

Program EVALPLOT
(Version 2017-1)

by

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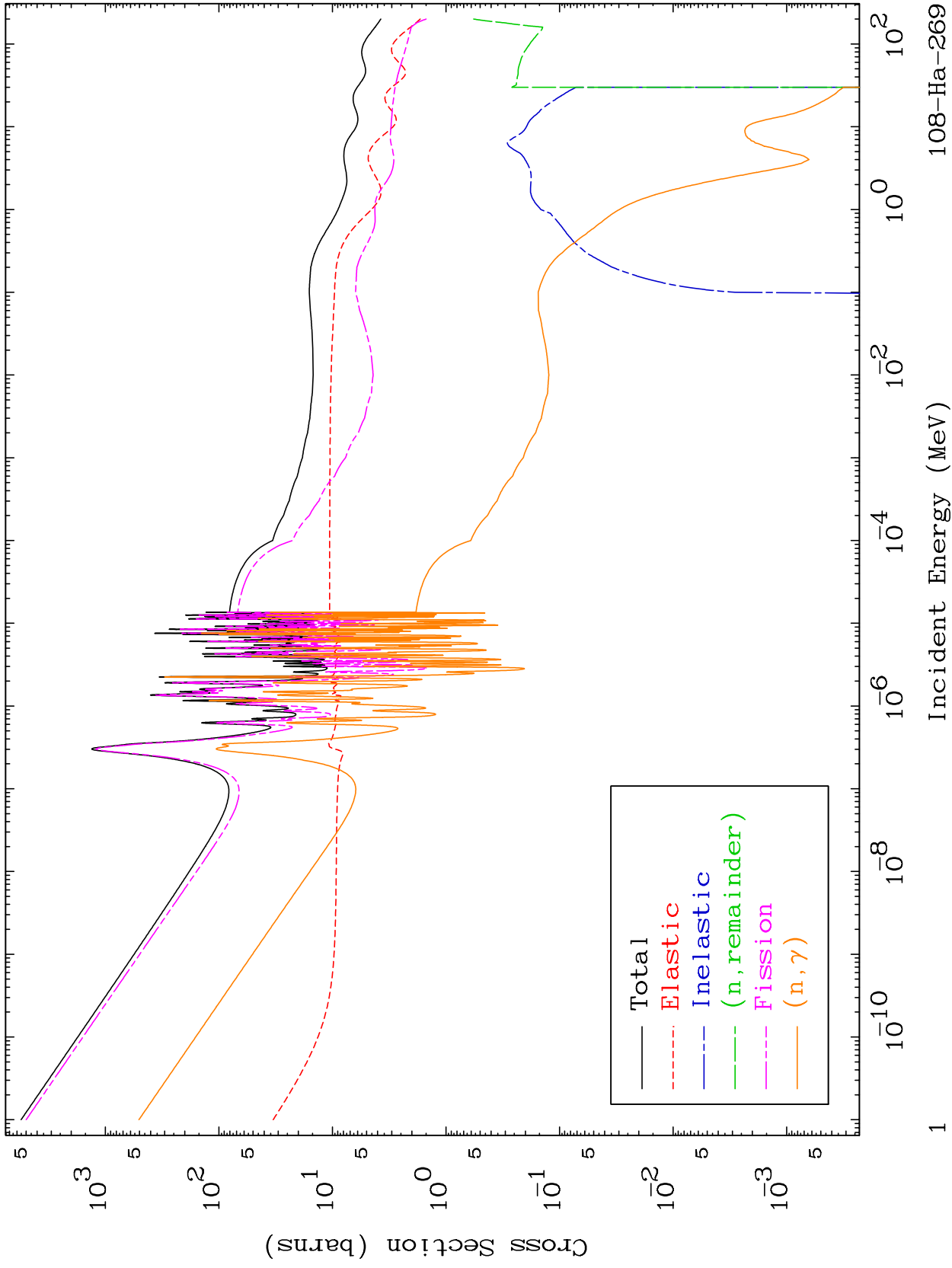
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Press Mouse Button to Start

