

Program EVALPLOT  
(Version 2017-1)

by

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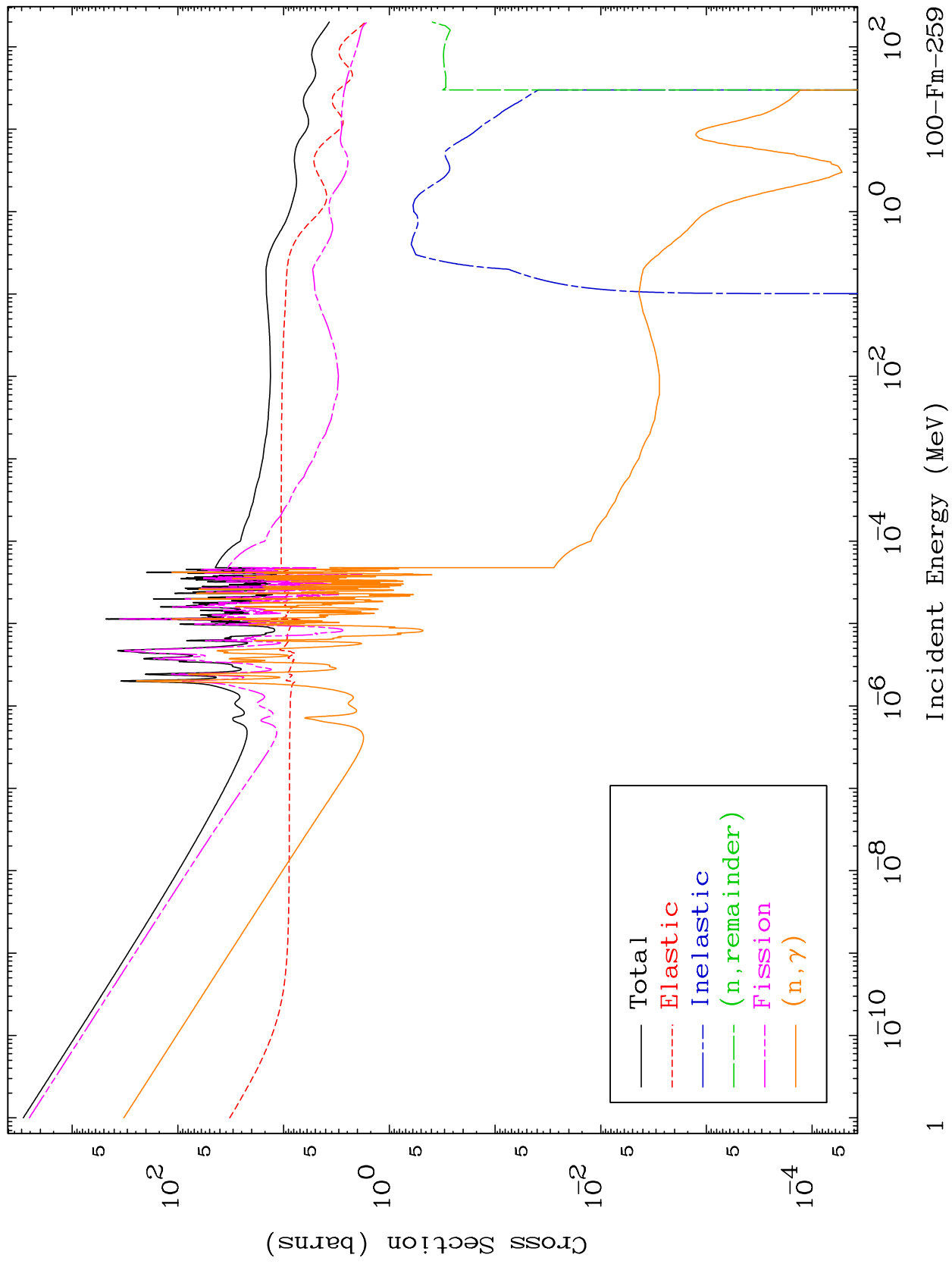
Press Mouse Button to Start

