

Program EVALPLOT
(Version 2017-1)

by

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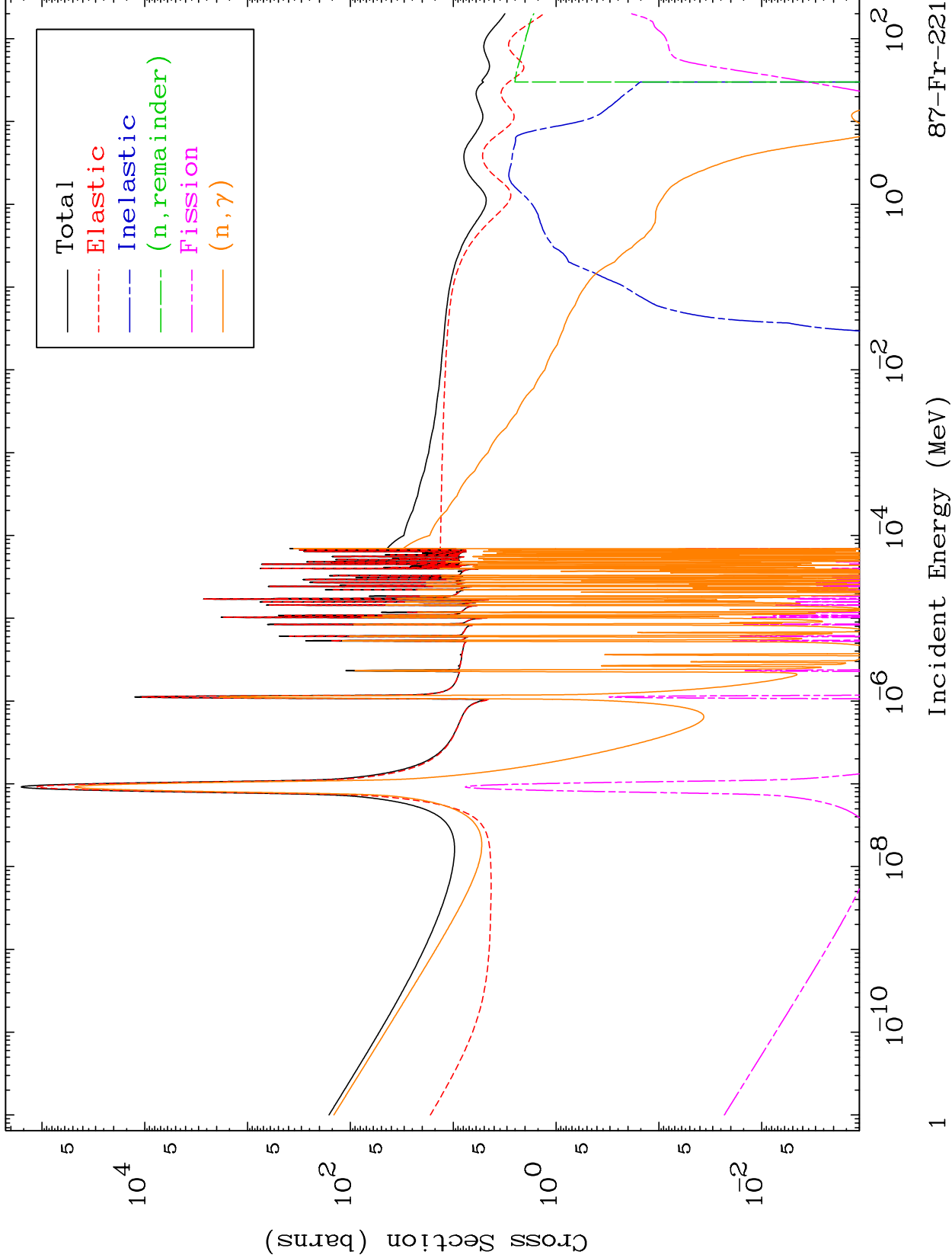
Press Mouse Button to Start

MAT 8752

Major

293 Kelvin Cross Sections

87-Fr-221



1

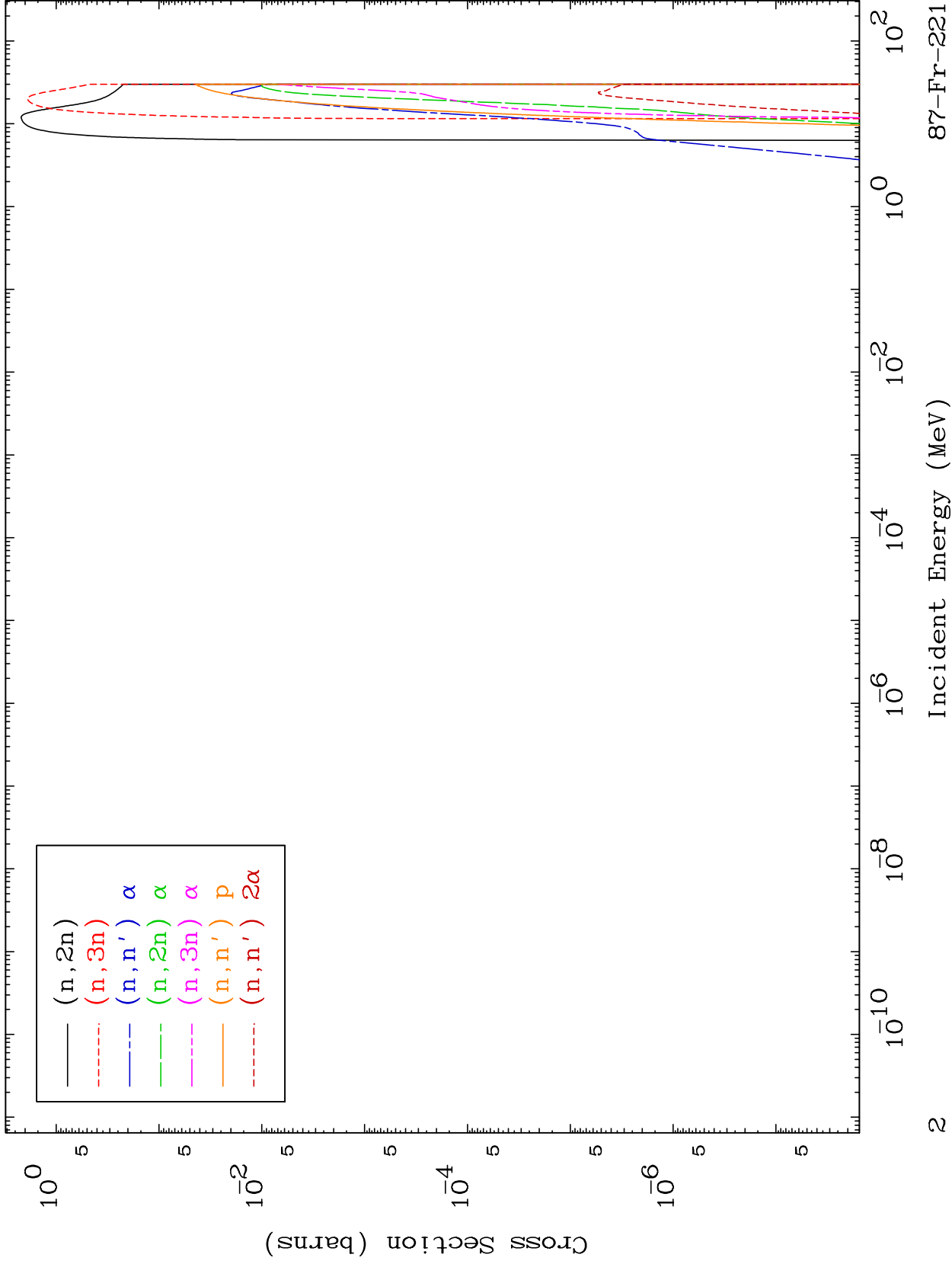
Incident Energy (MeV)

87-Fr-221

MAT 8752

Neutron Production
293 Kelvin Cross Sections

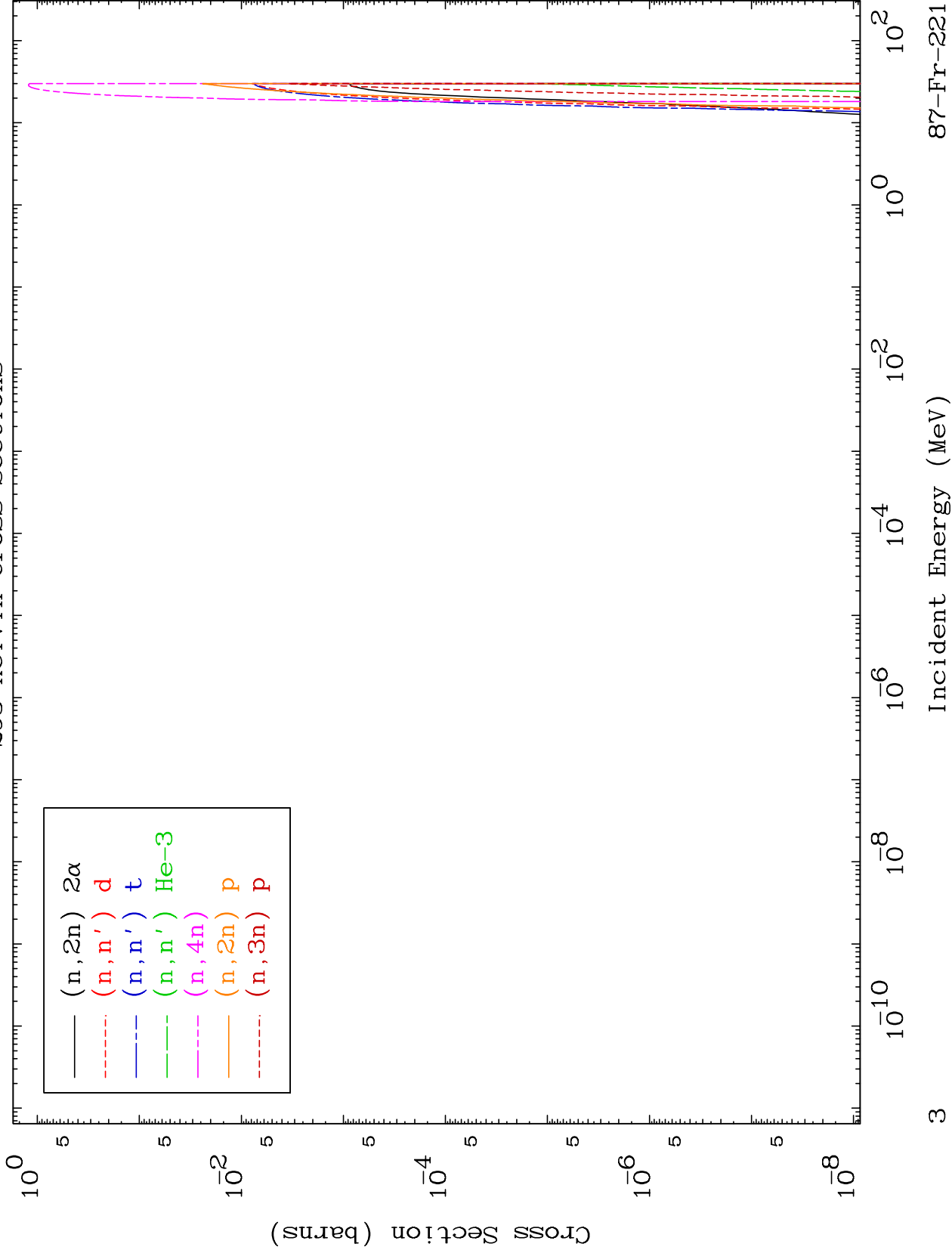
87-Fr-221



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Neutron Production
293 Kelvin Cross Sections