MAT 869

Major
293 Kelvin Cross Sections

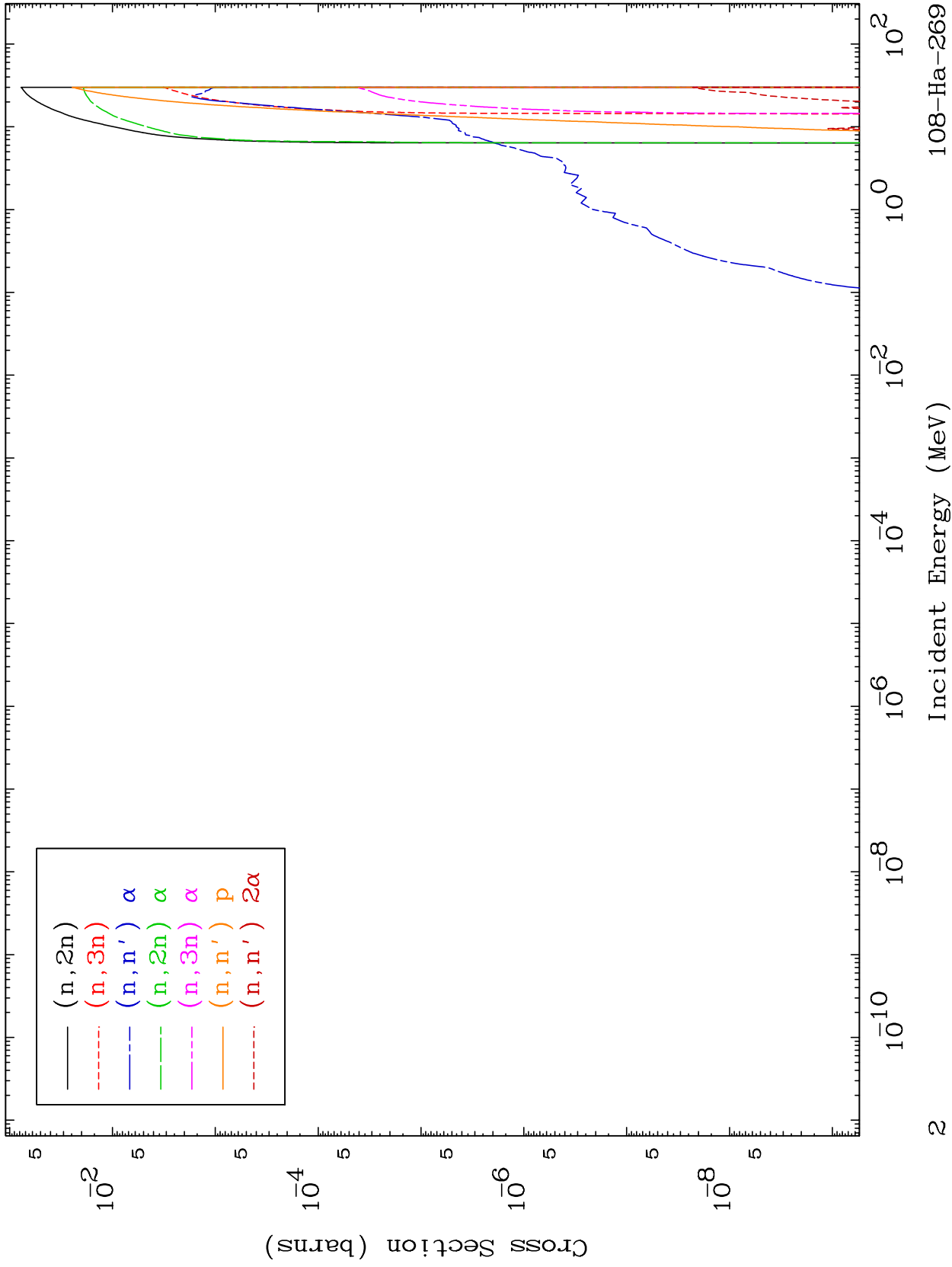
108-Ha-269



MAT 869

Neutron Production
293 Kelvin Cross Sections

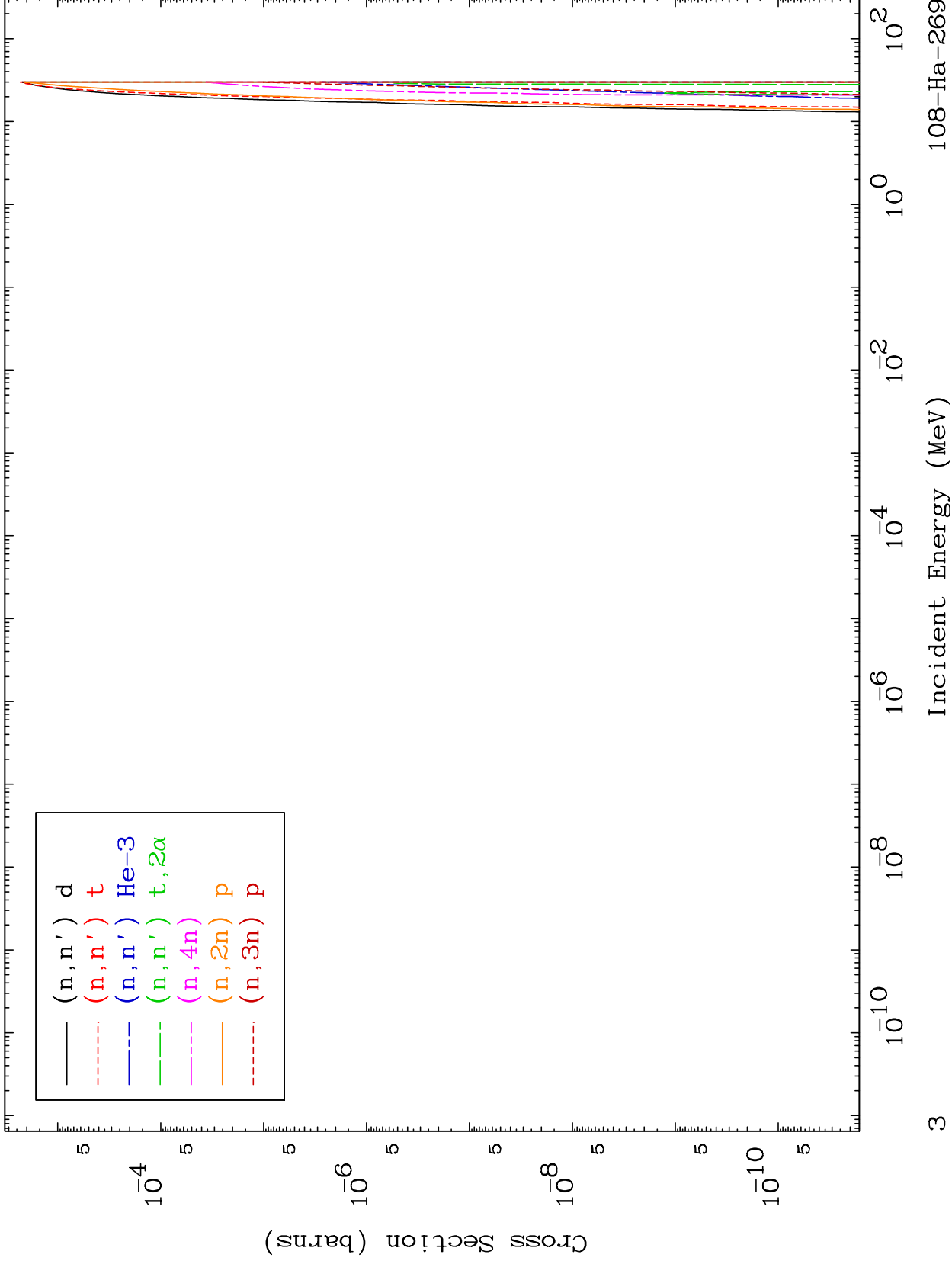
108-Ha-269



MAT 869

Neutron Production
293 Kelvin Cross Sections

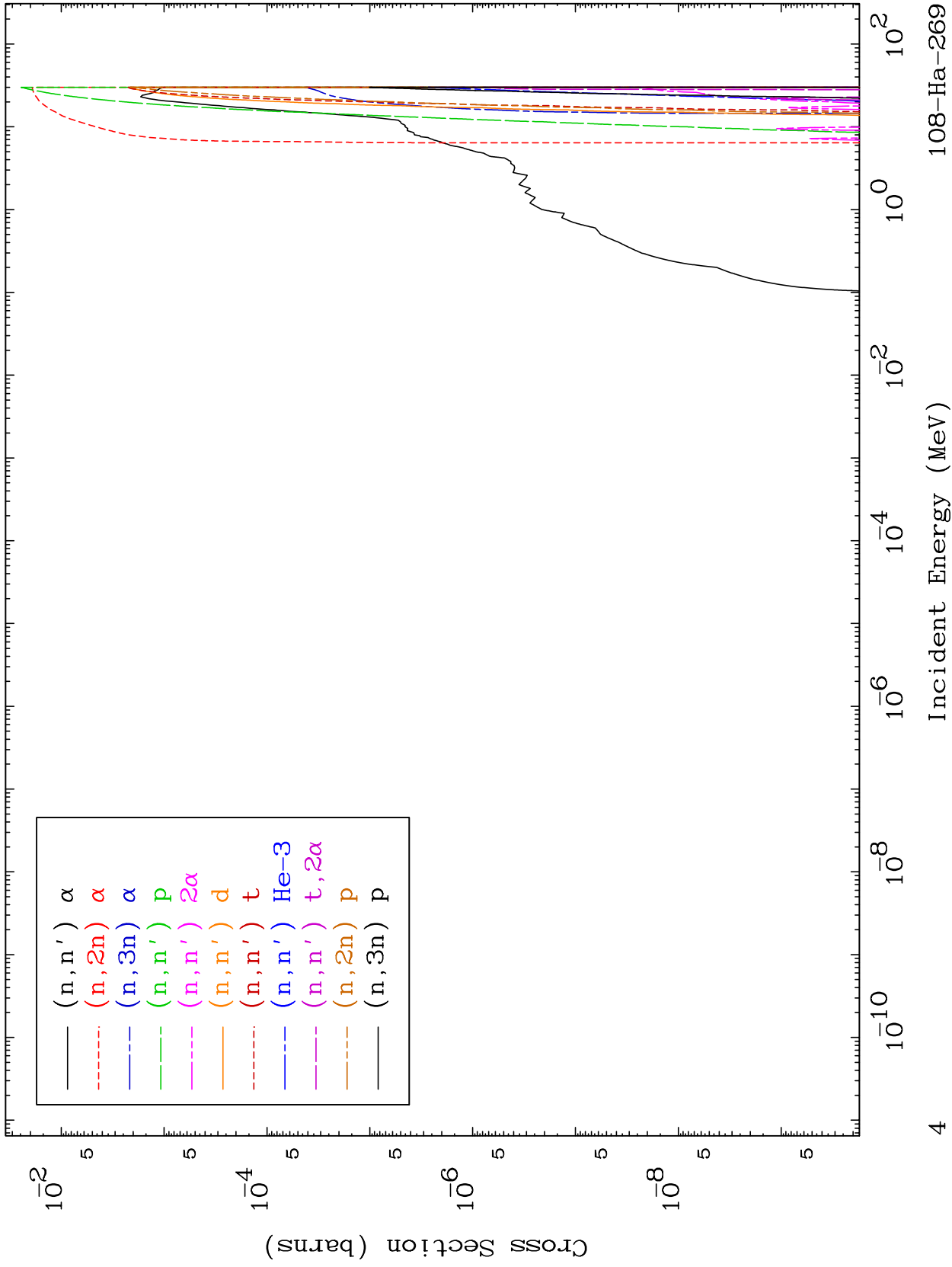
108-Ha-269



