

Program EVALPLOT  
(Version 2017-1)

by

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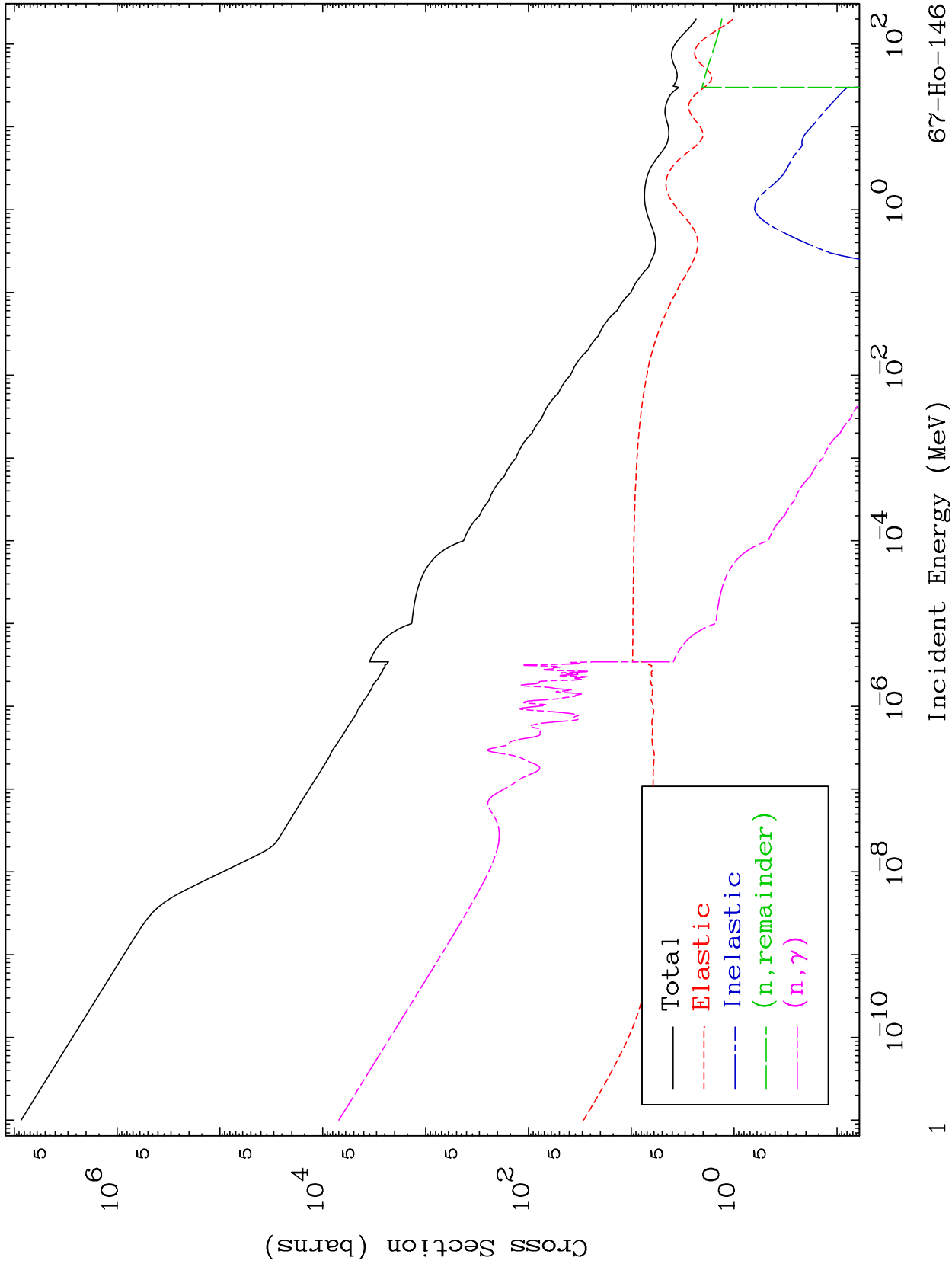
Press Mouse Button to Start

