

Program EVALPLOT
(Version 2017-1)

by

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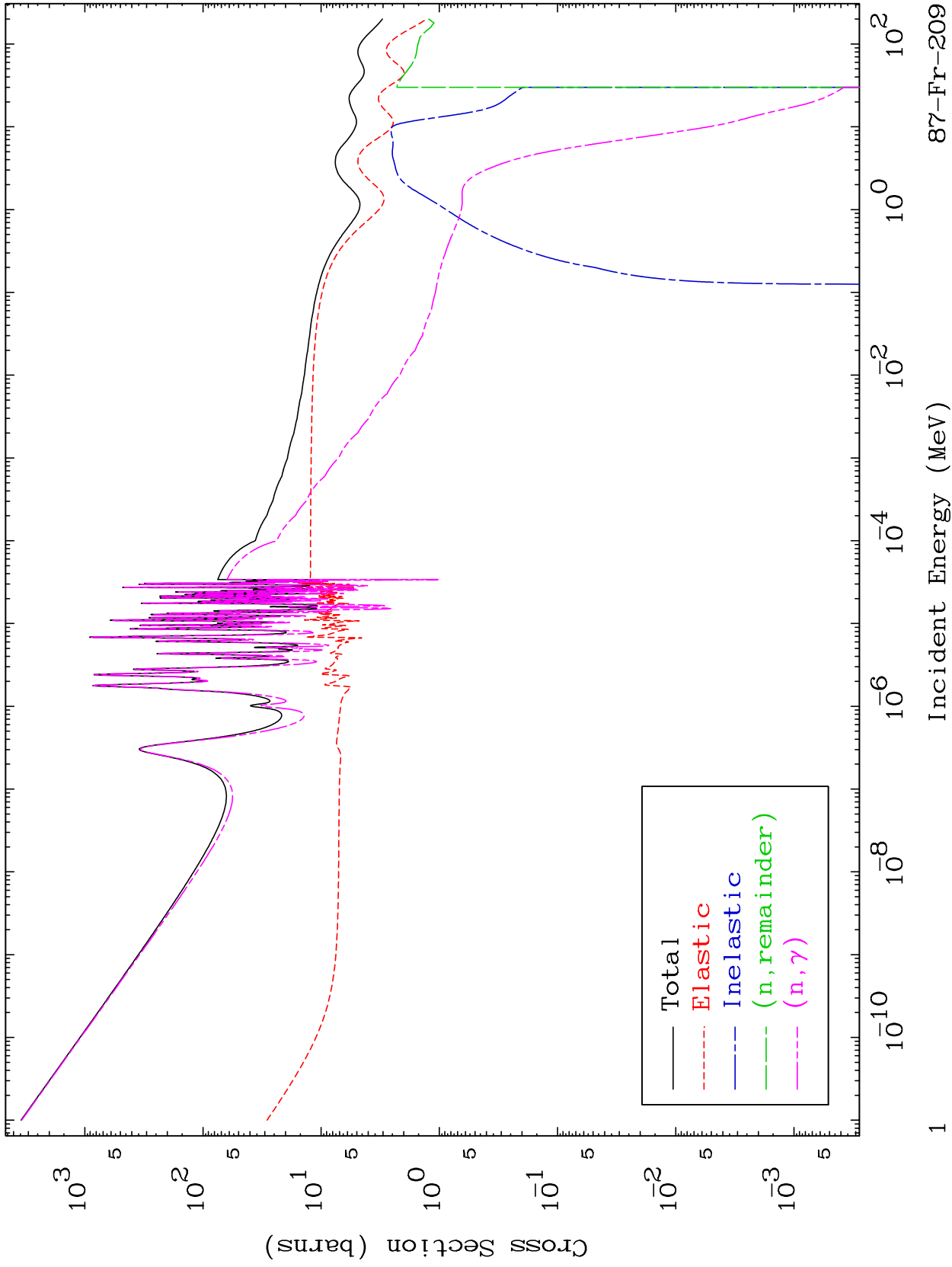
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Press Mouse Button to Start