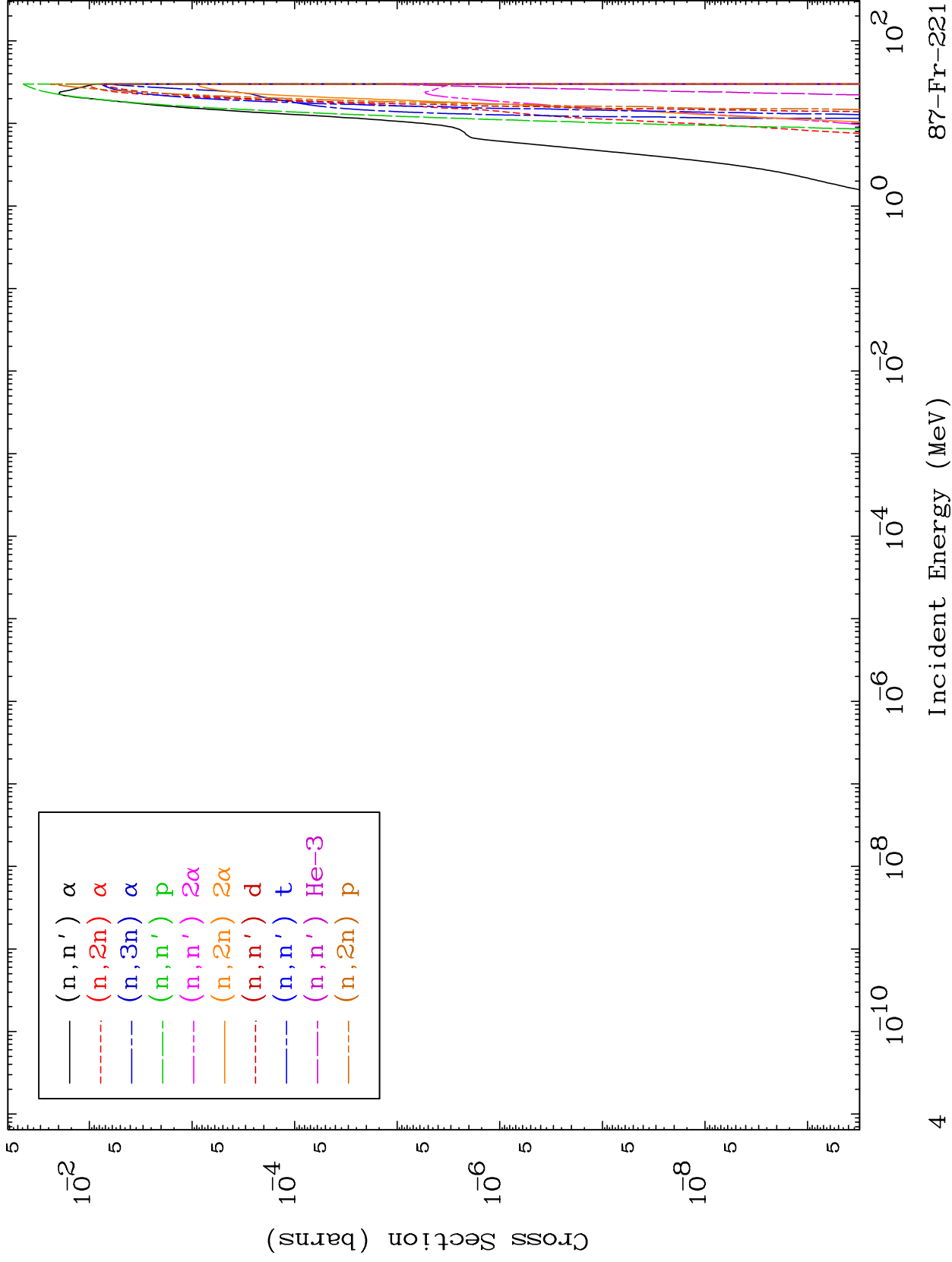
87-Fr-221



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Charged Particle
293 Kelvin Cross Sections

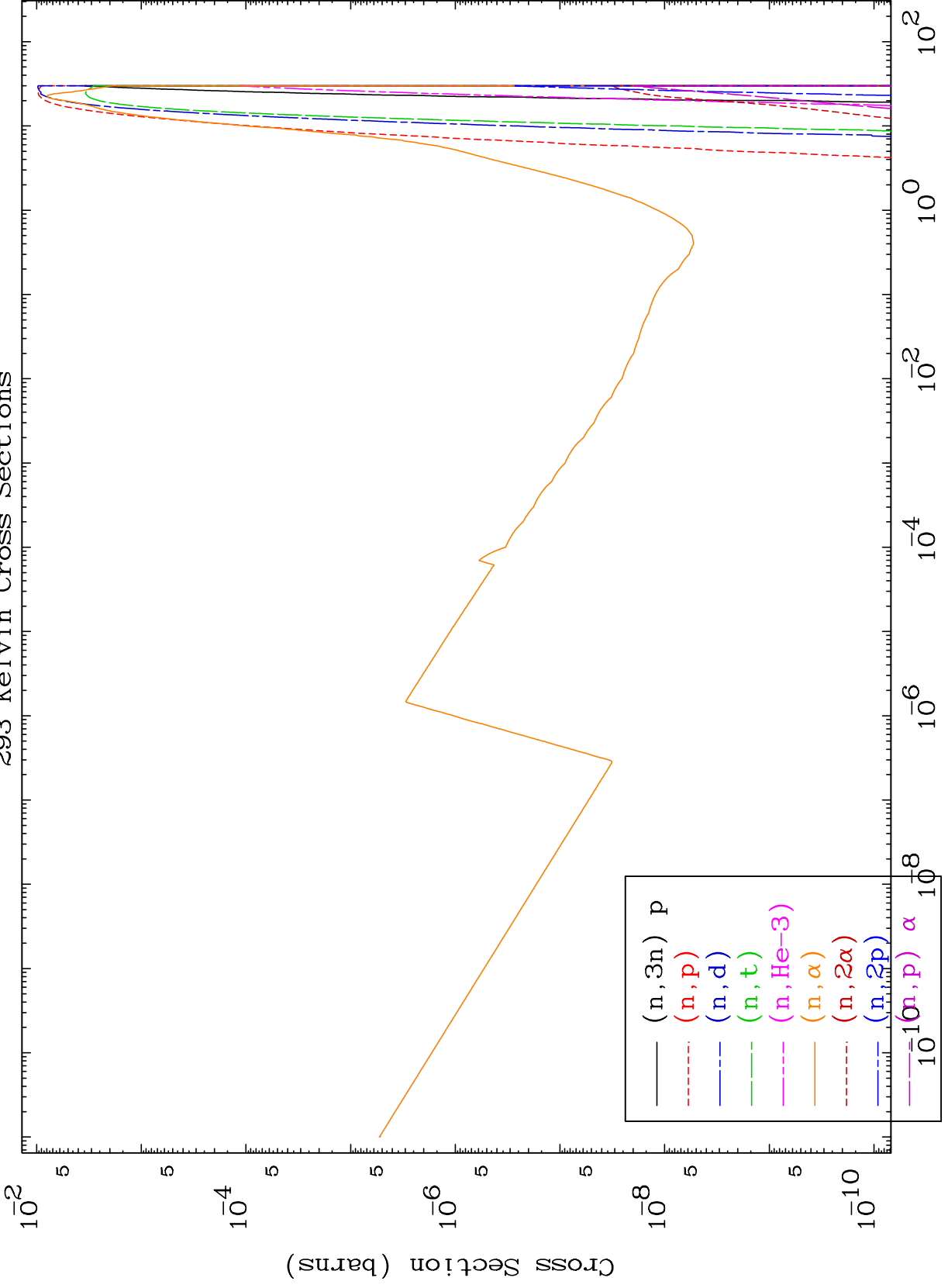
87-Fr-221



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Charged Particle
293 Kelvin Cross Sections

