

Program EVALPLOT
(Version 2017-1)

by

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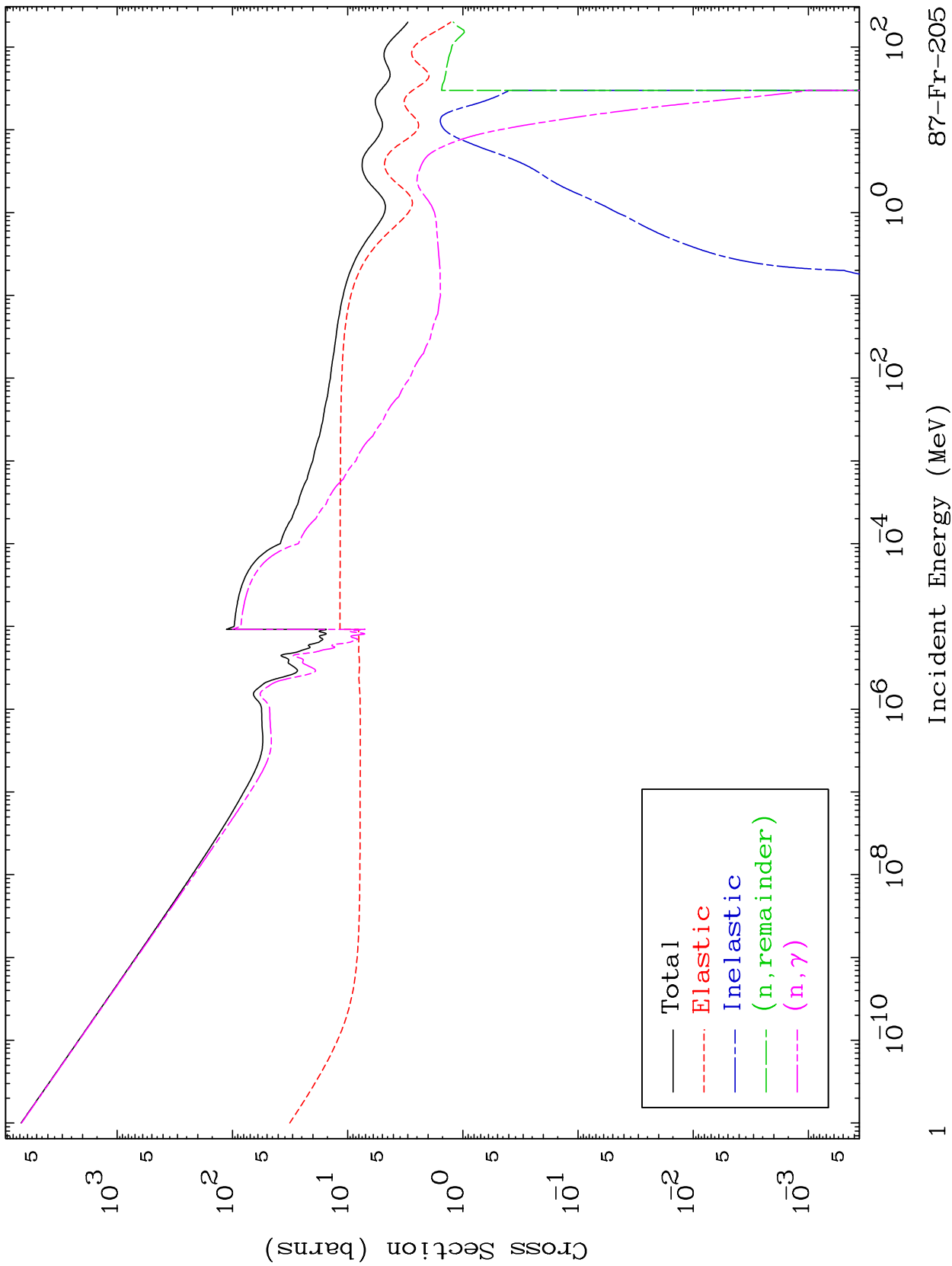
Press Mouse Button to Start