MAT 8716

Major
293 Kelvin Cross Sections

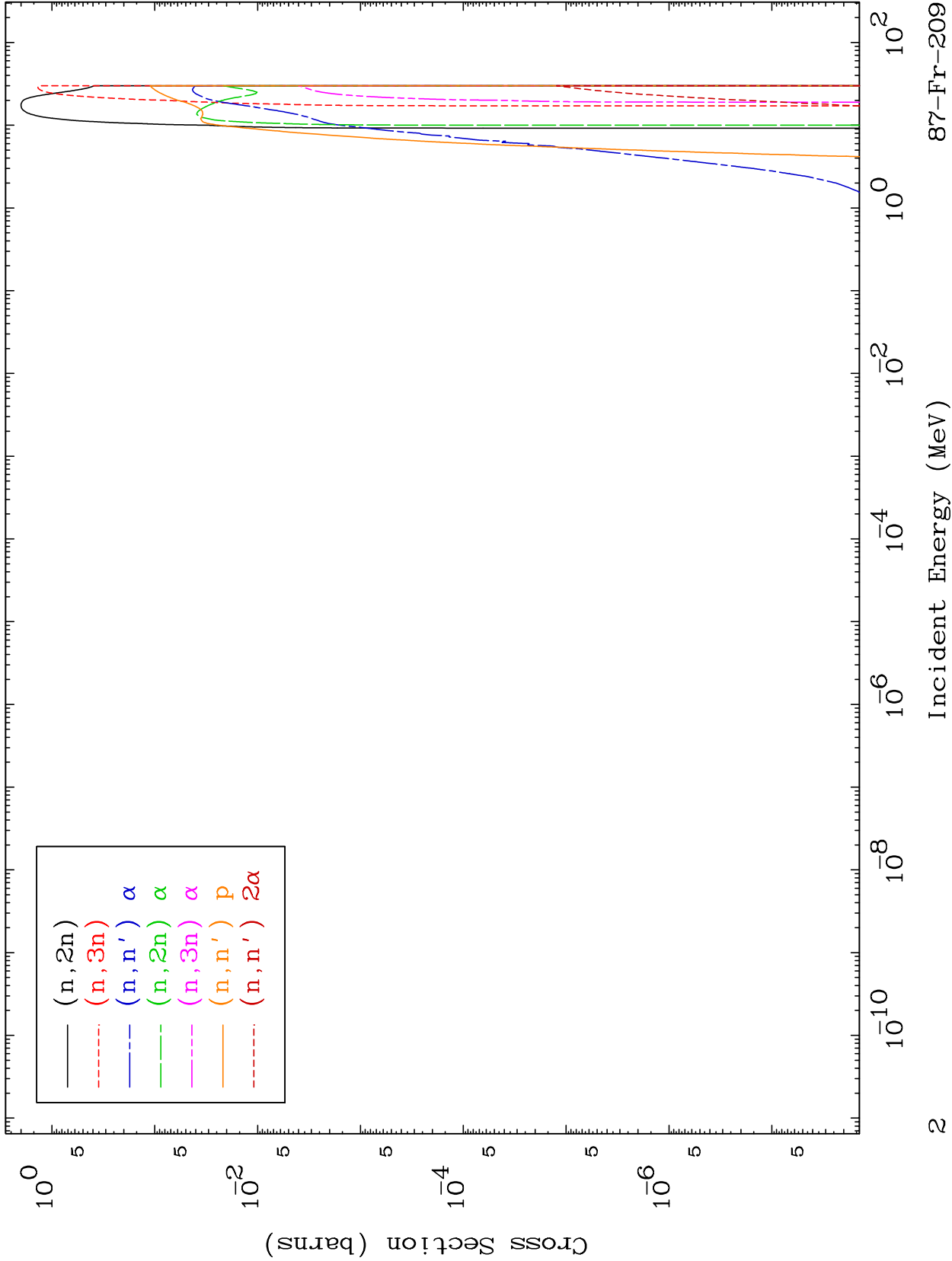
87-Fr-209



MAT 8716

Neutron Production
293 Kelvin Cross Sections

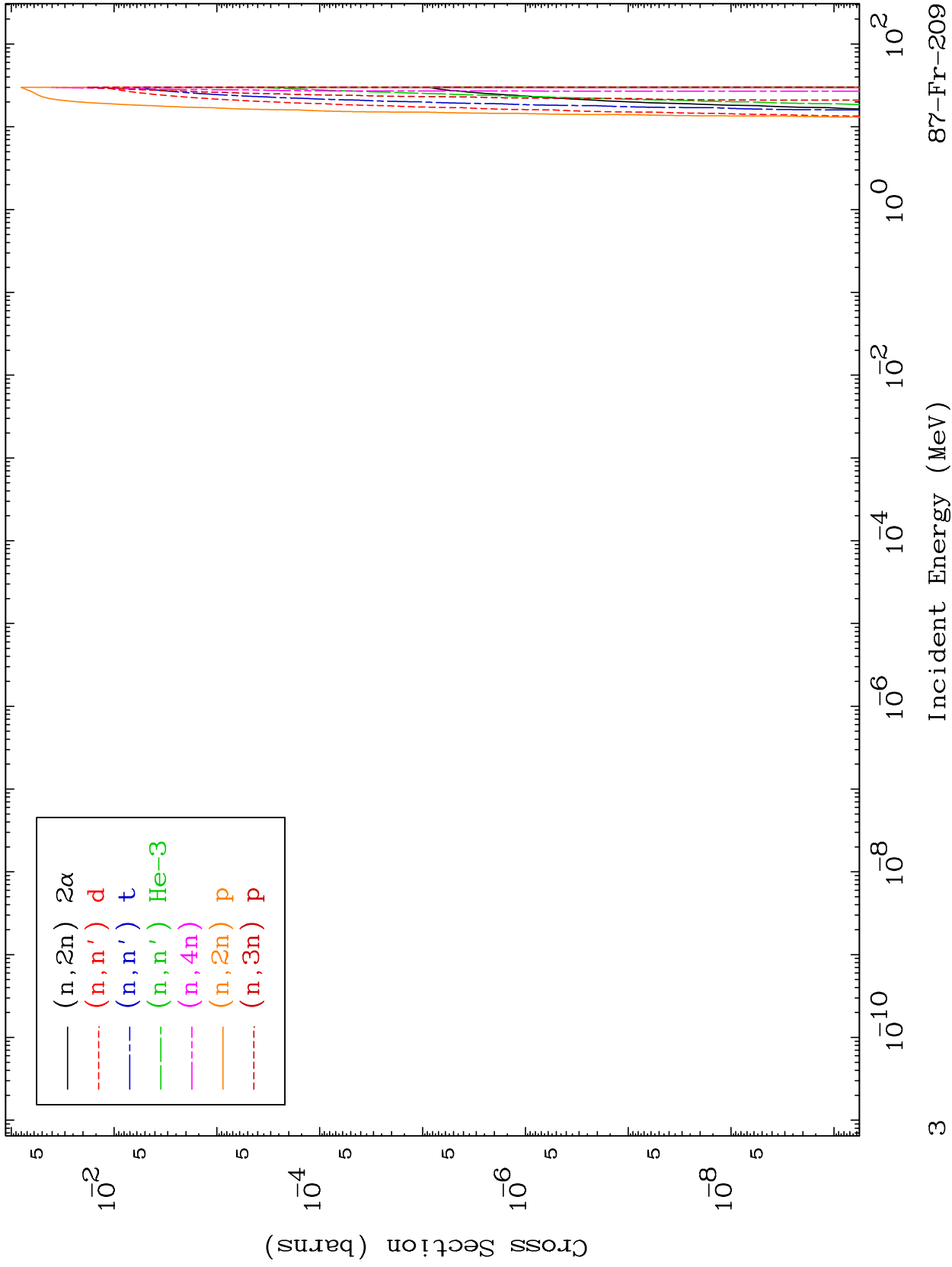
87-Fr-209



MAT 8716

Neutron Production
293 Kelvin Cross Sections

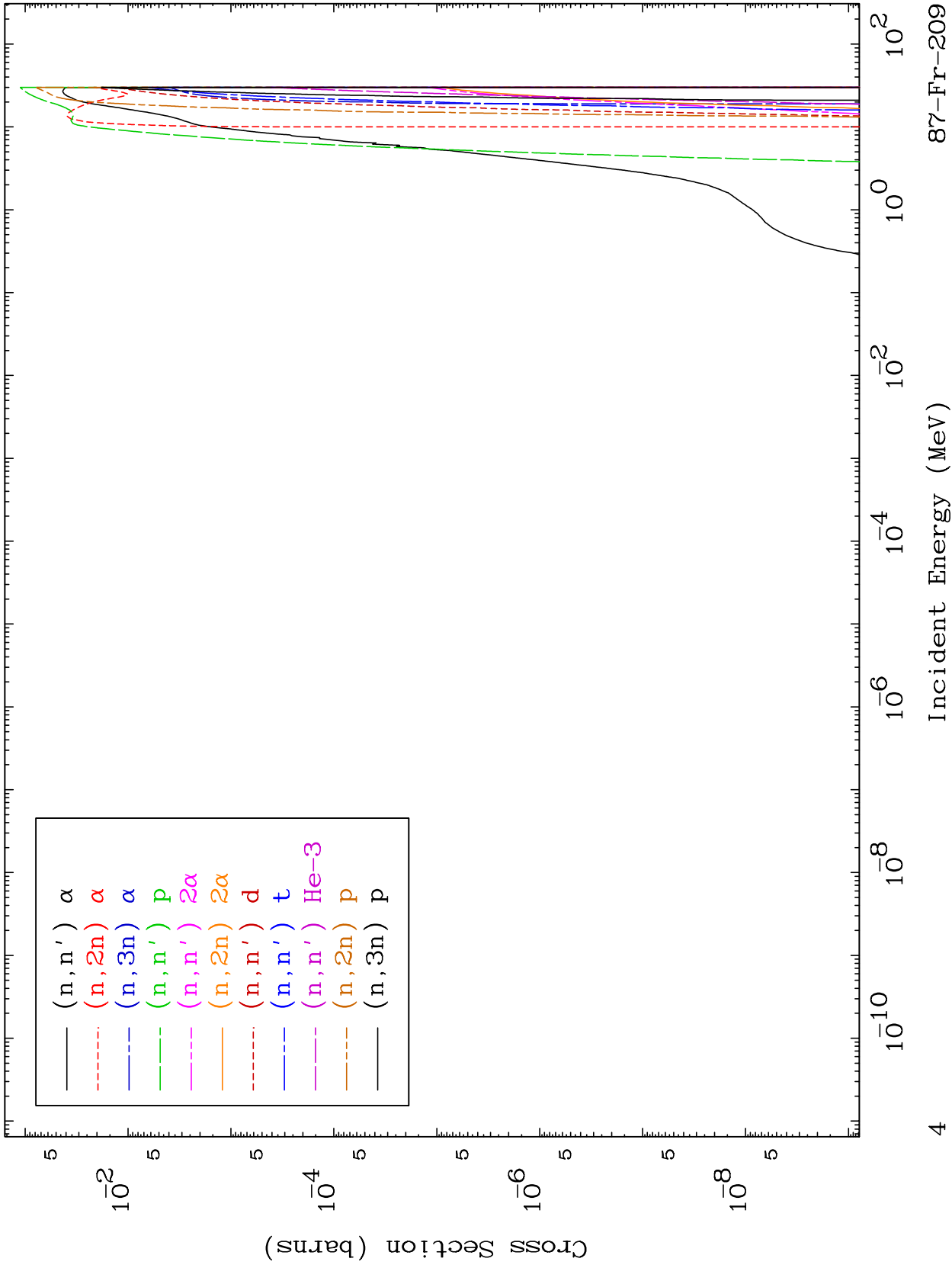
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MAT 8716

Charged Particle
293 Kelvin Cross Sections

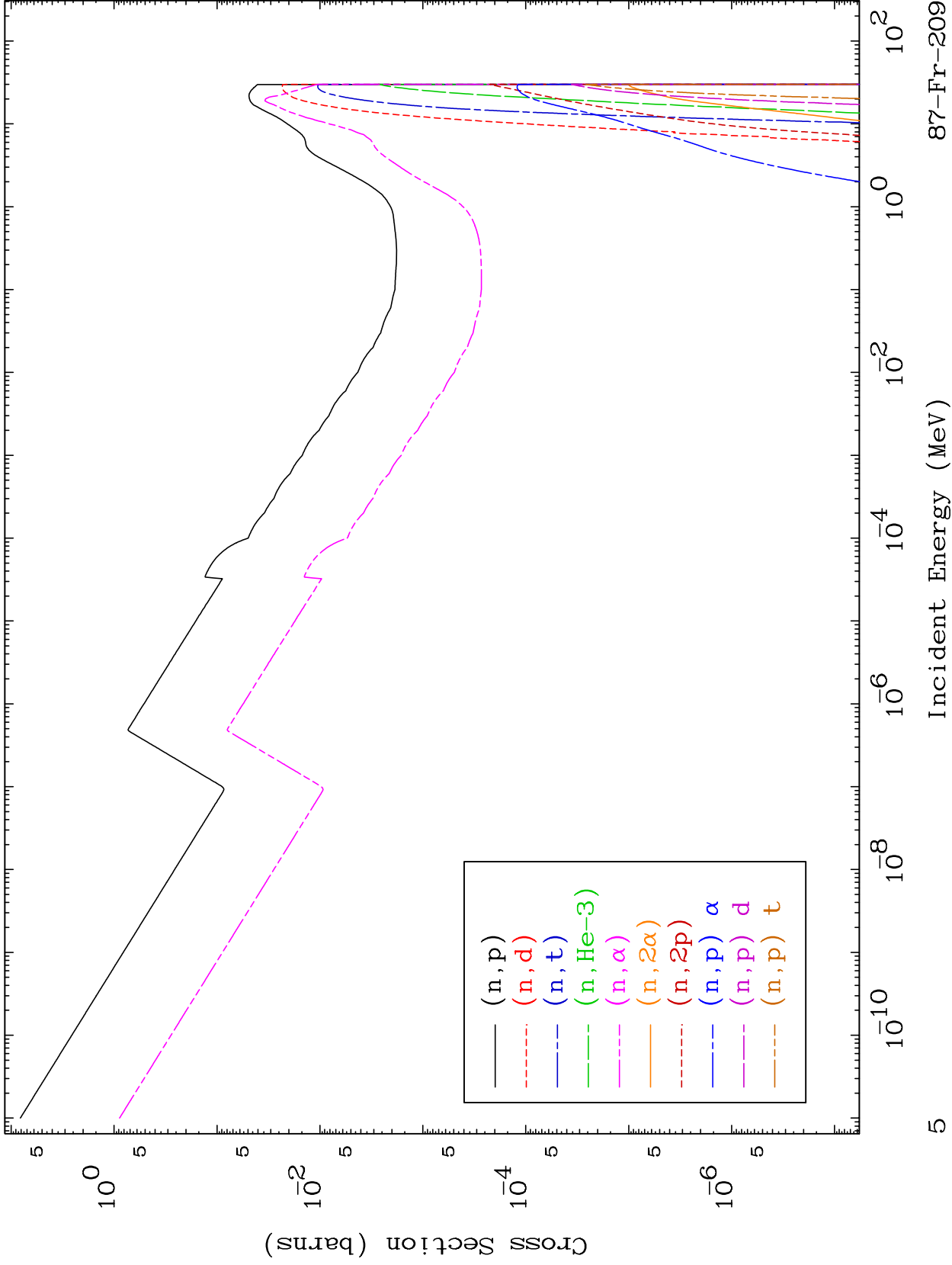
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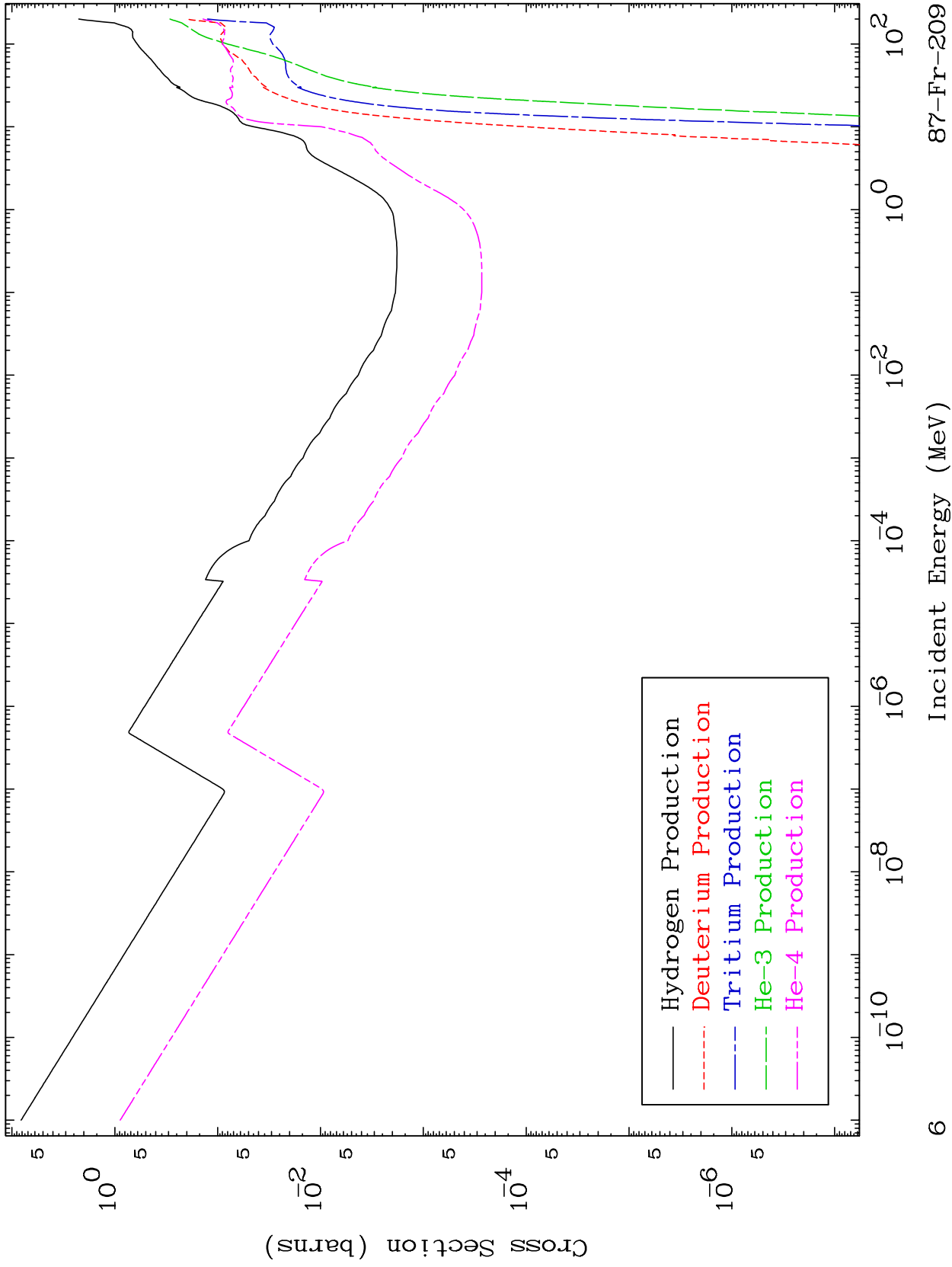


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Charged Particle
293 Kelvin Cross Sections

