

Program EVALPLOT
(Version 2017-1)

by

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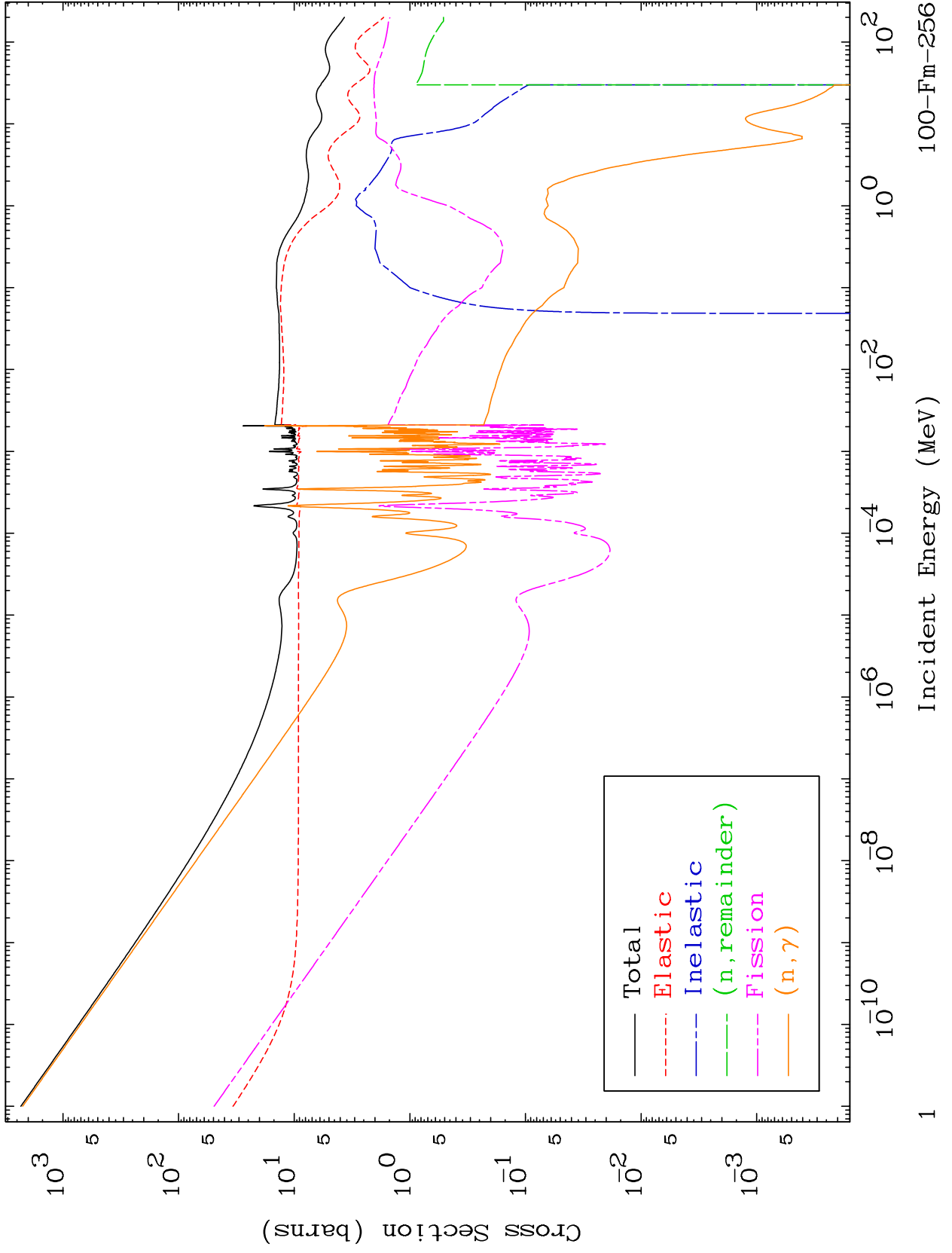
Press Mouse Button to Start