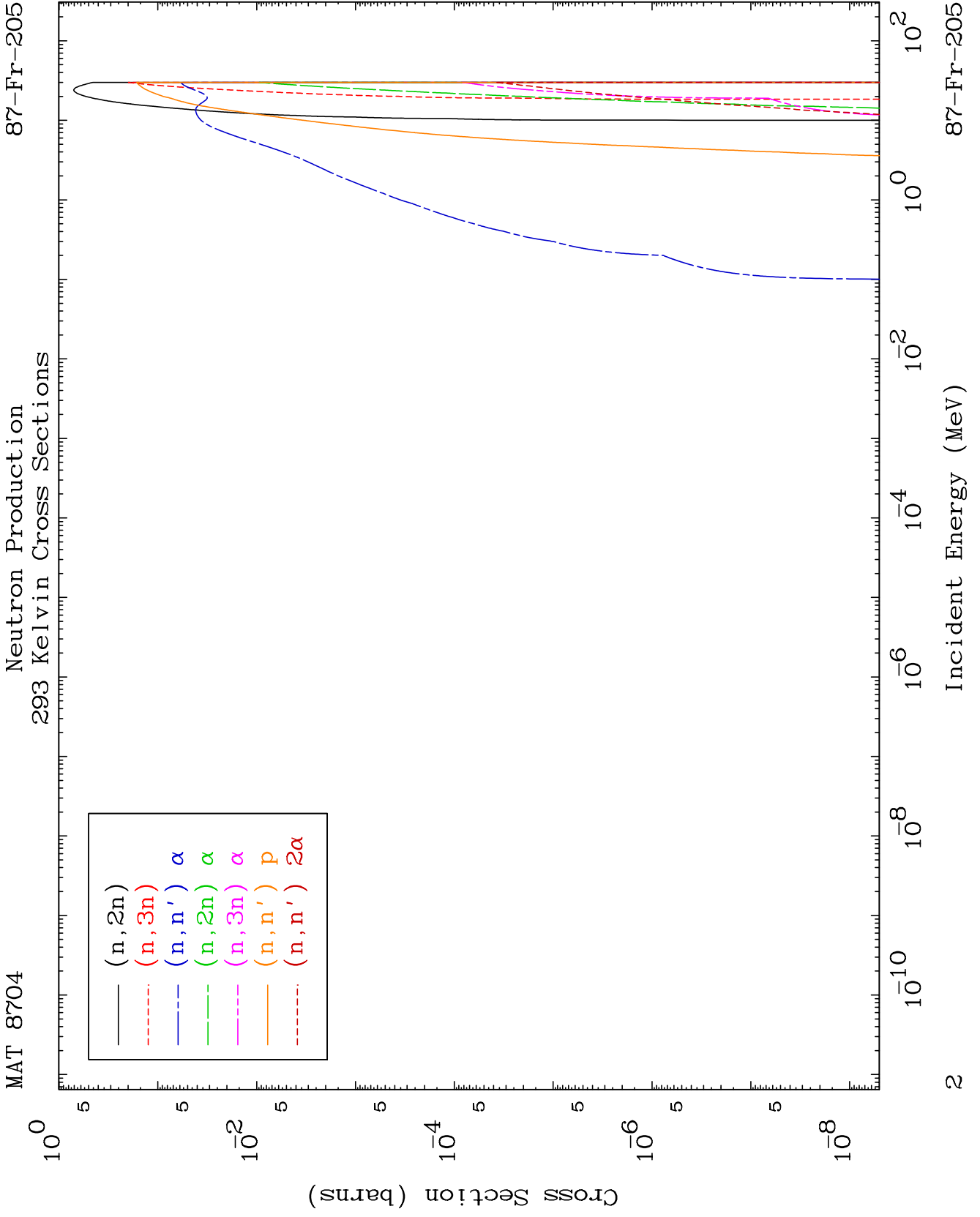
MAT 8704

Major

293 Kelvin Cross Sections

87-Fr-205

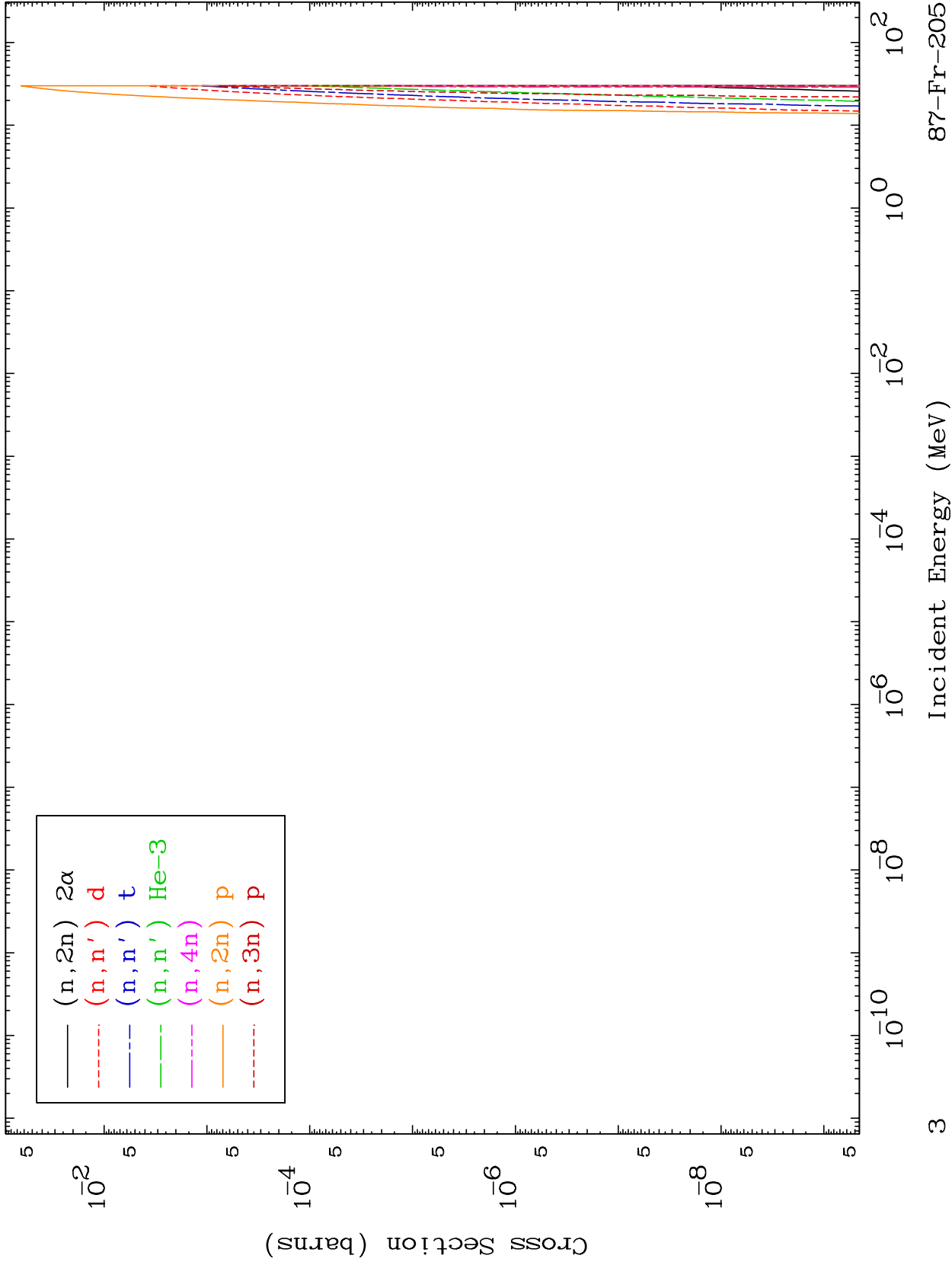




MAT 8704

Neutron Production
293 Kelvin Cross Sections

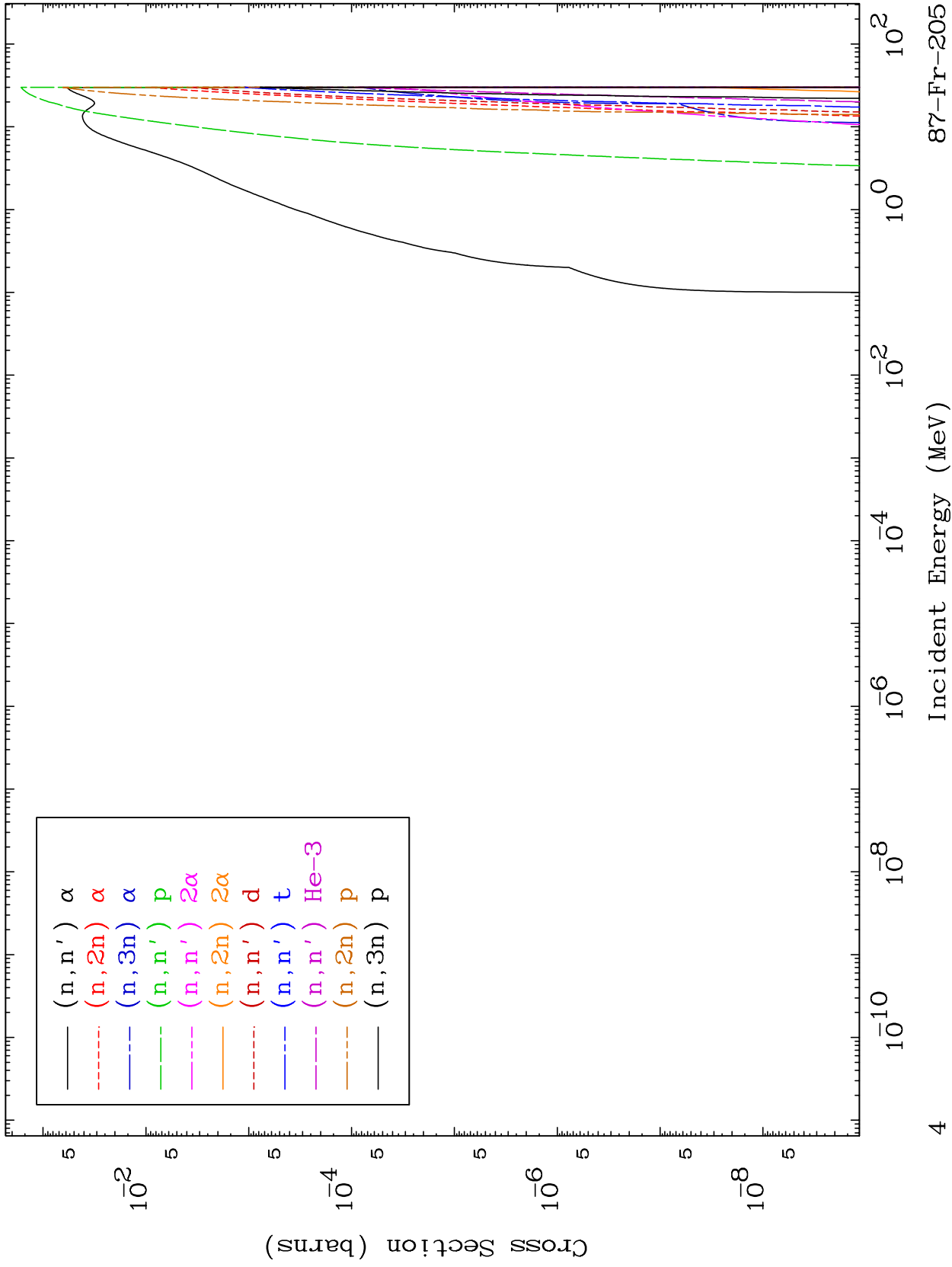
87-Fr-205



MAT 8704

Charged Particle
293 Kelvin Cross Sections

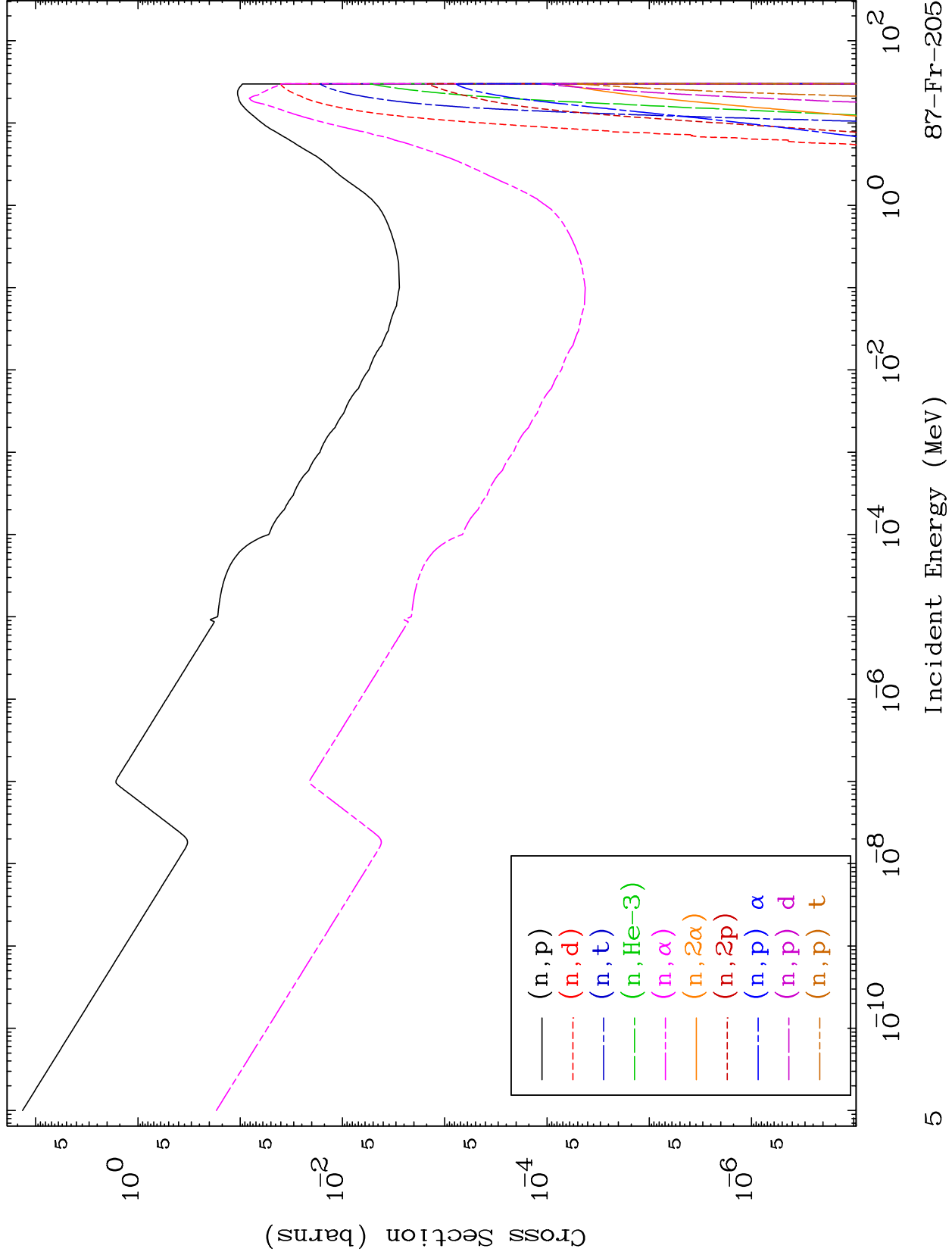
87-Fr-205



MAT 8704

Charged Particle
293 Kelvin Cross Sections

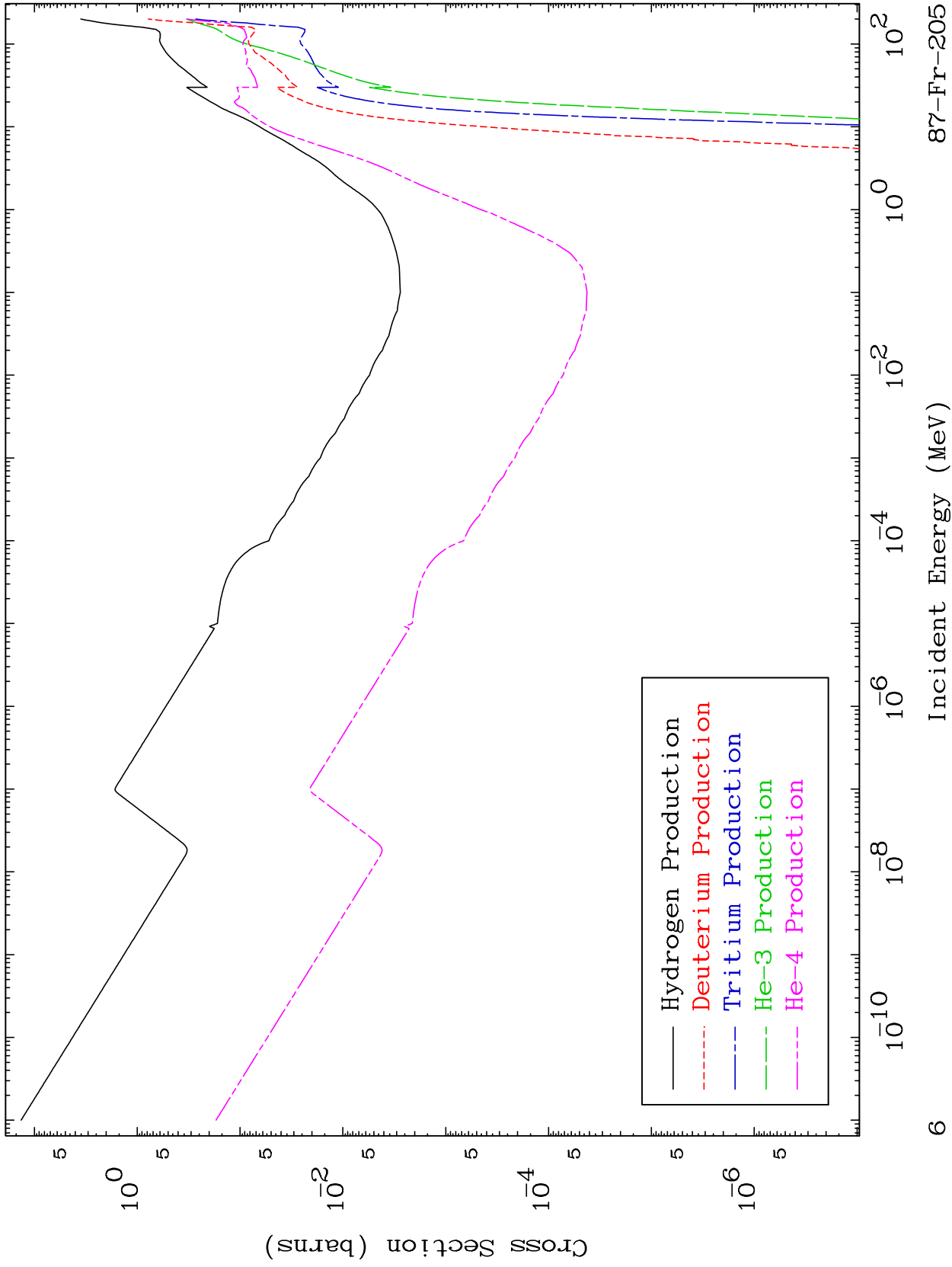
87-Fr-205



MAT 8704

Particle Production
293 Kelvin Cross Sections

87-Fr-205

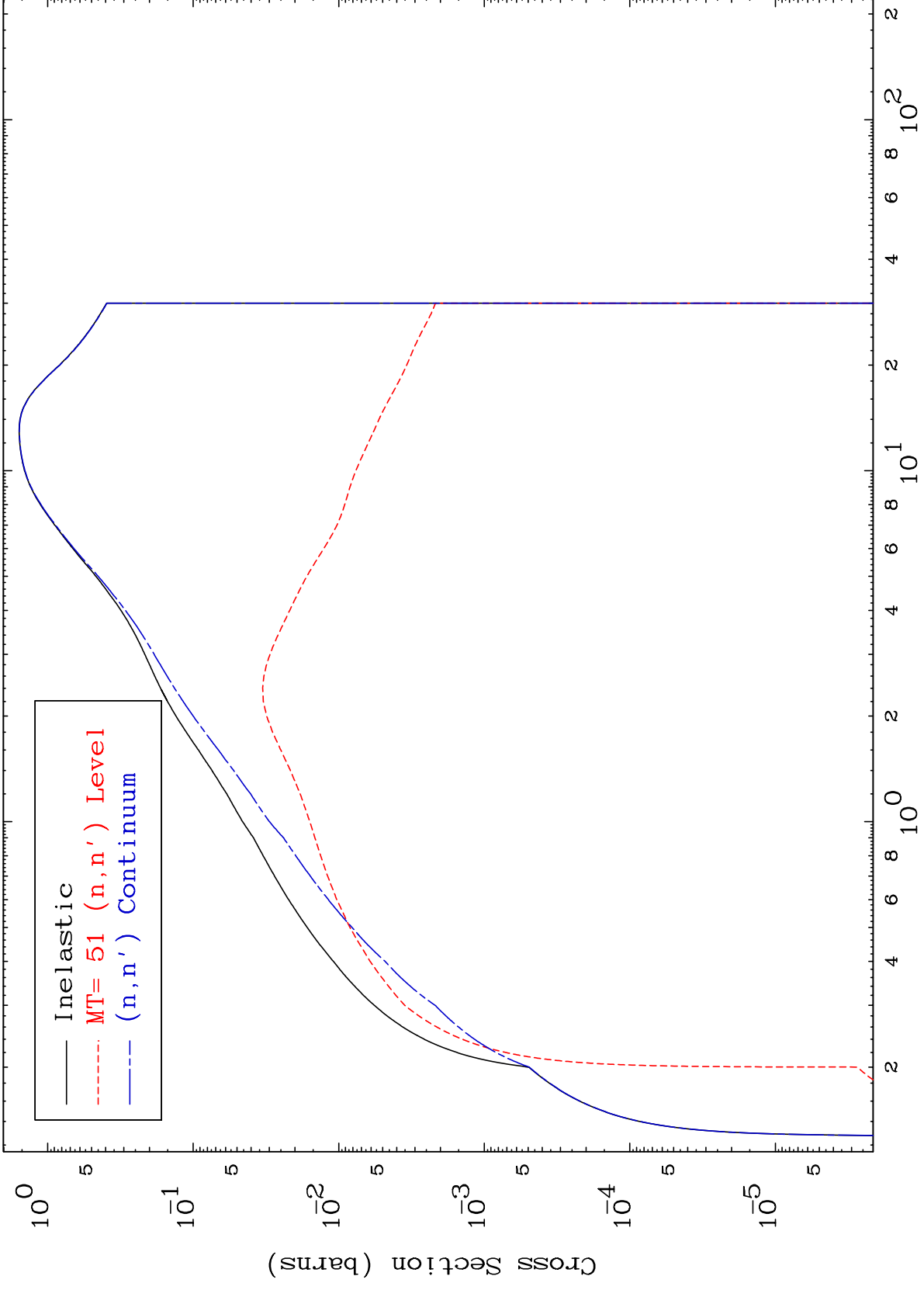


MAT 8704

(n,n') Level

87-Fr-205

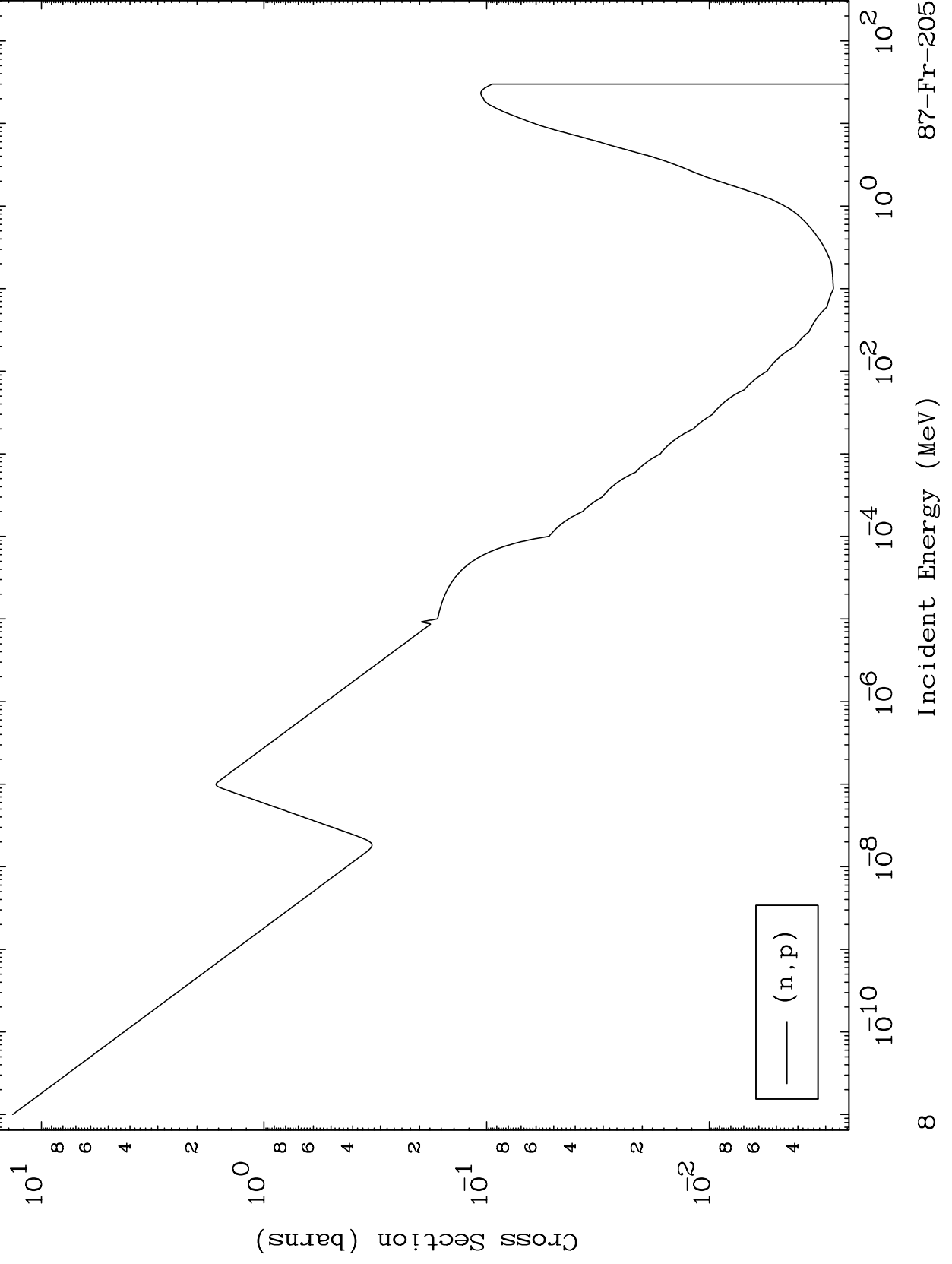
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

