

Program EVALPLOT  
(Version 2017-1)

by

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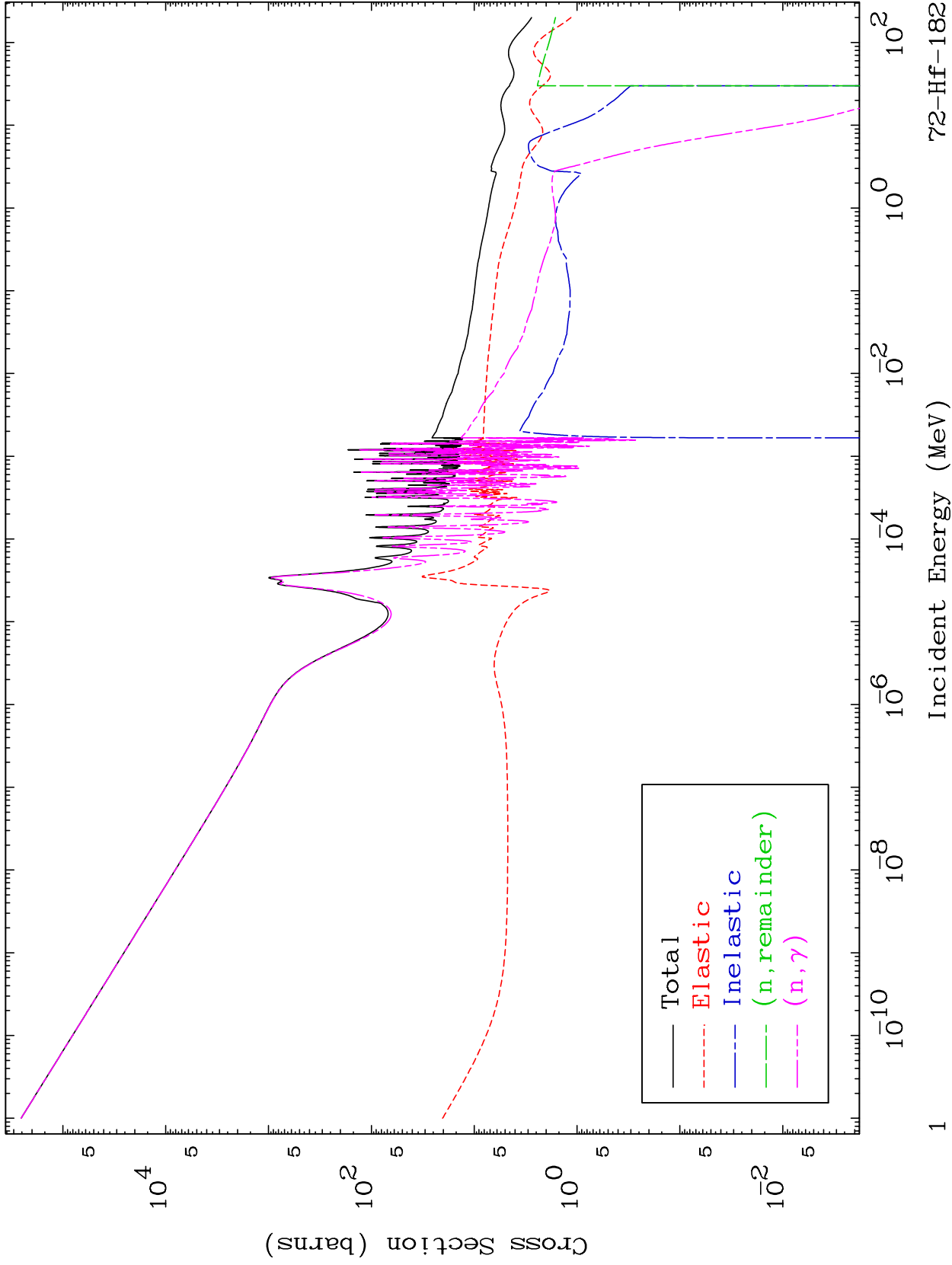
Press Mouse Button to Start

