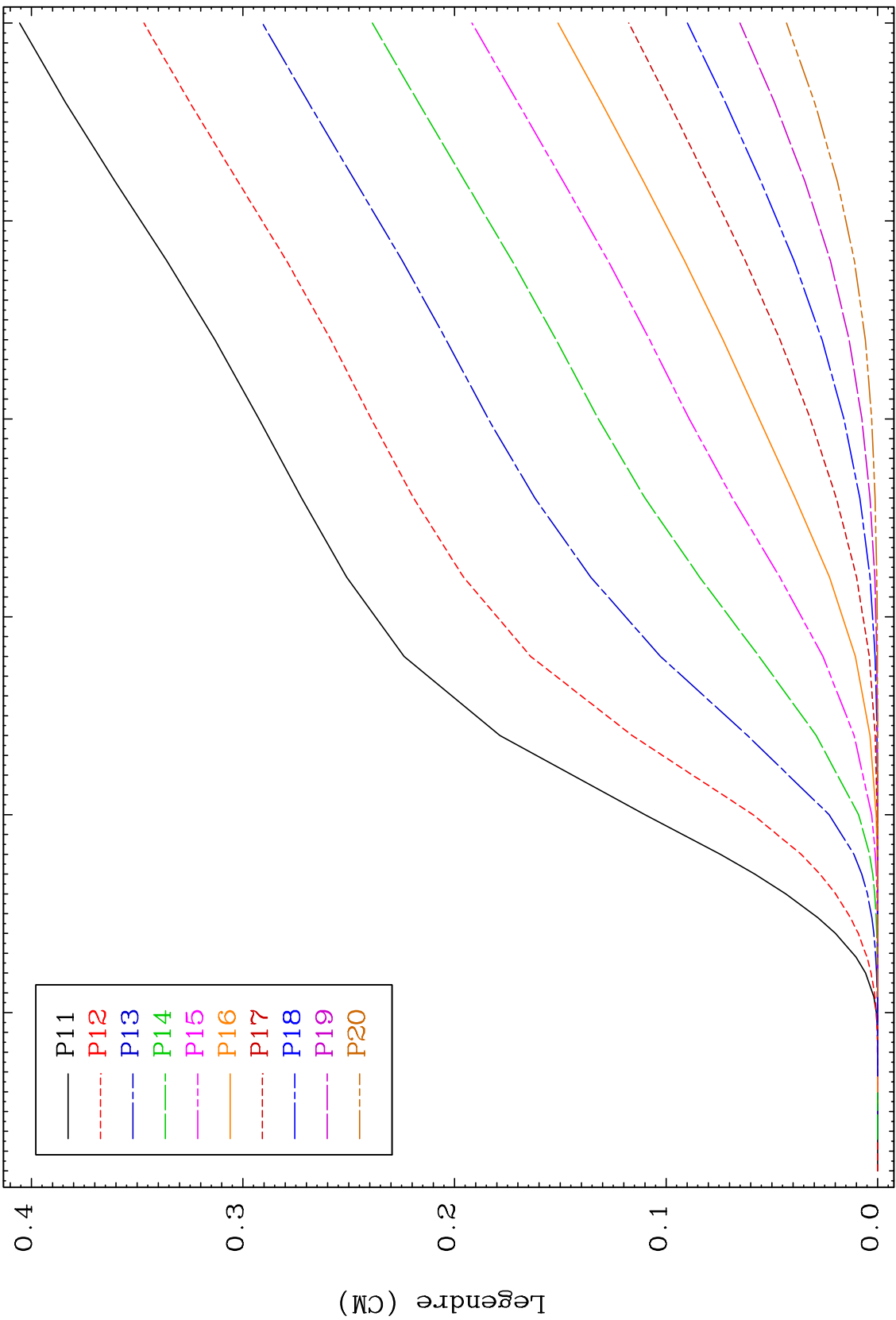
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Elastic Legendre Coefficients

100-Fm-252



19

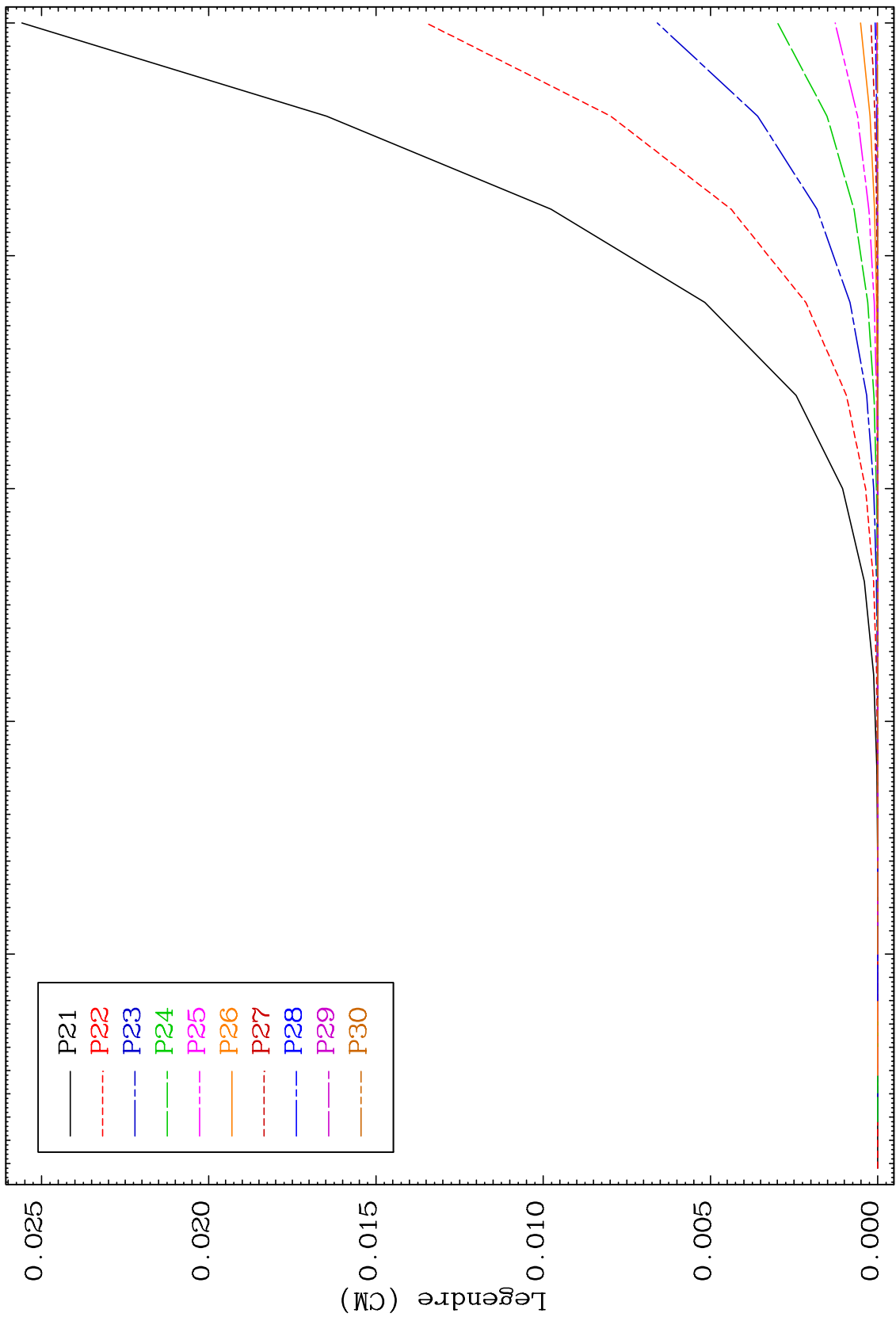
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic  
Legendre Coefficients