MAT 9940

Major

293 Kelvin Cross Sections

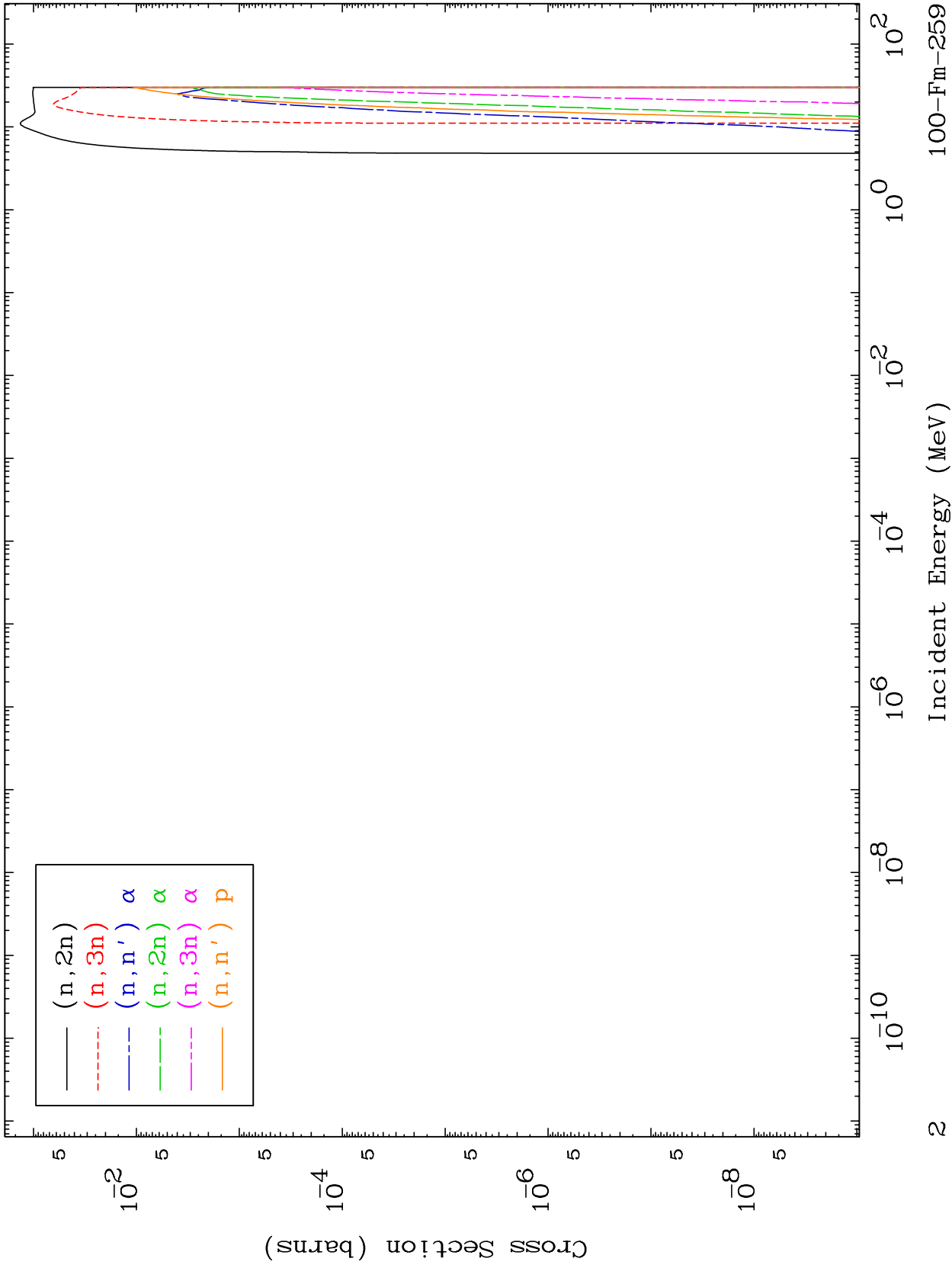
100-Fm-259



MAT 9940

Neutron Production  
293 Kelvin Cross Sections

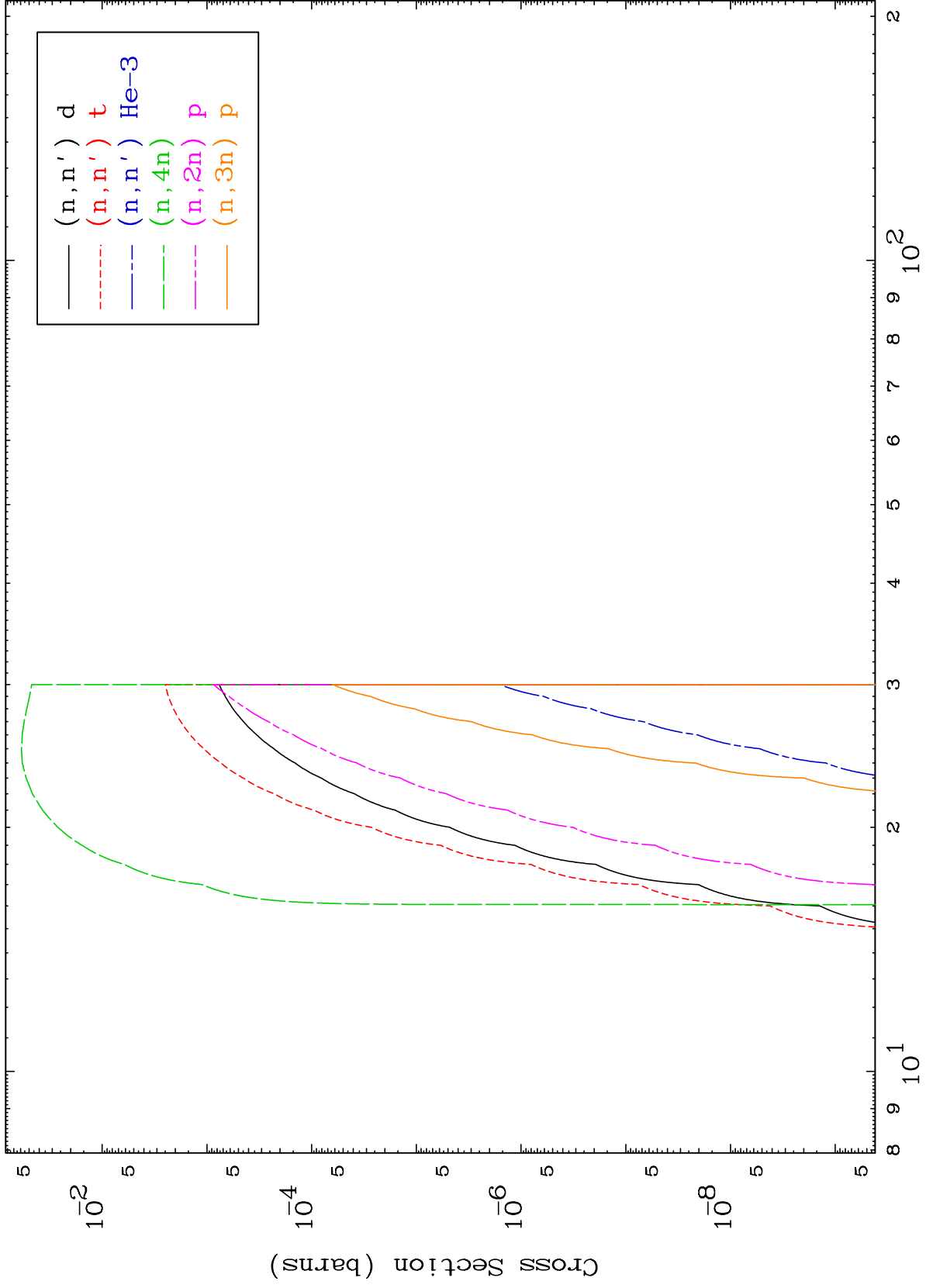
100-Fm-259



MAT 9940

Neutron Production  
293 Kelvin Cross Sections

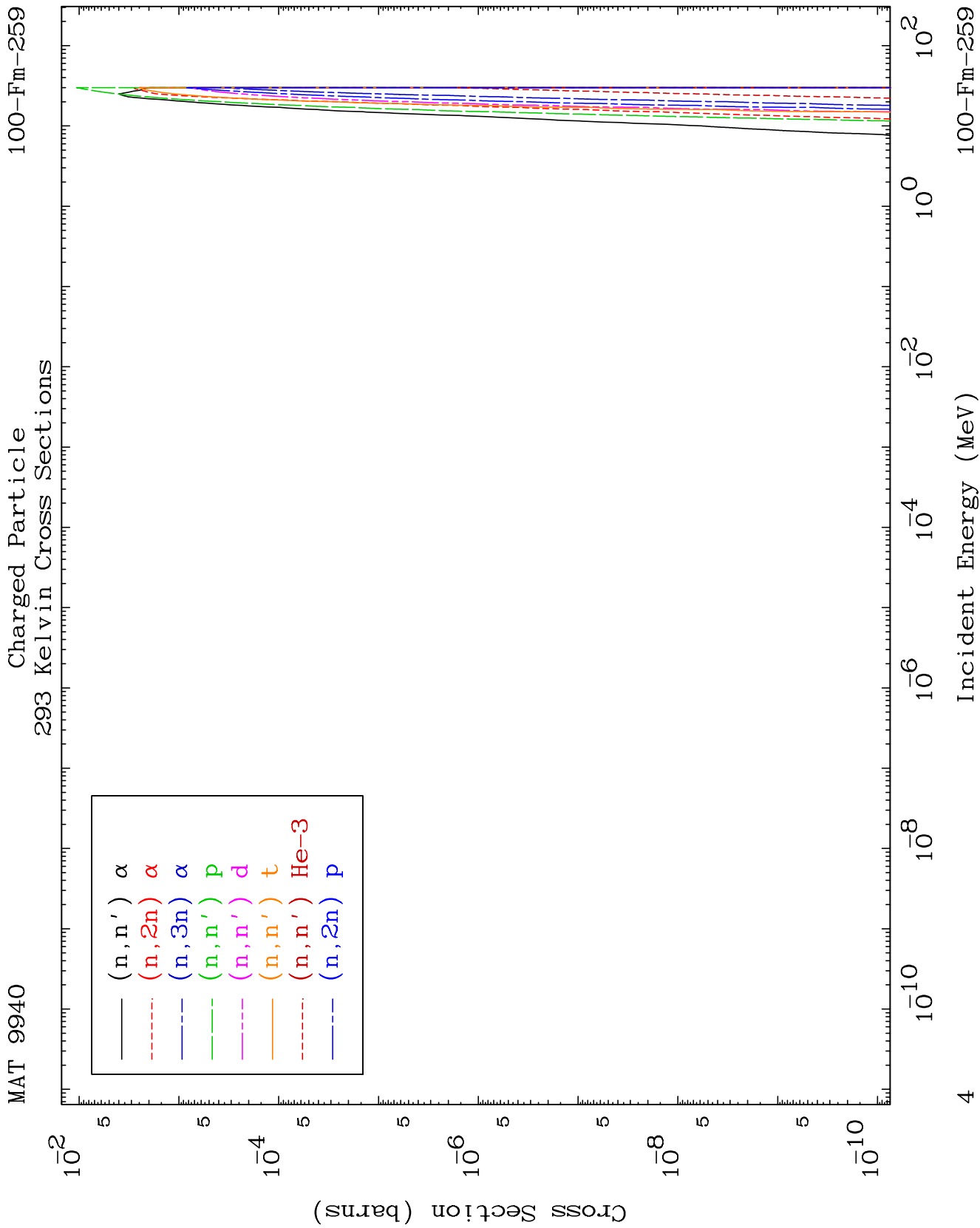
100-Fm-259



3

Incident Energy (MeV)

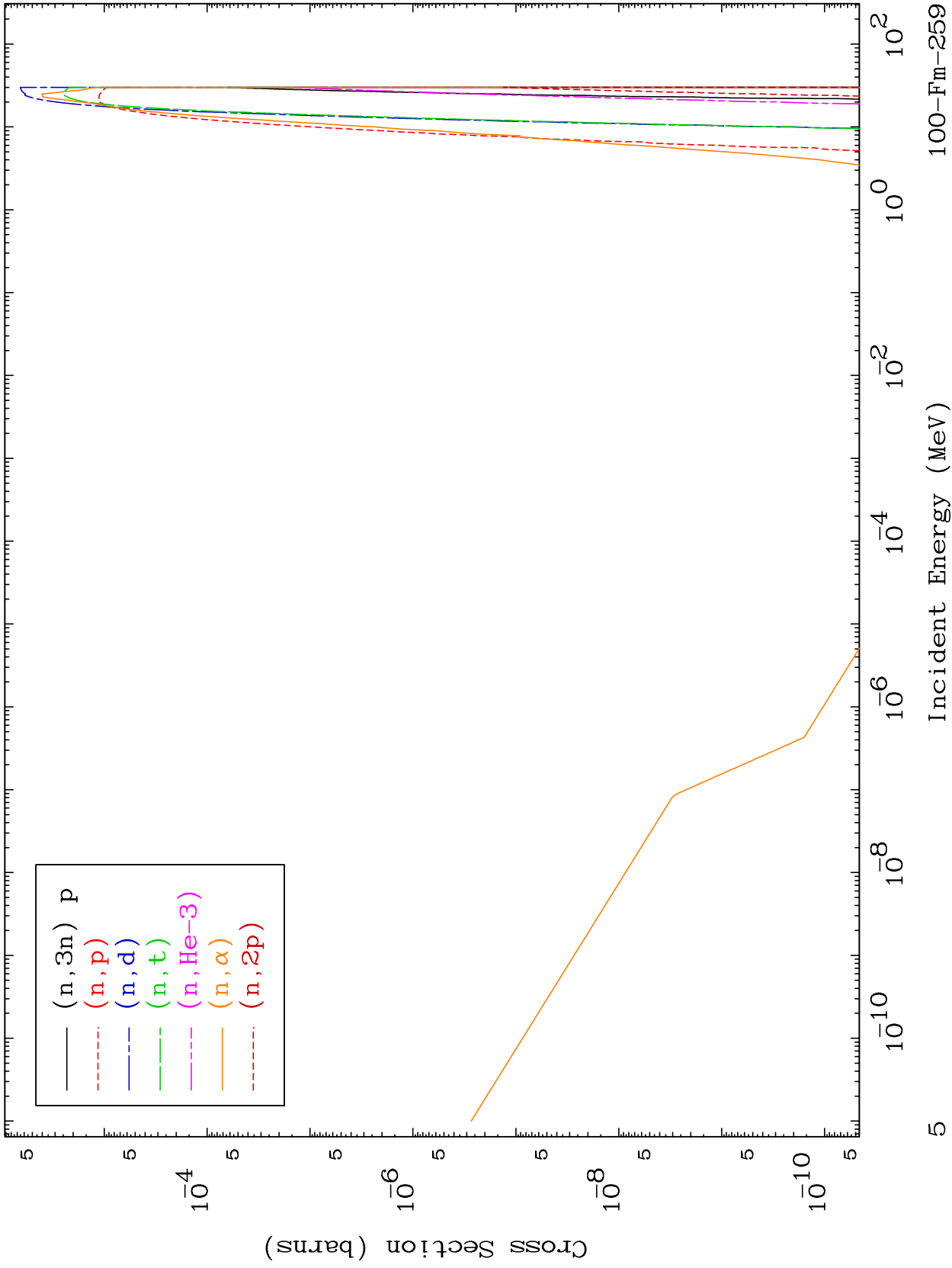
100-Fm-259



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Charged Particle  
293 Kelvin Cross Sections

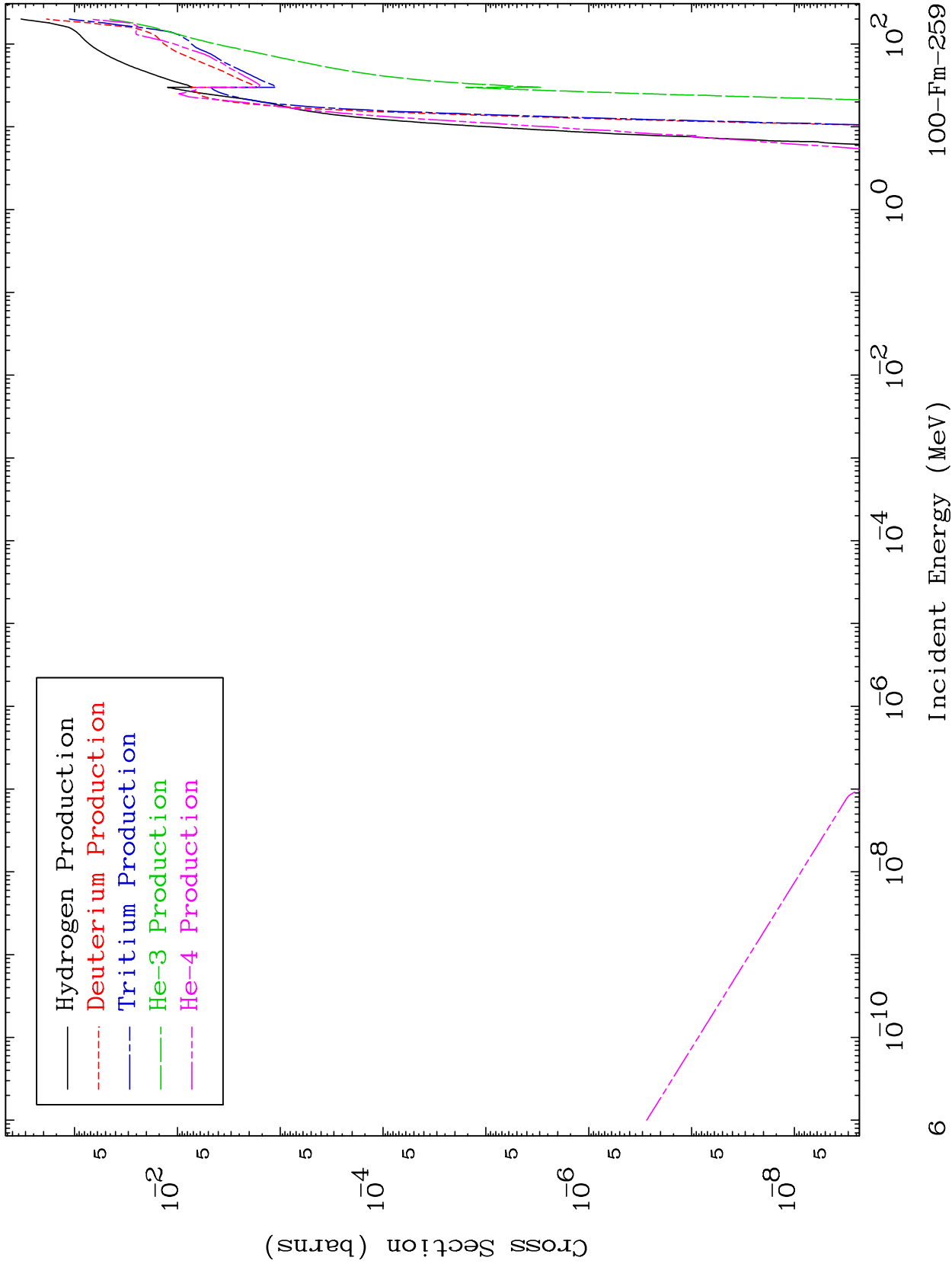
100-Fm-259



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Particle Production  
293 Kelvin Cross Sections

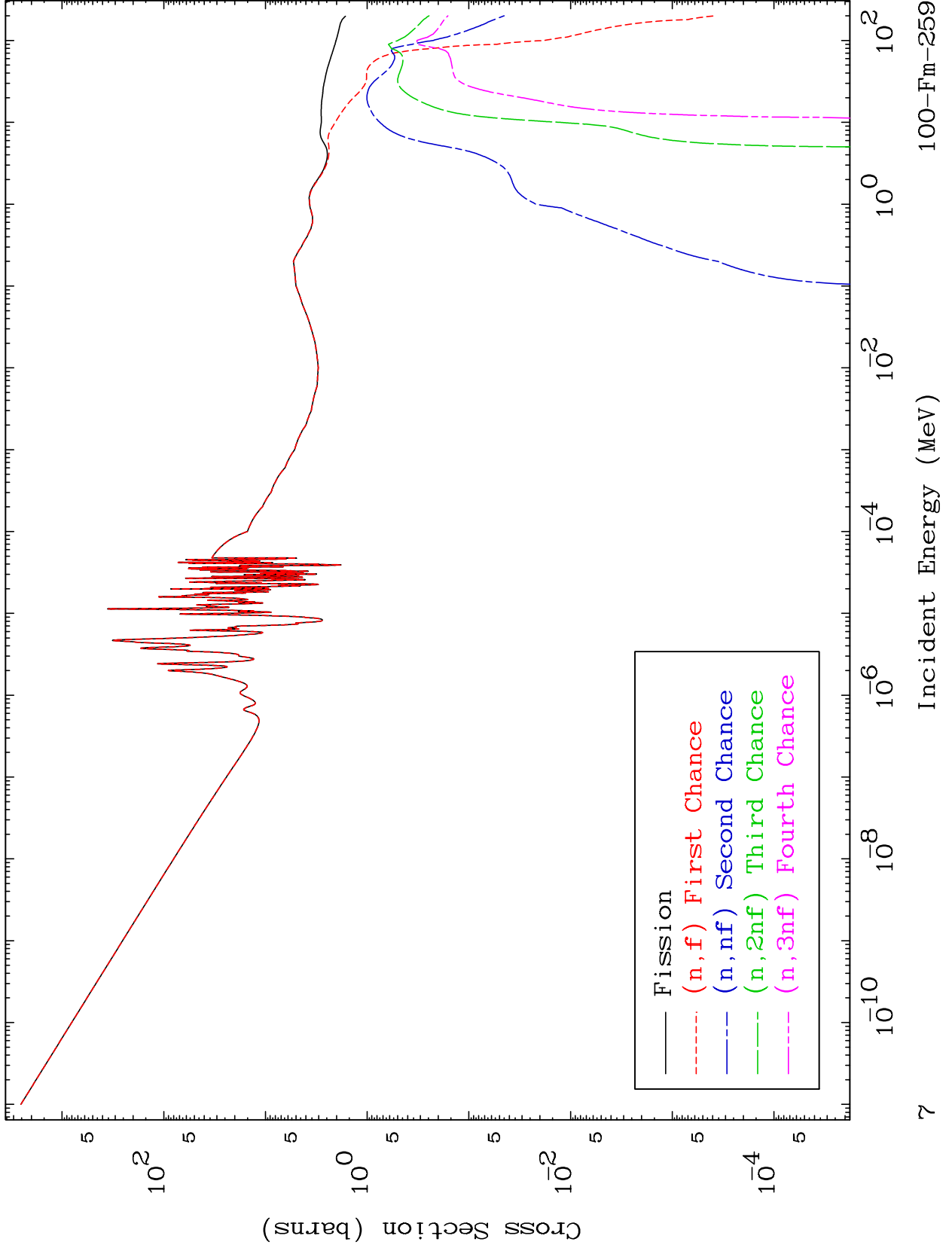
100-Fm-259



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Fission  
293 Kelvin Cross Sections