MAT 7250

Major

293 Kelvin Cross Sections

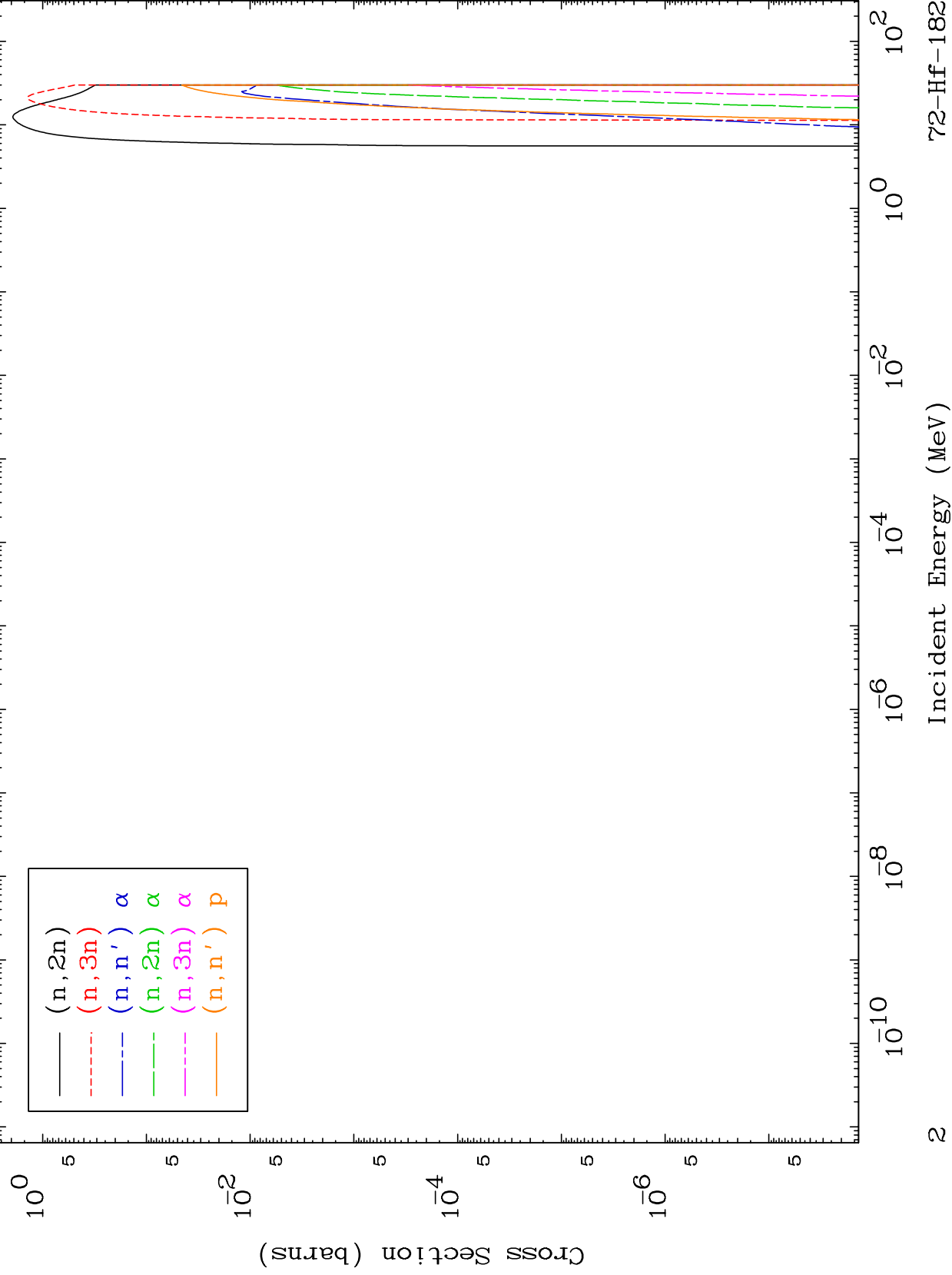
72-Hf-182

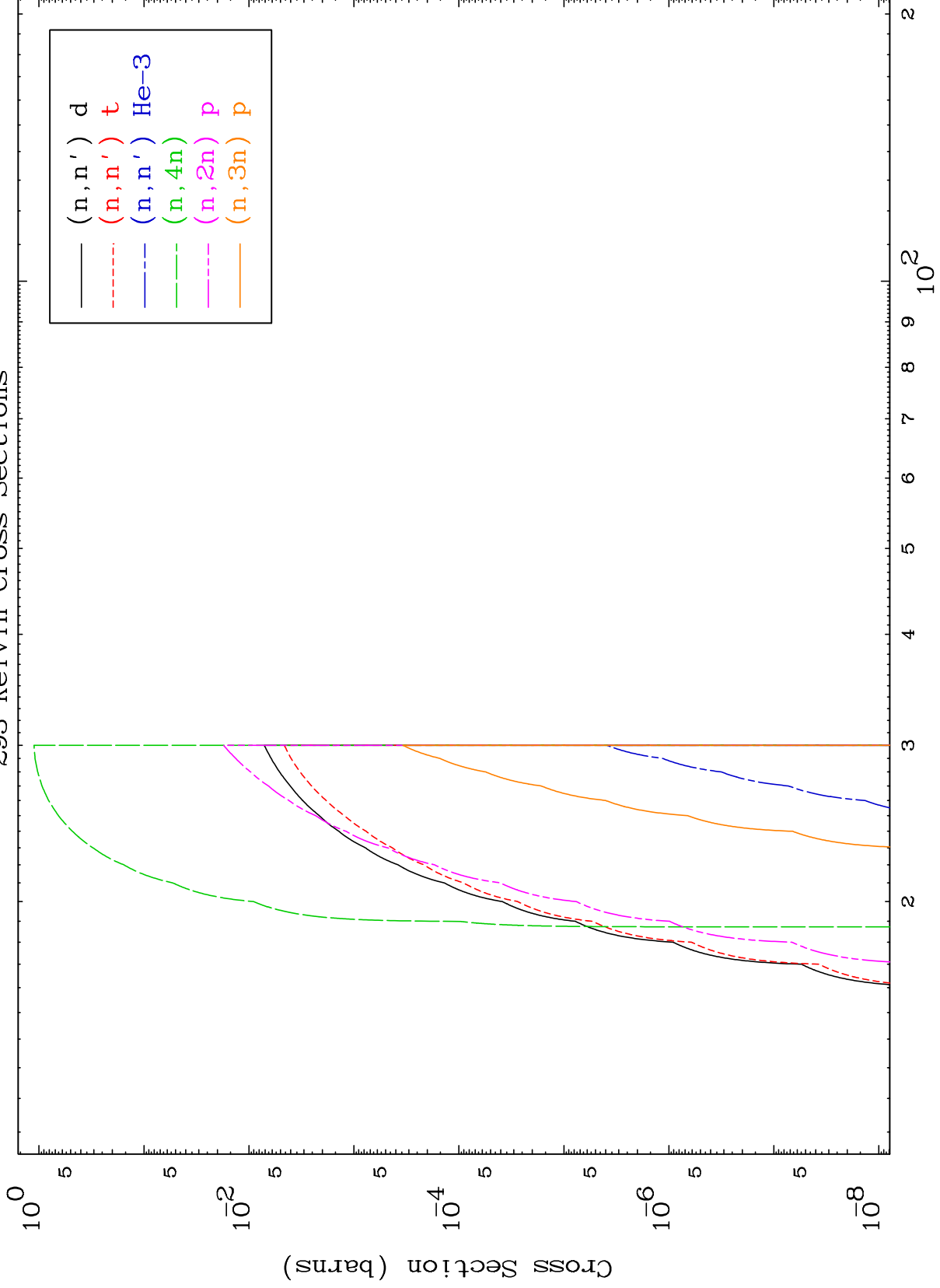


MAT 7250

Neutron Production  
293 Kelvin Cross Sections

72-Hf-182

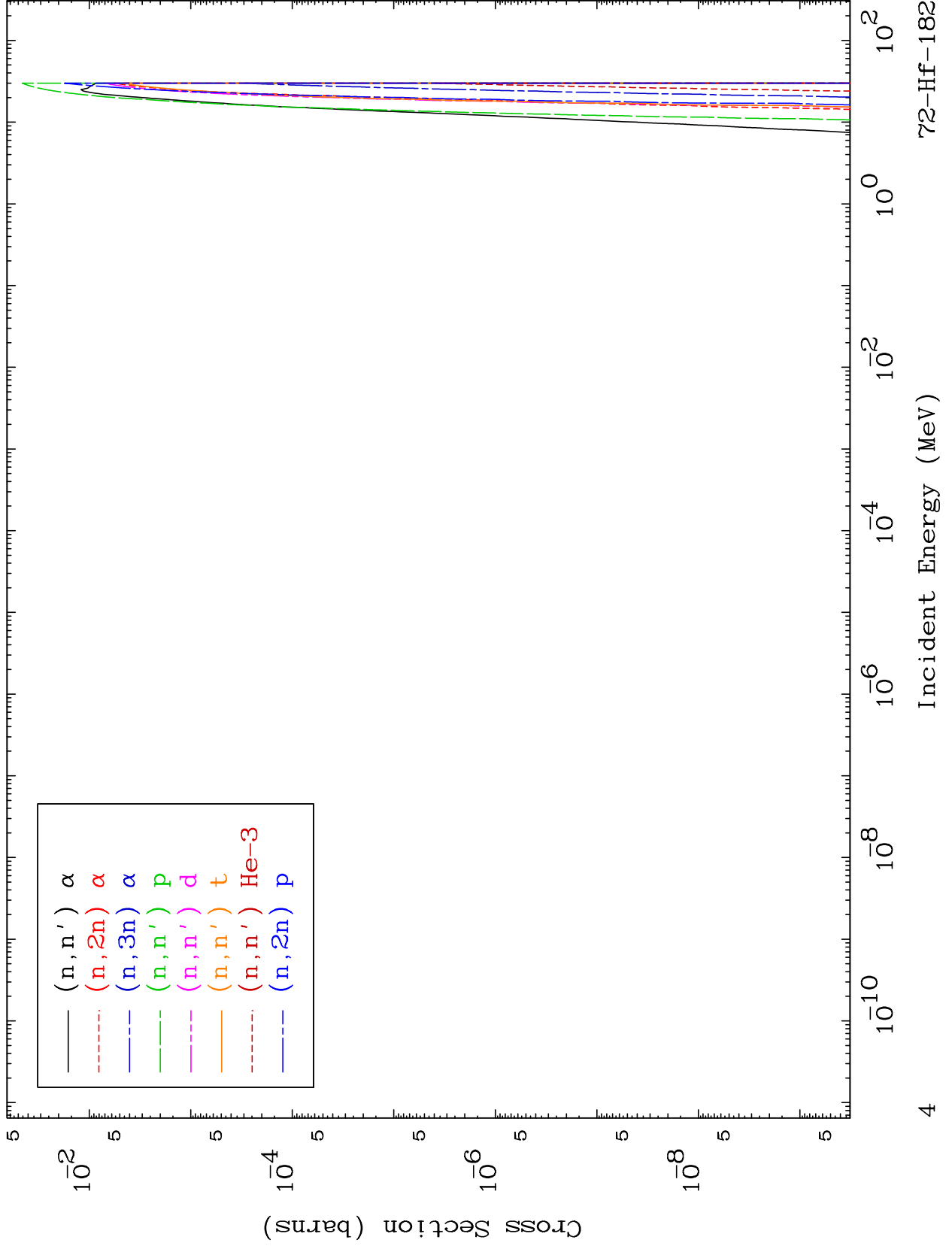




MAT 7250

Charged Particle  
293 Kelvin Cross Sections

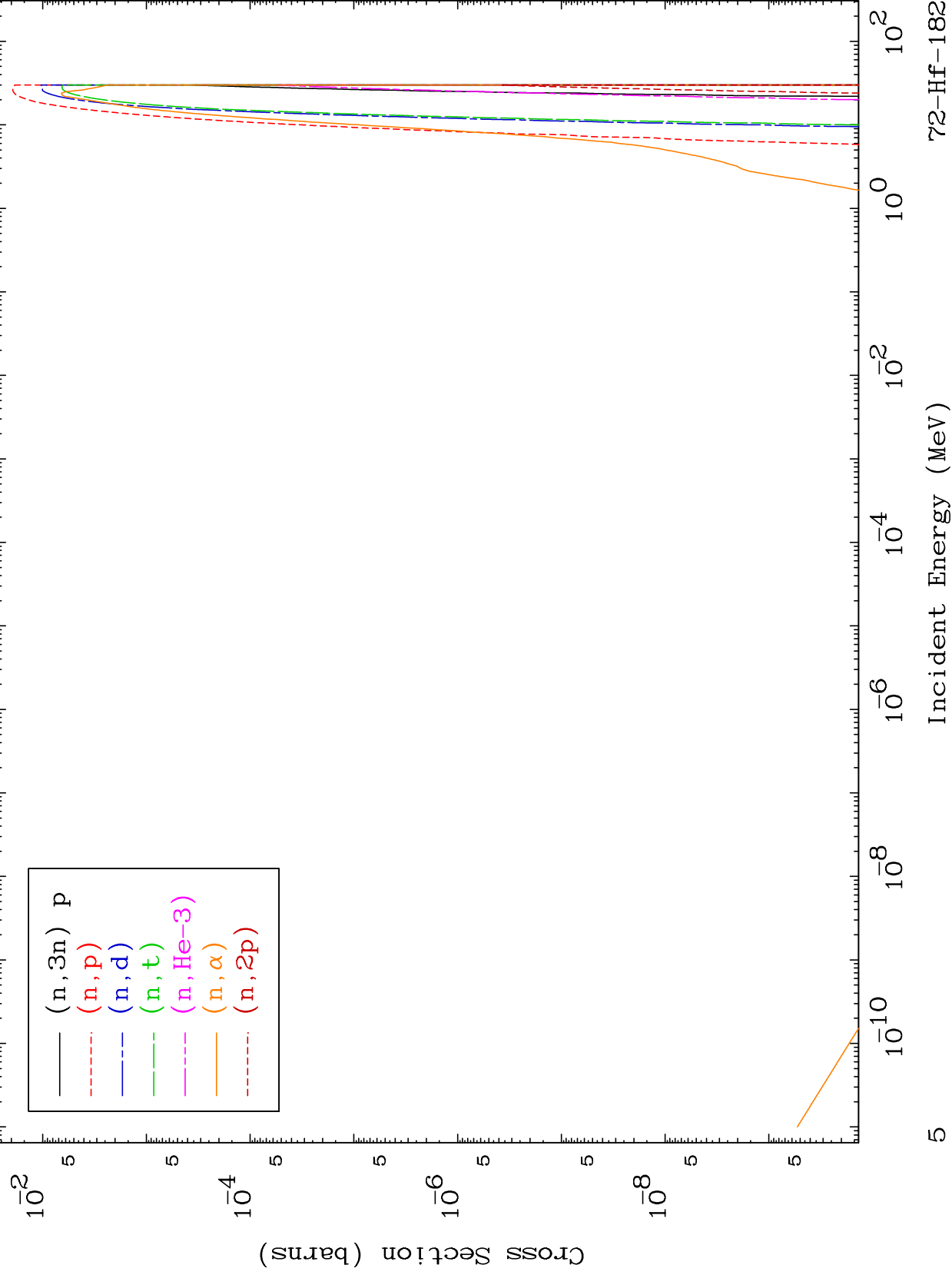
72-Hf-182



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Charged Particle  
293 Kelvin Cross Sections

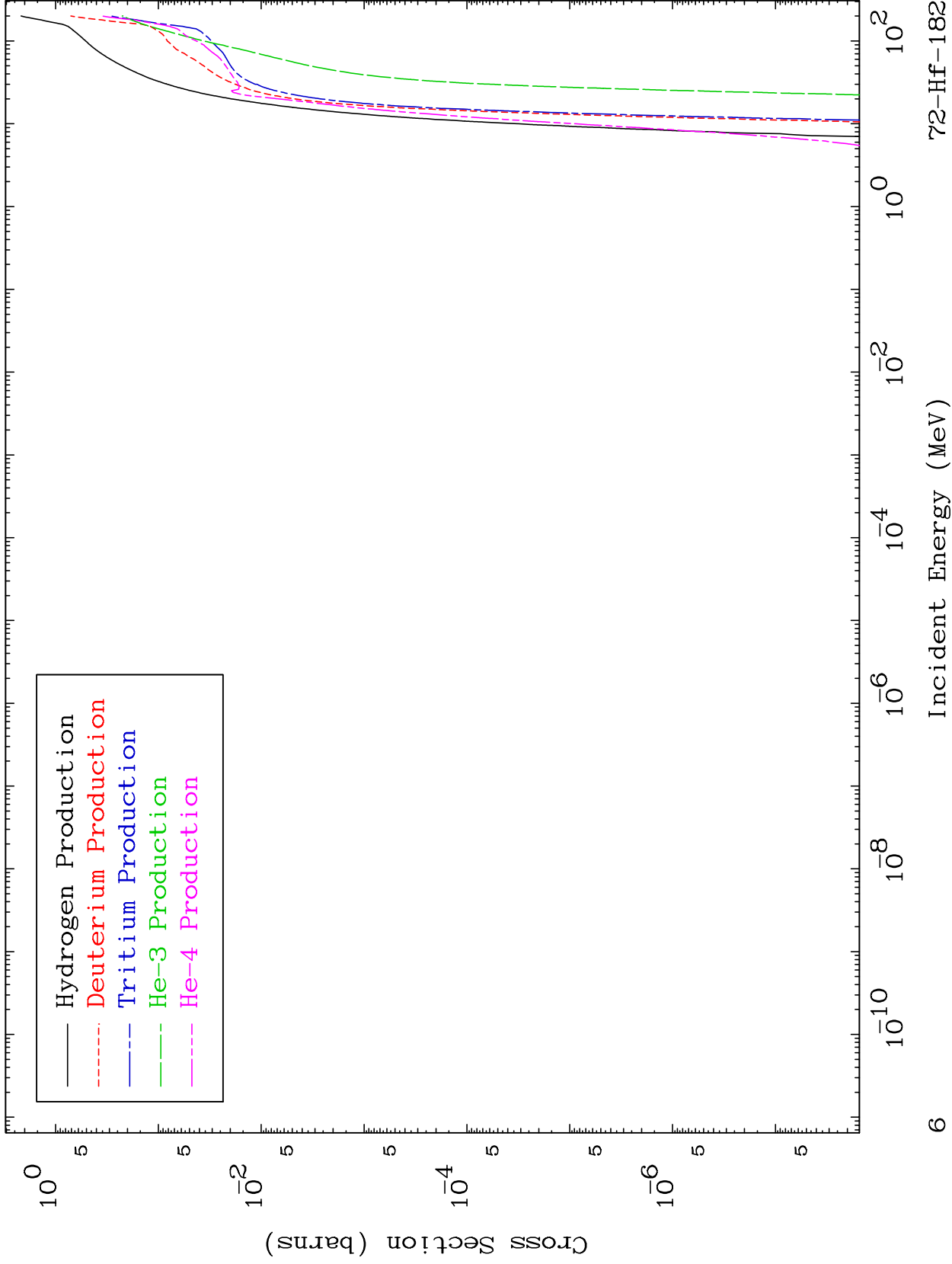
72-Hf-182



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Particle Production  
293 Kelvin Cross Sections