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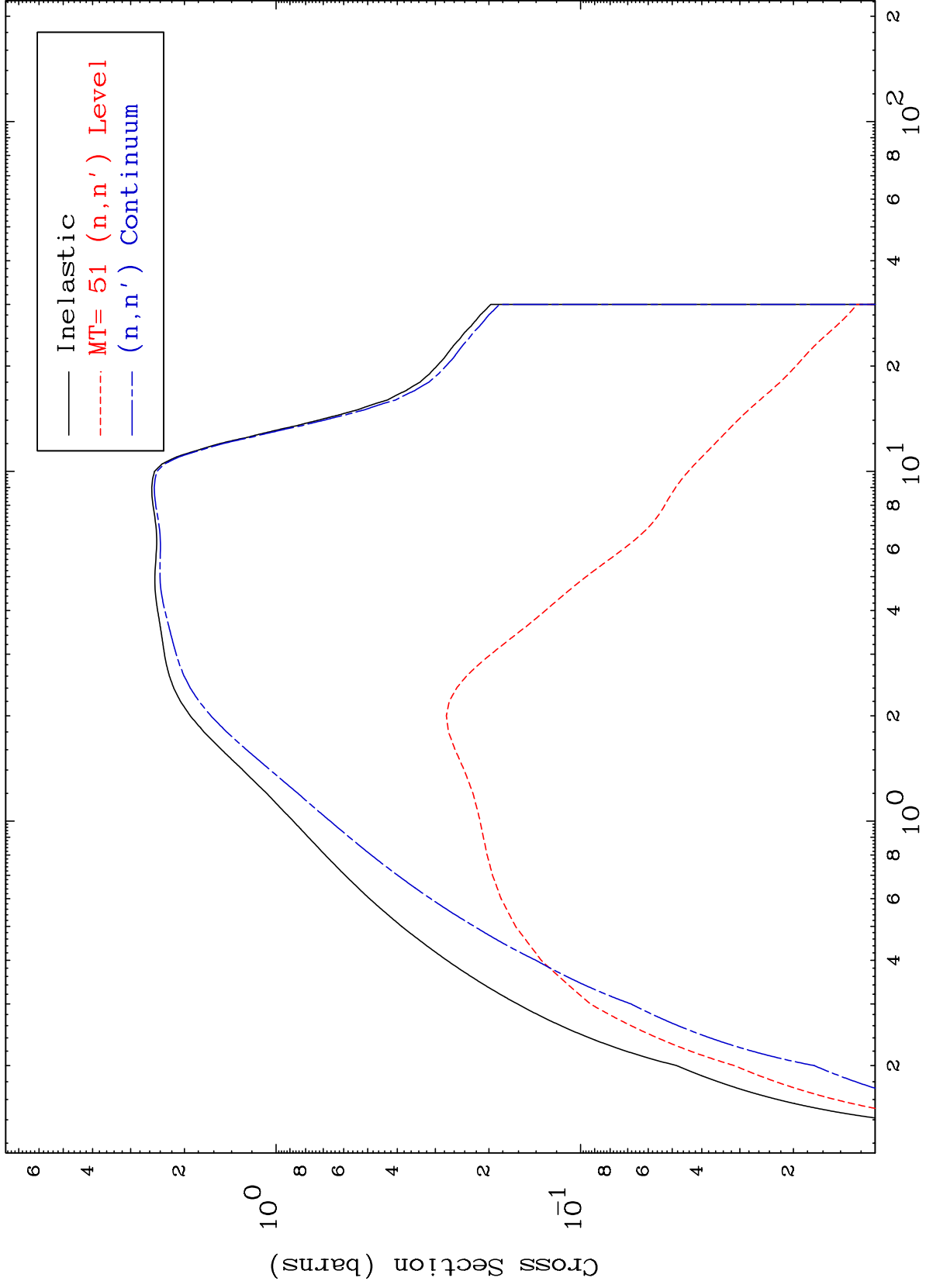




MAT 8716

(n,n') Level
293 Kelvin Cross Sections

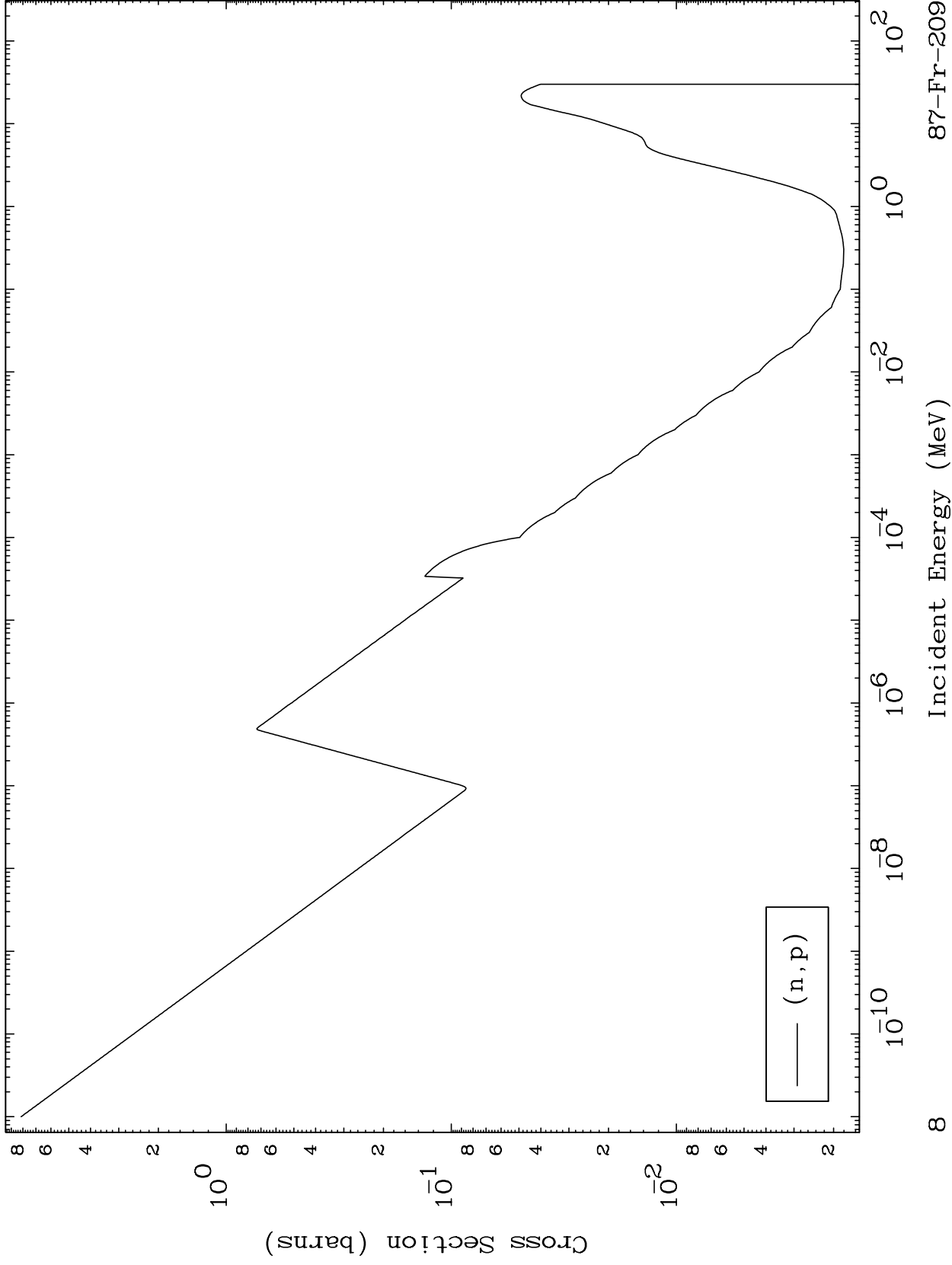
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(n,p) Levels
293 Kelvin Cross Sections

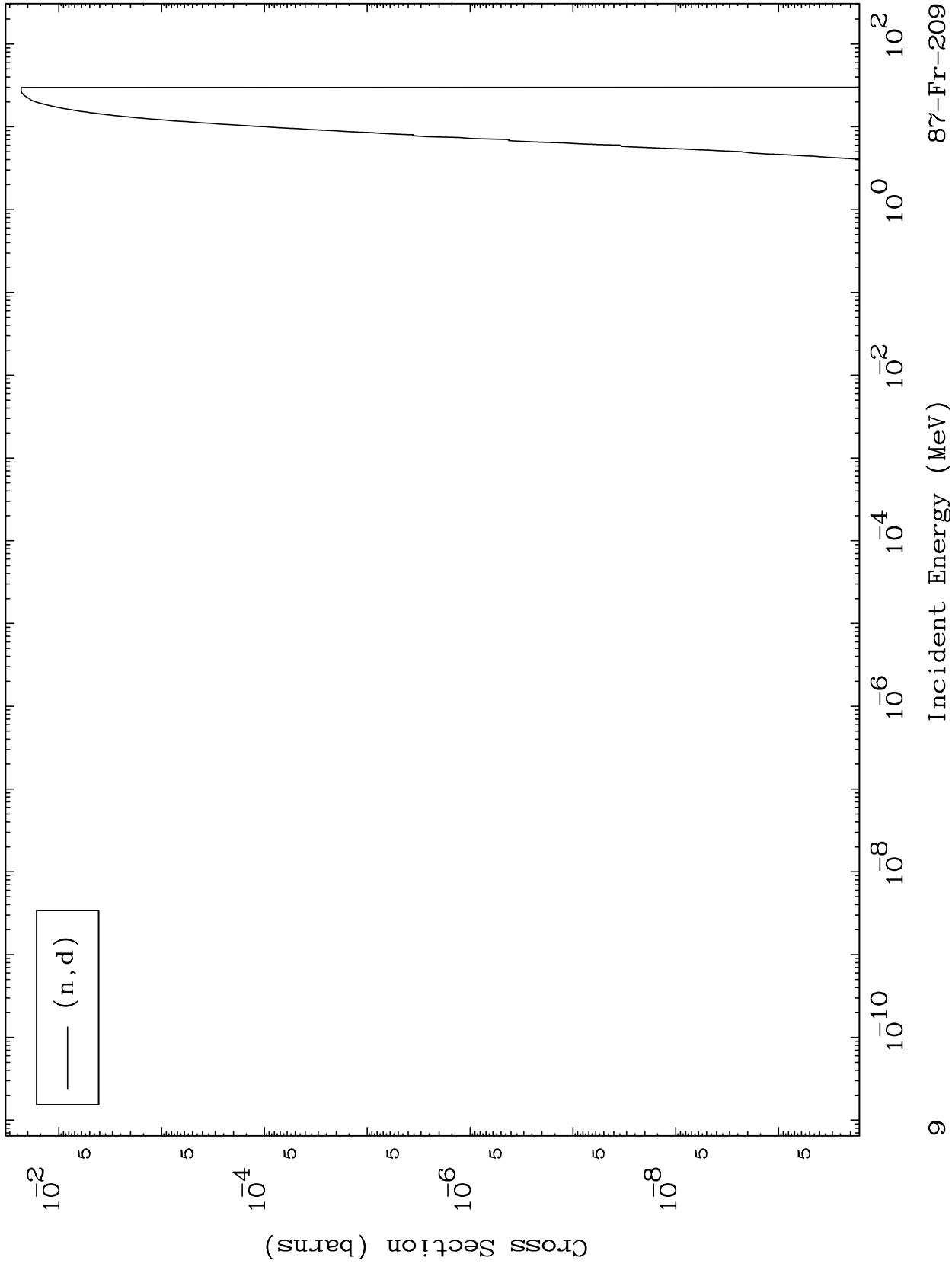
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(n,d) Levels
293 Kelvin Cross Sections

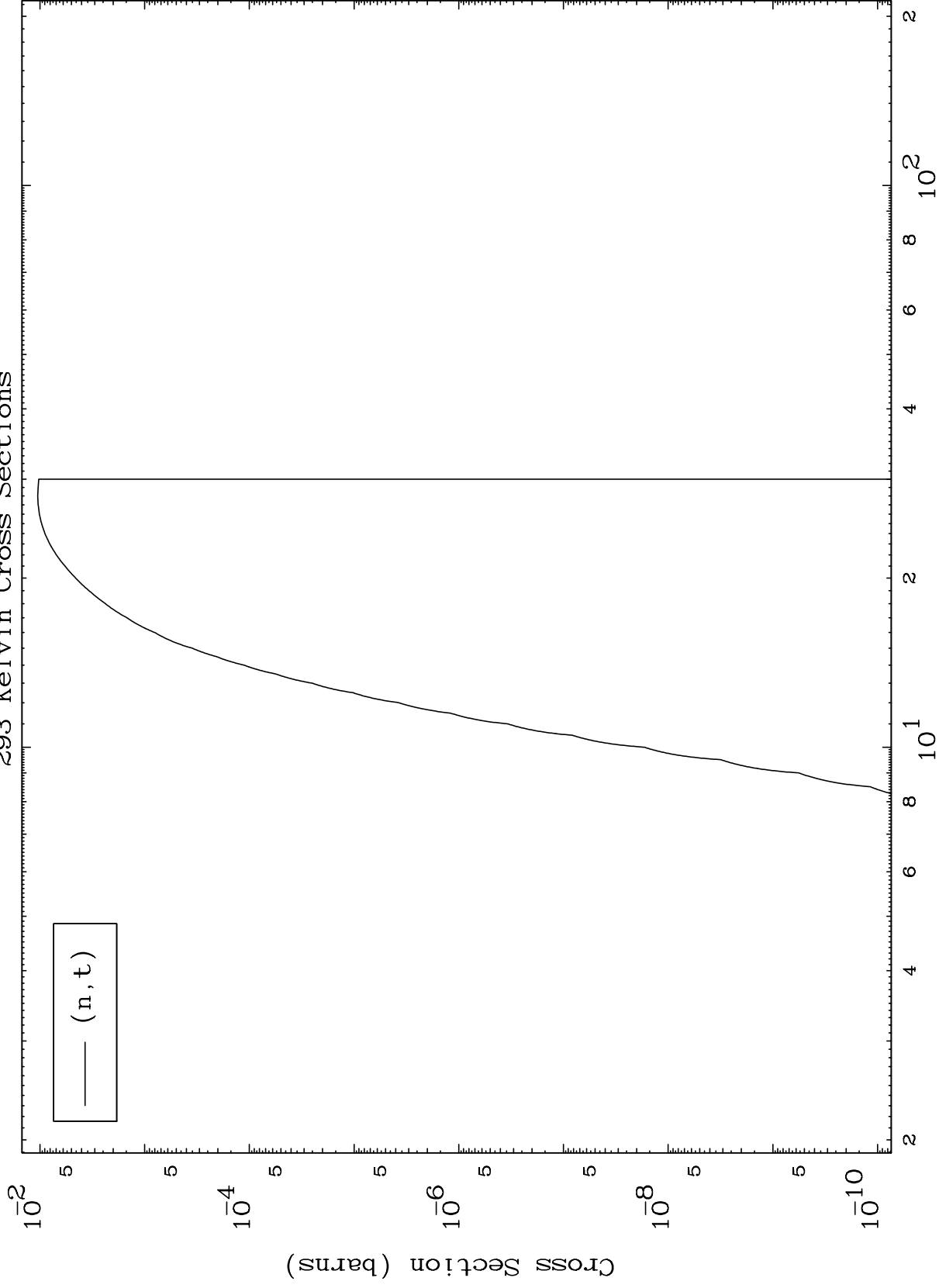
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MAT 8716