100-Fm-259



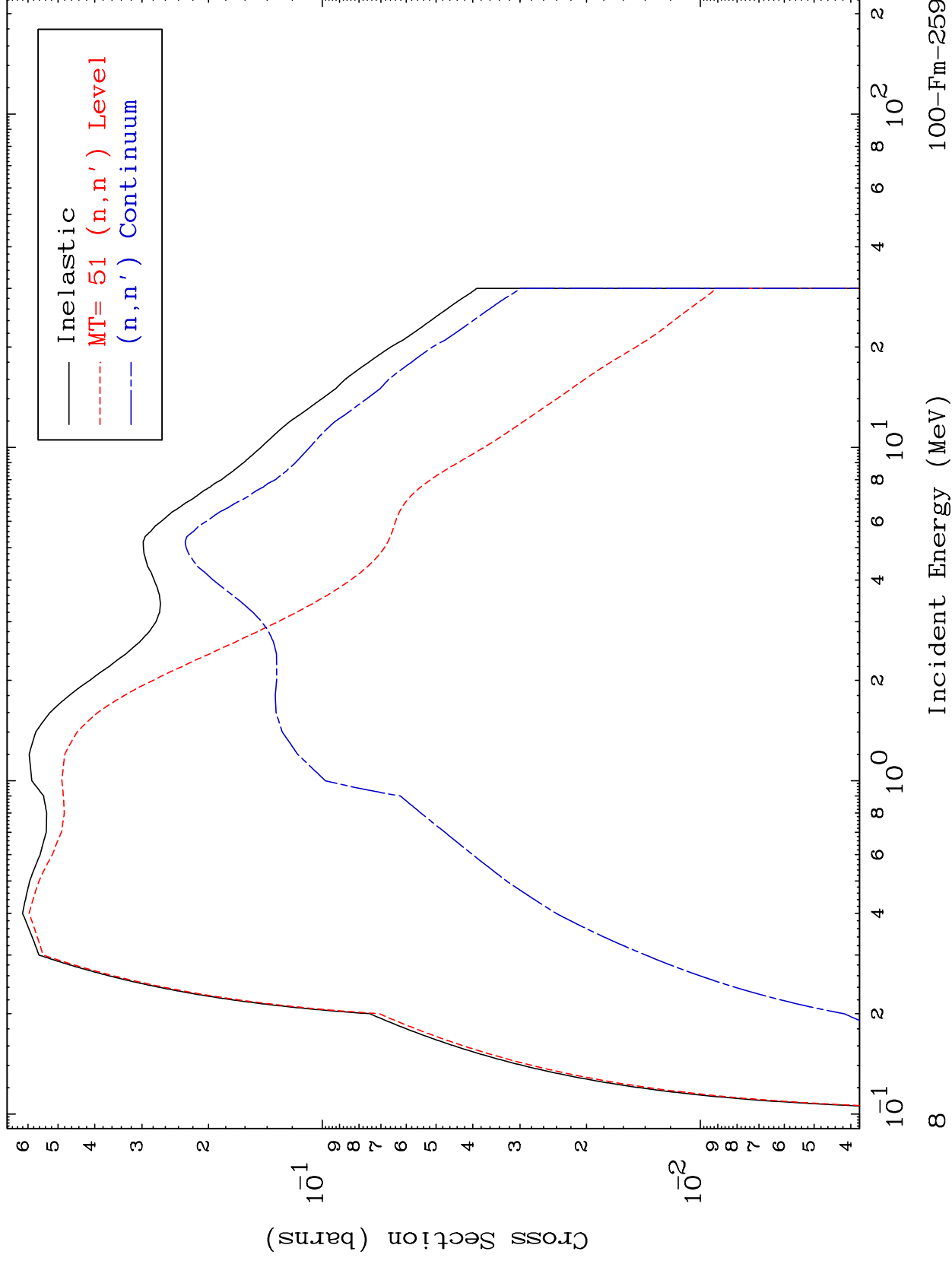


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(n,n') Level

100-Fm-259

293 Kelvin Cross Sections



100-Fm-259

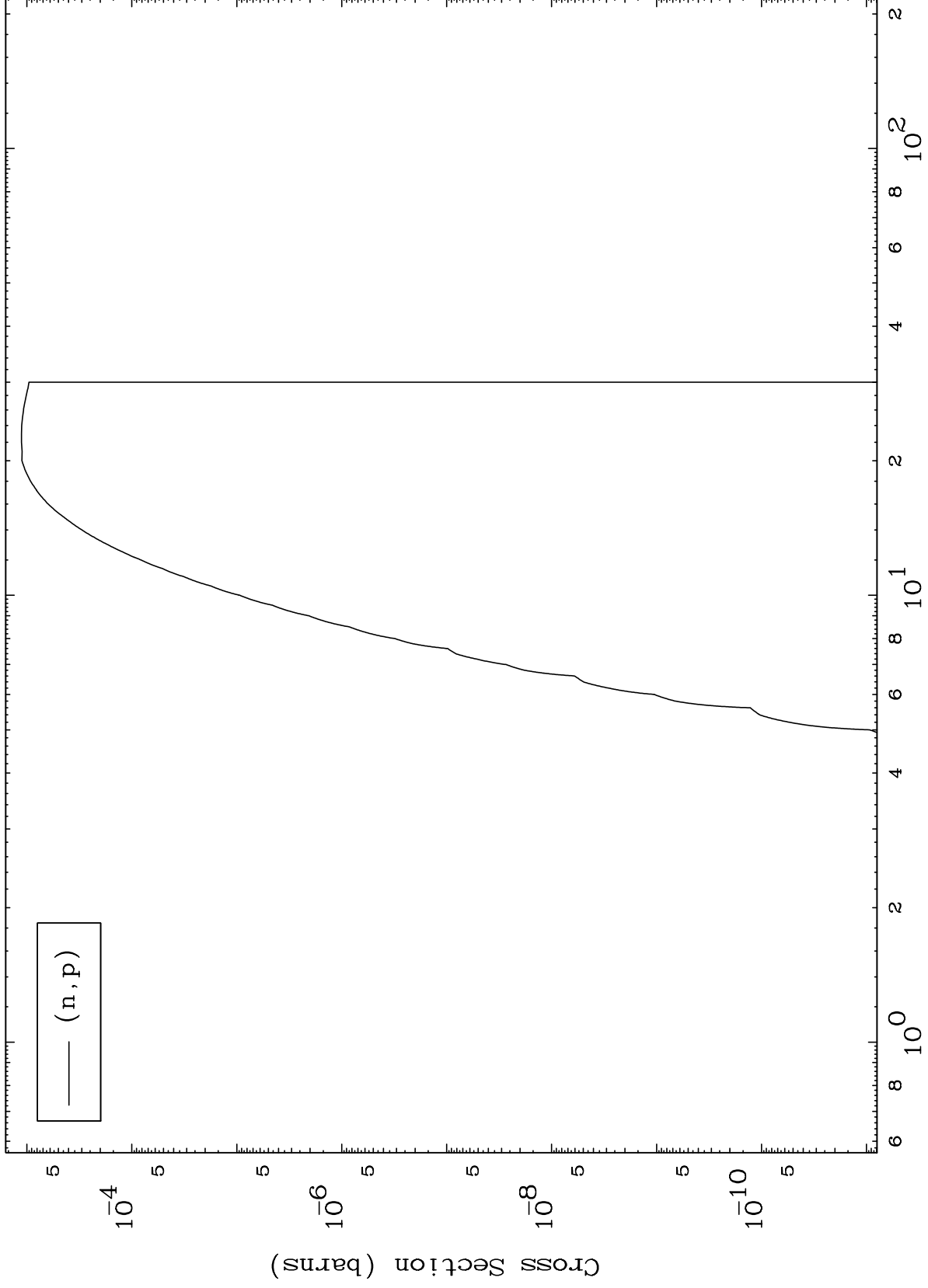
Incident Energy (MeV)

8

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(n,p) Levels  
293 Kelvin Cross Sections

100-Fm-259



9

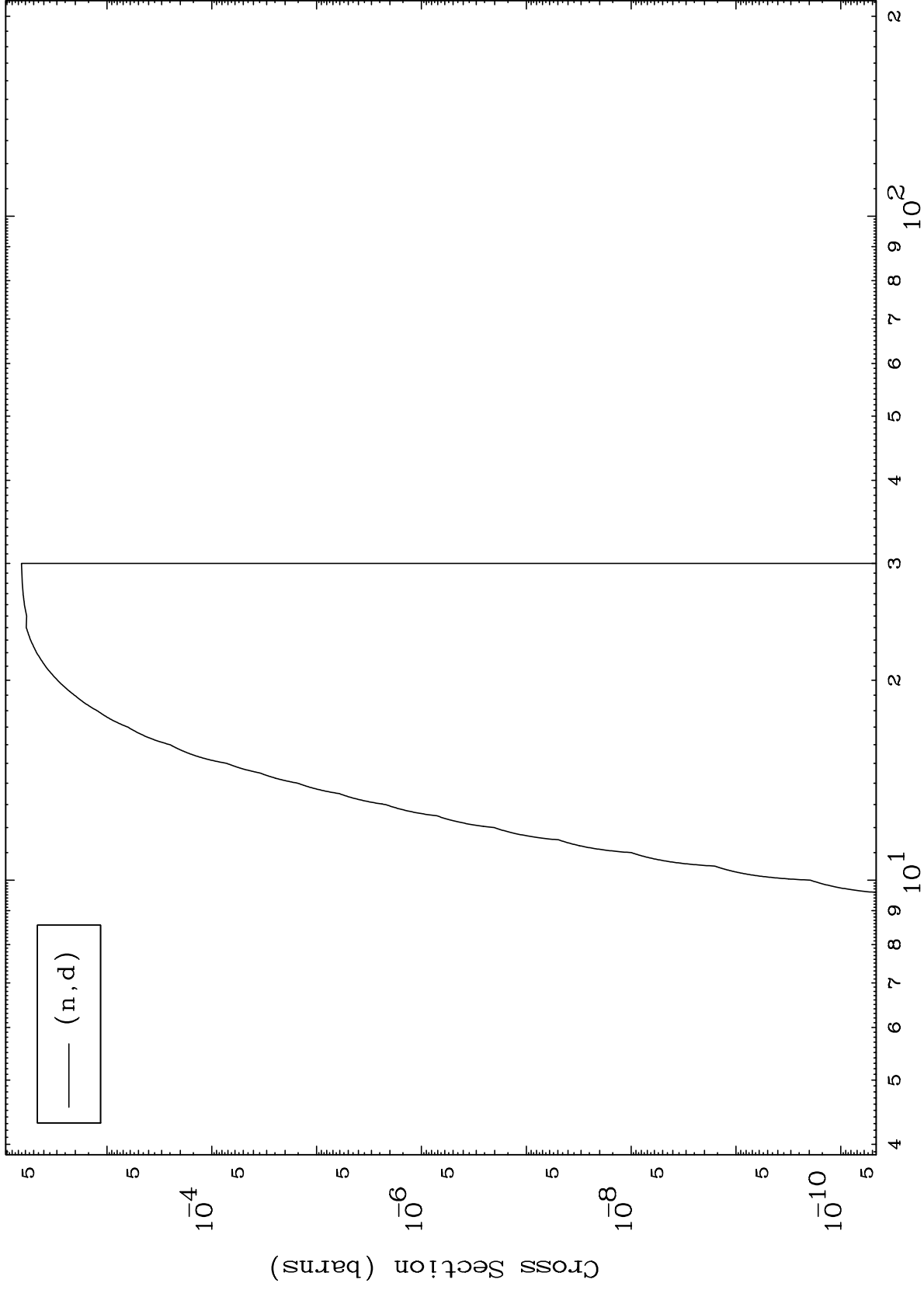
Incident Energy (MeV)

100-Fm-259

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(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-259