72-Hf-182

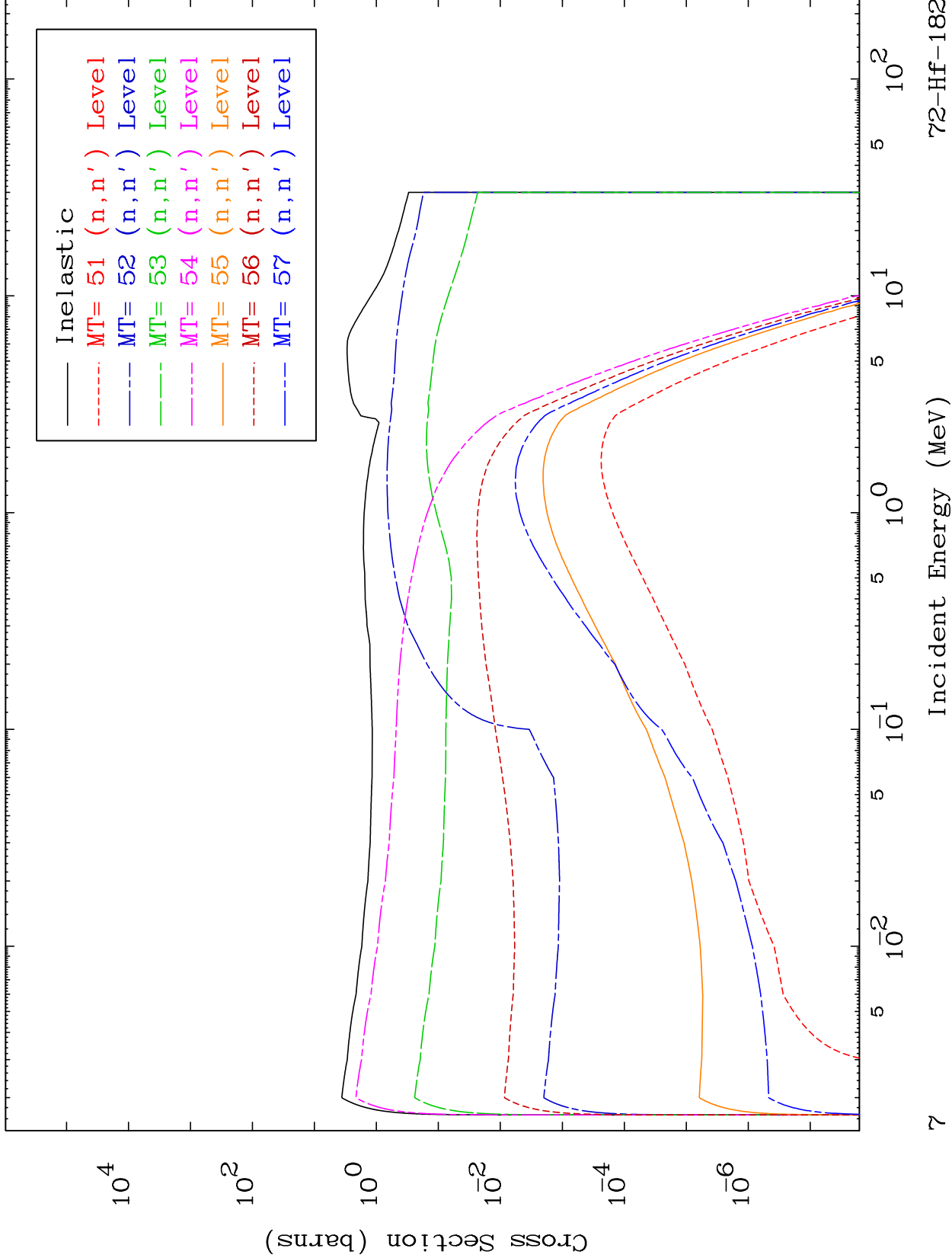


MAT 7250

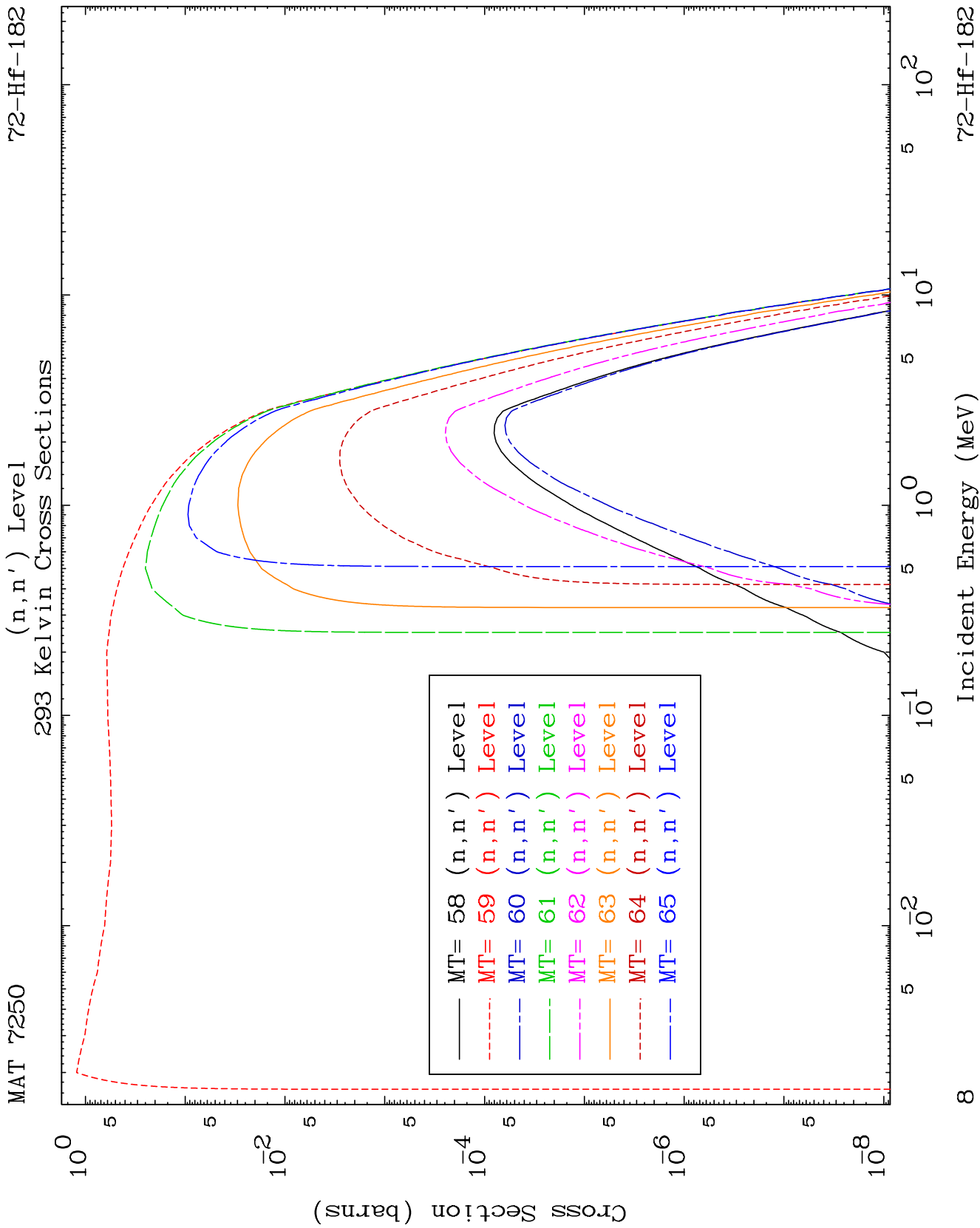
(n,n') Level

72-Hf-182

293 Kelvin Cross Sections





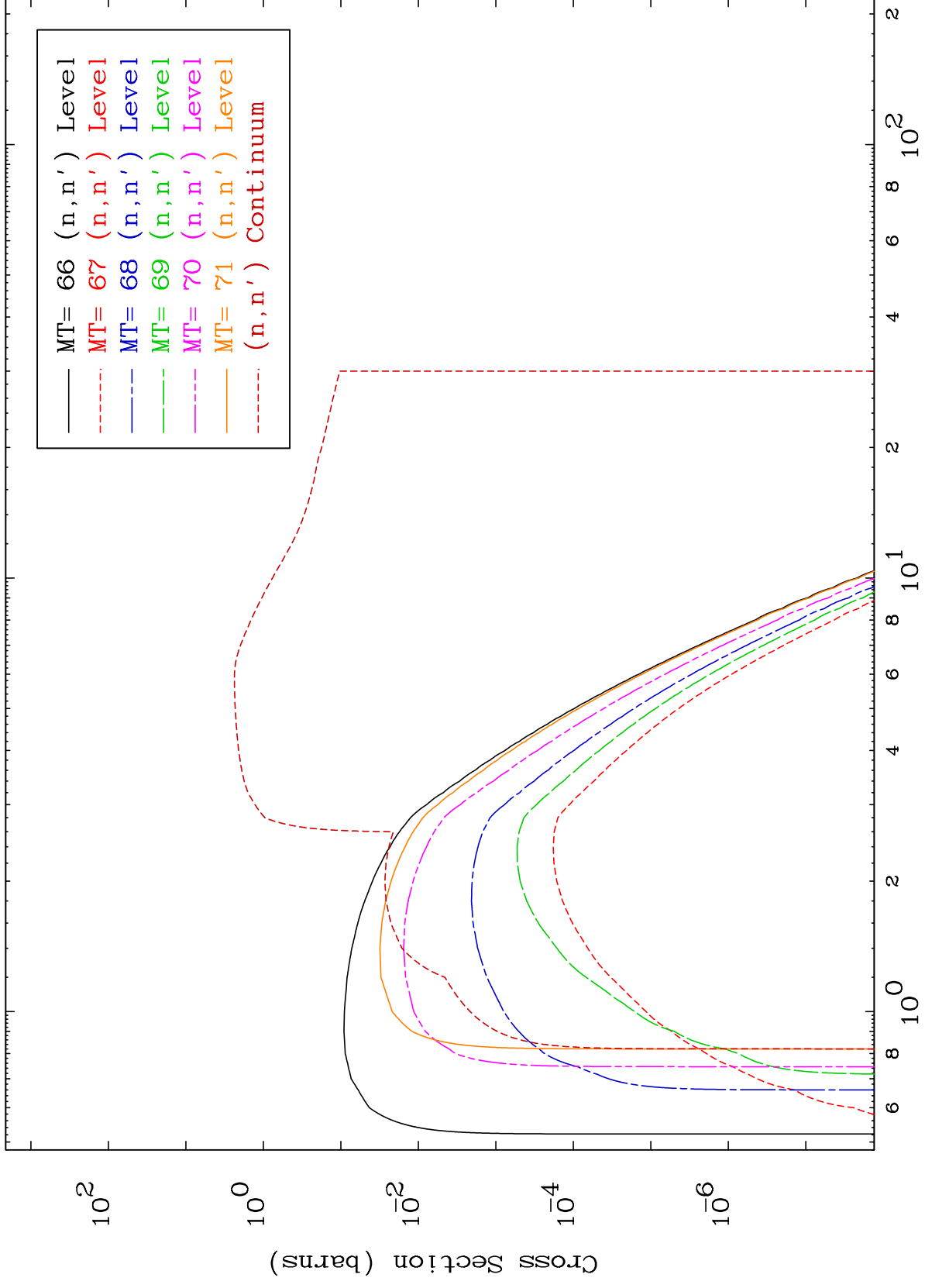


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(n,n') Level

72-Hf-182