100-Fm-252



20

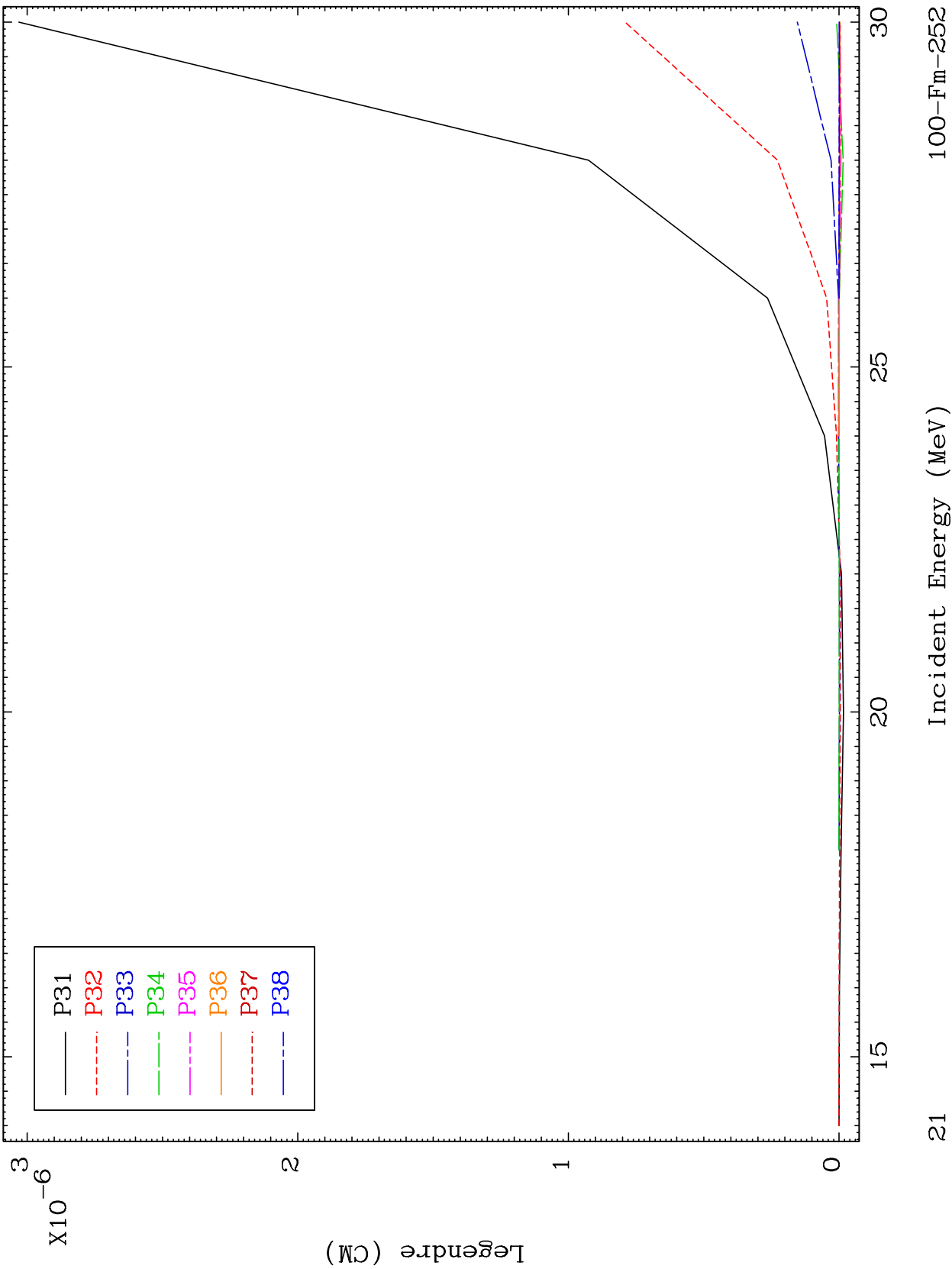
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic Legendre Coefficients