(n,t) Levels
293 Kelvin Cross Sections

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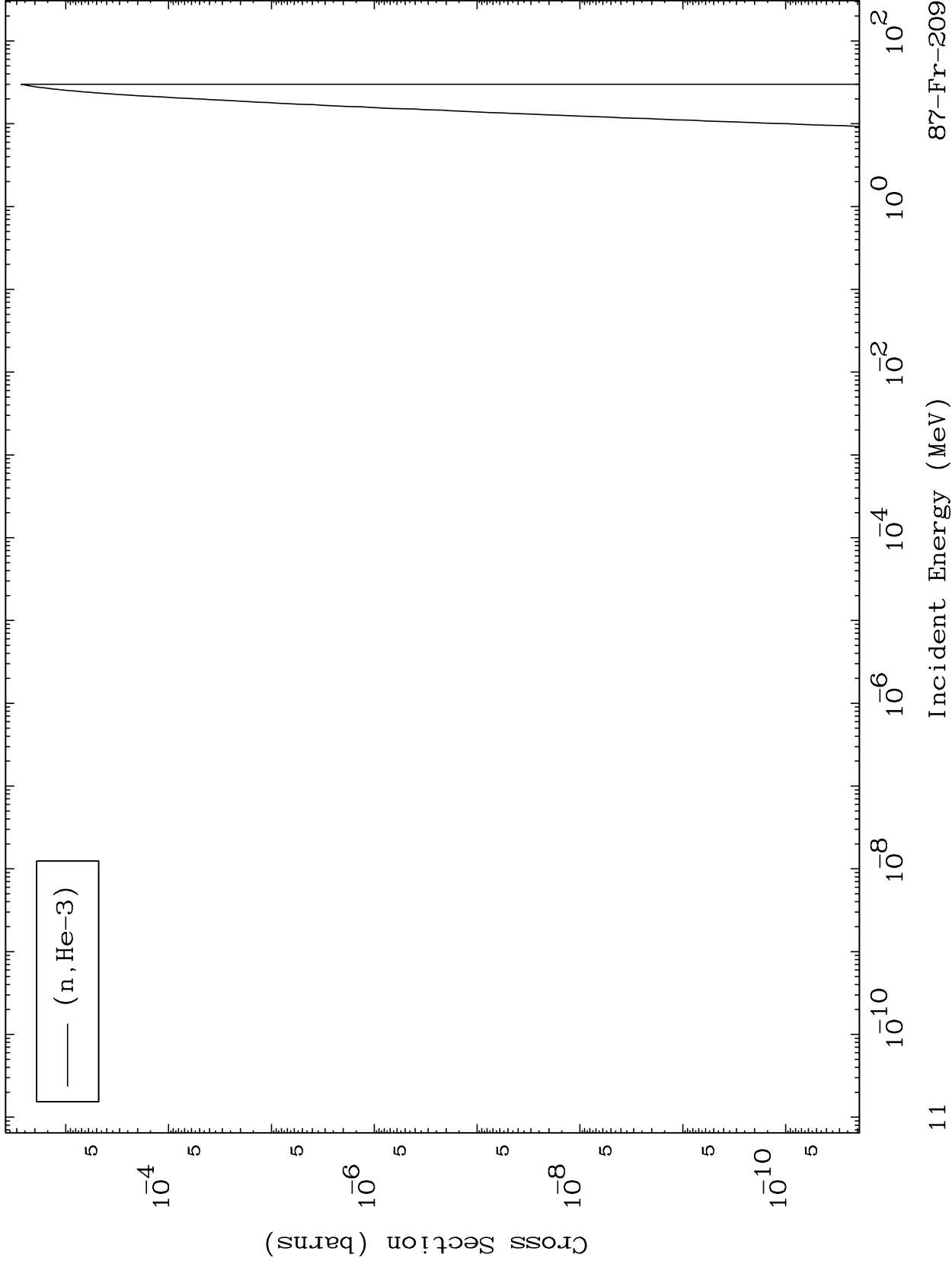
Incident Energy (MeV)

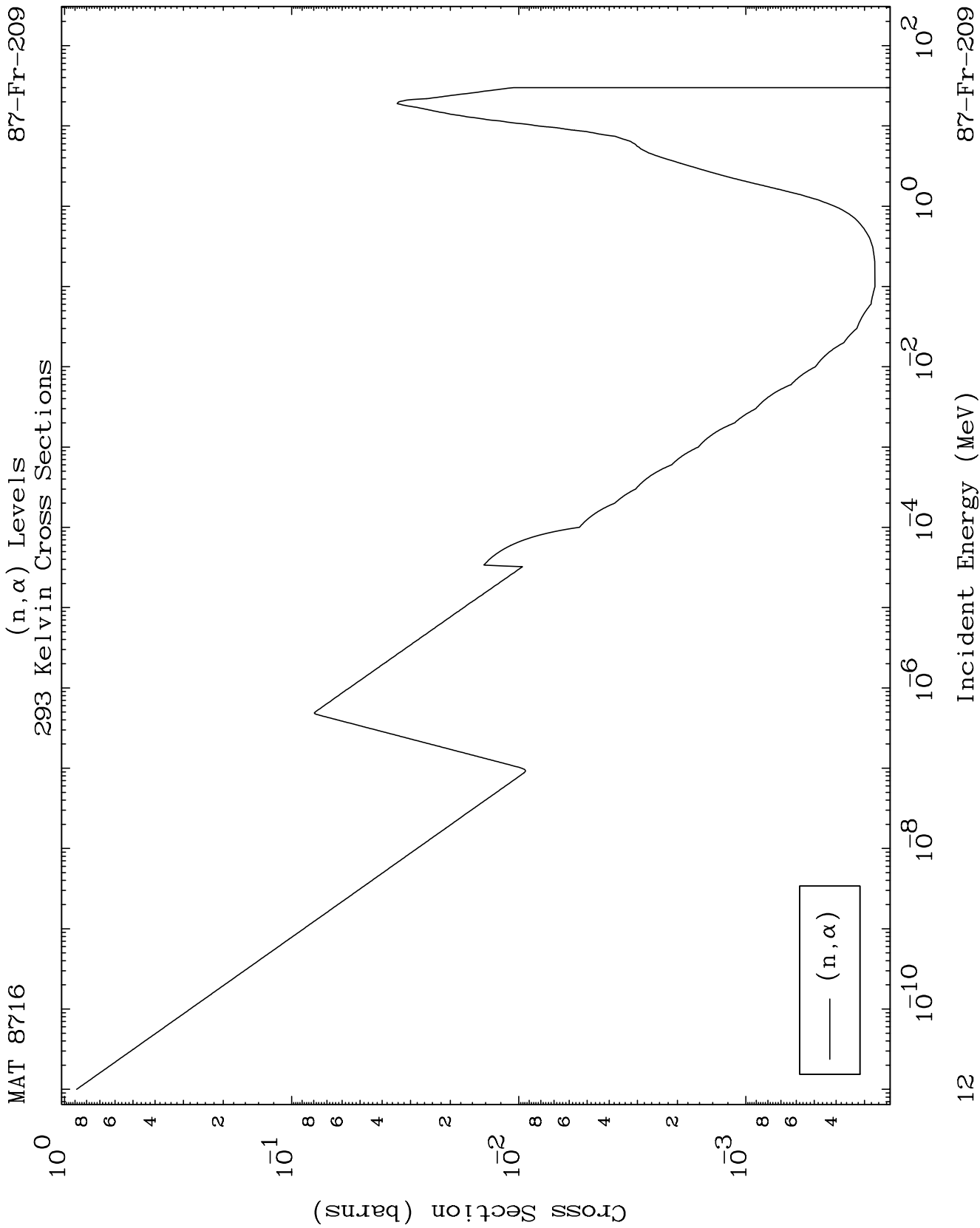
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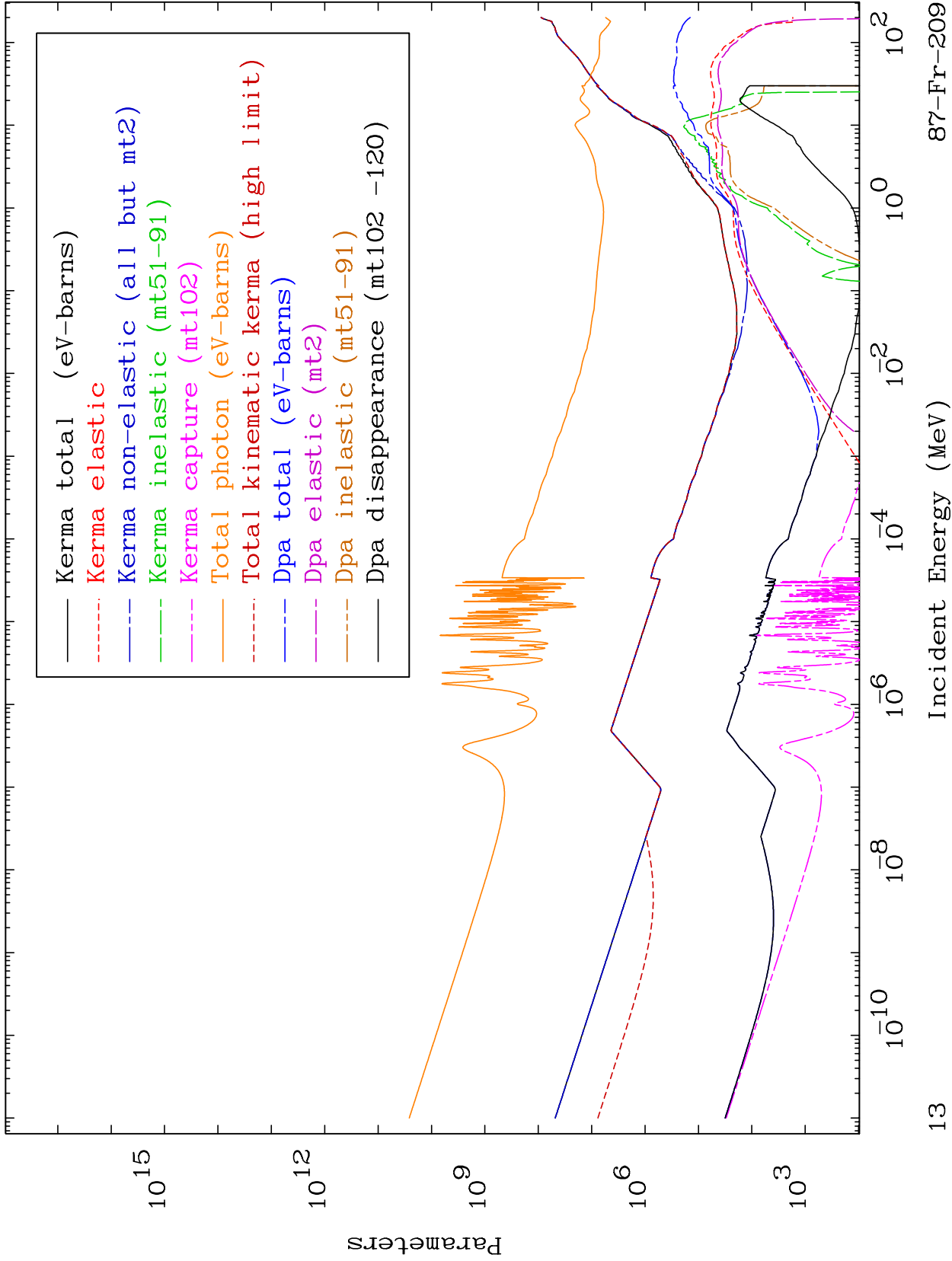
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(n,He3) Levels
293 Kelvin Cross Sections