10

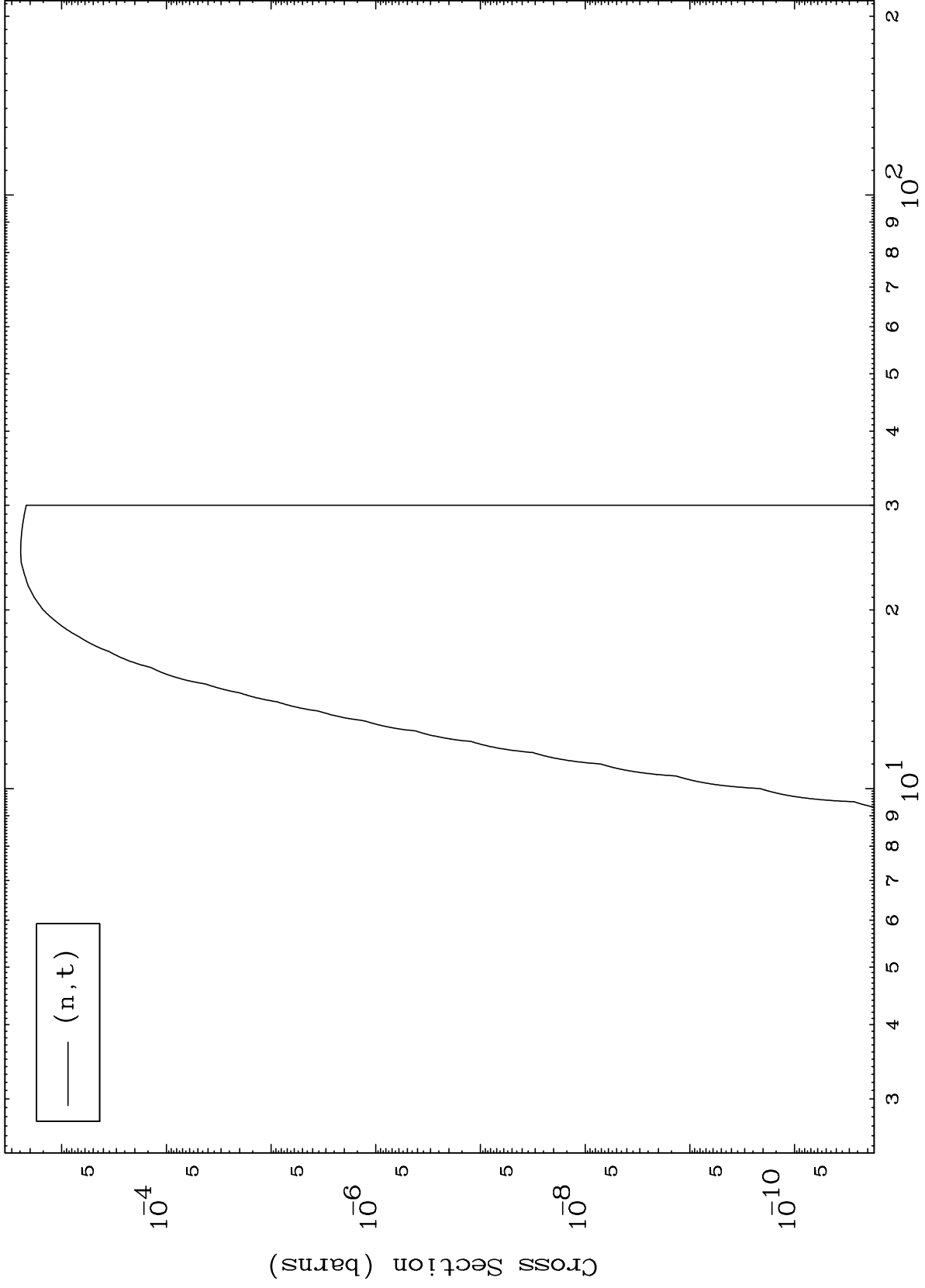
Incident Energy (MeV)

100-Fm-259

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(n,t) Levels  
293 Kelvin Cross Sections

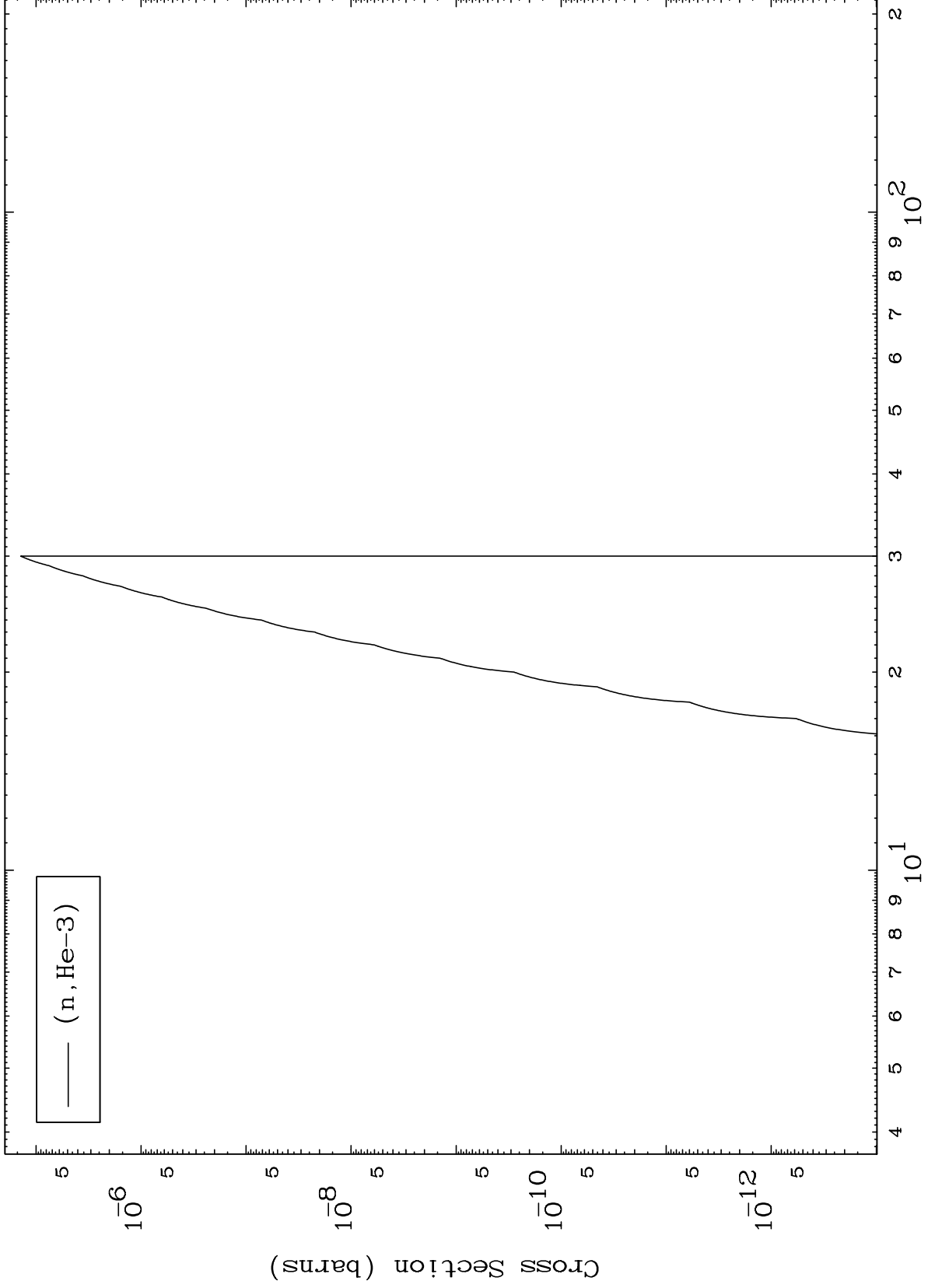
100-Fm-259



MAT 9940

(n,He3) Levels  
293 Kelvin Cross Sections

100-Fm-259



12

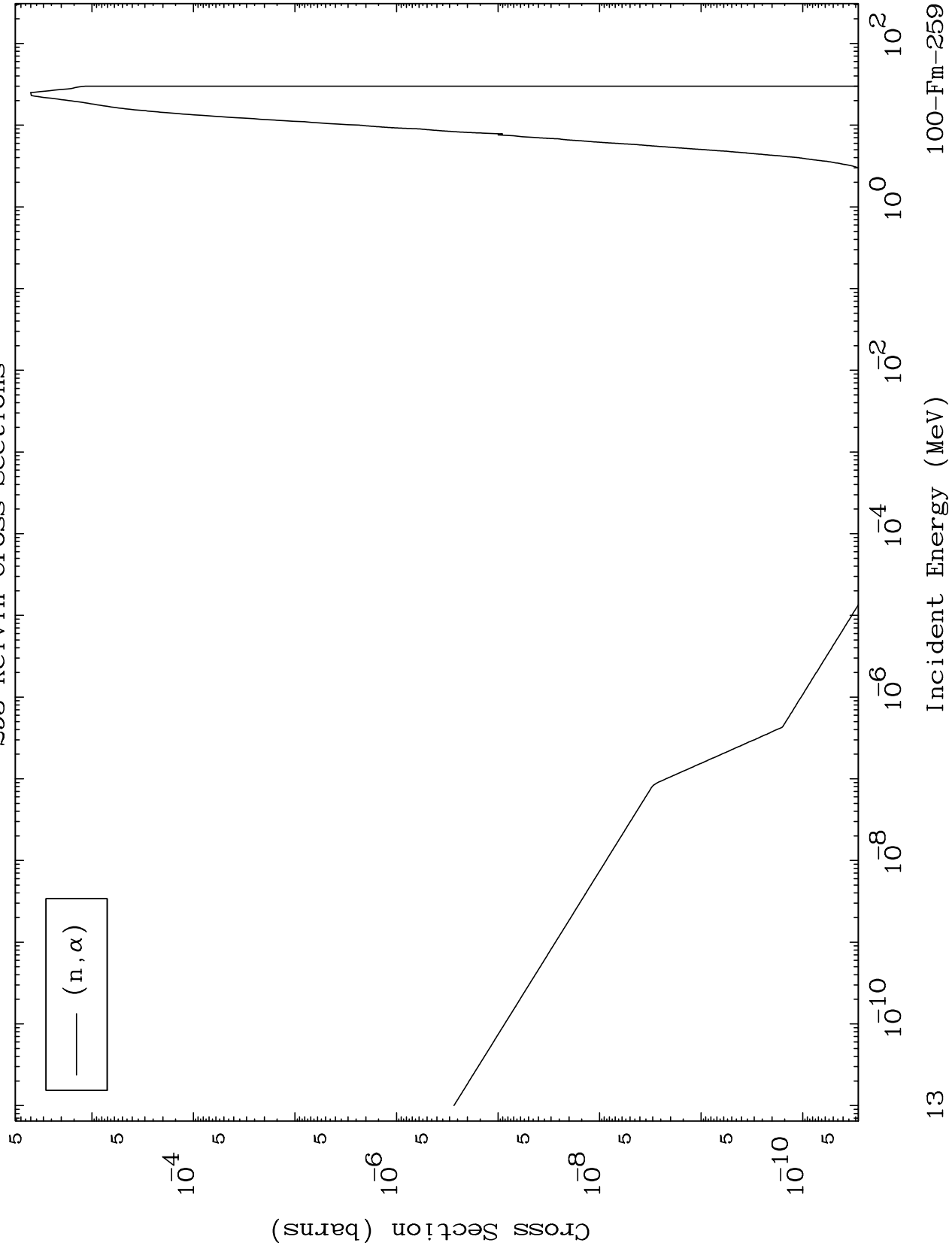
Incident Energy (MeV)