MAT 869

Charged Particle
293 Kelvin Cross Sections

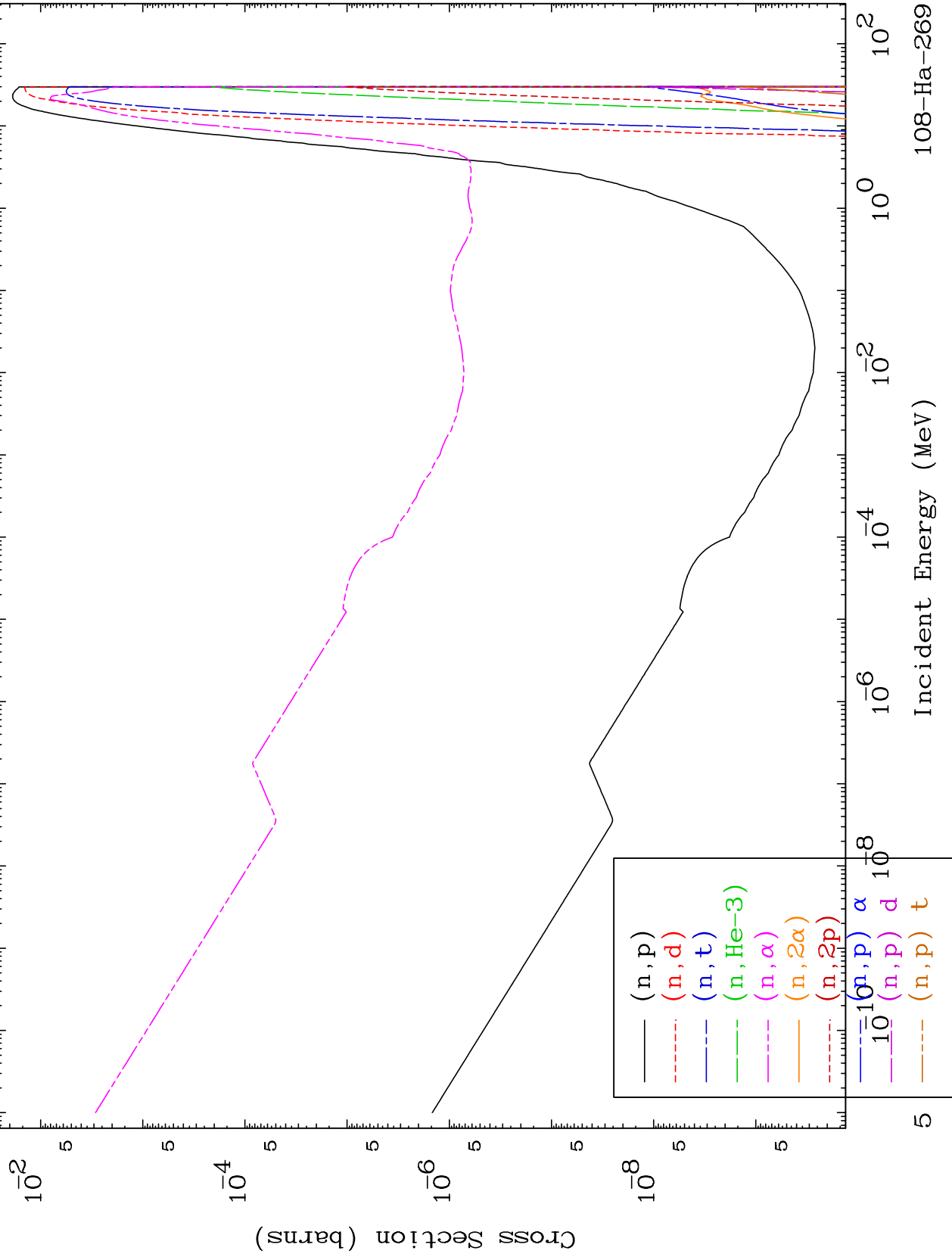
108-Ha-269



MAT 869

Charged Particle
293 Kelvin Cross Sections

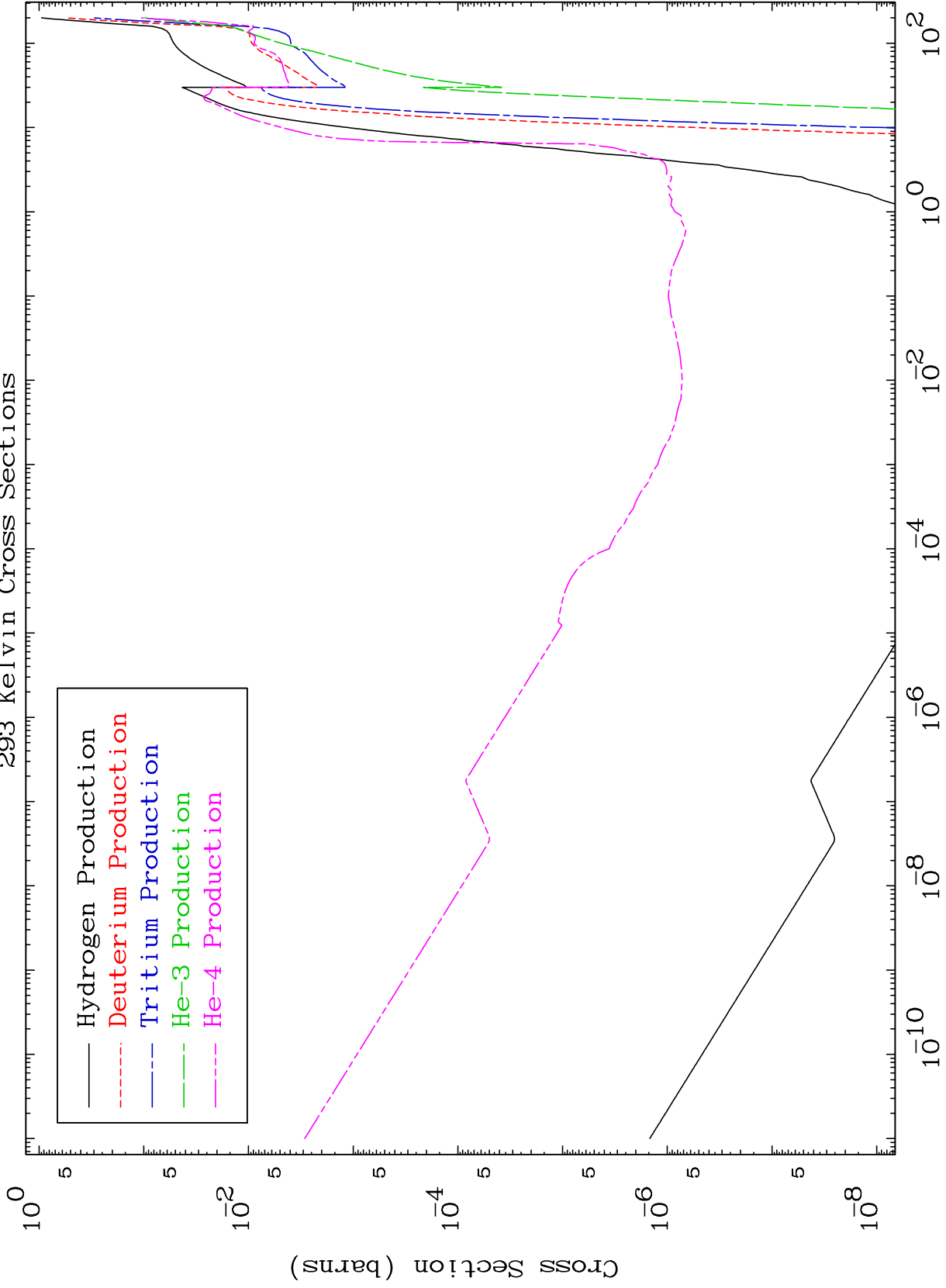
108-Ha-269



MAT 869

Particle Production
293 Kelvin Cross Sections

108-Ha-269