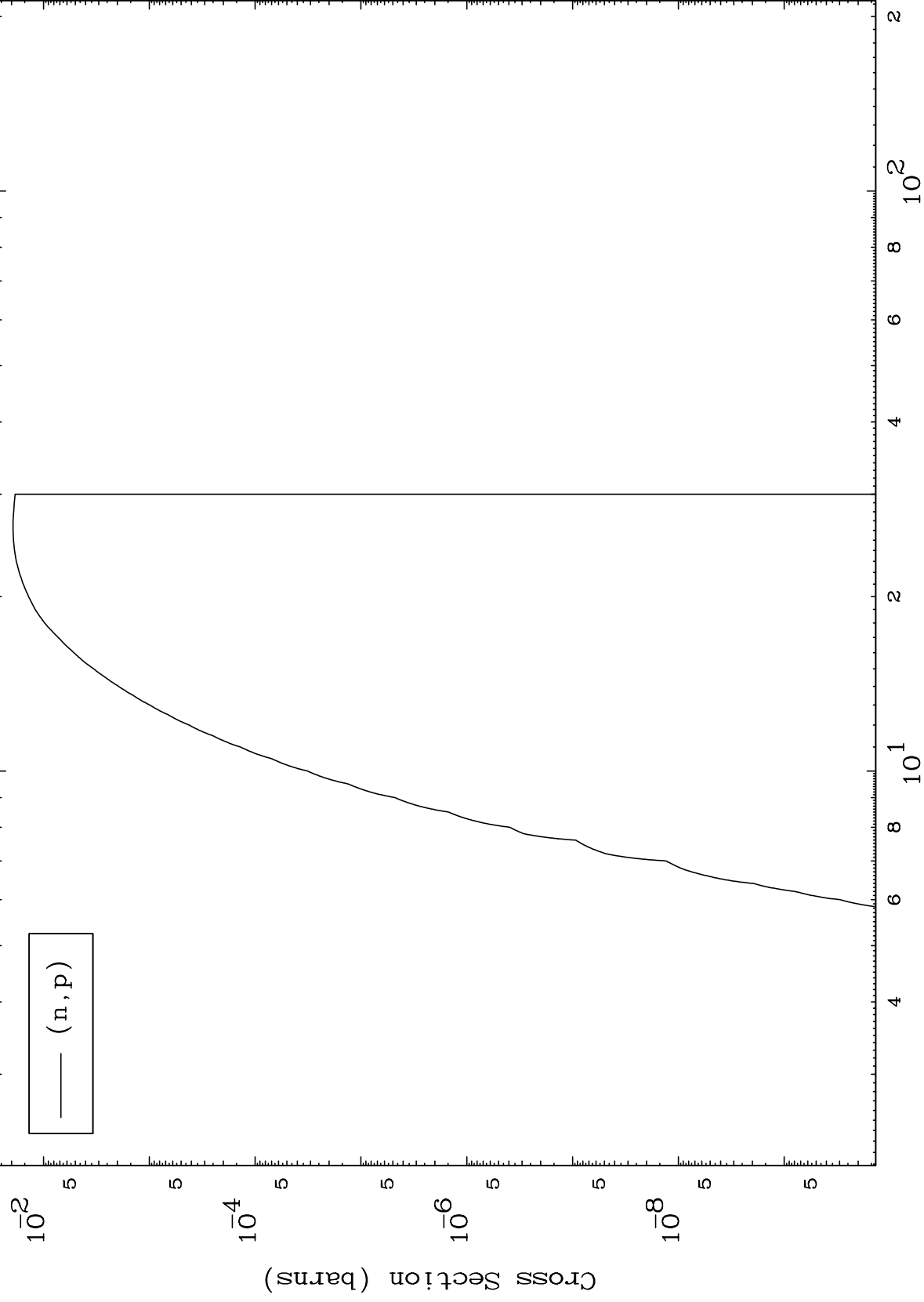
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

72-Hf-182



10

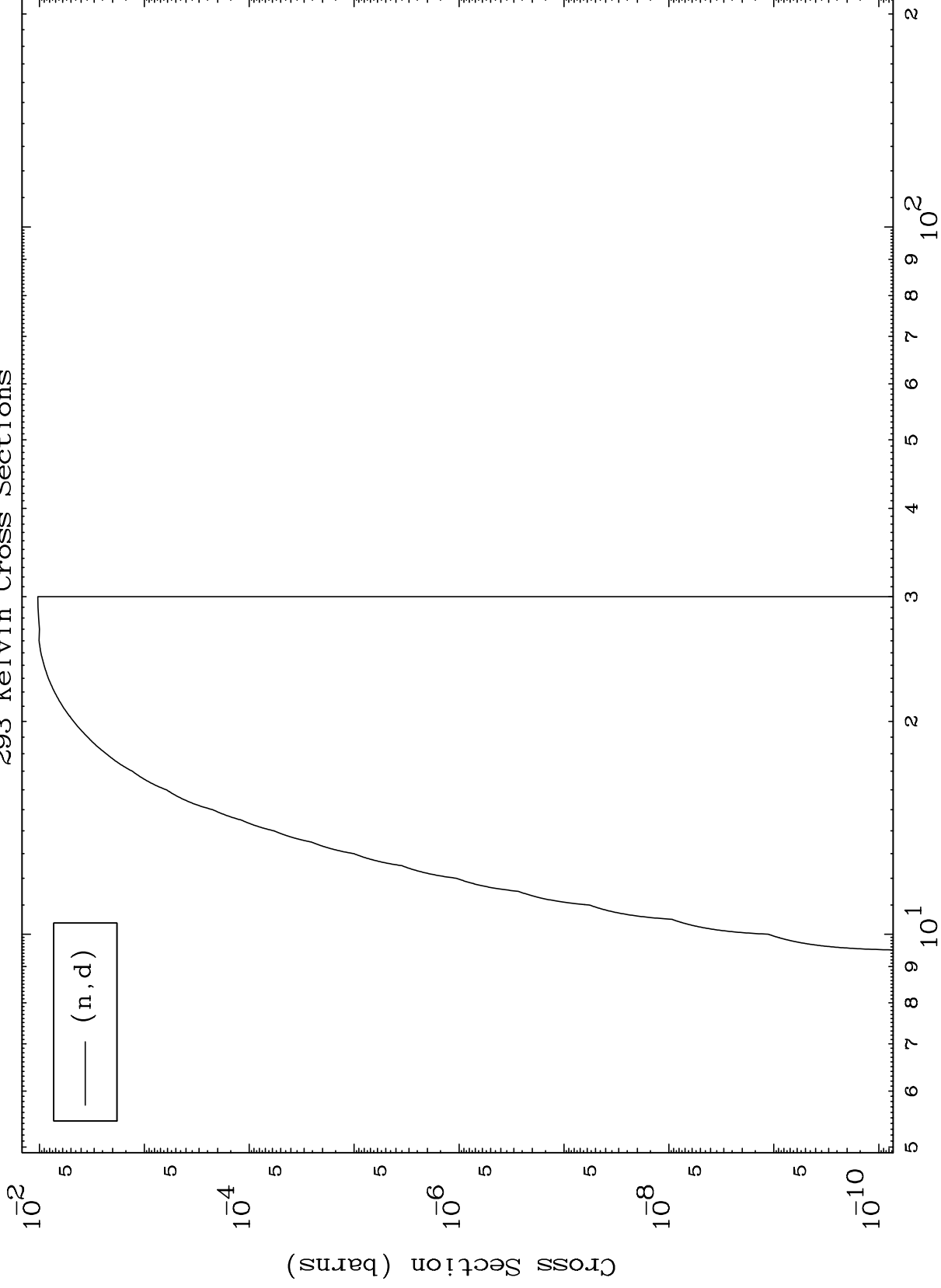
Incident Energy (MeV)

72-Hf-182

MAT 7250

(n,d) Levels  
293 Kelvin Cross Sections

72-Hf-182



11

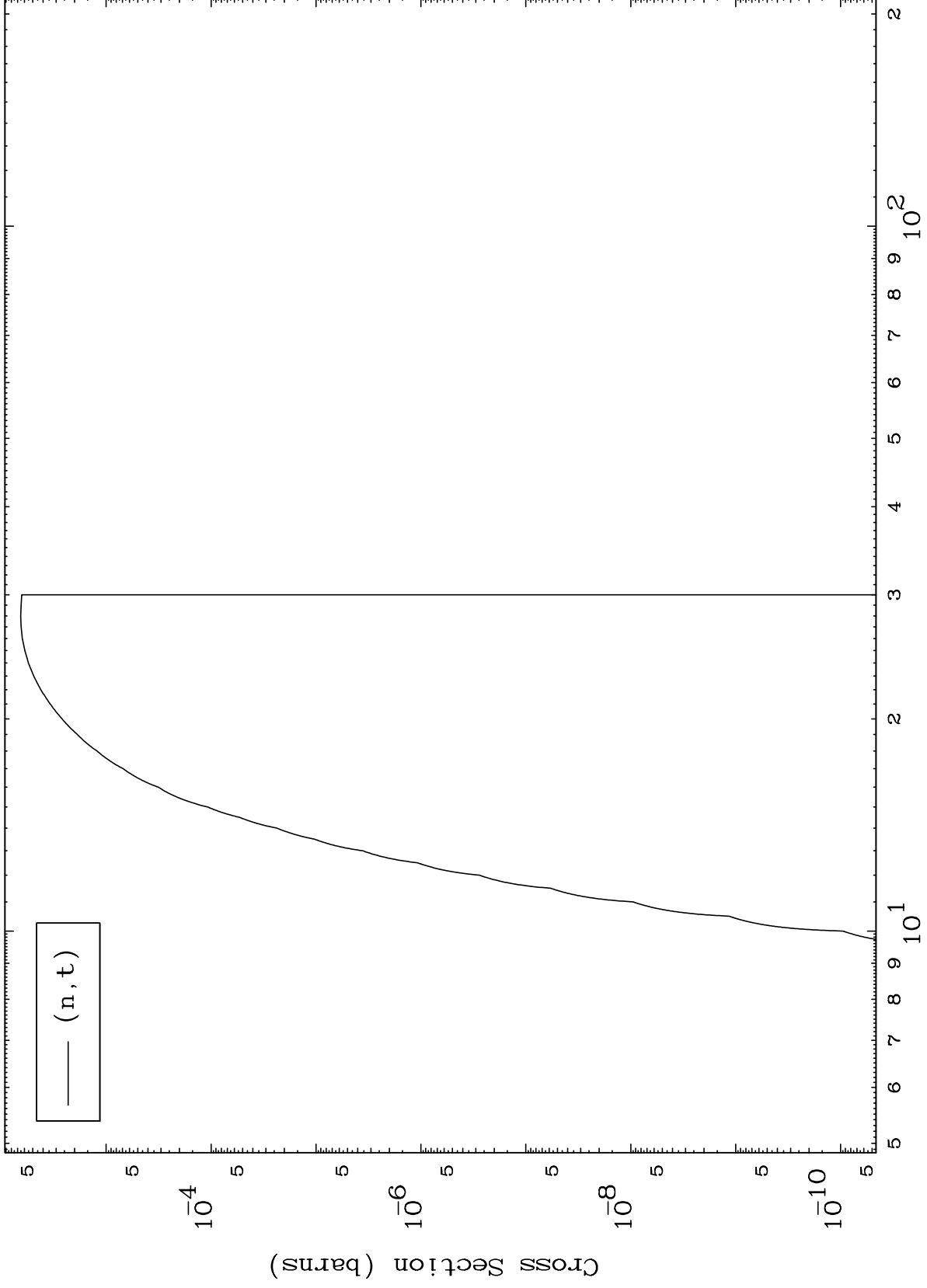
Incident Energy (MeV)