100-Fm-252



21

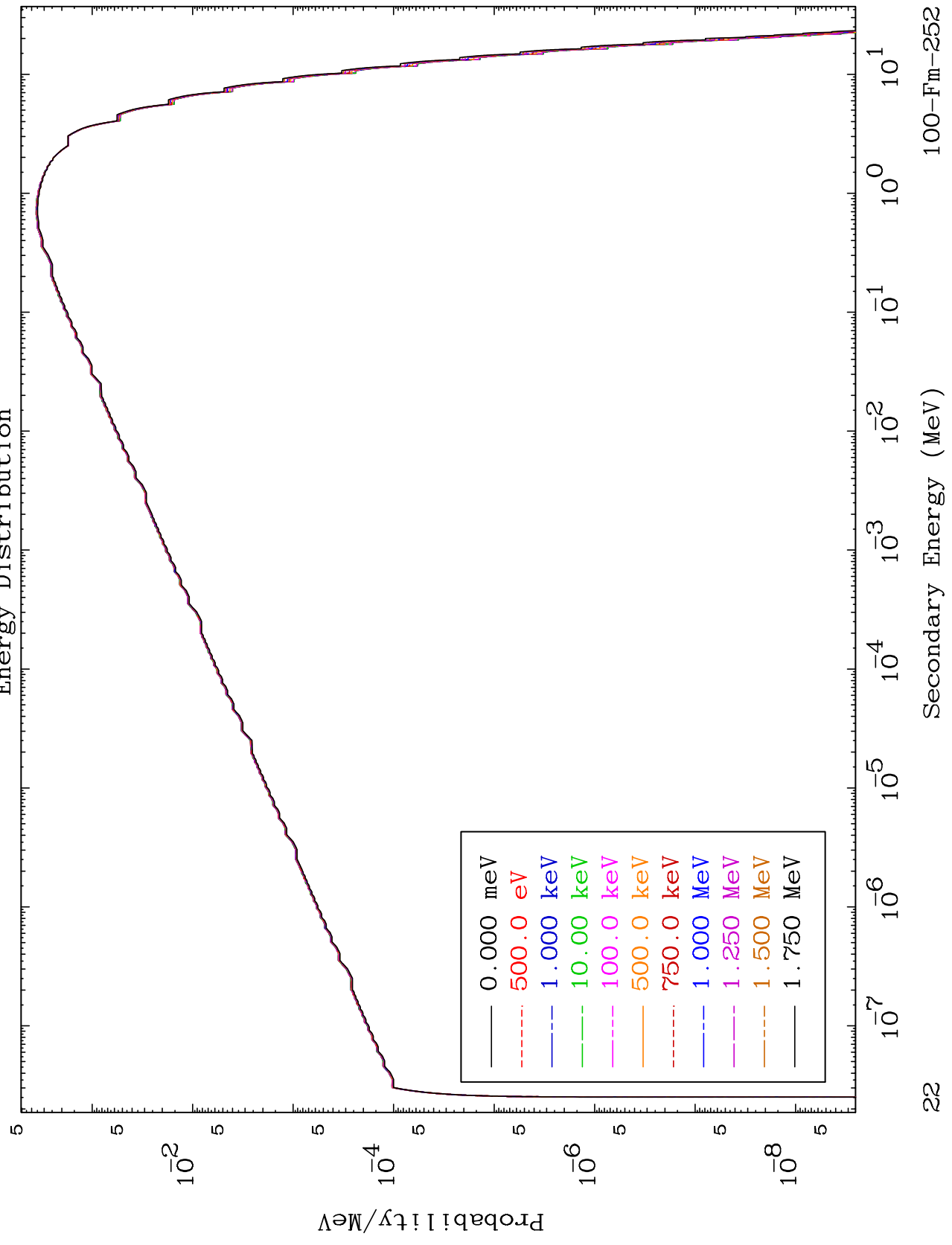
Incident Energy (MeV)

100-Fm-252

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### Fission Energy Distribution