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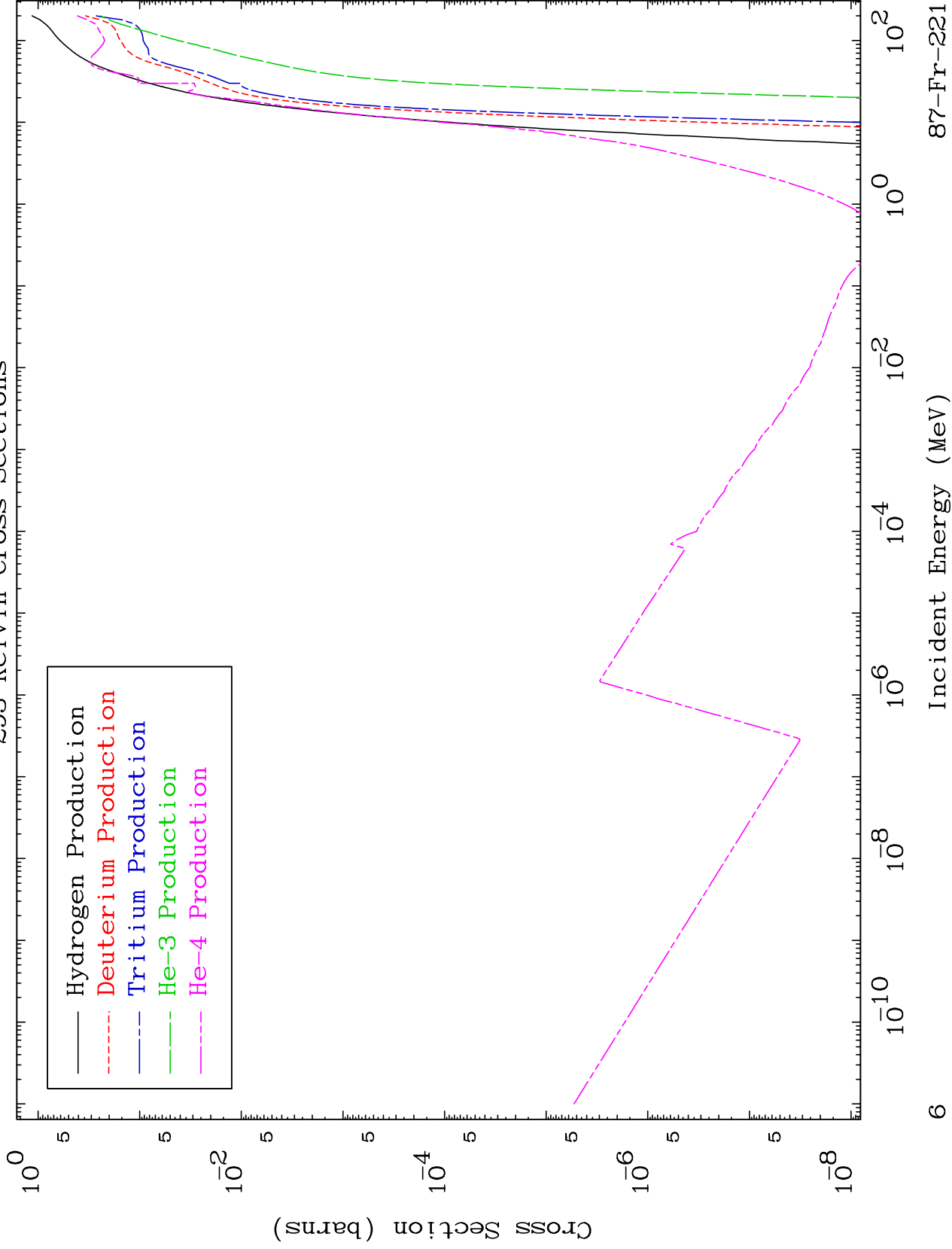
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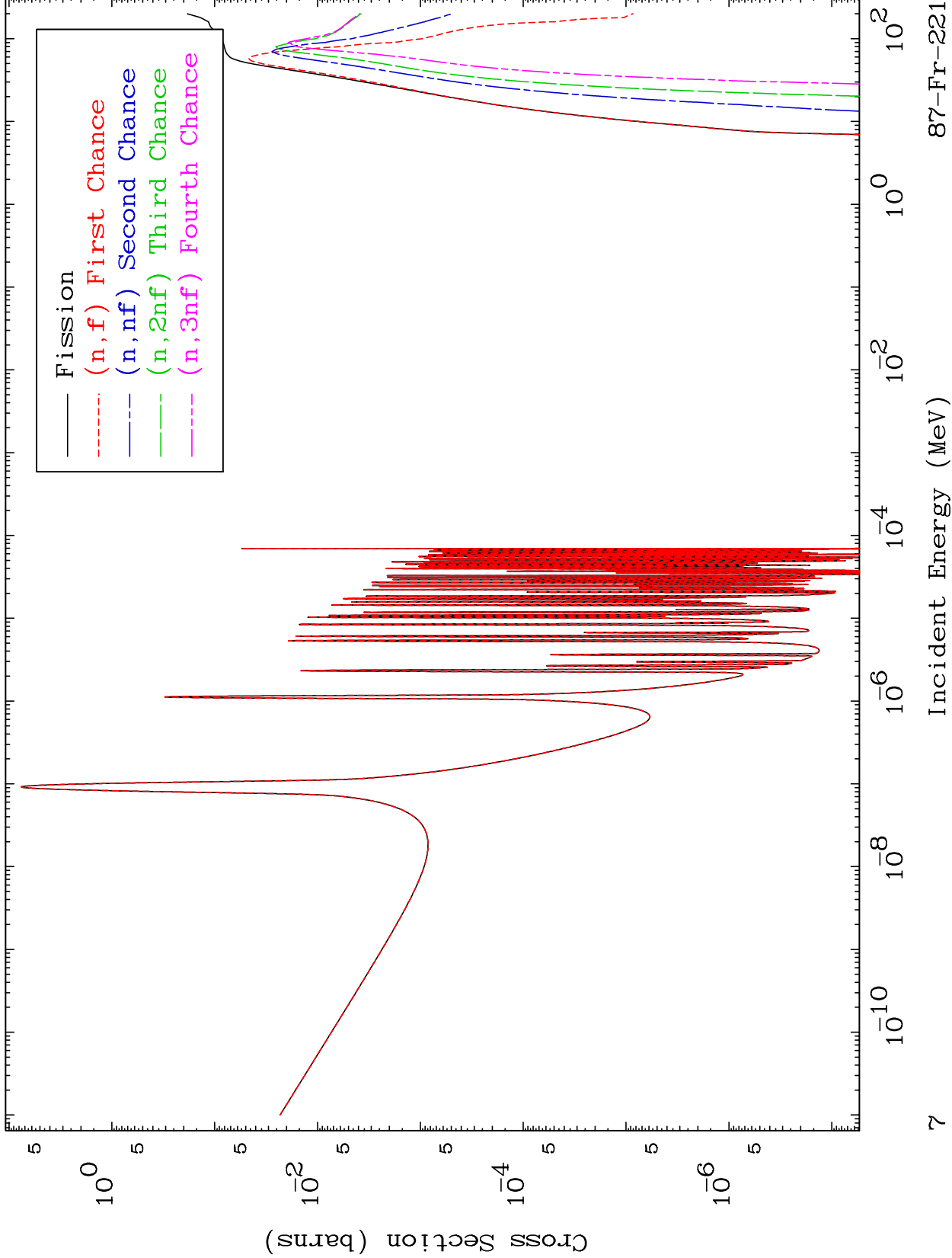
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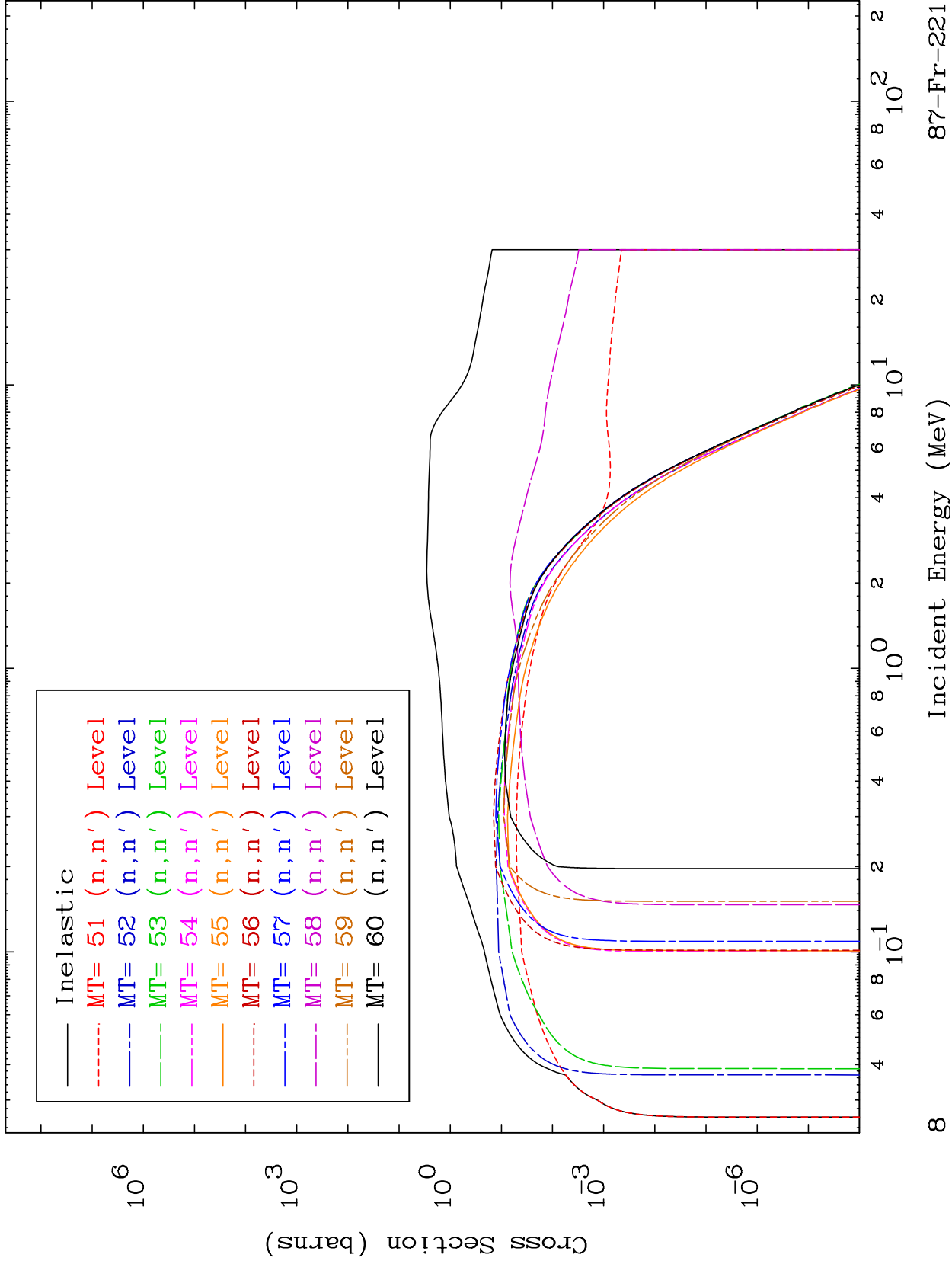
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Particle Production
293 Kelvin Cross Sections

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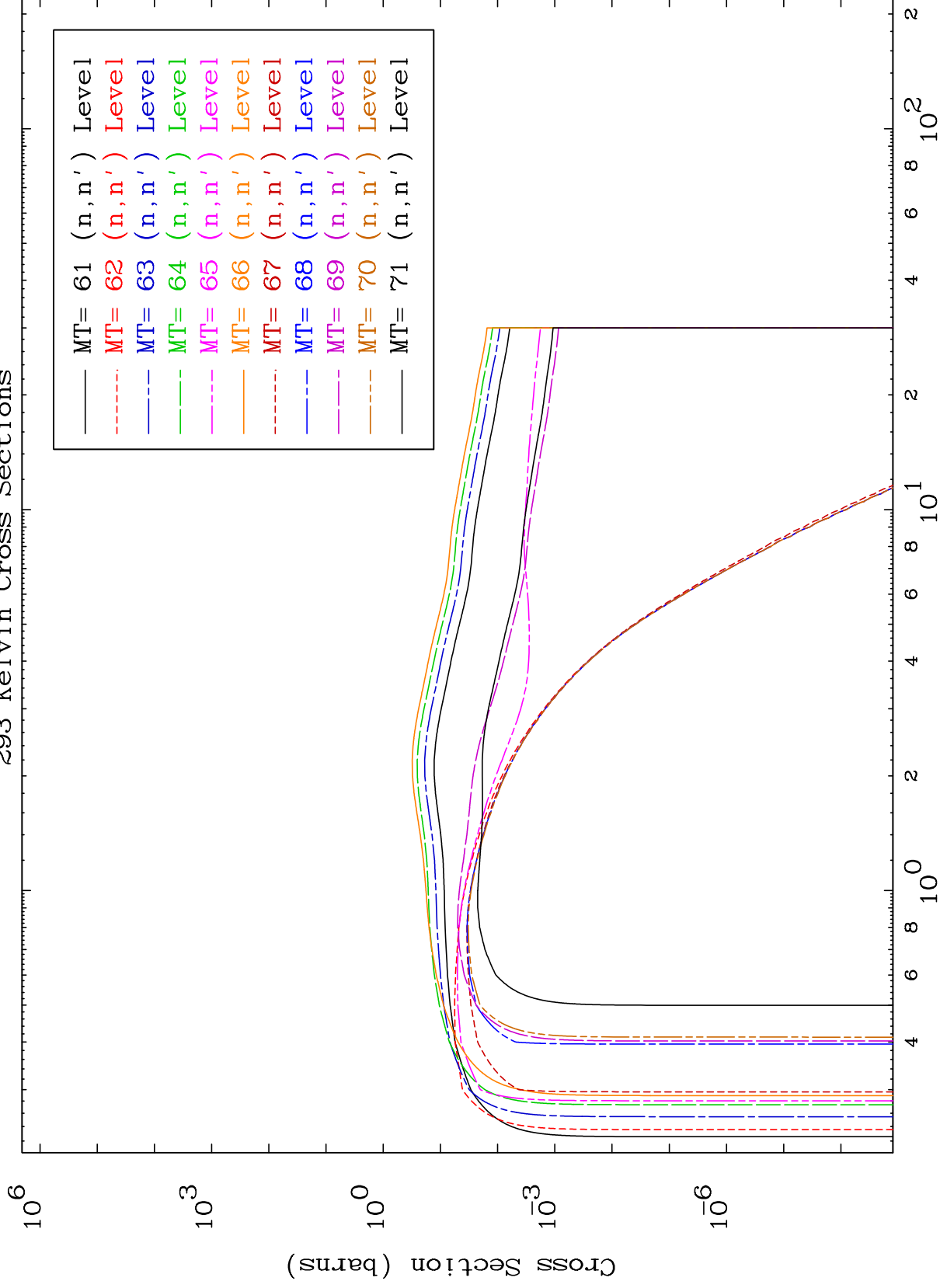