MAT 6668

Major

293 Kelvin Cross Sections

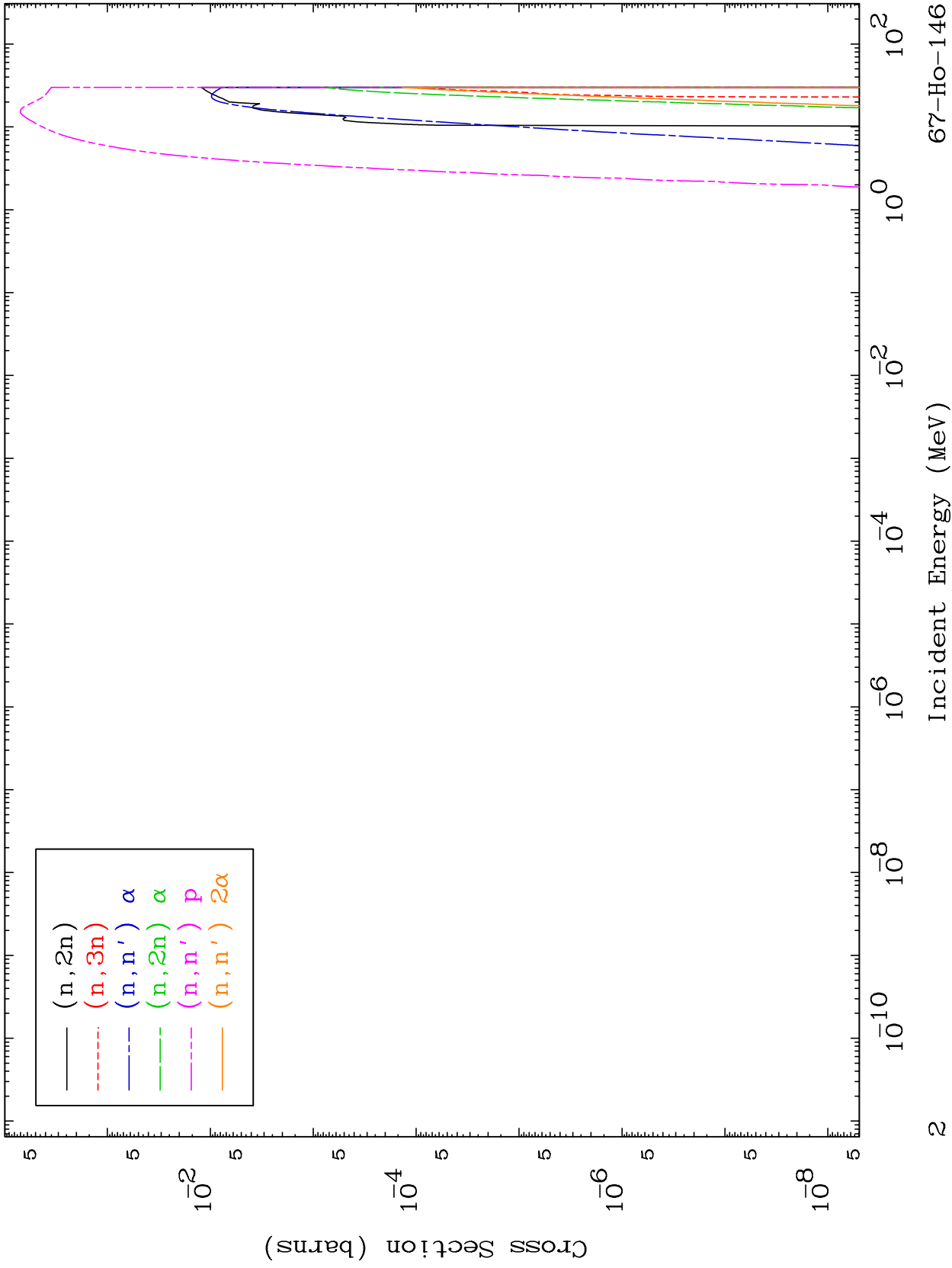
67-Ho-146

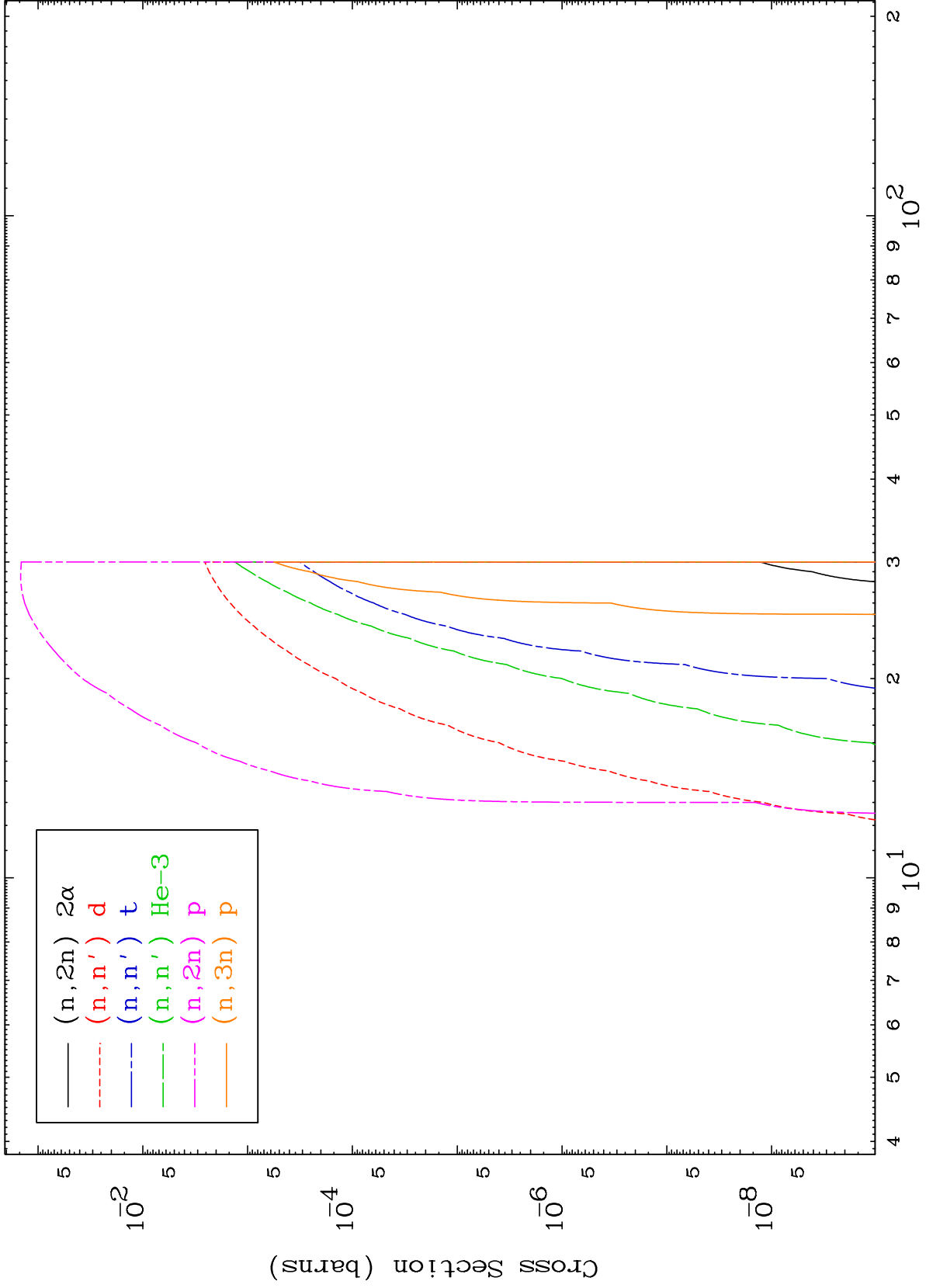


MAT 6668

Neutron Production  
293 Kelvin Cross Sections

67-Ho-146

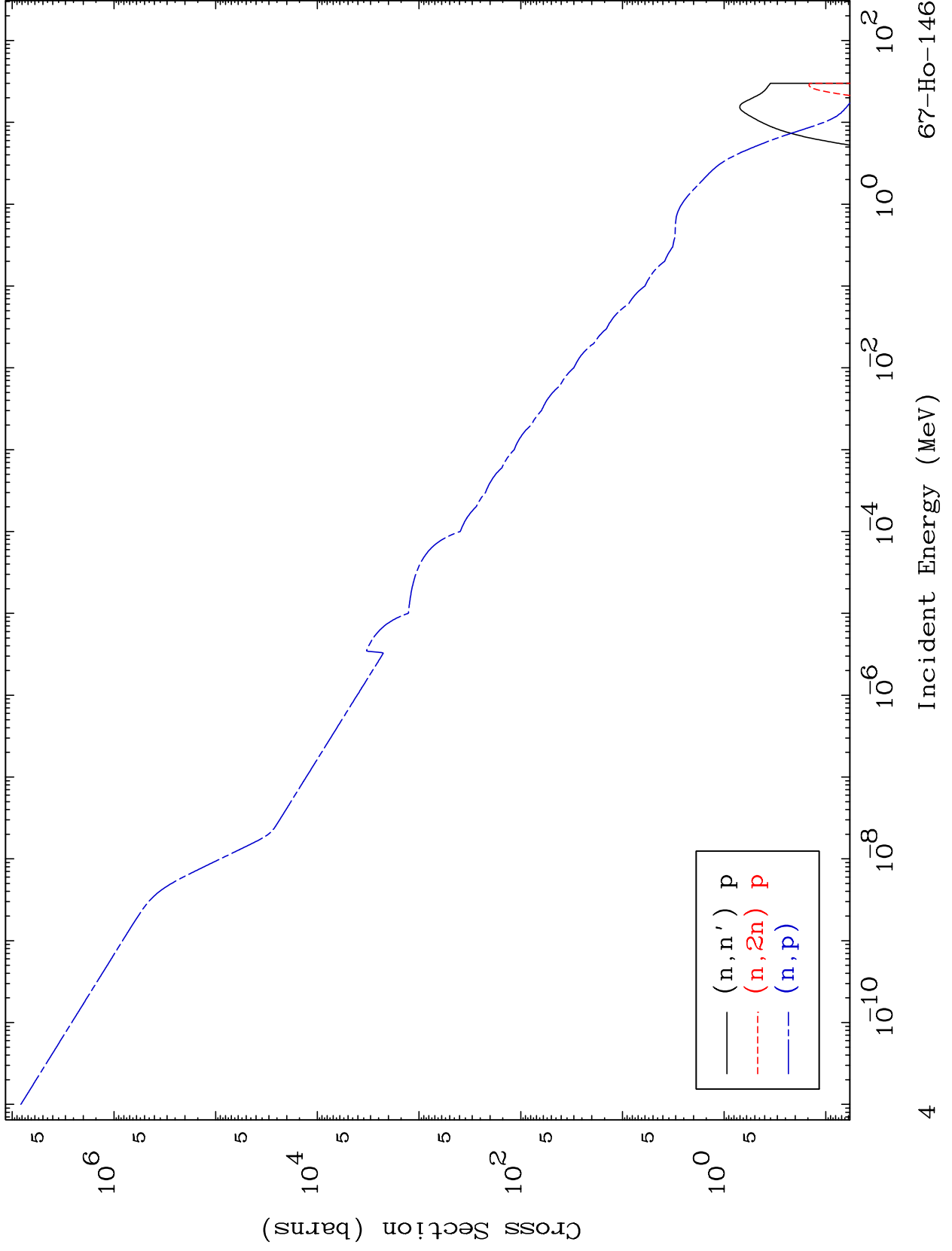




MAT 6668

Charged Particle  
293 Kelvin Cross Sections

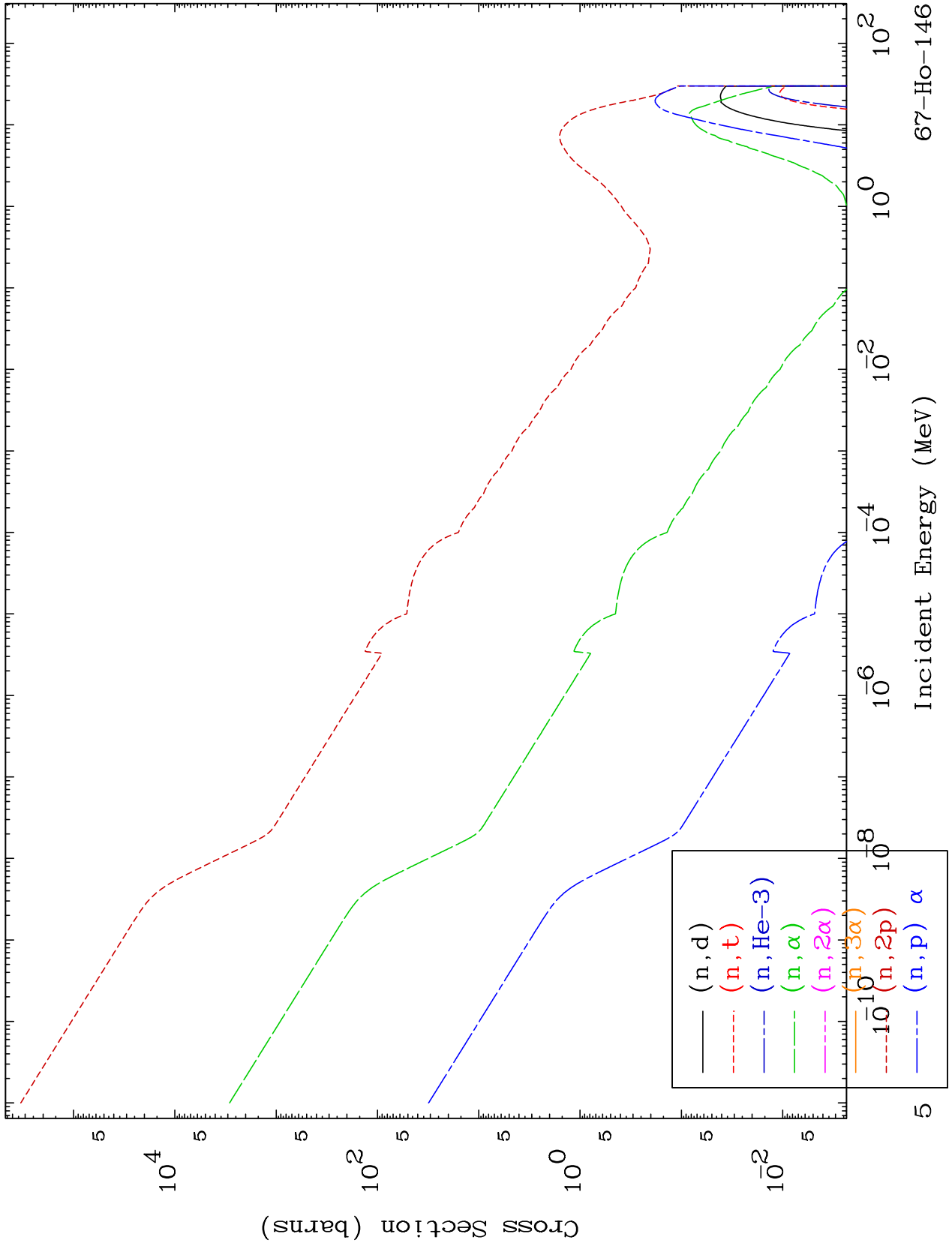
67-Ho-146



MAT 6668

Charged Particle  
293 Kelvin Cross Sections

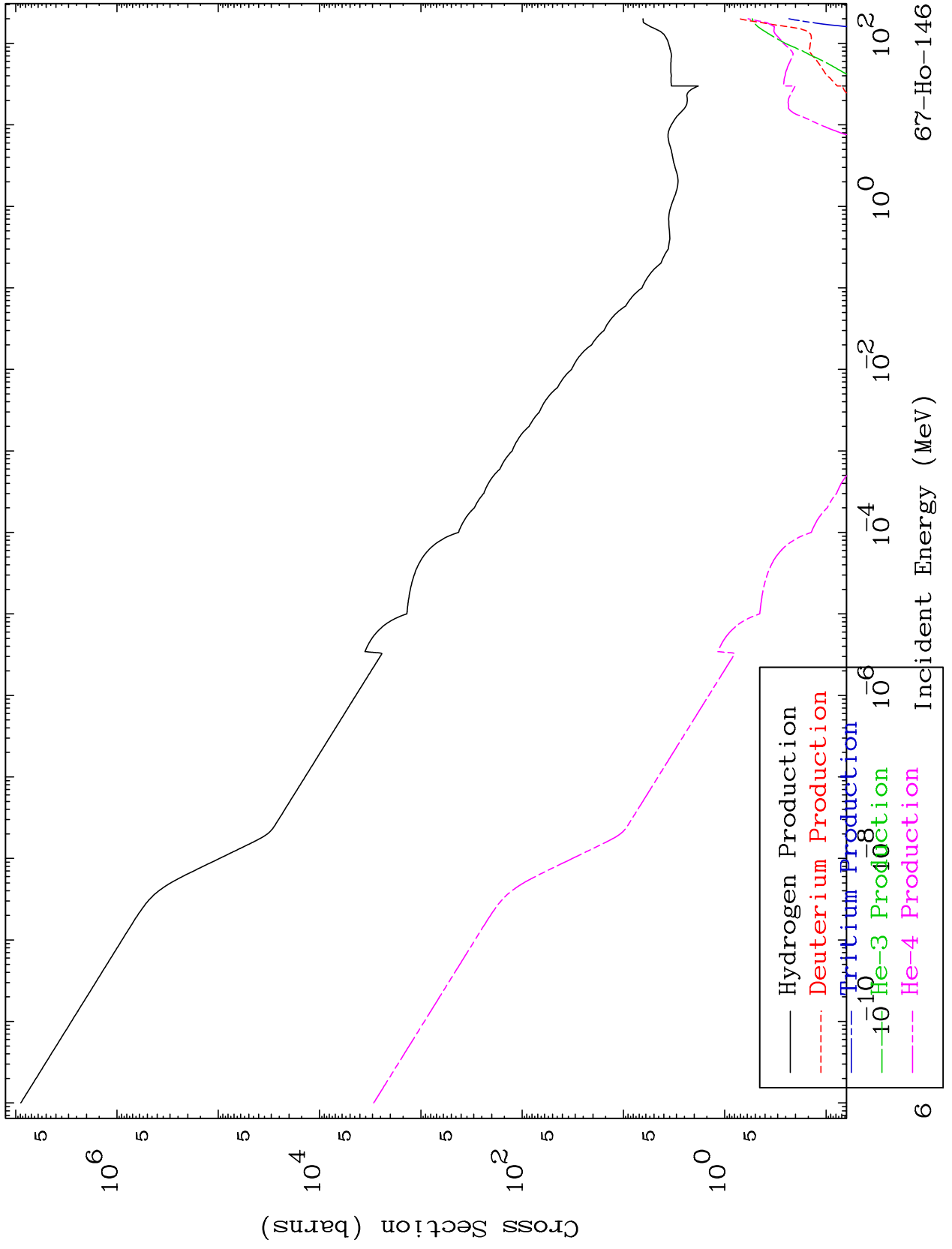
67-Ho-146



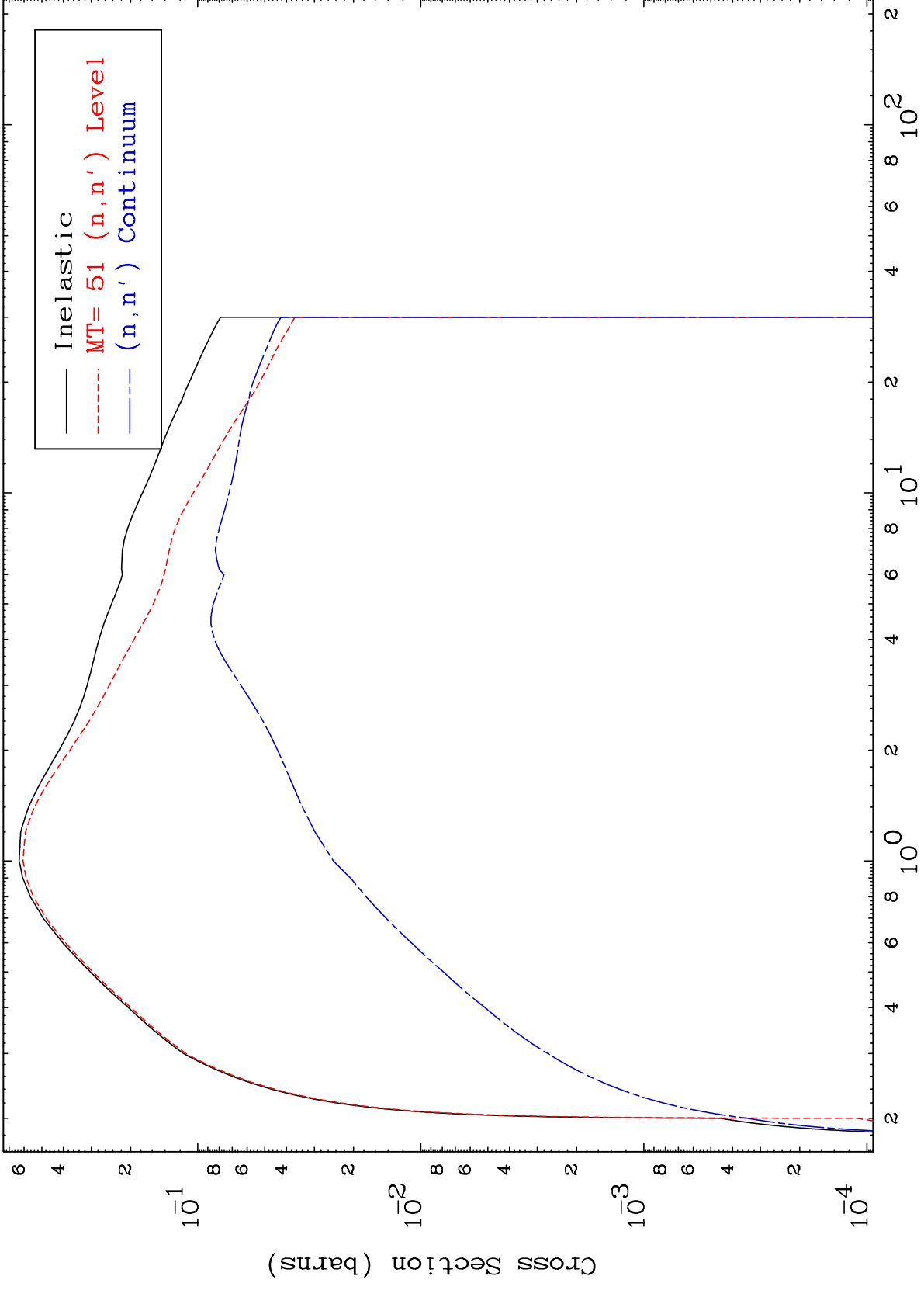
MAT 6668

Particle Production  
293 Kelvin Cross Sections

67-Ho-146



293 Kelvin Cross Sections