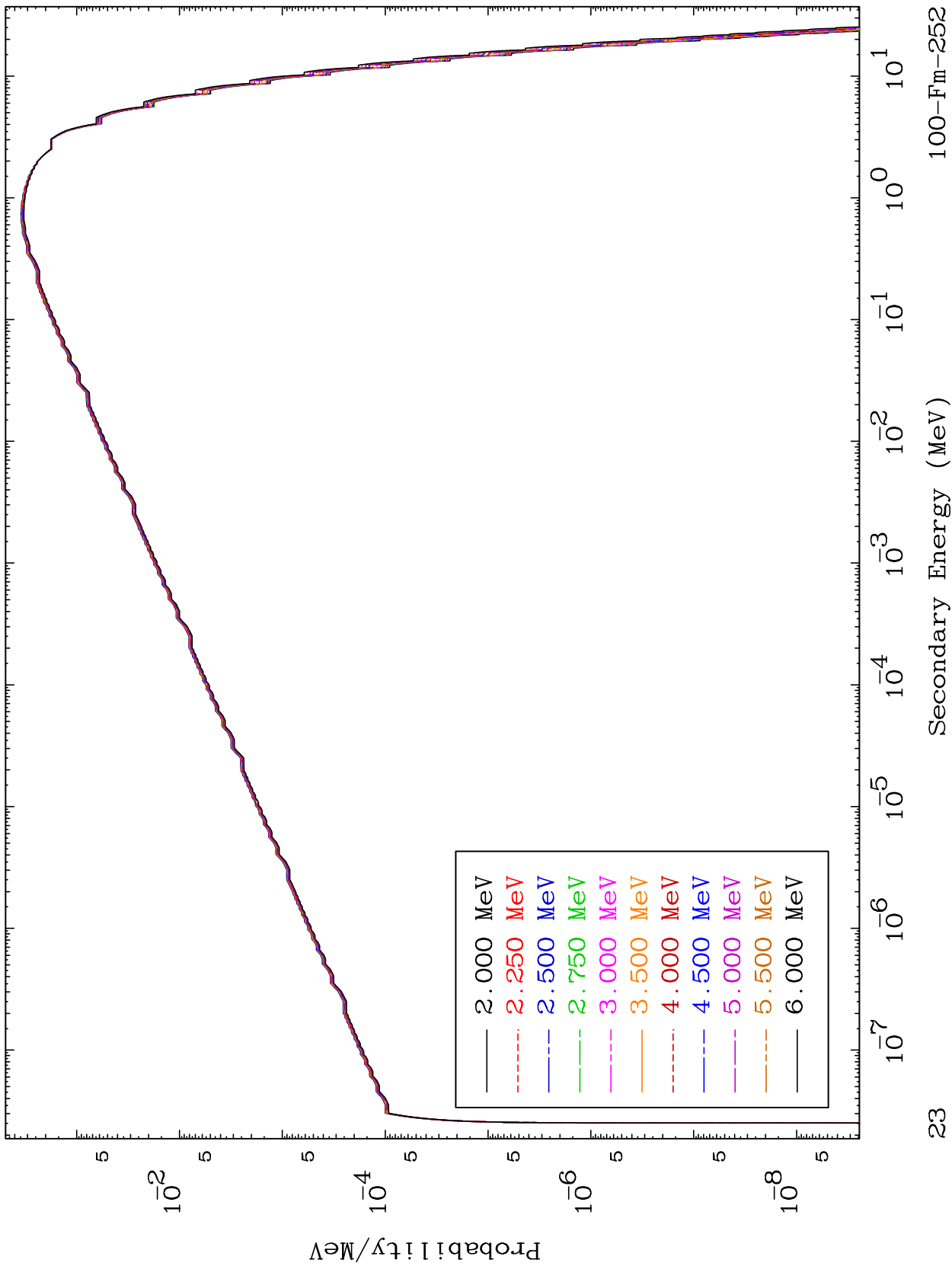
100-Fm-252



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### Fission Energy Distribution

100-Fm-252

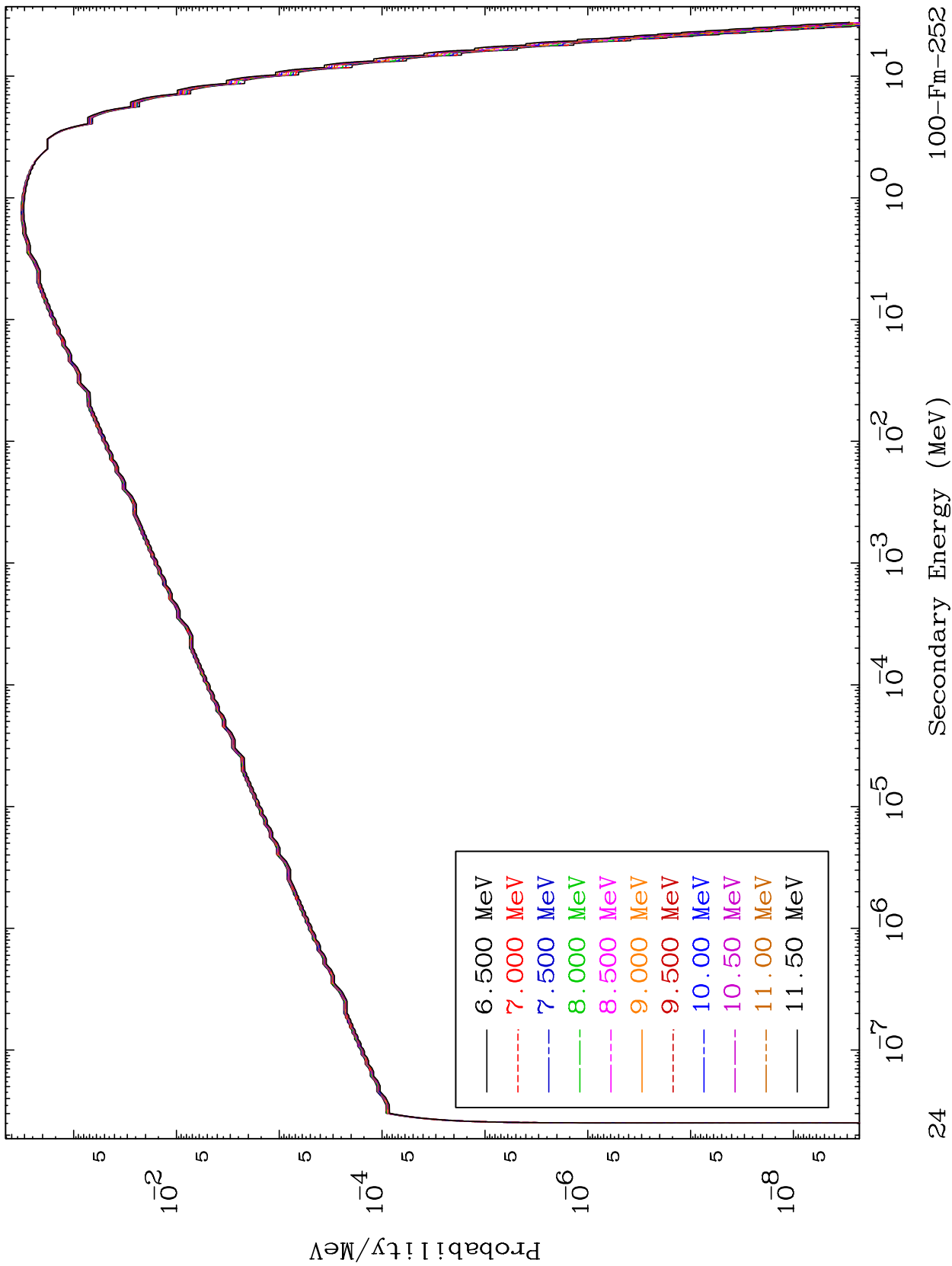




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### Fission Energy Distribution