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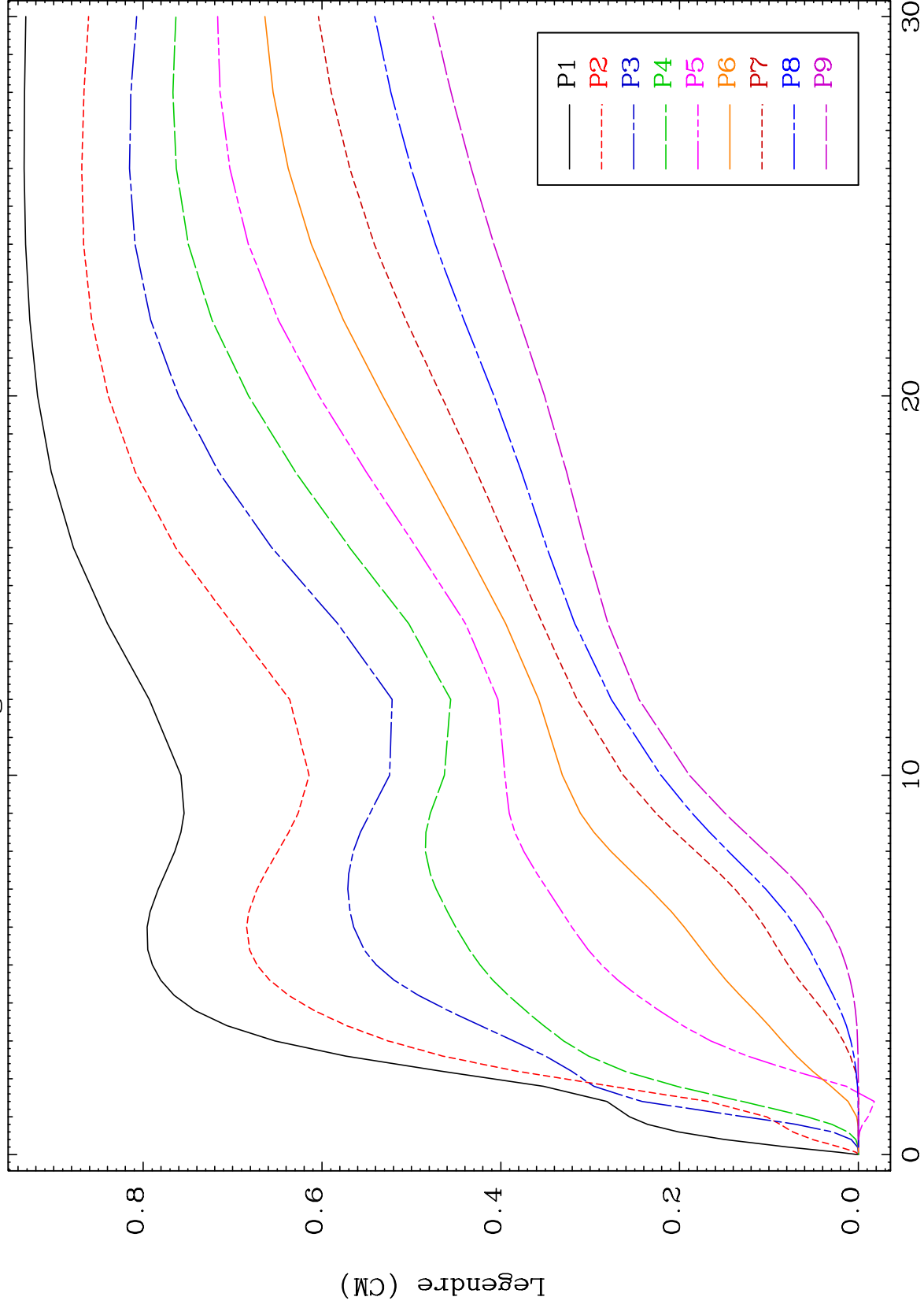




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Elastic Legendre Coefficients

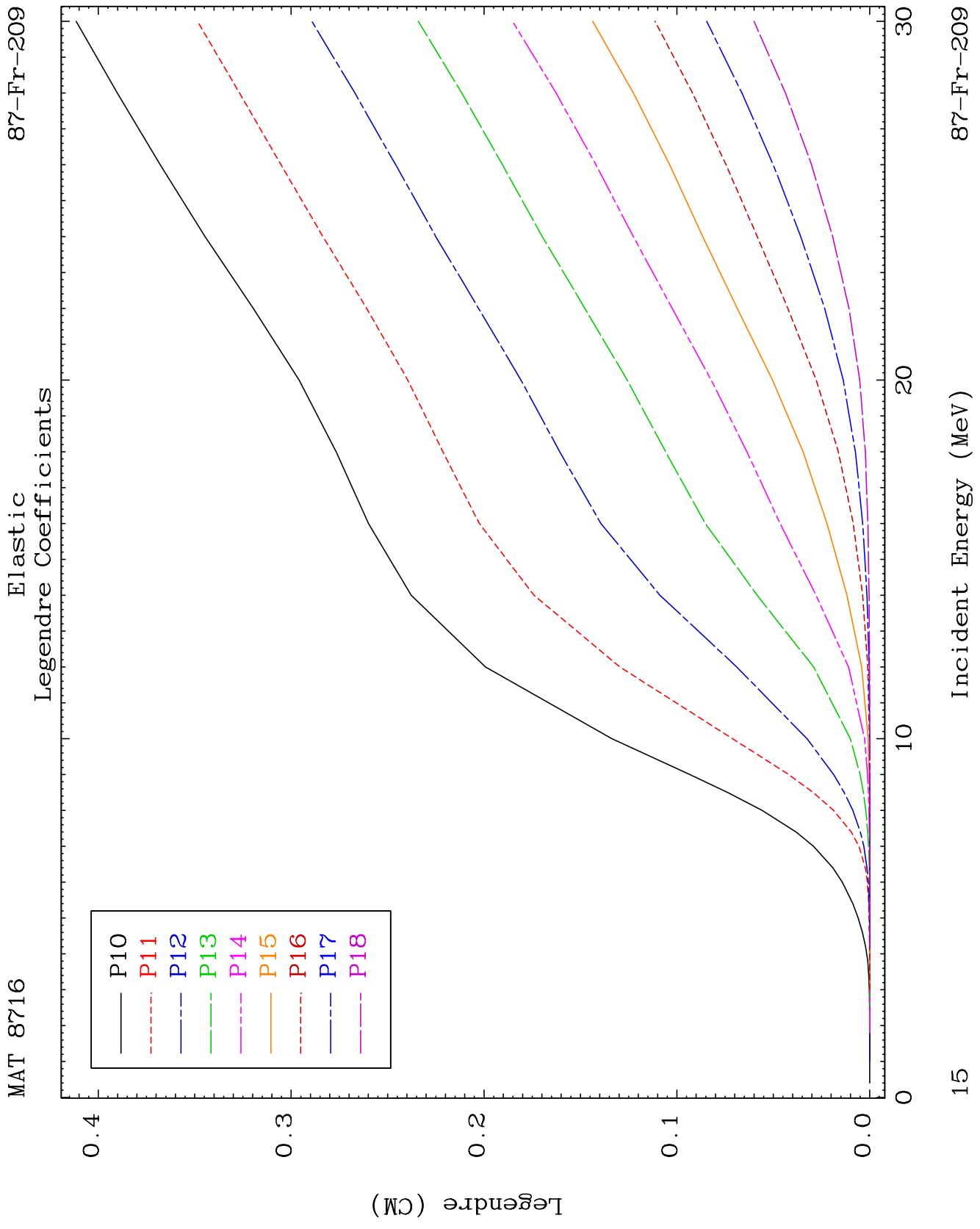
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Incident Energy (MeV)

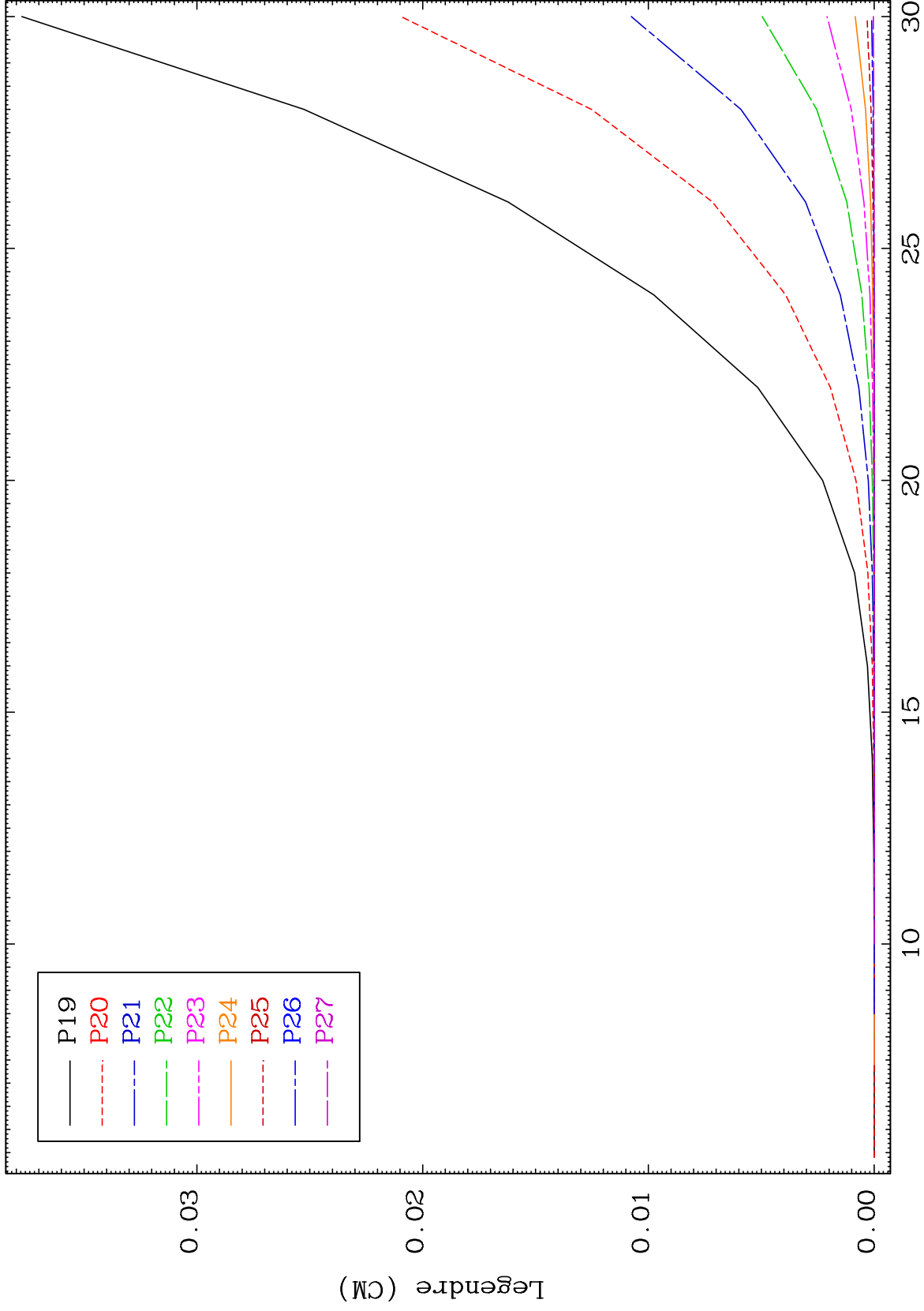
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Elastic Legendre Coefficients