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(n,n') Level

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293 Kelvin Cross Sections

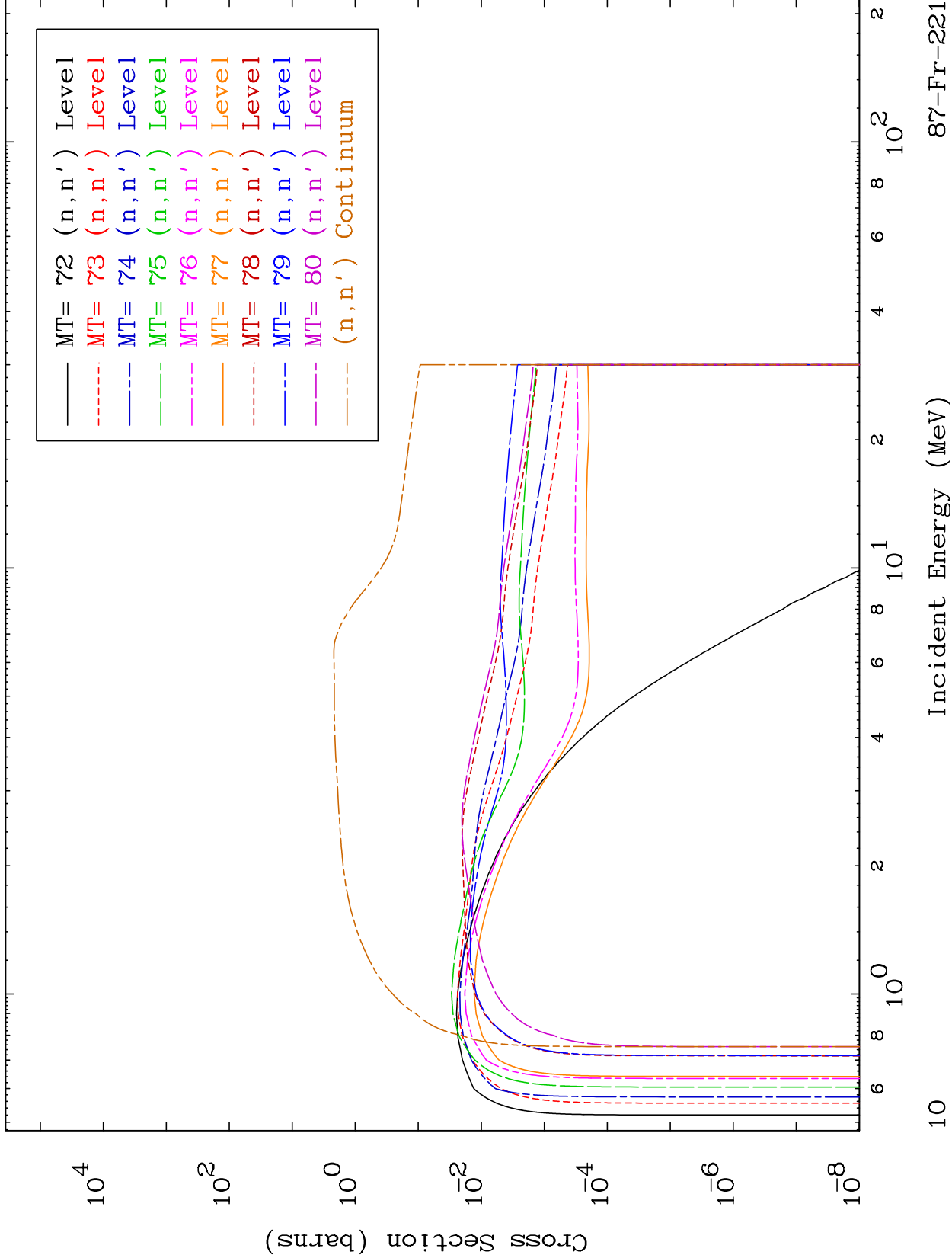


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(n,n') Level

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293 Kelvin Cross Sections



10

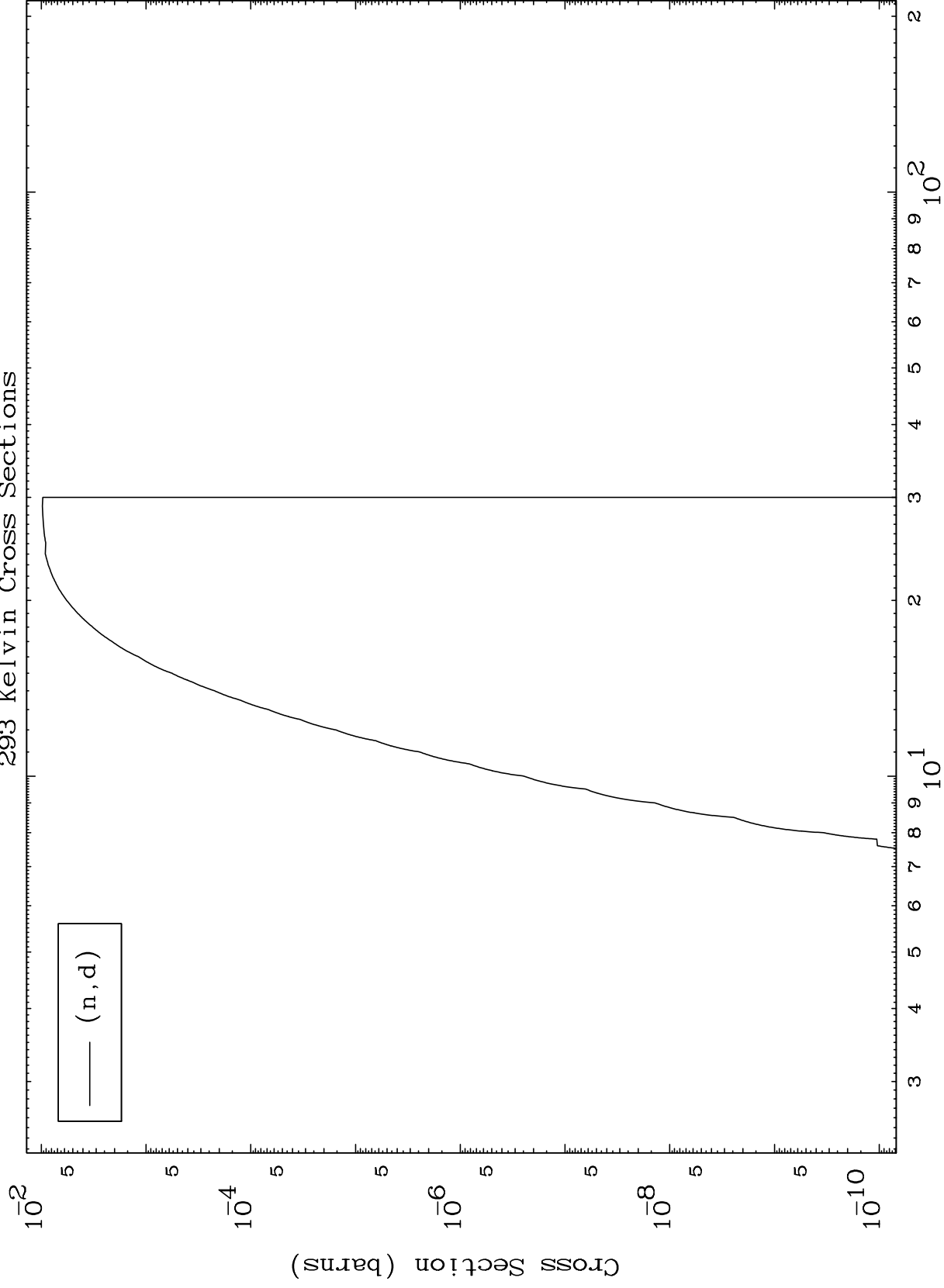
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

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12

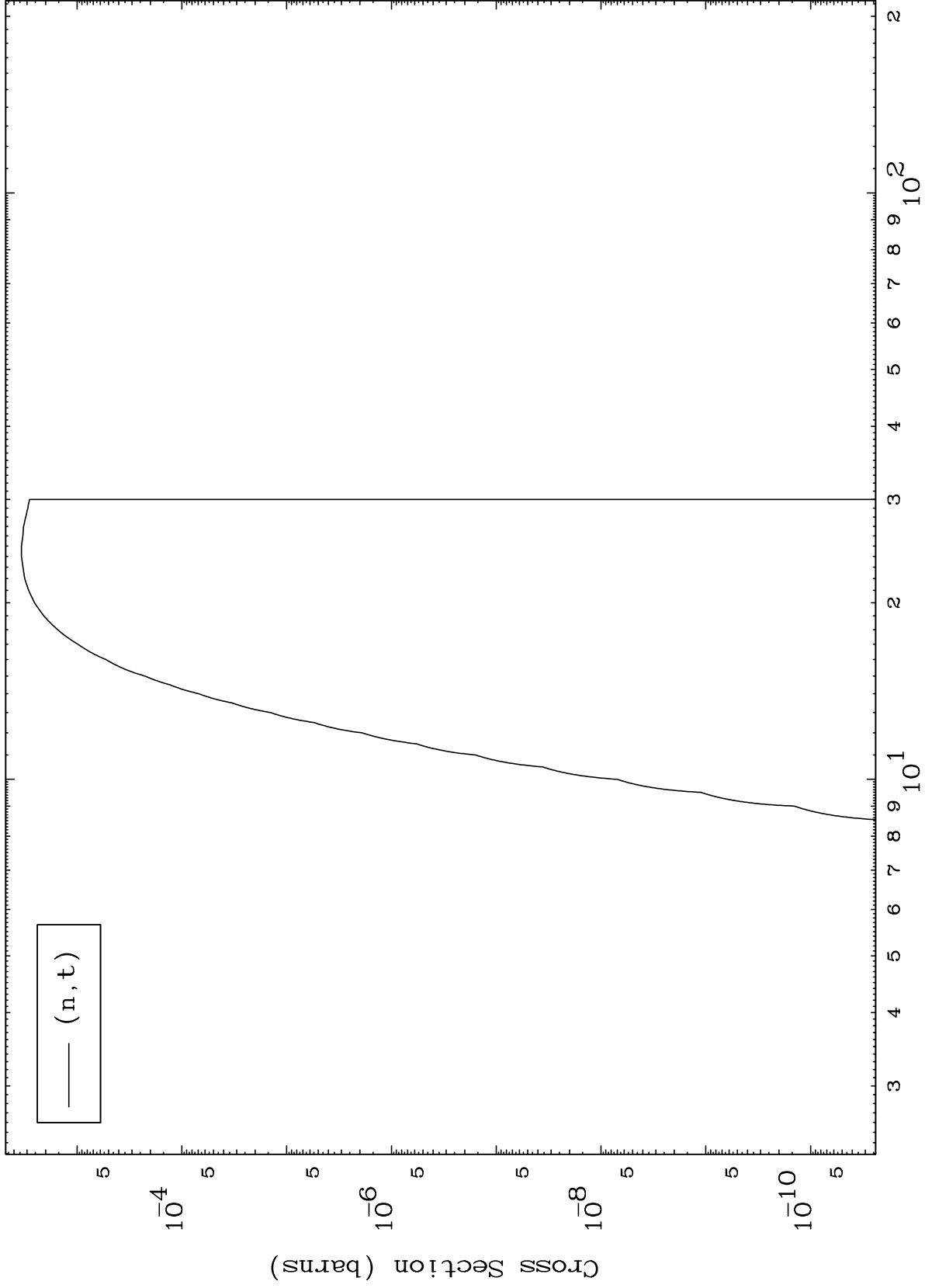
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

87-Fr-221



13

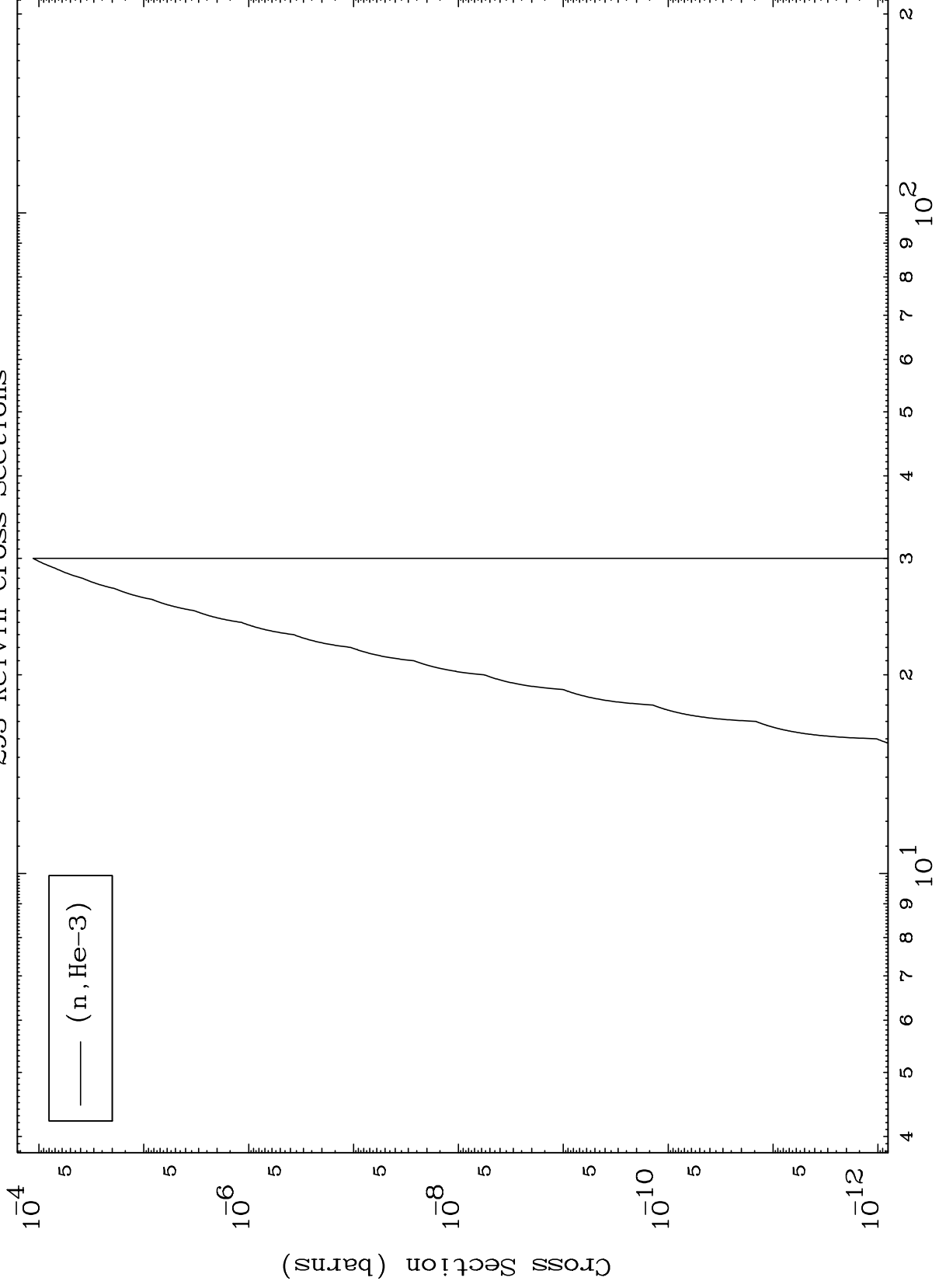
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