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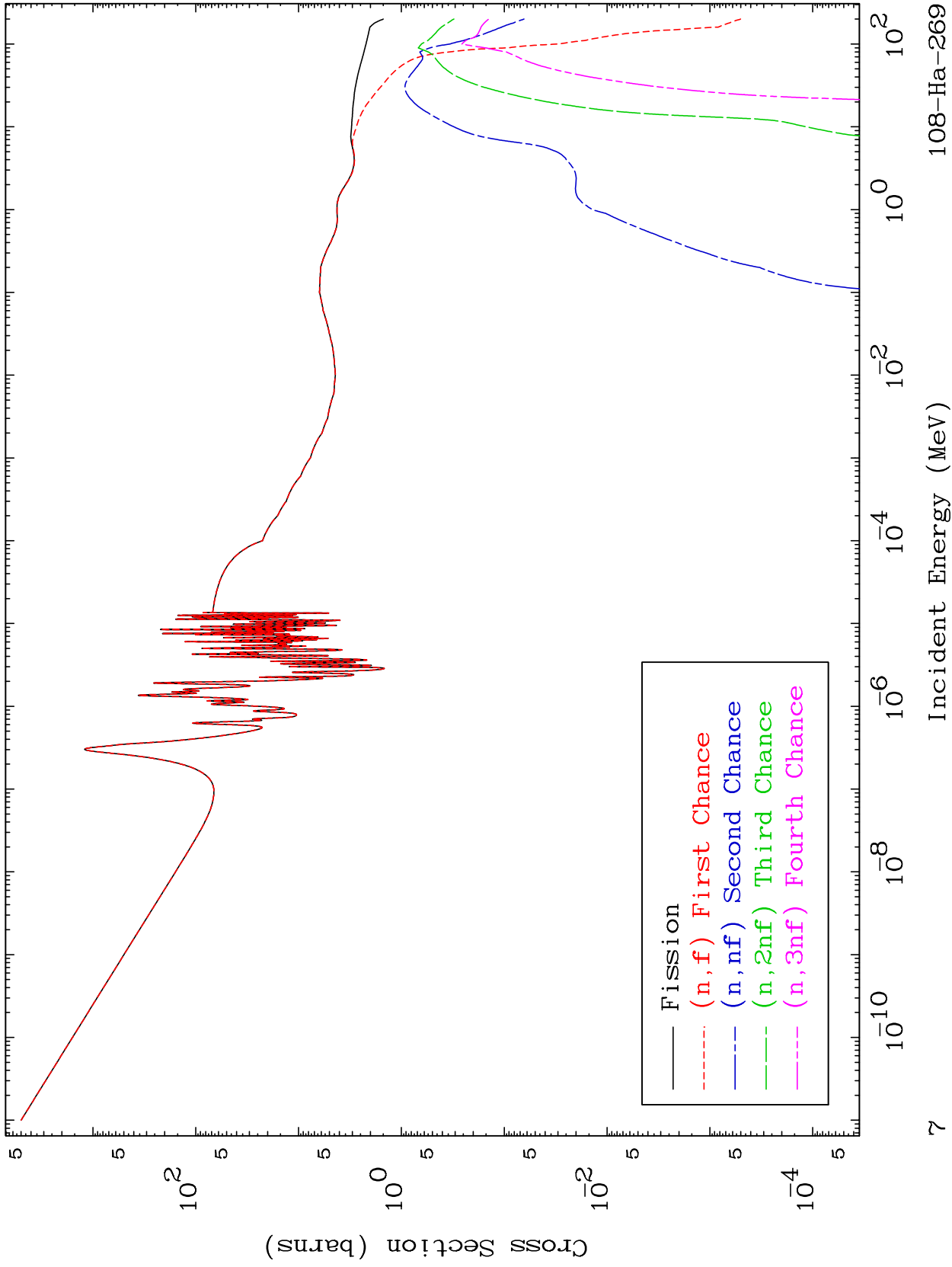
Incident Energy (MeV)

108-Ha-269

MAT 869

Fission
293 Kelvin Cross Sections

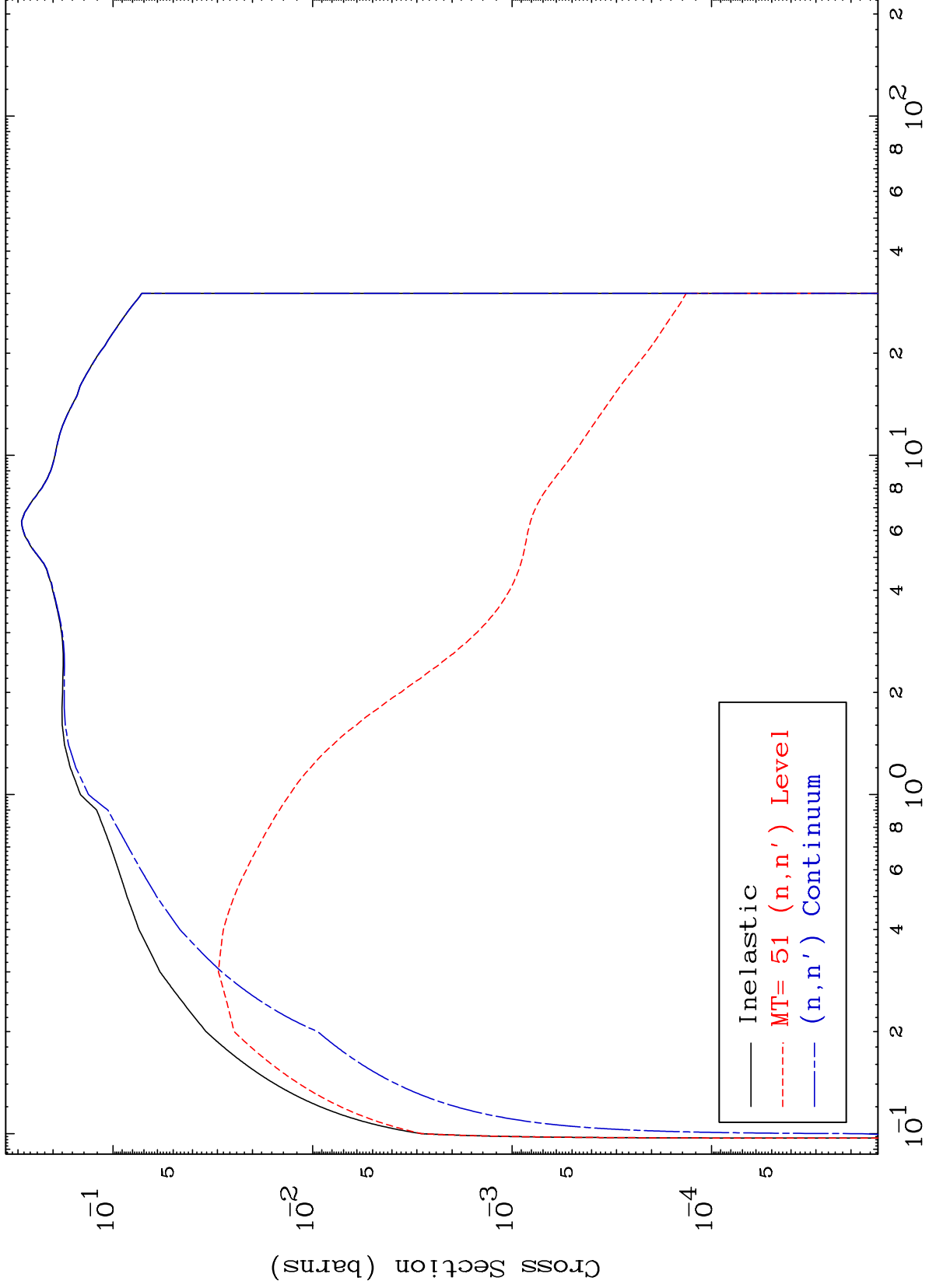
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MAT 869

(n,n') Level
293 Kelvin Cross Sections

108-Ha-269



Incident Energy (MeV)