MAT 9937

Major

293 Kelvin Cross Sections

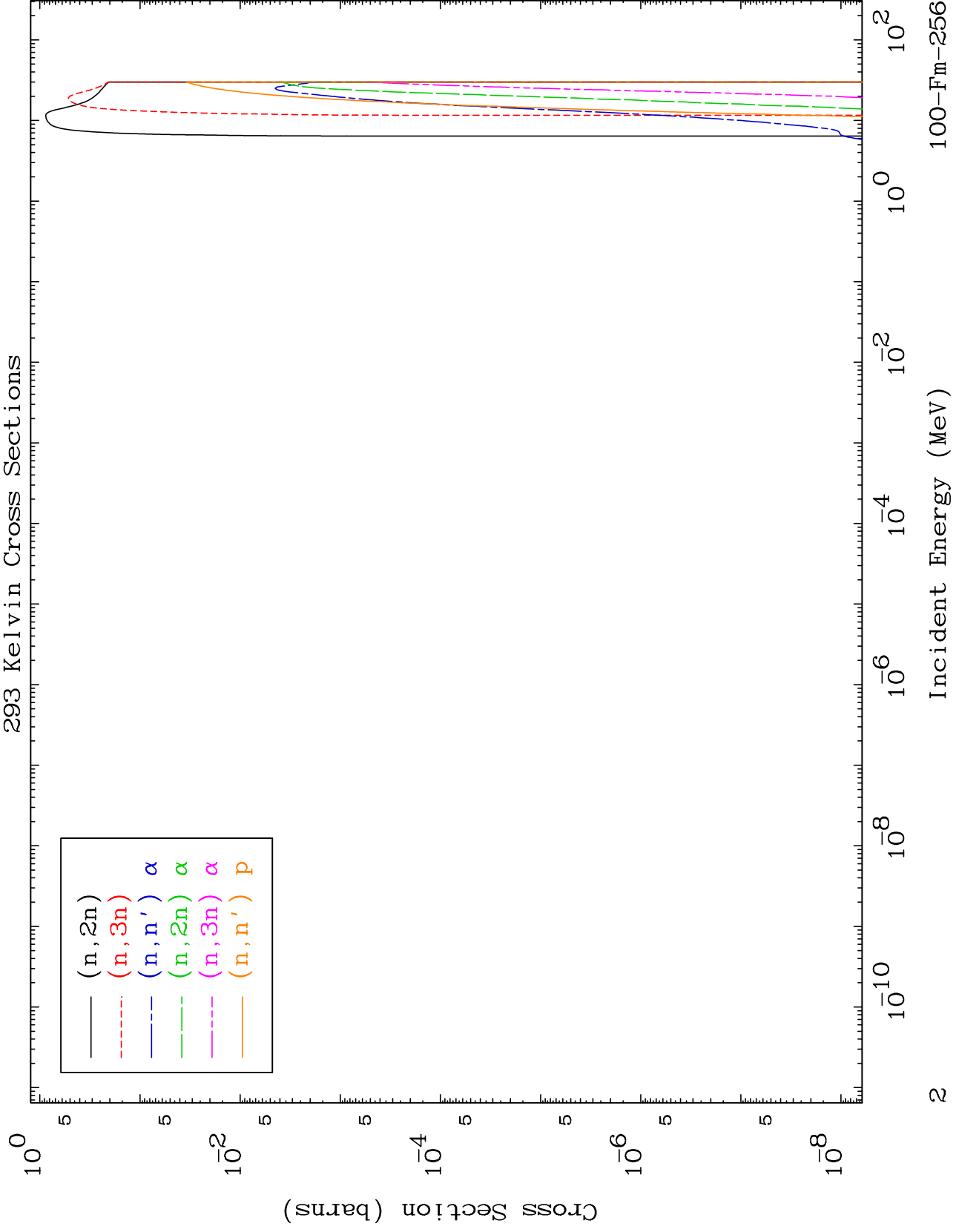
100-Fm-256

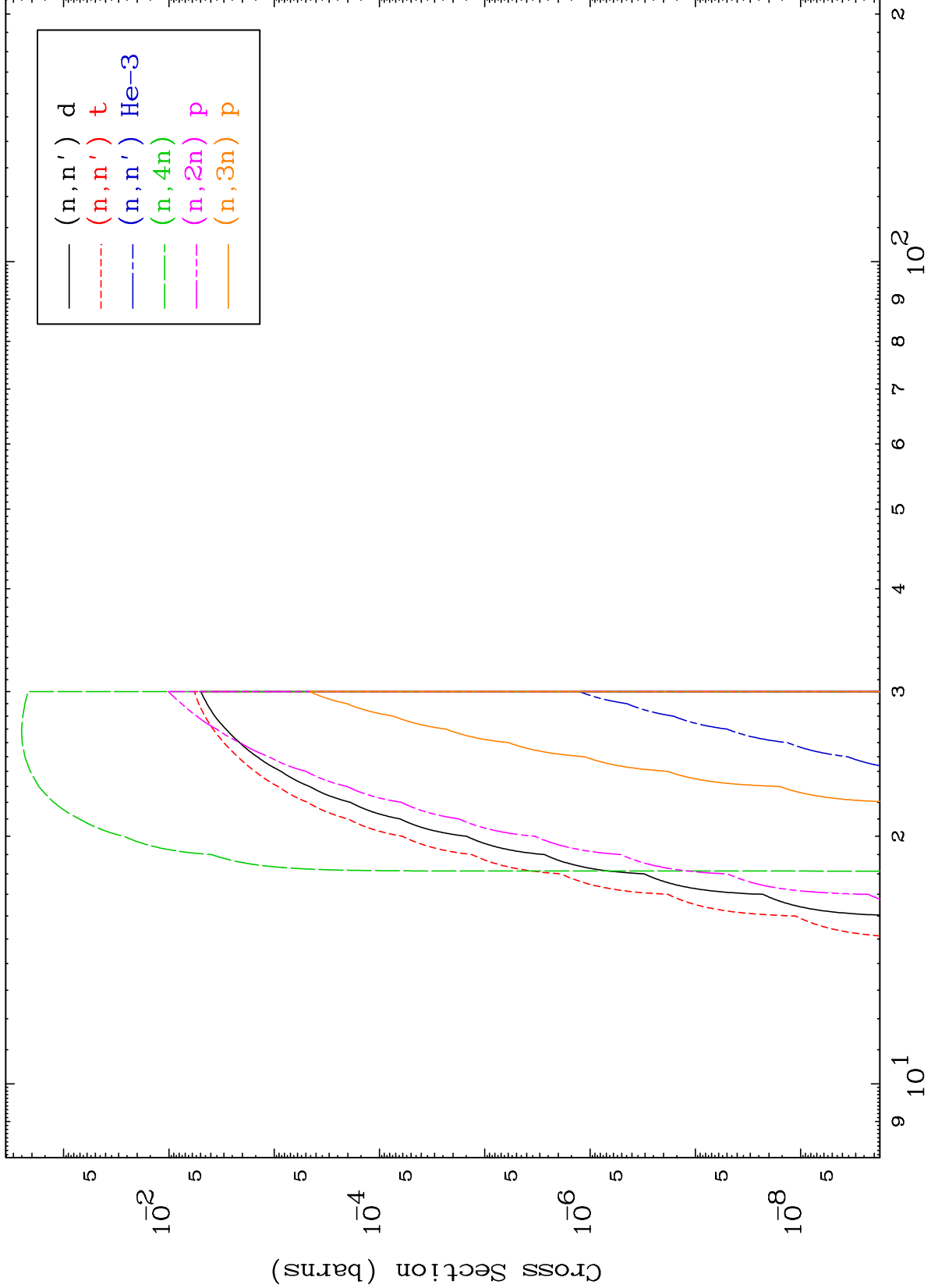


MAT 9937

Neutron Production
293 Kelvin Cross Sections

100-Fm-256

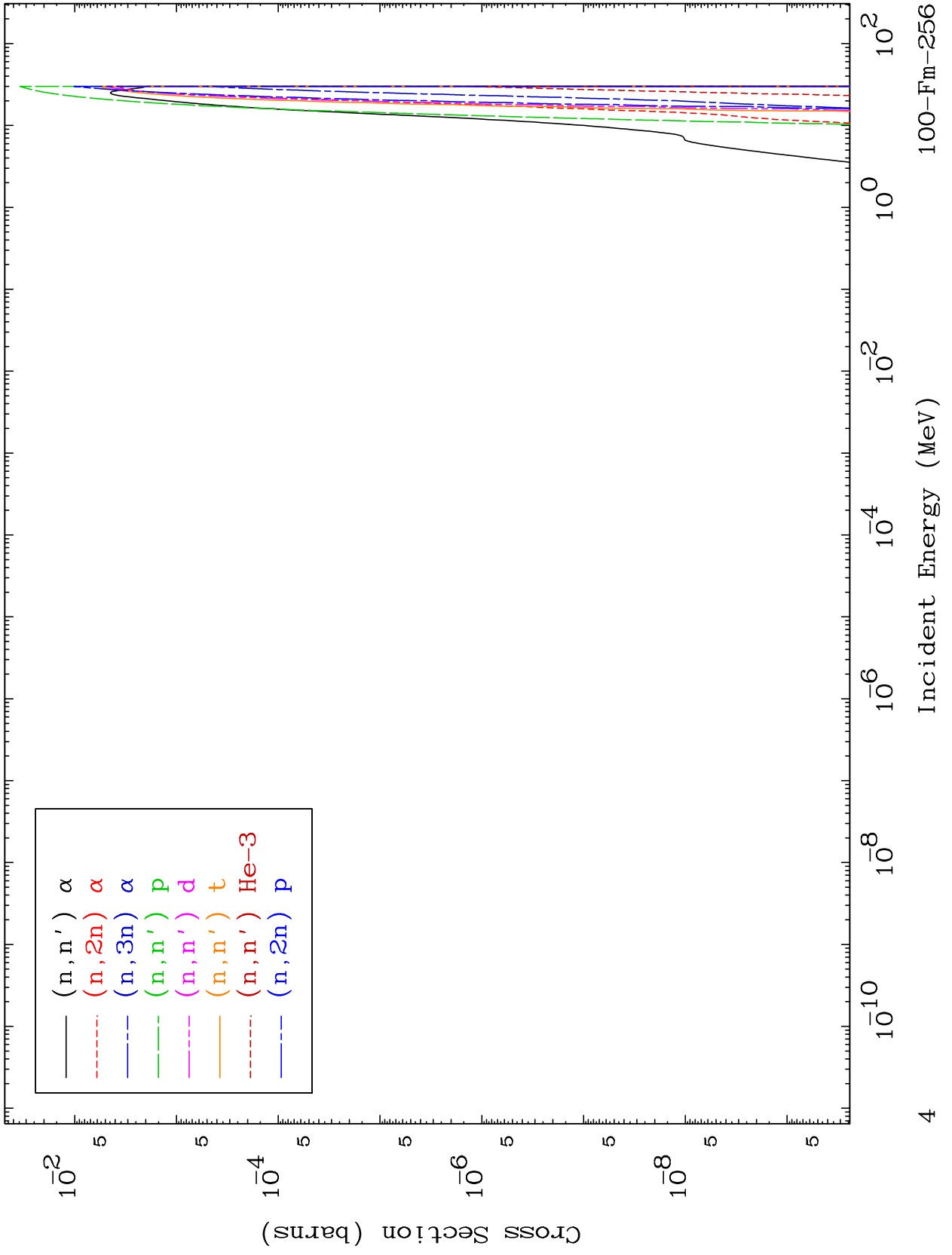




MAT 9937

Charged Particle
293 Kelvin Cross Sections

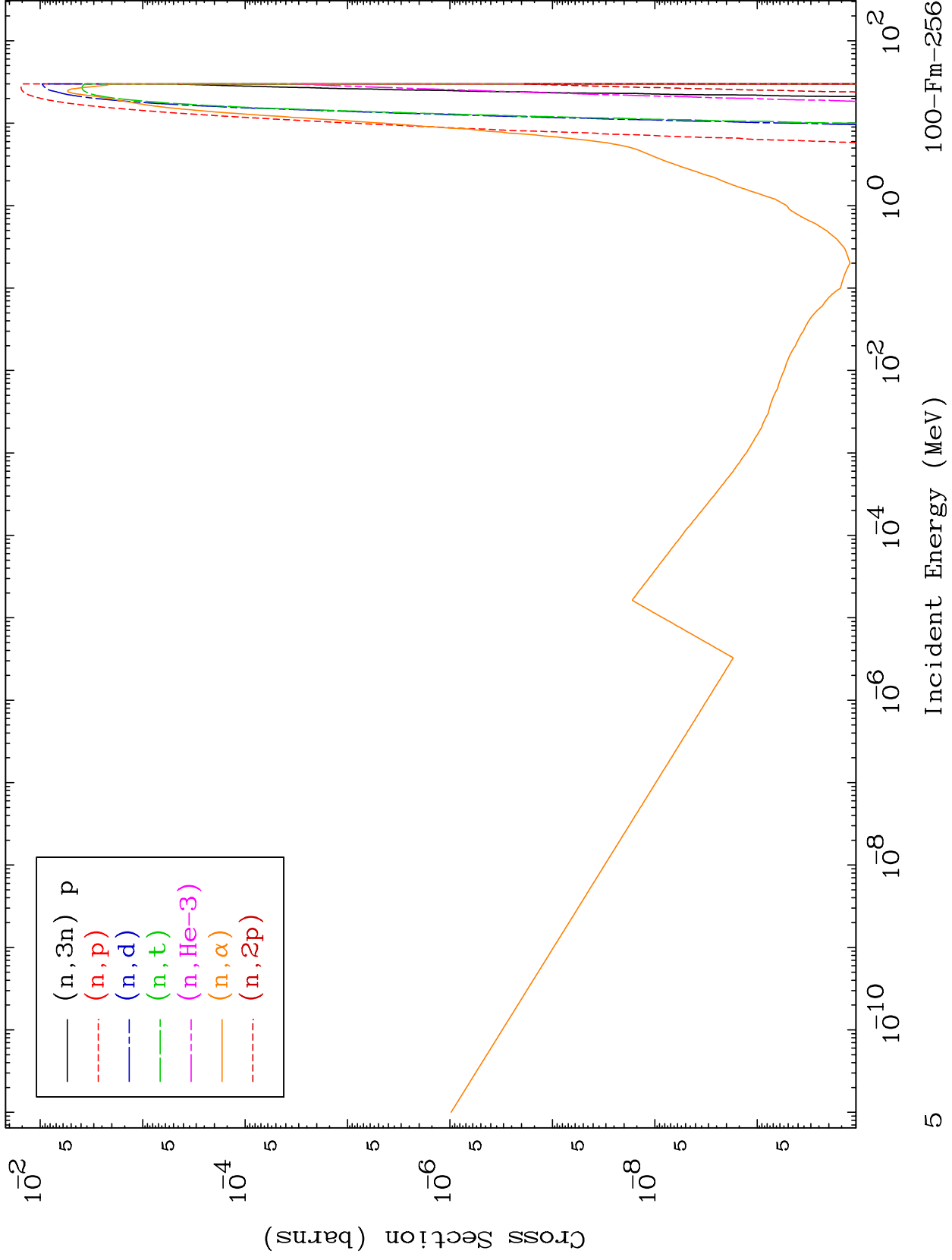
100-Fm-256



MAT 9937

Charged Particle
293 Kelvin Cross Sections

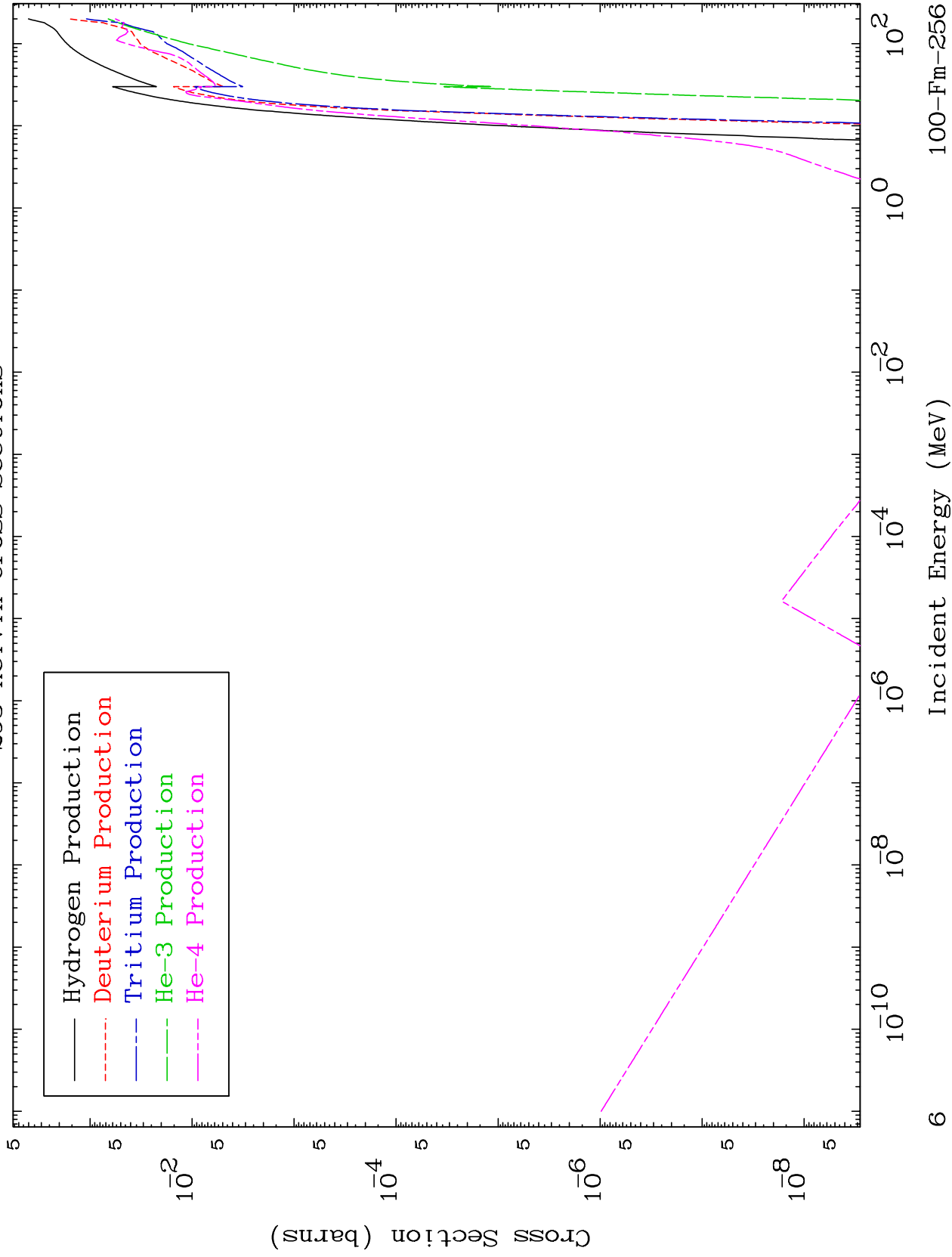
100-Fm-256



MAT 9937

Particle Production
293 Kelvin Cross Sections

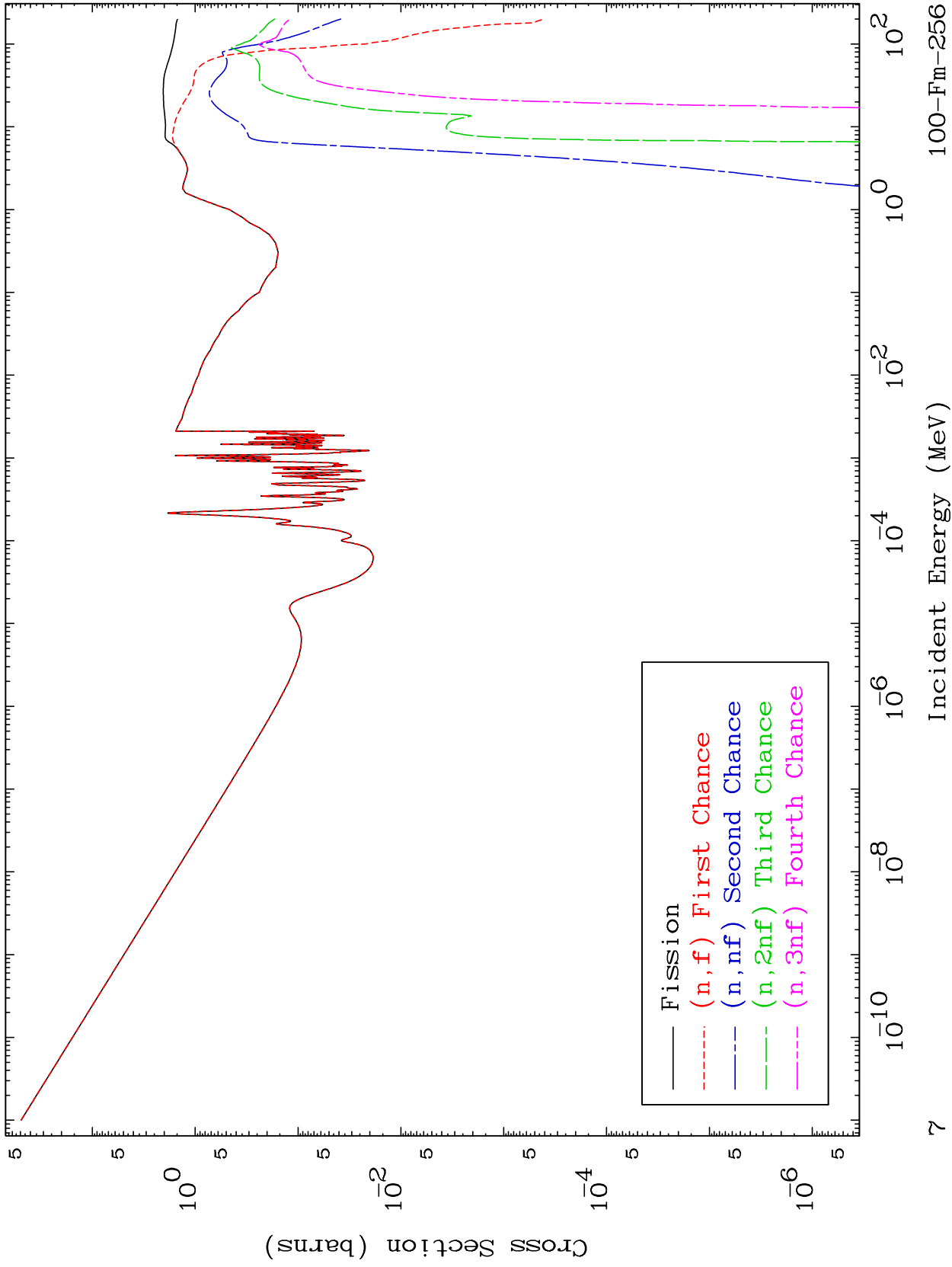
100-Fm-256