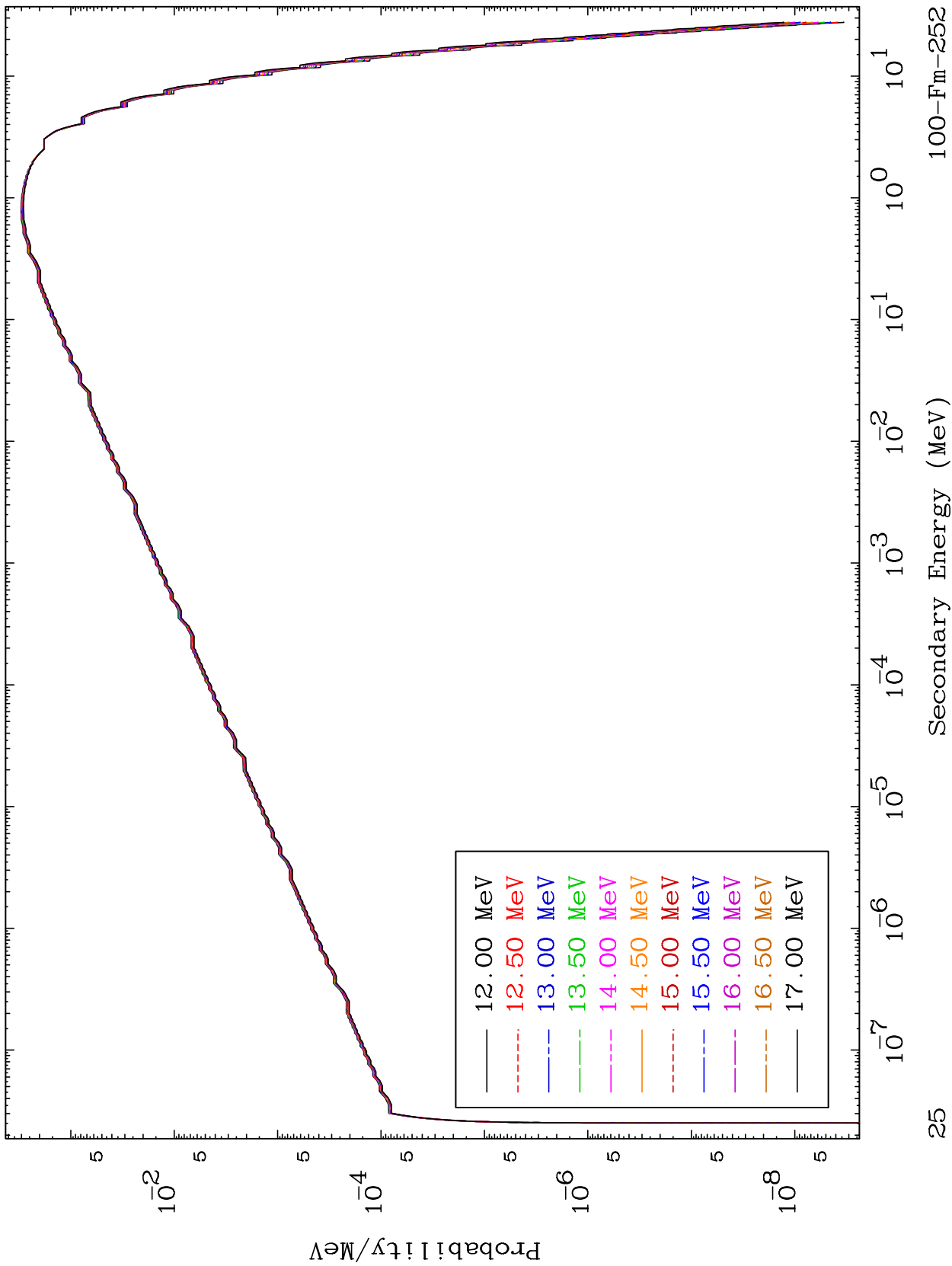
100-Fm-252



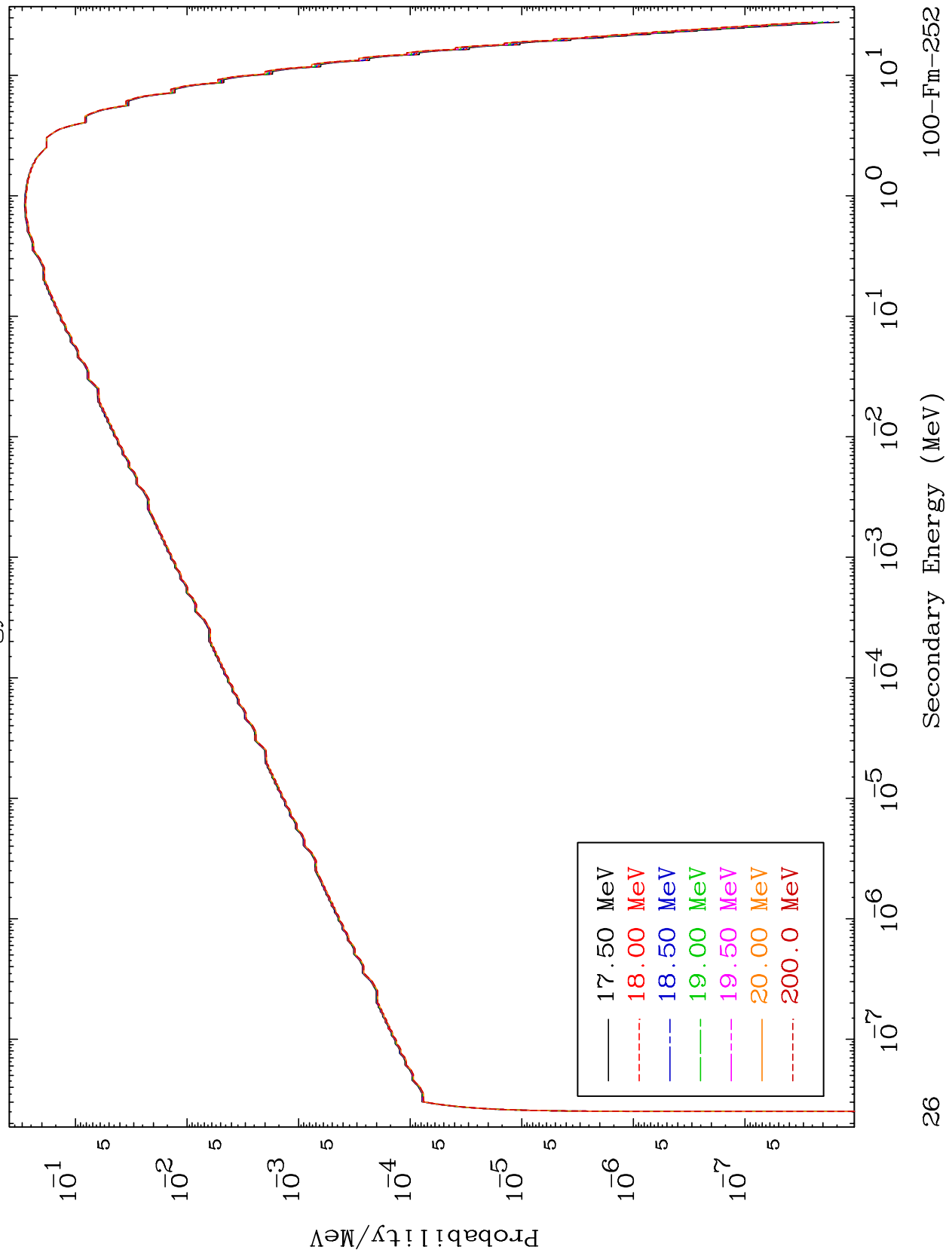
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### Fission Energy Distribution