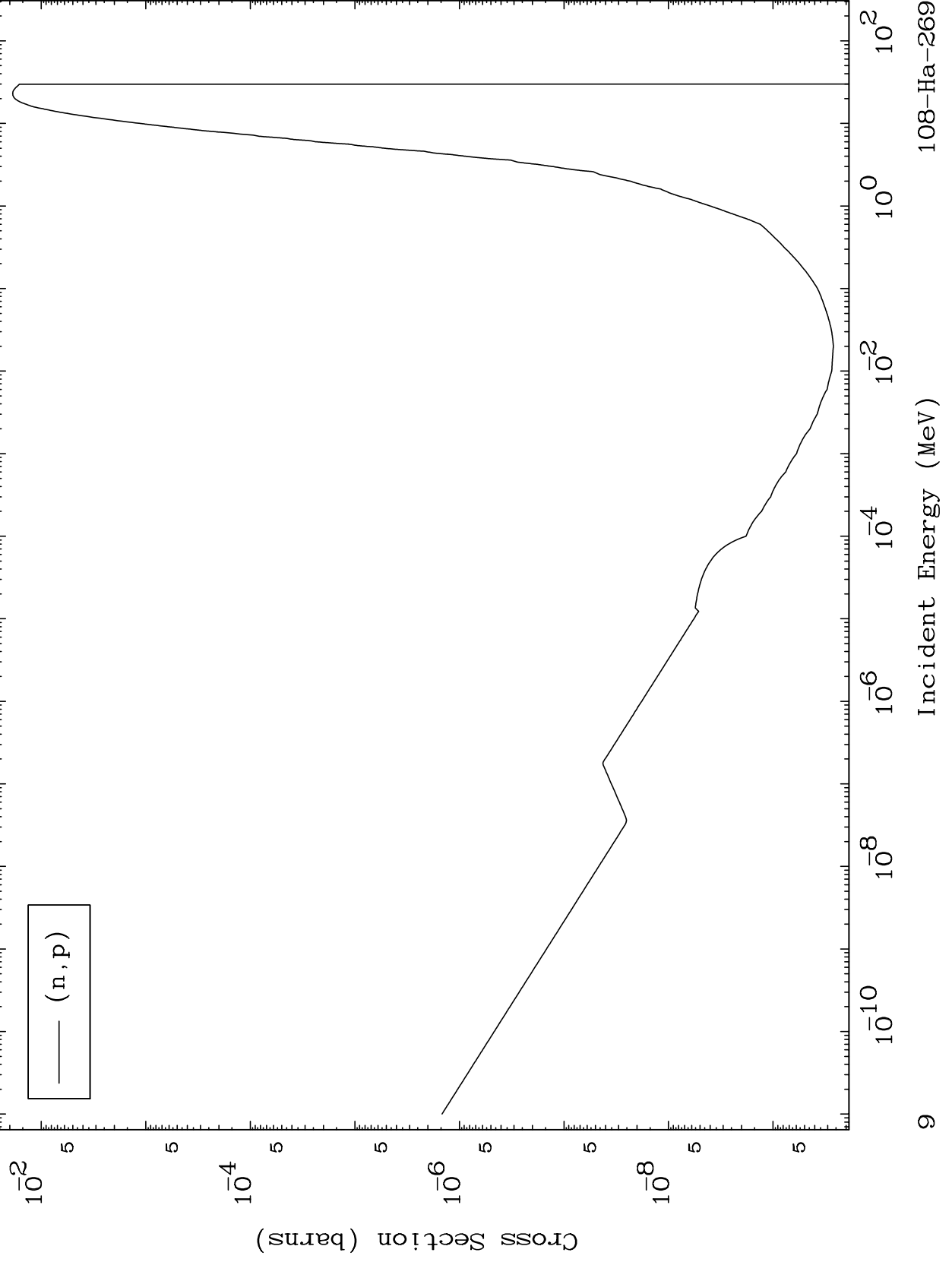
108-Ha-269

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(n,p) Levels
293 Kelvin Cross Sections

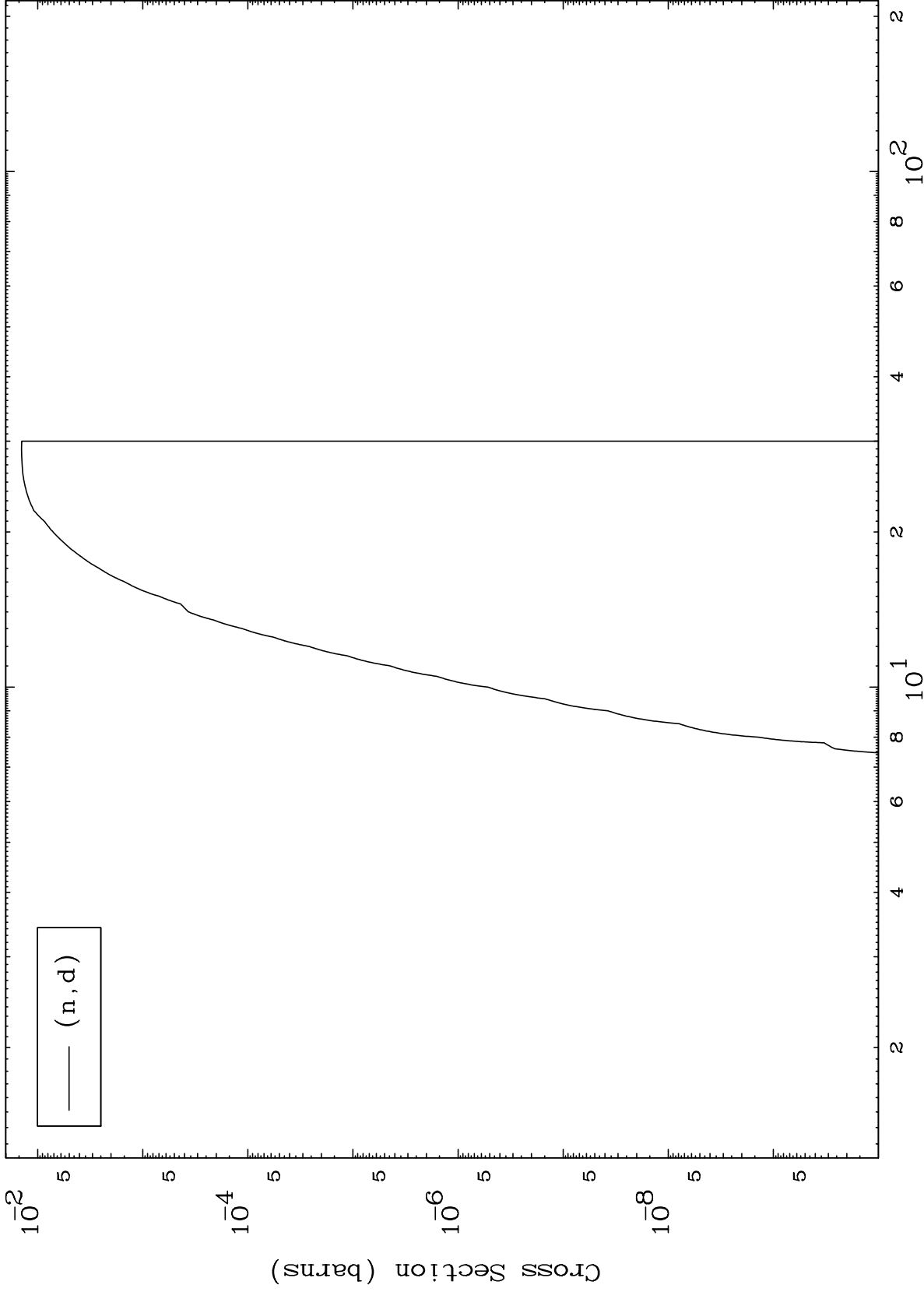
108-Ha-269



MAT 869

(n,d) Levels
293 Kelvin Cross Sections

108-Ha-269



10

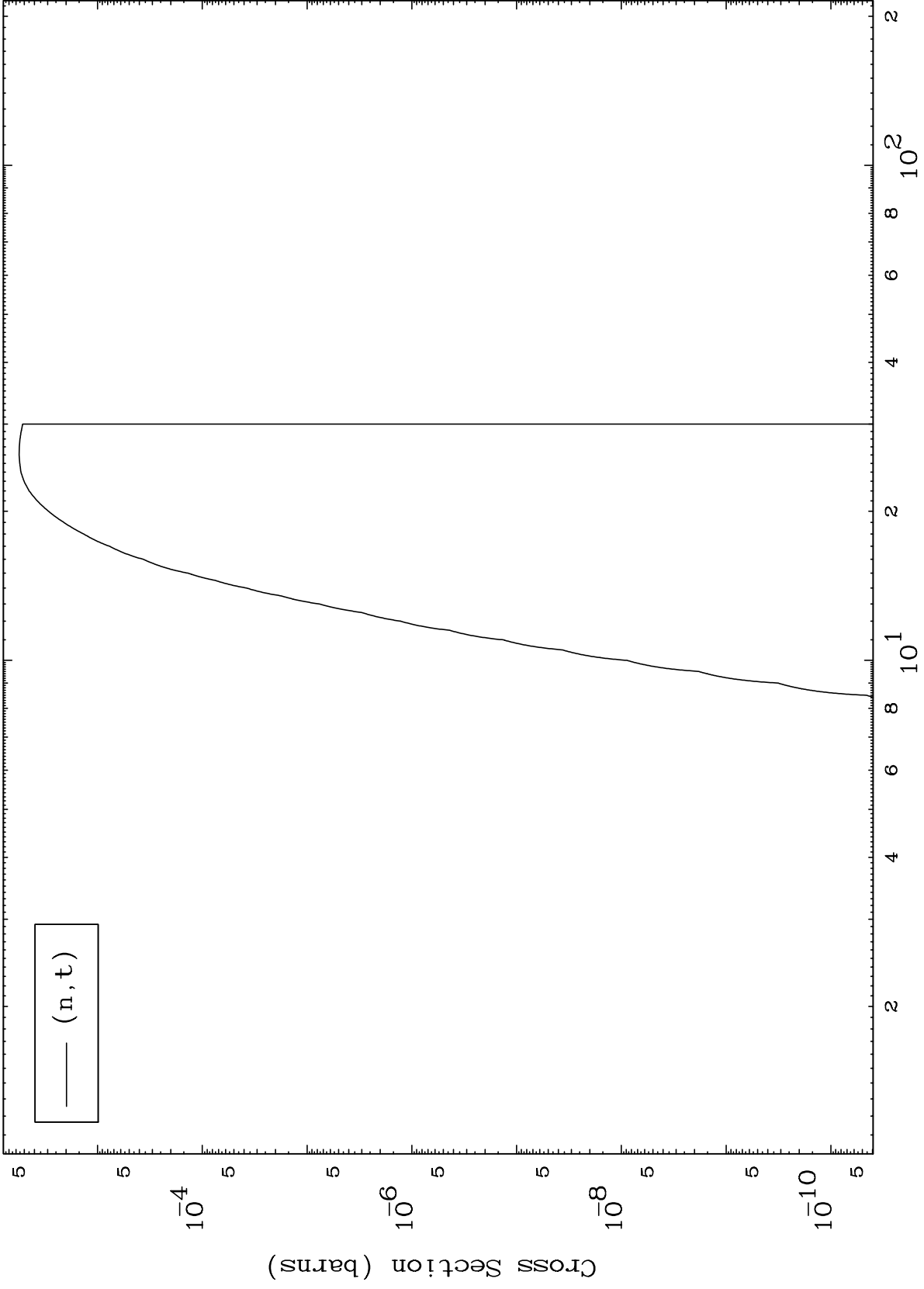
Incident Energy (MeV)