MAT 9937

Fission
293 Kelvin Cross Sections

100-Fm-256

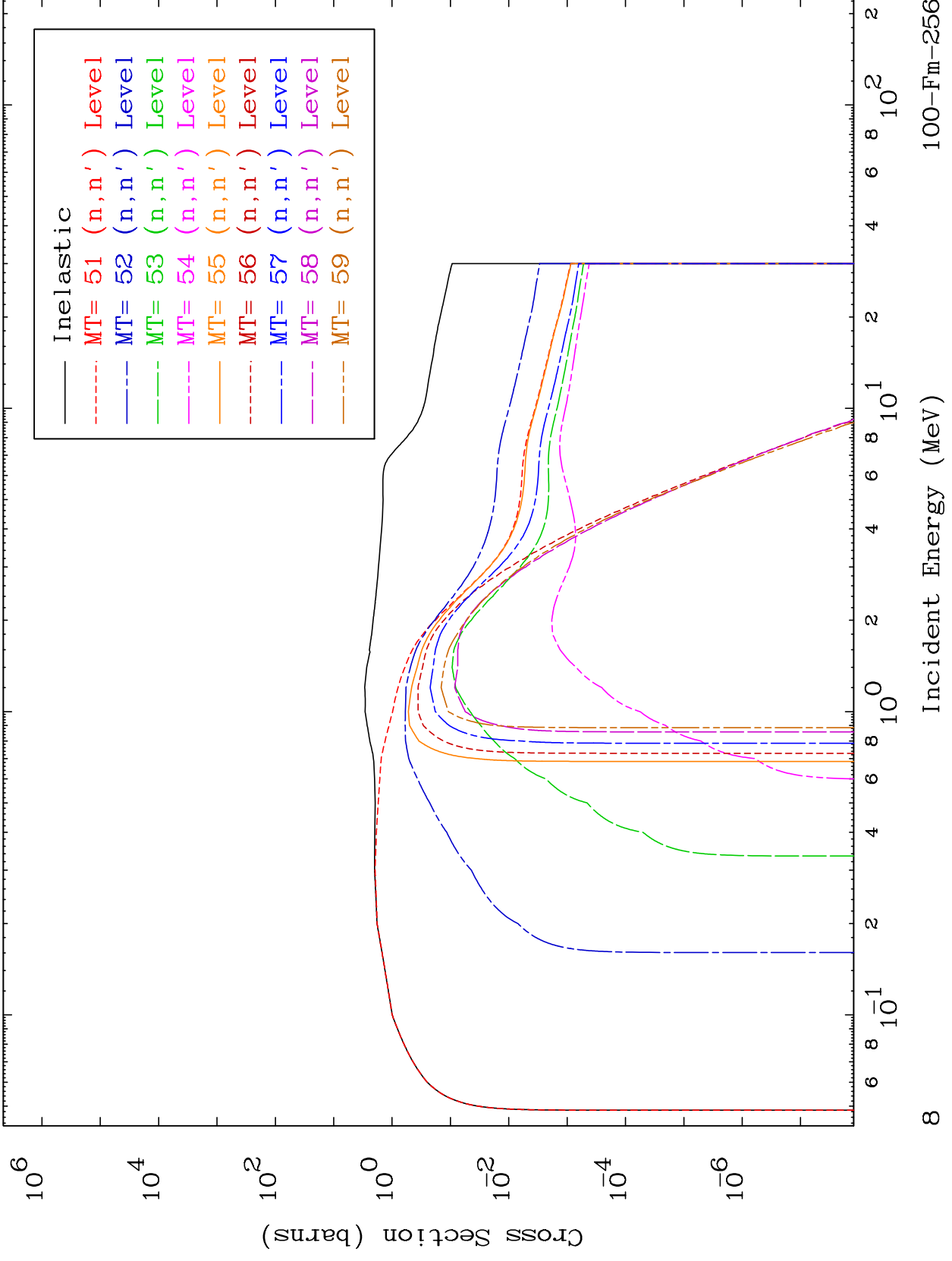


MAT 9937

(n,n') Level

100-Fm-256

293 Kelvin Cross Sections

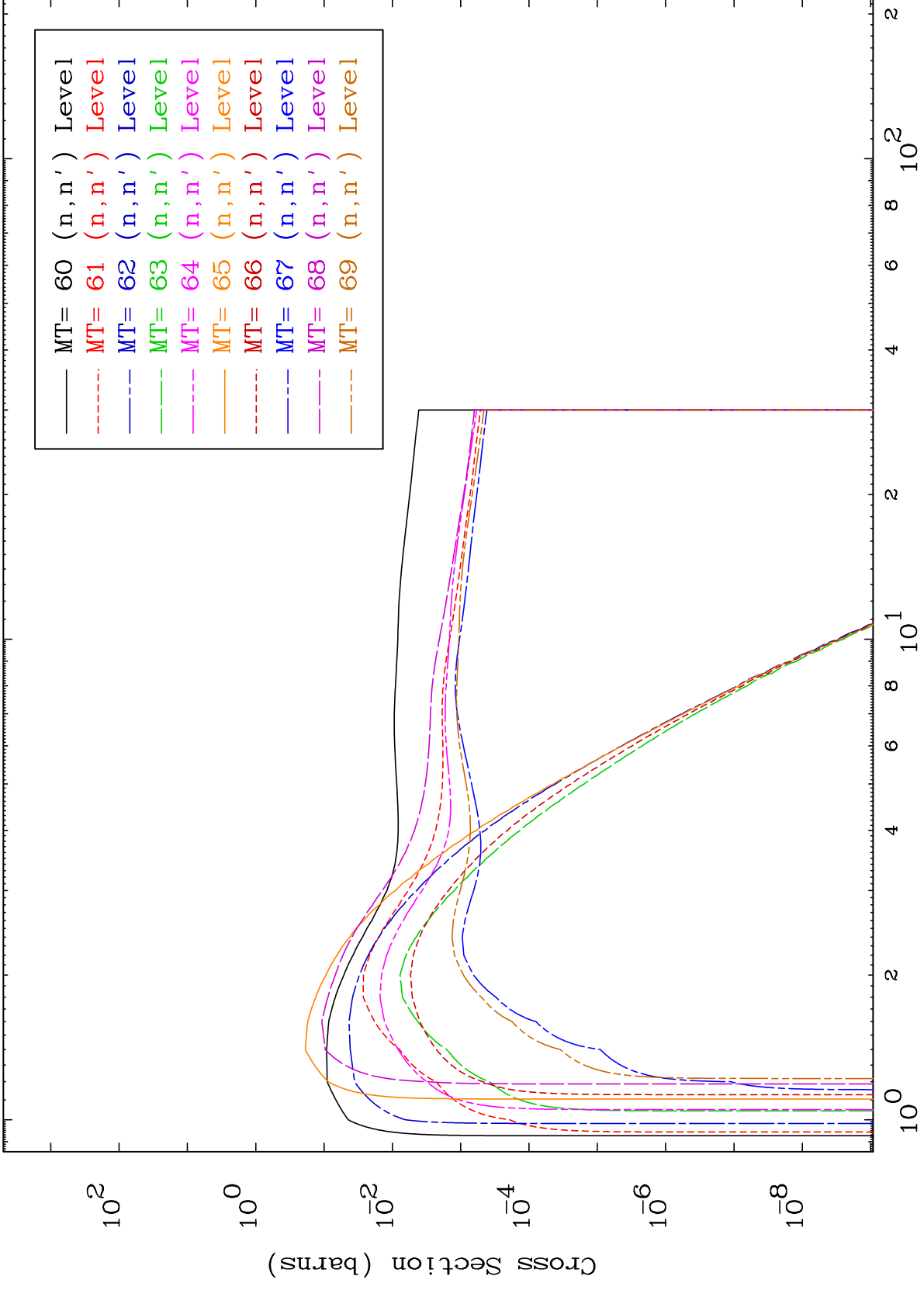


MAT 9937

(n,n') Level

100-Fm-256

293 Kelvin Cross Sections



Incident Energy (MeV)

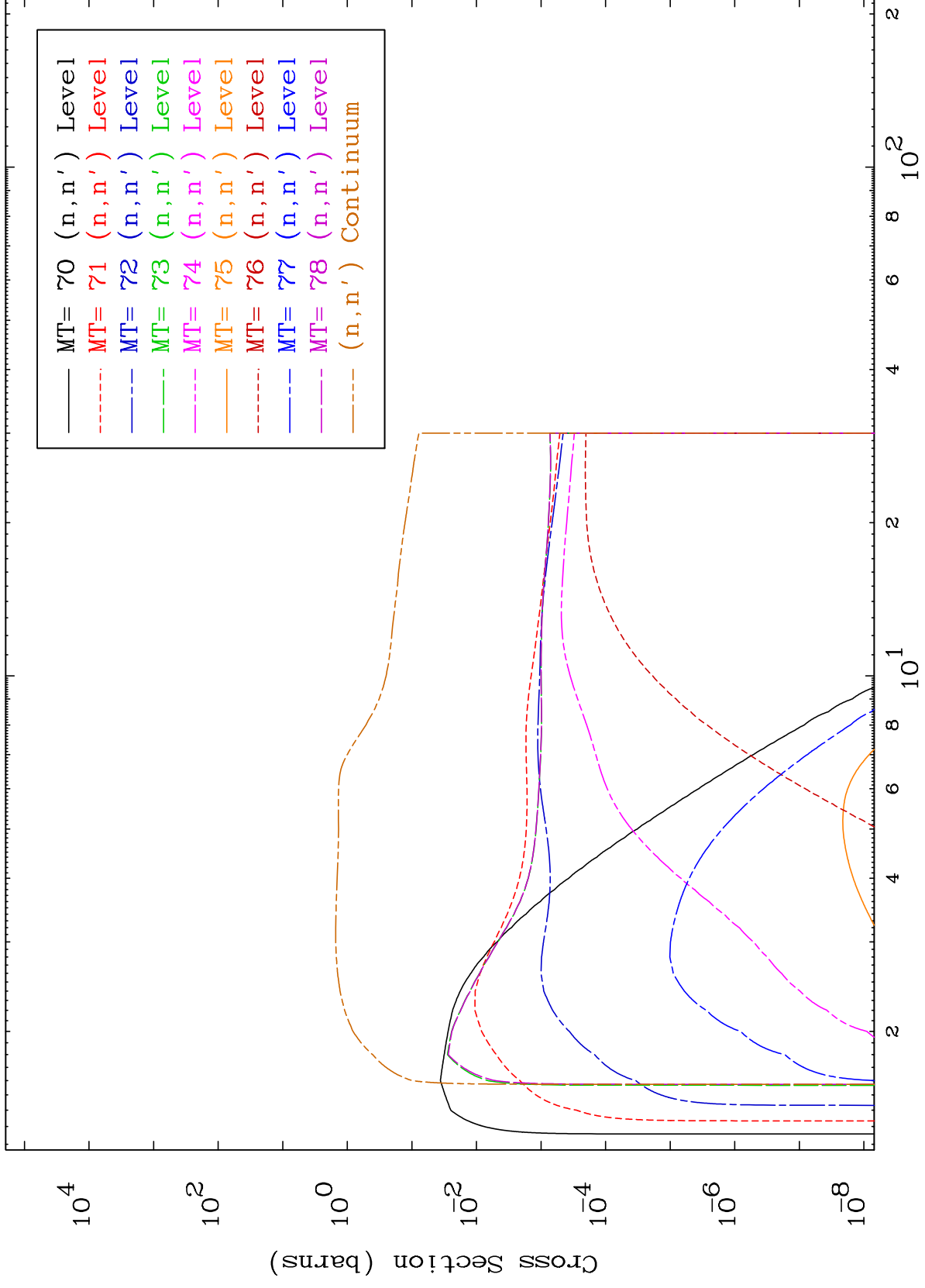
100-Fm-256

MAT 9937

(n,n') Level

100-Fm-256

293 Kelvin Cross Sections



10

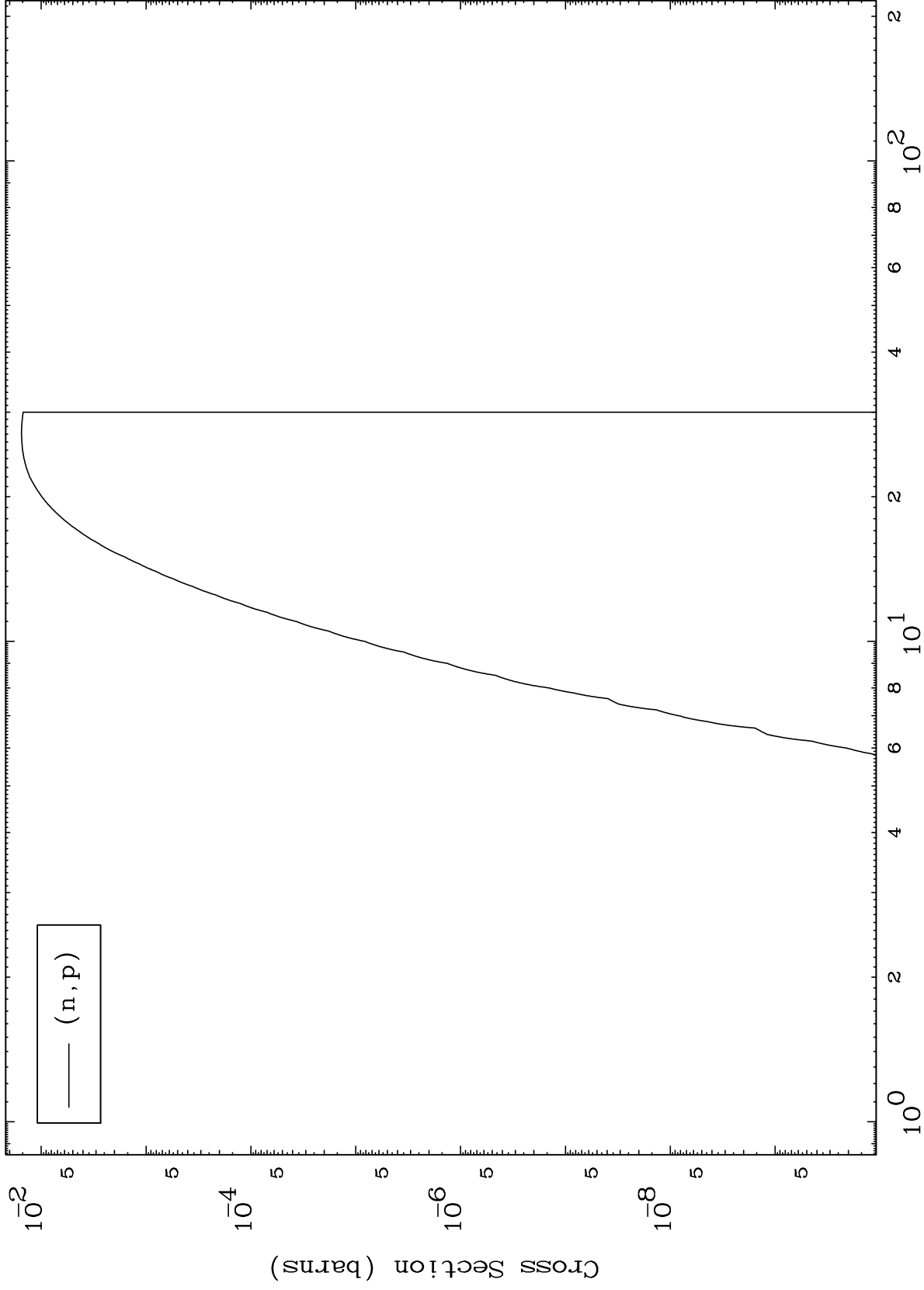
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,p) Levels
293 Kelvin Cross Sections

100-Fm-256



(n,p)

Incident Energy (MeV)