100-Fm-252



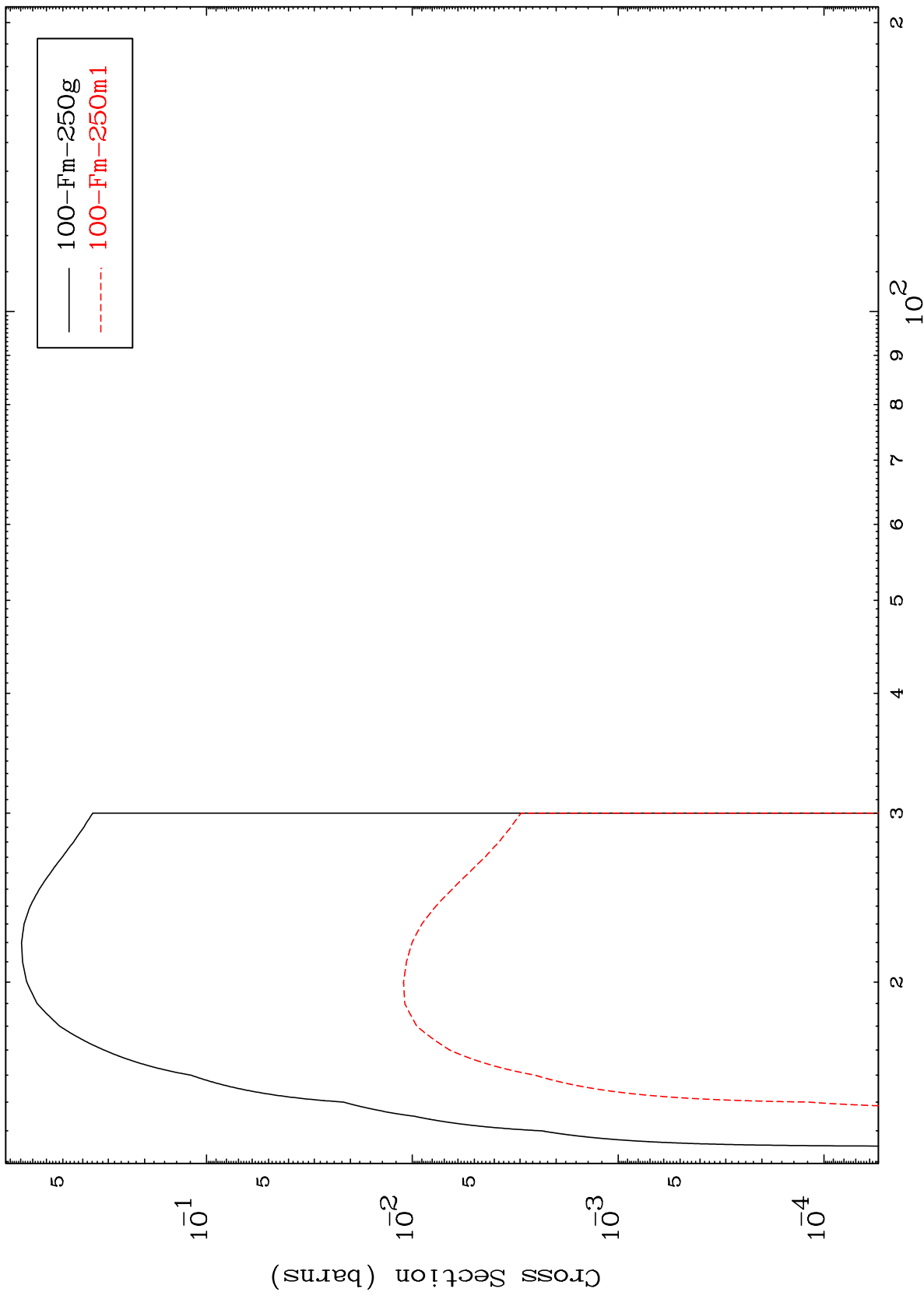
Fission  
Energy Distribution



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100-Fm-252

(n,3n)  
Radionuclide Production Cross Section



27

100-Fm-252

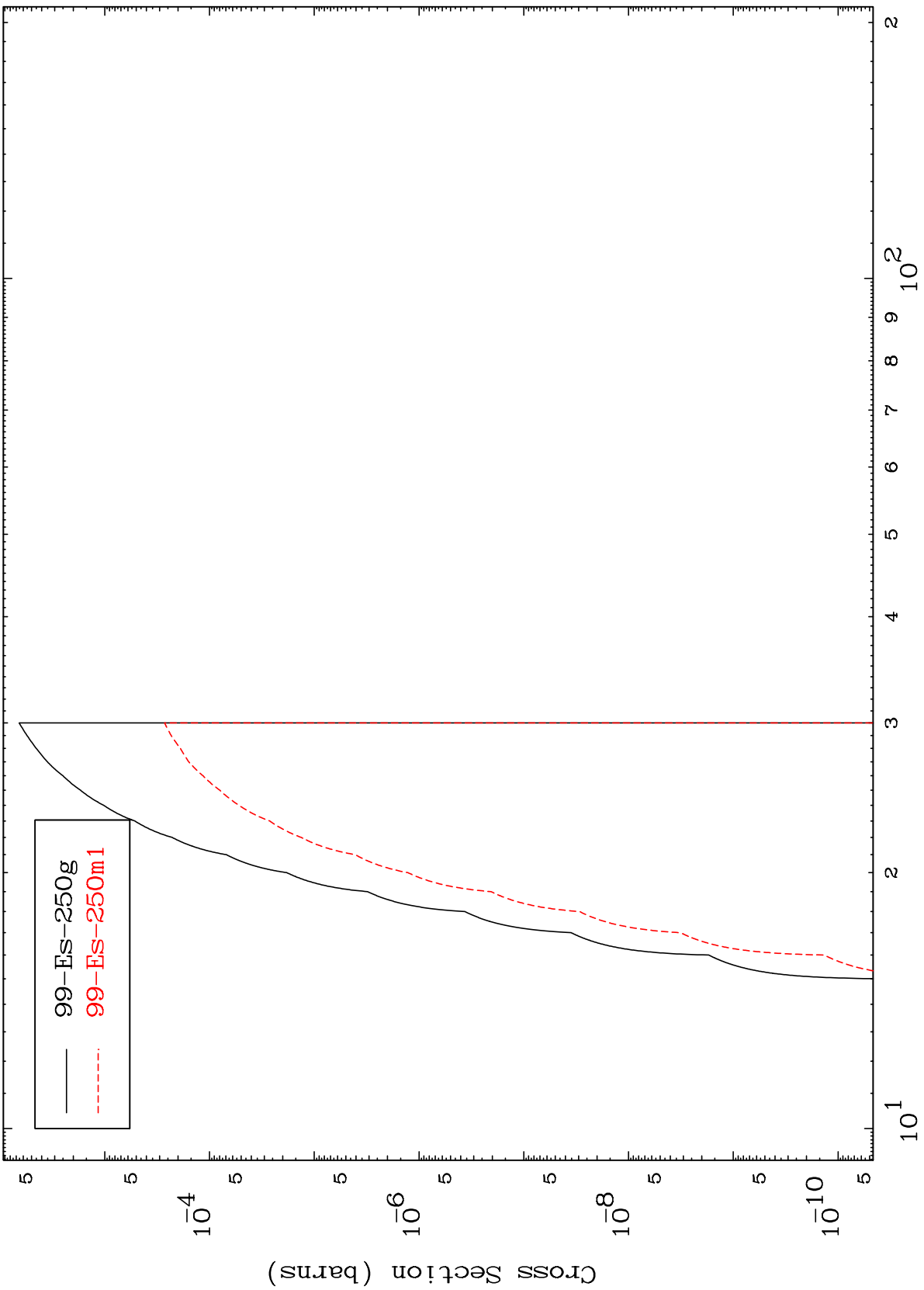
Incident Energy (MeV)

MAT 9933

(n,n') d

100-Fm-252

Radionuclide Production Cross Section



28

Incident Energy (MeV)