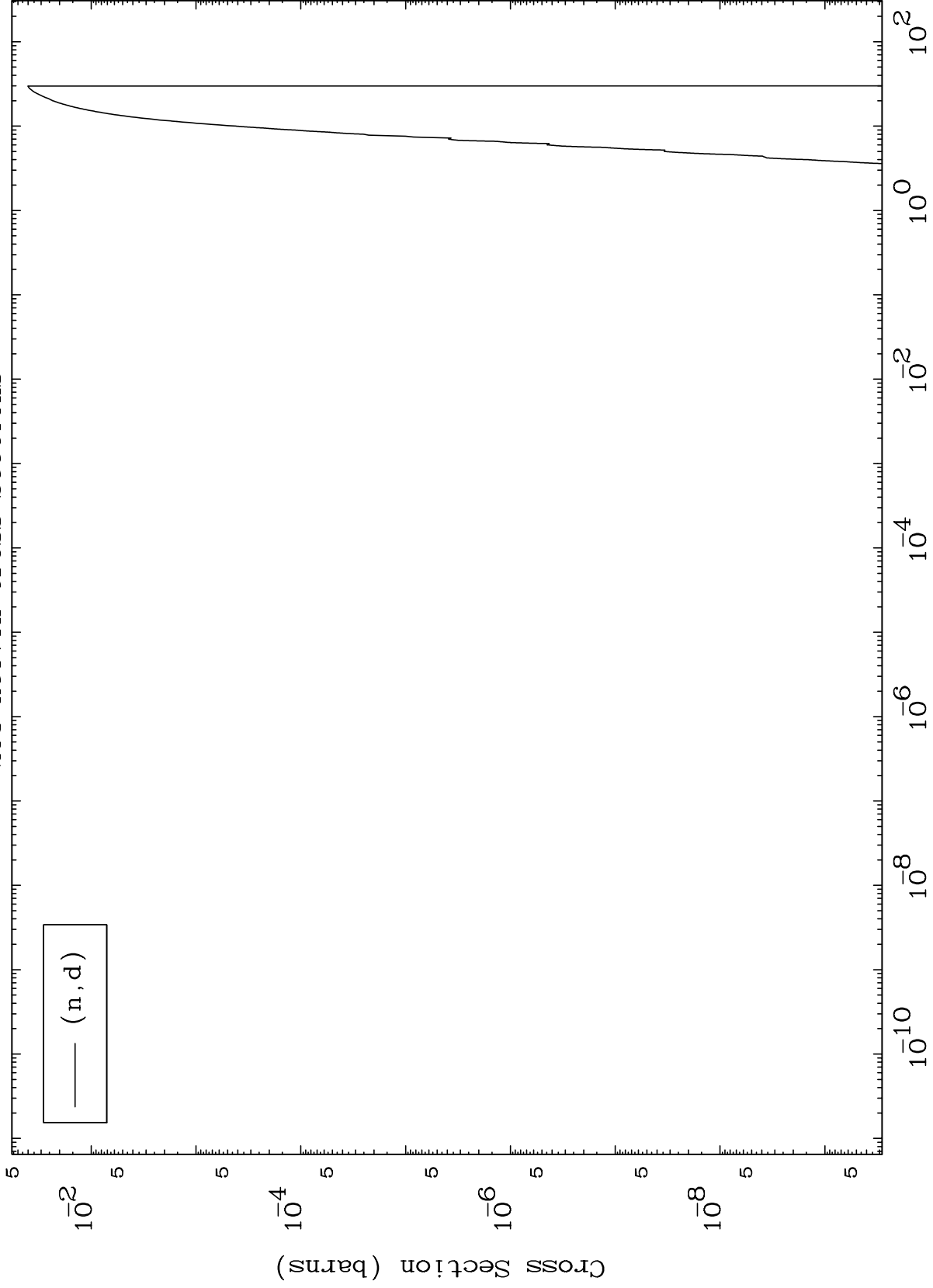
87-Fr-205



MAT 8704

(n,d) Levels
293 Kelvin Cross Sections

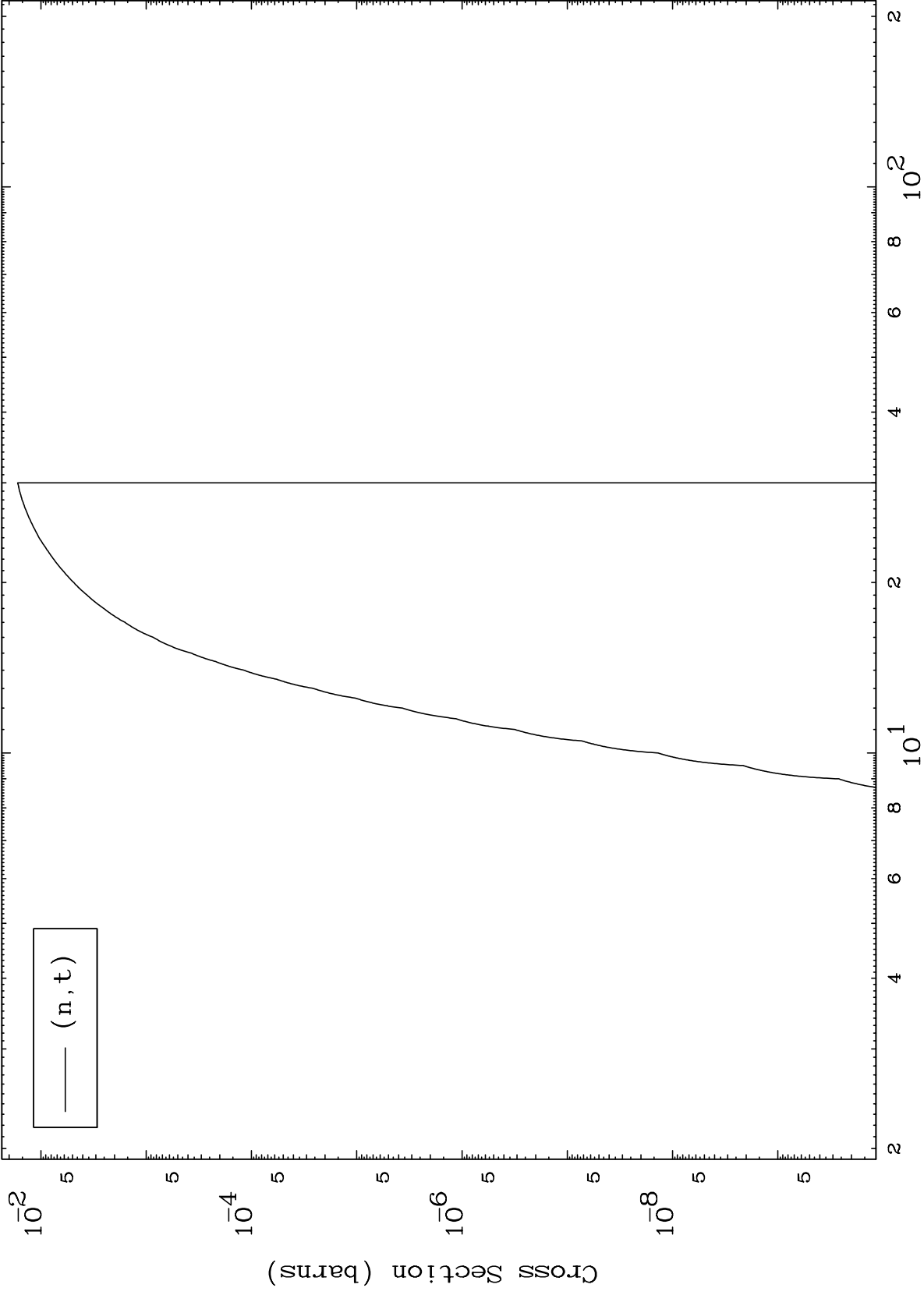
87-Fr-205



MAT 8704

(n,t) Levels
293 Kelvin Cross Sections

87-Fr-205



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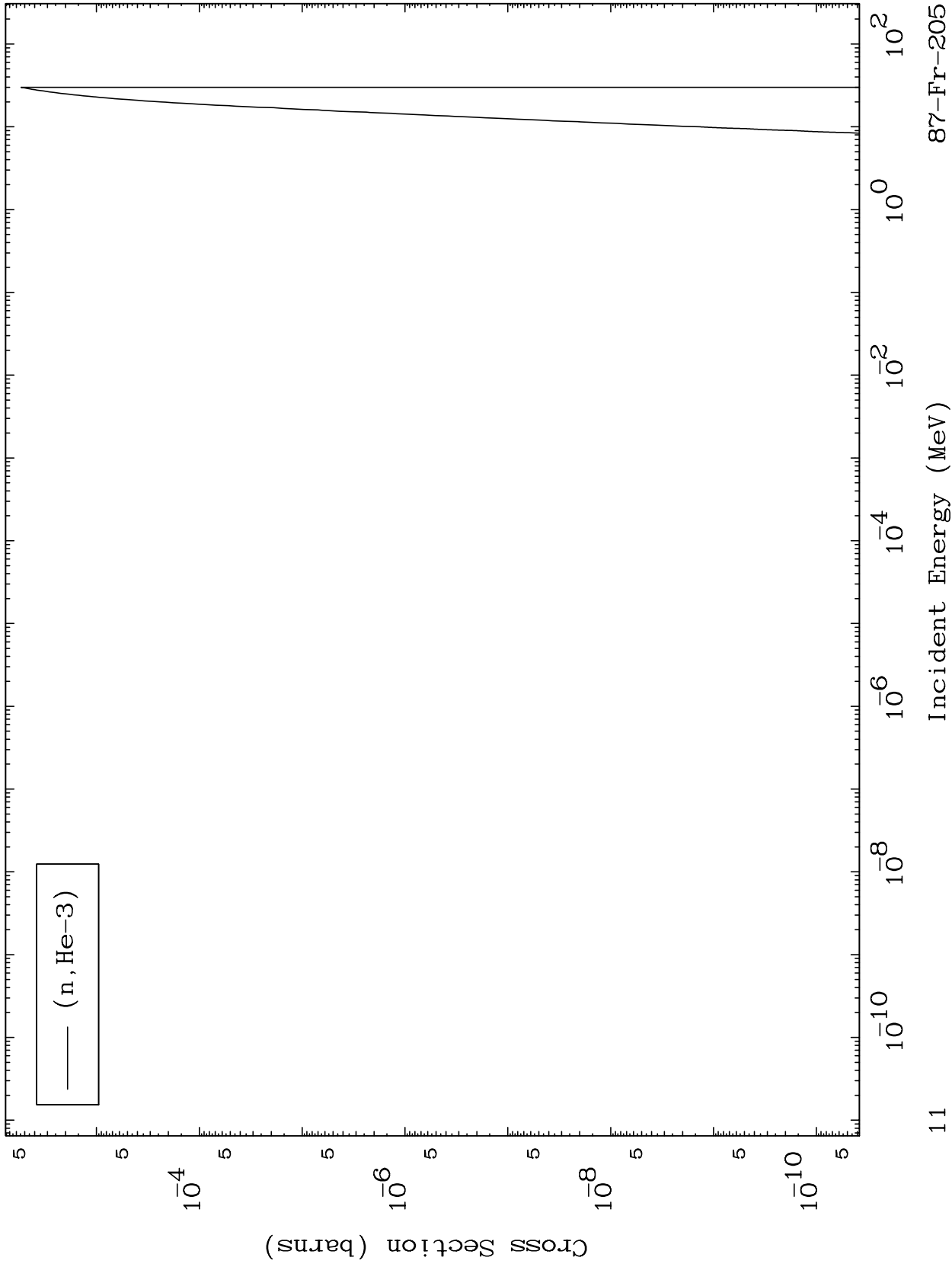
Incident Energy (MeV)

87-Fr-205

MAT 8704

(n,He3) Levels
293 Kelvin Cross Sections

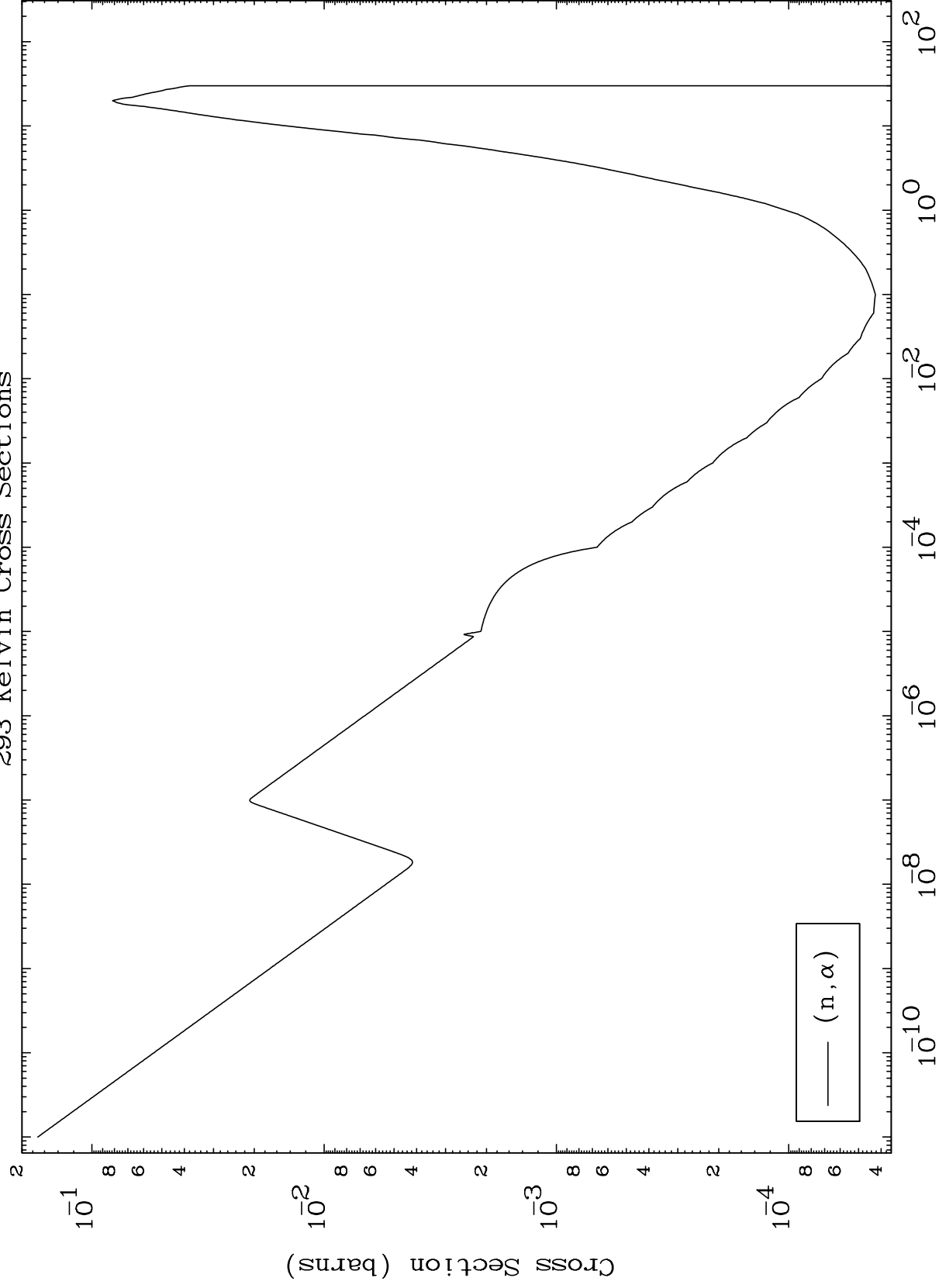
87-Fr-205



MAT 8704

(n,α) Levels
293 Kelvin Cross Sections

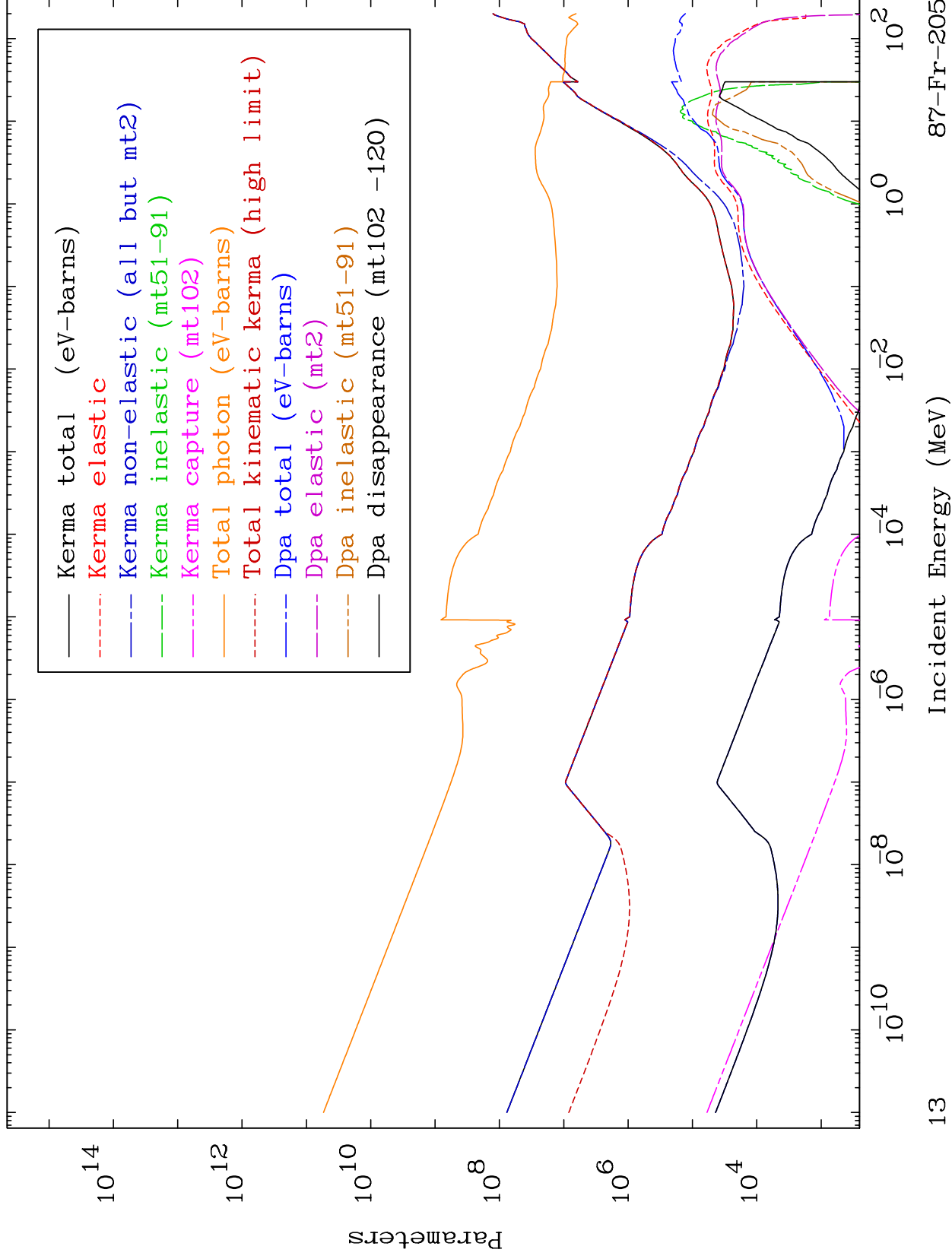
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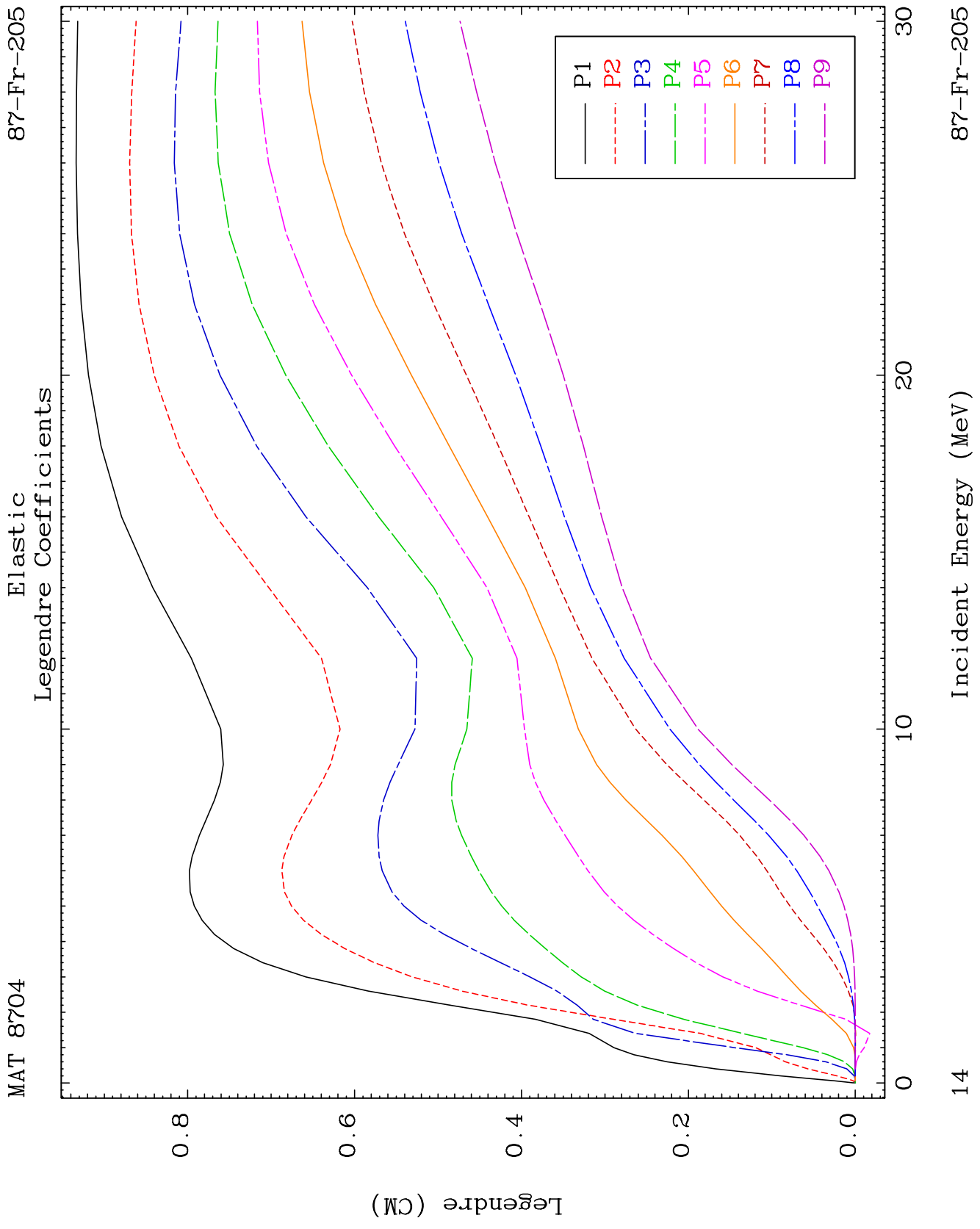