100-Fm-259

MAT 9940

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

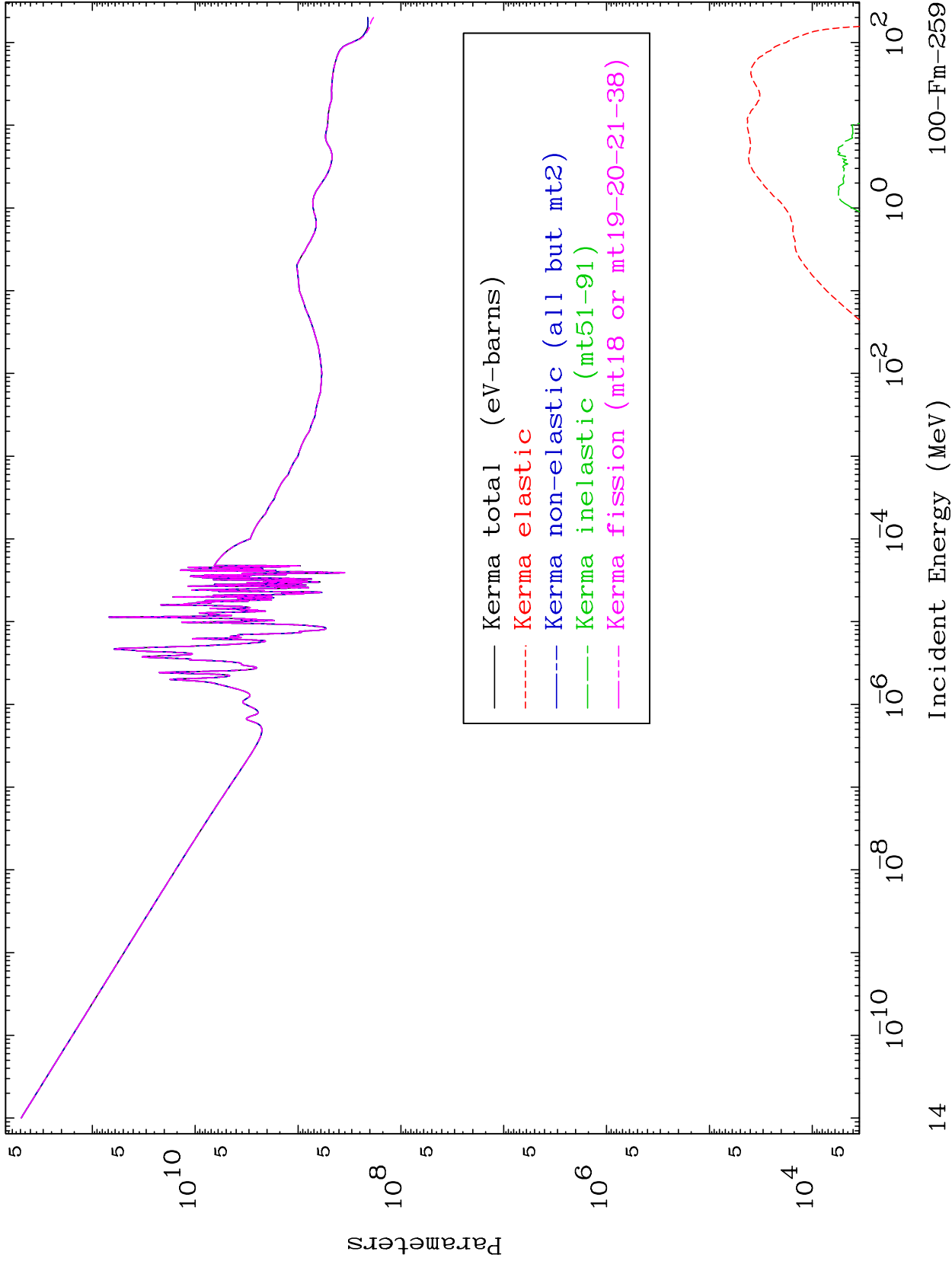
100-Fm-259



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Energy Release  
Parameters

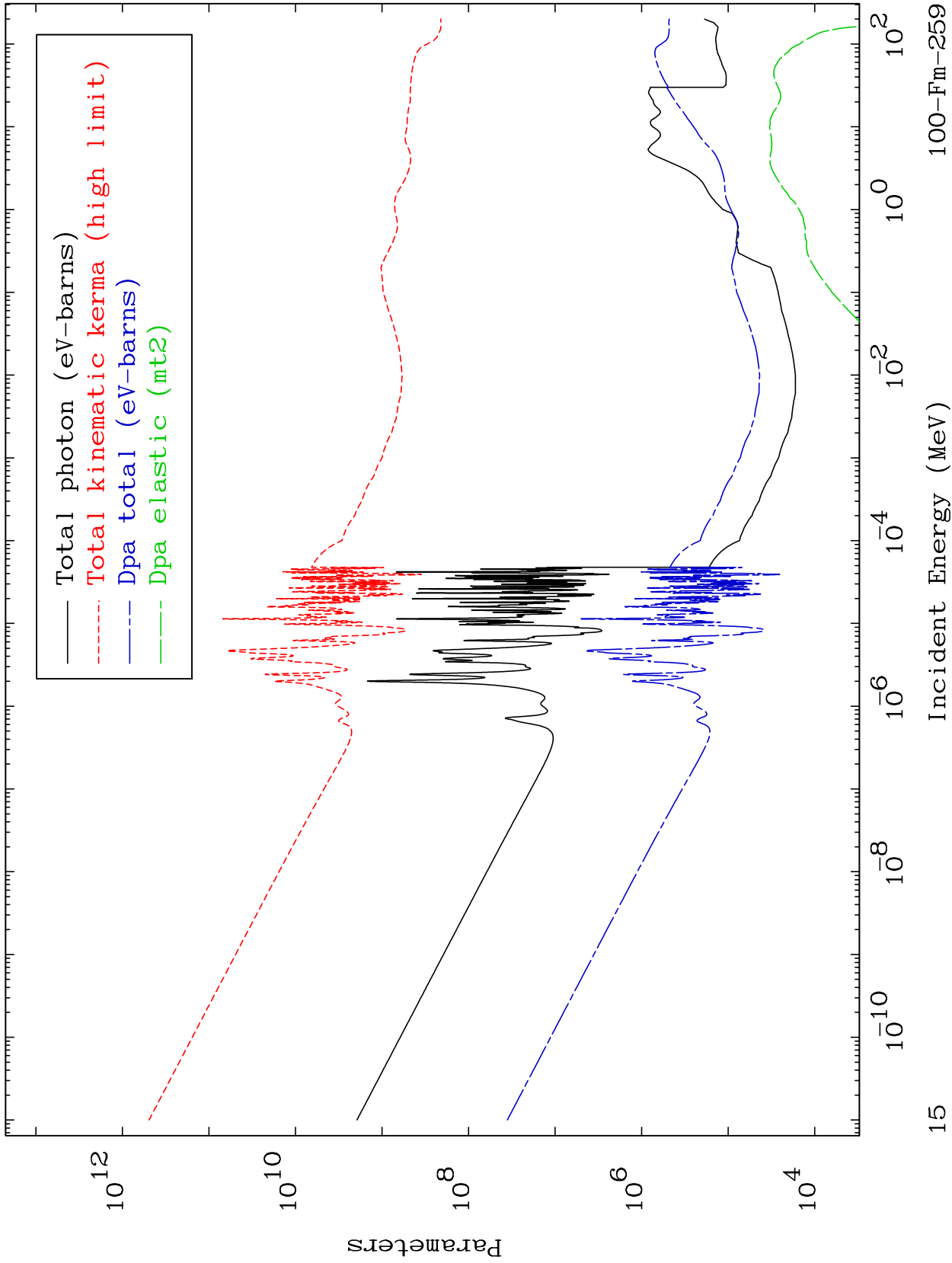
100-Fm-259



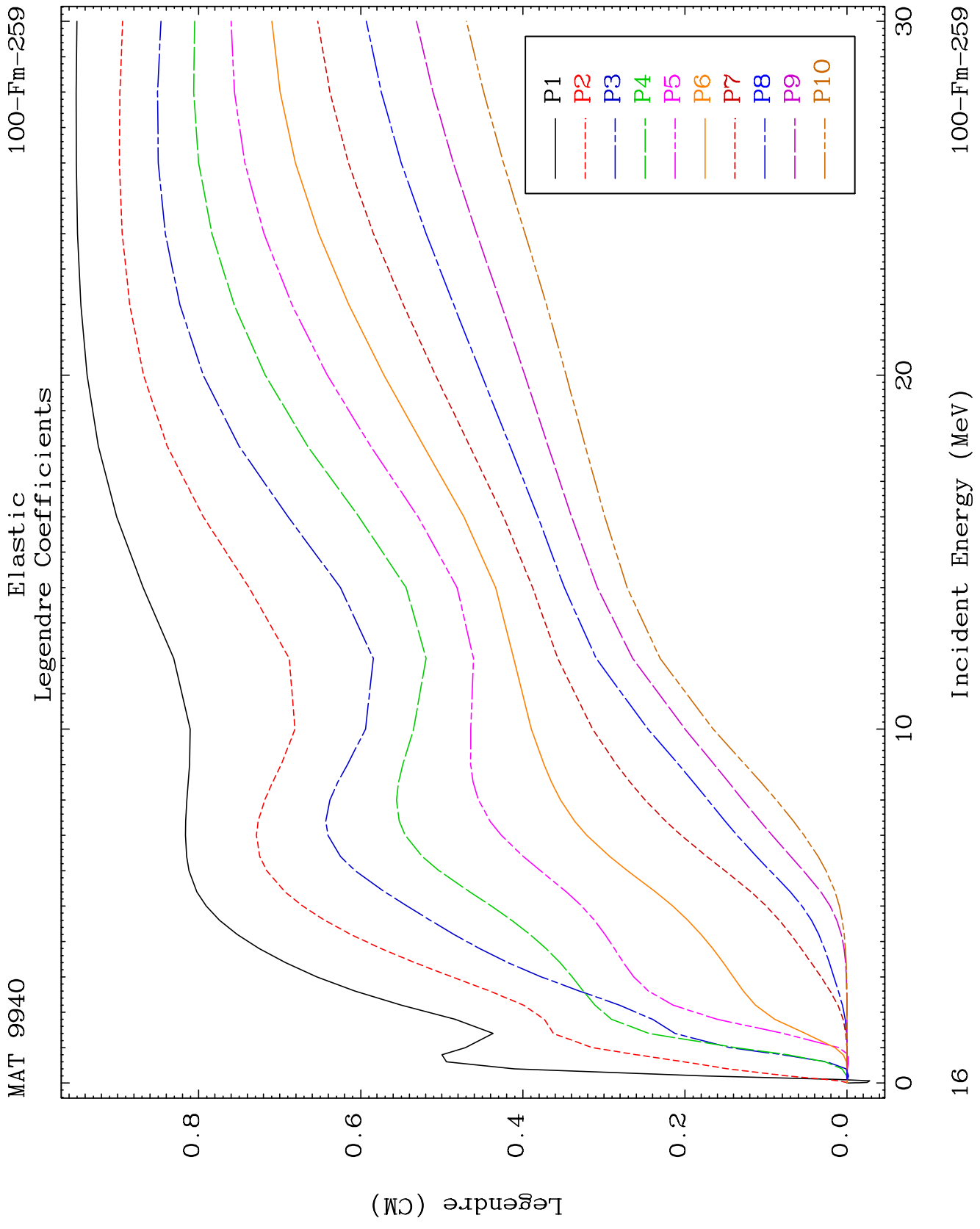
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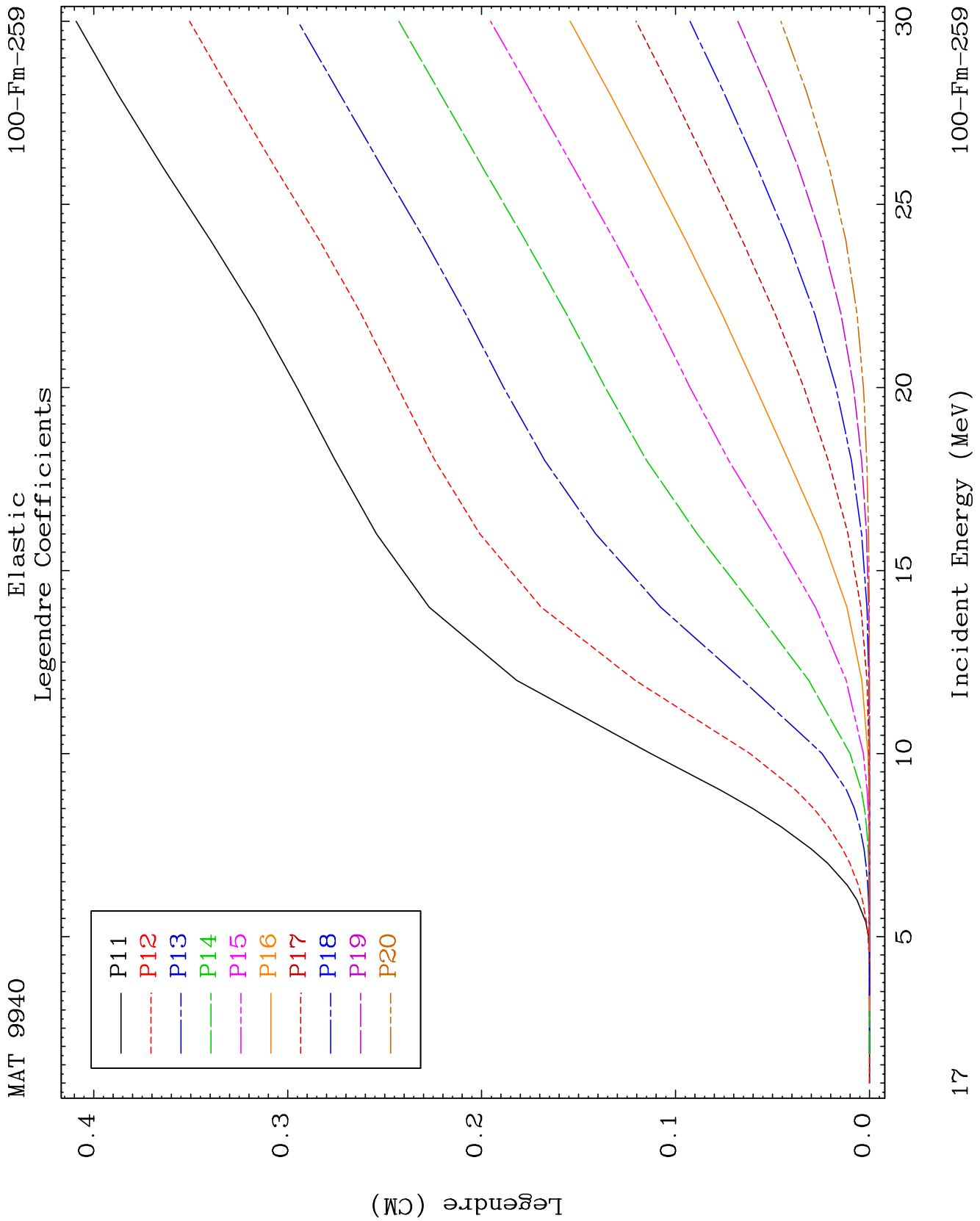
Energy Release  
Parameters