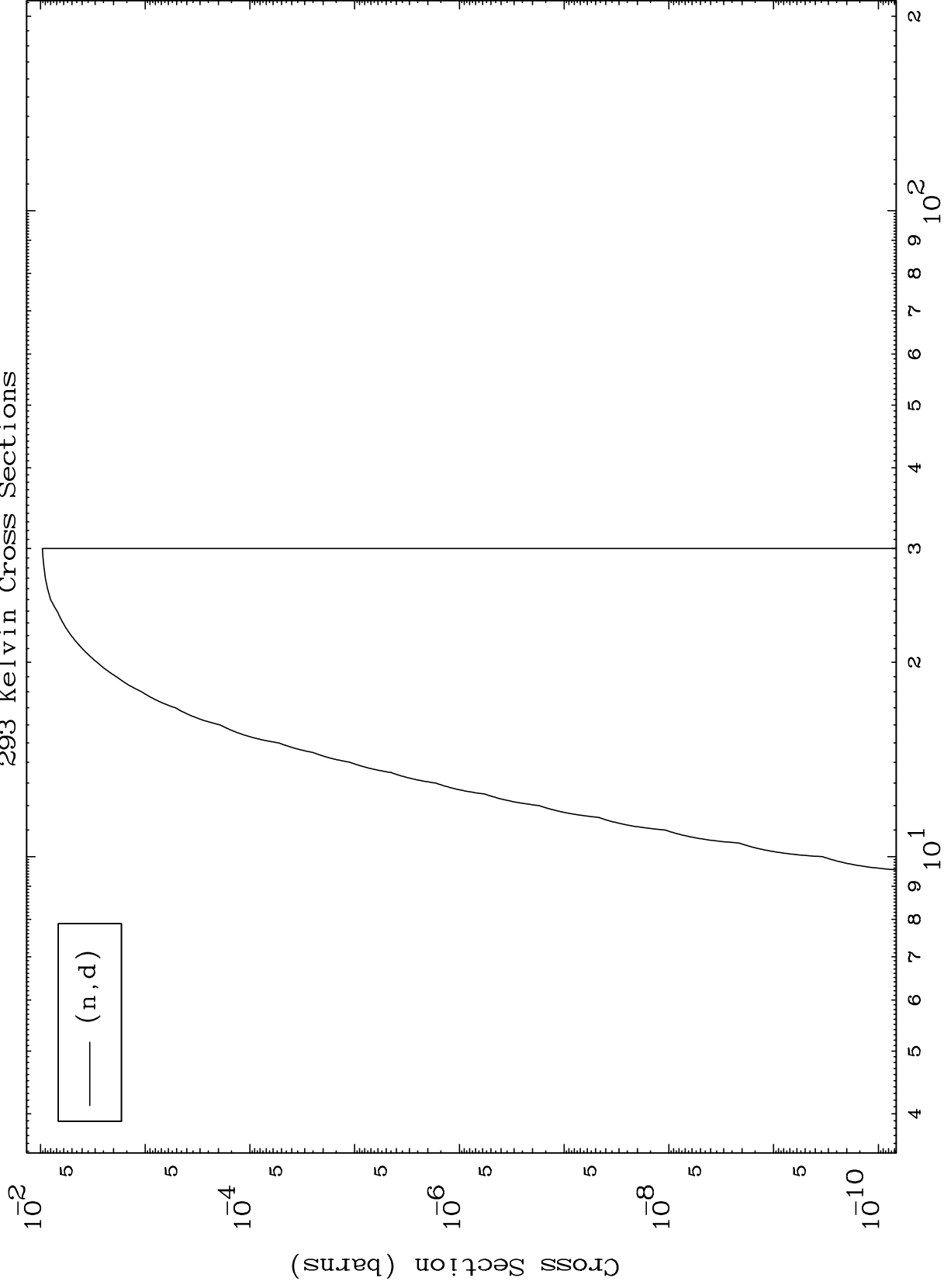
100-Fm-256

11

MAT 9937

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-256



12

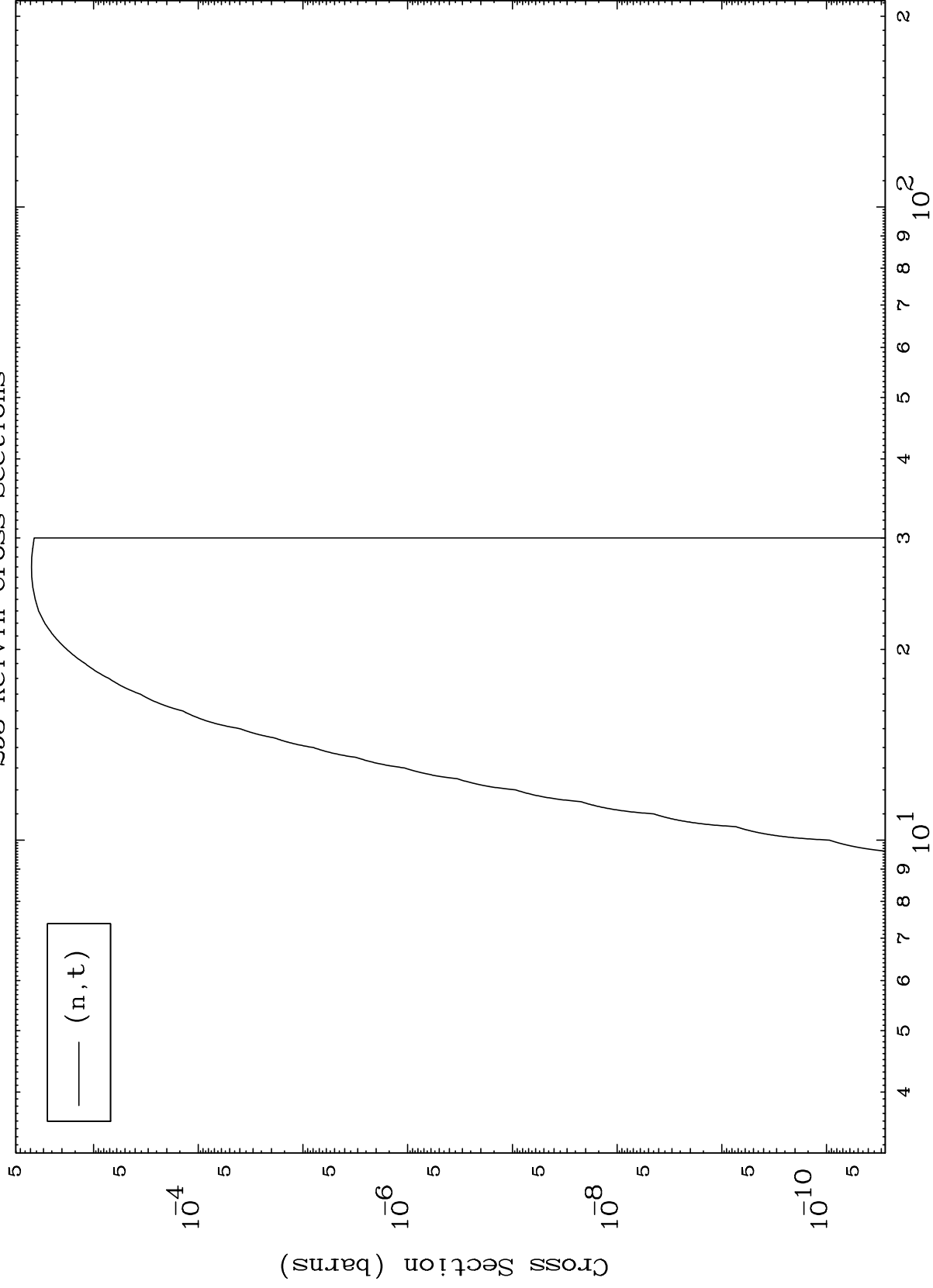
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-256



13

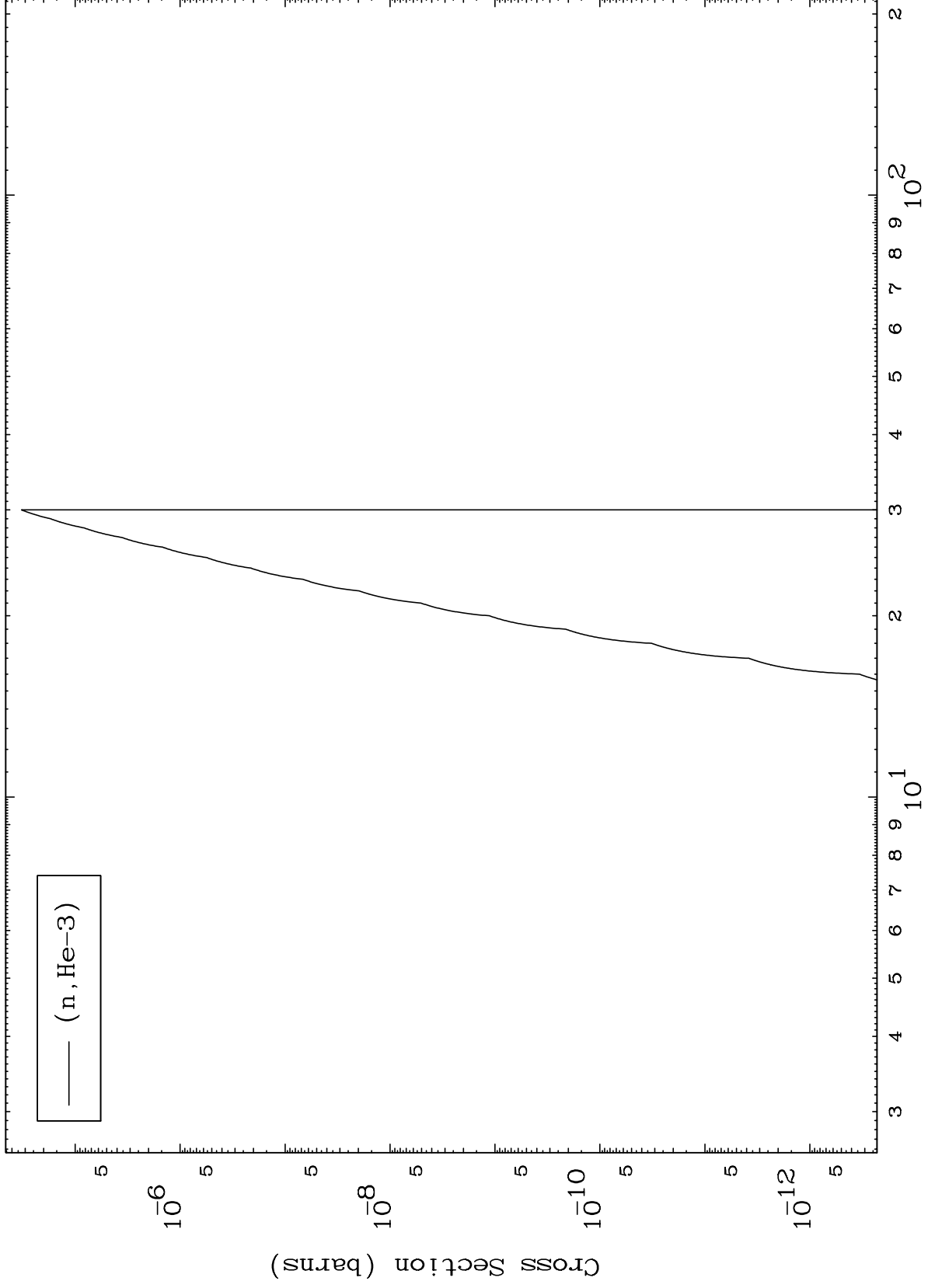
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-256



14

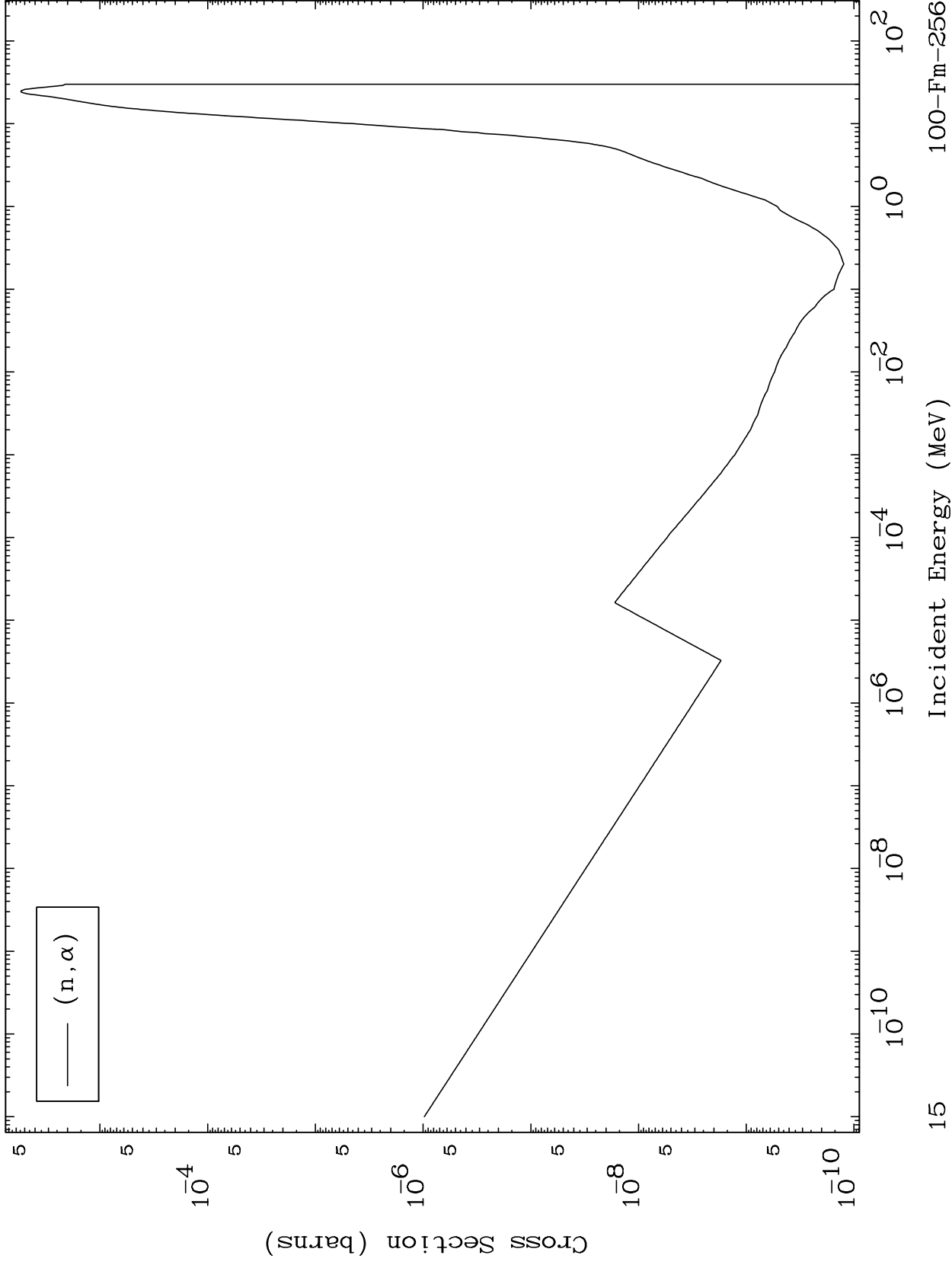
Incident Energy (MeV)