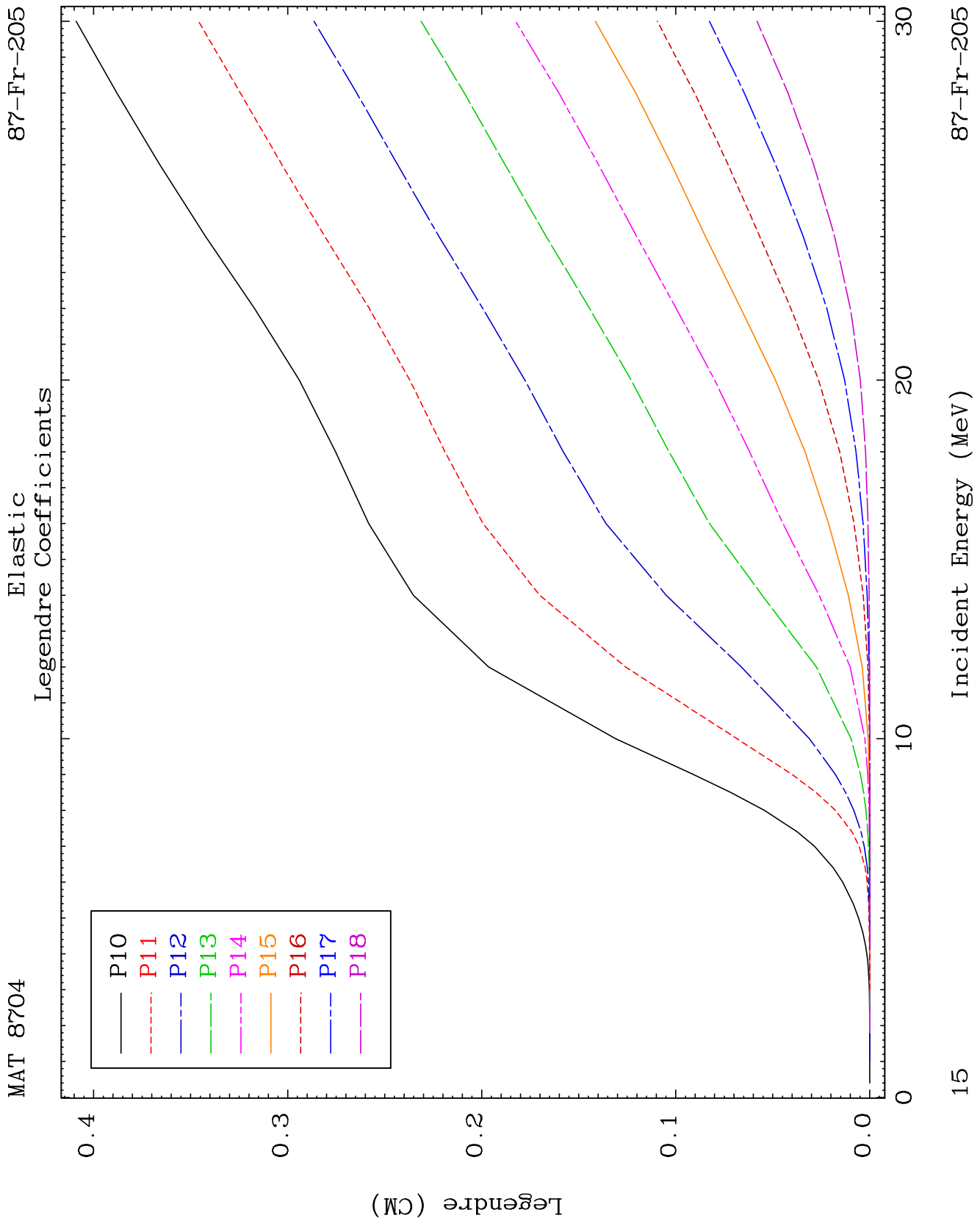
12

Incident Energy (MeV)

87-Fr-205



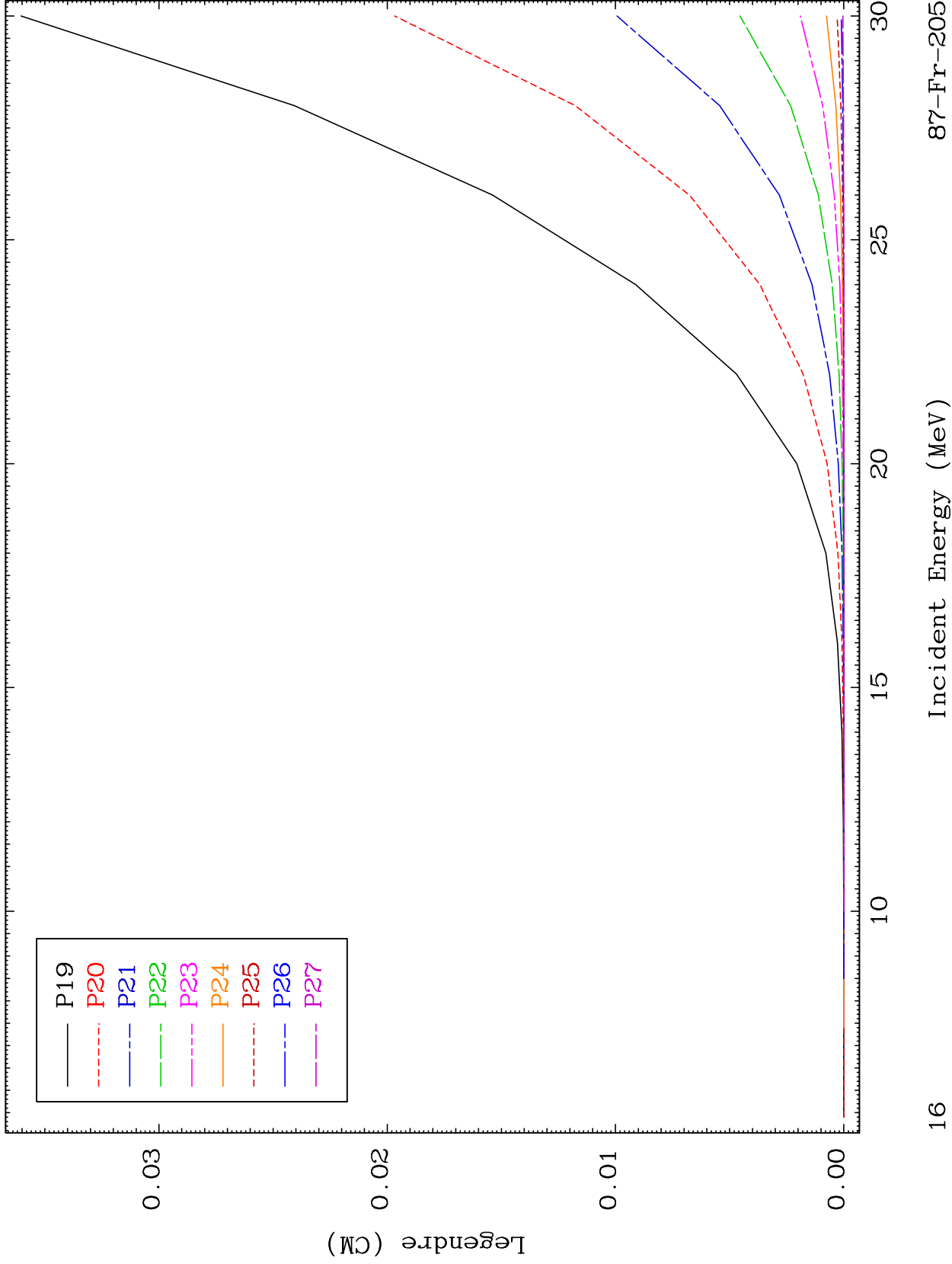




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Elastic Legendre Coefficients

87-Fr-205



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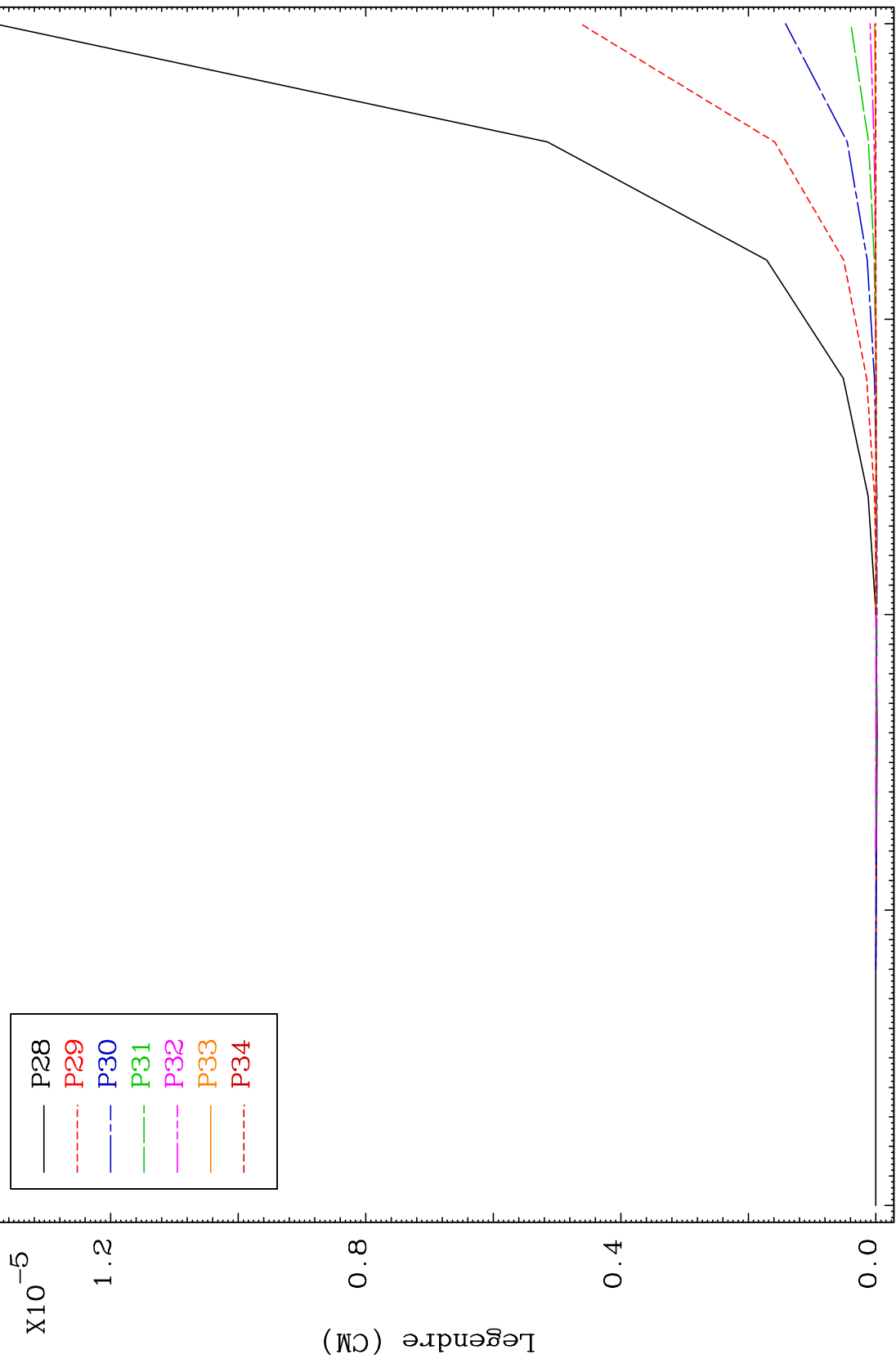
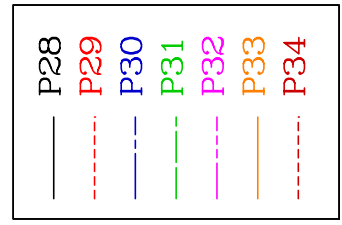
Incident Energy (MeV)

87-Fr-205

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Elastic
Legendre Coefficients

87-Fr-205



87-Fr-205

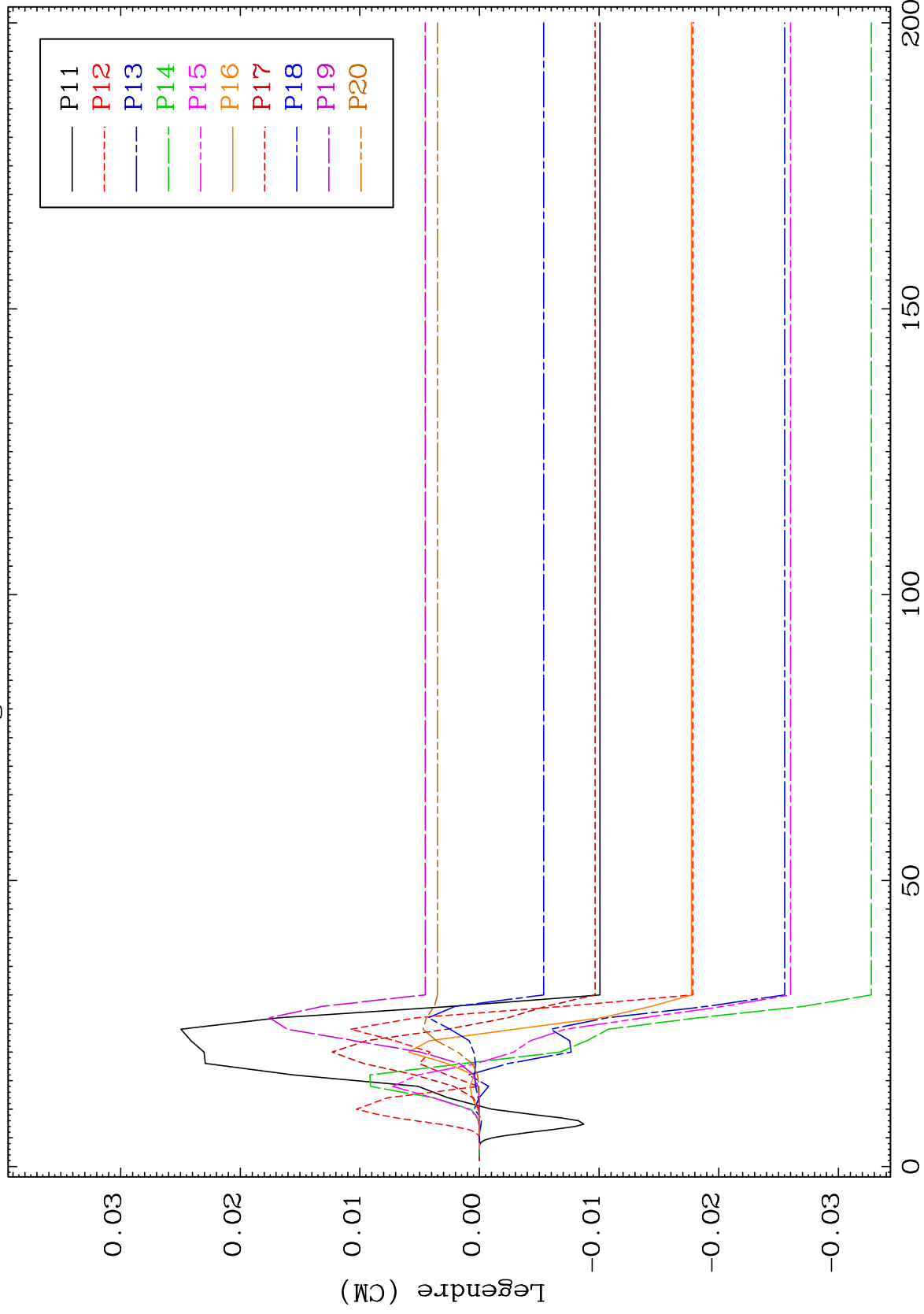
Incident Energy (MeV)