100-Fm-259





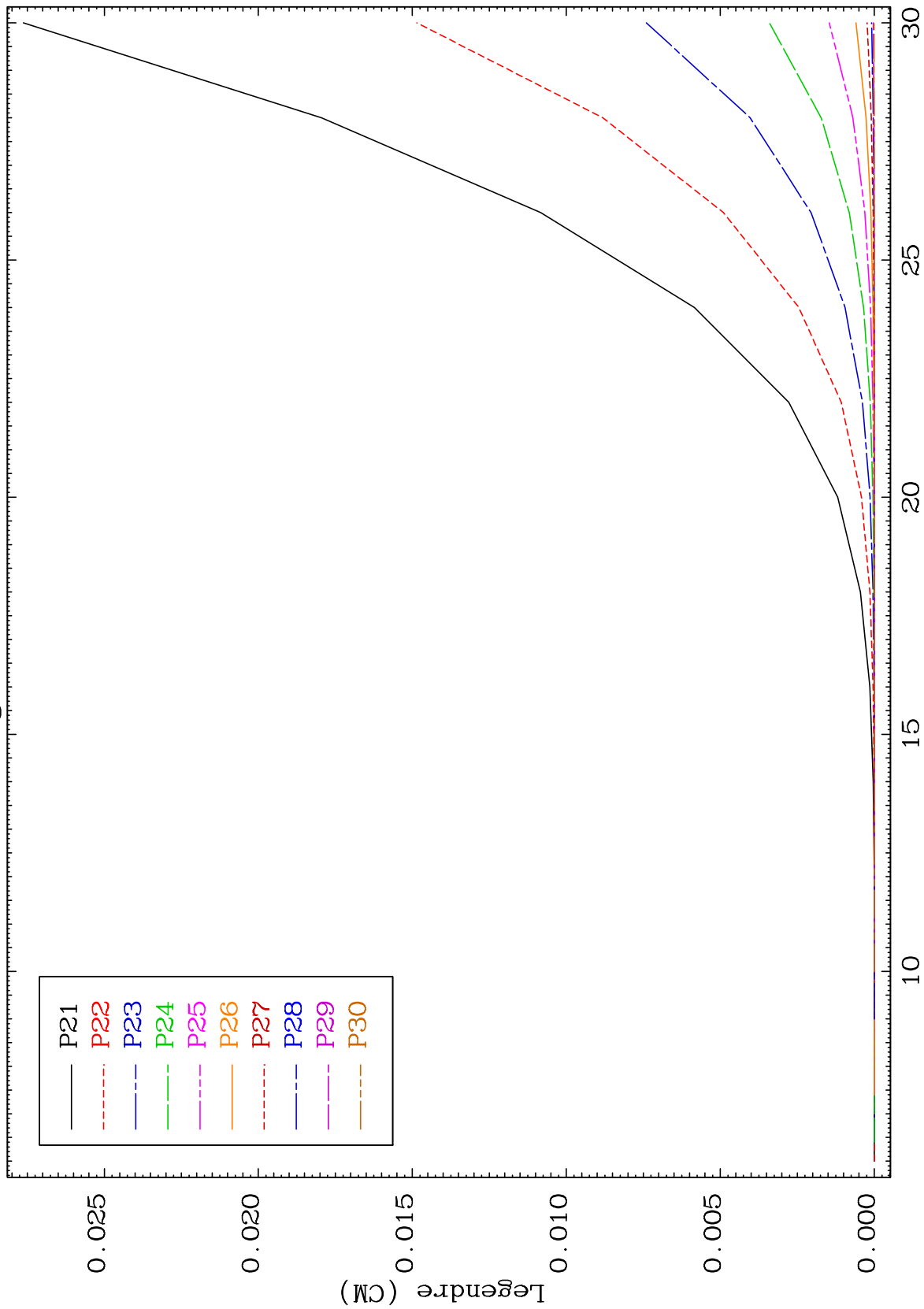




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Elastic Legendre Coefficients

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18

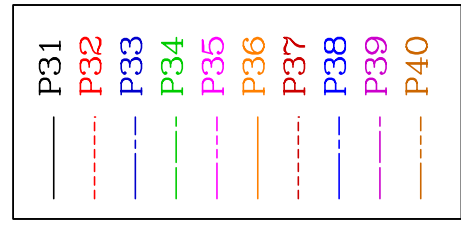
Incident Energy (MeV)

100-Fm-259

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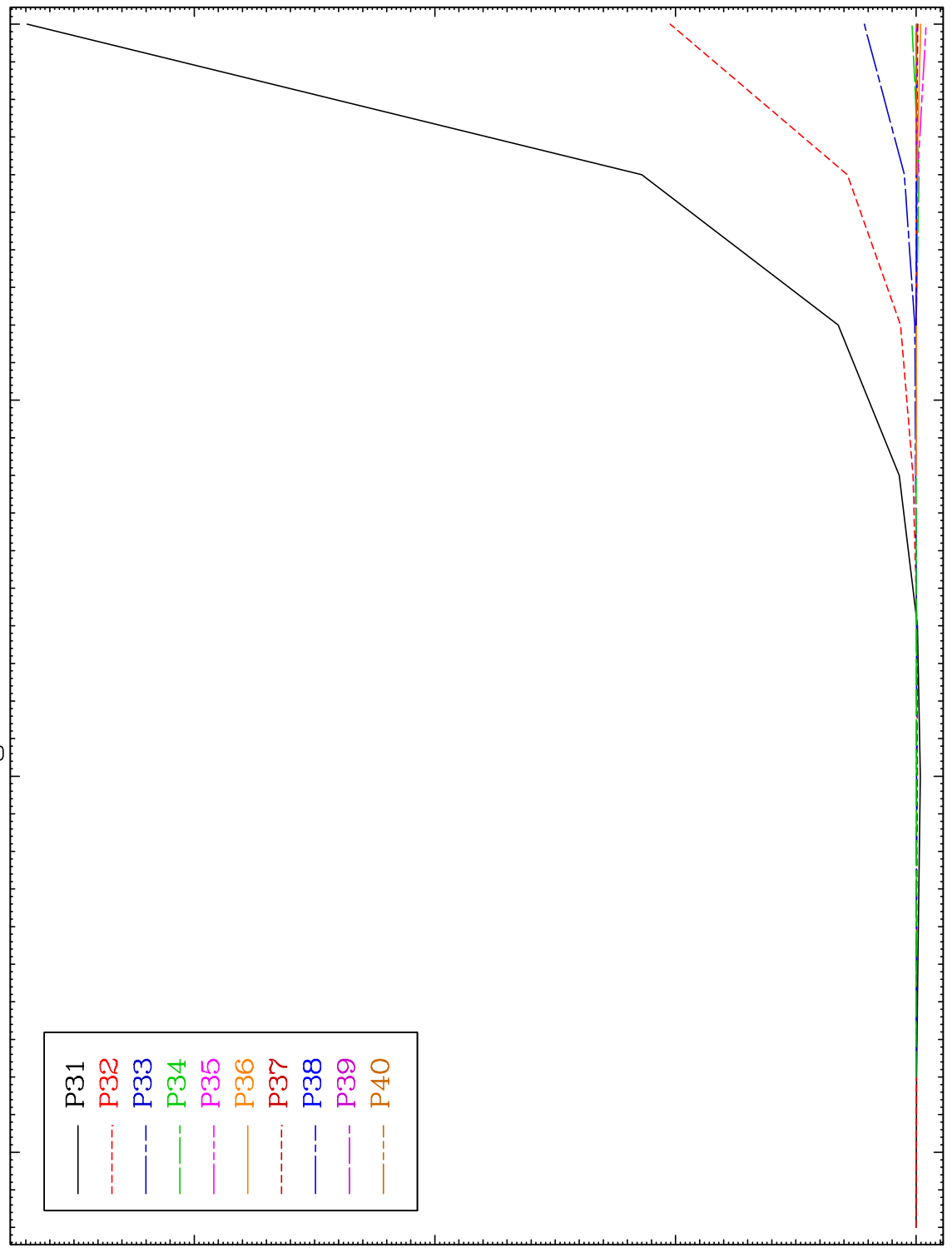
Elastic Legendre Coefficients

100-Fm-259



$\times 10^{-6}$

Legendre (CM)



19

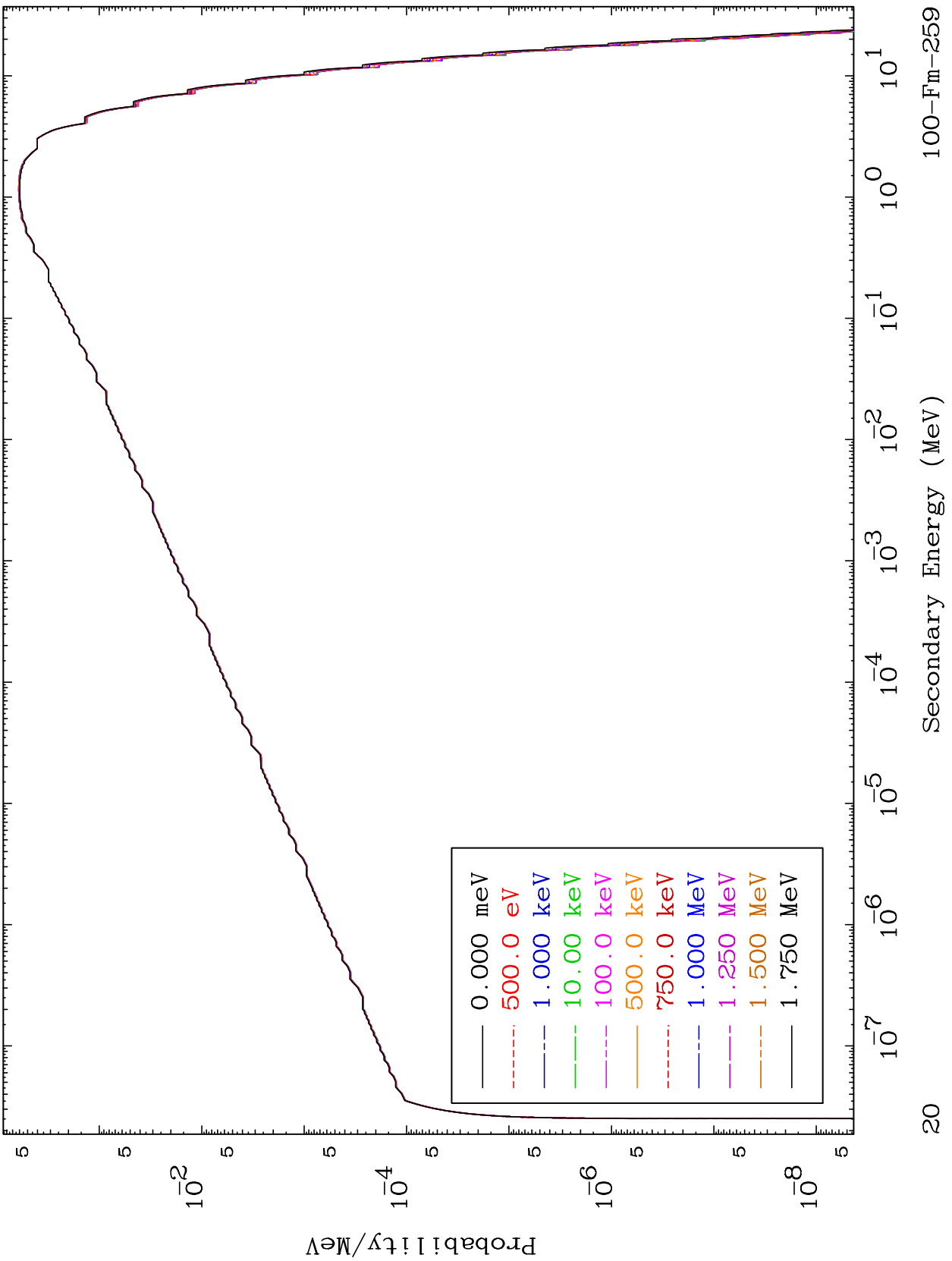
Incident Energy (MeV)

100-Fm-259

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### Fission Energy Distribution