72-Hf-182

MAT 7250

(n,t) Levels  
293 Kelvin Cross Sections

72-Hf-182



12

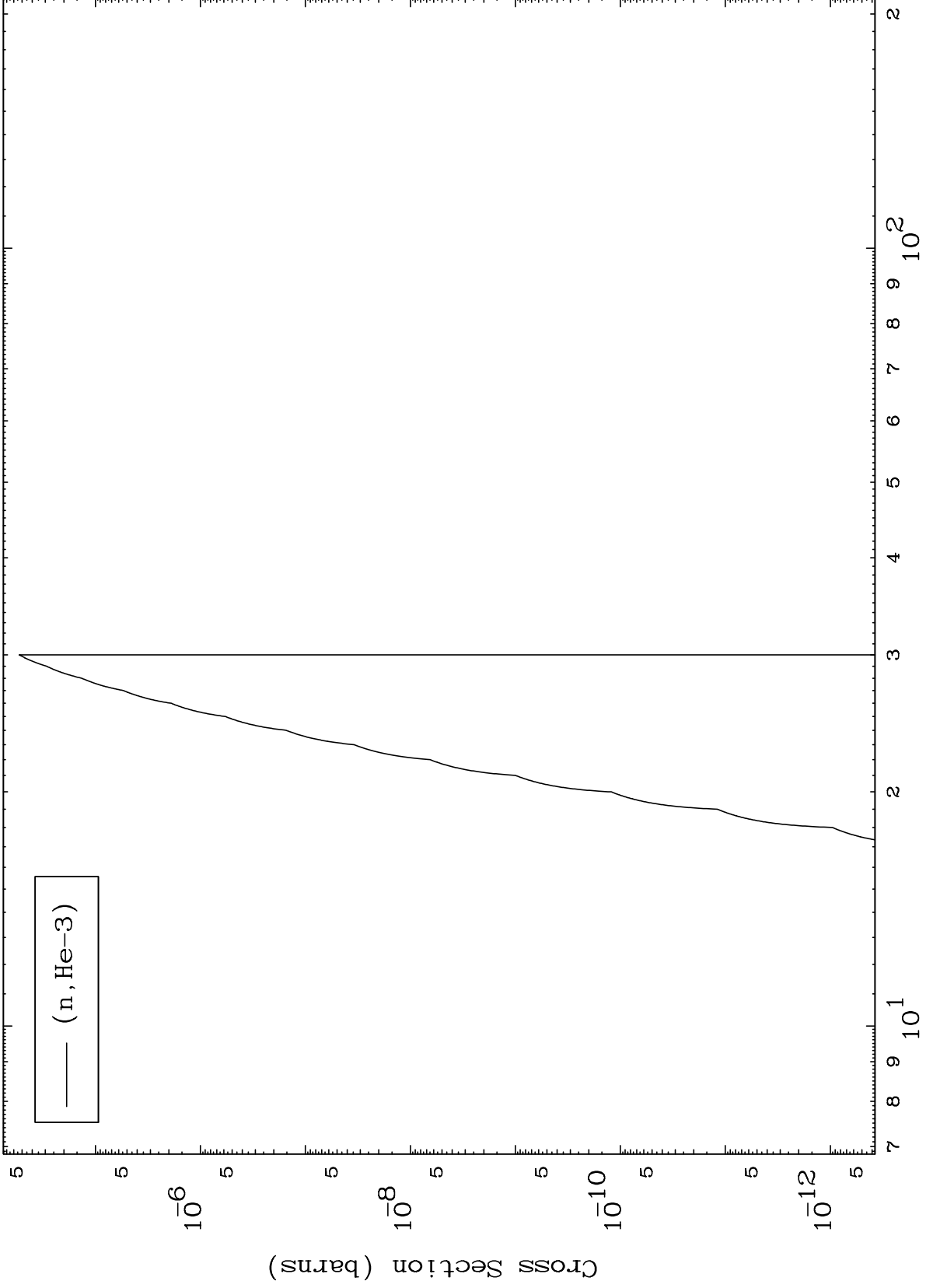
Incident Energy (MeV)

72-Hf-182

MAT 7250

(n,He3) Levels  
293 Kelvin Cross Sections

72-Hf-182



13

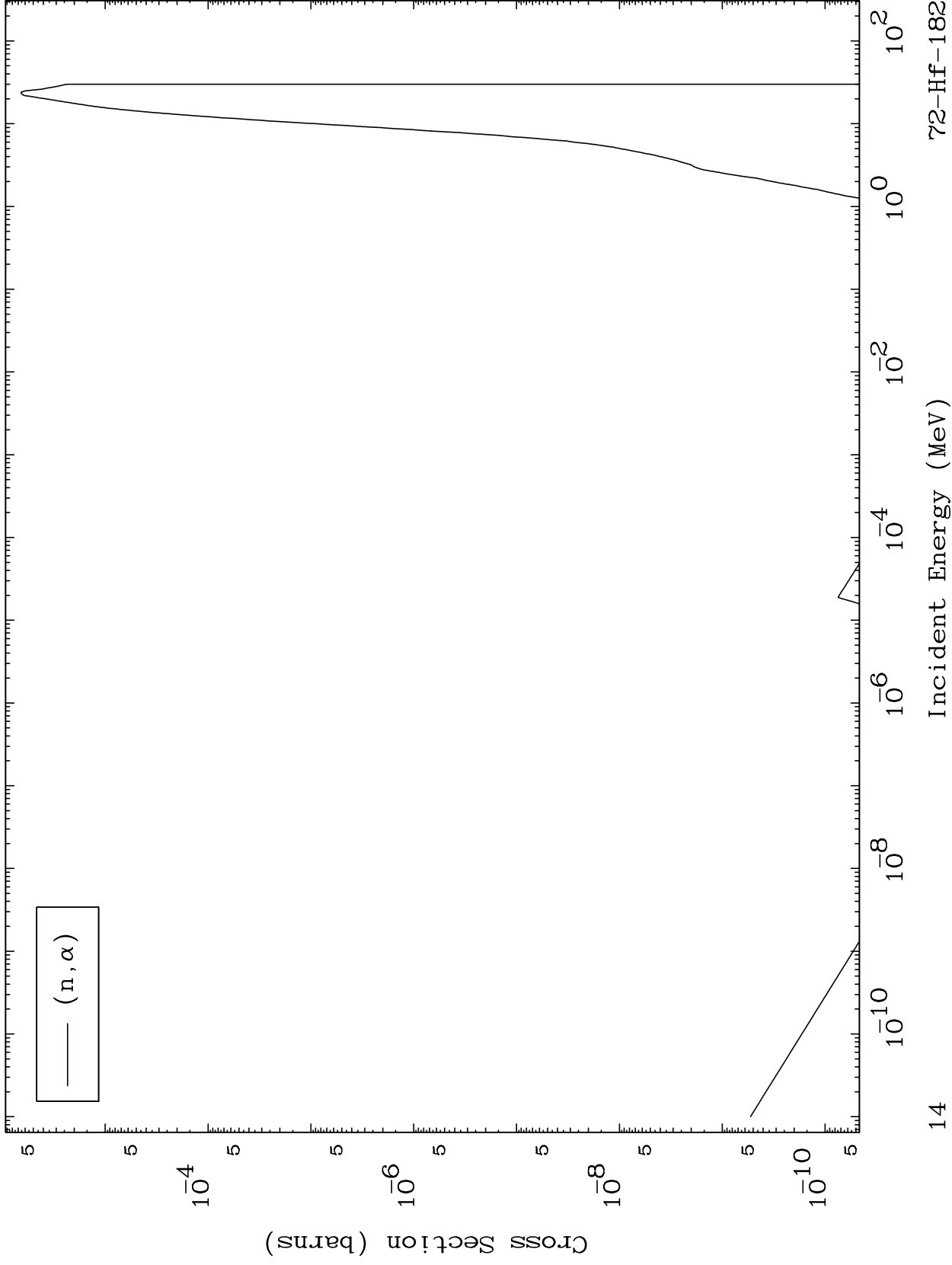
Incident Energy (MeV)