17

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MT= 51 (n,n') Level
Legendre Coefficients

87-Fr-205



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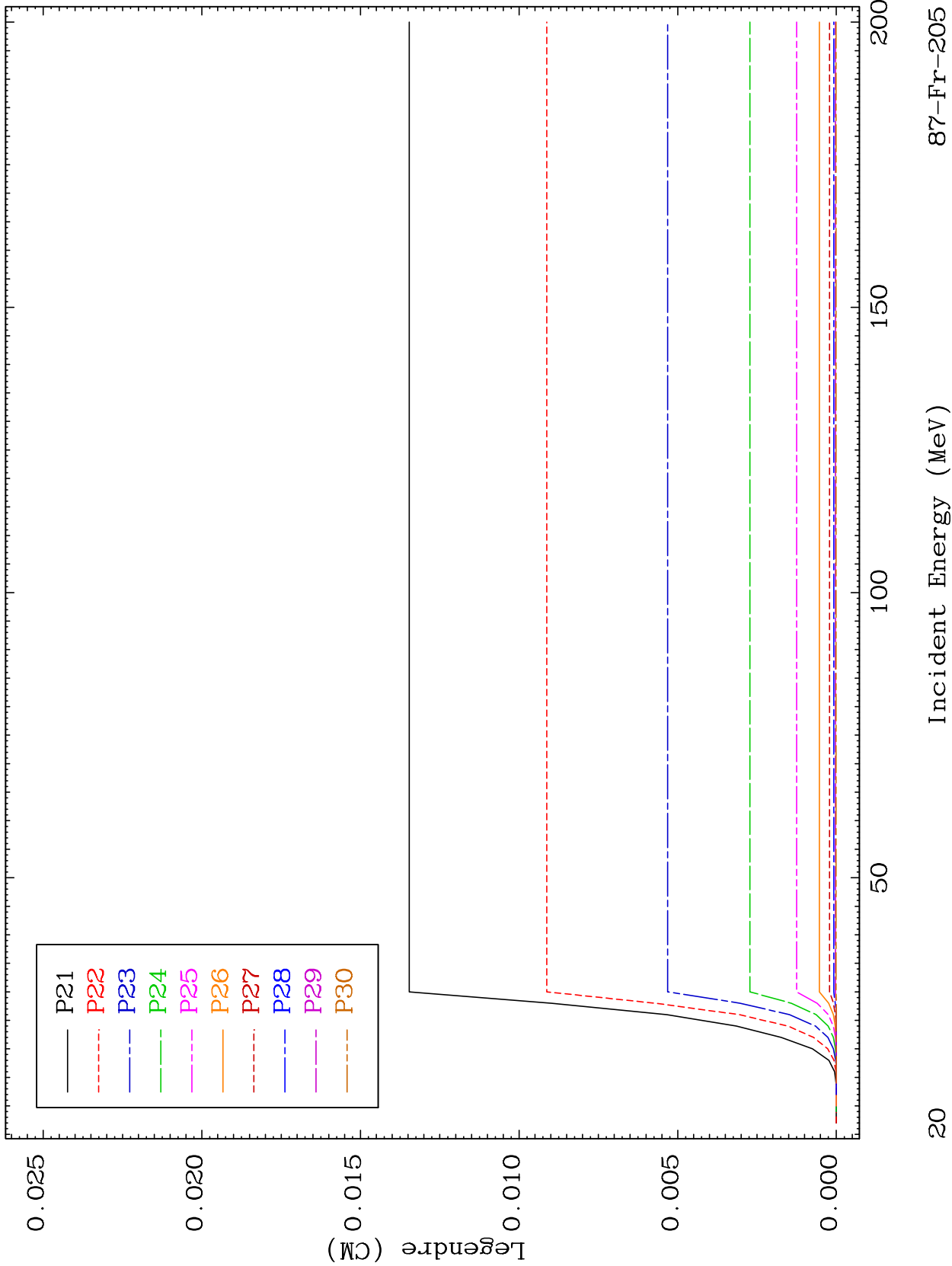
Incident Energy (MeV)

87-Fr-205

MAT 8704

MT= 51 (n,n') Level
Legendre Coefficients

87-Fr-205



87-Fr-205

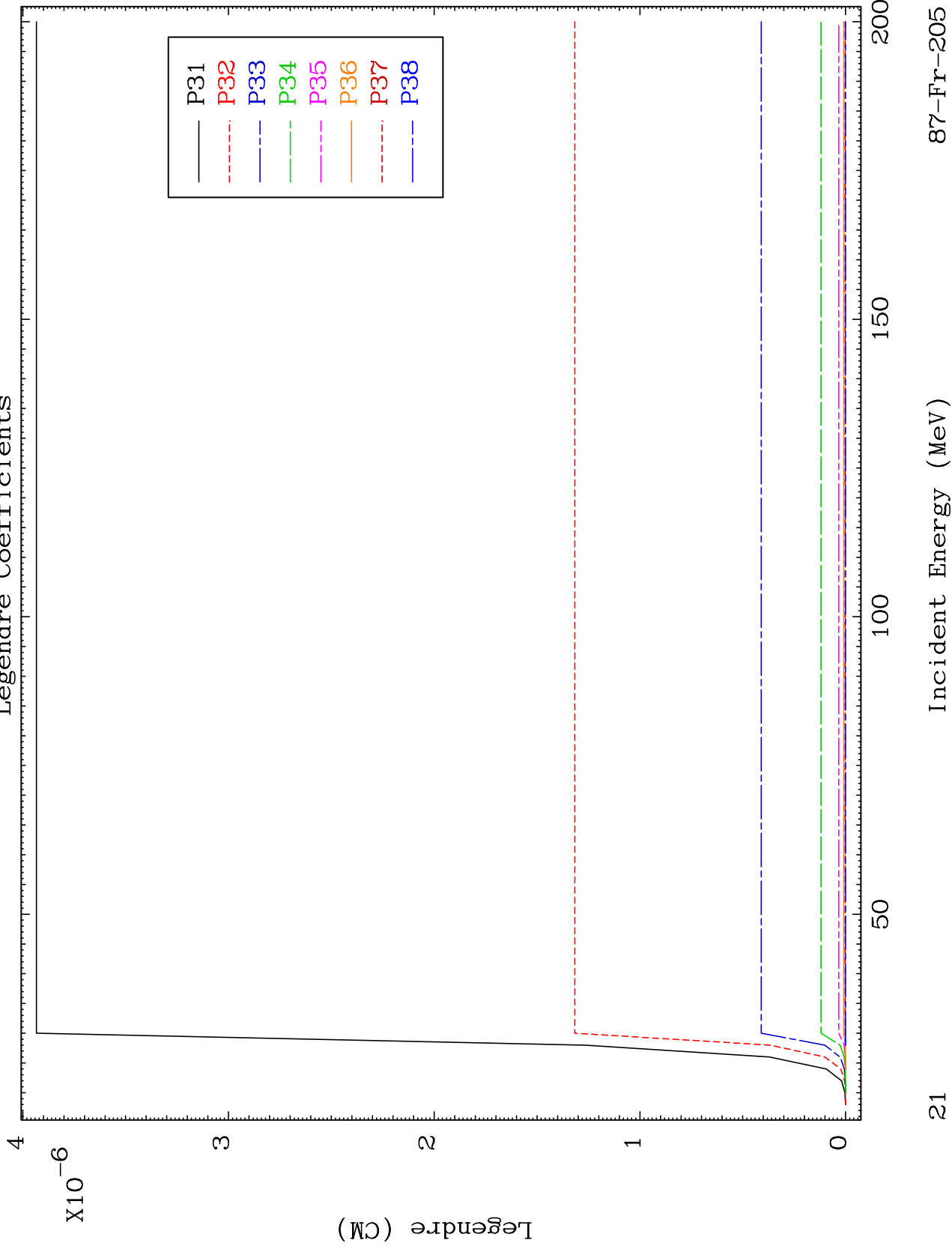
Incident Energy (MeV)

20

MAT 8704

MT= 51 (n,n') Level
Legendre Coefficients

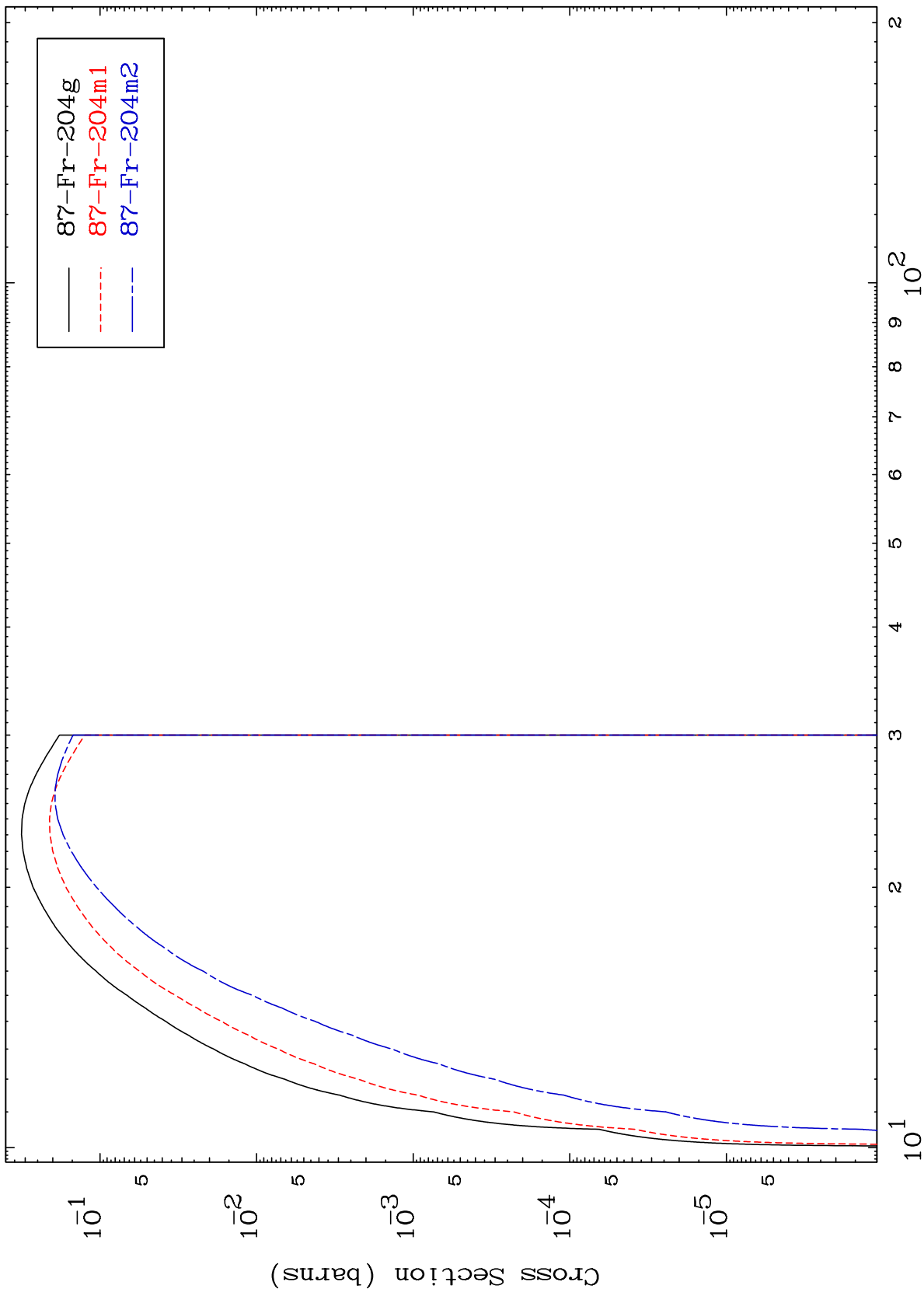
87-Fr-205



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87-Fr-205

(n,2n)
Radionuclide Production Cross Section



87-Fr-205

Incident Energy (MeV)