108-Ha-269

MAT 869

(n,t) Levels
293 Kelvin Cross Sections

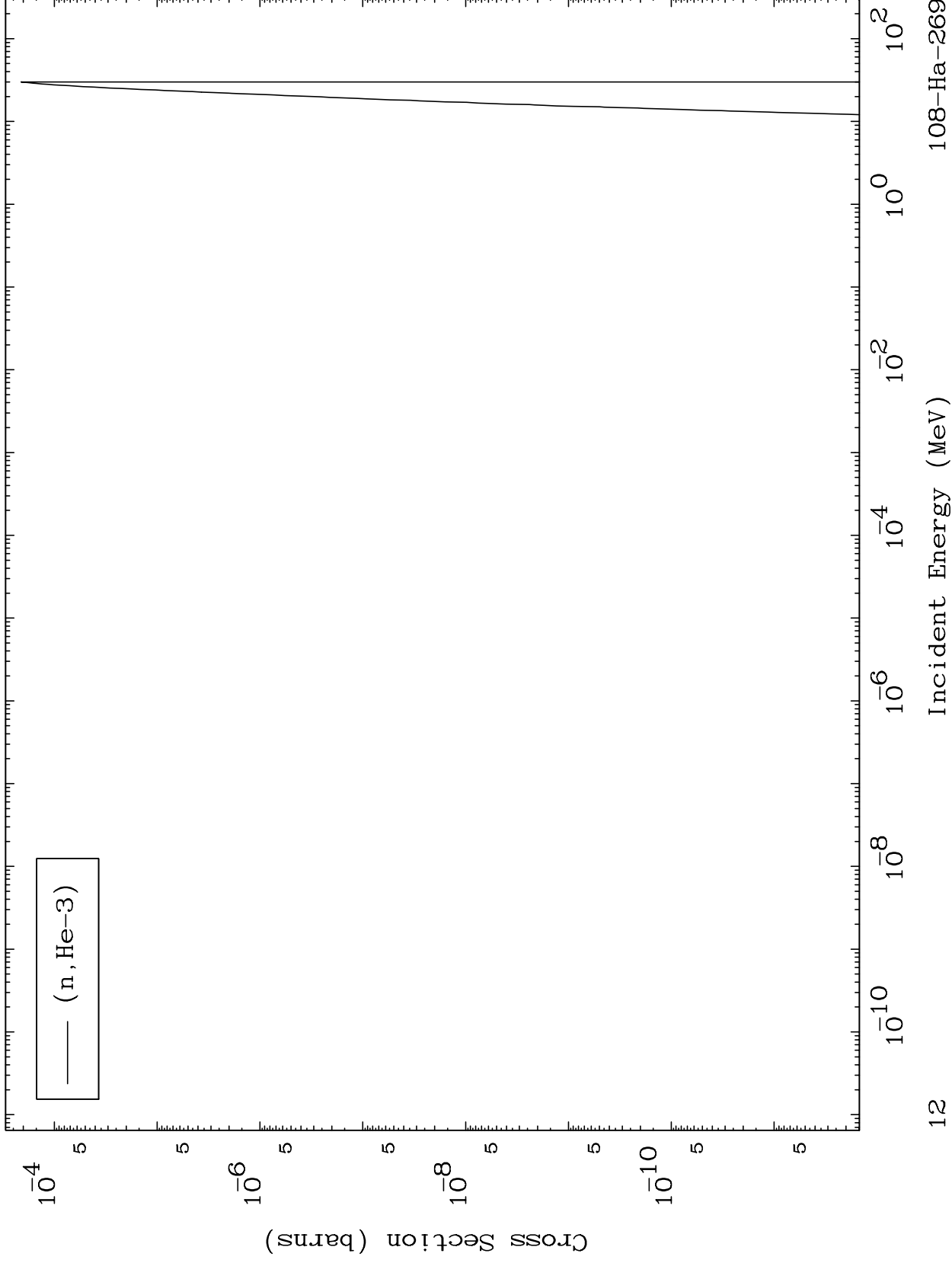
108-Ha-269



MAT 869

(n,He3) Levels
293 Kelvin Cross Sections

108-Ha-269



12

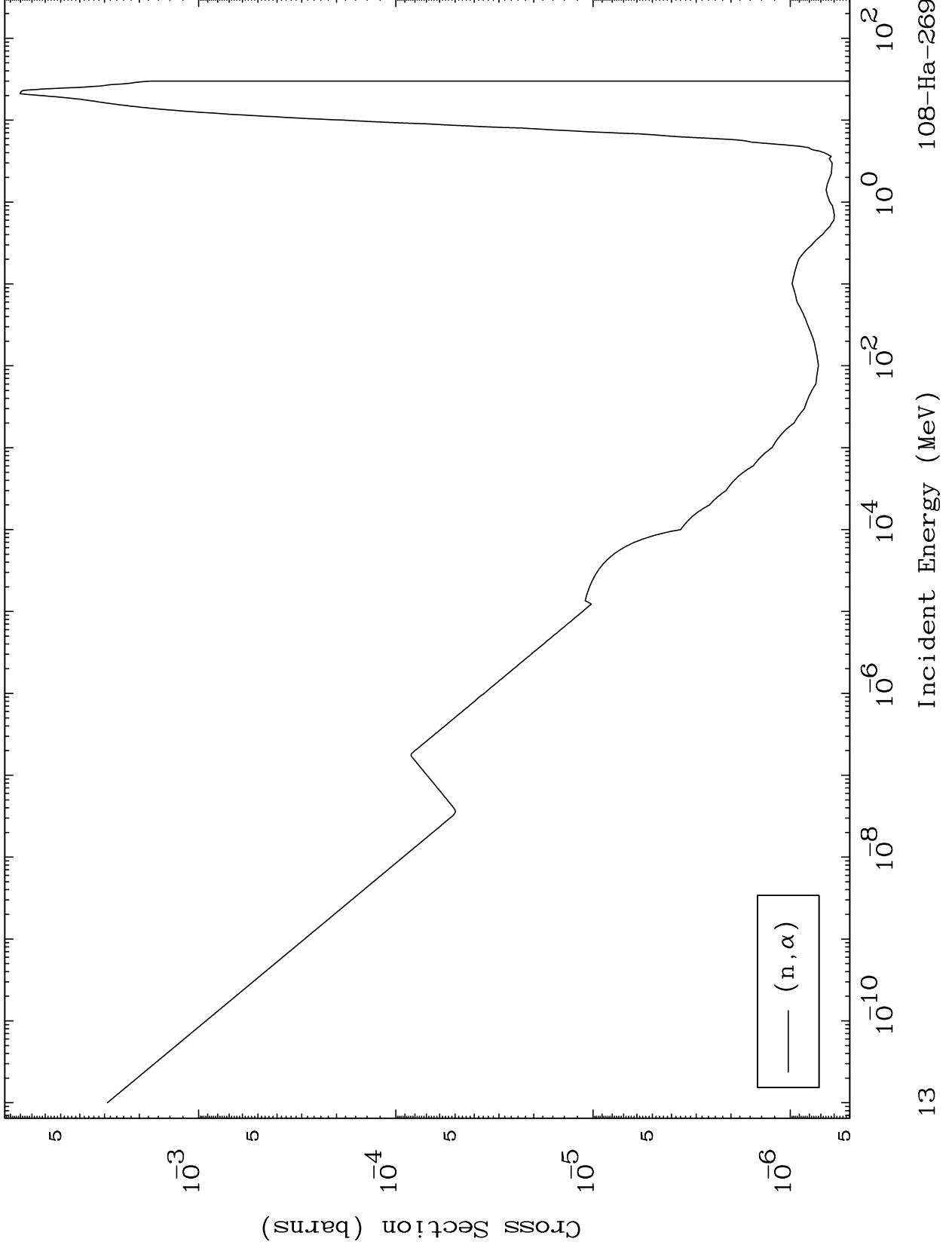