87-Fr-221

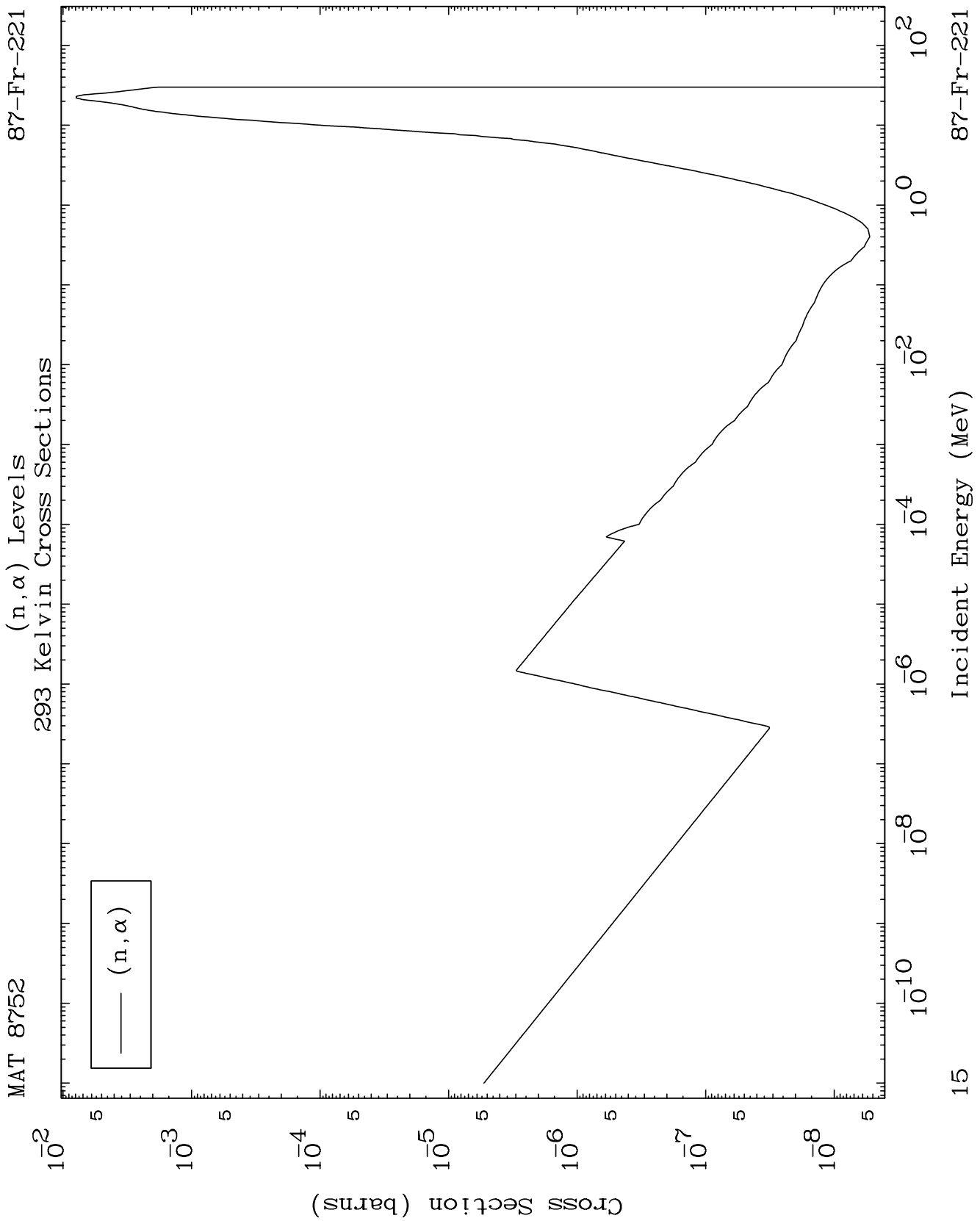


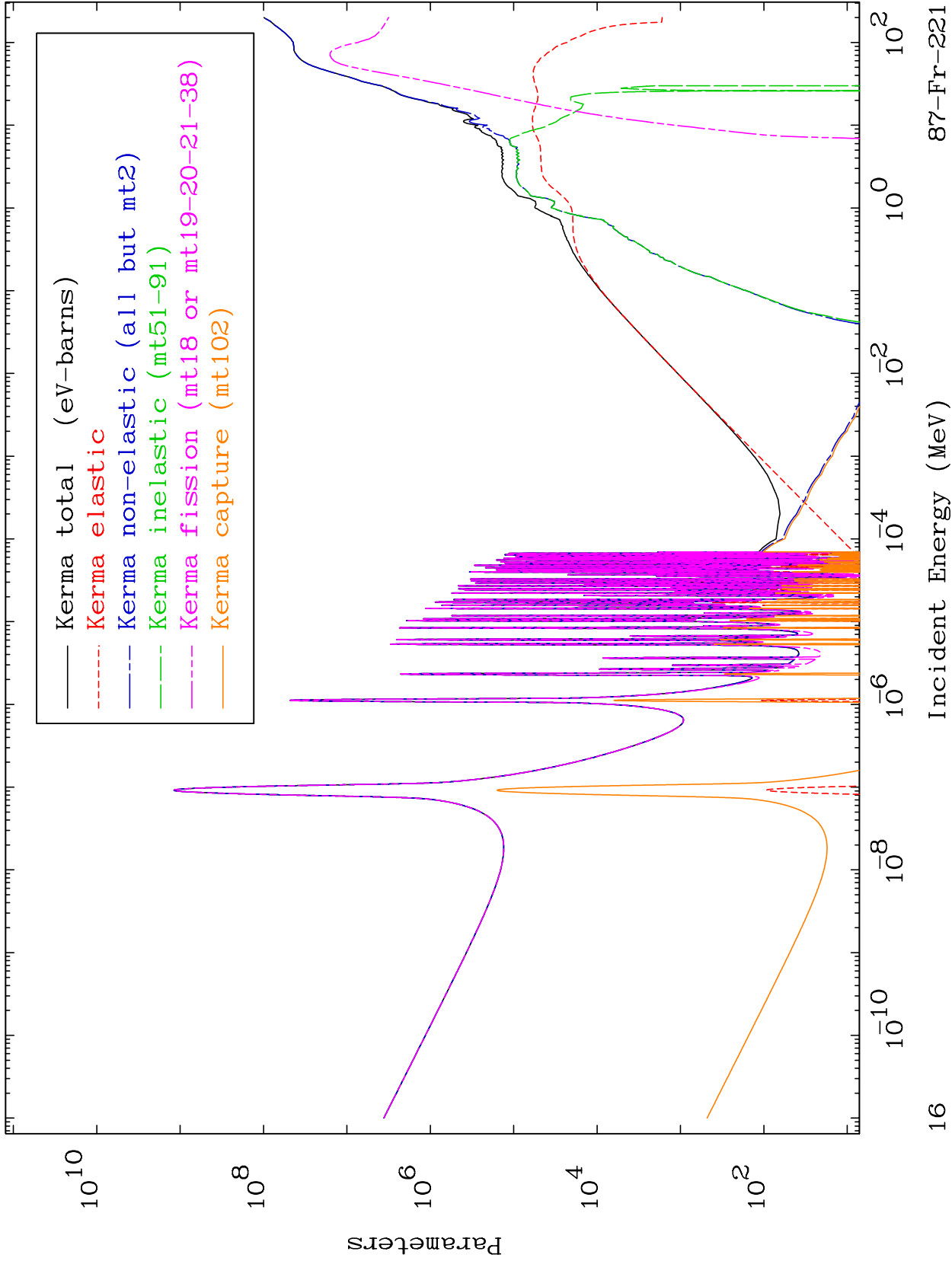
(n, He-3)

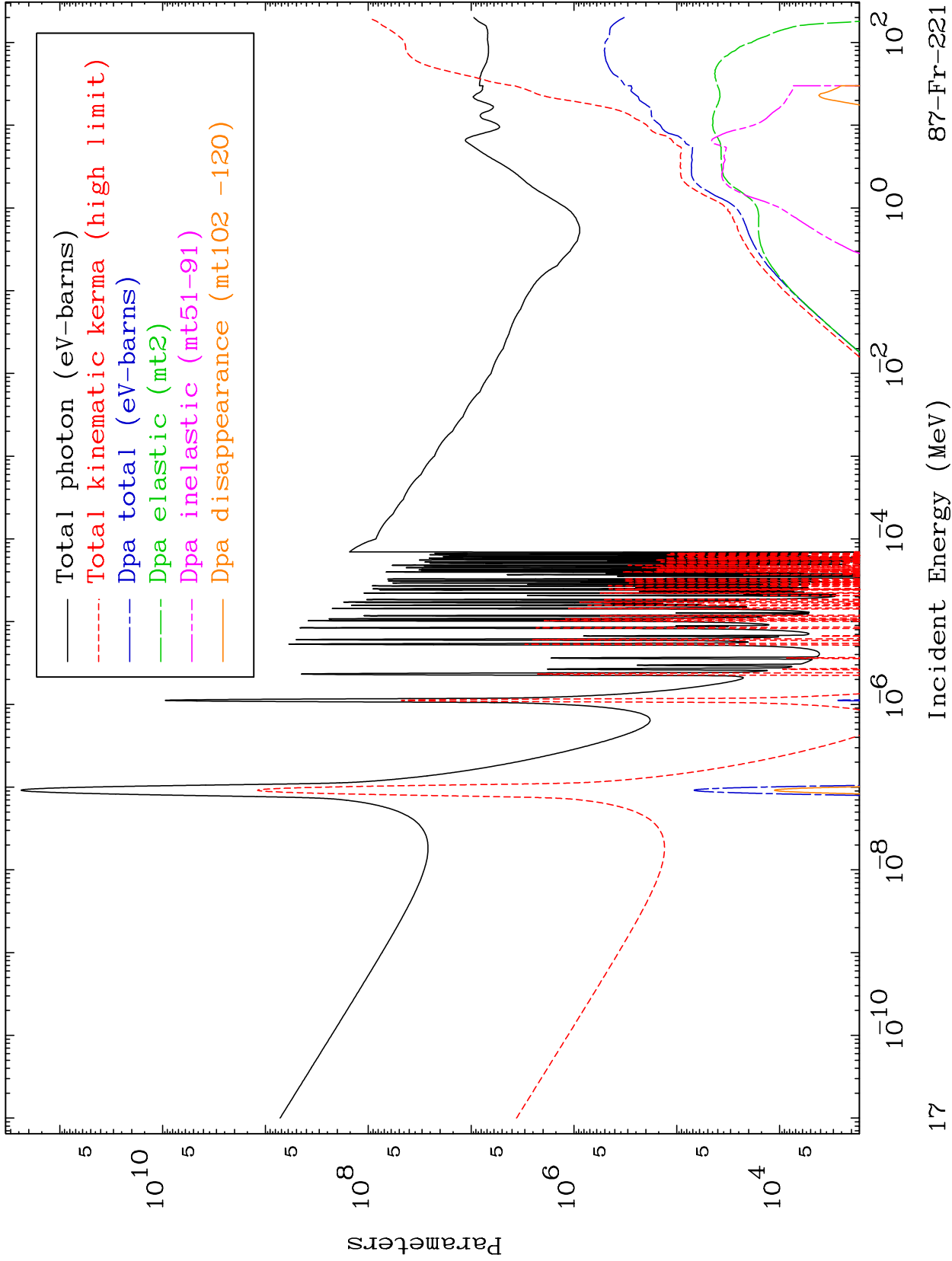
14

Incident Energy (MeV)