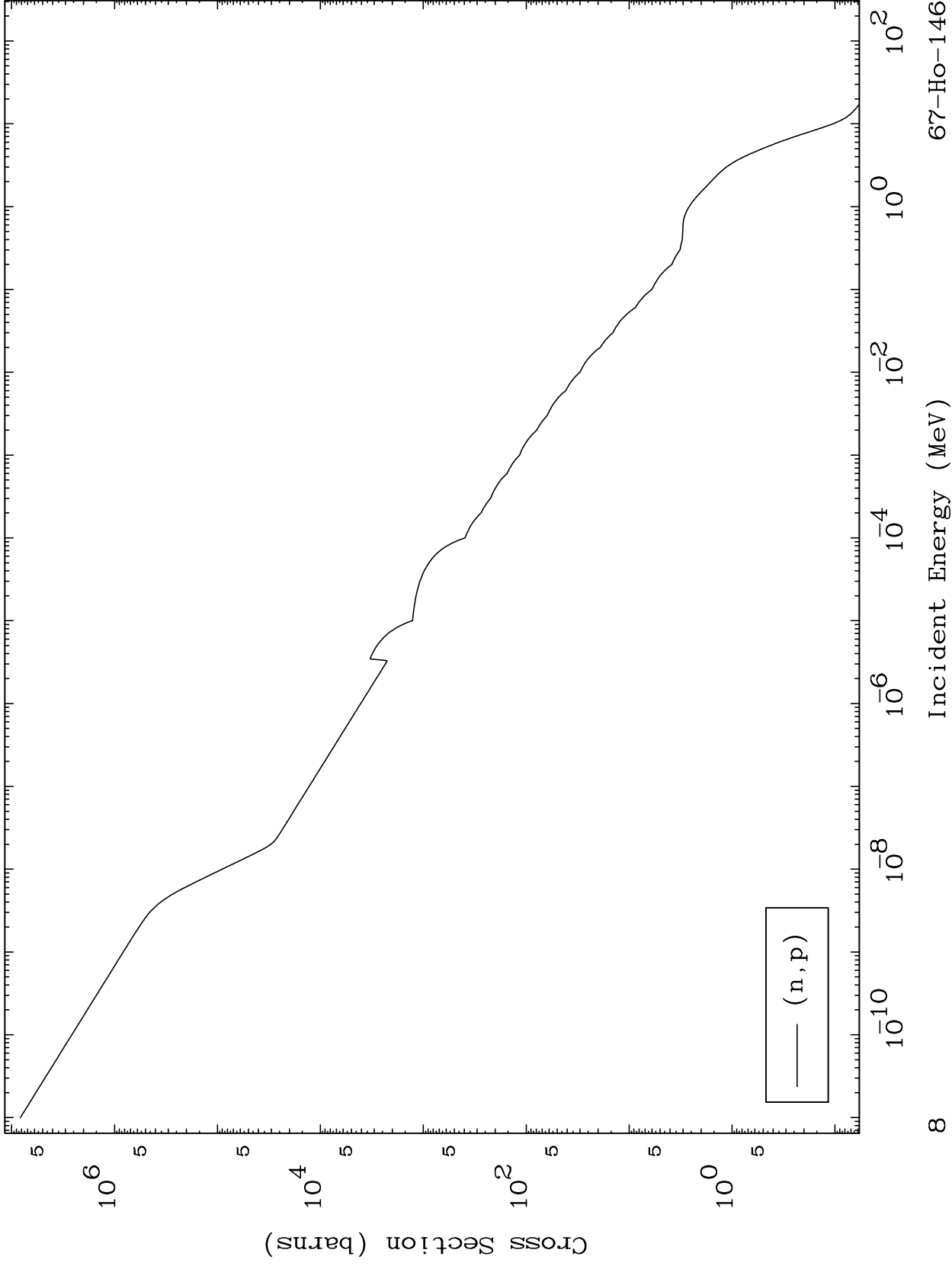




MAT 6668

(n,p) Levels  
293 Kelvin Cross Sections

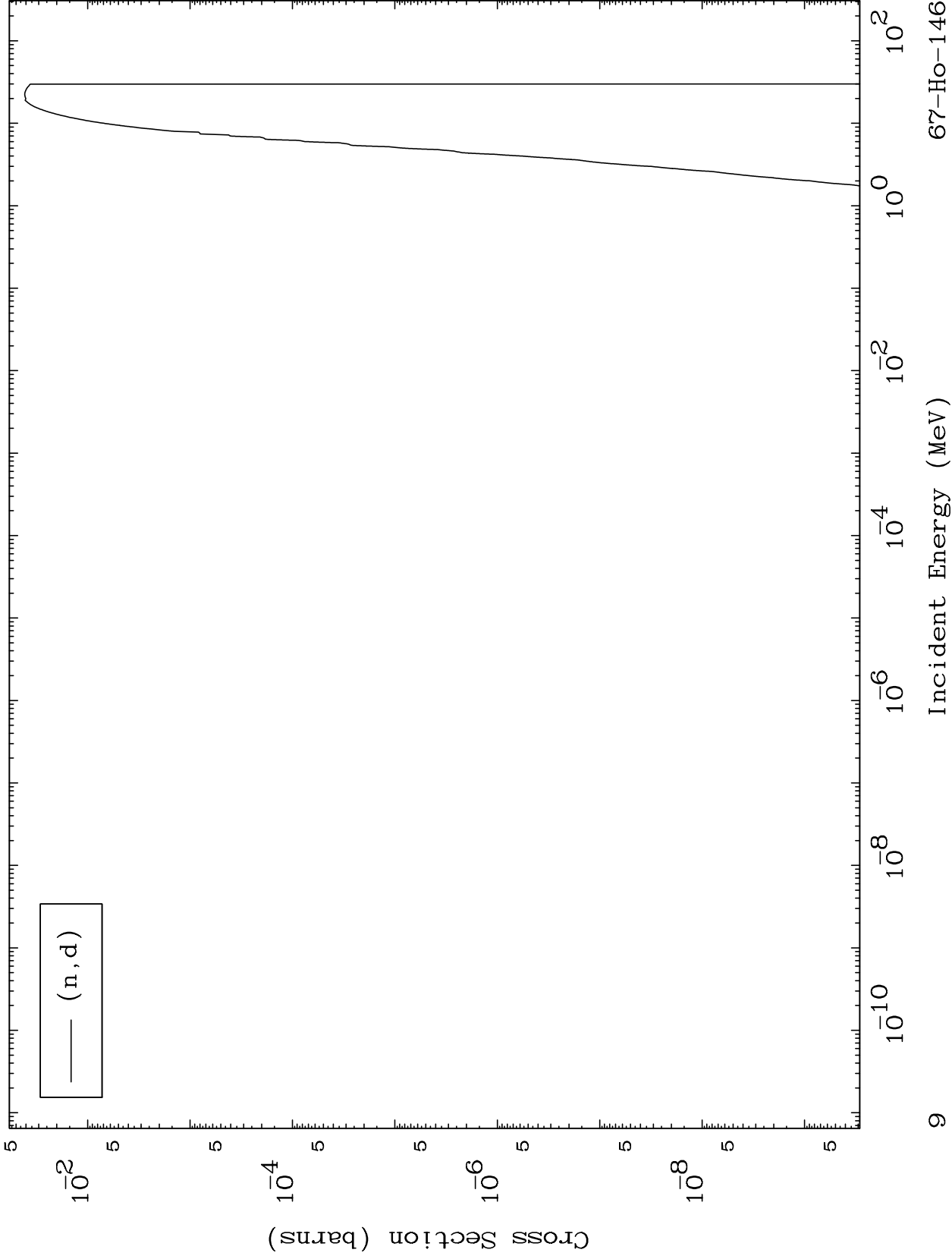
67-Ho-146



MAT 6668

(n,d) Levels  
293 Kelvin Cross Sections

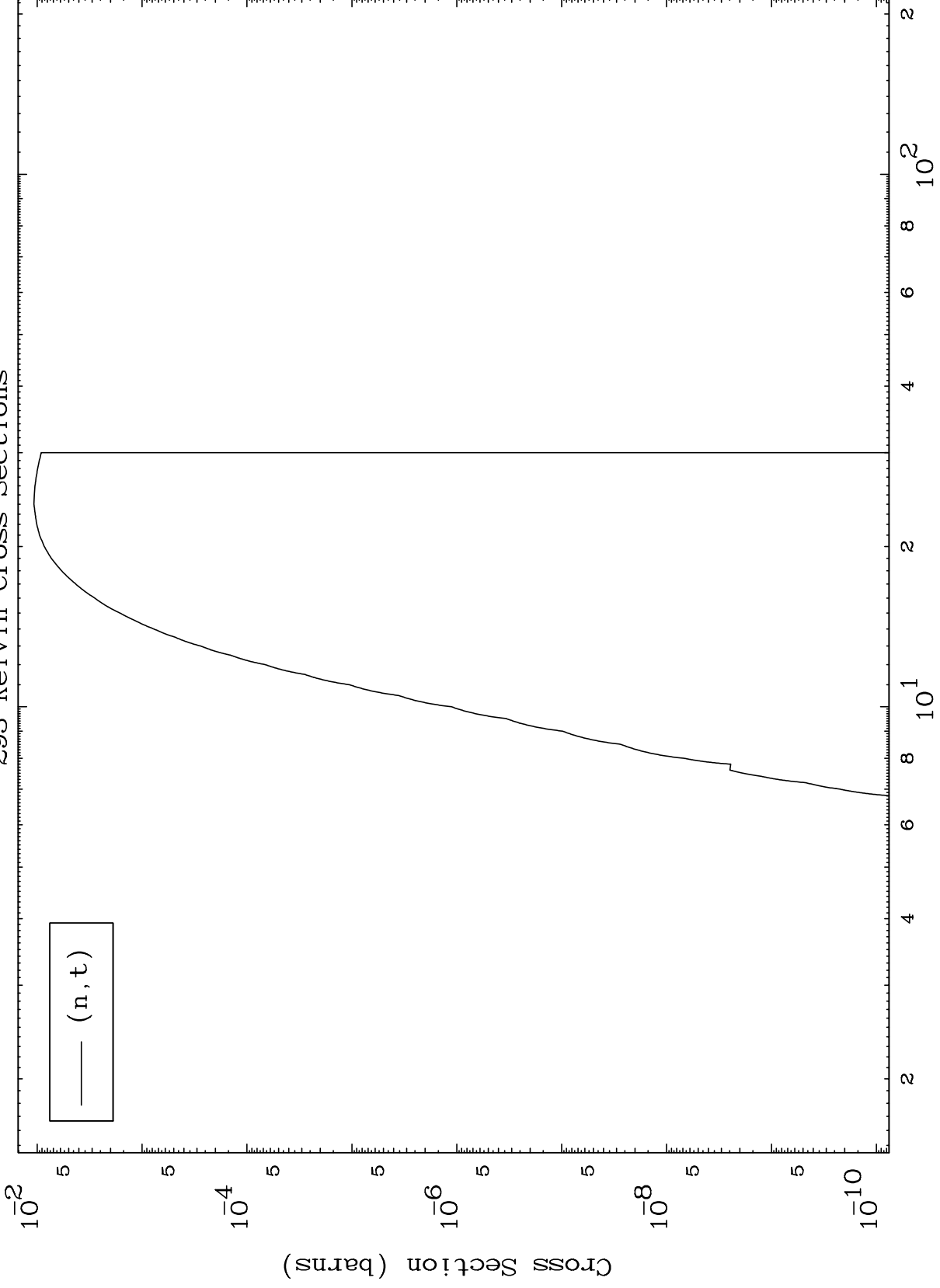
67-Ho-146



MAT 6668

(n,t) Levels  
293 Kelvin Cross Sections

67-Ho-146



10

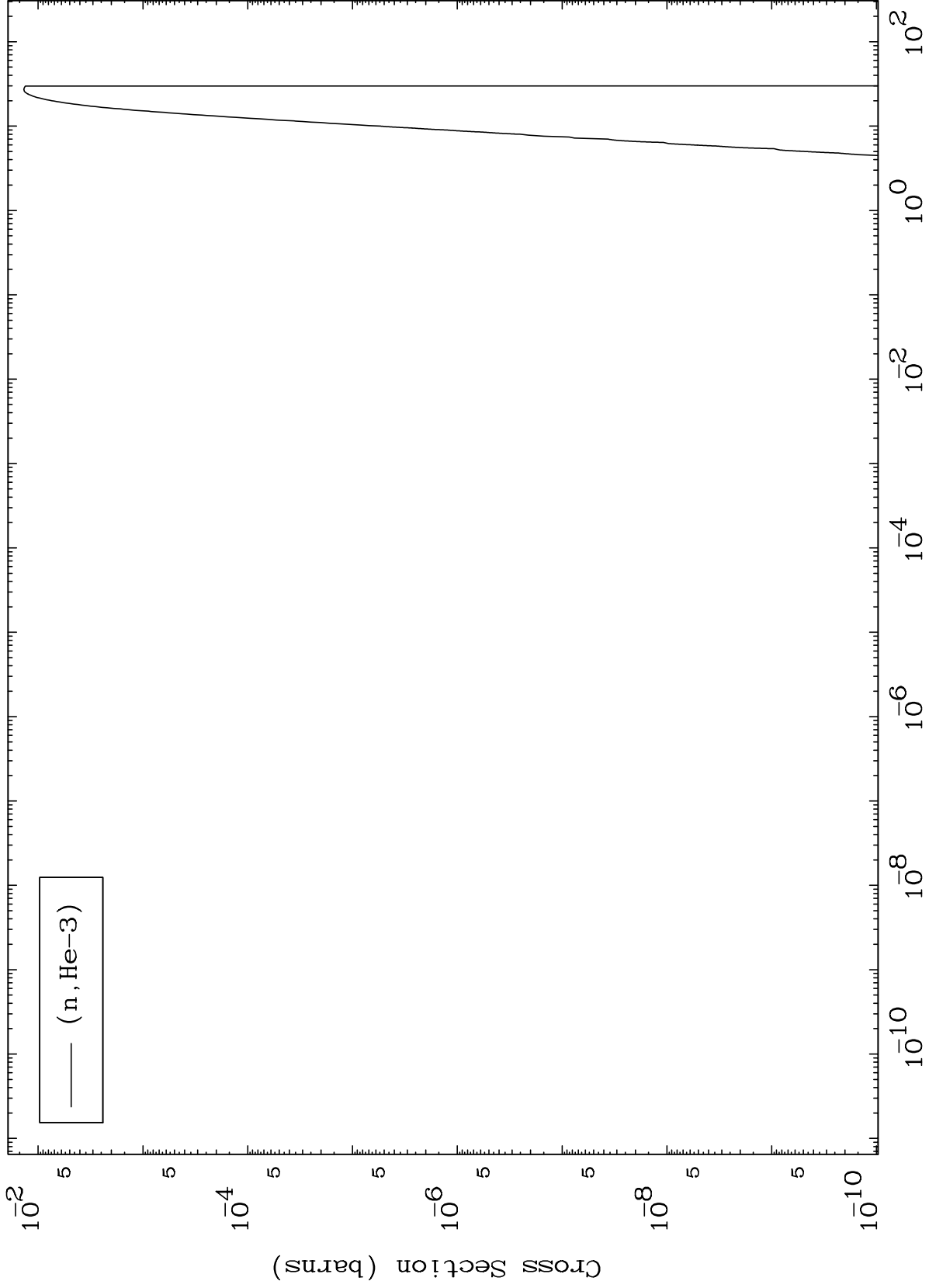
Incident Energy (MeV)

67-Ho-146

MAT 6668

(n,He3) Levels  
293 Kelvin Cross Sections

67-Ho-146

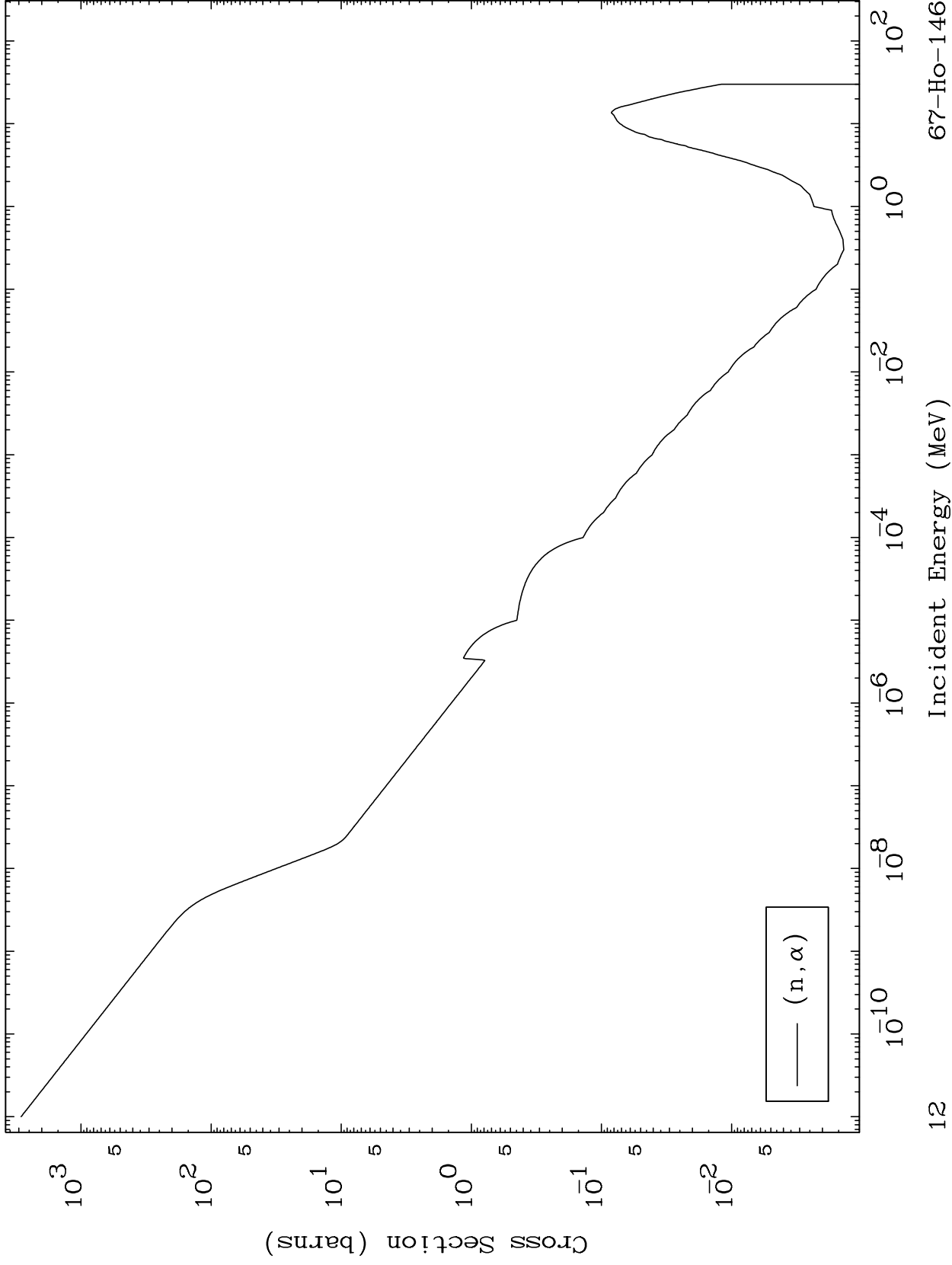


(n, He-3)

MAT 6668

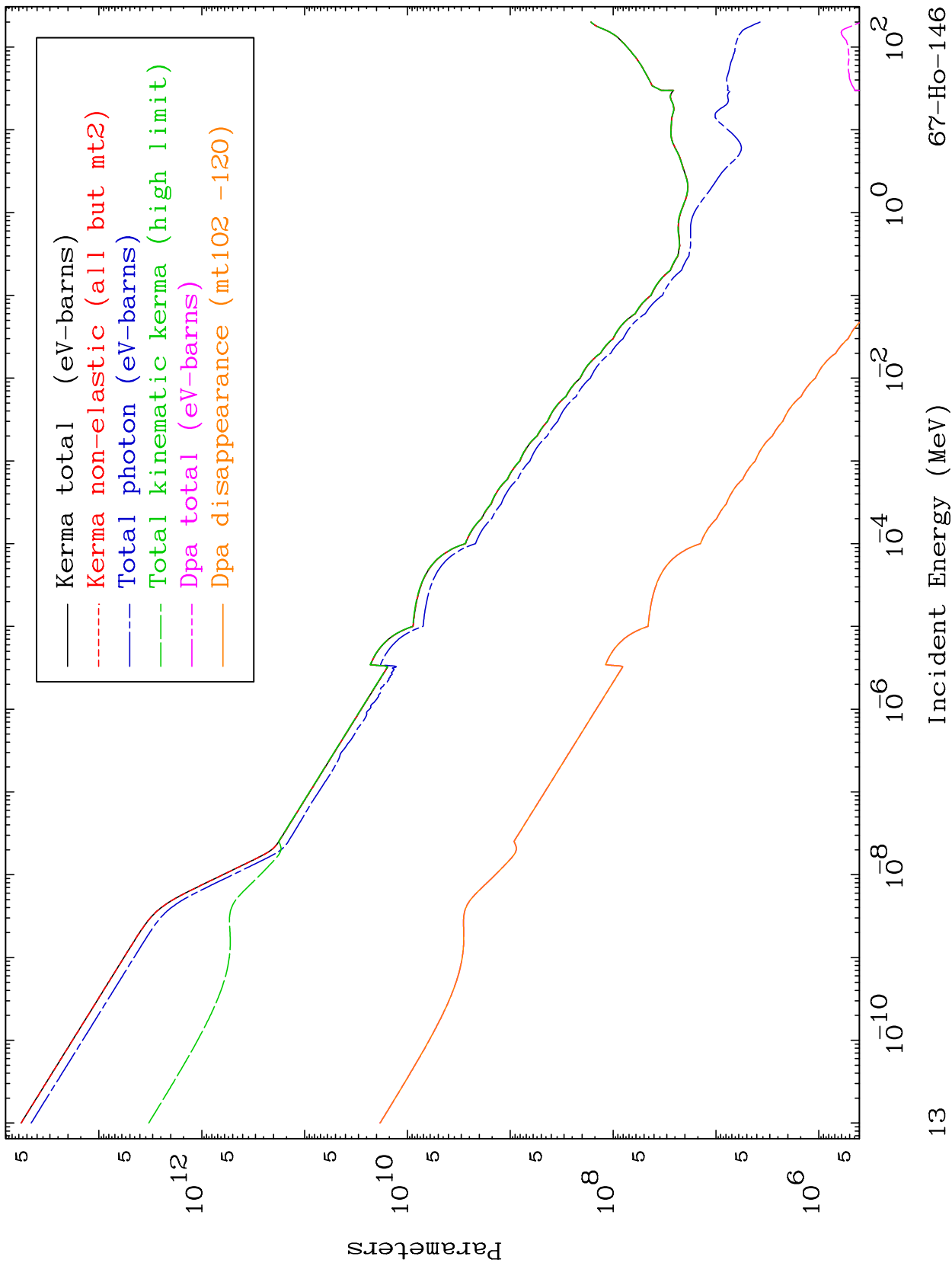
(n,α) Levels  
293 Kelvin Cross Sections