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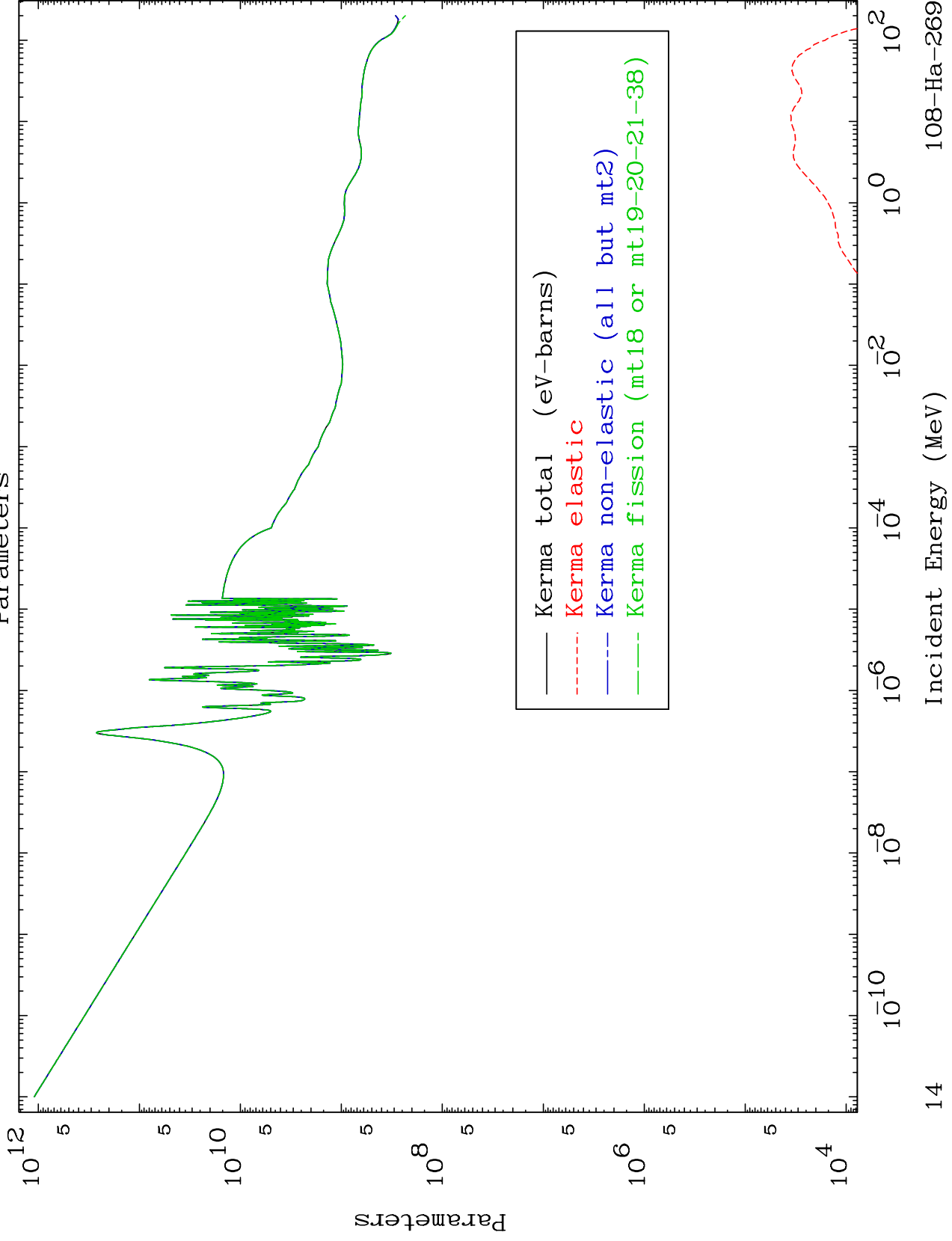
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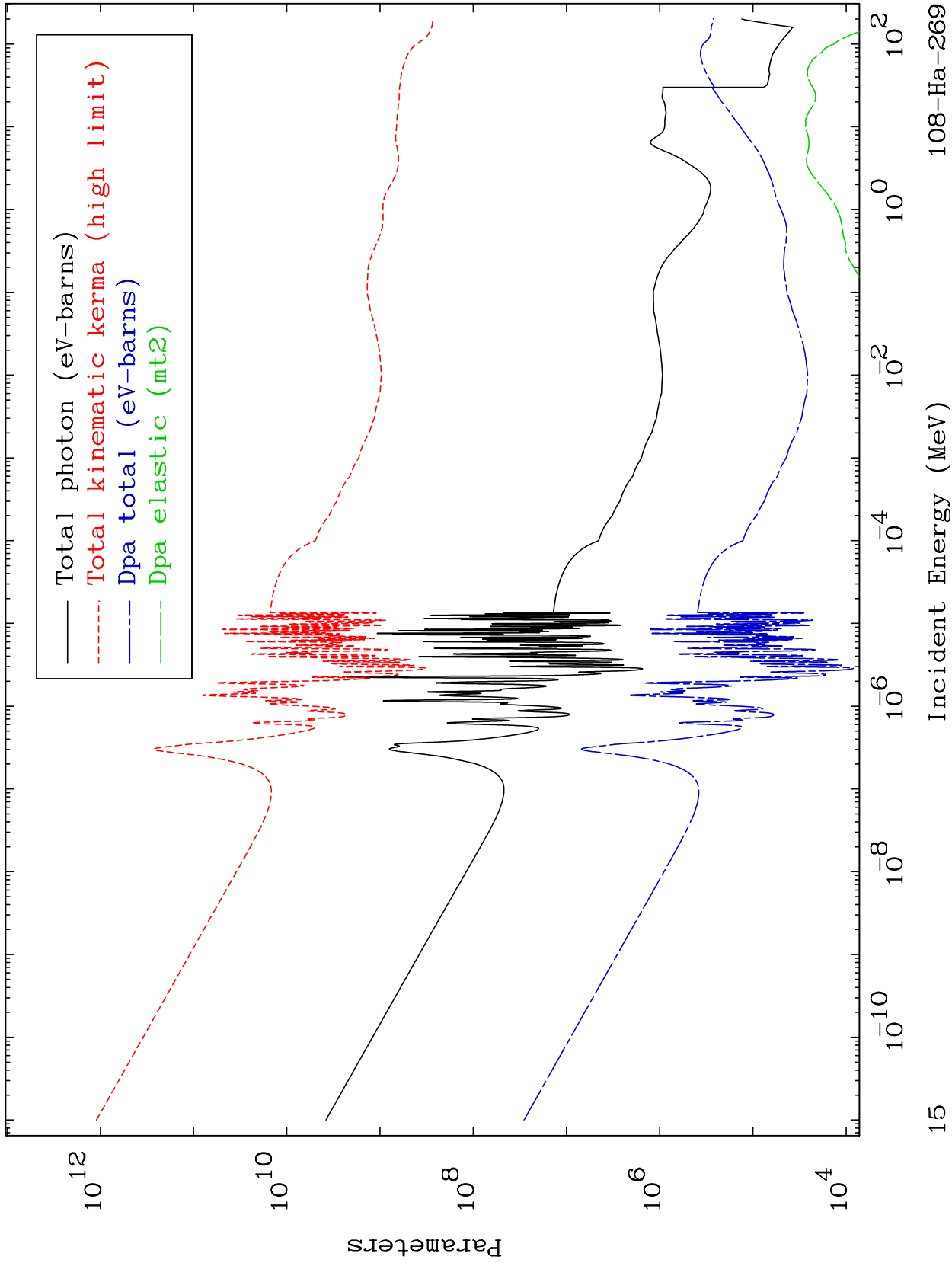
MAT 869