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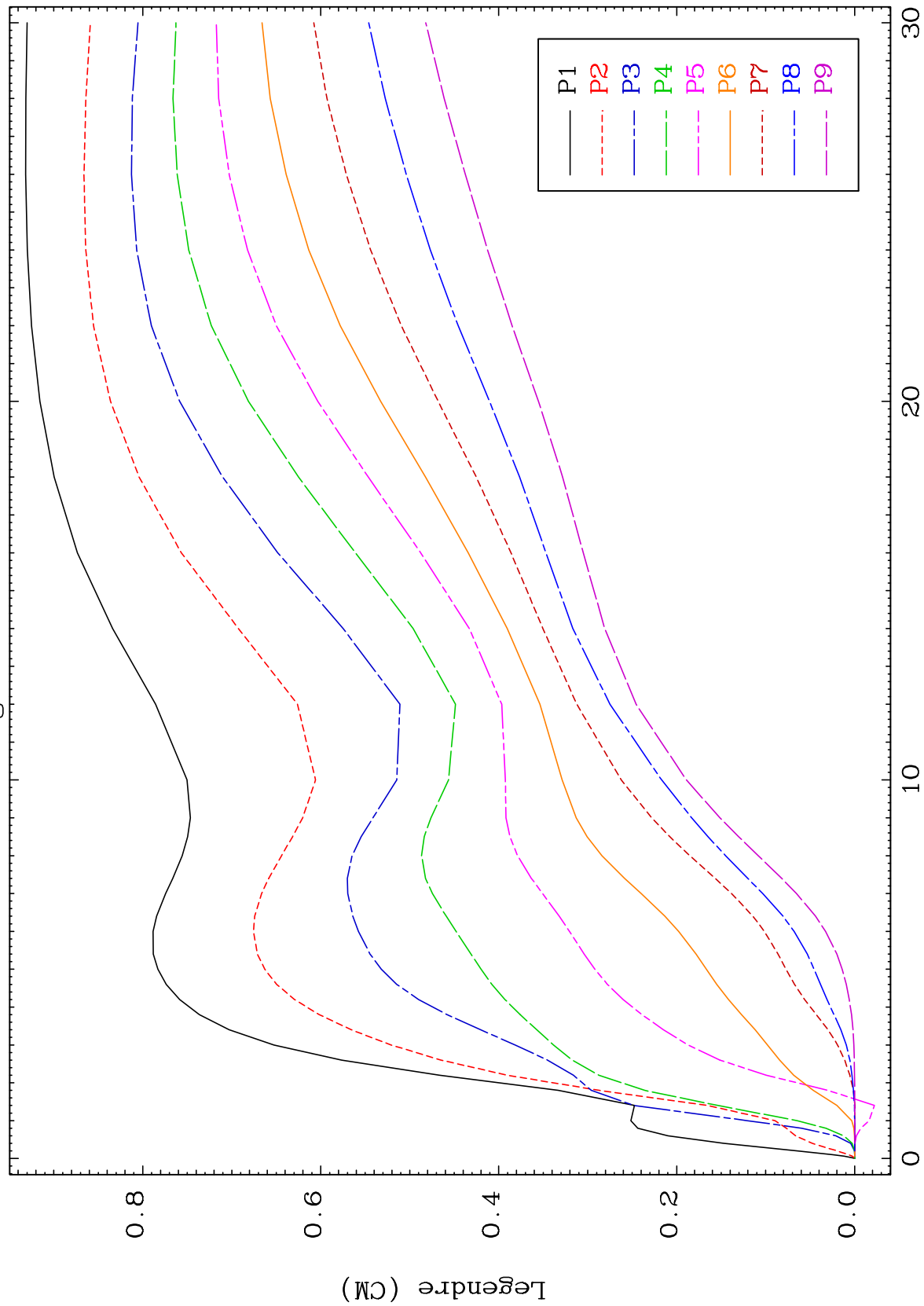




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Elastic Legendre Coefficients

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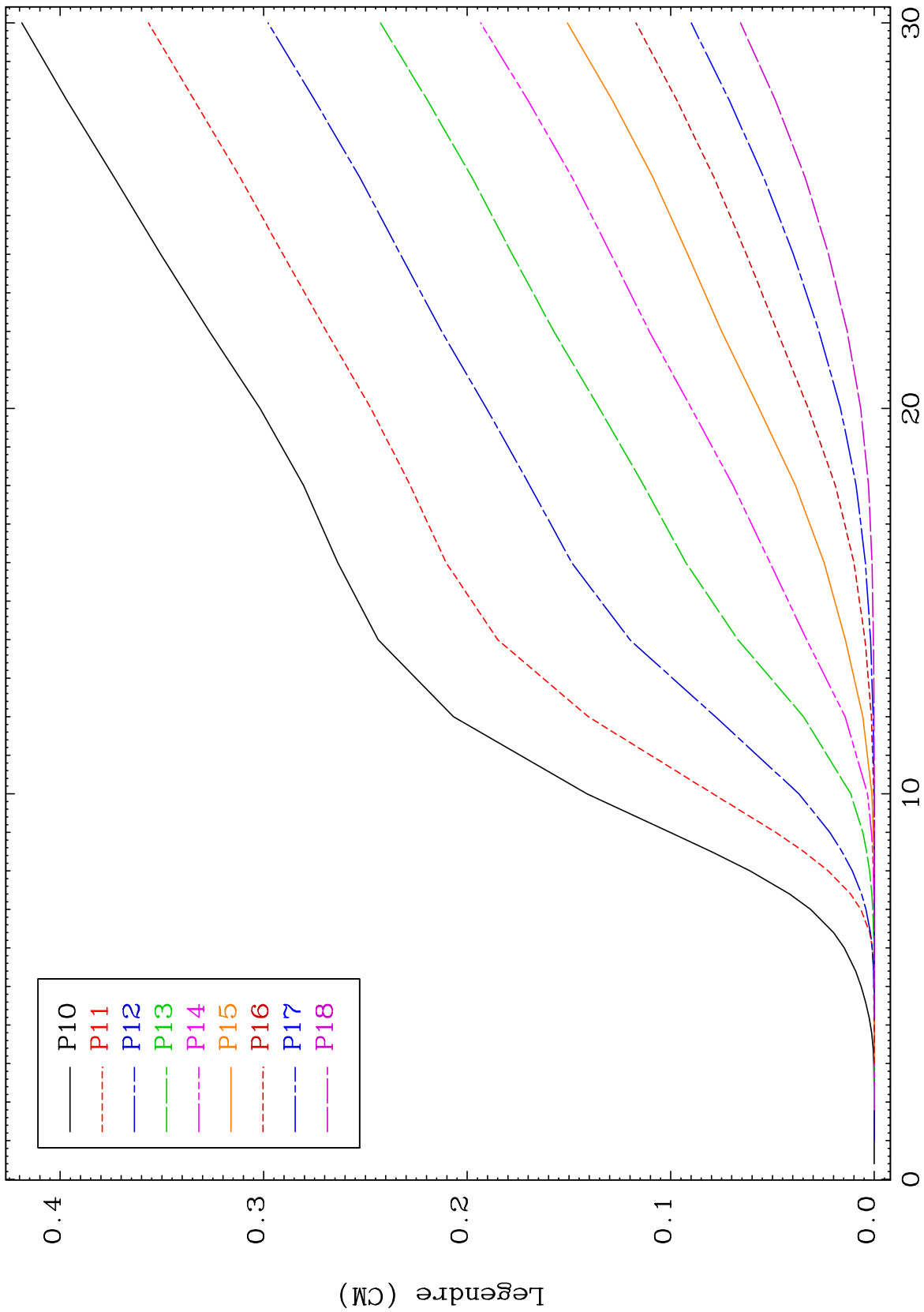
Incident Energy (MeV)

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MAT 8752

Elastic Legendre Coefficients

87-Fr-221



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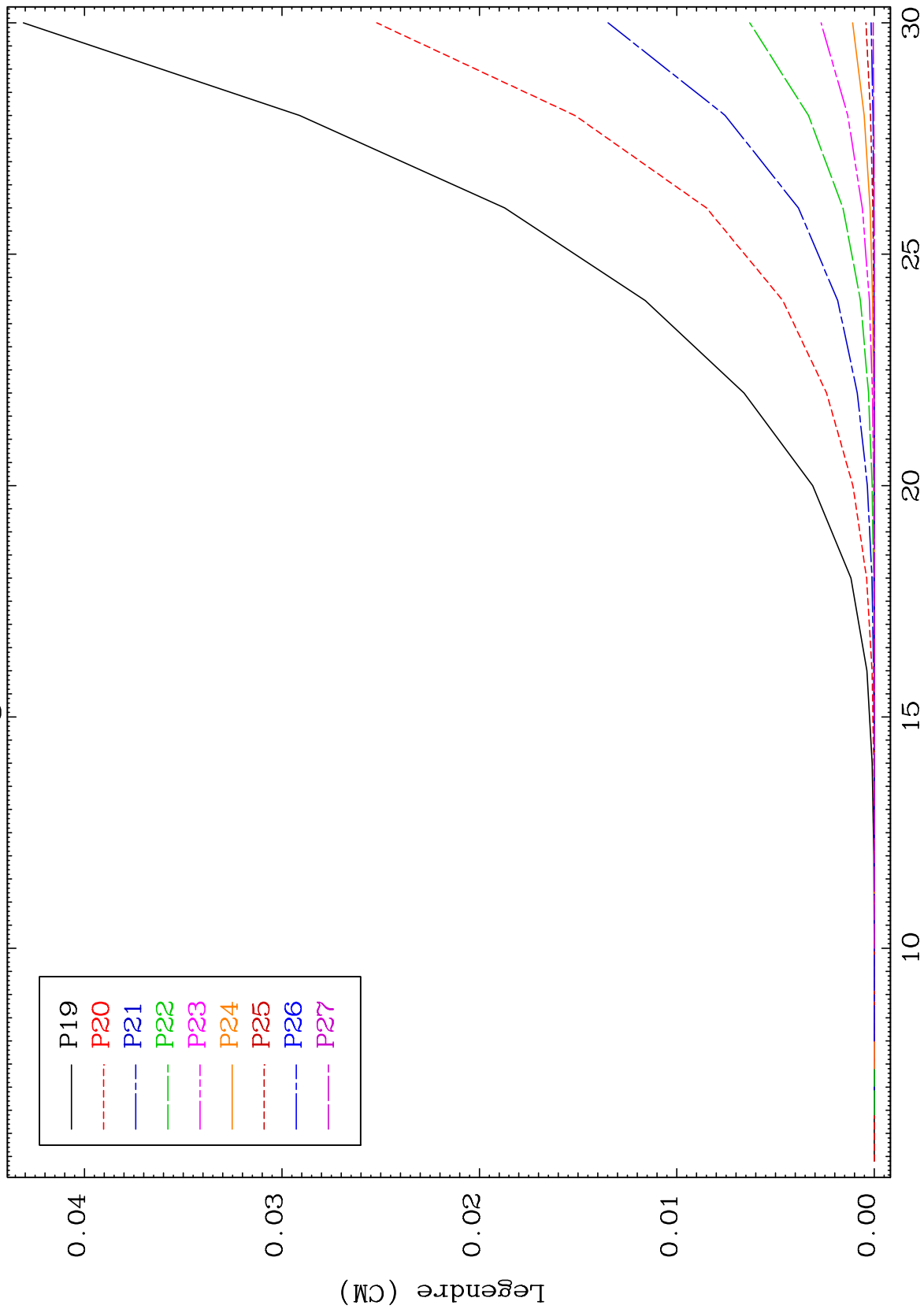
Incident Energy (MeV)

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Elastic Legendre Coefficients

87-Fr-221



20

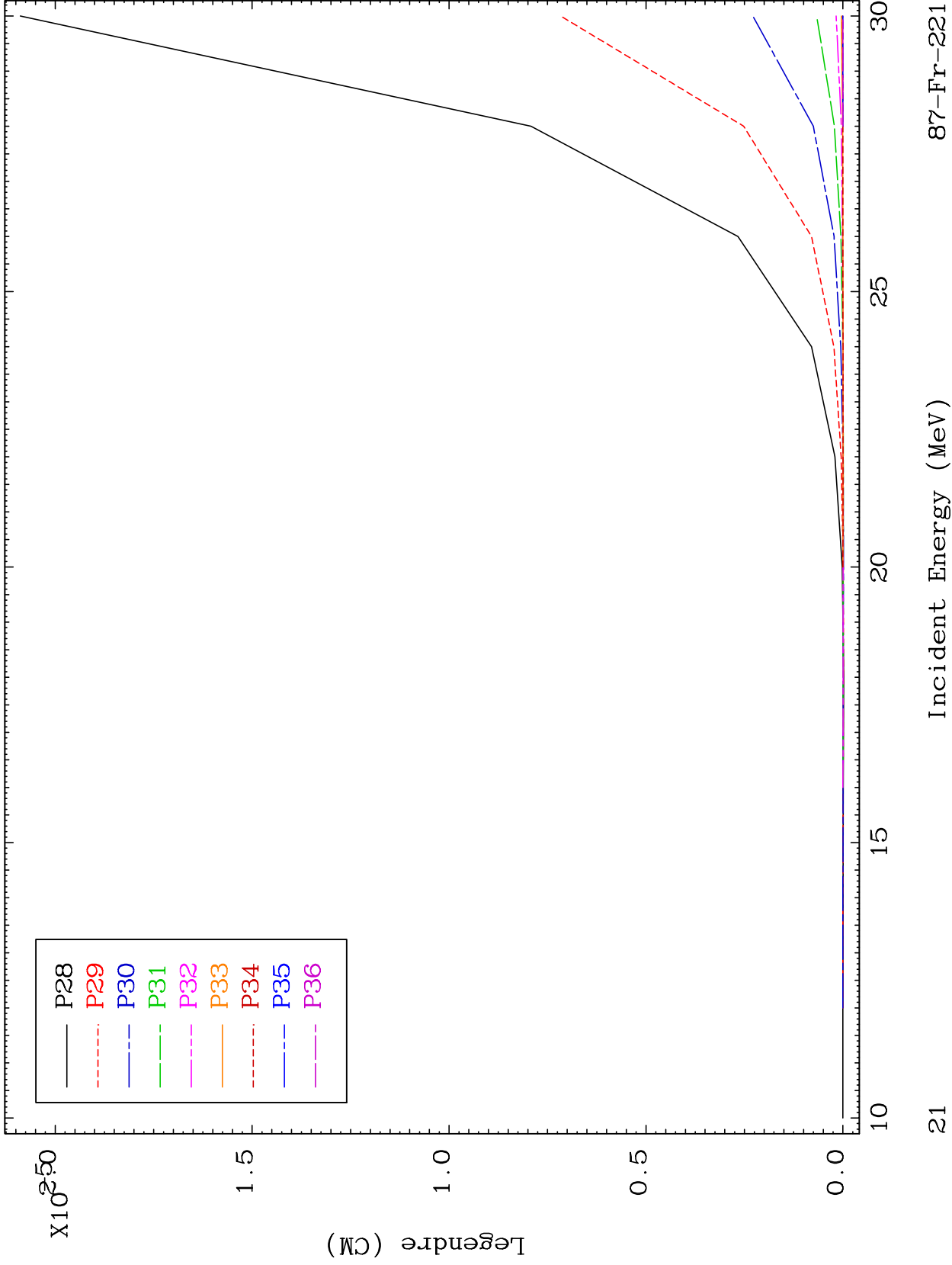
Incident Energy (MeV)