67-Ho-146



12

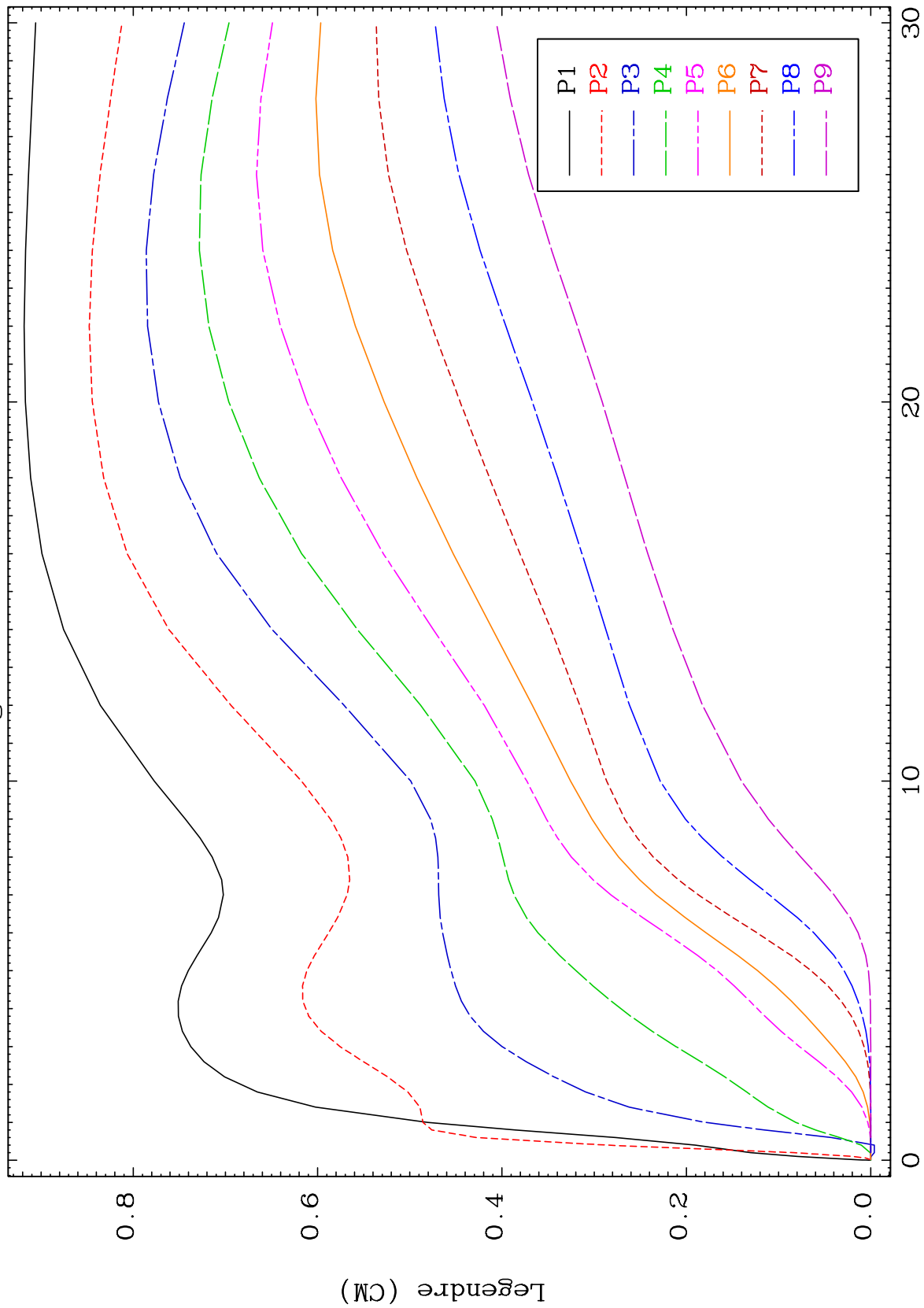
67-Ho-146



MAT 6668

Elastic Legendre Coefficients

67-Ho-146



14

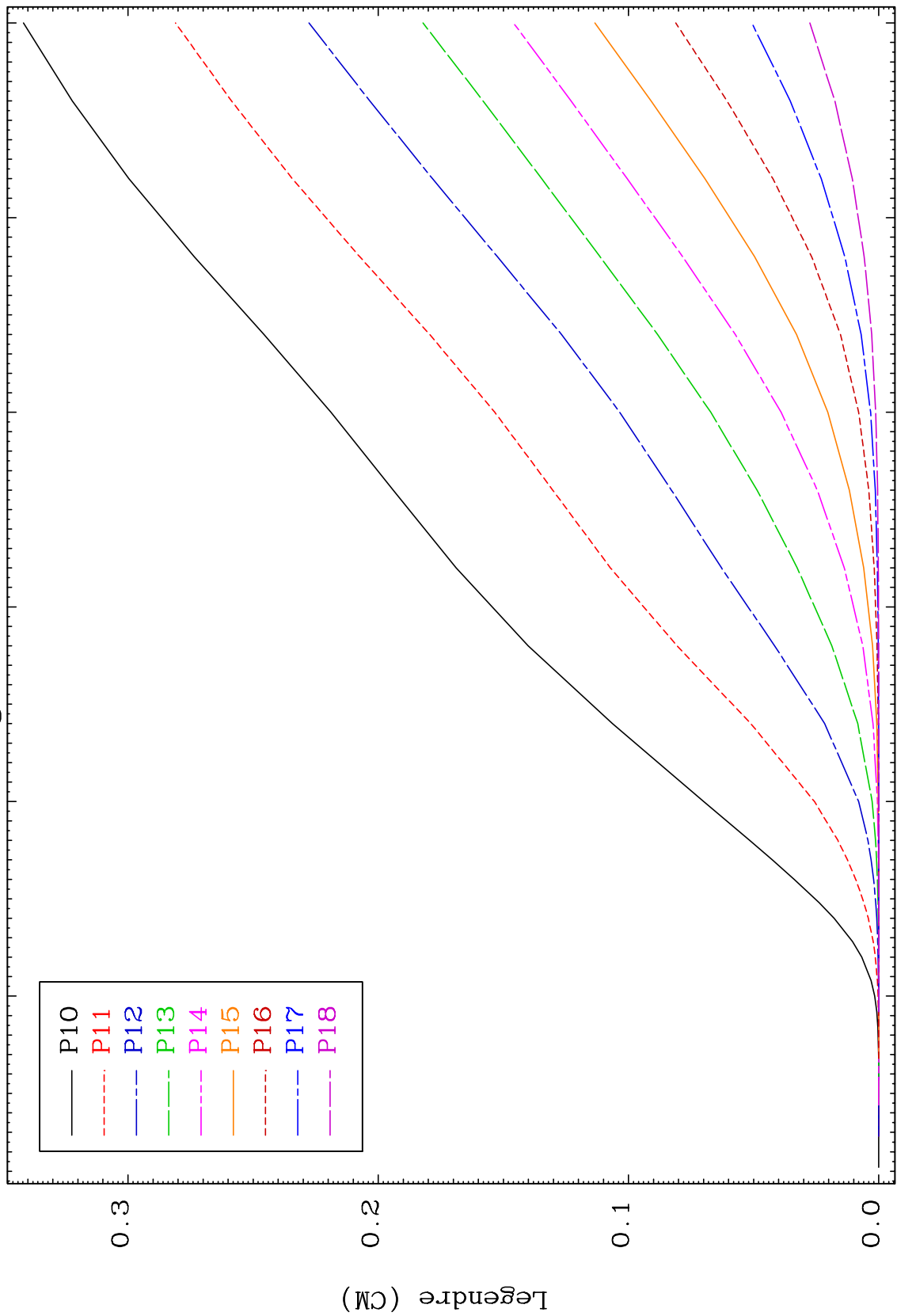
Incident Energy (MeV)

67-Ho-146

MAT 6668

Elastic Legendre Coefficients

67-Ho-146



15

Incident Energy (MeV)

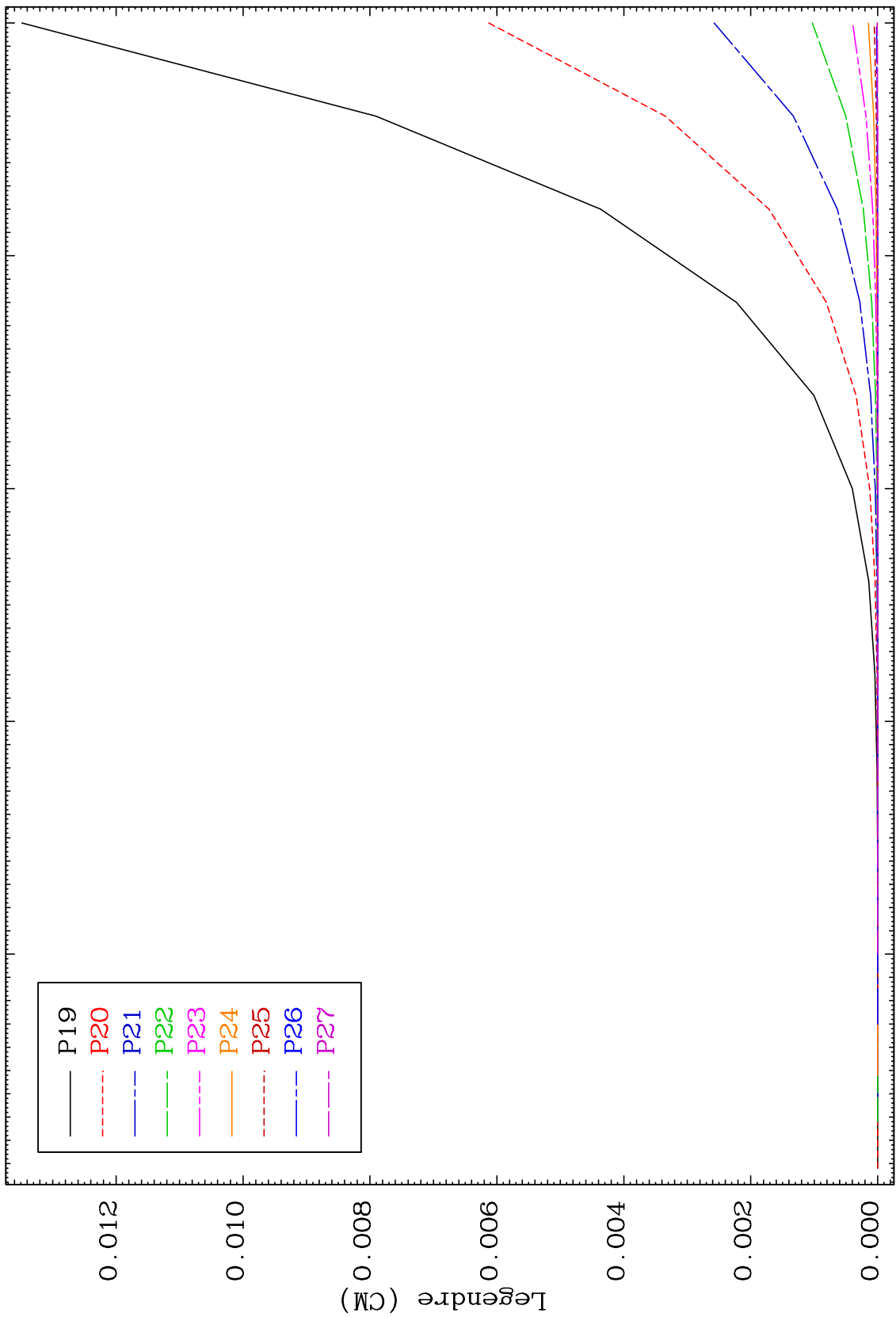
67-Ho-146



MAT 6668

### Elastic Legendre Coefficients

67-Ho-146



16

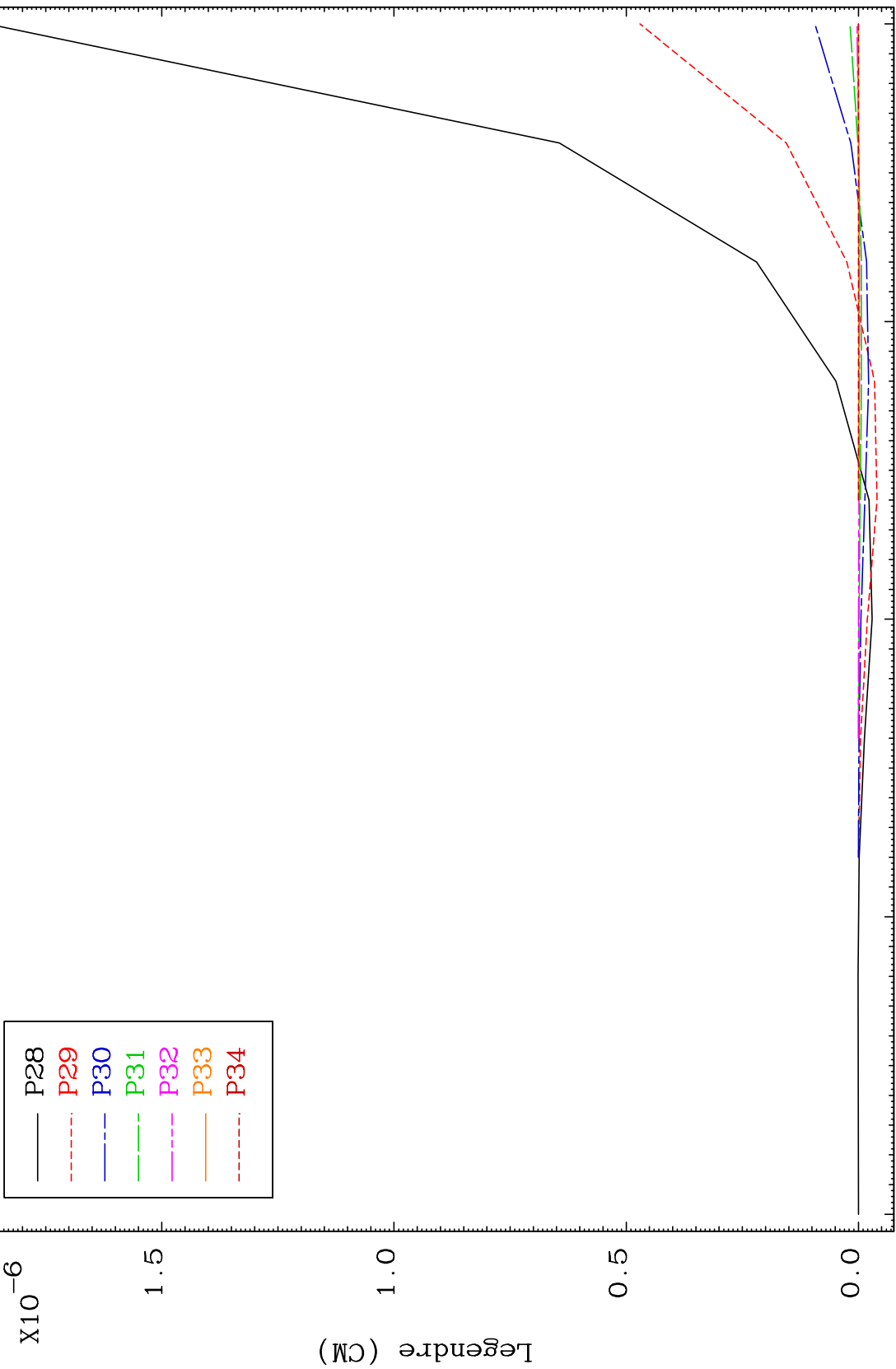
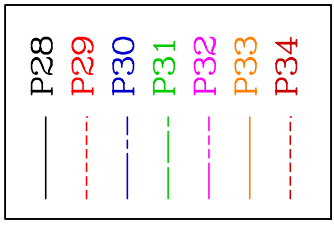
Incident Energy (MeV)

67-Ho-146

MAT 6668

Elastic Legendre Coefficients

67-Ho-146



67-Ho-146

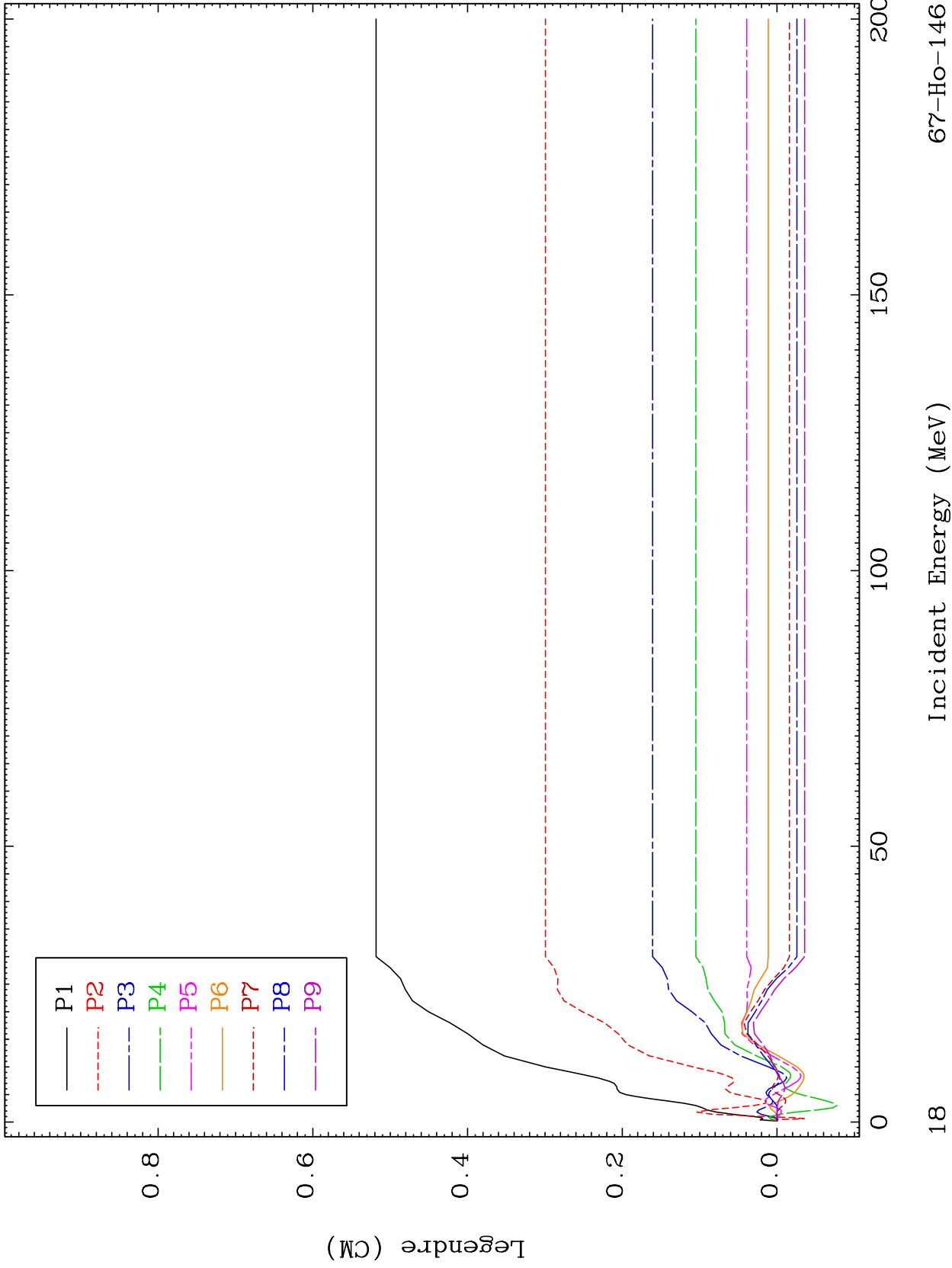
Incident Energy (MeV)

17

MAT 6668

MT= 51 (n,n') Level  
Legendre Coefficients

67-Ho-146



18

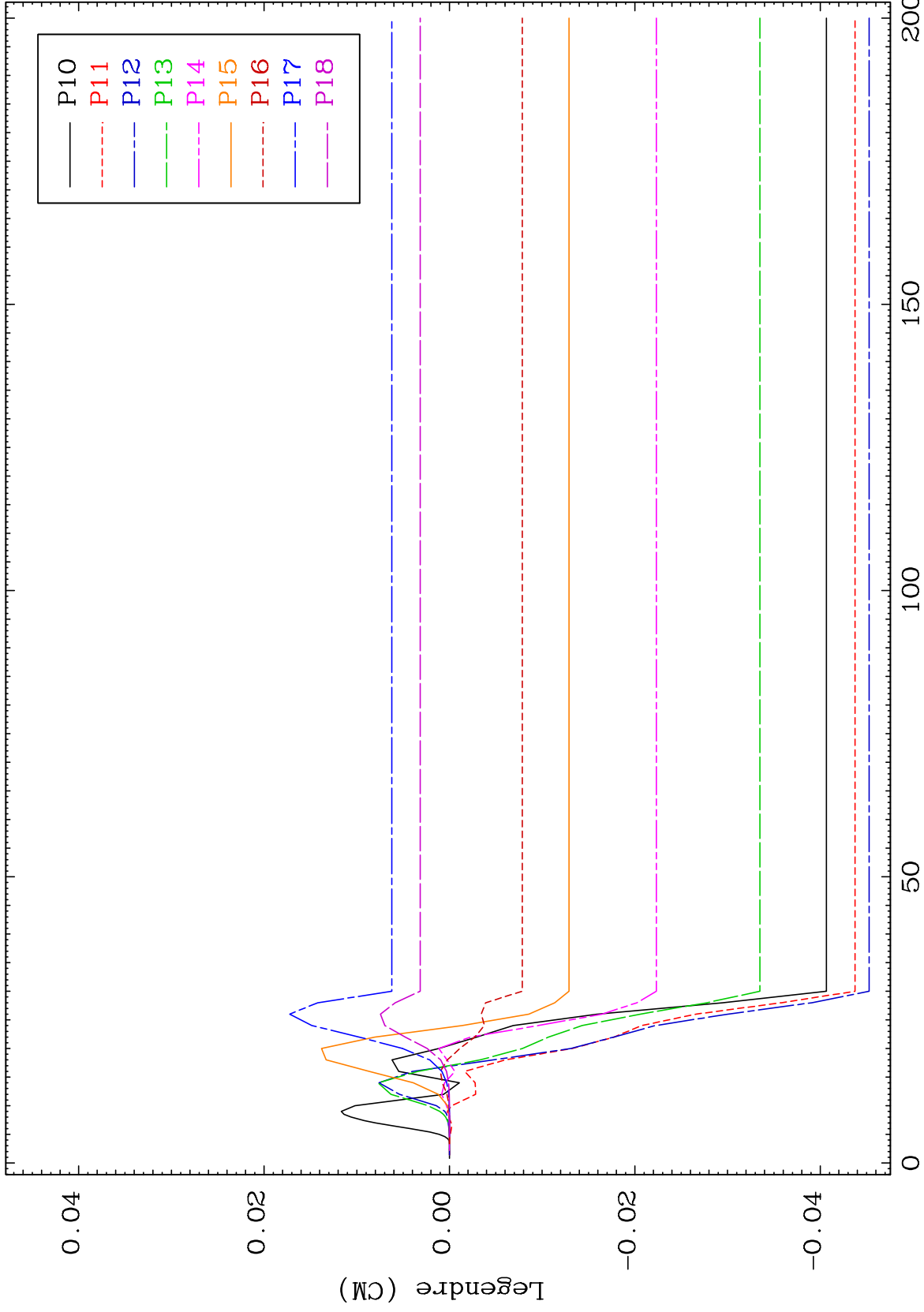
Incident Energy (MeV)