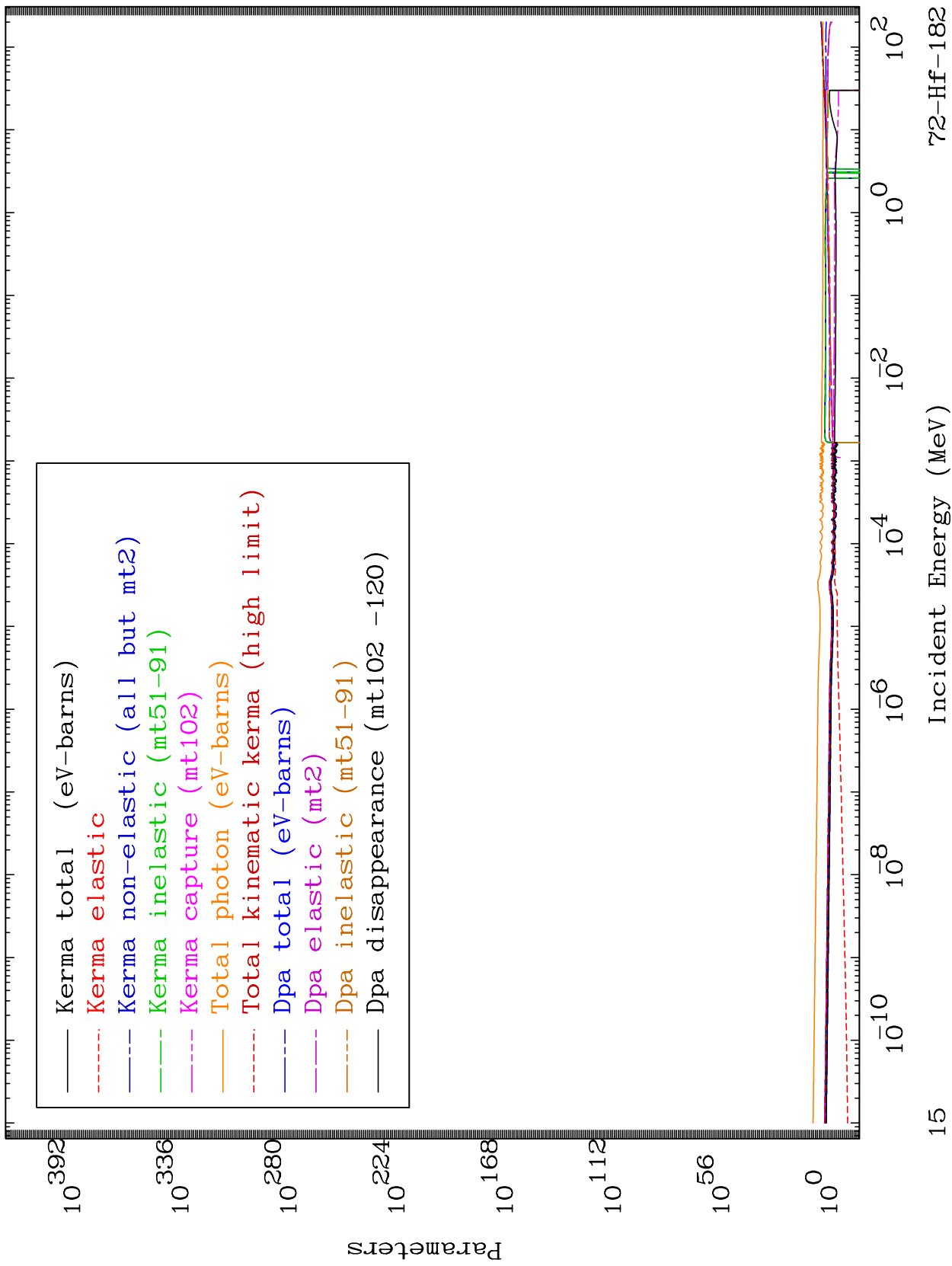
72-Hf-182

MAT 7250

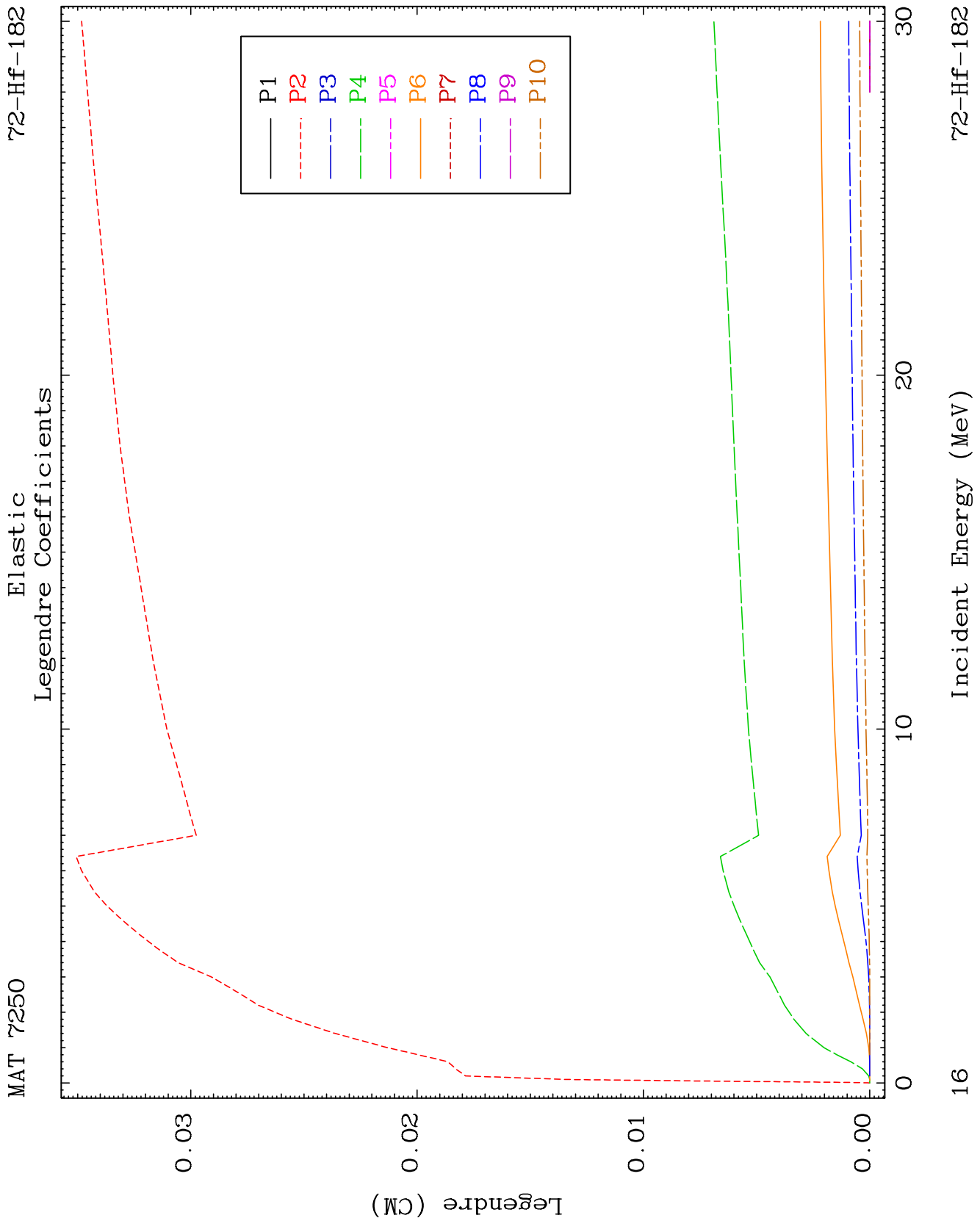
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

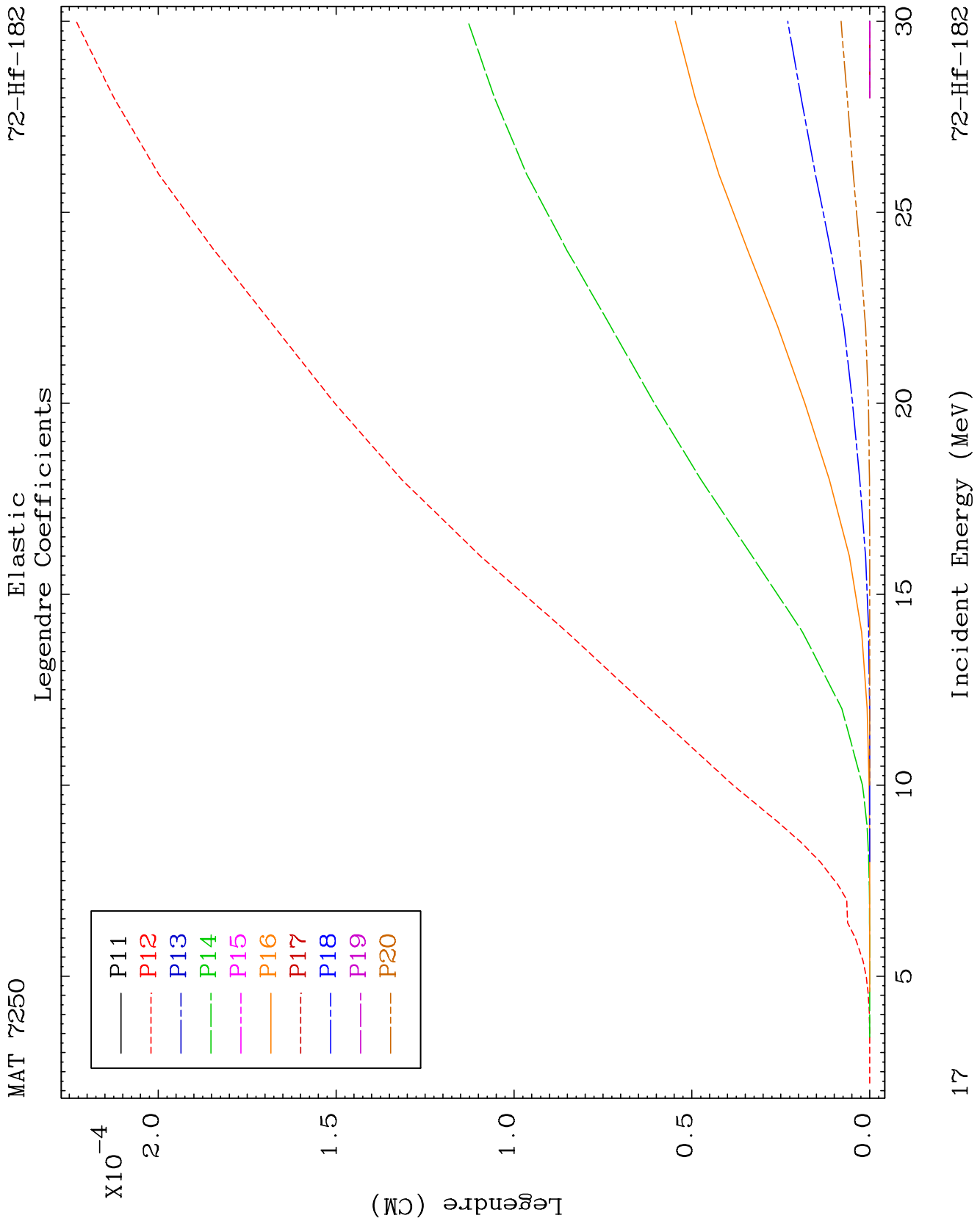
72-Hf-182







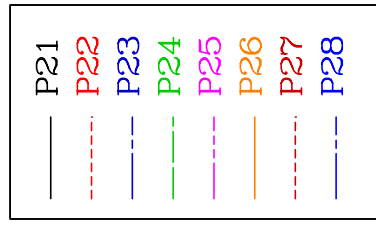




MAT 7250

Elastic  
Legendre Coefficients

72-Hf-182



$\times 10^{-6}$   
1.2  
1.0  
0.8  
0.6  
0.4  
0.2  
0.0

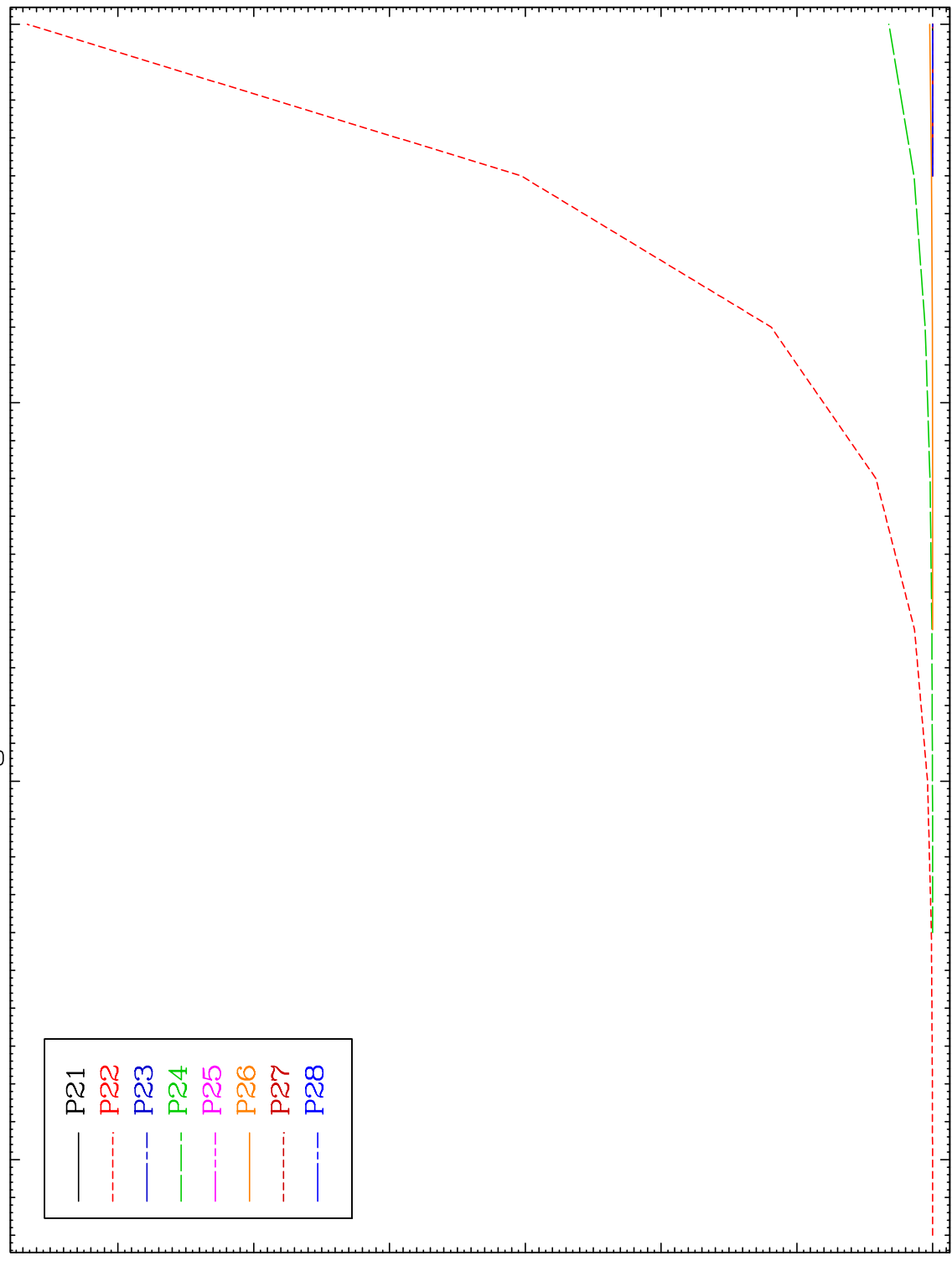
Legendre (CM)