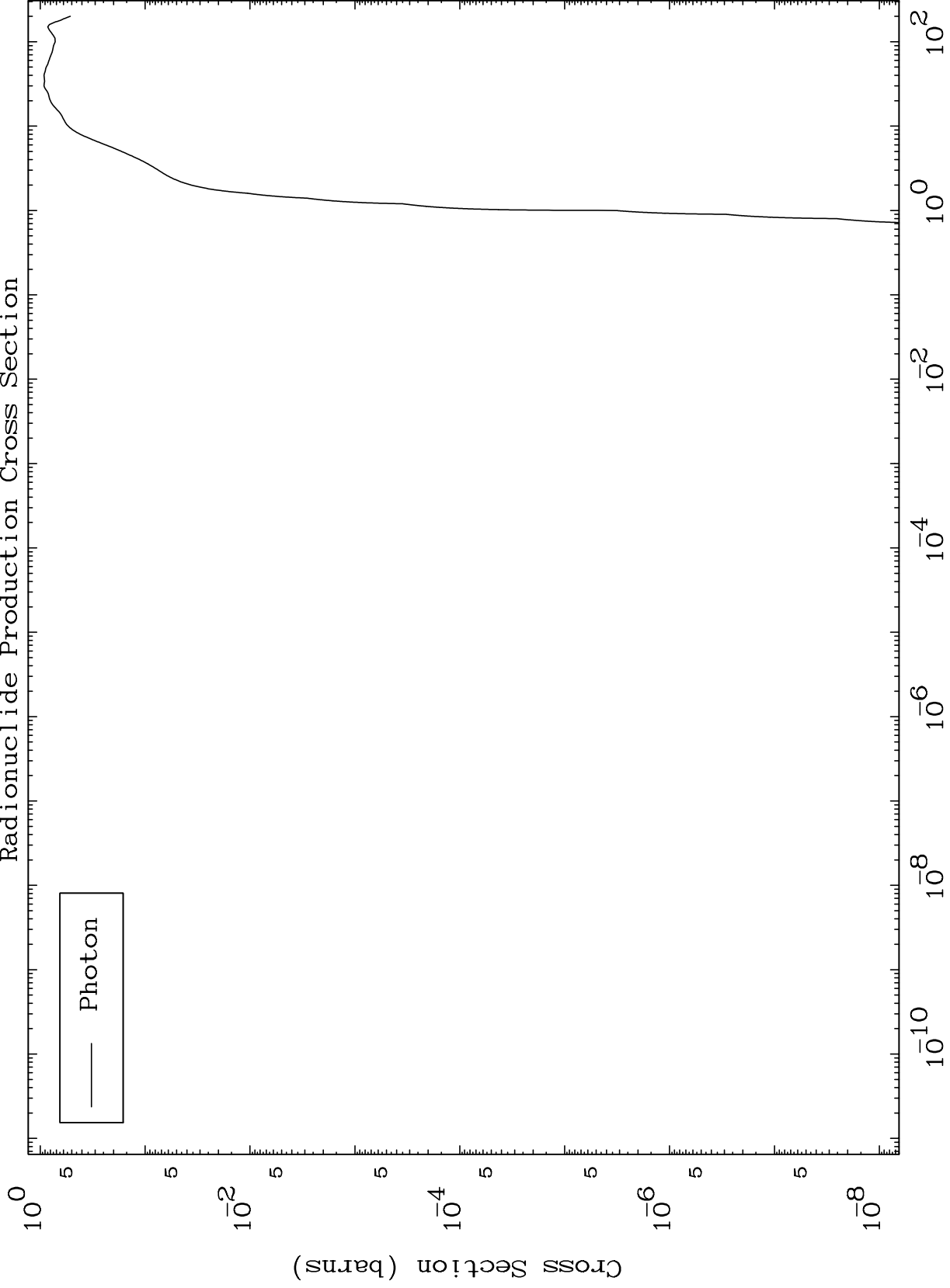
22

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Fission

87-Fr-205

Radionuclide Production Cross Section

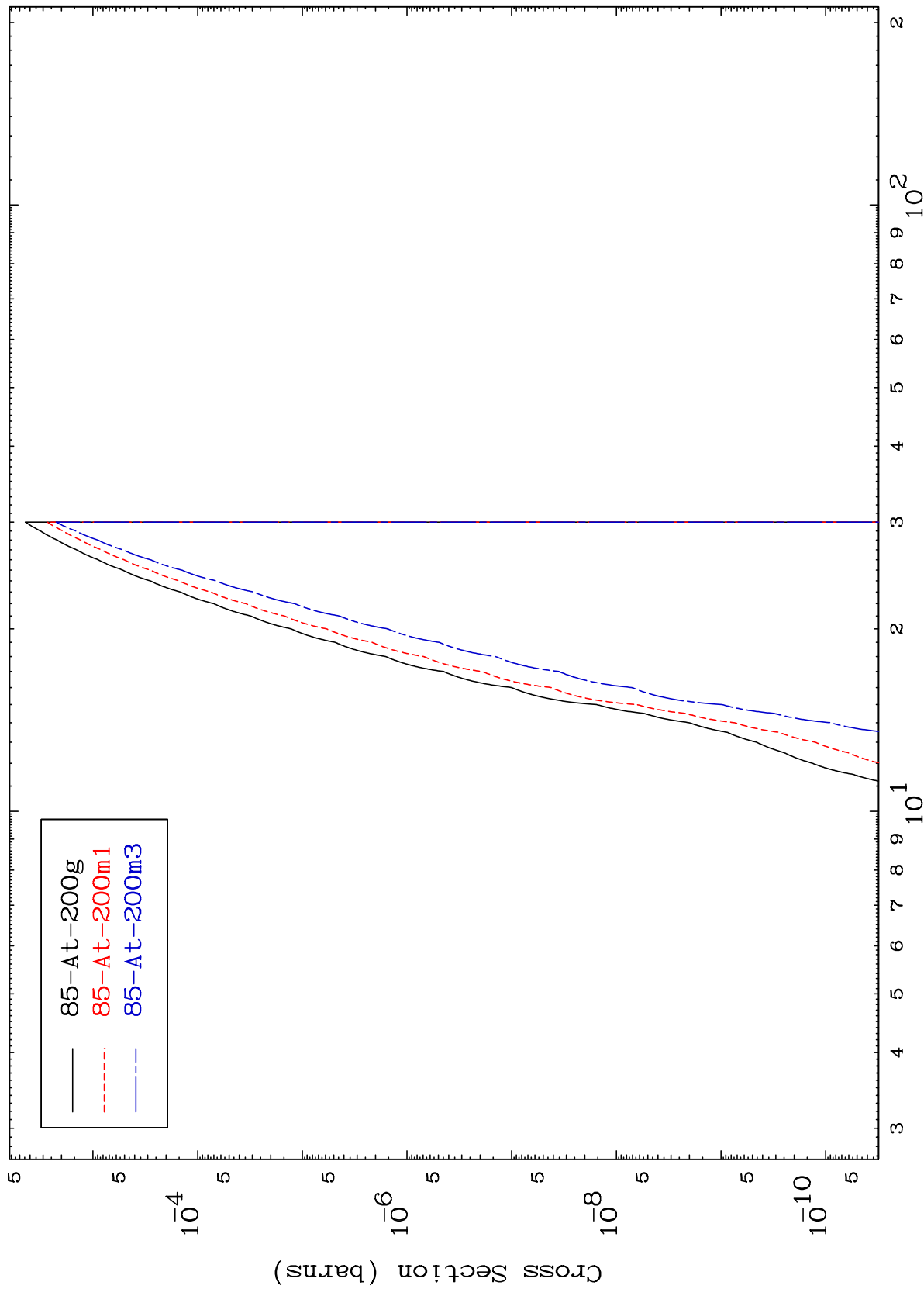


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$(n,2n) \alpha$

87-Fr-205

Radionuclide Production Cross Section



24

Incident Energy (MeV)

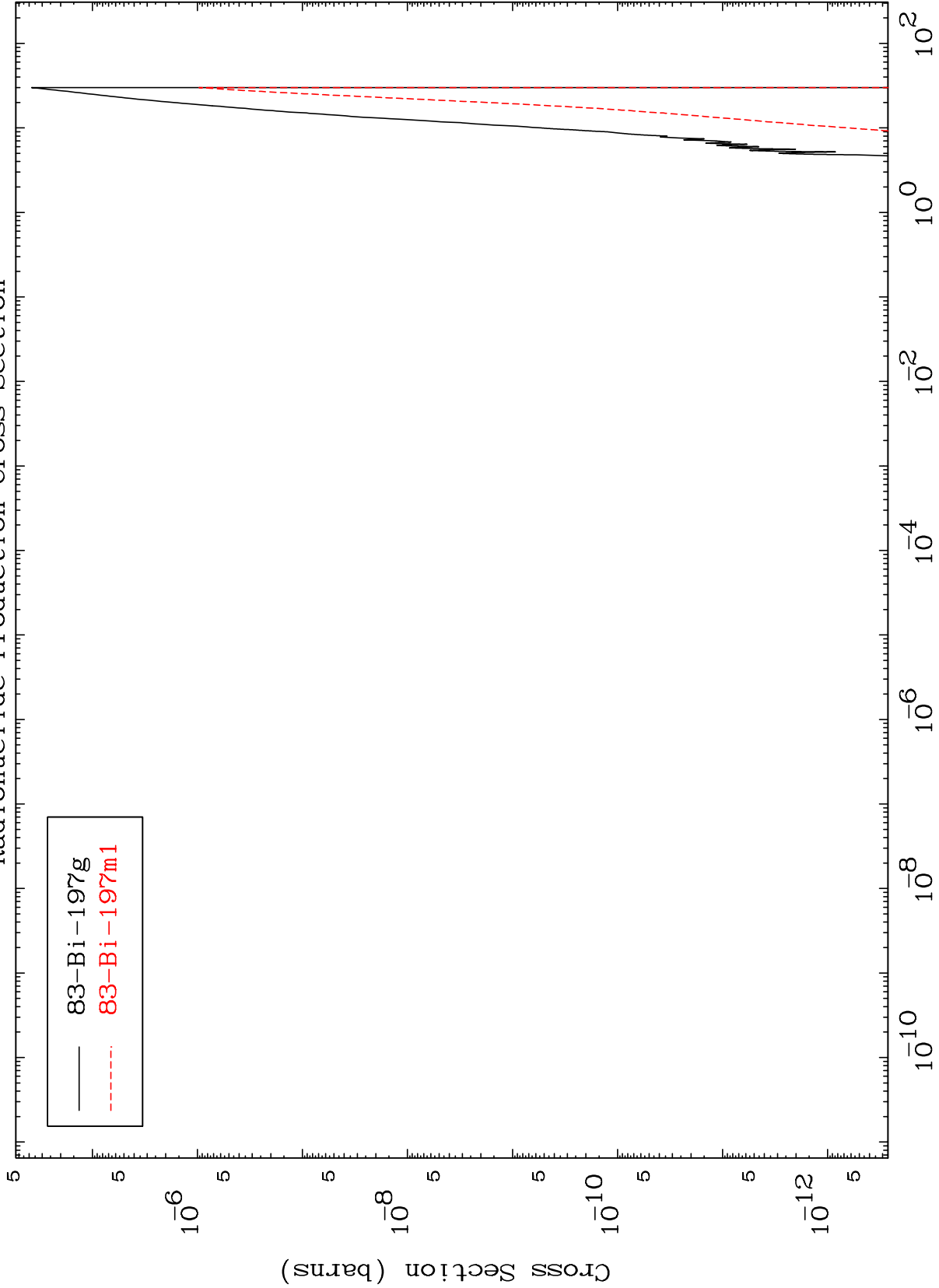
87-Fr-205

MAT 8704

(n,n') 2α

87-Fr-205

Radionuclide Production Cross Section



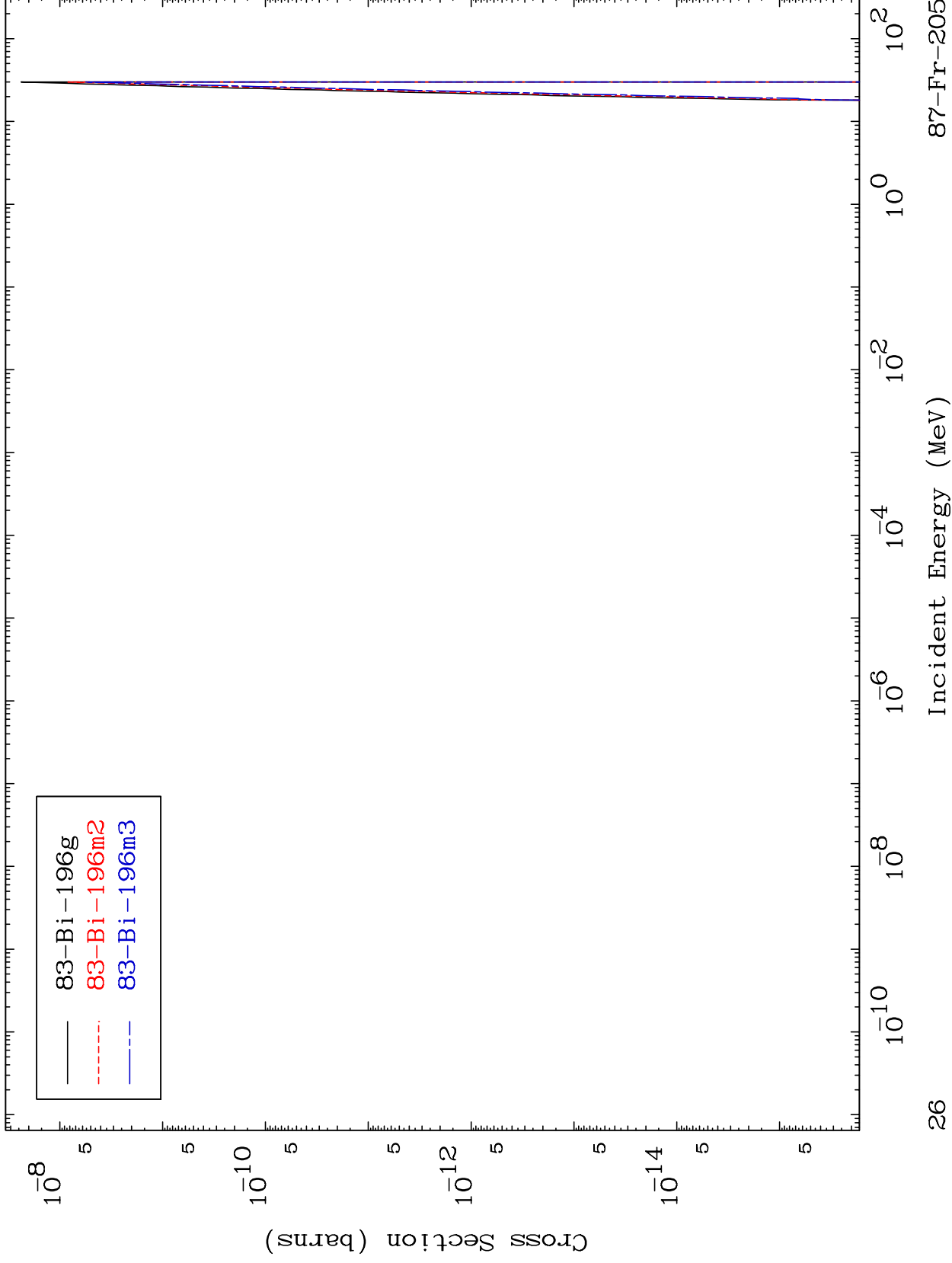
83-Bi-197g
83-Bi-197m1

MAT 8704

(n,2n) 2 α

87-Fr-205

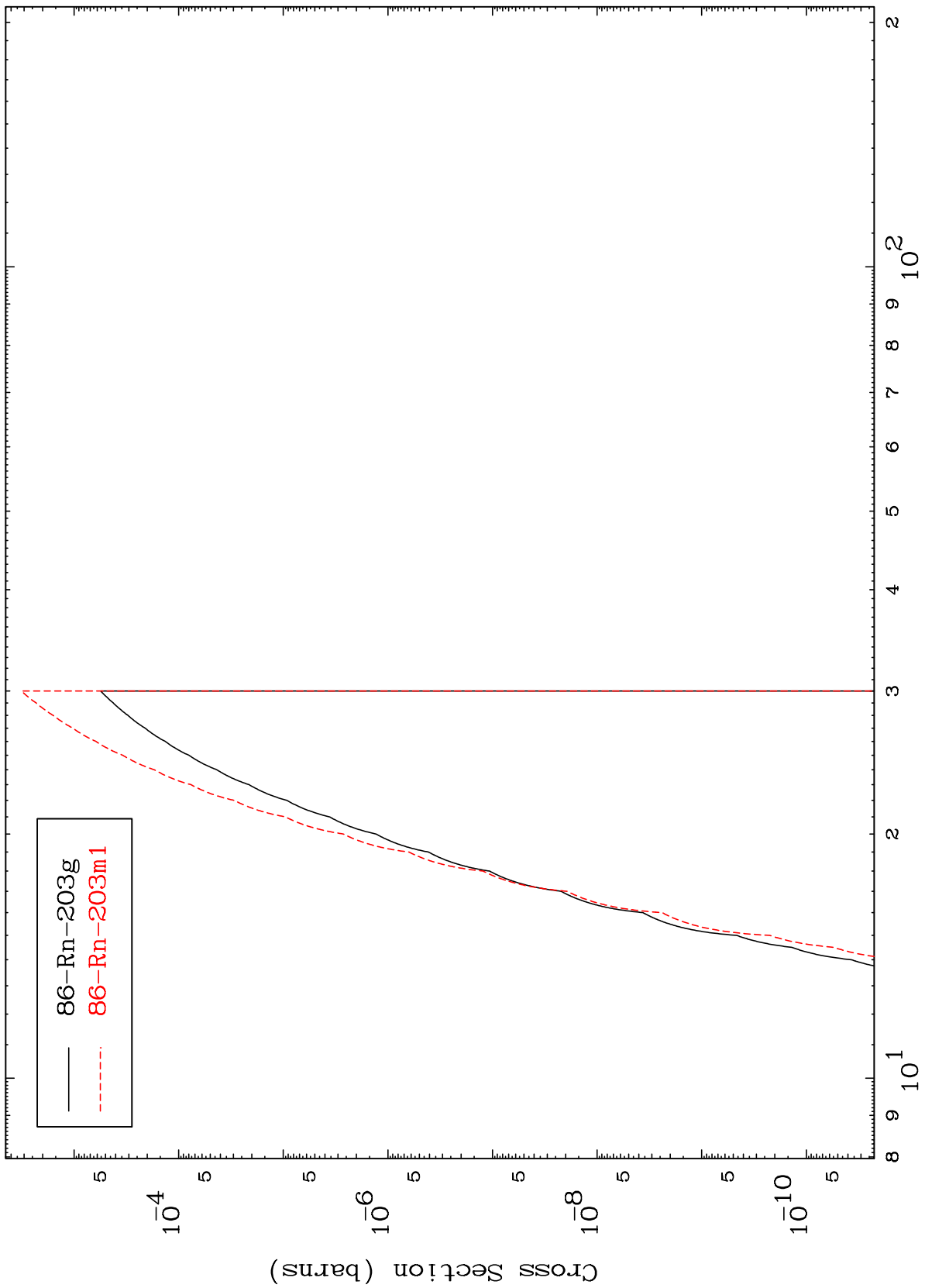
Radionuclide Production Cross Section



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87-Fr-205

(n,n') d
Radionuclide Production Cross Section



27

Incident Energy (MeV)