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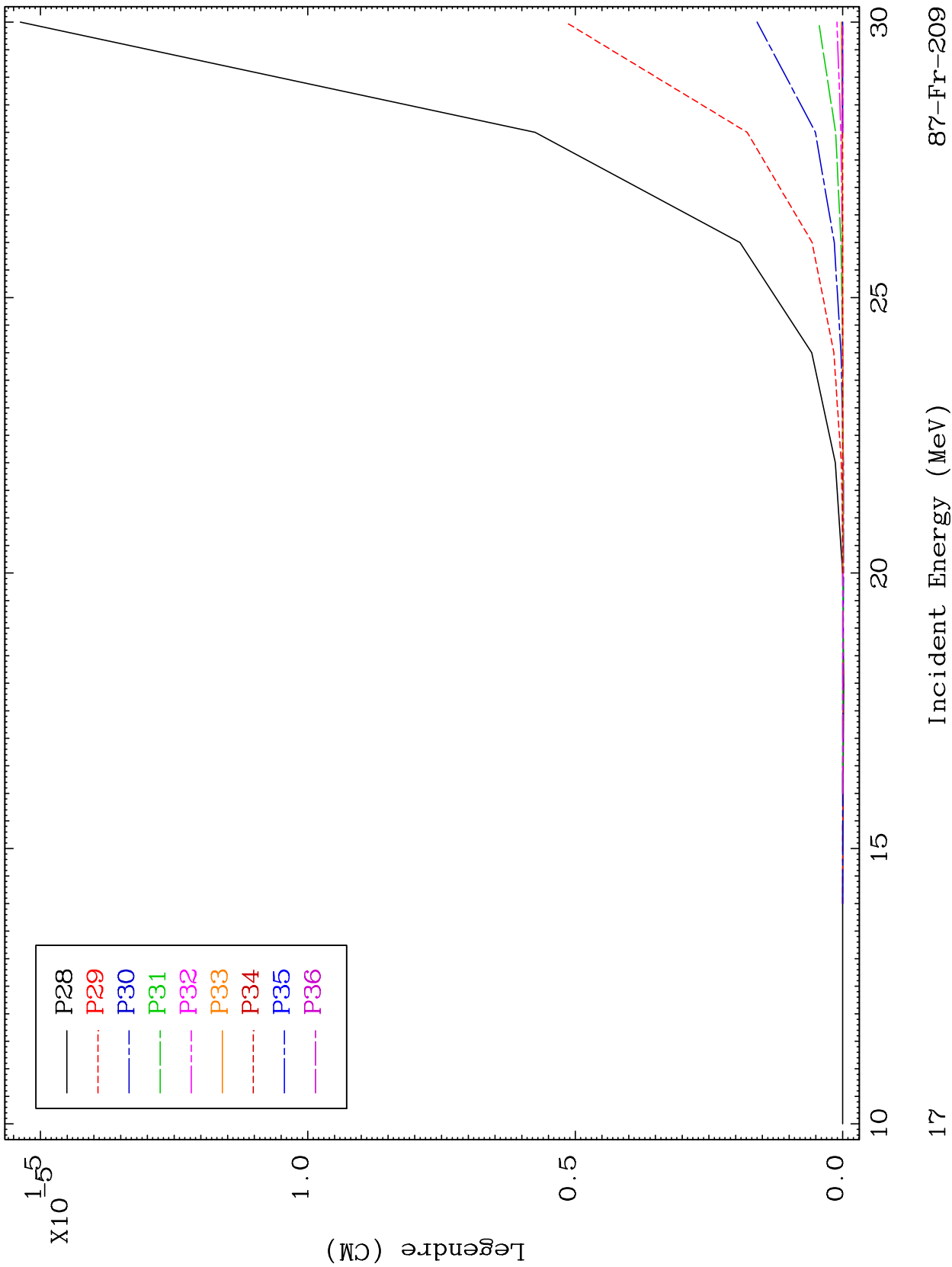
Incident Energy (MeV)

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Elastic
Legendre Coefficients

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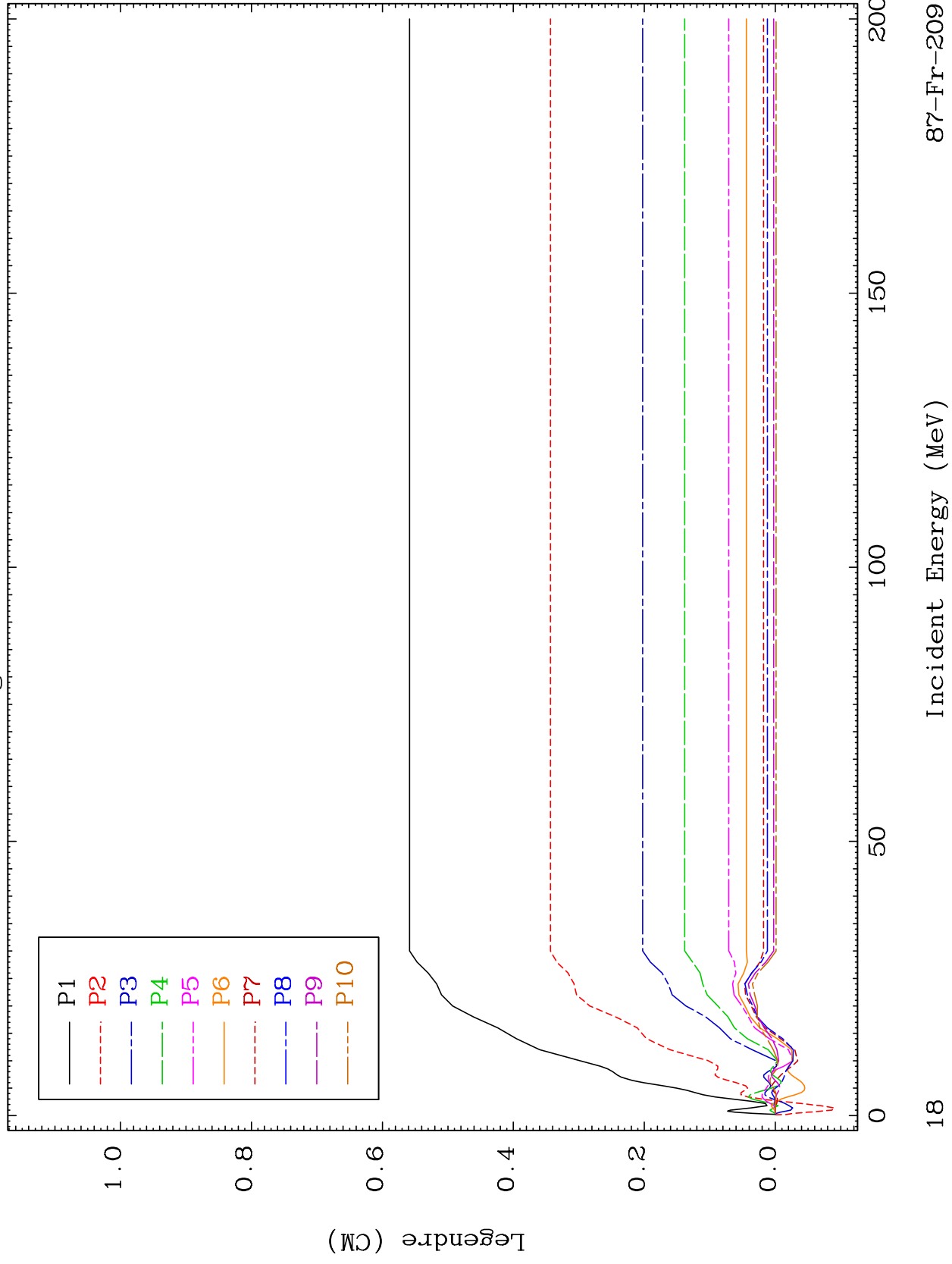
Incident Energy (MeV)

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MT= 51 (n,n') Level
Legendre Coefficients

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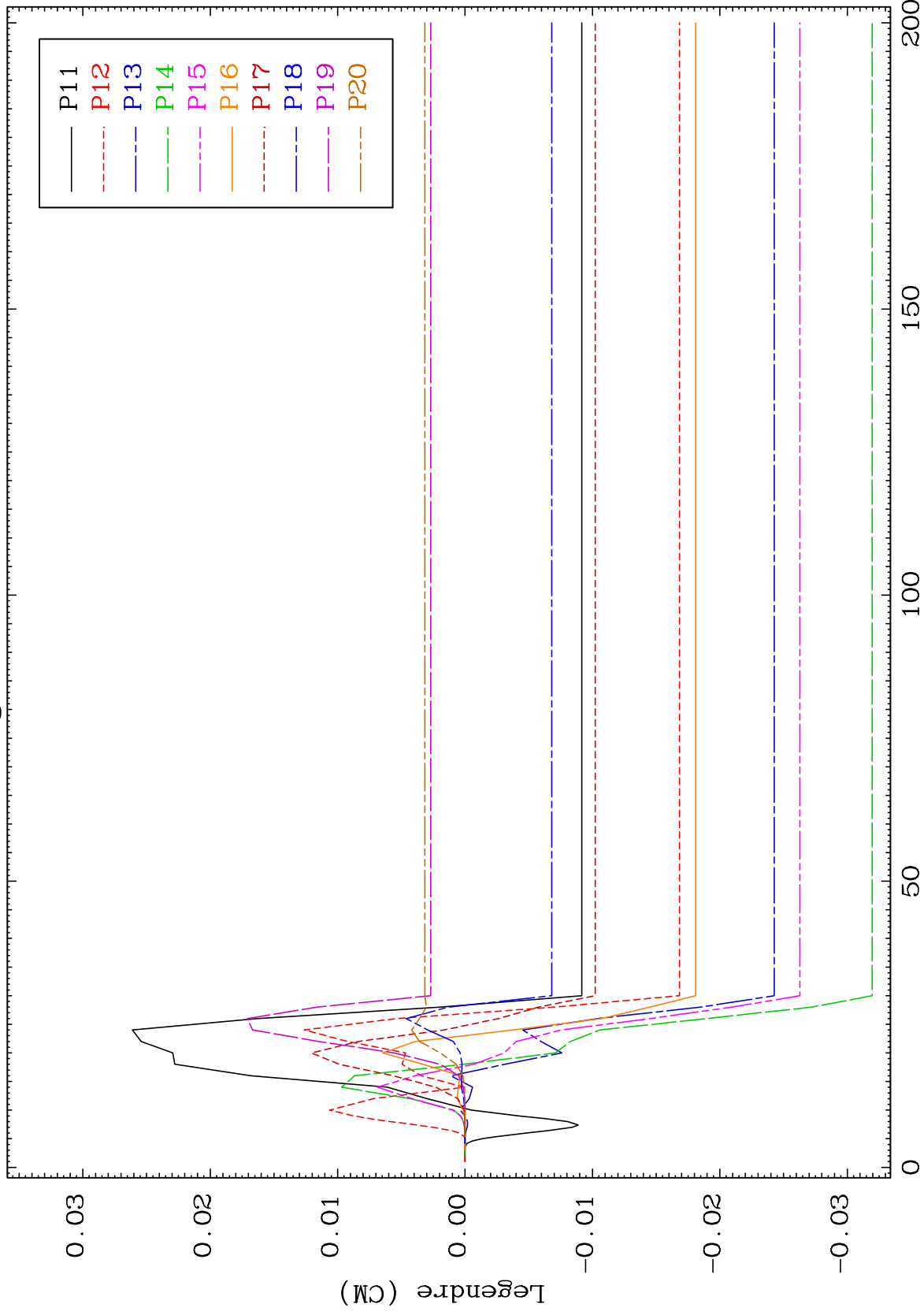
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MT= 51 (n,n') Level
Legendre Coefficients

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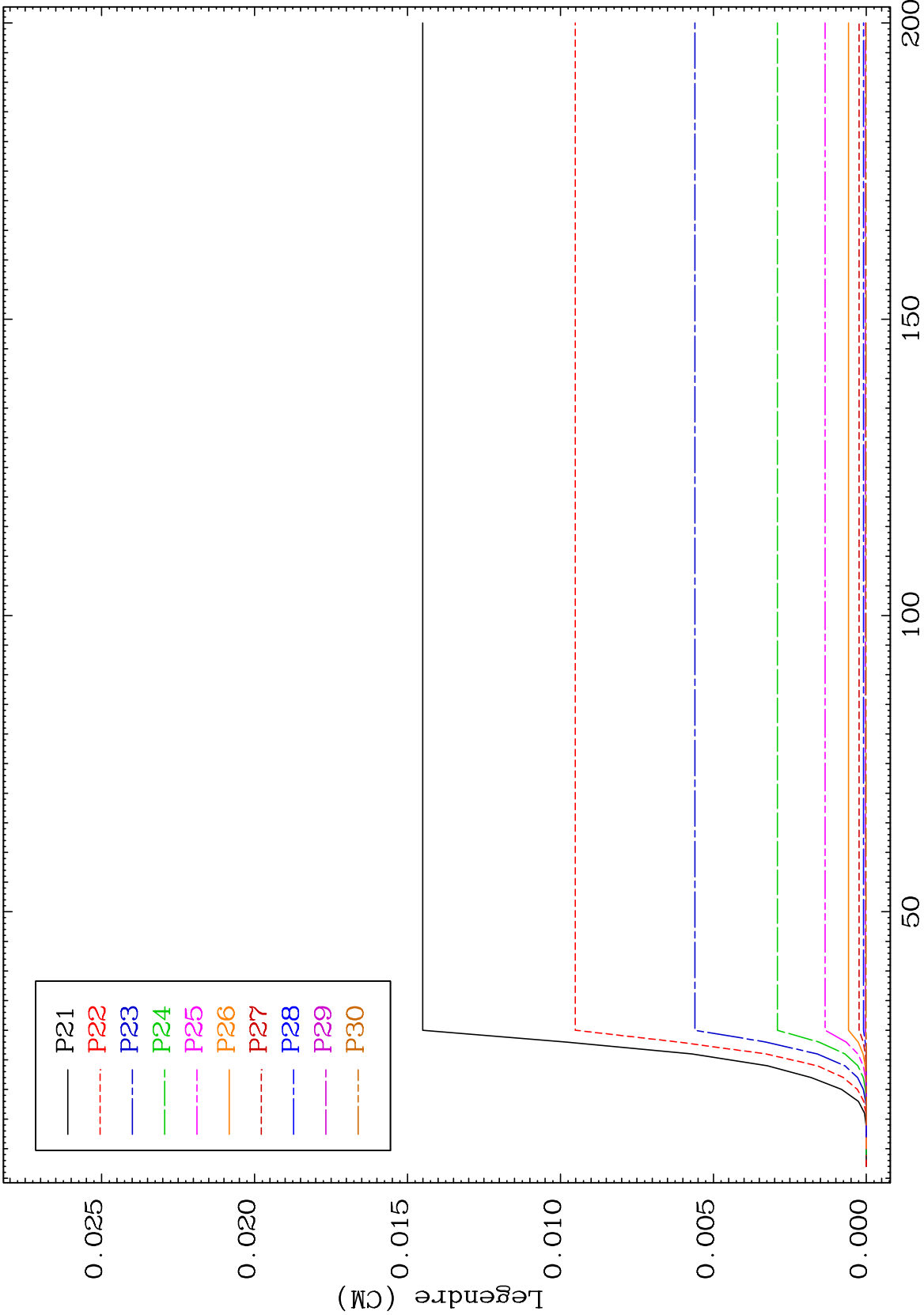
Incident Energy (MeV)