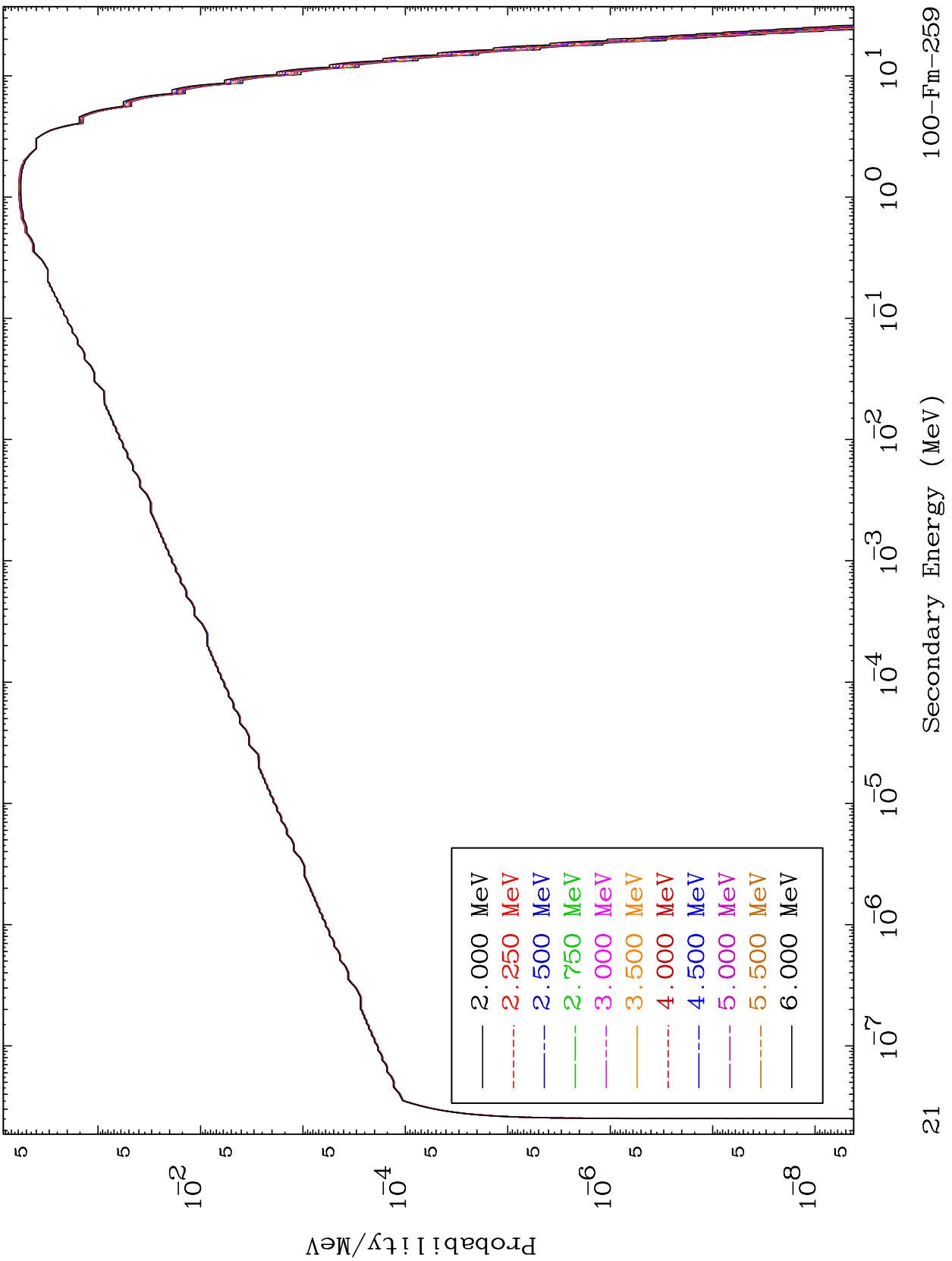
100-Fm-259



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### Fission Energy Distribution

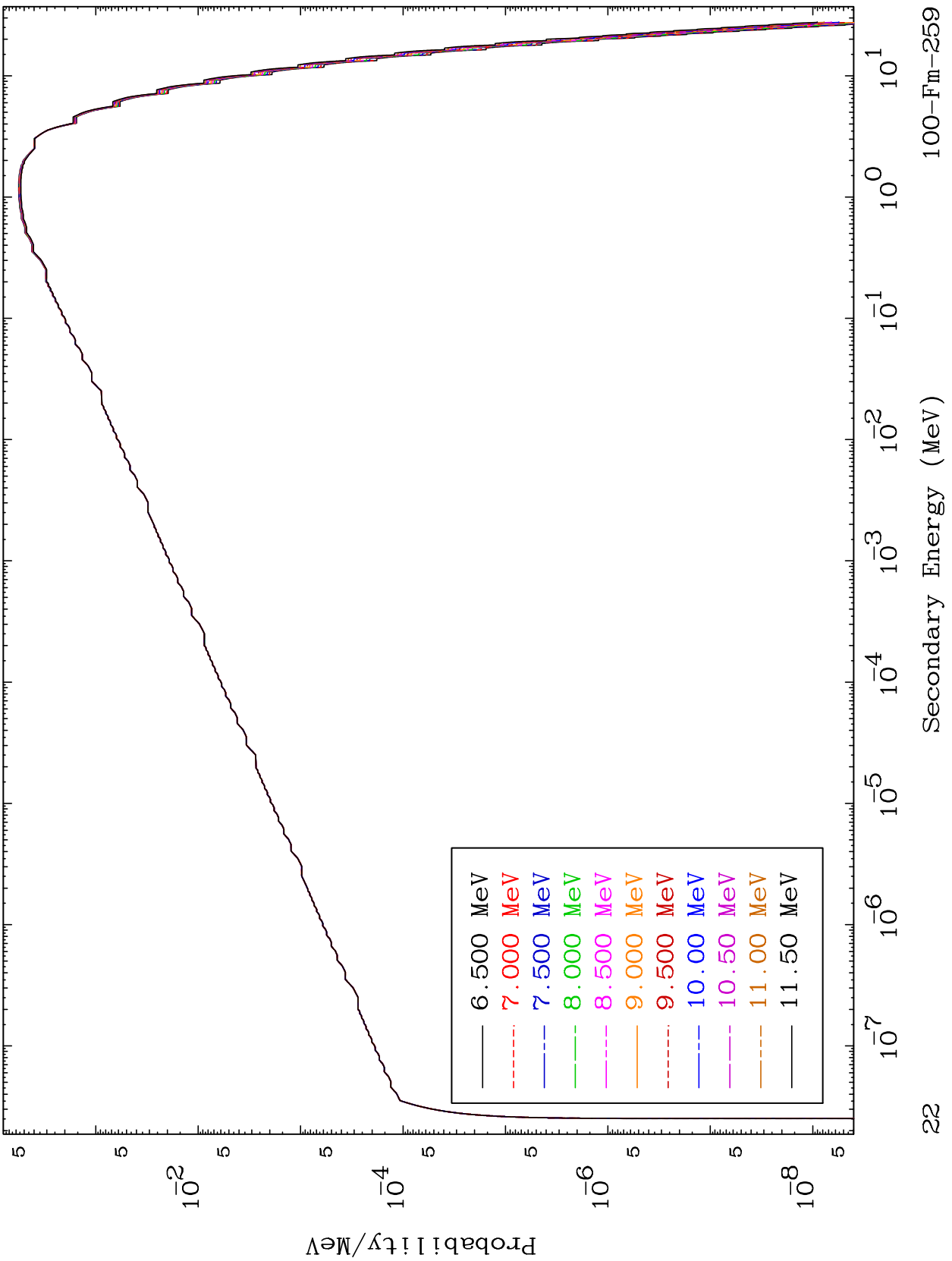
100-Fm-259



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### Fission Energy Distribution

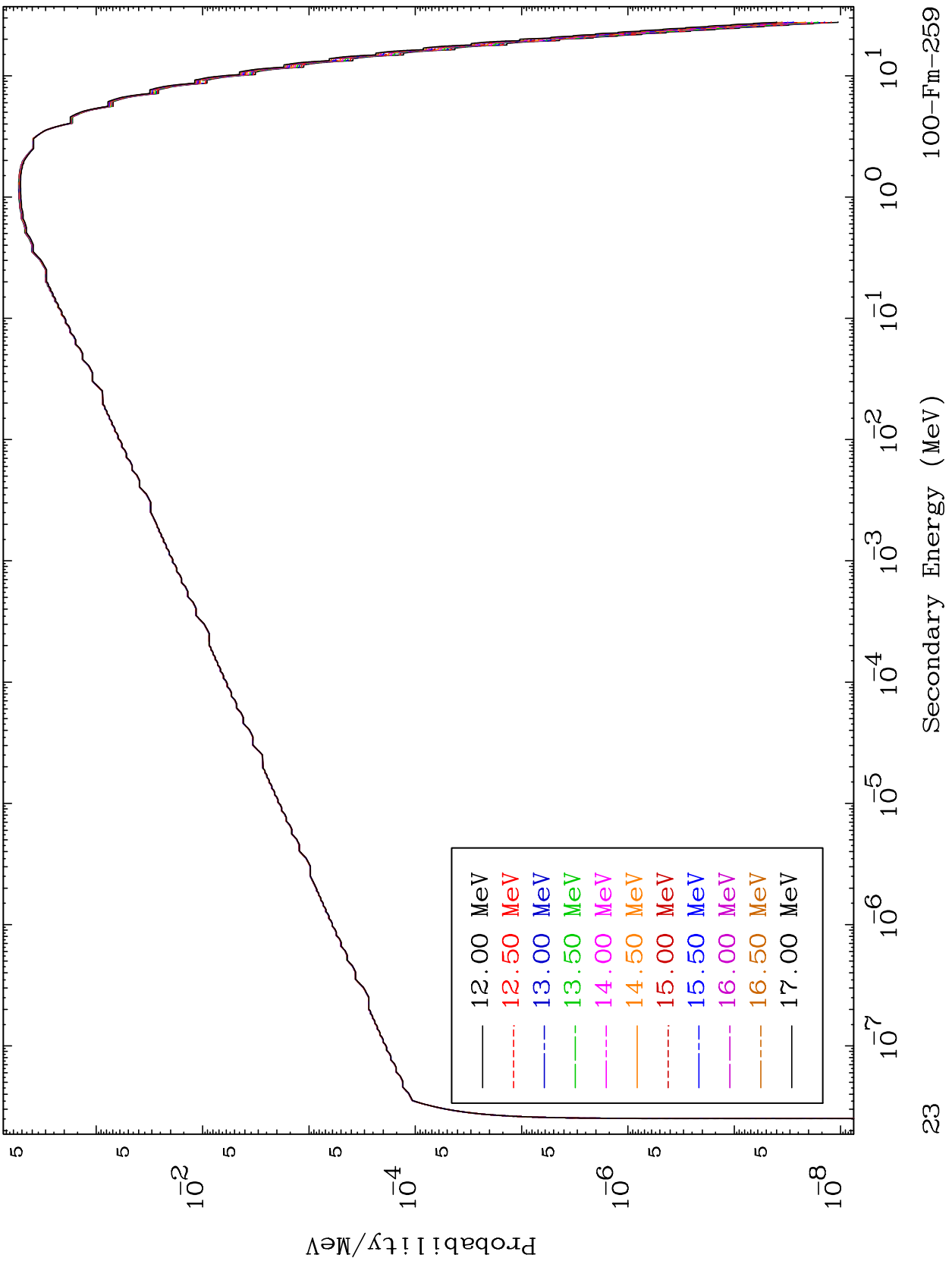
100-Fm-259



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### Fission Energy Distribution

100-Fm-259

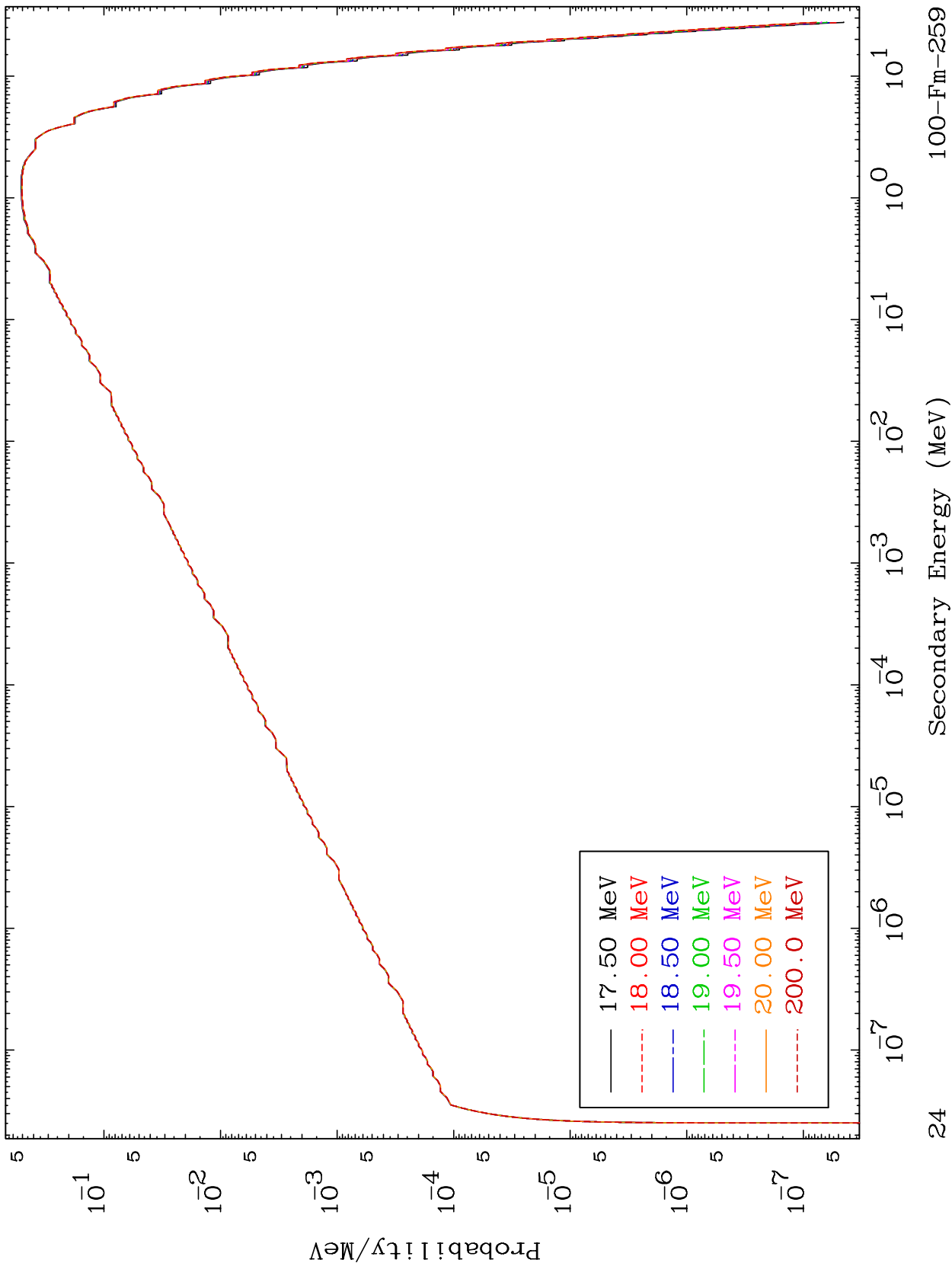




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### Fission Energy Distribution