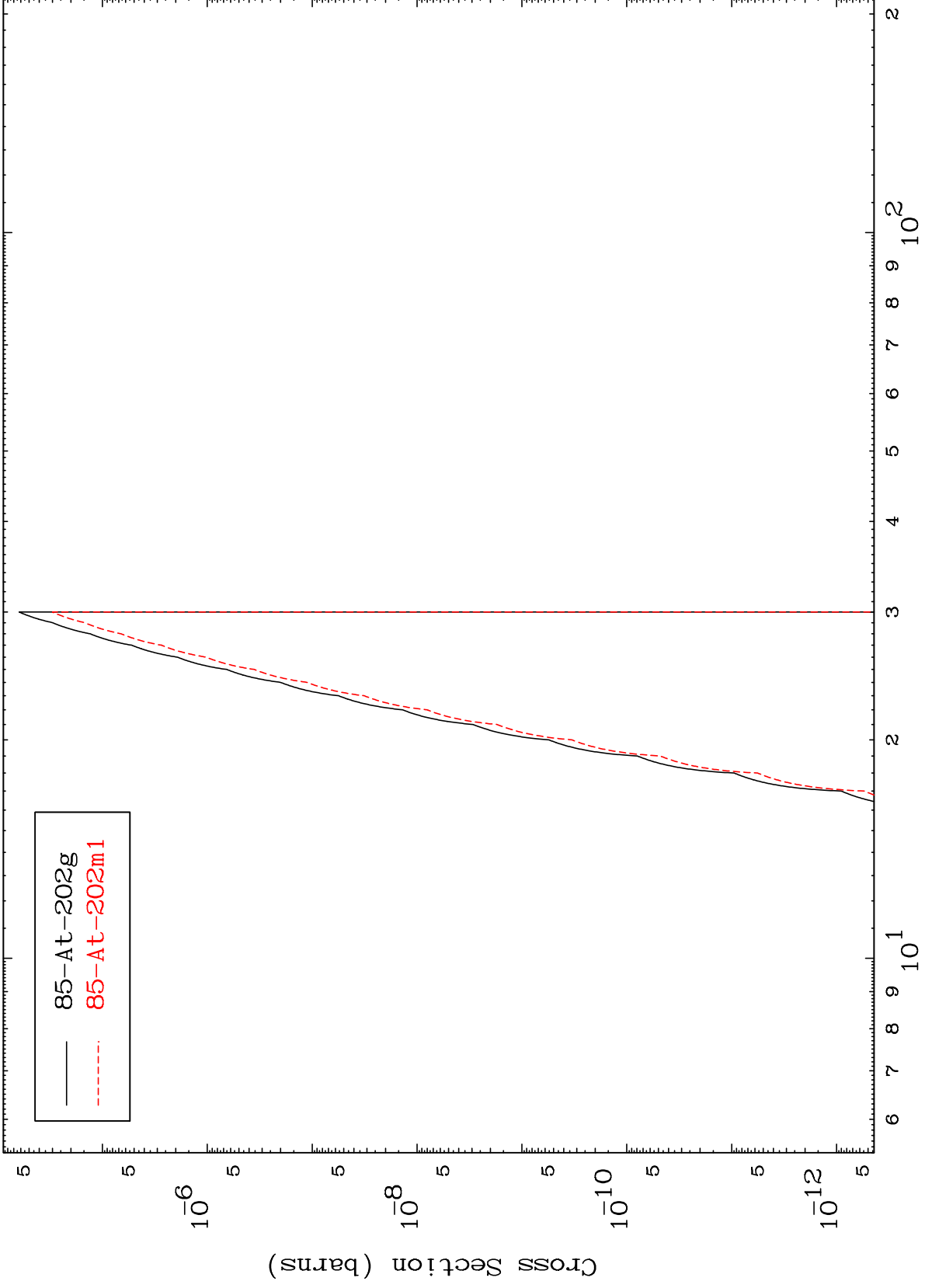
87-Fr-205

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(n,n') He-3

87-Fr-205

Radionuclide Production Cross Section

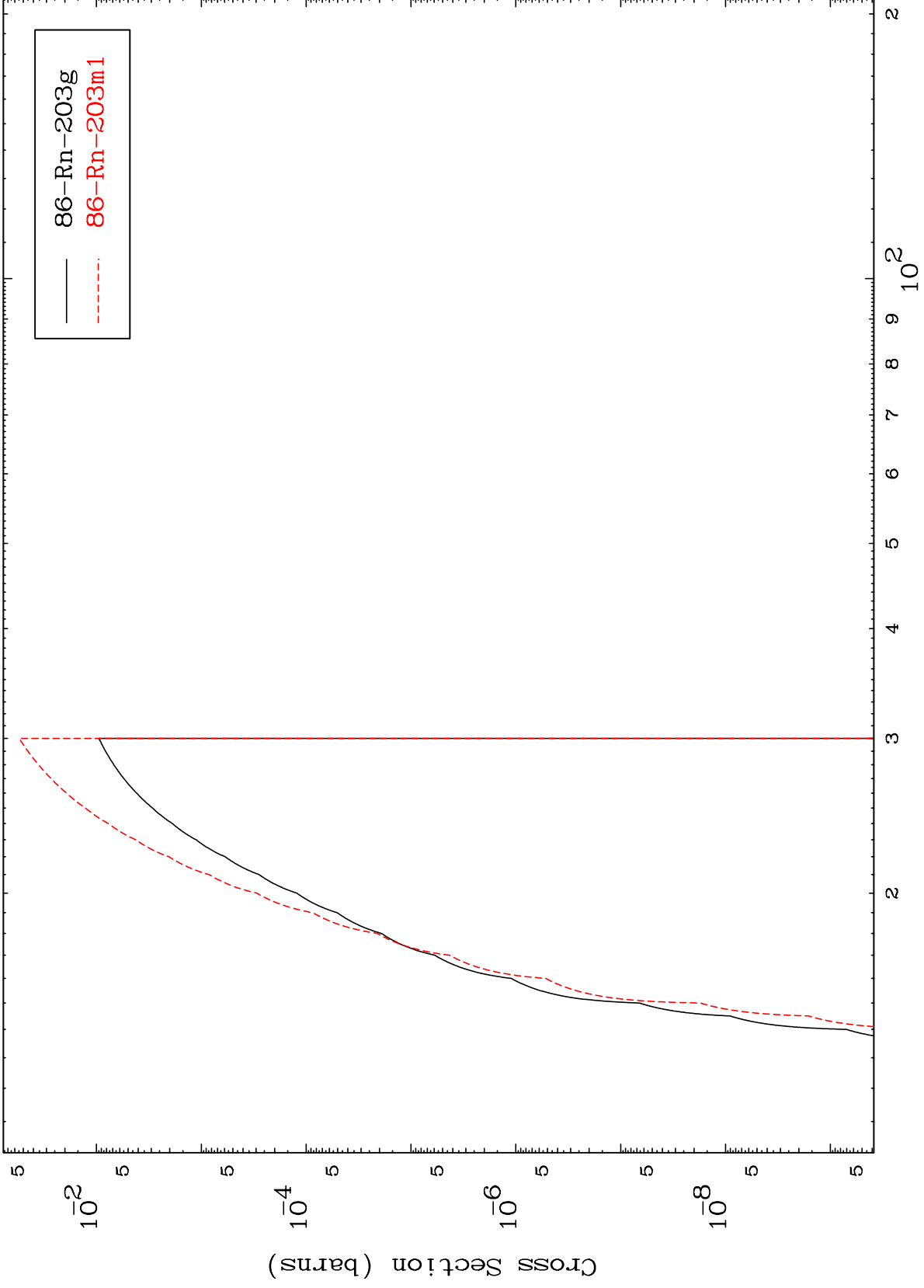


28

Incident Energy (MeV)

87-Fr-205

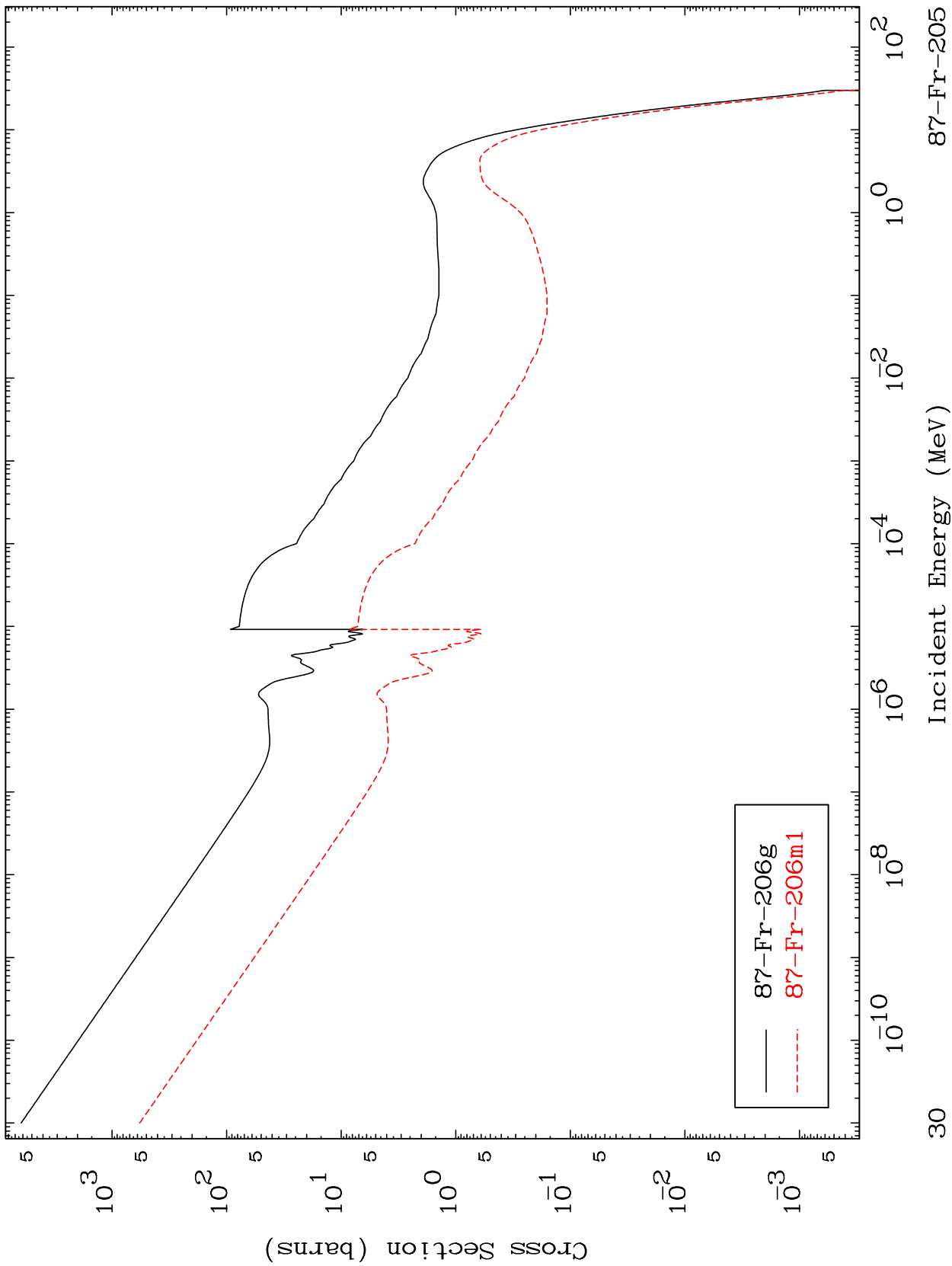
Radionuclide Production Cross Section



MAT 8704

87-Fr-205

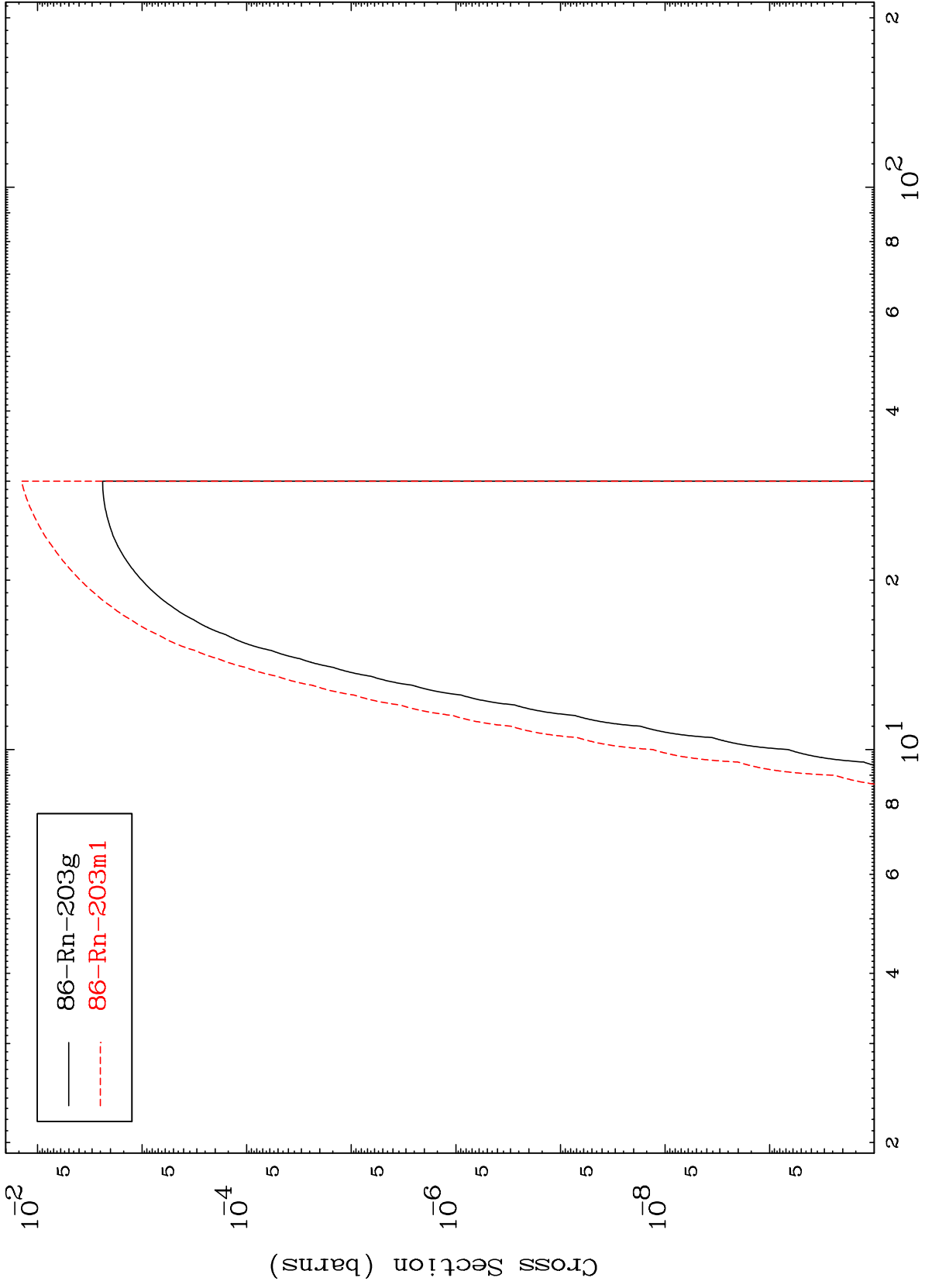
(n, γ)
Radionuclide Production Cross Section



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87-Fr-205

Radionuclide Production Cross Section



86-Rn-203g
86-Rn-203m1

Incident Energy (MeV)

87-Fr-205

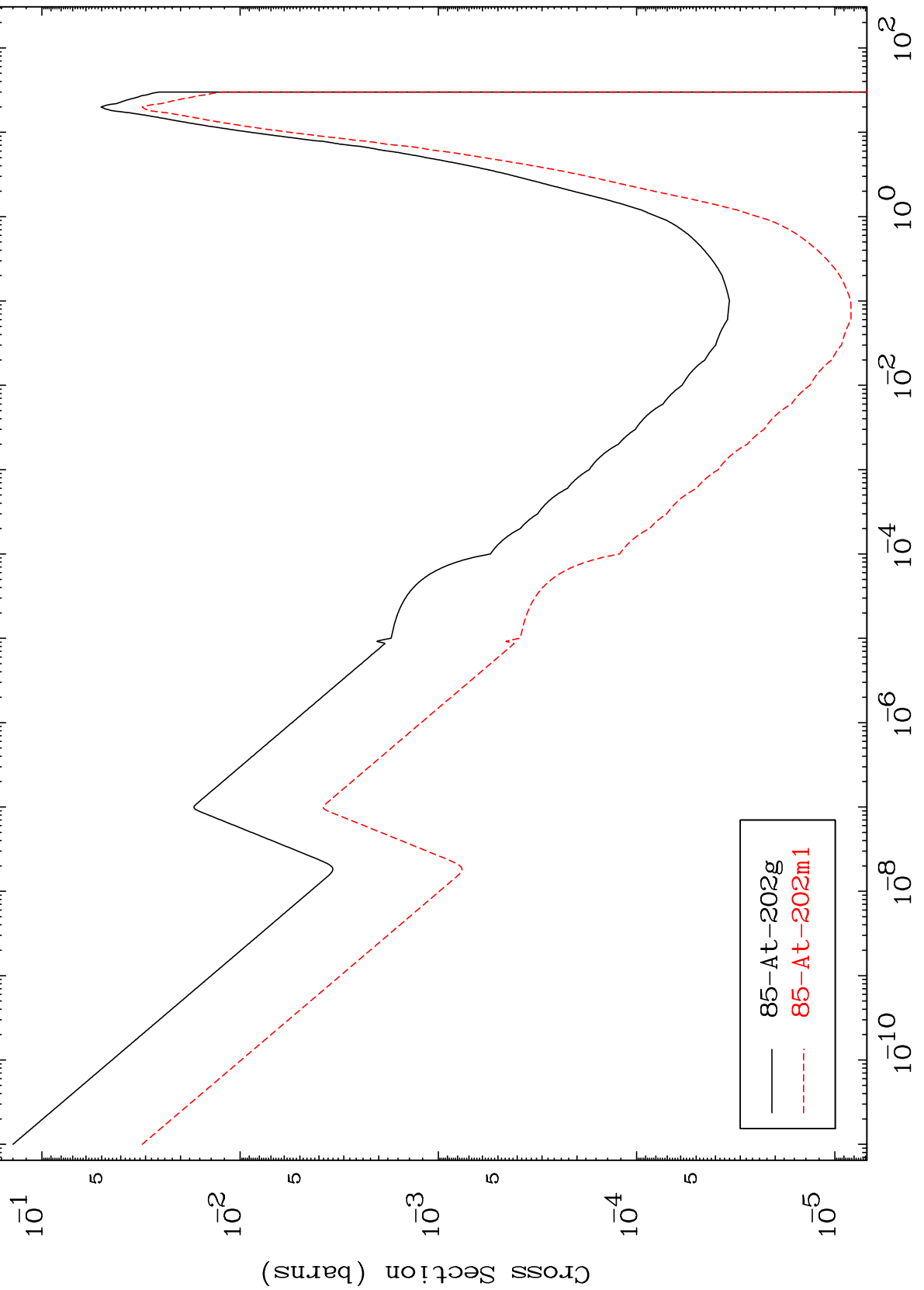
31

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(n, α)