15 20 25 30

18

Incident Energy (MeV)

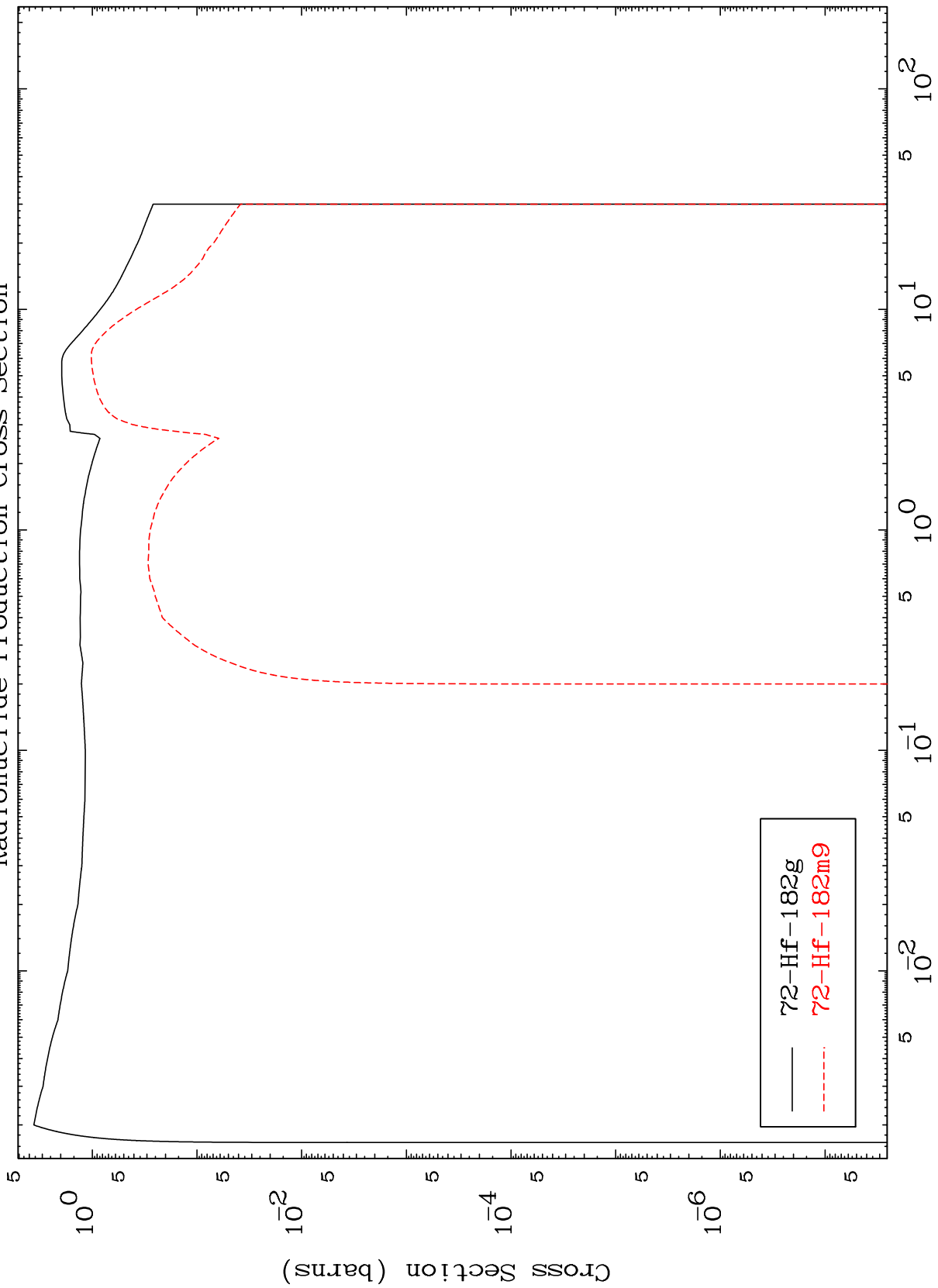
72-Hf-182

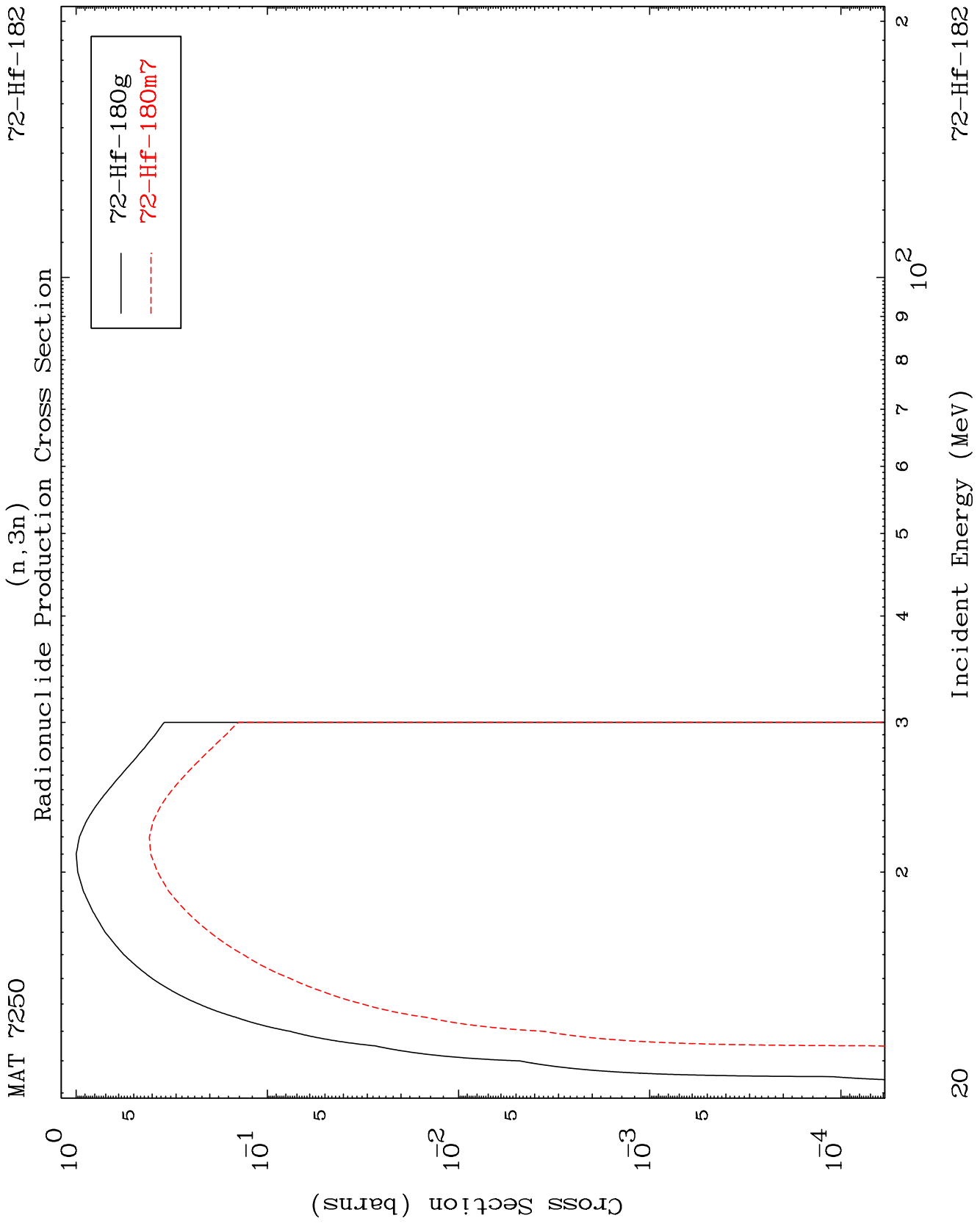


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### Inelastic Radionuclide Production Cross Section



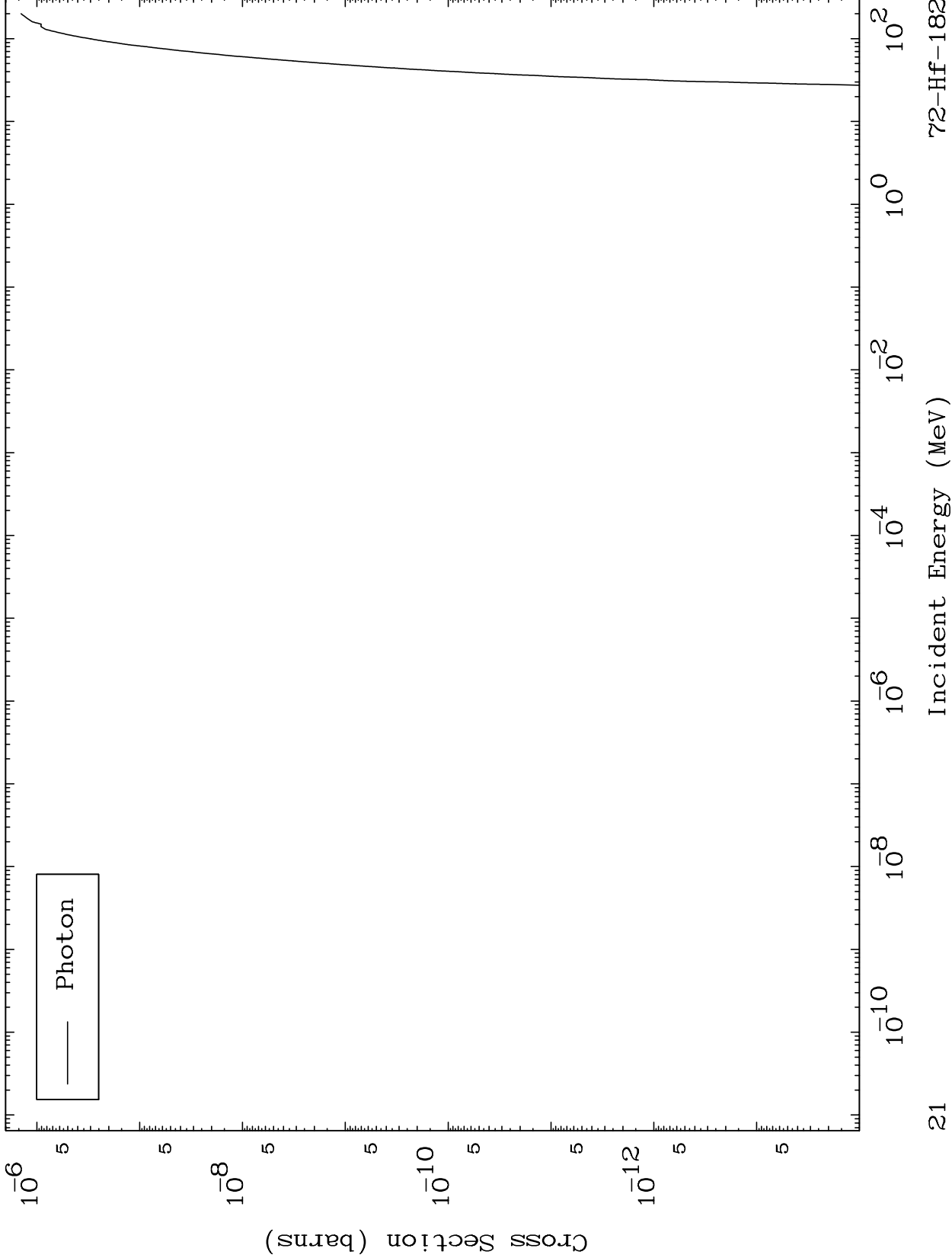


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Fission

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Radionuclide Production Cross Section