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MT= 51 (n,n') Level
Legendre Coefficients

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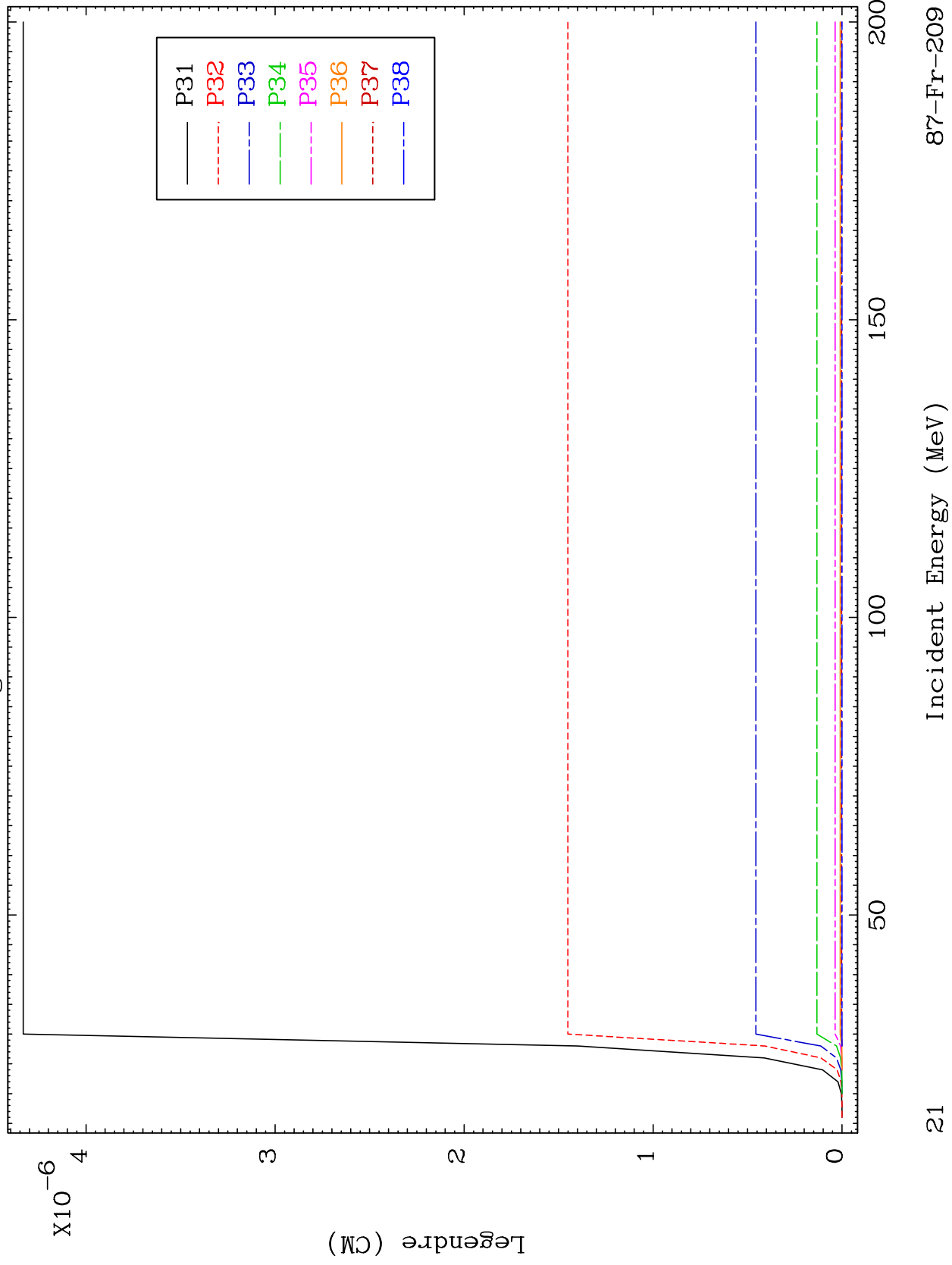
Incident Energy (MeV)

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MAT 8716

MT= 51 (n,n') Level
Legendre Coefficients

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Incident Energy (MeV)

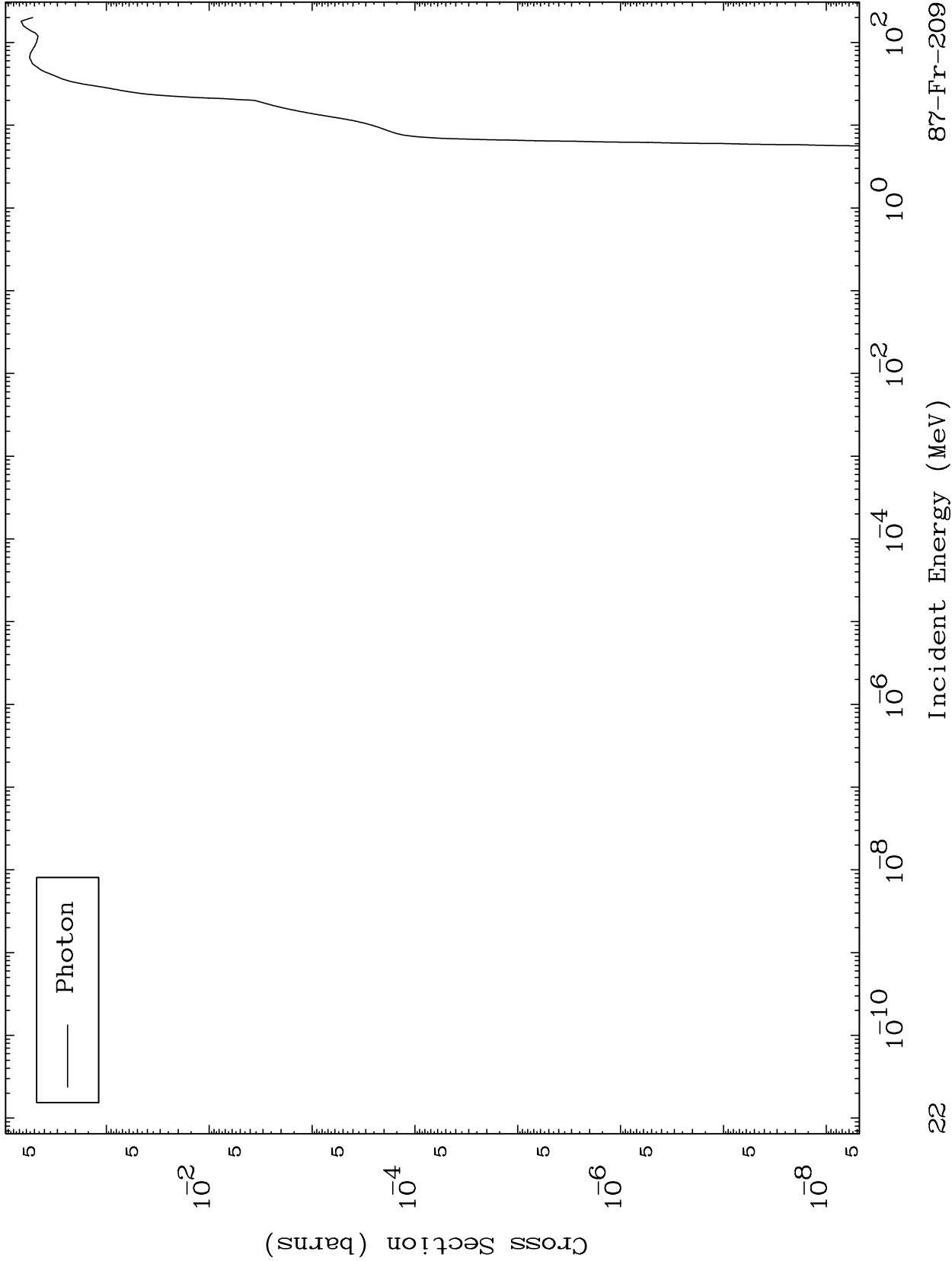
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Fission

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Radionuclide Production Cross Section

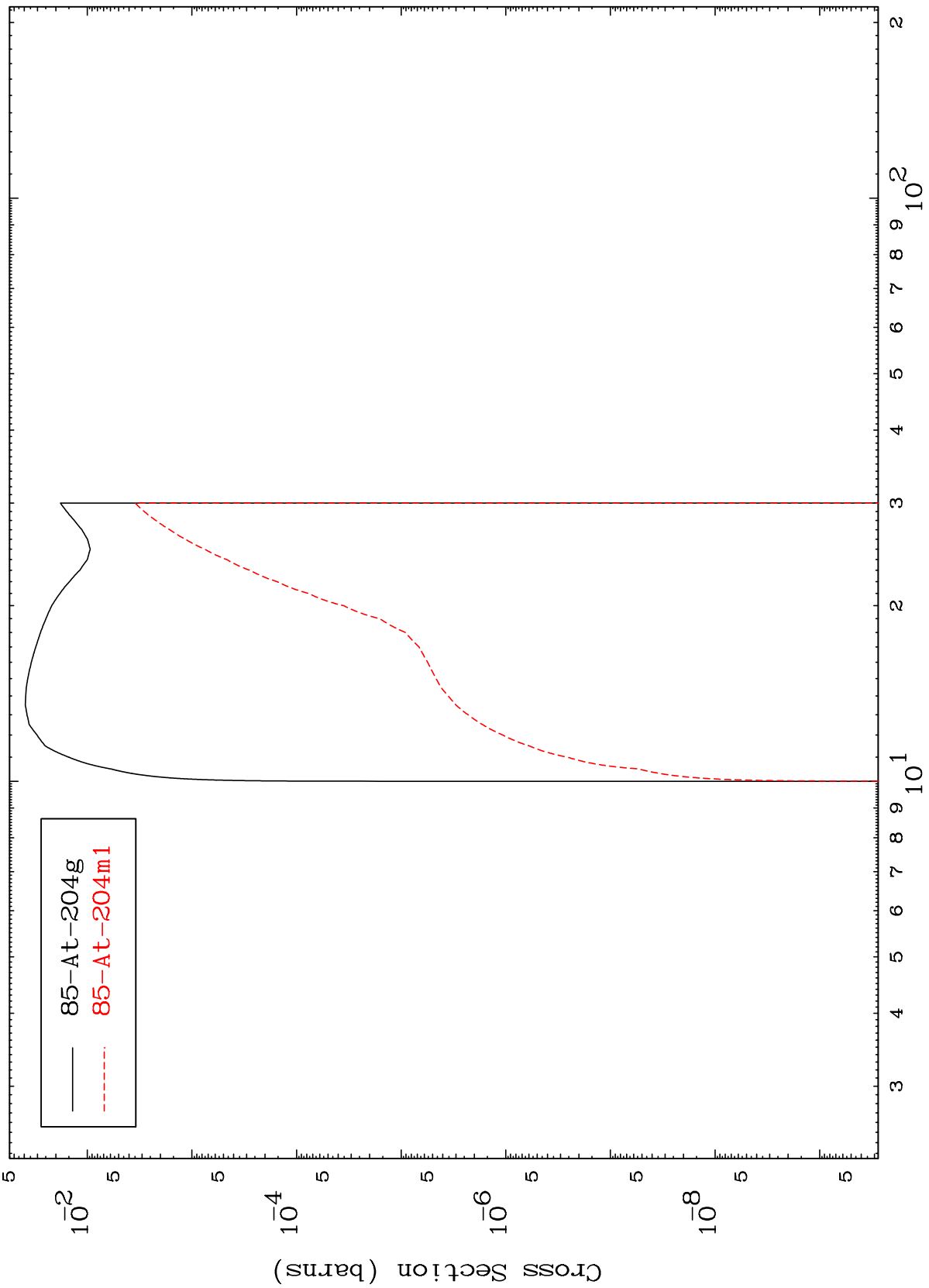


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(n,2n) α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