87-Fr-205

Radionuclide Production Cross Section



32

Incident Energy (MeV)

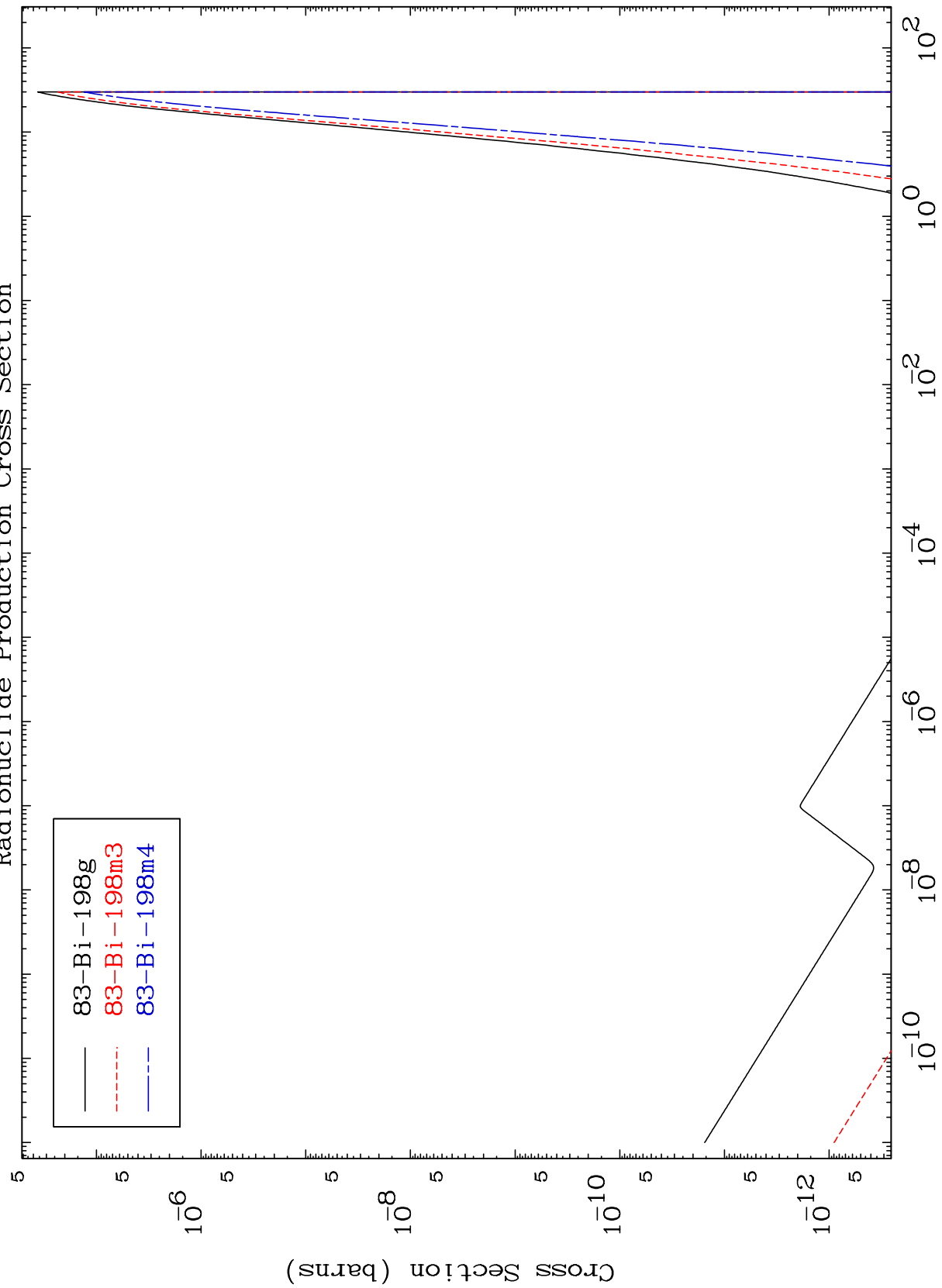
87-Fr-205

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87-Fr-205

87-Fr-205

Radionuclide Production Cross Section
(n,2α)



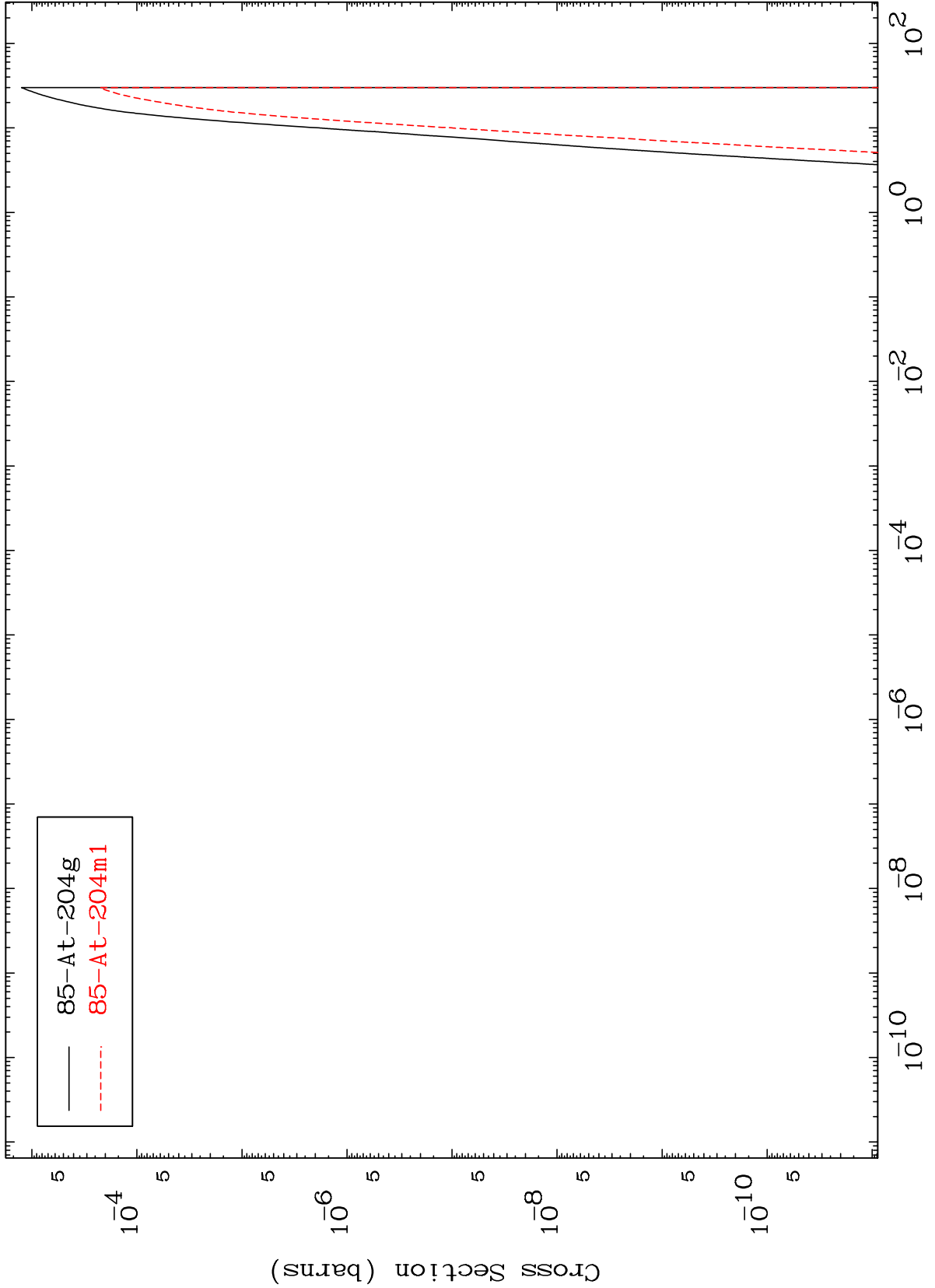
33

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(n,2p)

87-Fr-205

Radionuclide Production Cross Section

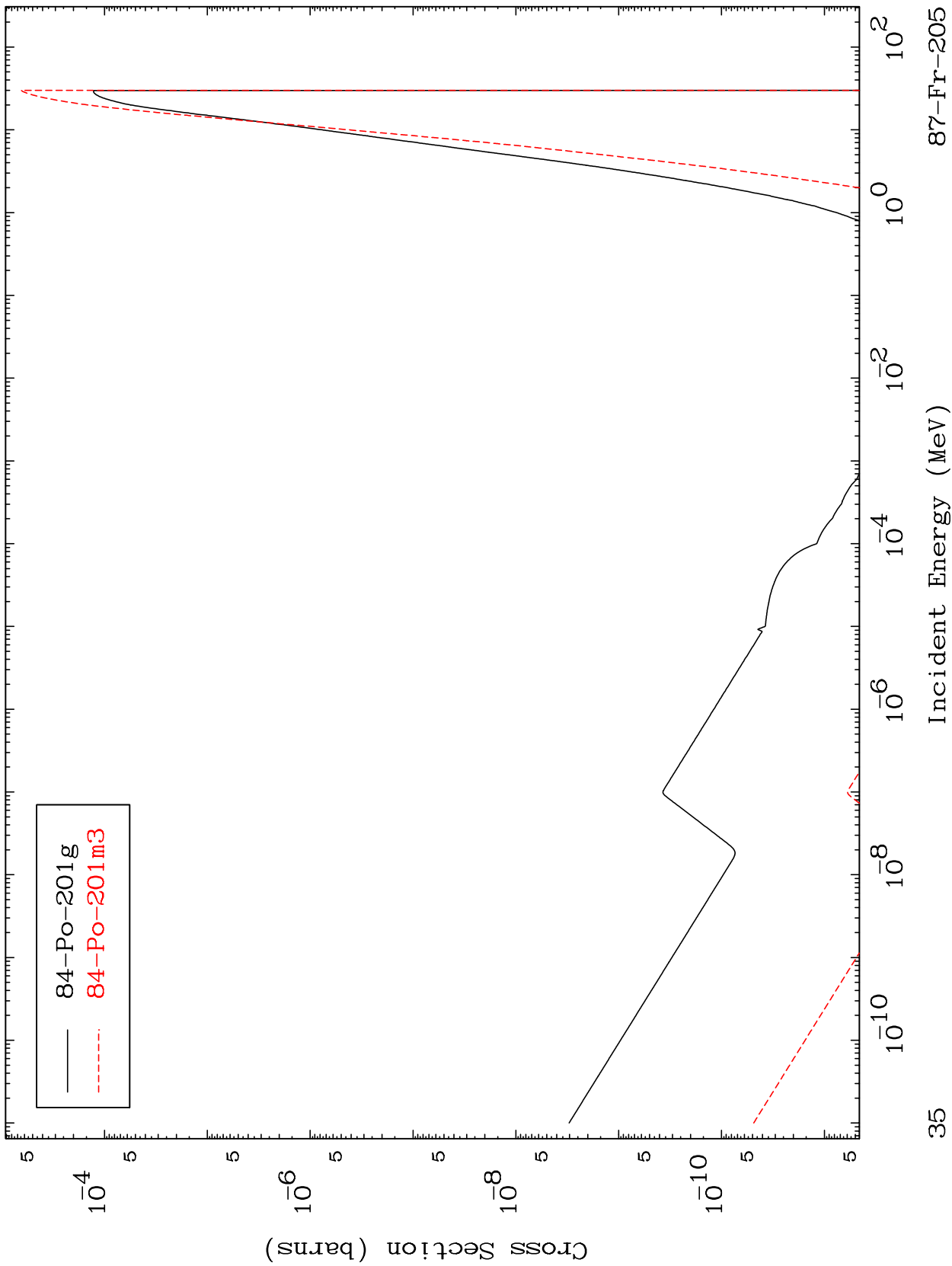


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(n,p) α

87-Fr-205

Radionuclide Production Cross Section

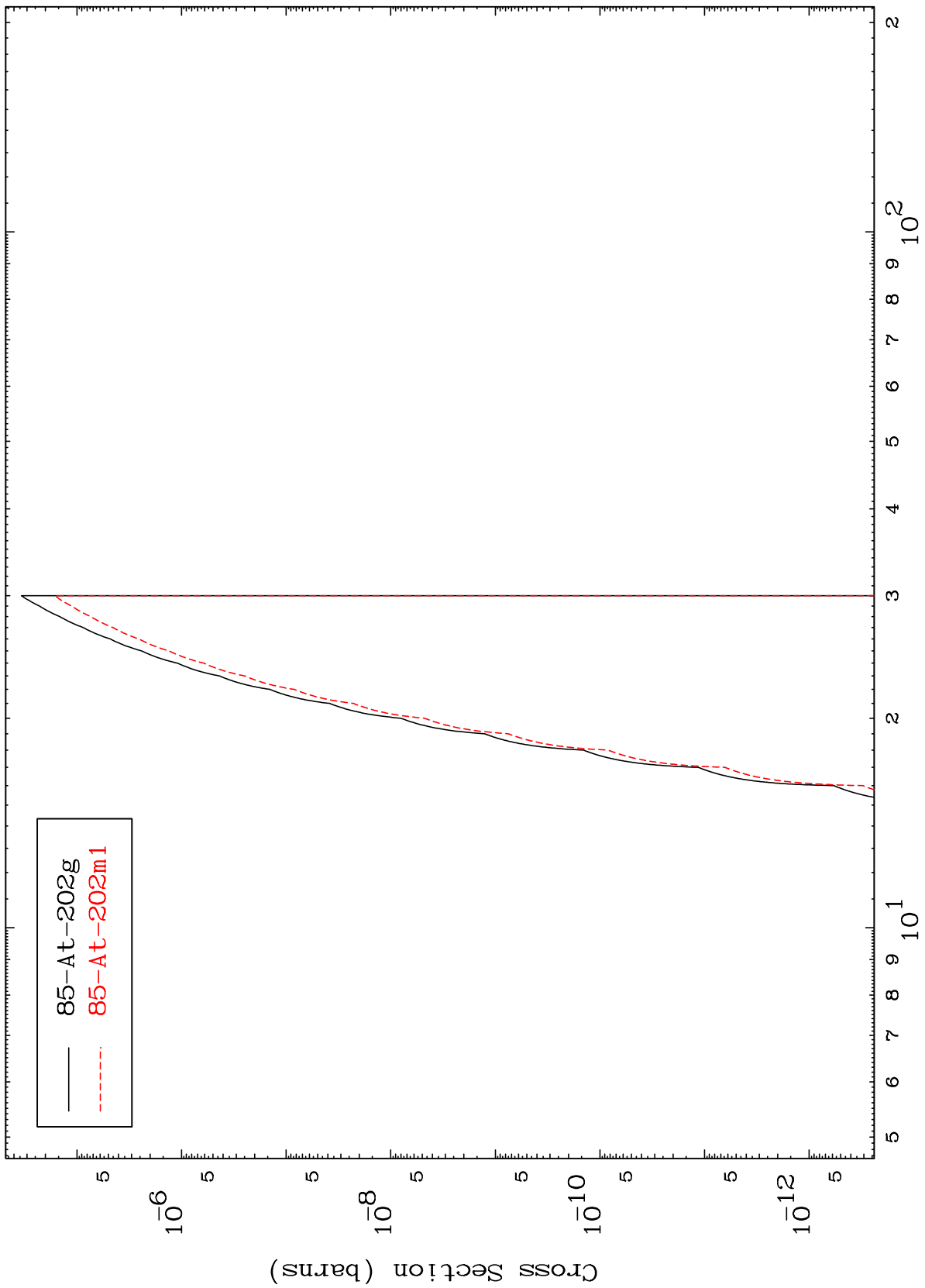


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(n,p) t

87-Fr-205

Radionuclide Production Cross Section



36

Incident Energy (MeV)

87-Fr-205