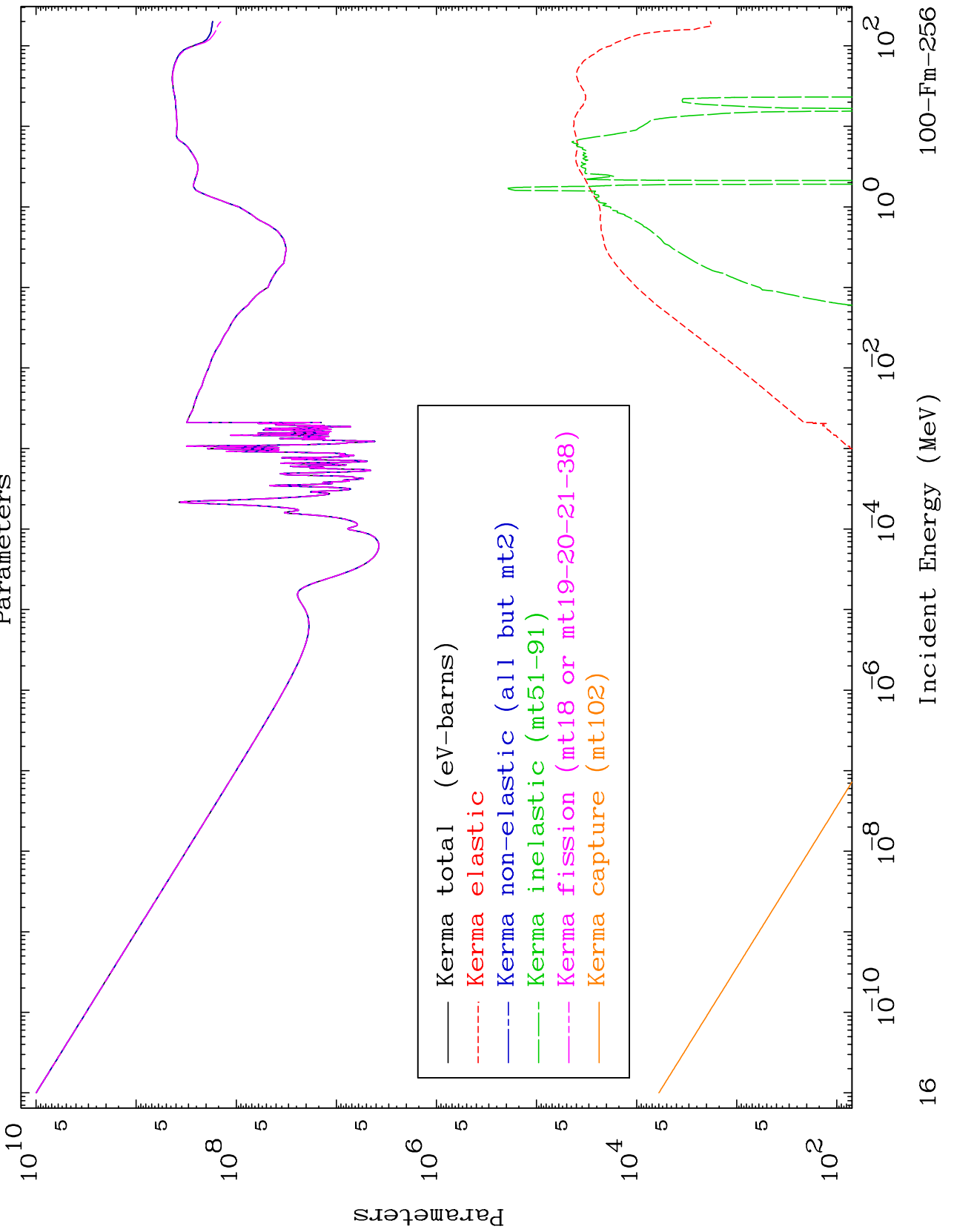
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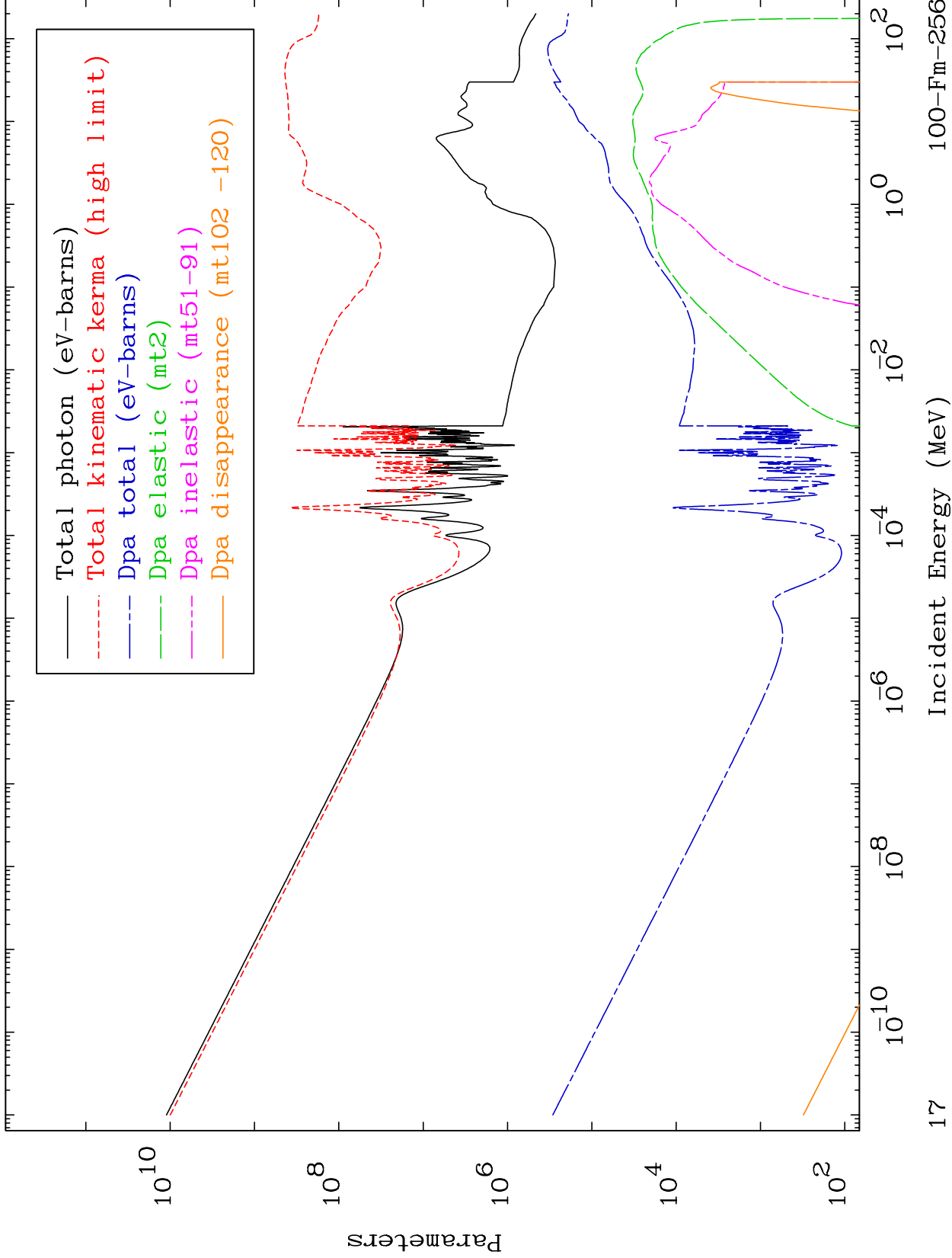
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(n, α) Levels
293 Kelvin Cross Sections

100-Fm-256



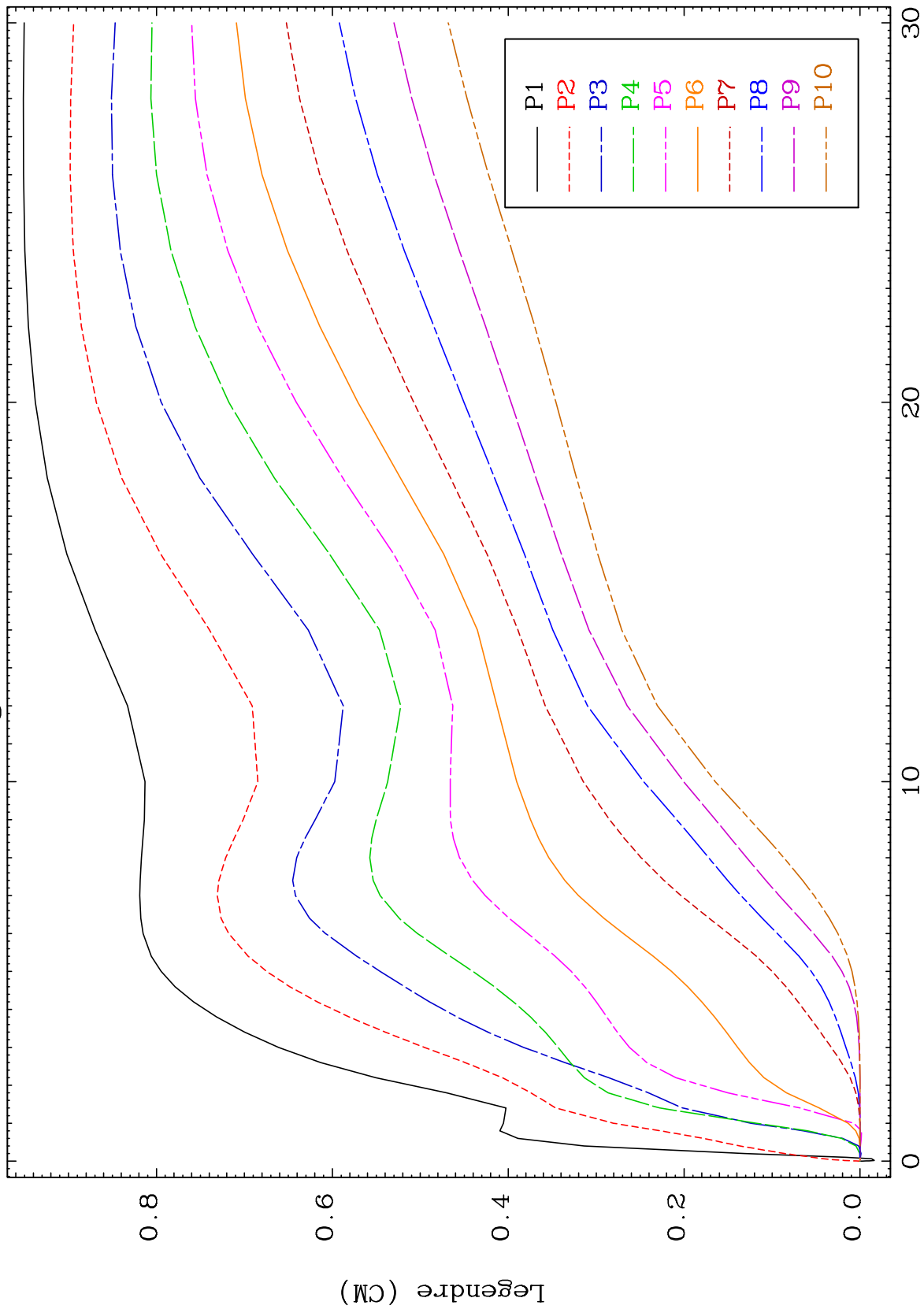




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Elastic Legendre Coefficients

100-Fm-256



18

Incident Energy (MeV)

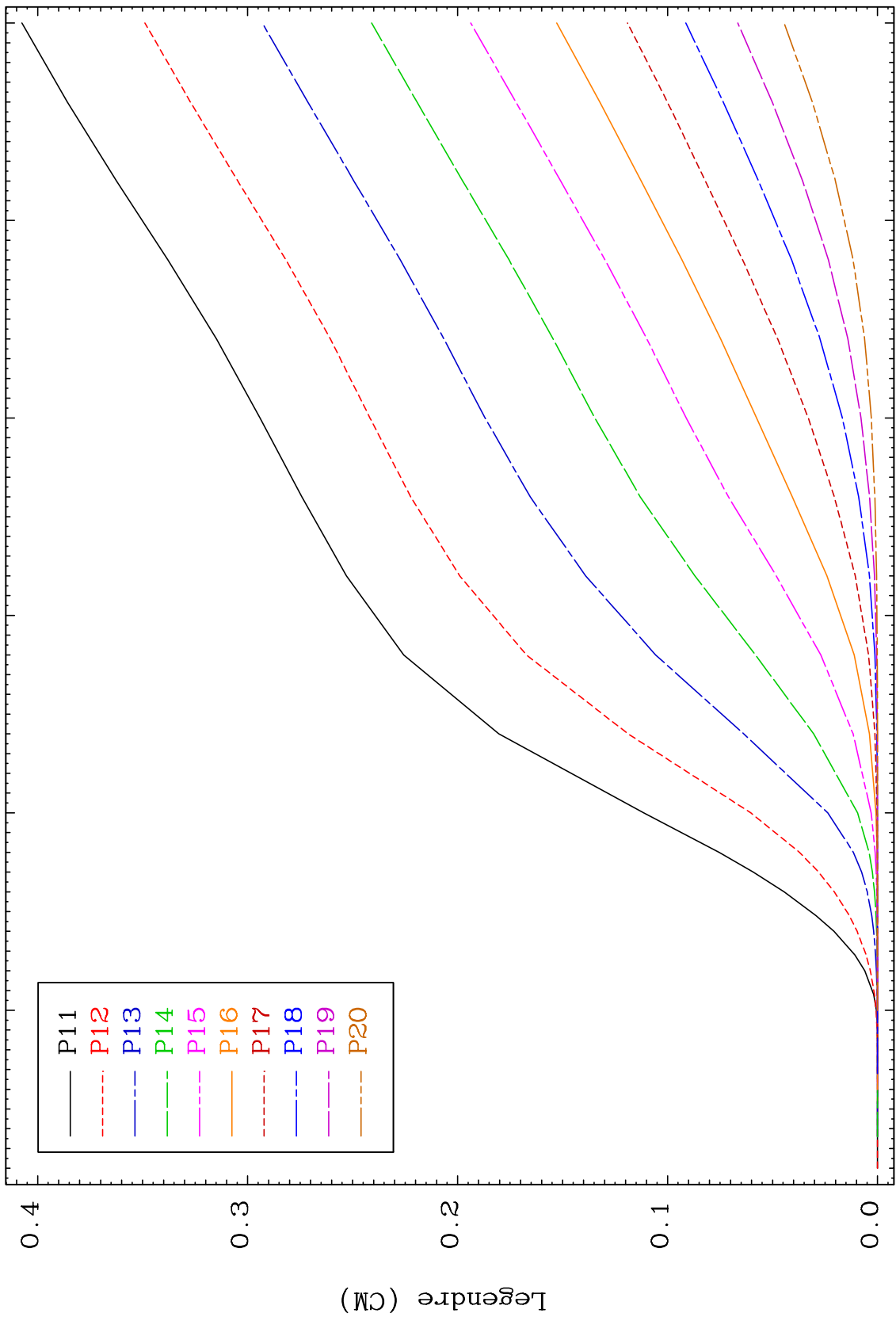
100-Fm-256

30

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Elastic Legendre Coefficients