67-Ho-146

MAT 6668

MT= 51 (n,n') Level  
Legendre Coefficients

67-Ho-146



19

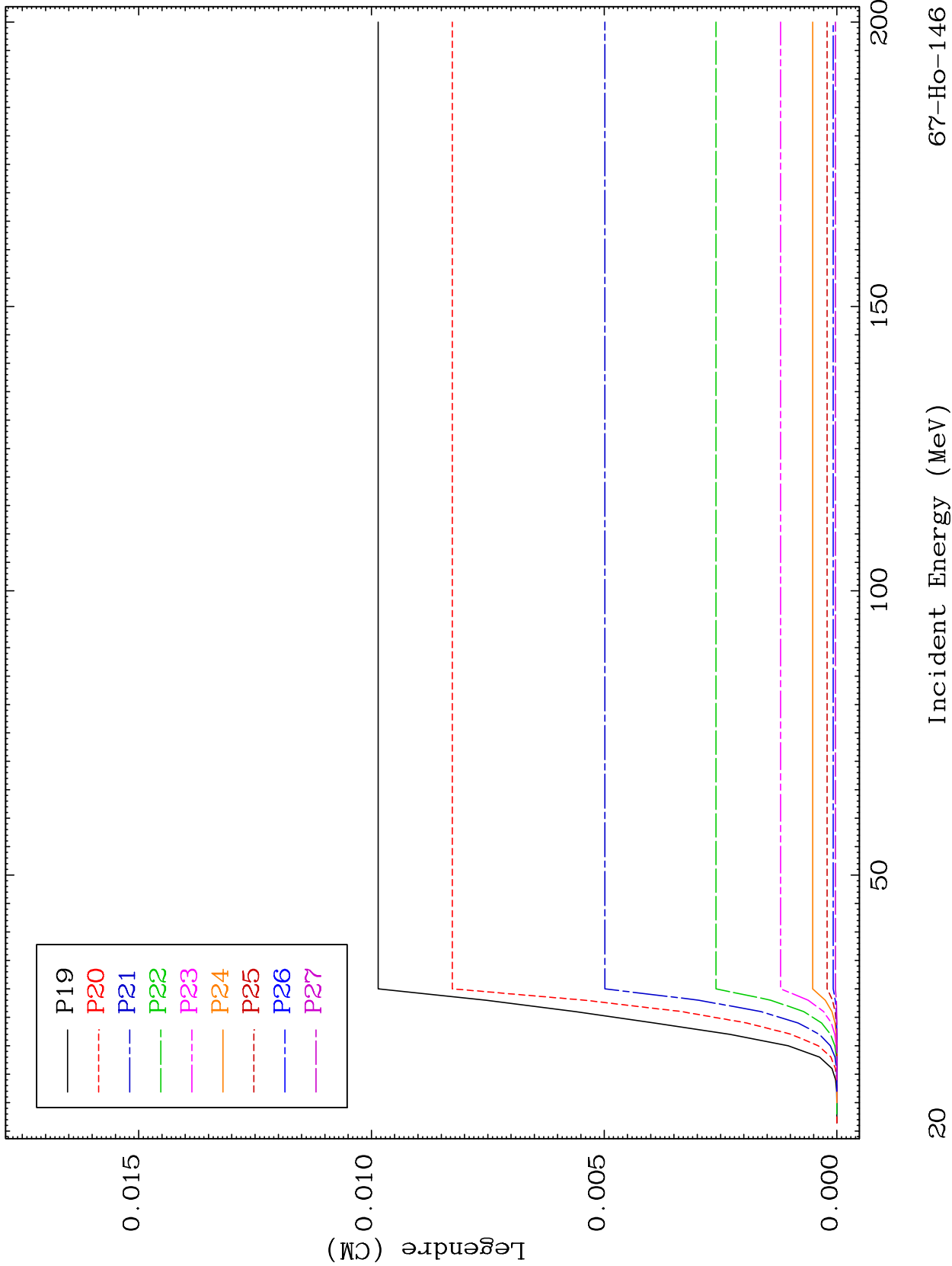
Incident Energy (MeV)

67-Ho-146

MAT 6668

MT= 51 (n,n') Level  
Legendre Coefficients

67-Ho-146



67-Ho-146

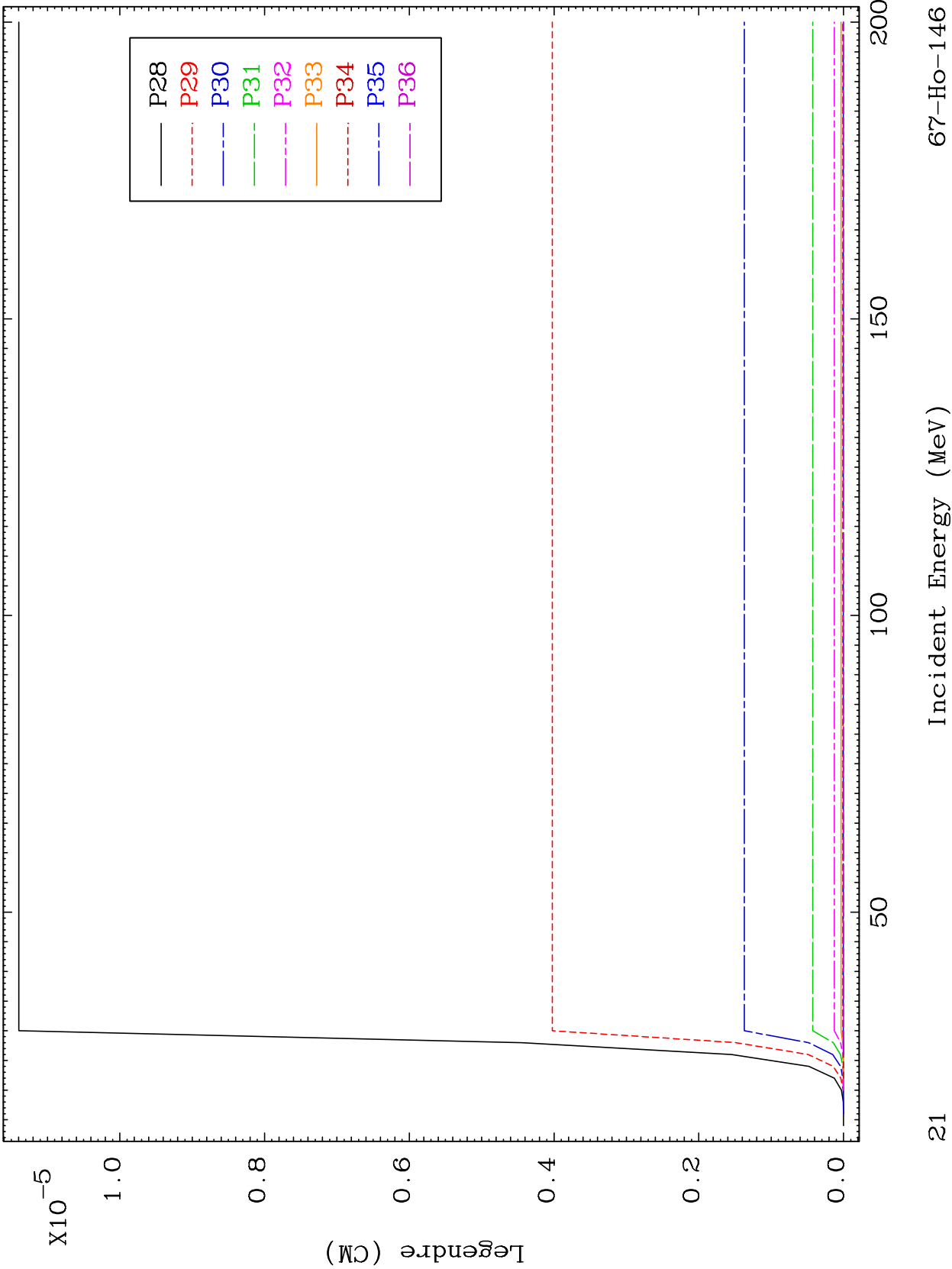
Incident Energy (MeV)