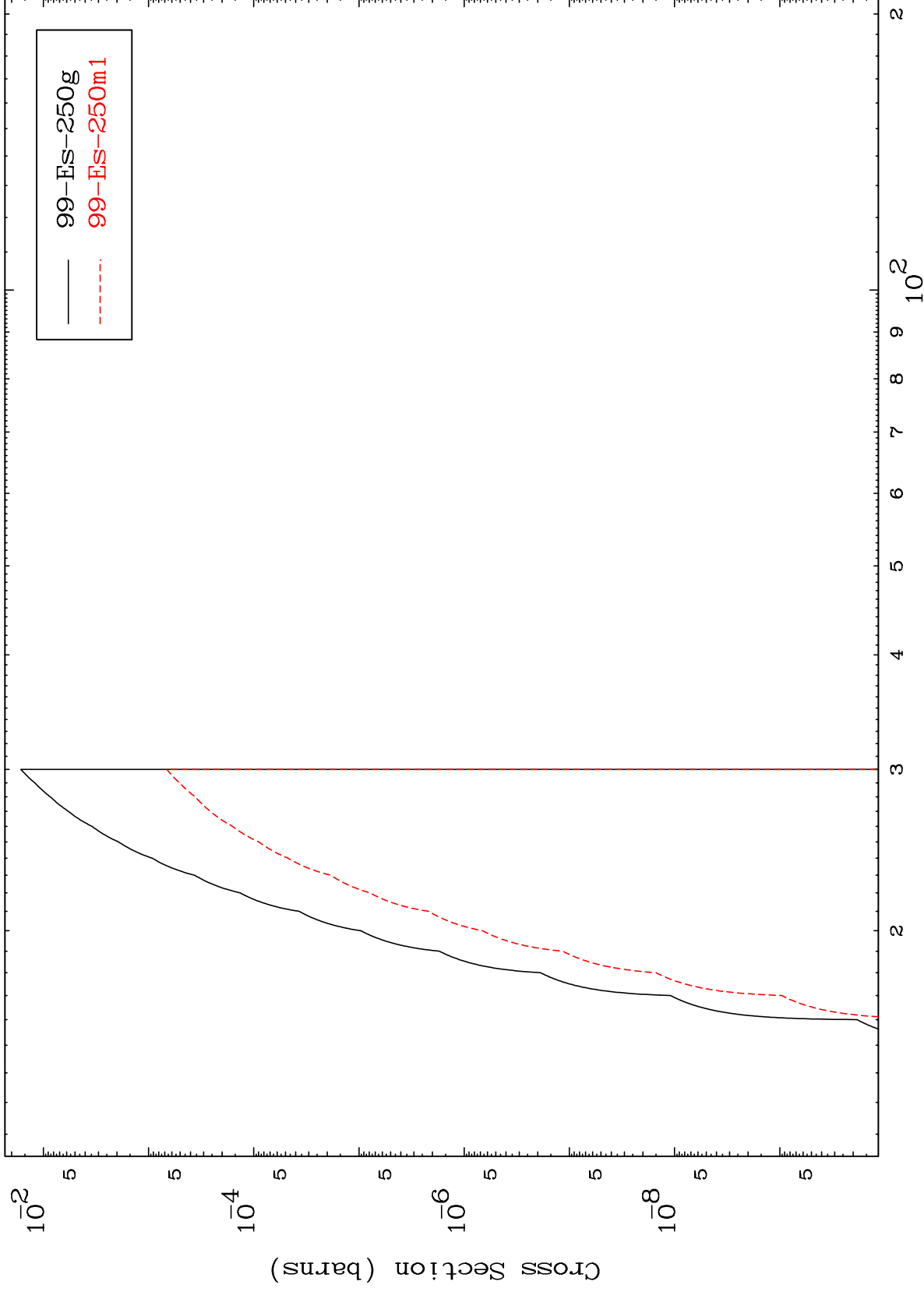
100-Fm-252

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(n,2n) p

100-Fm-252

Radionuclide Production Cross Section



29

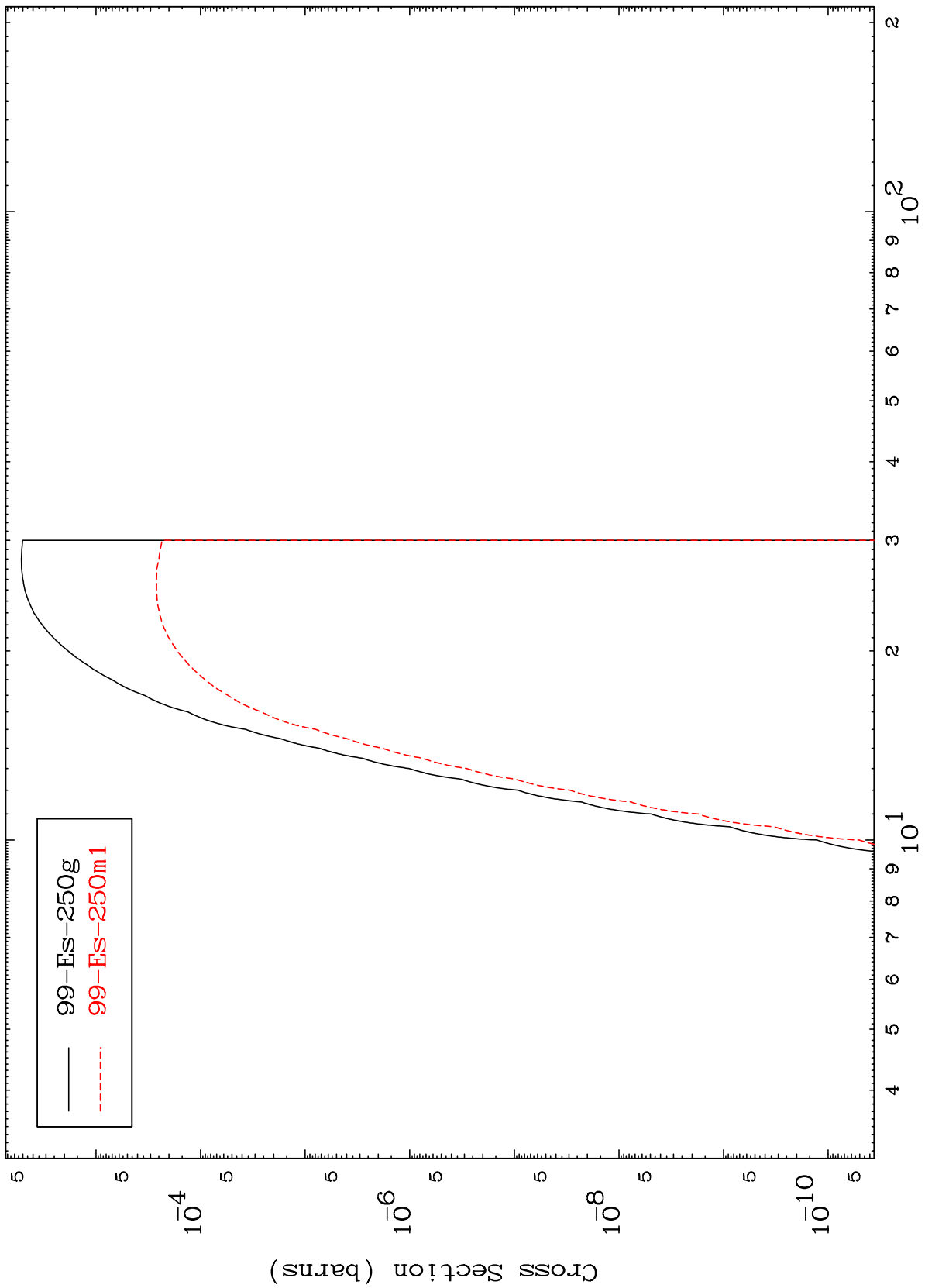
Incident Energy (MeV)

100-Fm-252

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100-Fm-252

(n, t)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

100-Fm-252

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(n,p)  $\alpha$

100-Fm-252

Radionuclide Production Cross Section