100-Fm-256



19

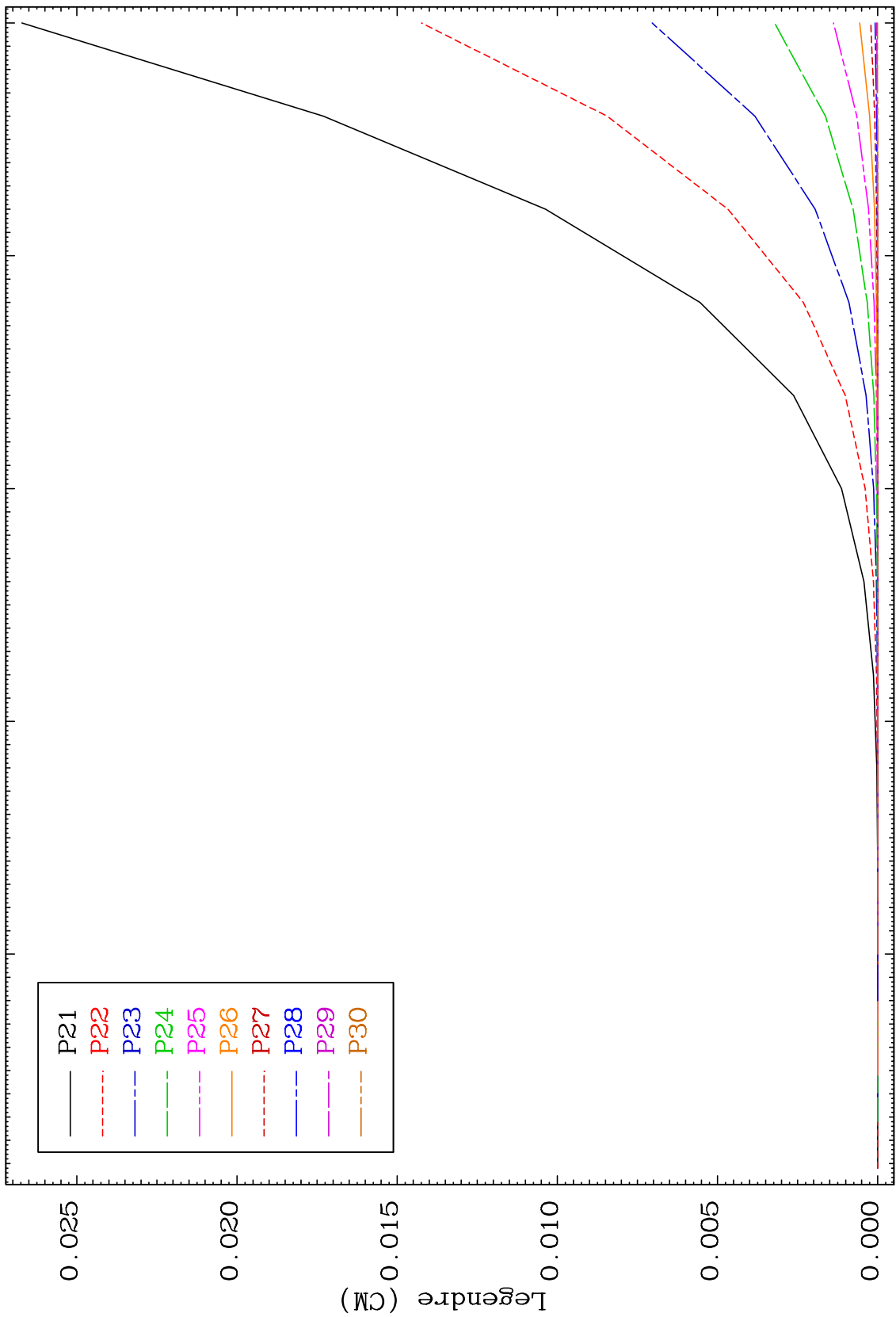
Incident Energy (MeV)

100-Fm-256

MAT 9937

Elastic Legendre Coefficients

100-Fm-256



20

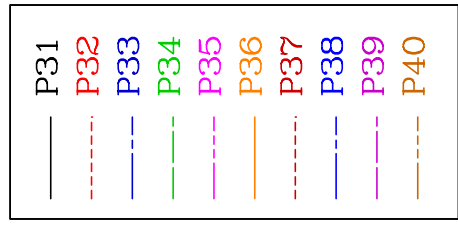
Incident Energy (MeV)

100-Fm-256

MAT 9937

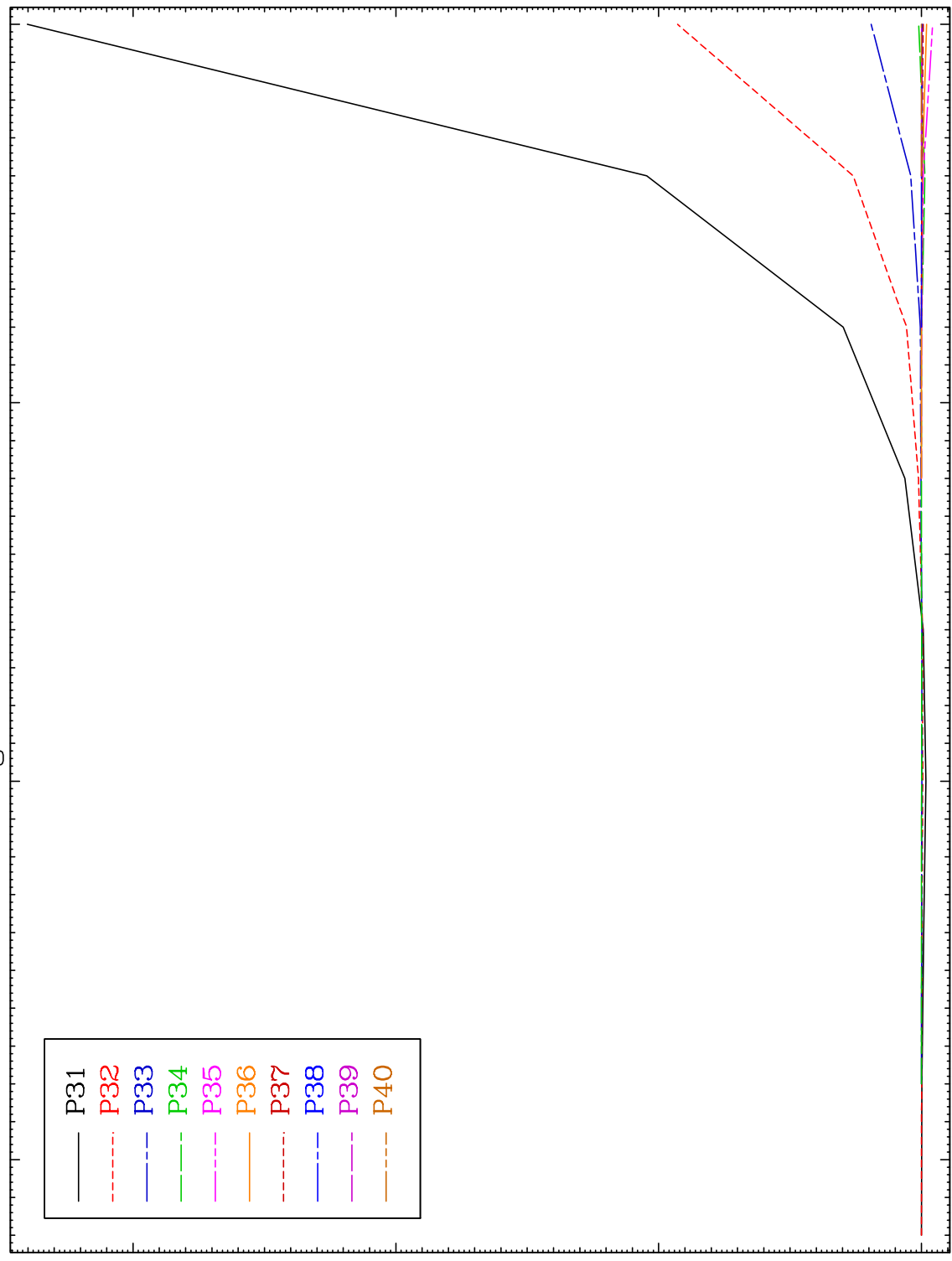
Elastic
Legendre Coefficients

100-Fm-256



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

21

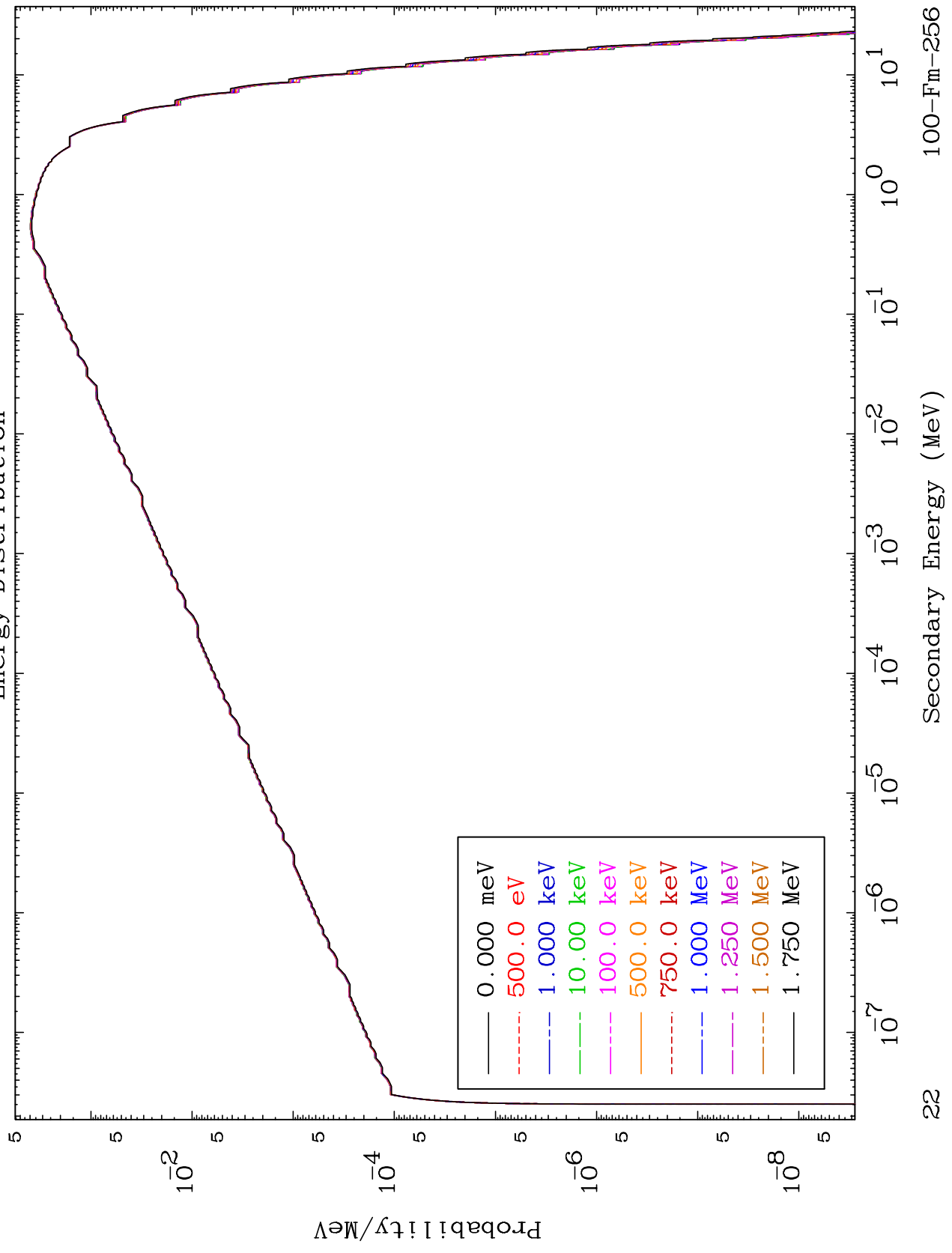
Incident Energy (MeV)