20

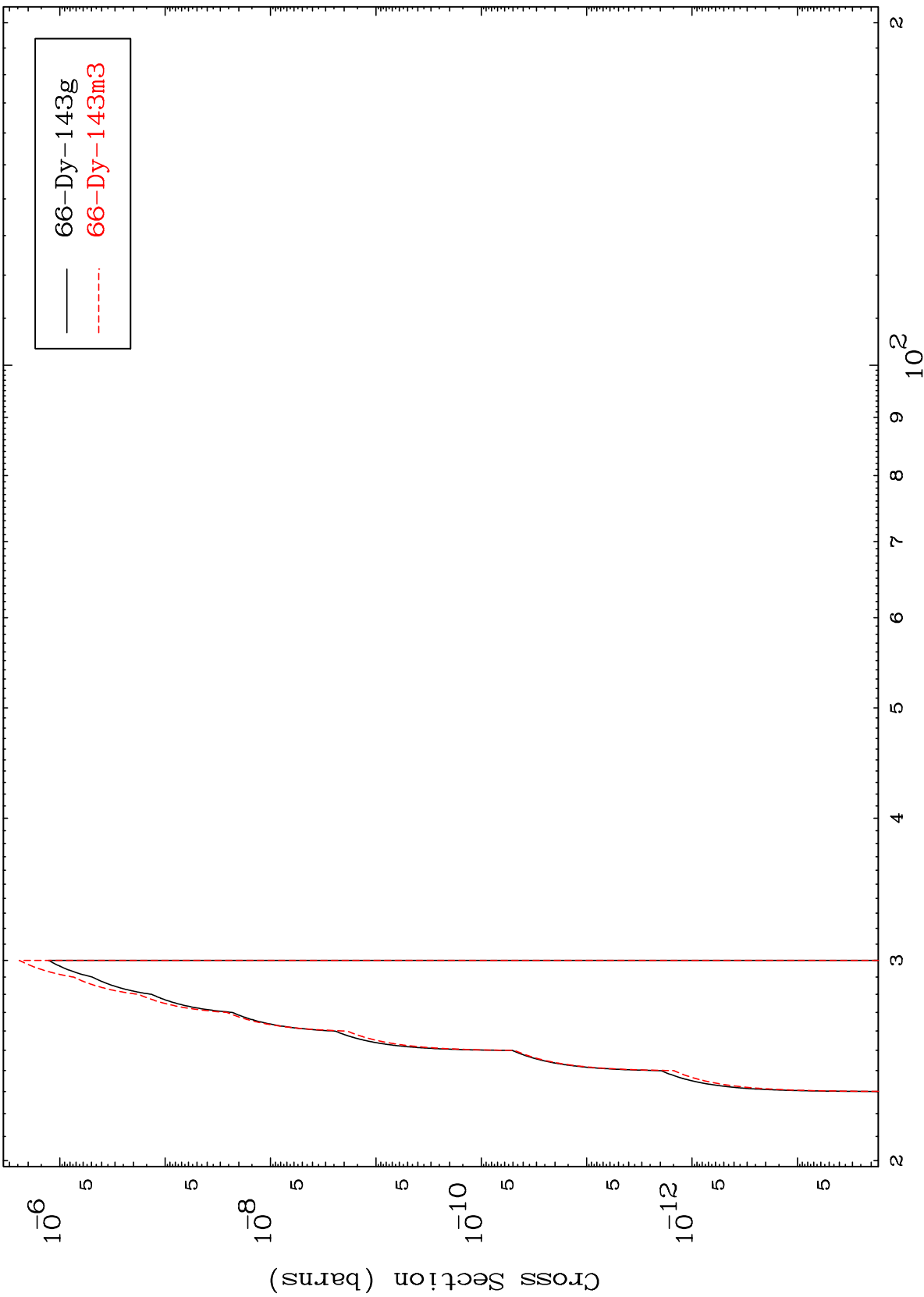
MAT 6668

MT= 51 (n,n') Level  
Legendre Coefficients

67-Ho-146



Radionuclide Production Cross Section

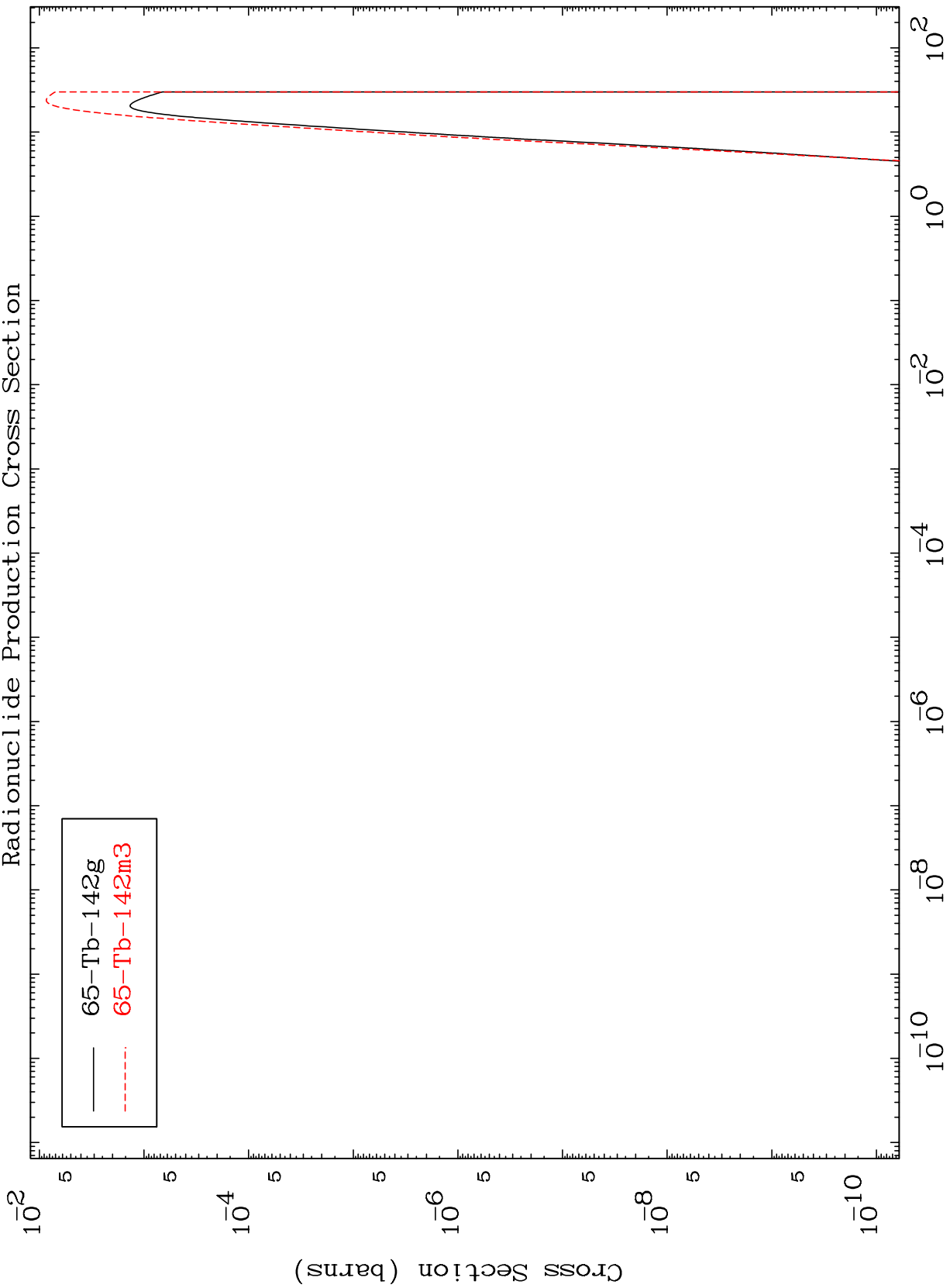


MAT 6668

$(n, n') \alpha$

$^{67}\text{Ho-146}$

Radionuclide Production Cross Section



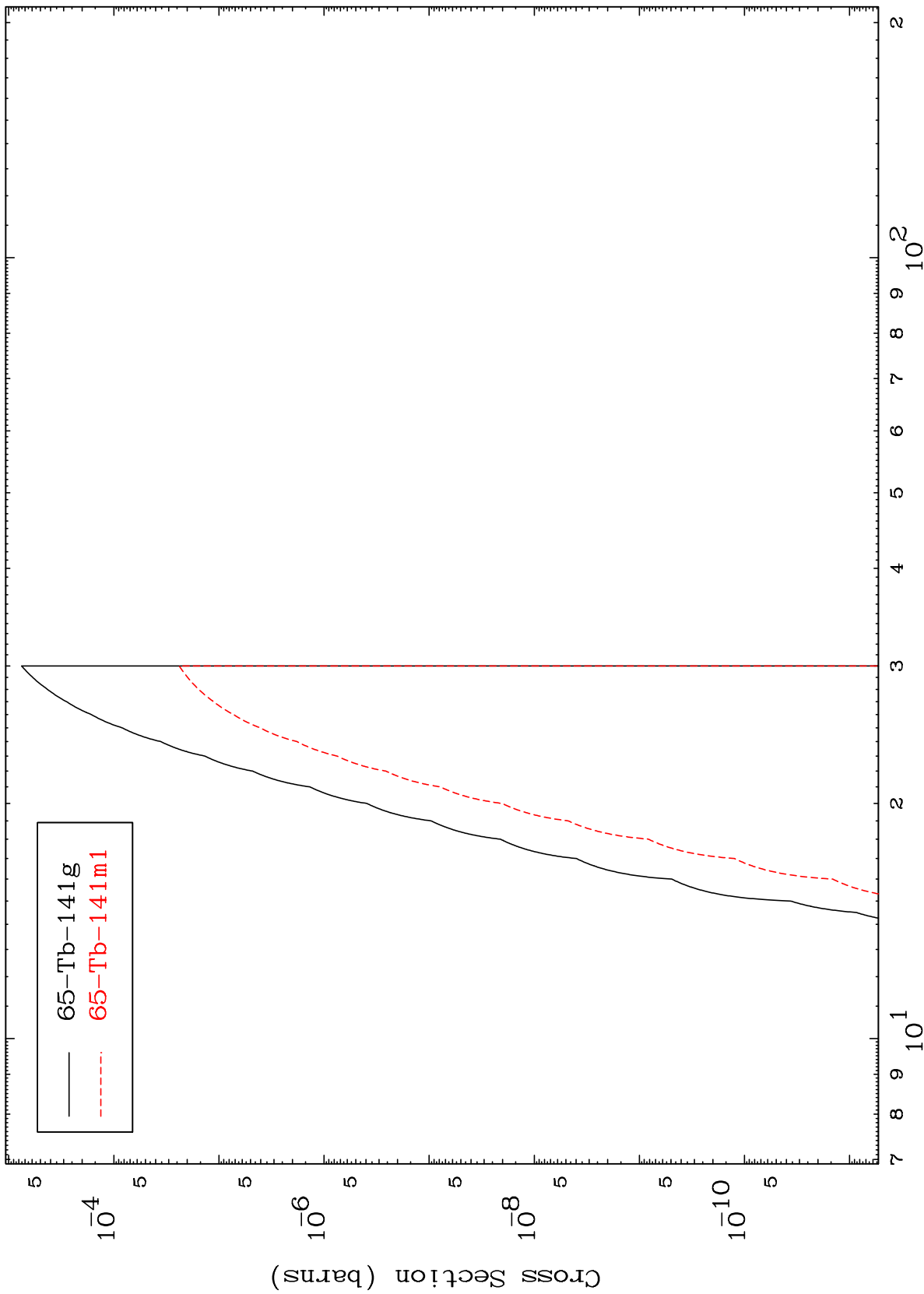


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(n,2n)  $\alpha$

67-Ho-146

Radionuclide Production Cross Section



65-Tb-141g  
65-Tb-141m1

24

Incident Energy (MeV)

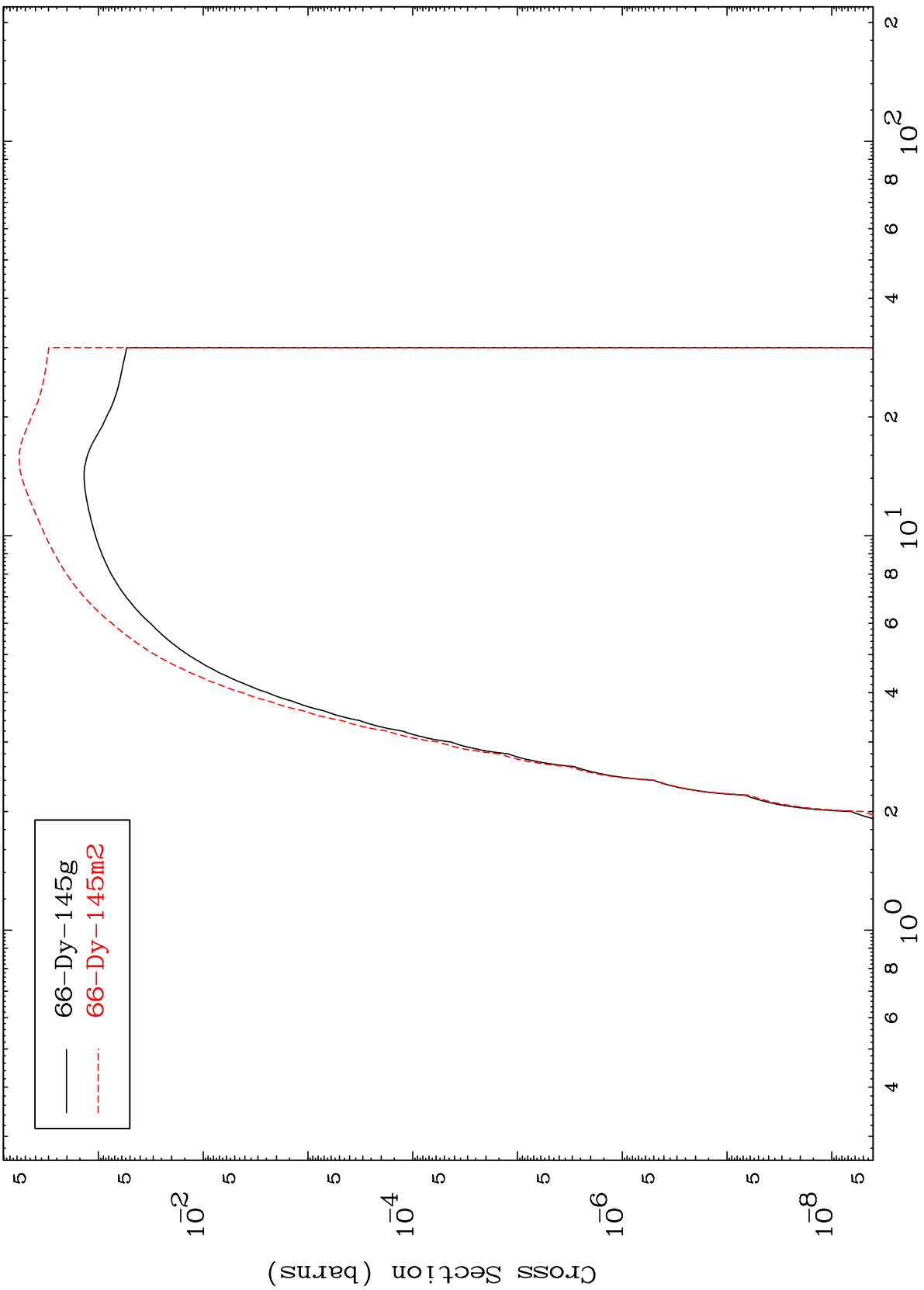
67-Ho-146

MAT 6668

(n,n') p

67-Ho-146

Radionuclide Production Cross Section

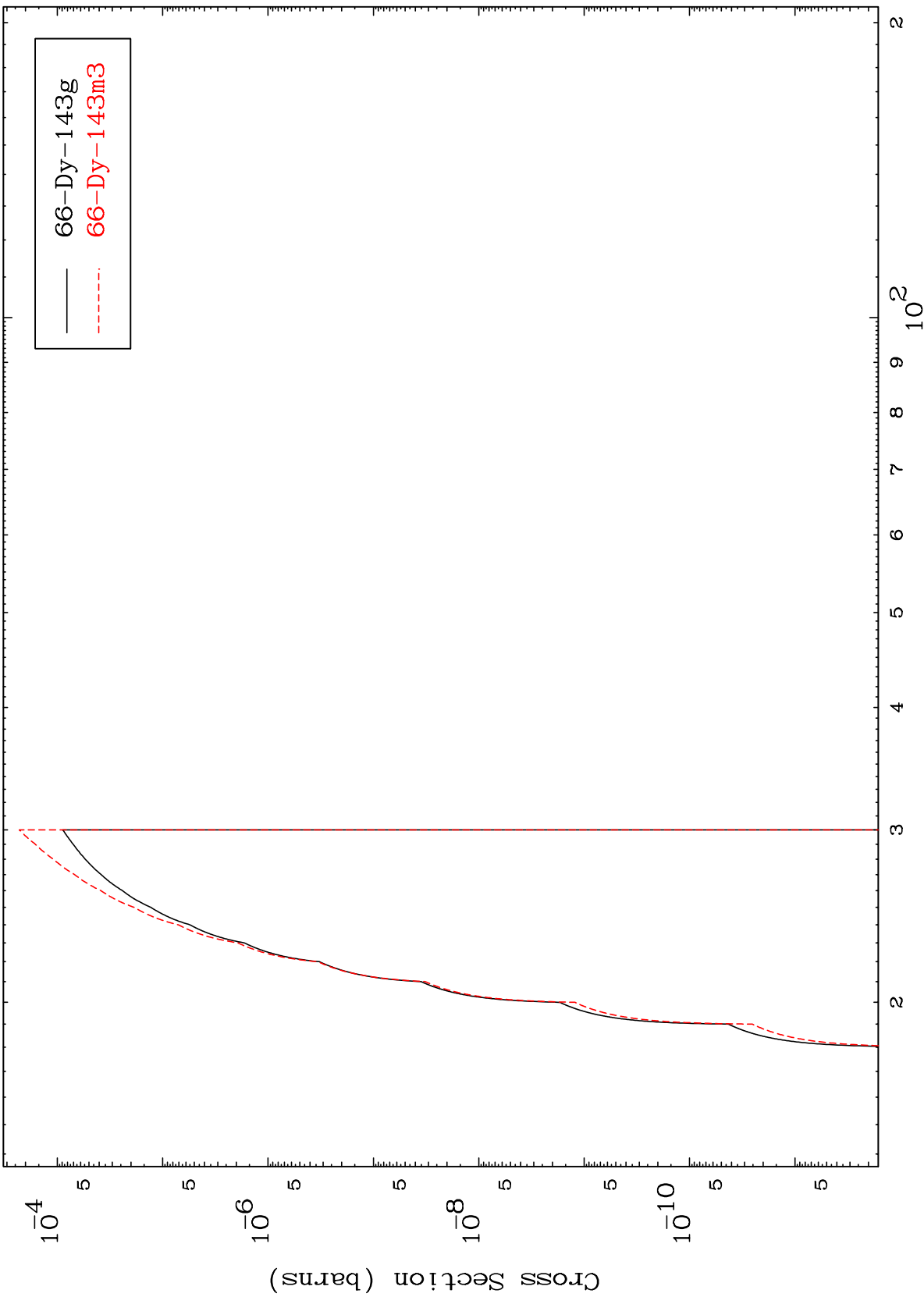


25

Incident Energy (MeV)

67-Ho-146

Radionuclide Production Cross Section

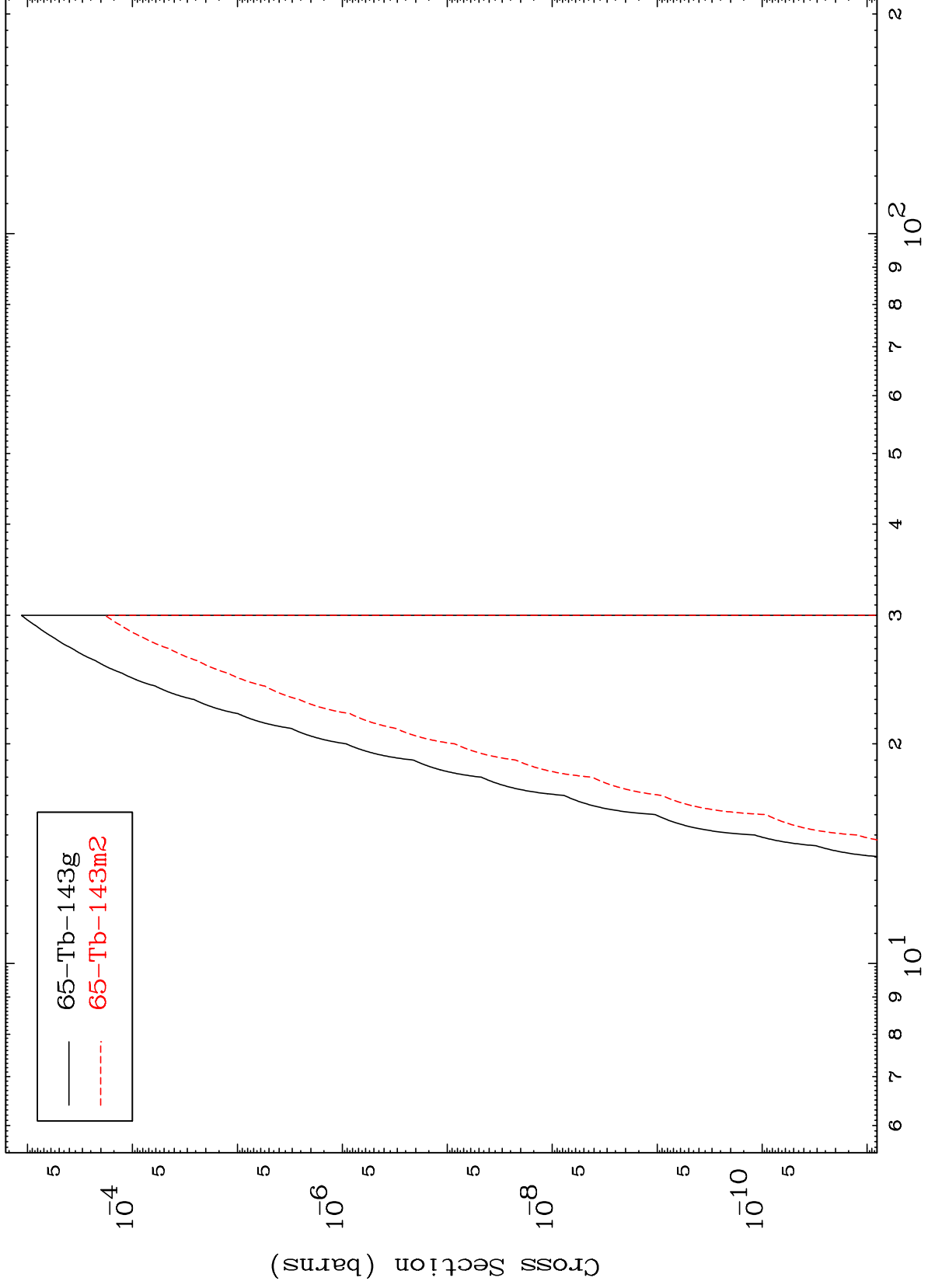


MAT 6668

(n,n') He-3

67-Ho-146

Radionuclide Production Cross Section



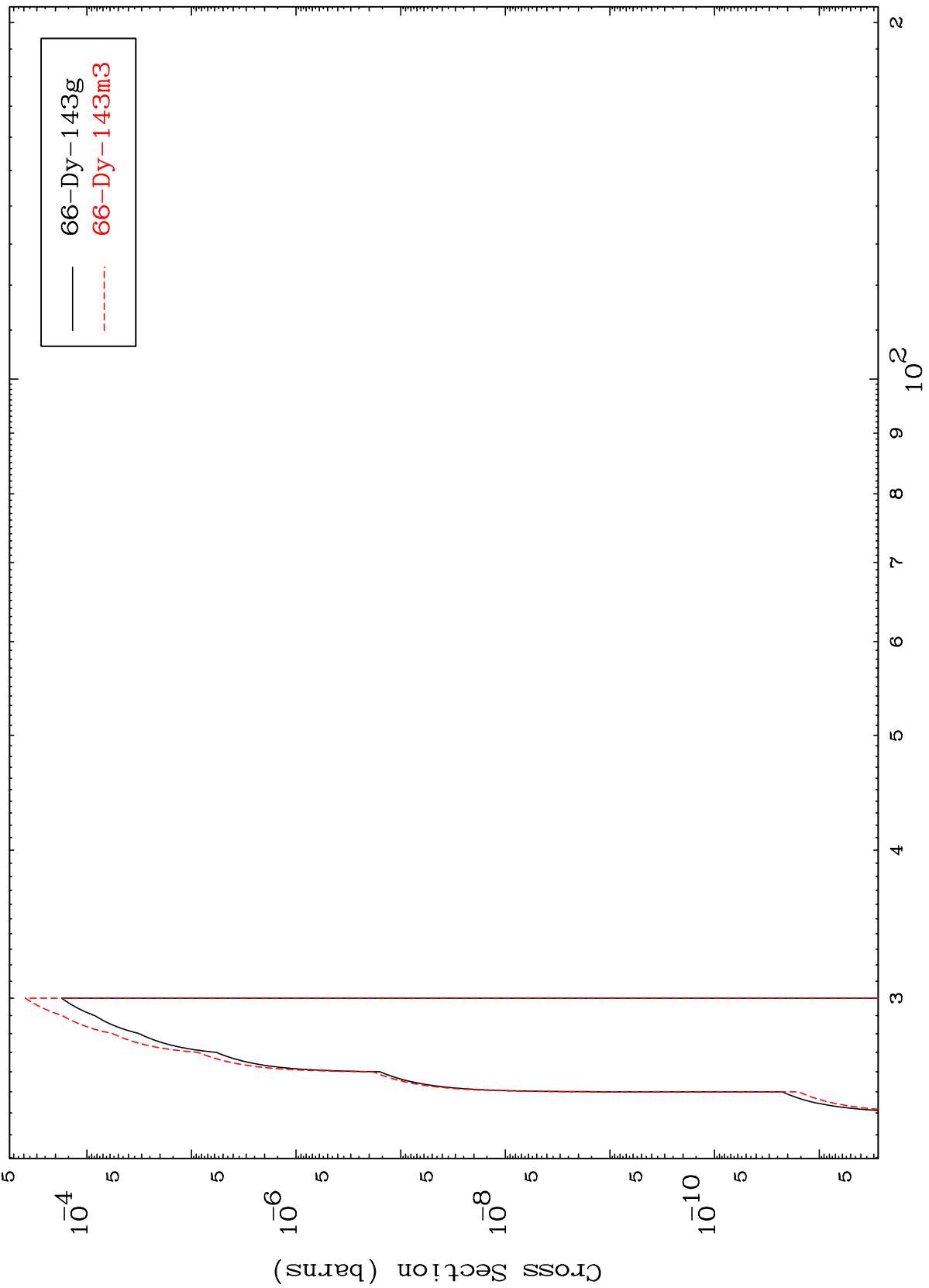
65-Tb-143g  
65-Tb-143m2

27

Incident Energy (MeV)