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Elastic
Legendre Coefficients

87-Fr-221

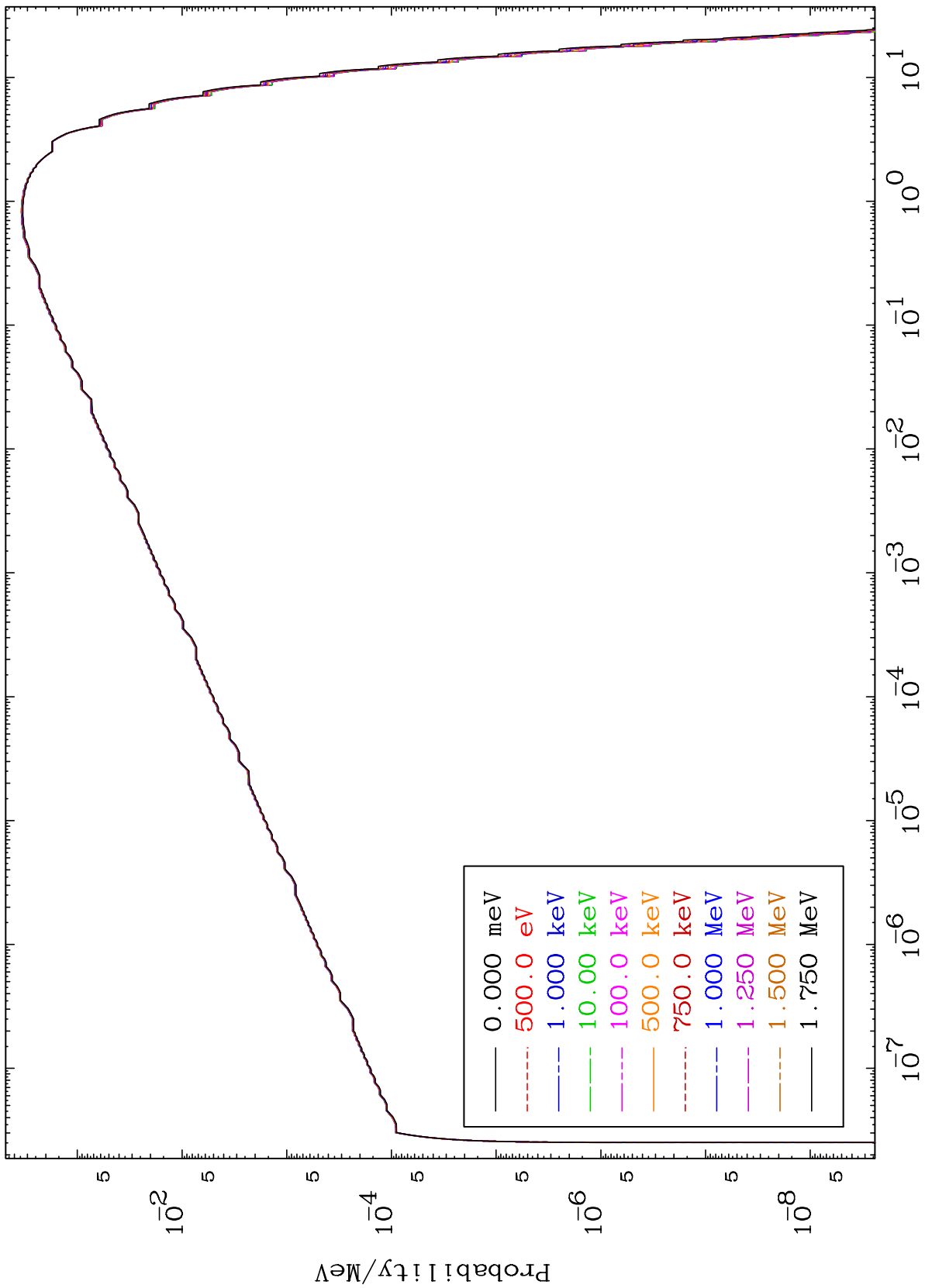


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Incident Energy (MeV)

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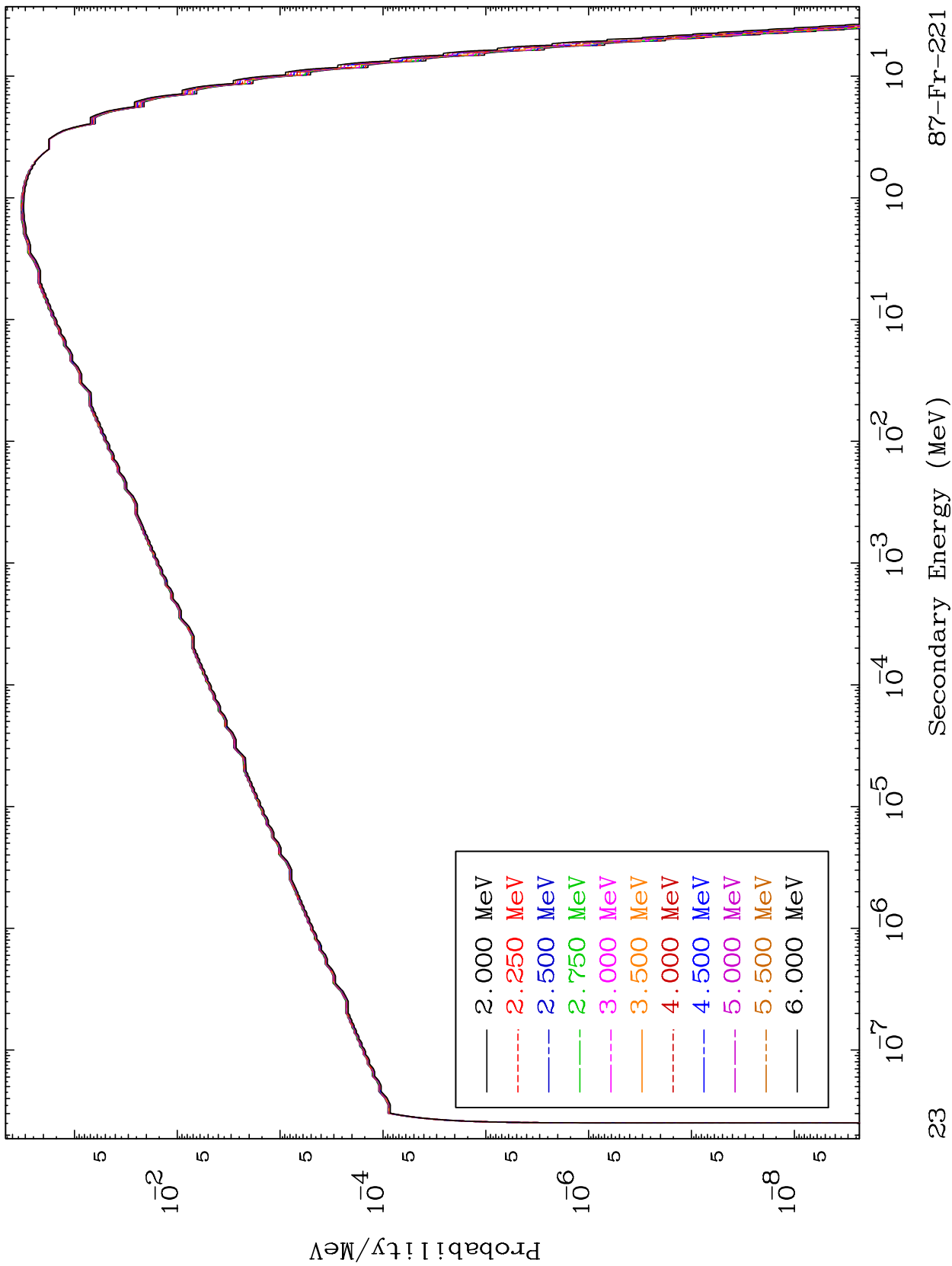
Fission
Energy Distribution



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Fission Energy Distribution

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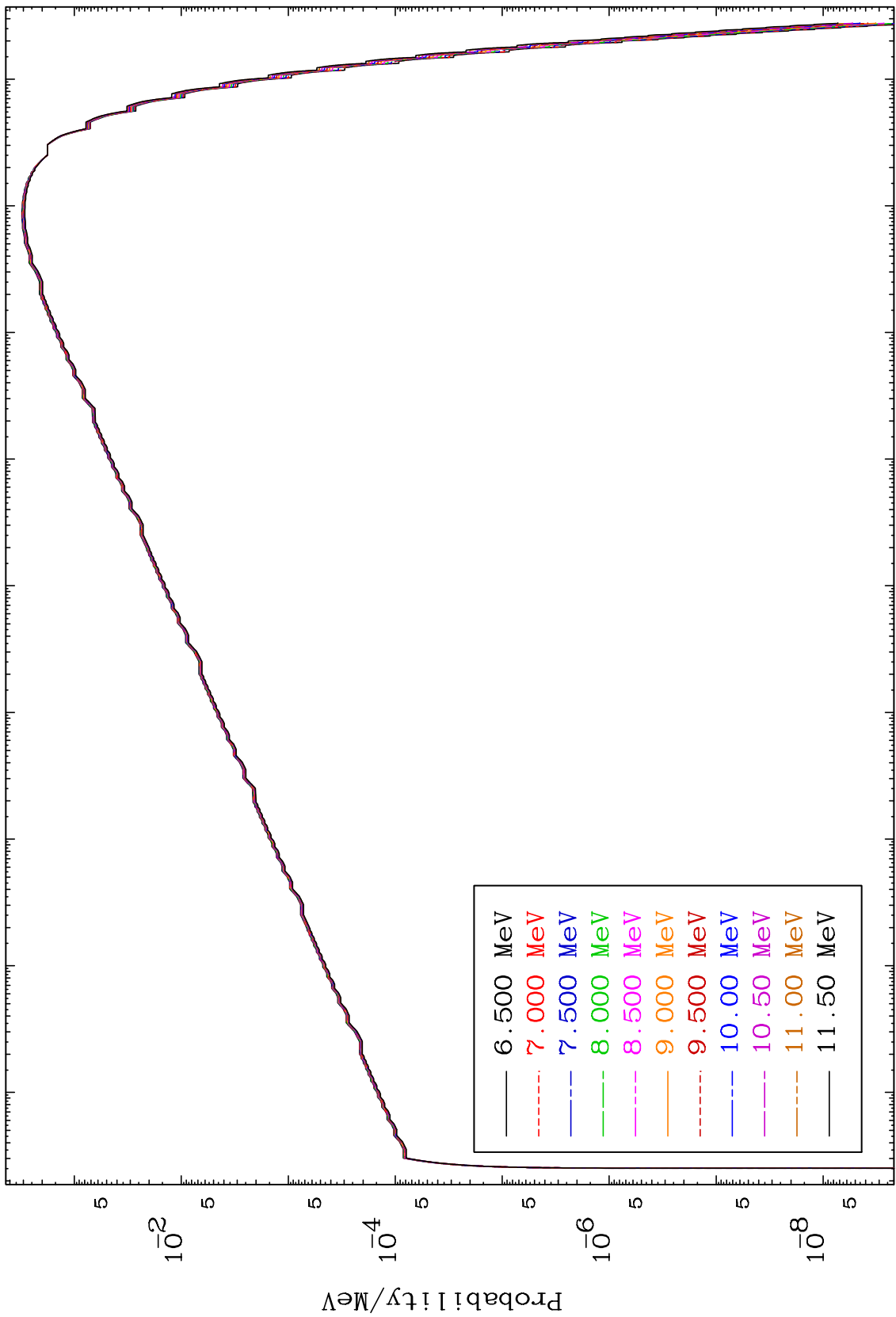