100-Fm-256

MAT 9937

Fission Energy Distribution

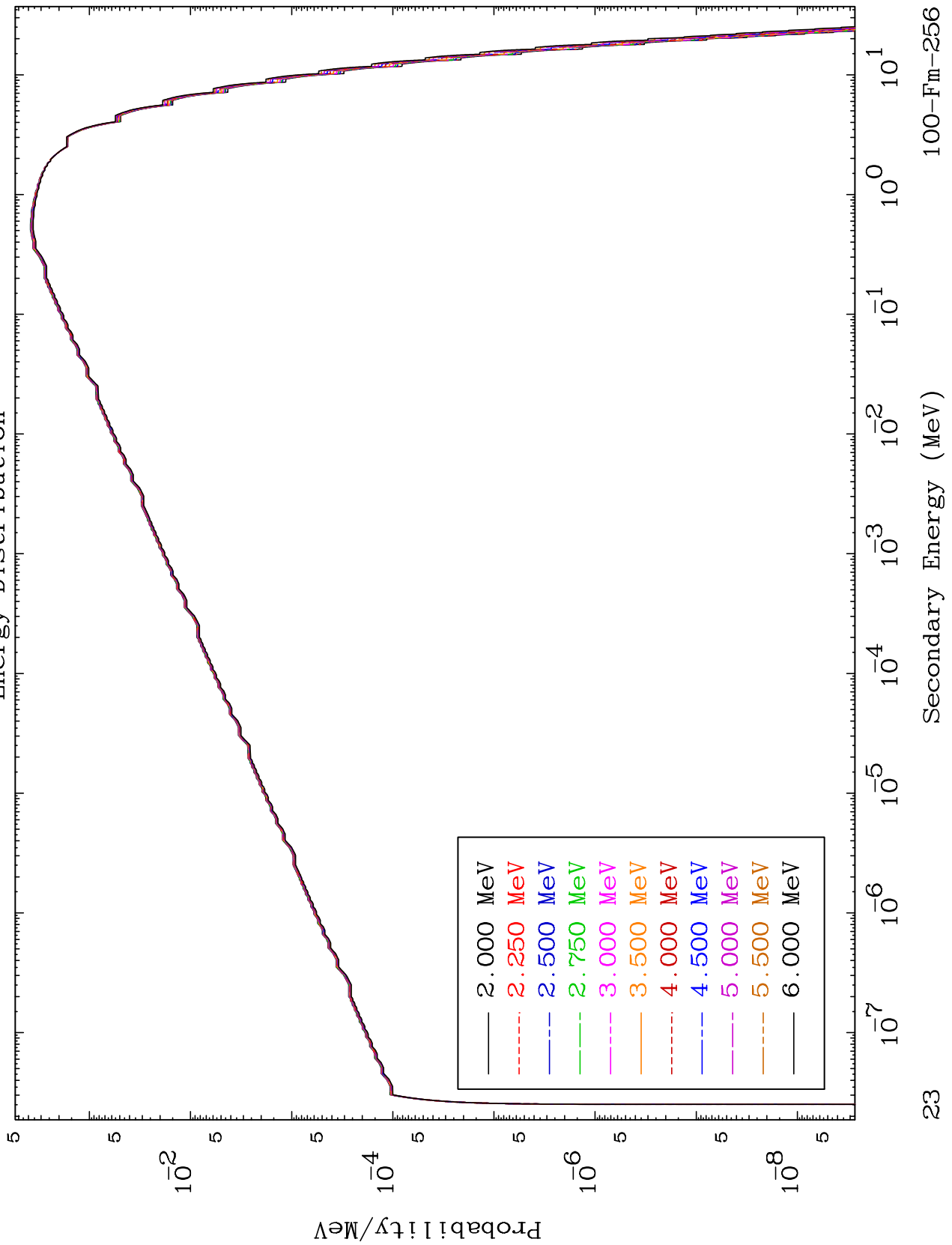
100-Fm-256



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Fission Energy Distribution

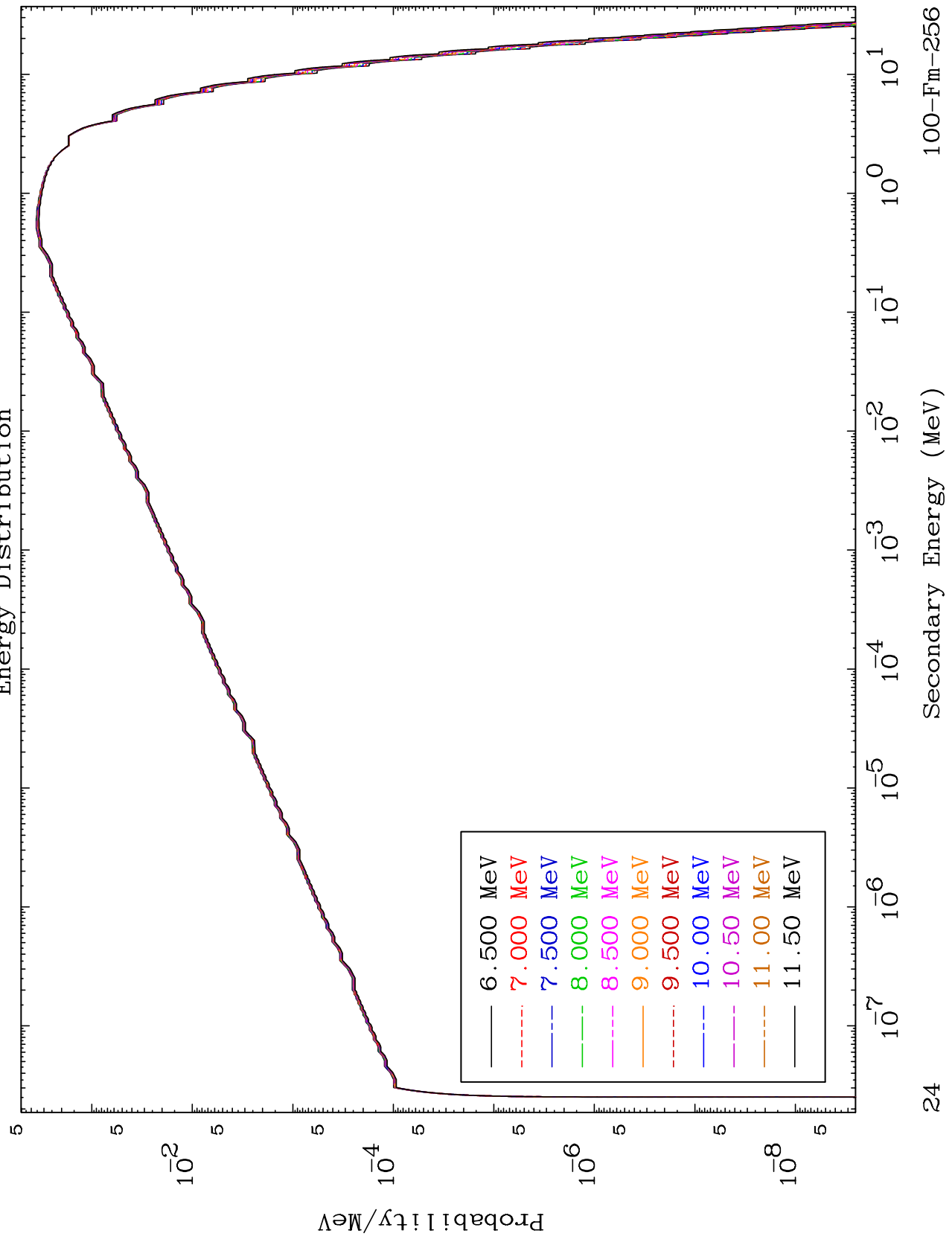
100-Fm-256



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Fission Energy Distribution

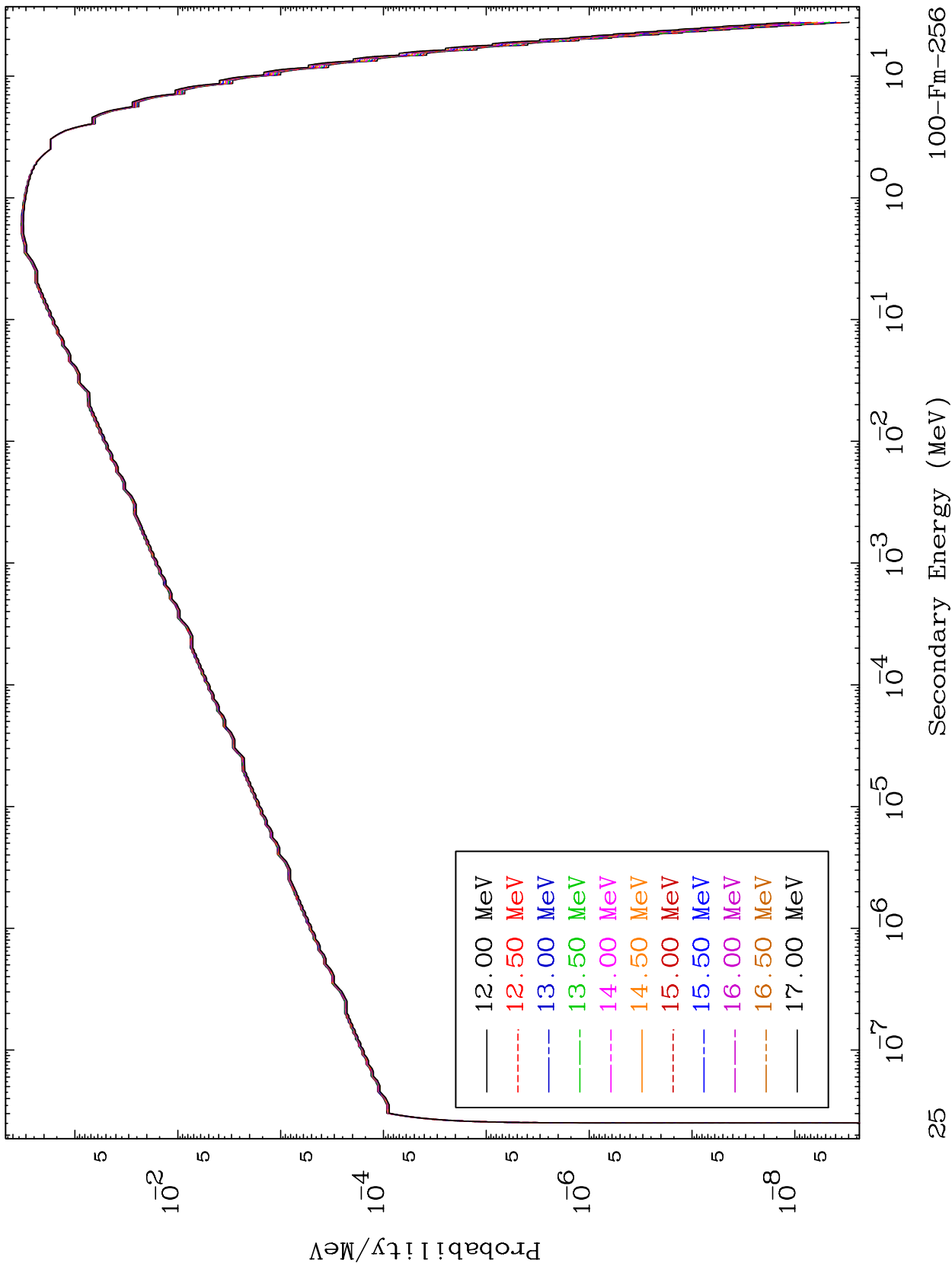
100-Fm-256



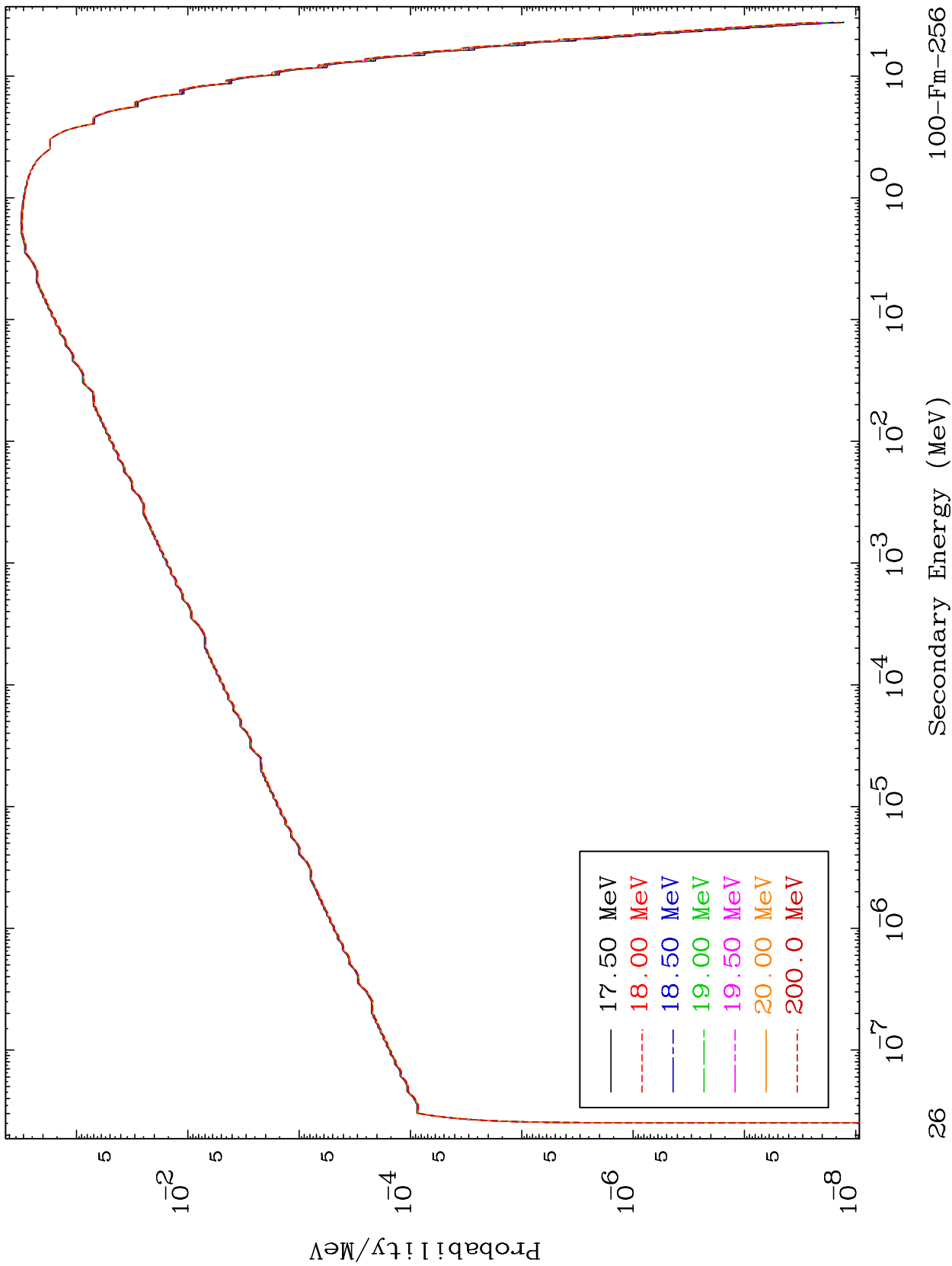
MAT 9937

Fission Energy Distribution

100-Fm-256



Fission
Energy Distribution

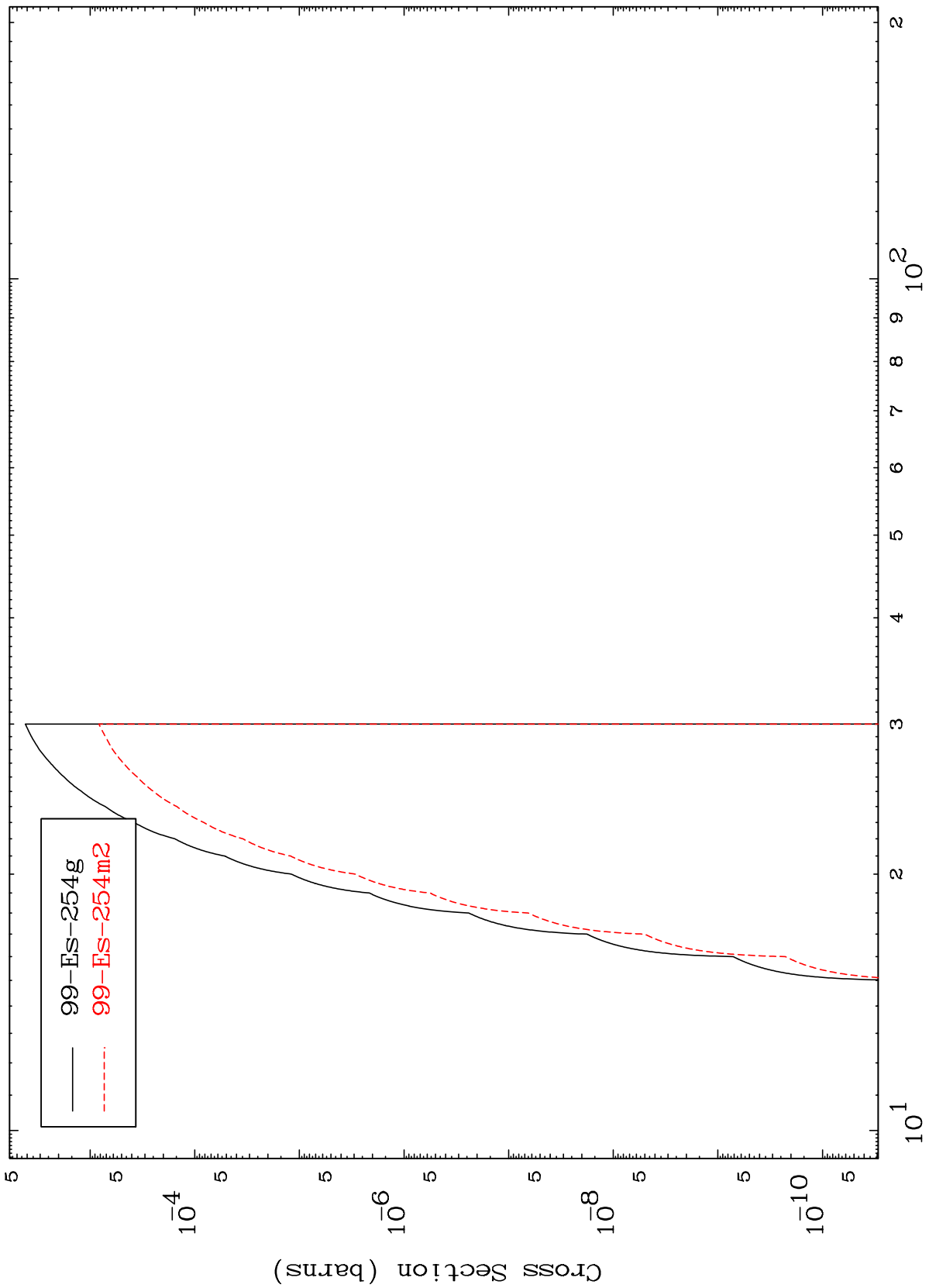


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(n,n') d