67-Ho-146

Radionuclide Production Cross Section

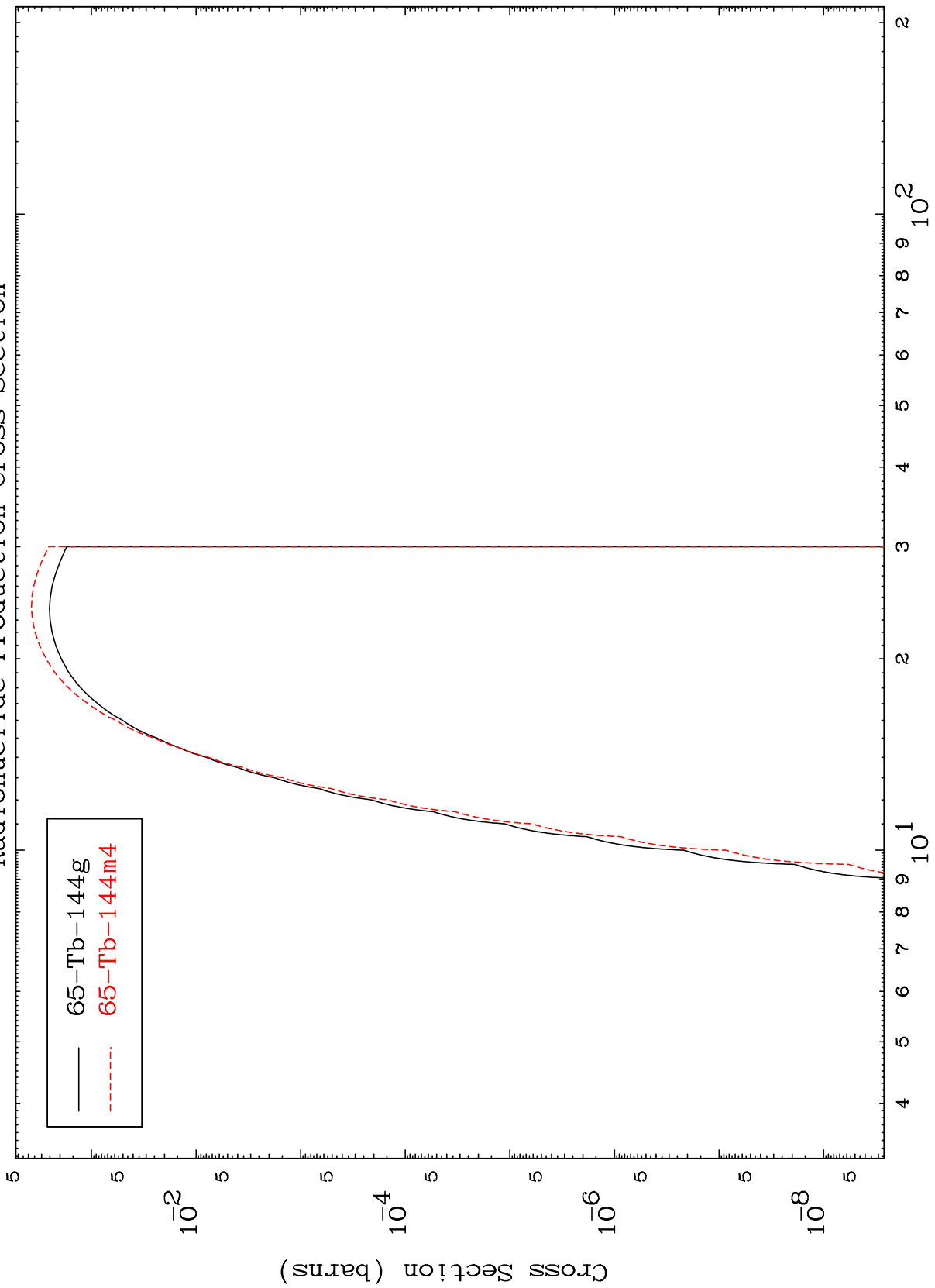


MAT 6668

$(n,2n)$  p

67-Ho-146

Radionuclide Production Cross Section



29

Incident Energy (MeV)

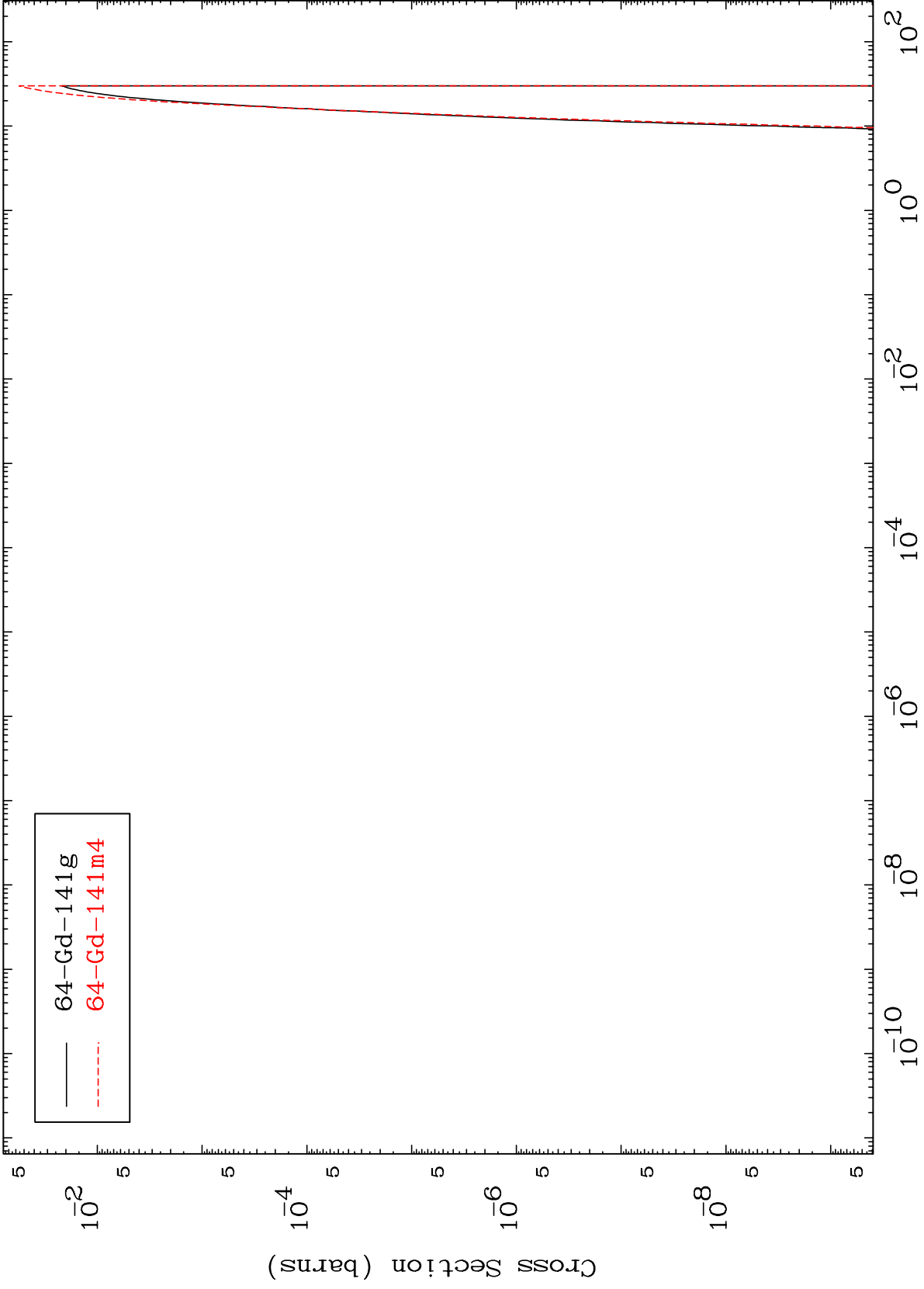
67-Ho-146

MAT 6668

(n,n') p  $\alpha$

<sup>67</sup>Ho-146

Radionuclide Production Cross Section



30

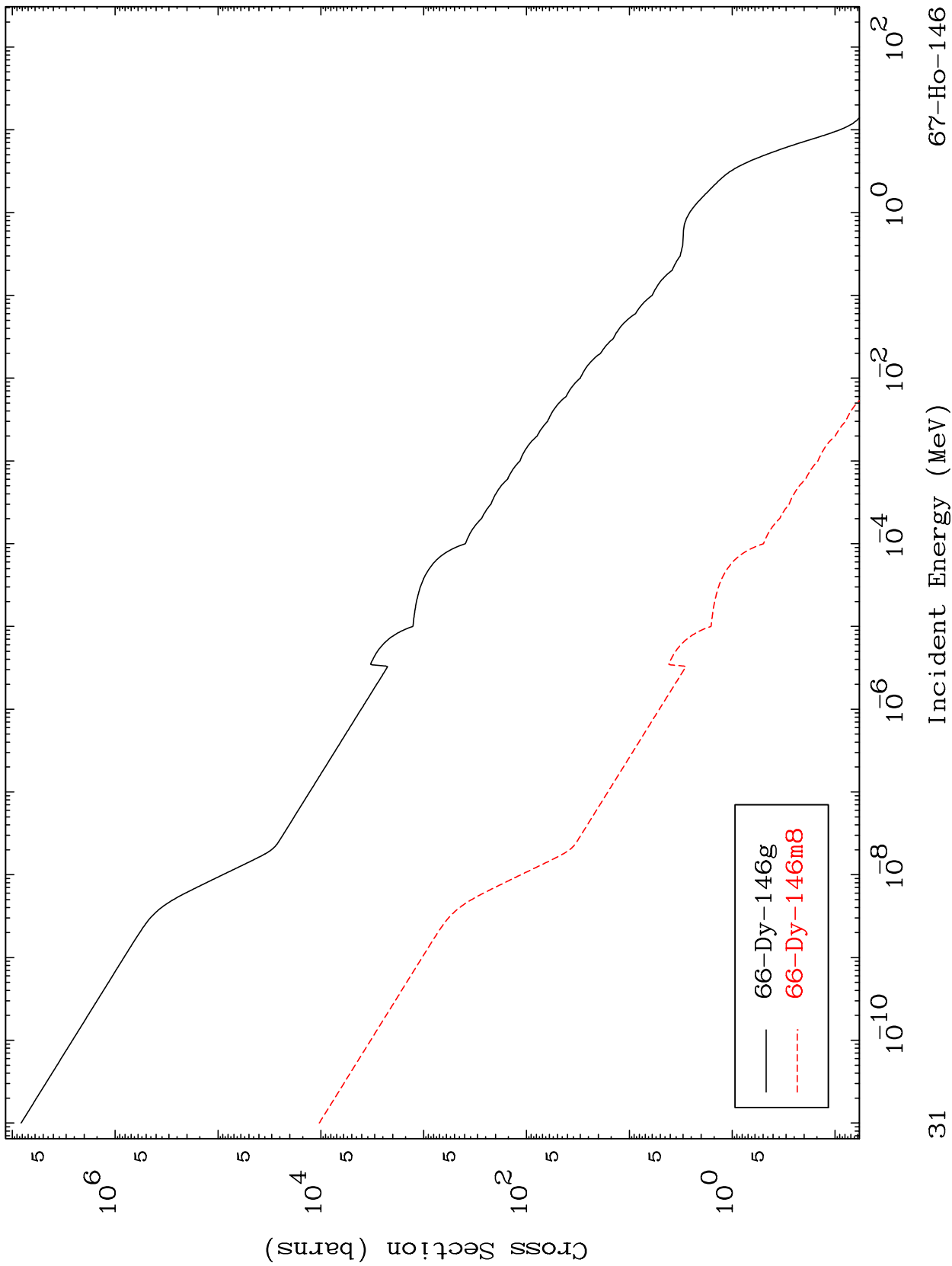
Incident Energy (MeV)

<sup>67</sup>Ho-146

MAT 6668

67-Ho-146

(n,p)  
Radionuclide Production Cross Section



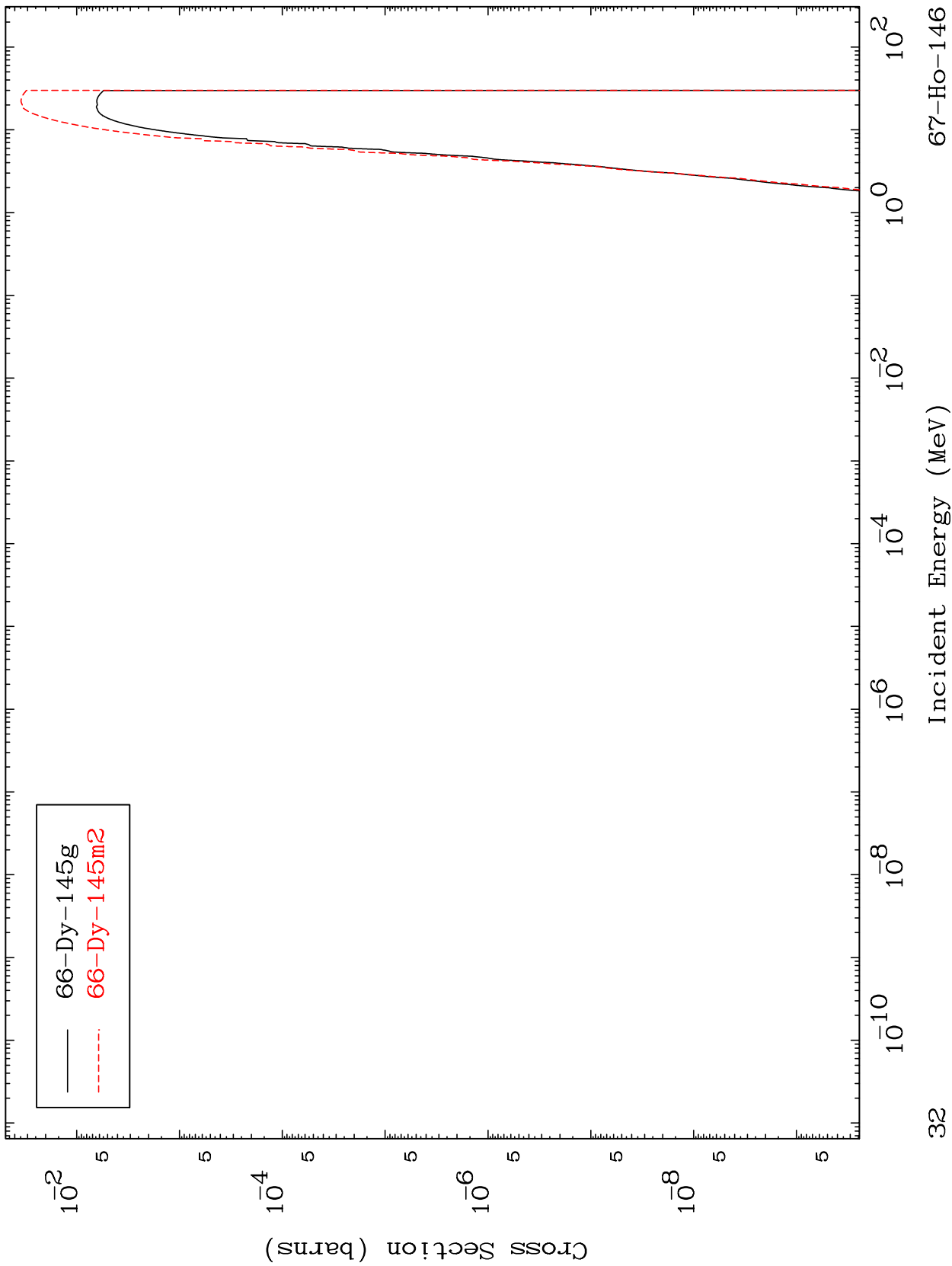


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(n,d)

<sup>67</sup>Ho-146

Radionuclide Production Cross Section



— 66-Dy-145g  
- - - 66-Dy-145m2

Incident Energy (MeV)