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Fission
Energy Distribution

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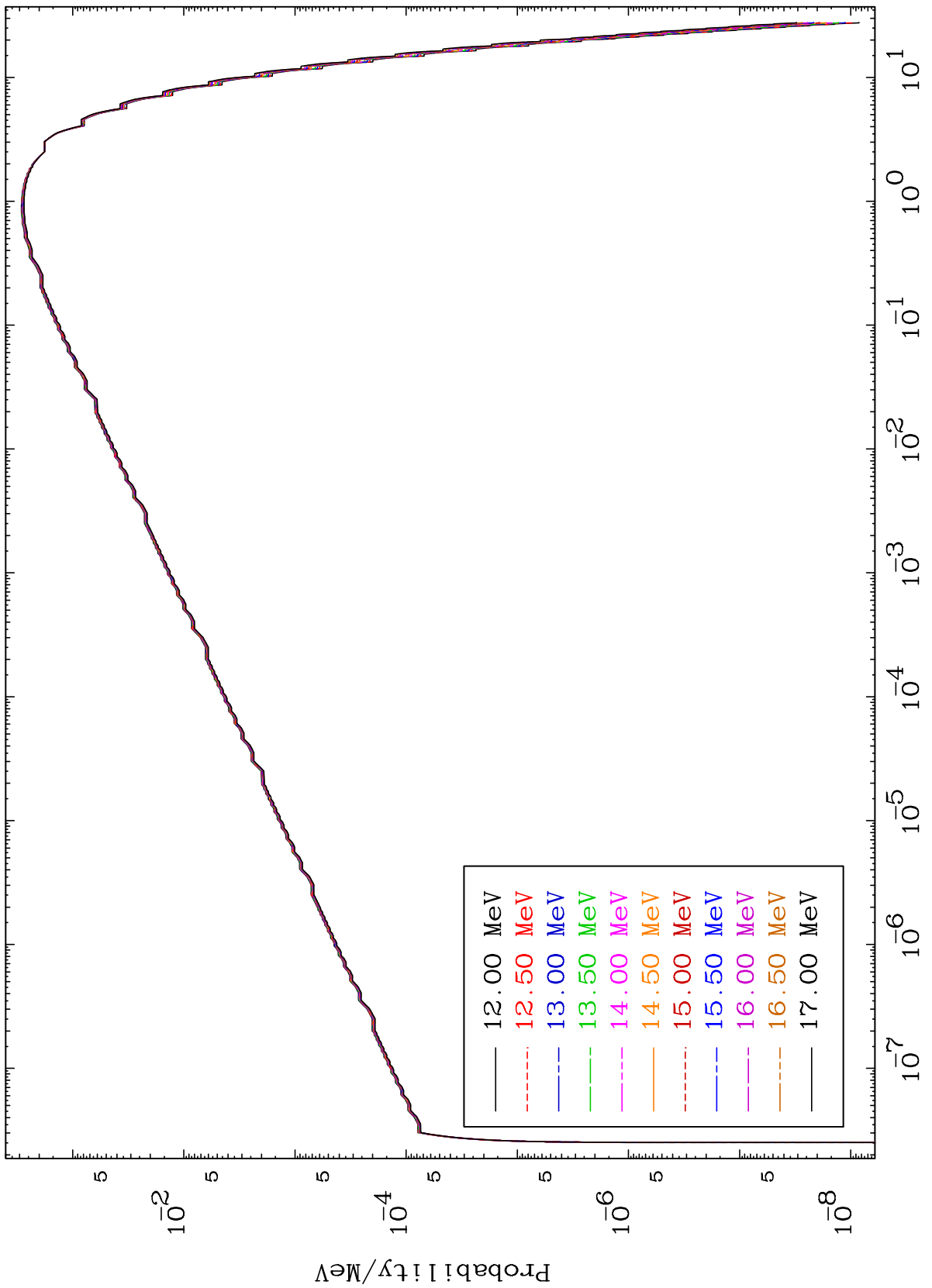


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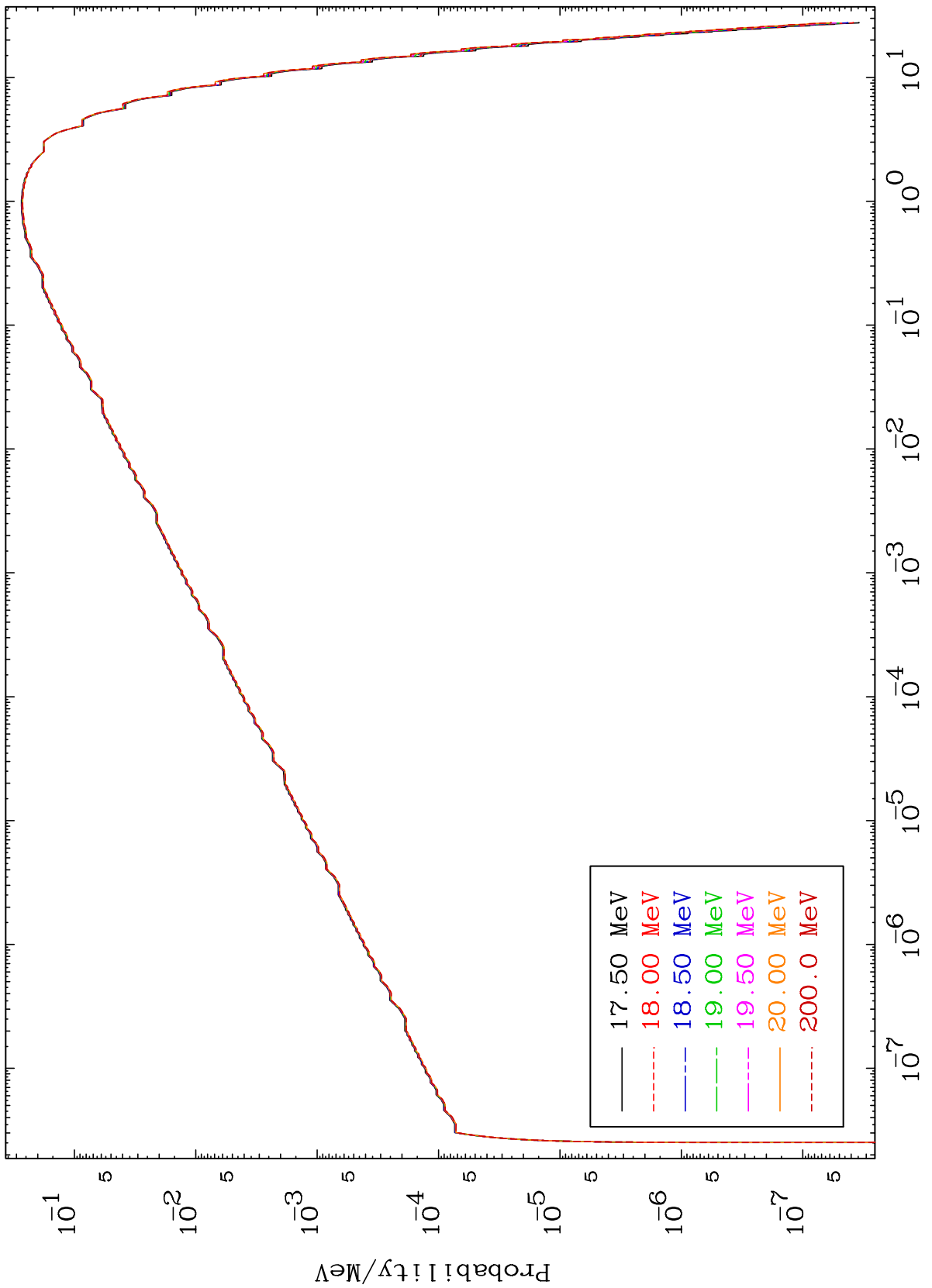
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Fission Energy Distribution

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Fission
Energy Distribution

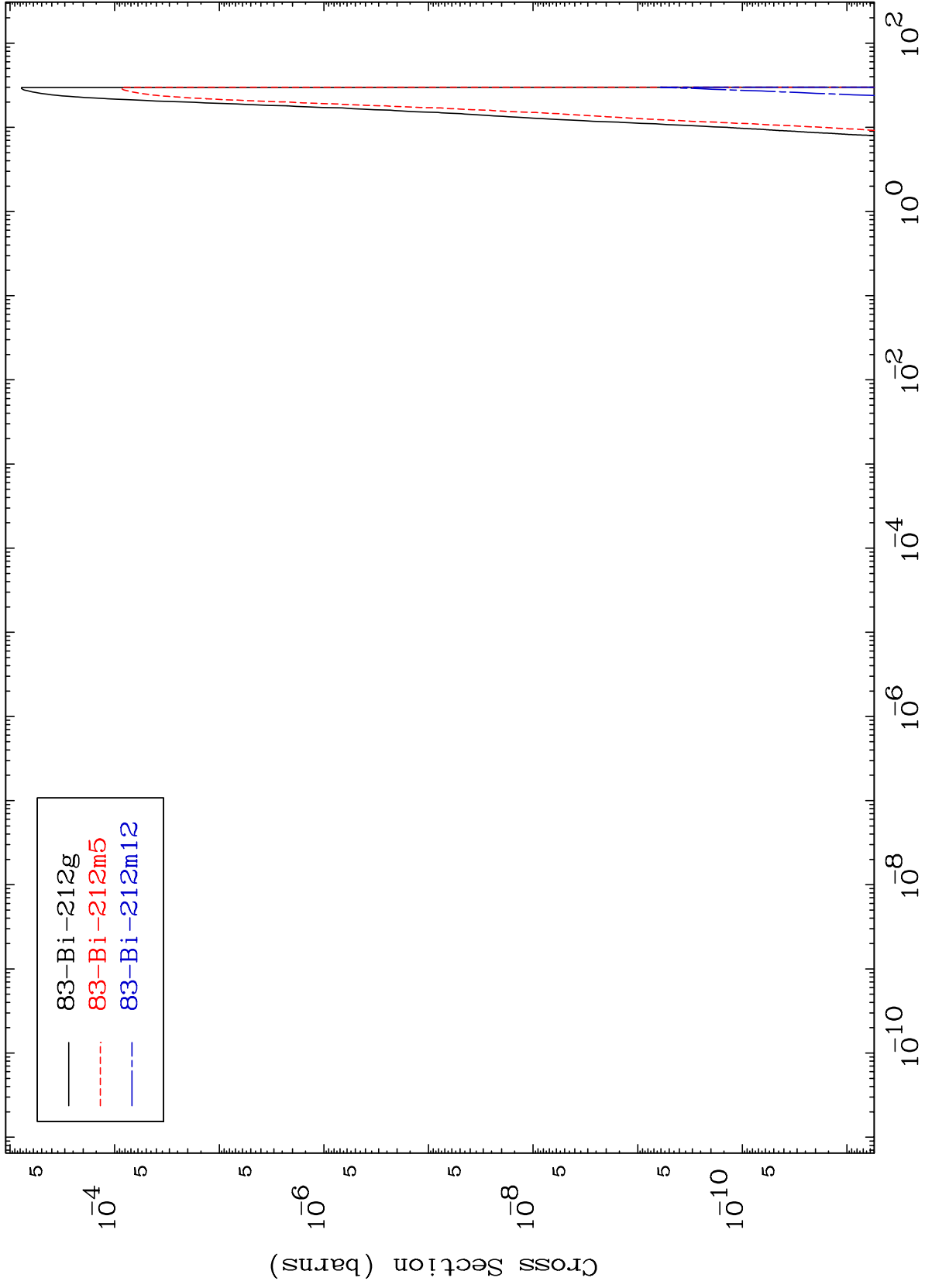


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(n,2n) 2 α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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