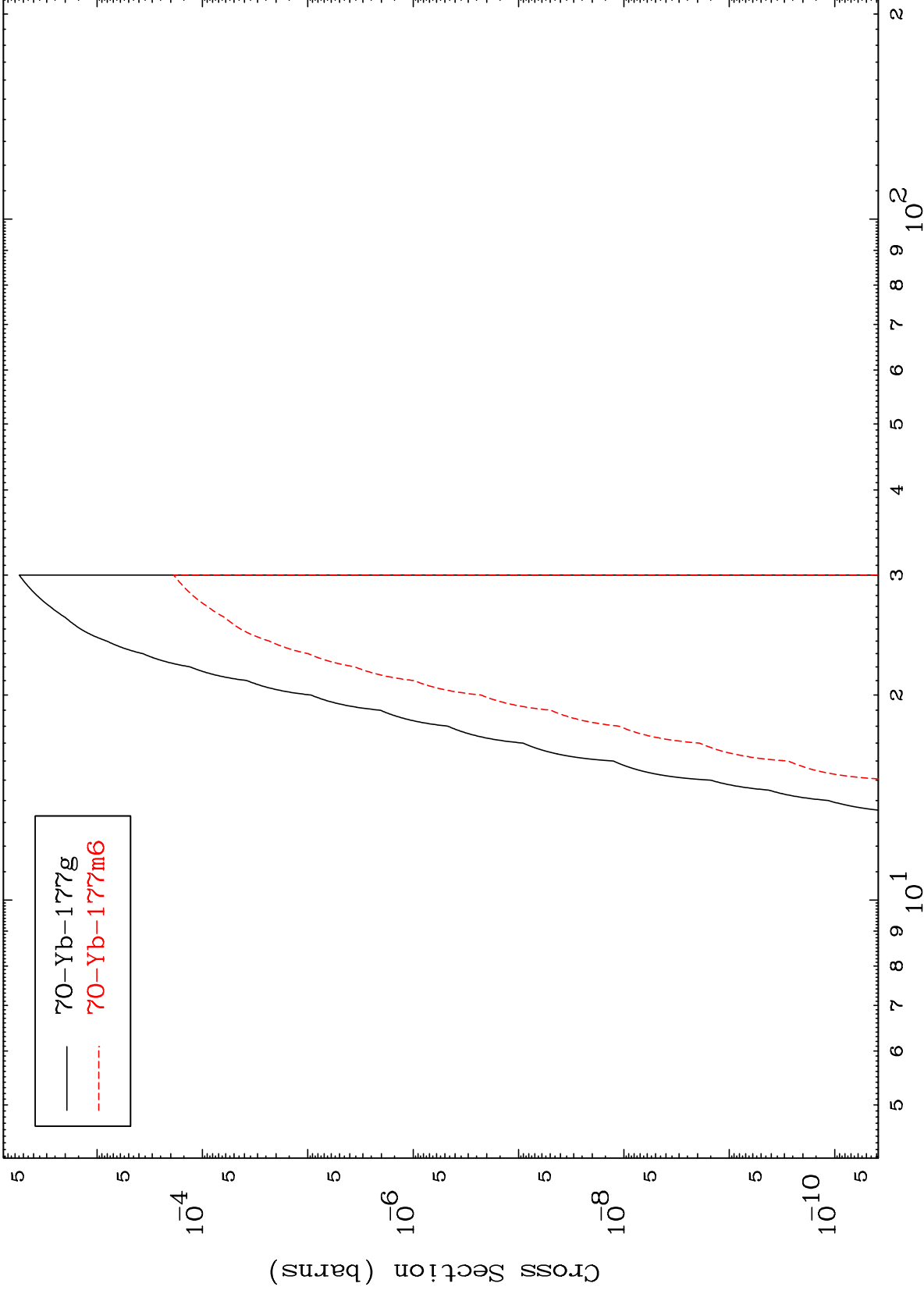


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(n,2n)  $\alpha$

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Radionuclide Production Cross Section

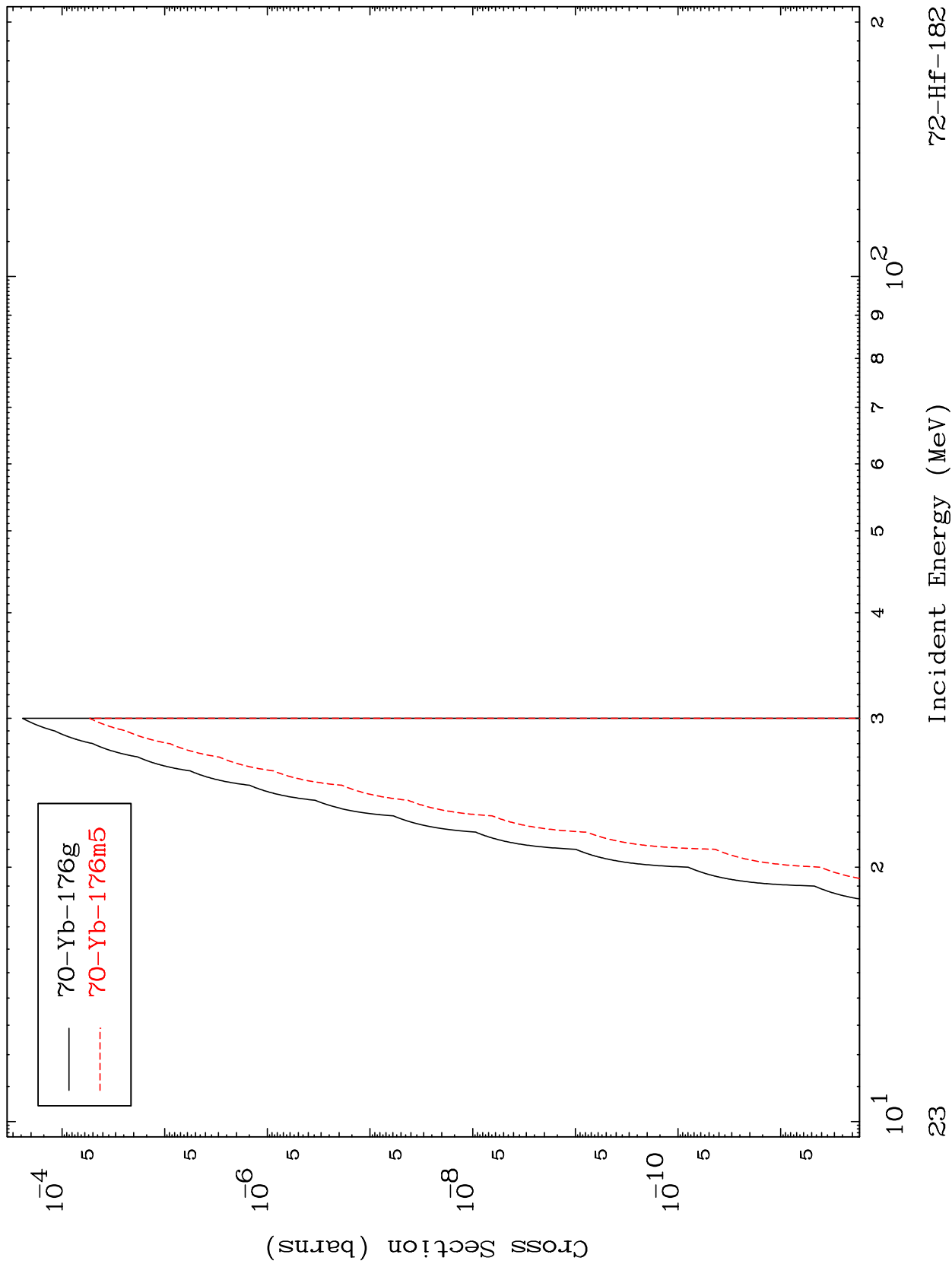


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(n,3n)  $\alpha$

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Radionuclide Production Cross Section



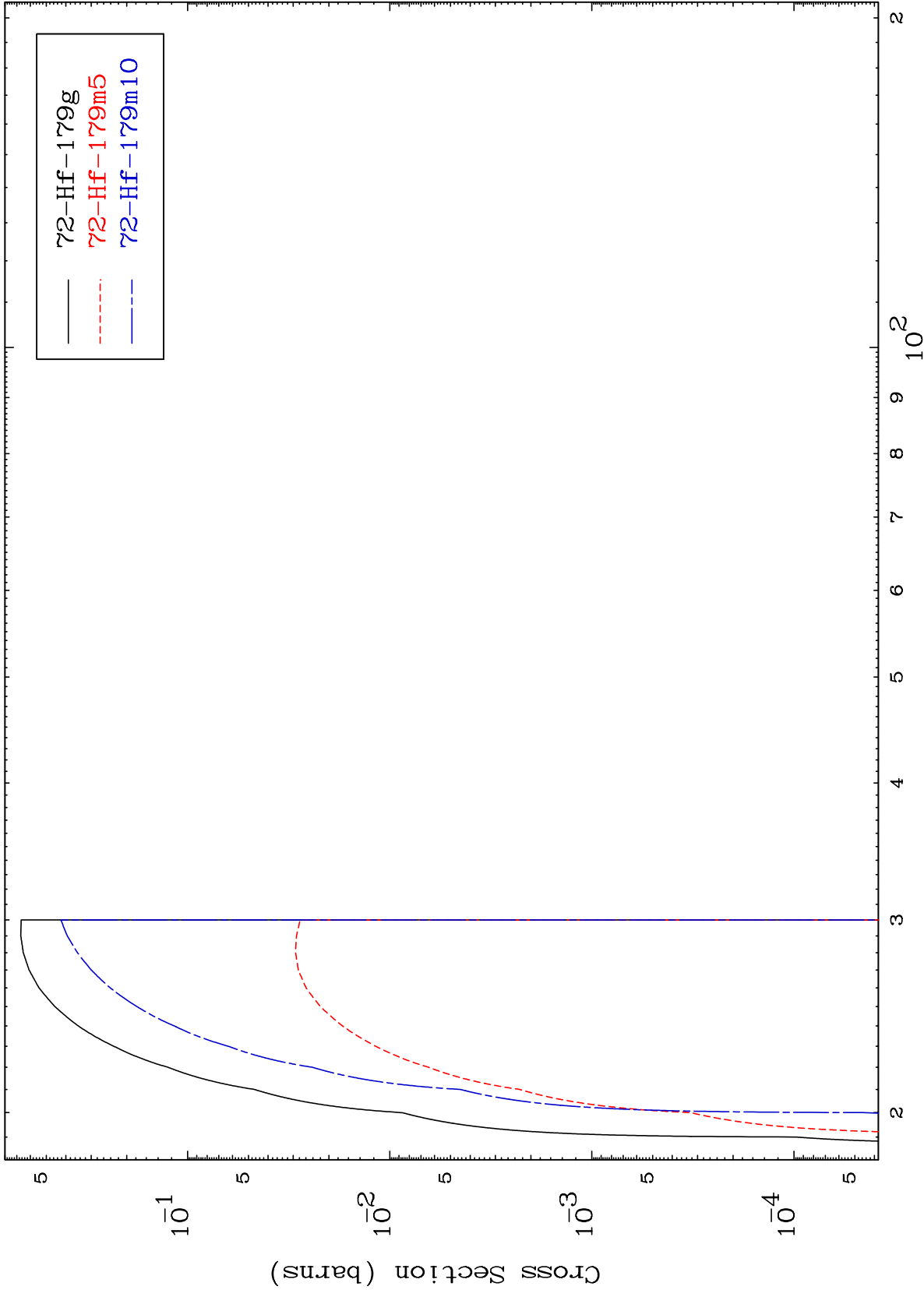


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(n,4n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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