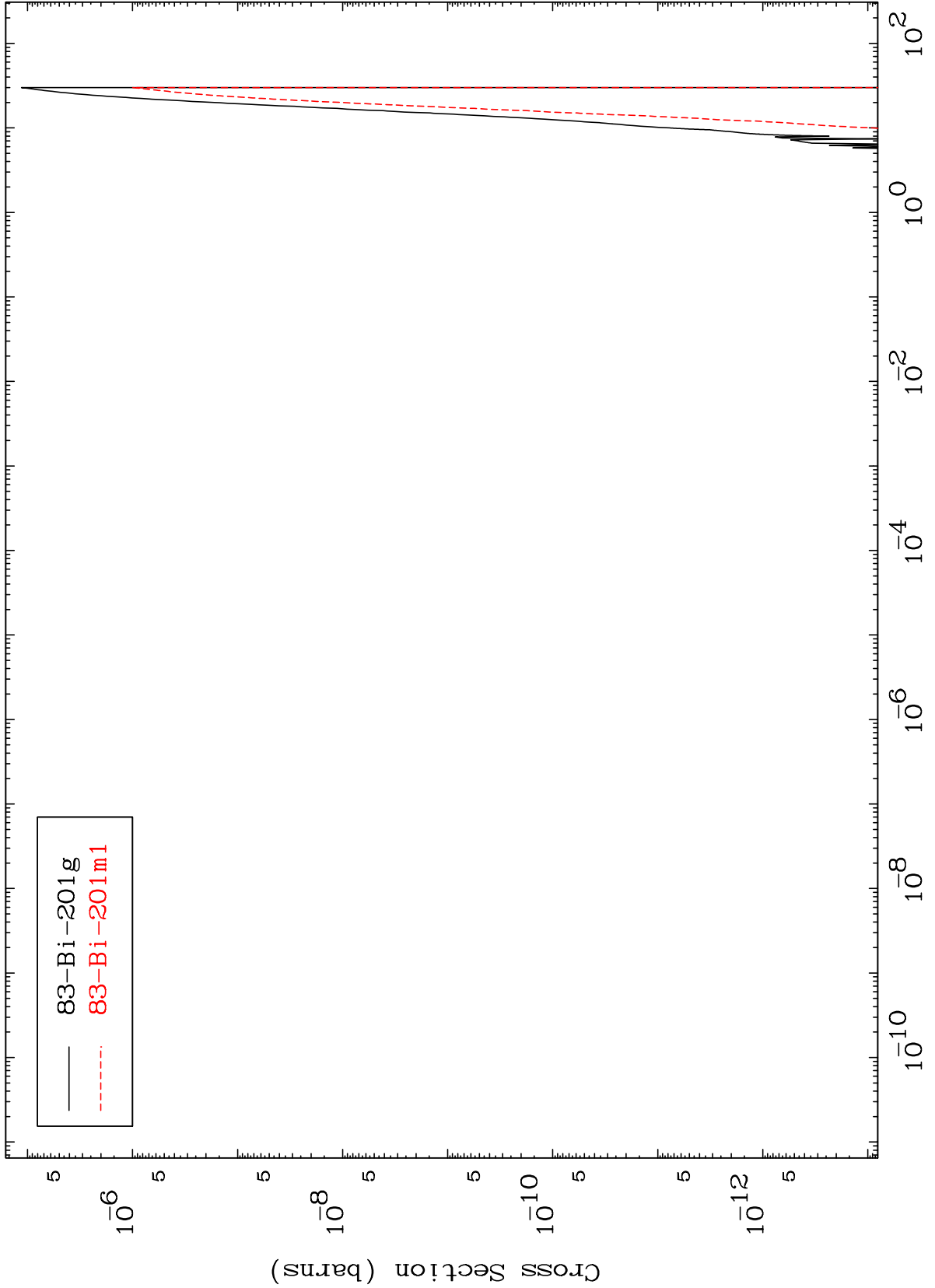
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MAT 8716

(n,n') 2 α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

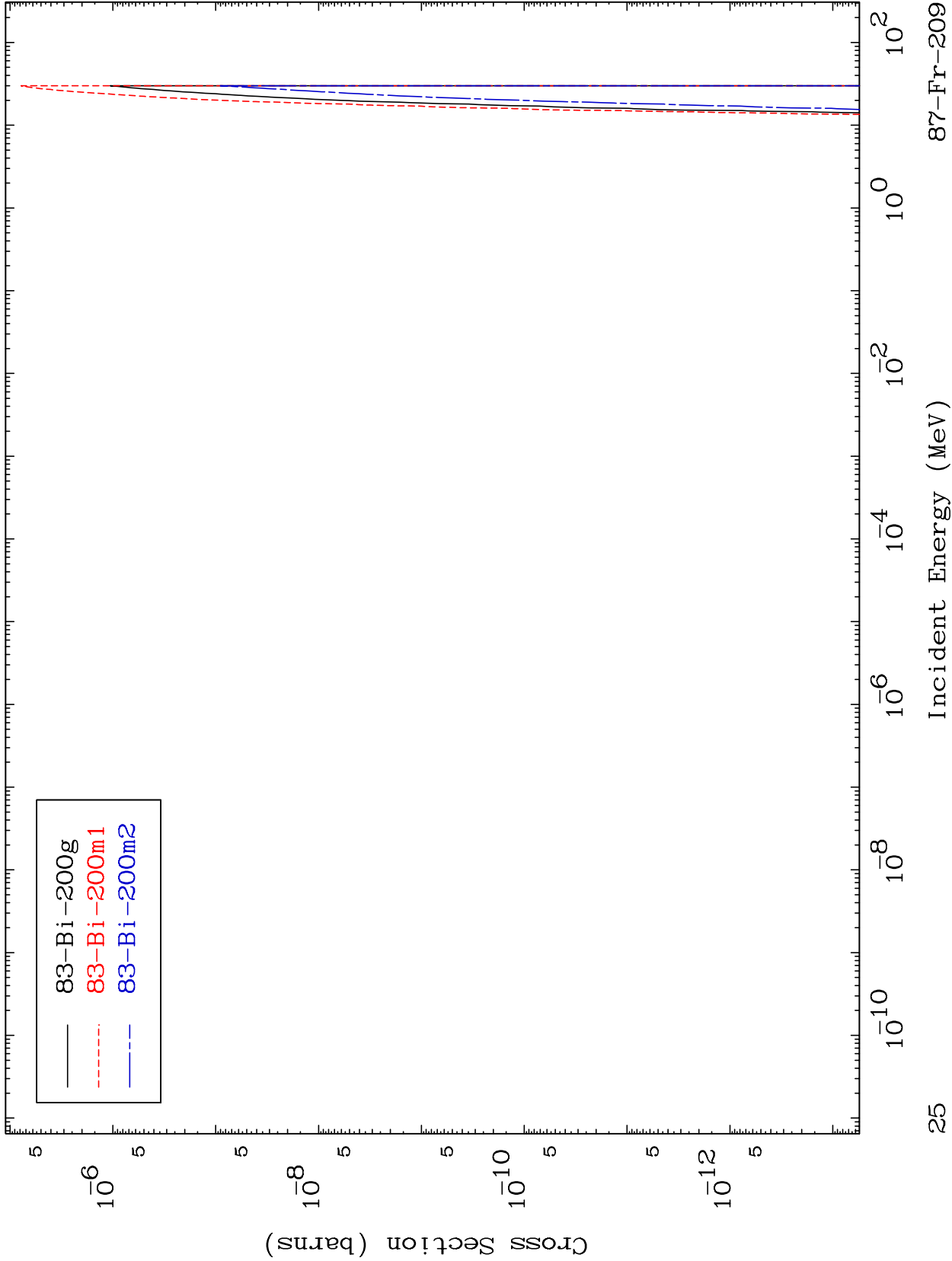
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MAT 8716

$(n,2n) 2\alpha$

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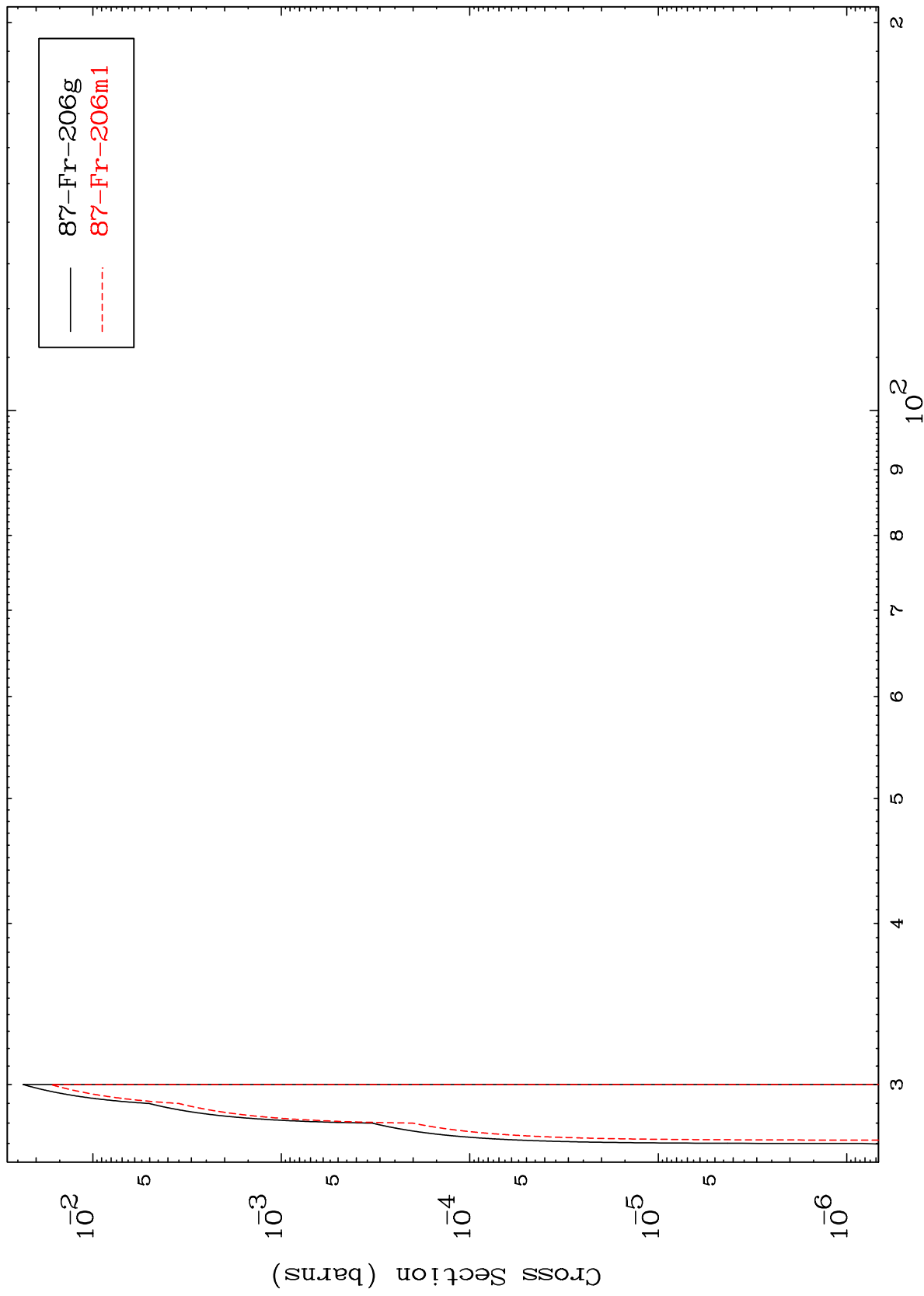
Radionuclide Production Cross Section



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(n,4n)
Radionuclide Production Cross Section



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Incident Energy (MeV)

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