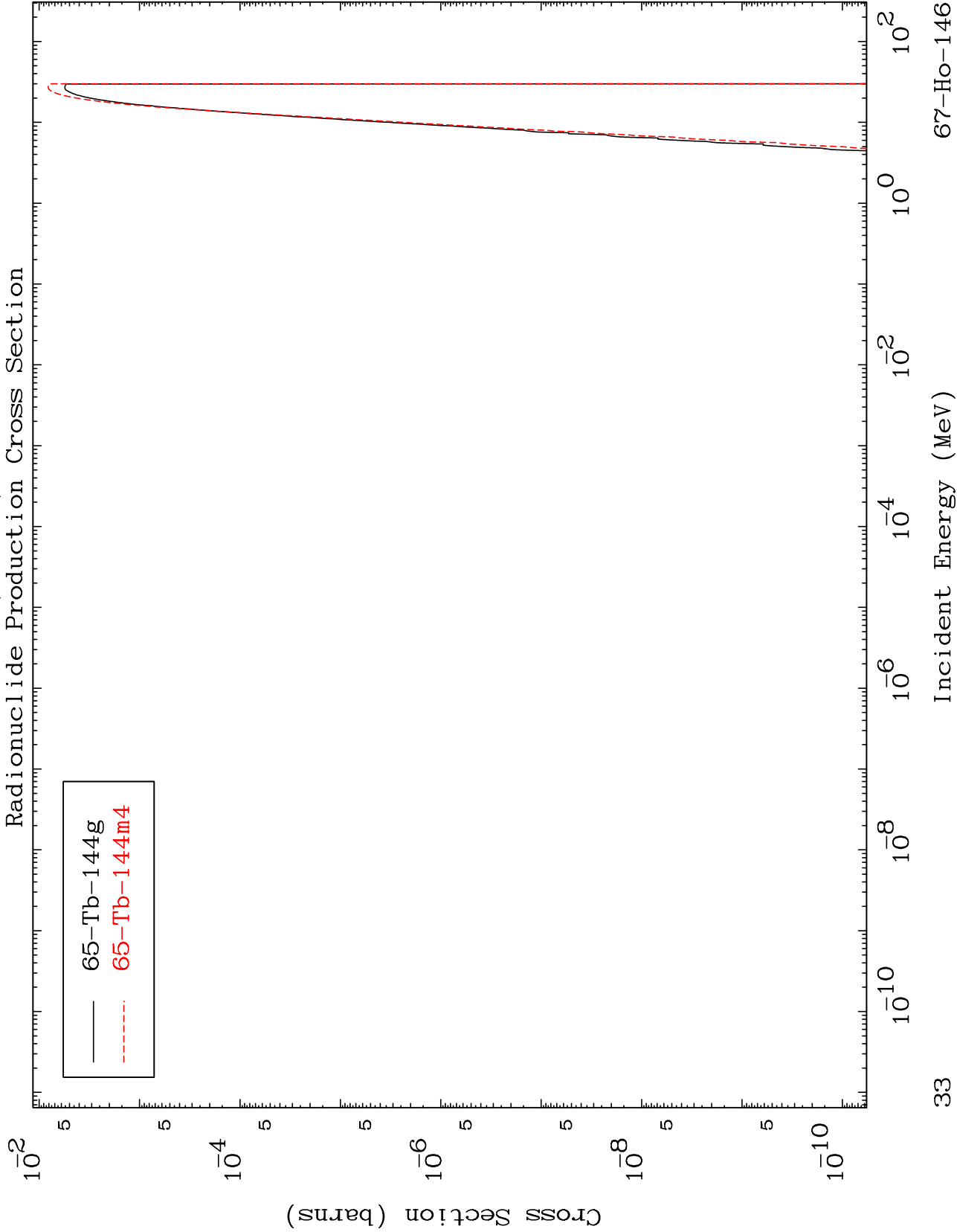
<sup>67</sup>Ho-146

MAT 6668

(n,He-3)

67-Ho-146

Radionuclide Production Cross Section

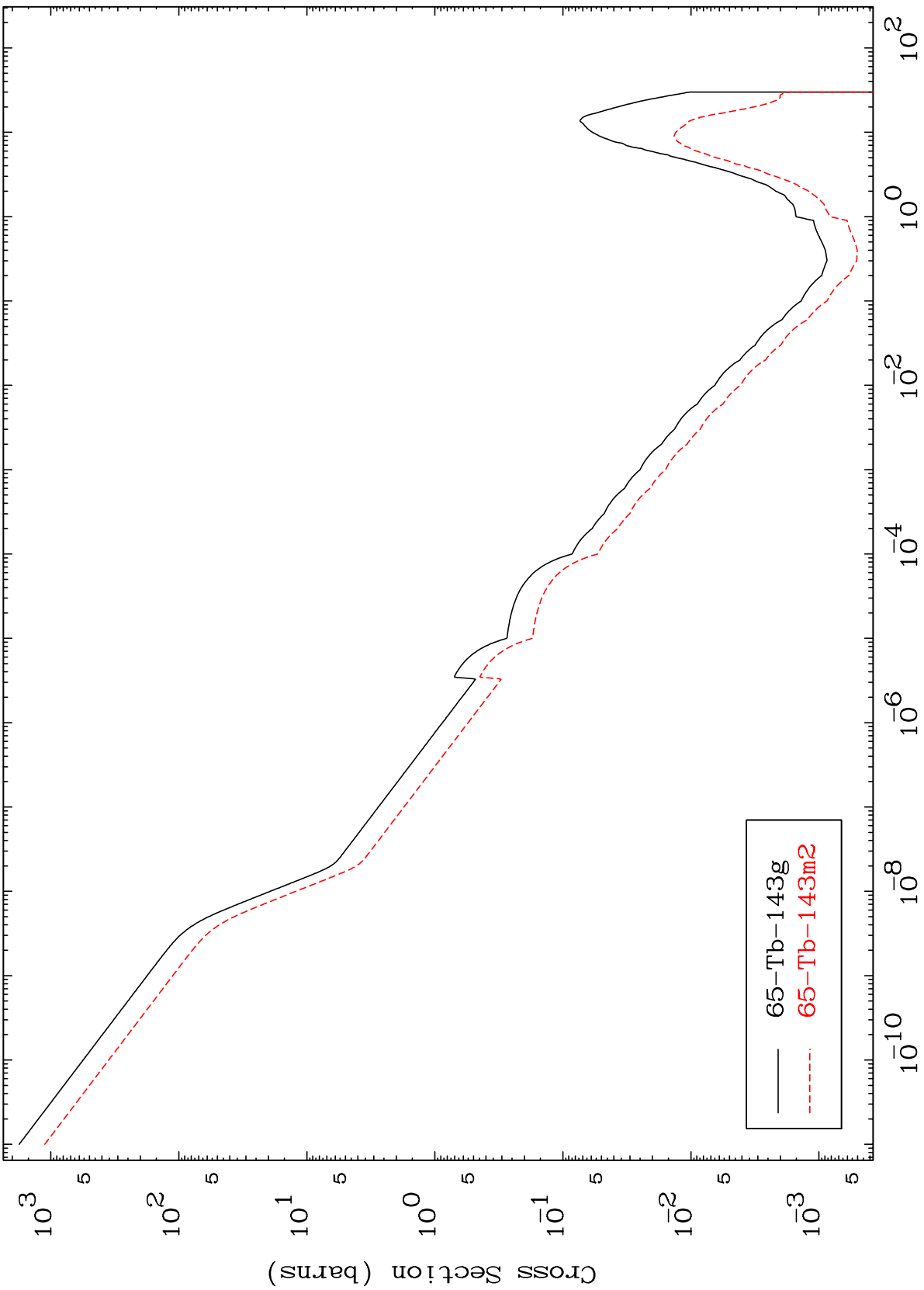


MAT 6668

67-Ho-146

Radionuclide Production Cross Section

(n,  $\alpha$ )



67-Ho-146

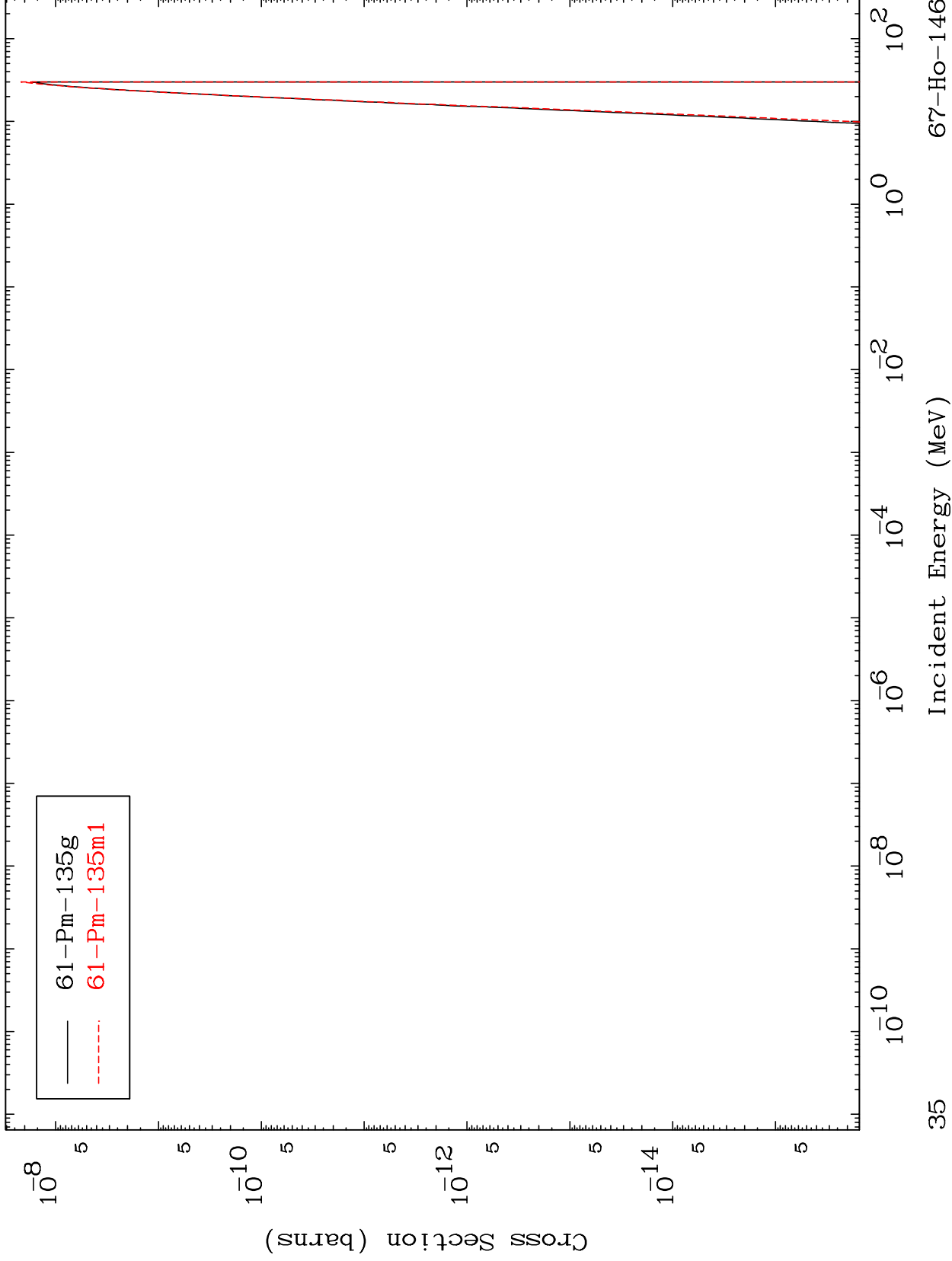
Incident Energy (MeV)

MAT 6668

(n,3α)

67-Ho-146

Radionuclide Production Cross Section



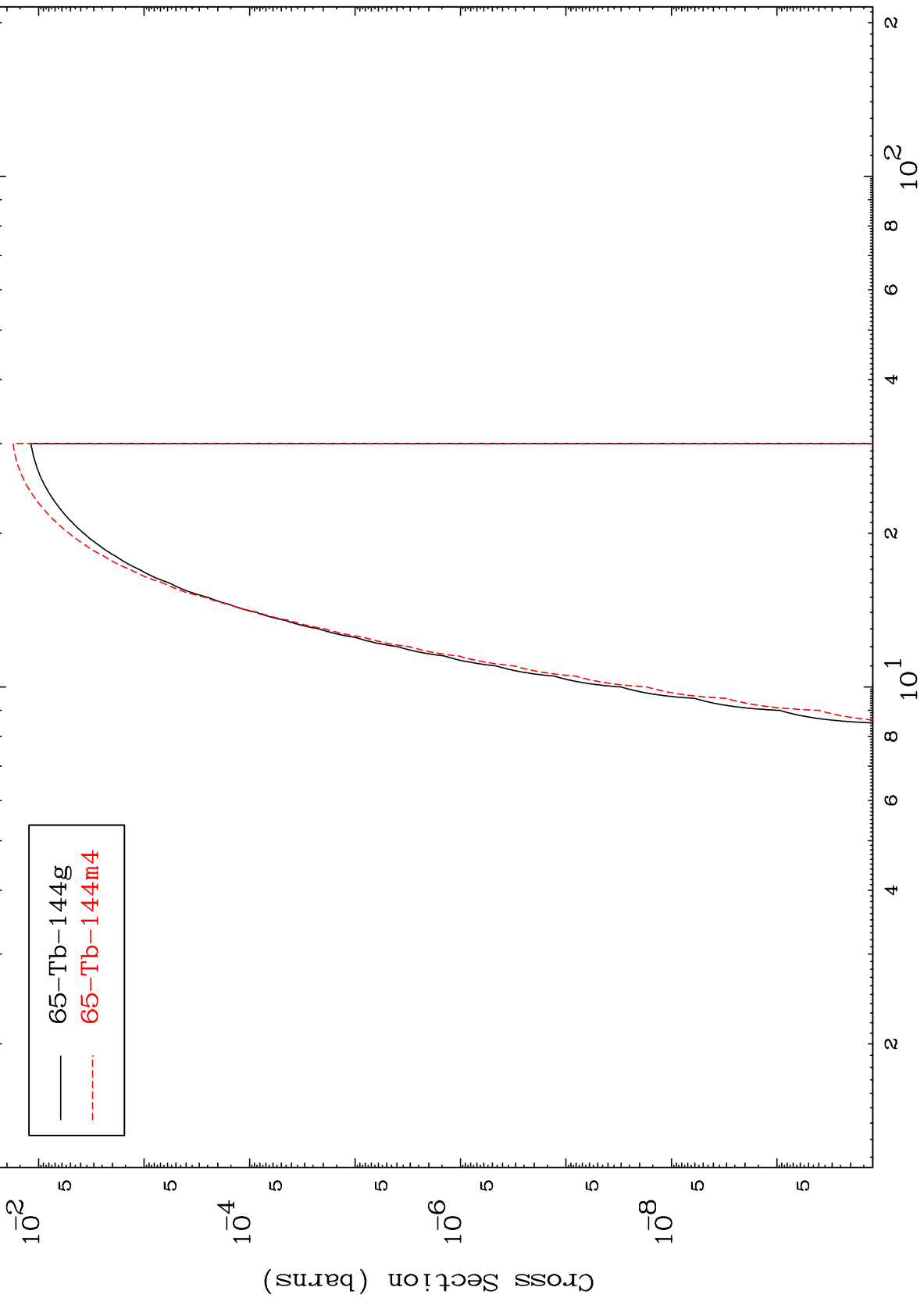
61-Pm-135g  
61-Pm-135m1

MAT 6668

(n,p) d

<sup>67</sup>Ho-146

Radionuclide Production Cross Section



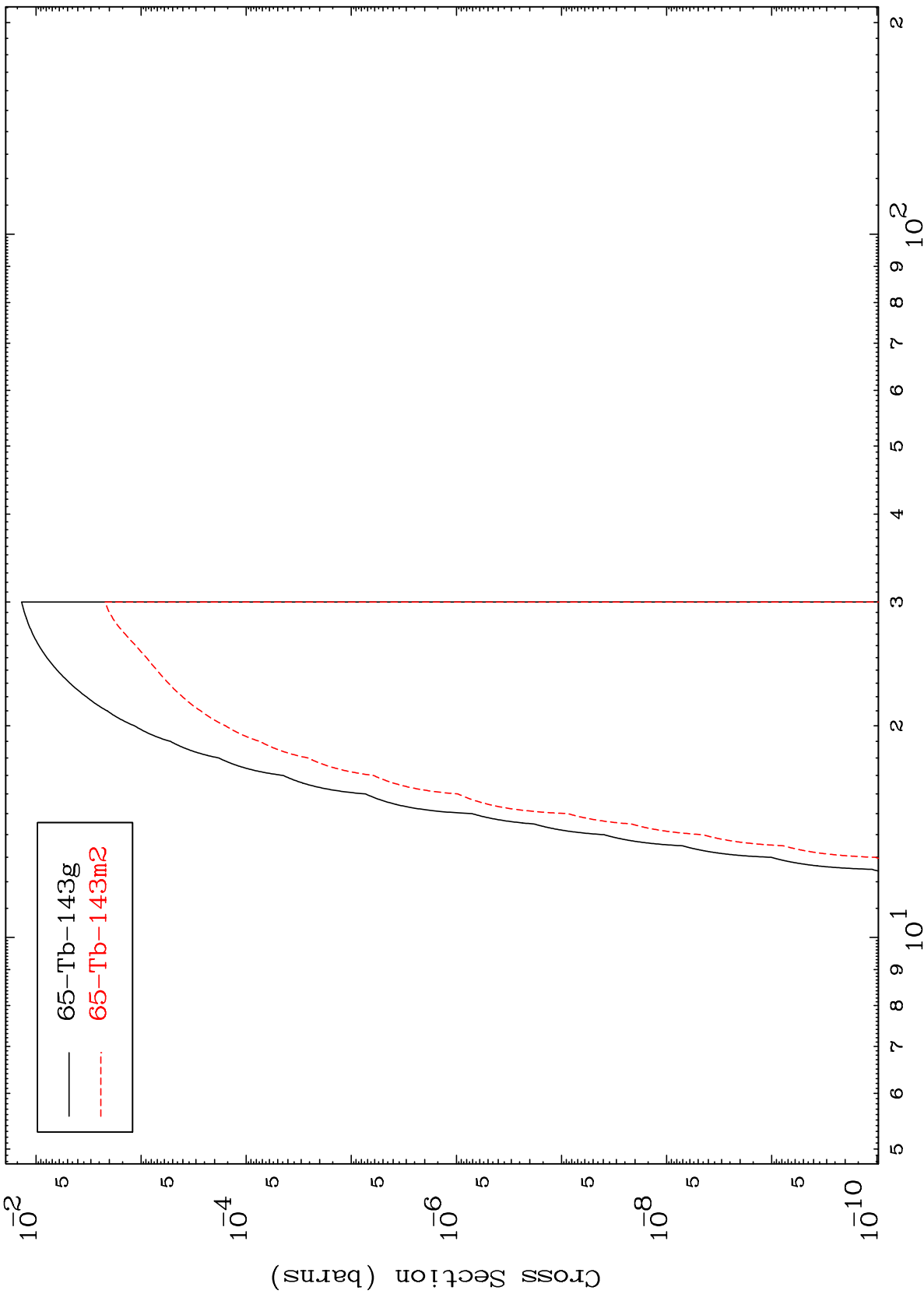
— 65-Tb-144g  
- - - 65-Tb-144m4

MAT 6668

(n,p) t

67-Ho-146

Radionuclide Production Cross Section



37

Incident Energy (MeV)

67-Ho-146

MAT 6668

(n,d)  $\alpha$

<sup>67</sup>Ho-146

Radionuclide Production Cross Section