100-Fm-256

Radionuclide Production Cross Section



99-Es-254g
99-Es-254m2

27

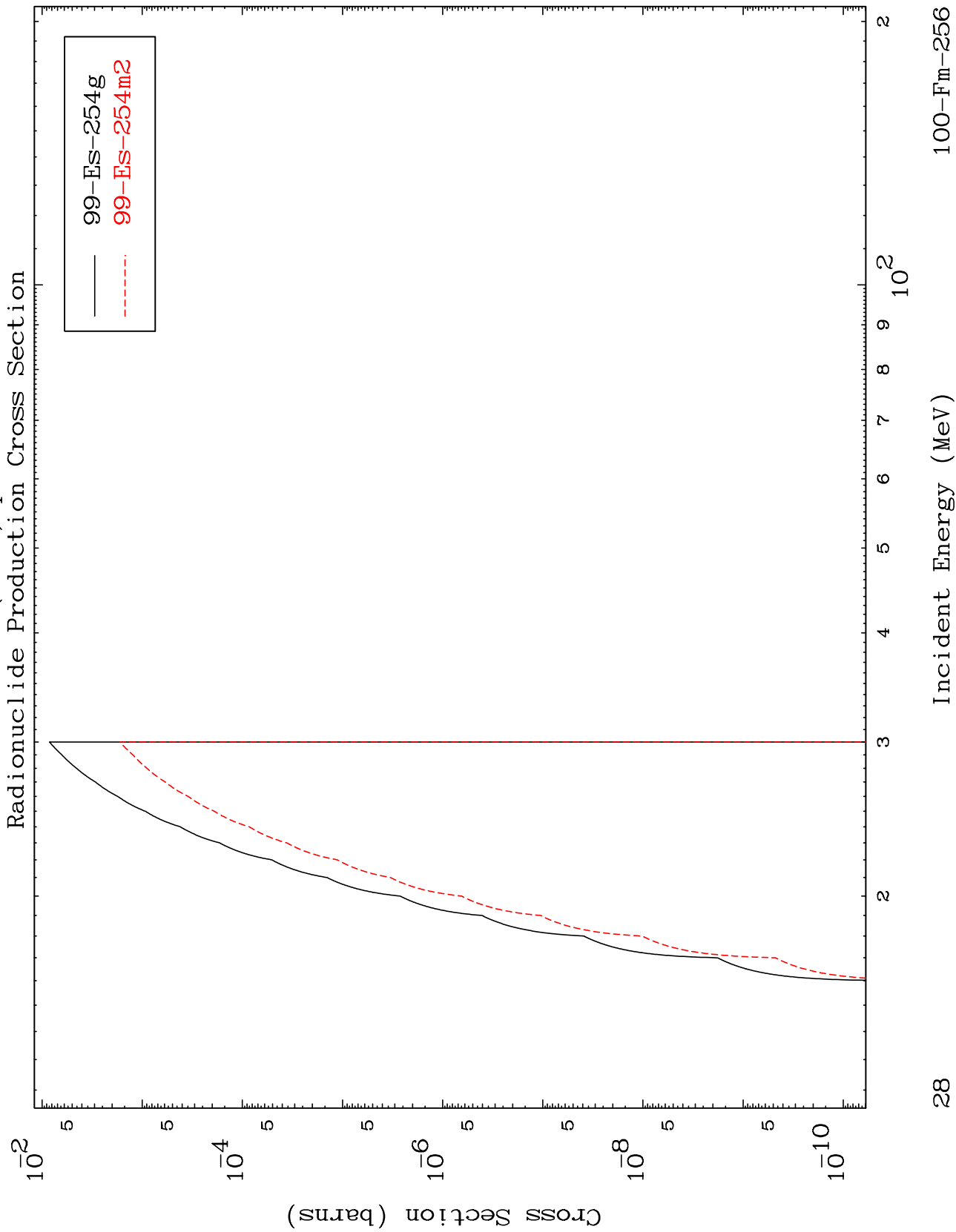
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,2n) p

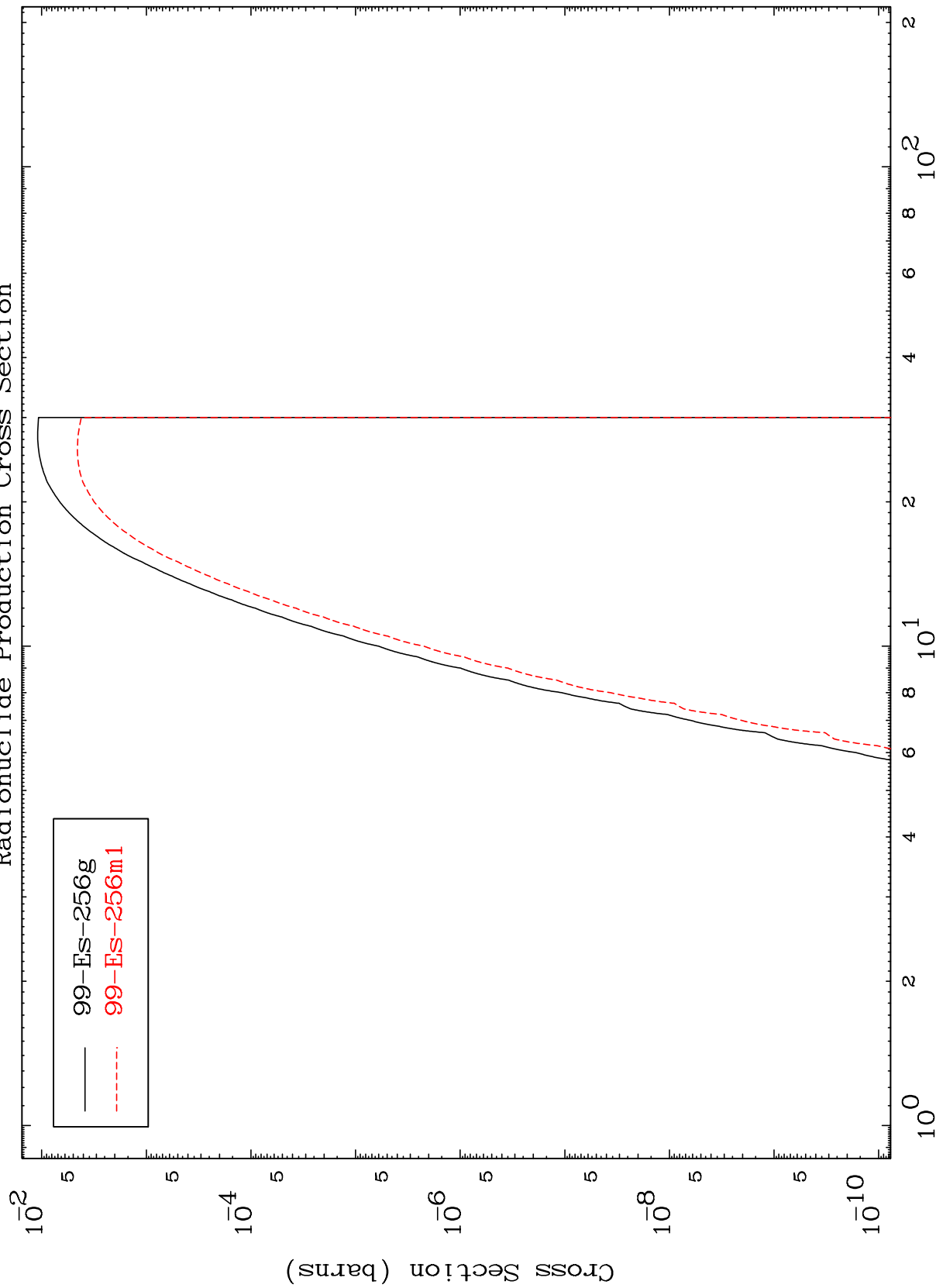
100-Fm-256



MAT 9937

100-Fm-256

Radionuclide Production Cross Section (n,p)



99-Es-256g
99-Es-256m1

Incident Energy (MeV)

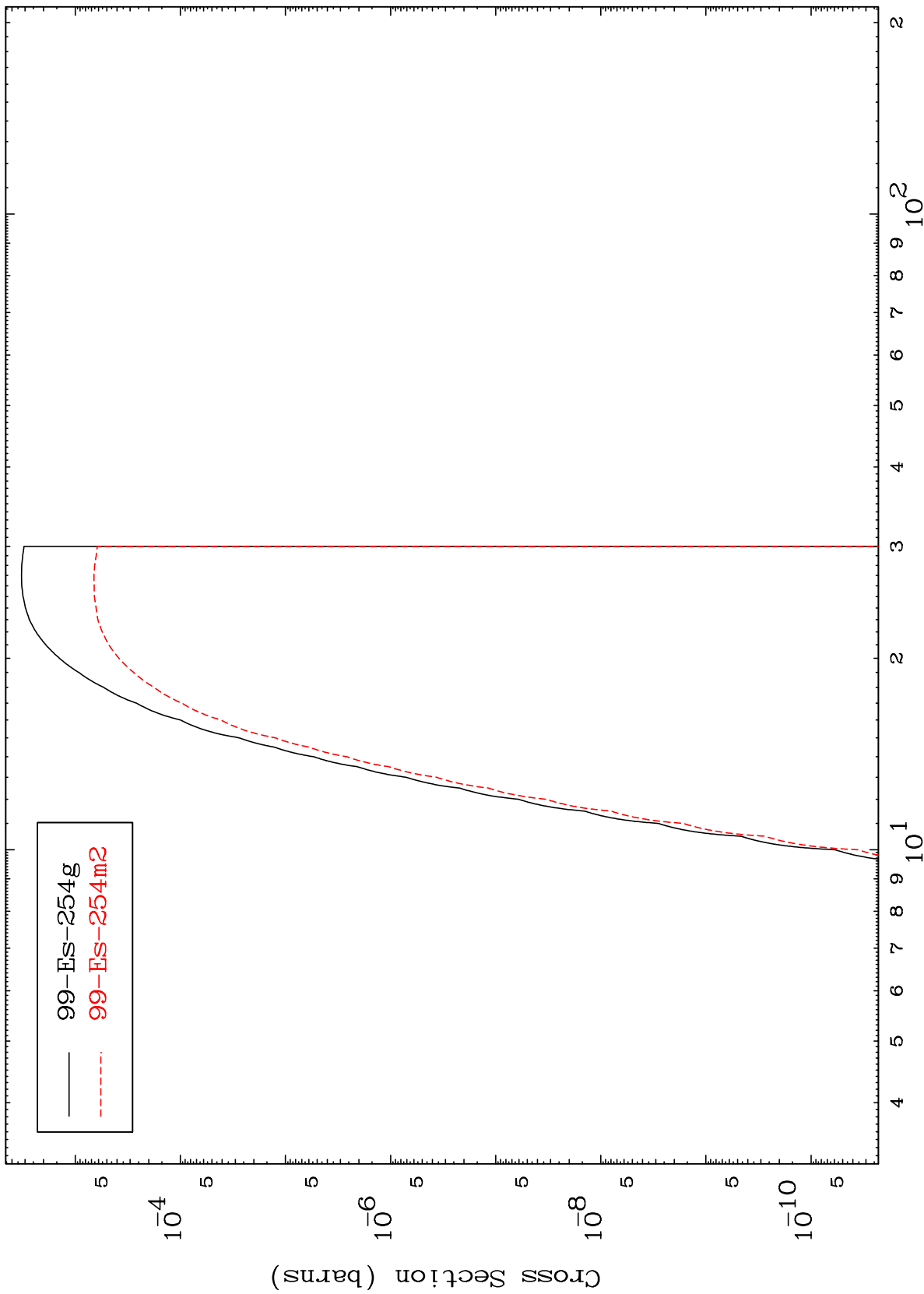
100-Fm-256

29

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100-Fm-256

(n, t)
Radionuclide Production Cross Section



100-Fm-256

Incident Energy (MeV)

30