100-Fm-259

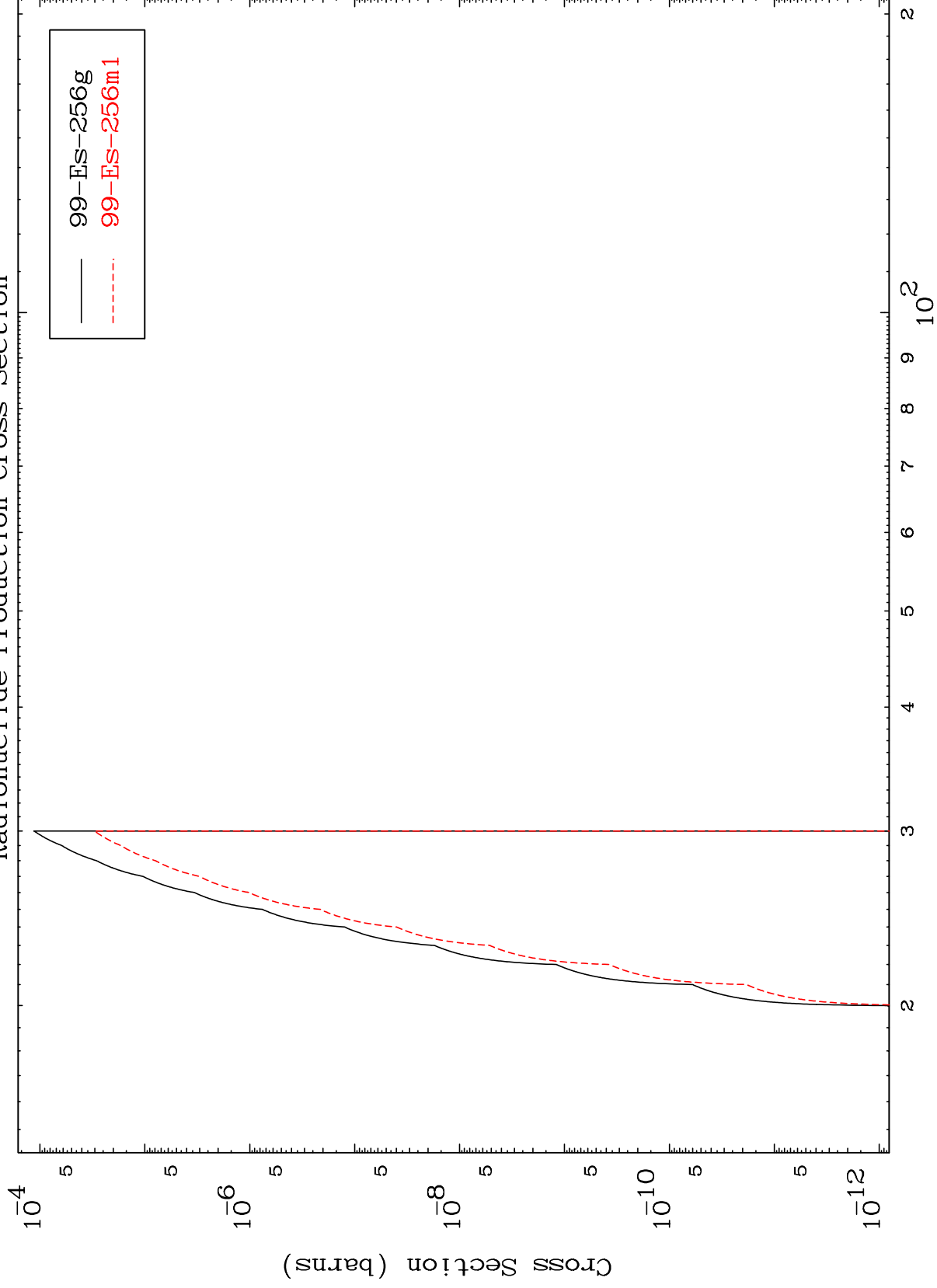


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(n,2n) d

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

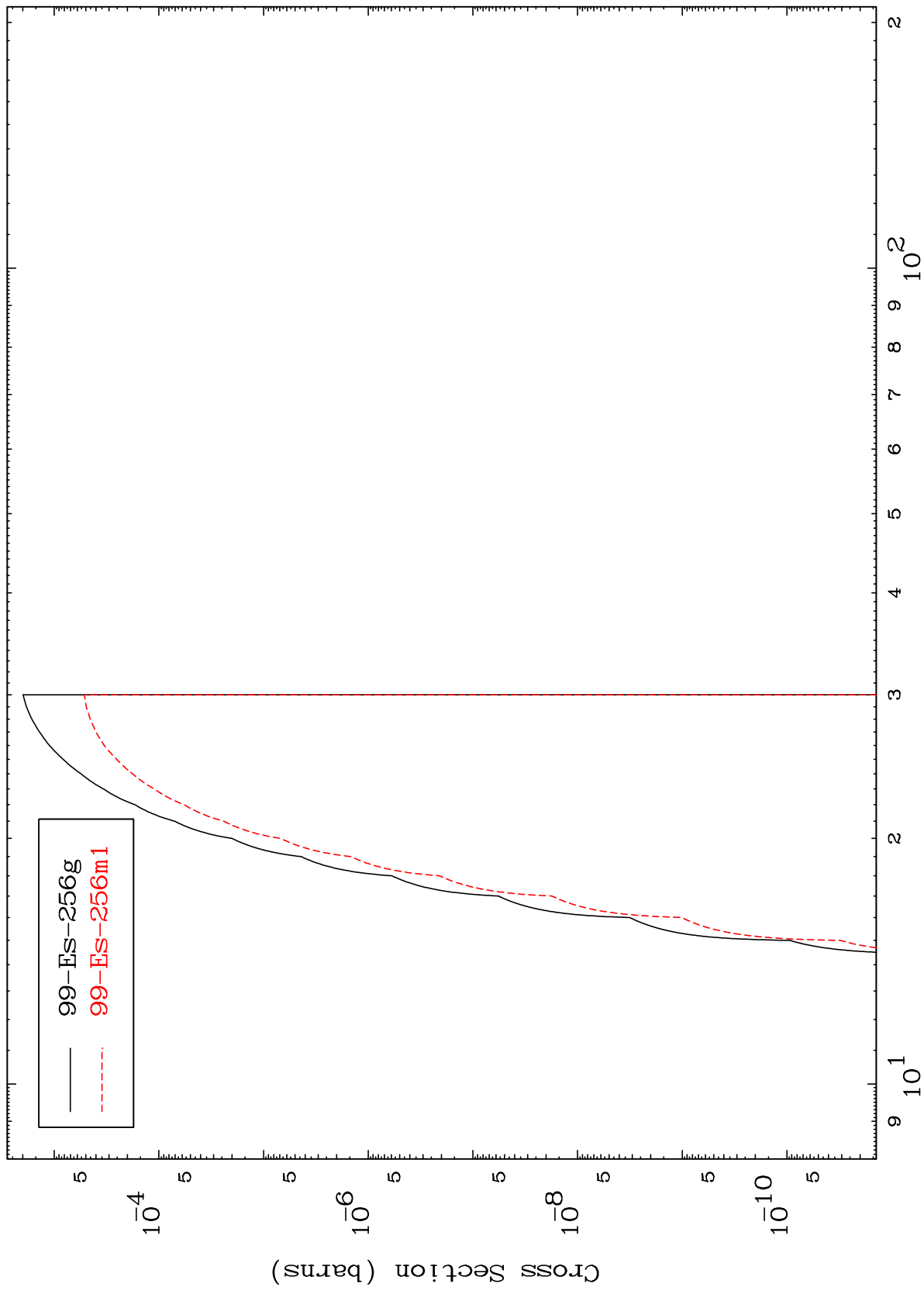
100-Fm-259

MAT 9940

(n,n') t

100-Fm-259

Radionuclide Production Cross Section



26

Incident Energy (MeV)

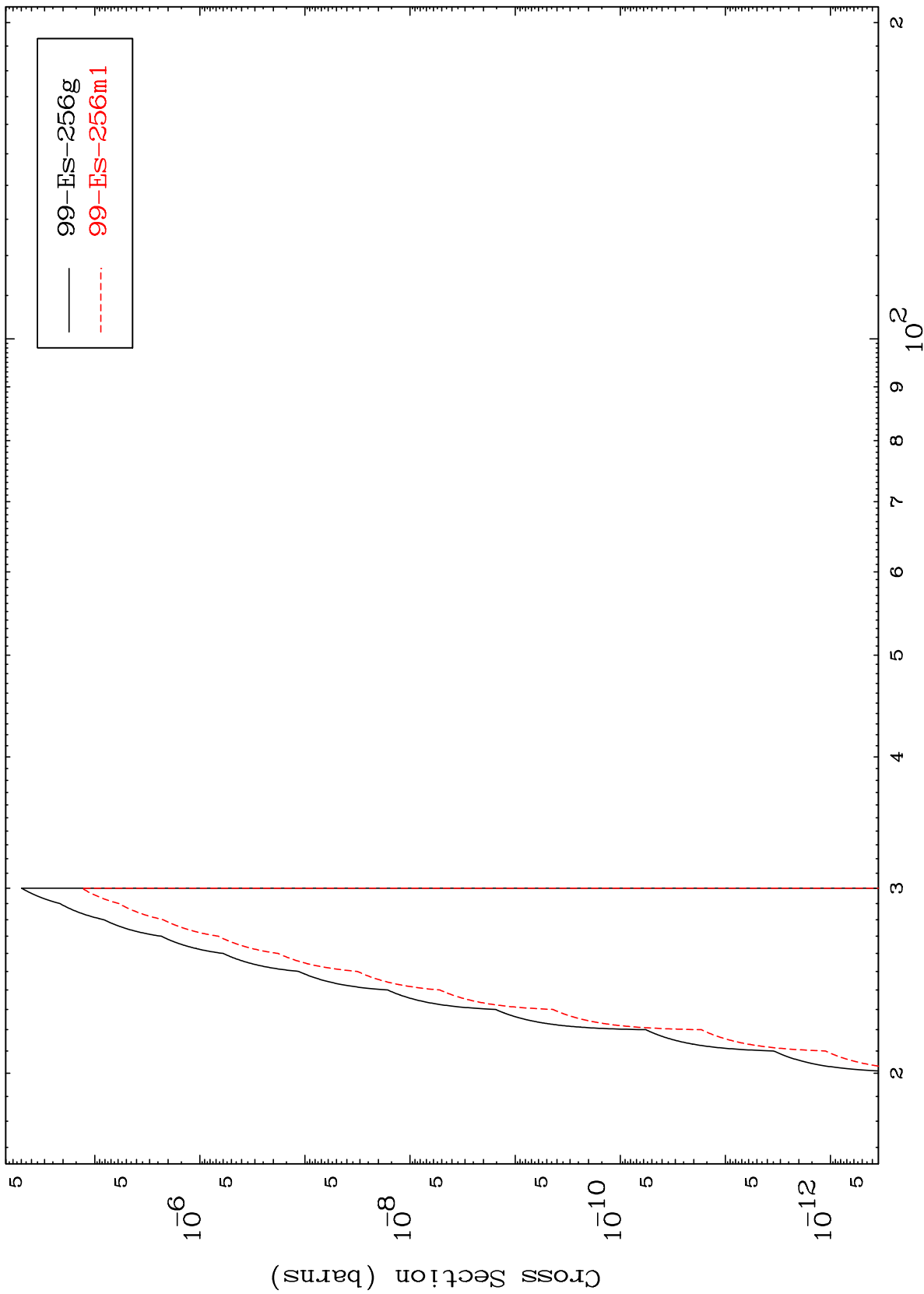
100-Fm-259

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(n,3n) p

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Radionuclide Production Cross Section



99-Es-256g  
99-Es-256m1