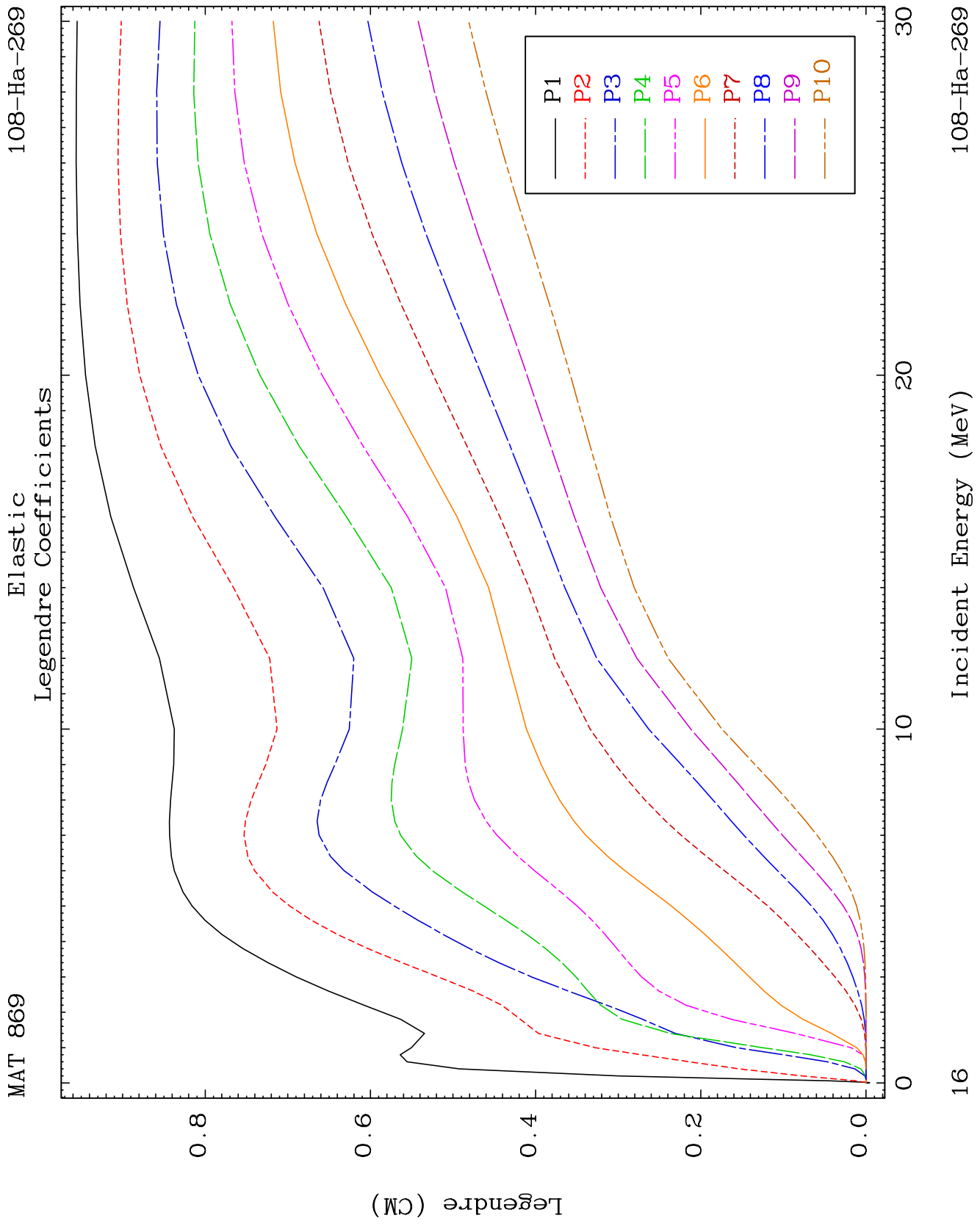
(n, α) Levels
293 Kelvin Cross Sections

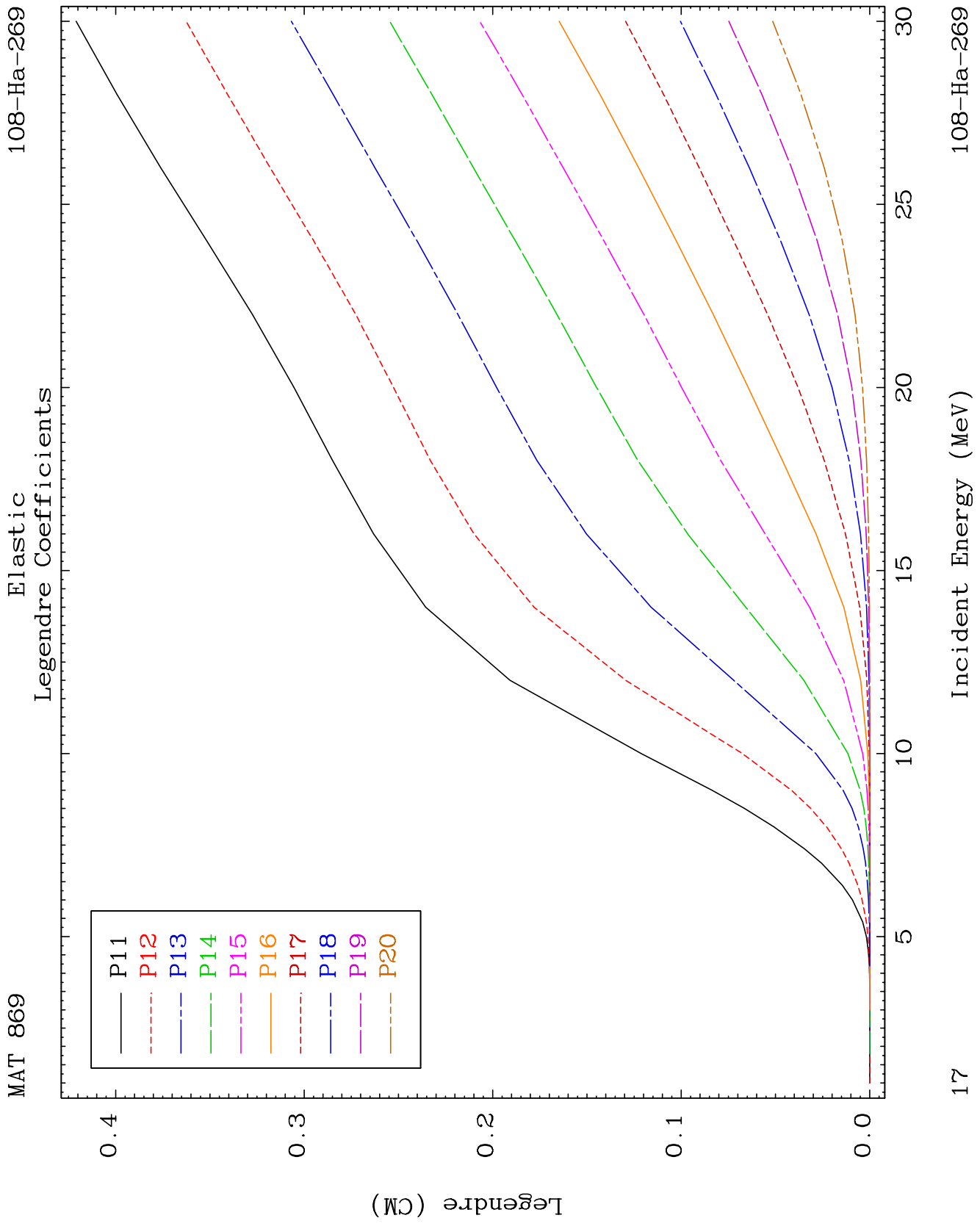
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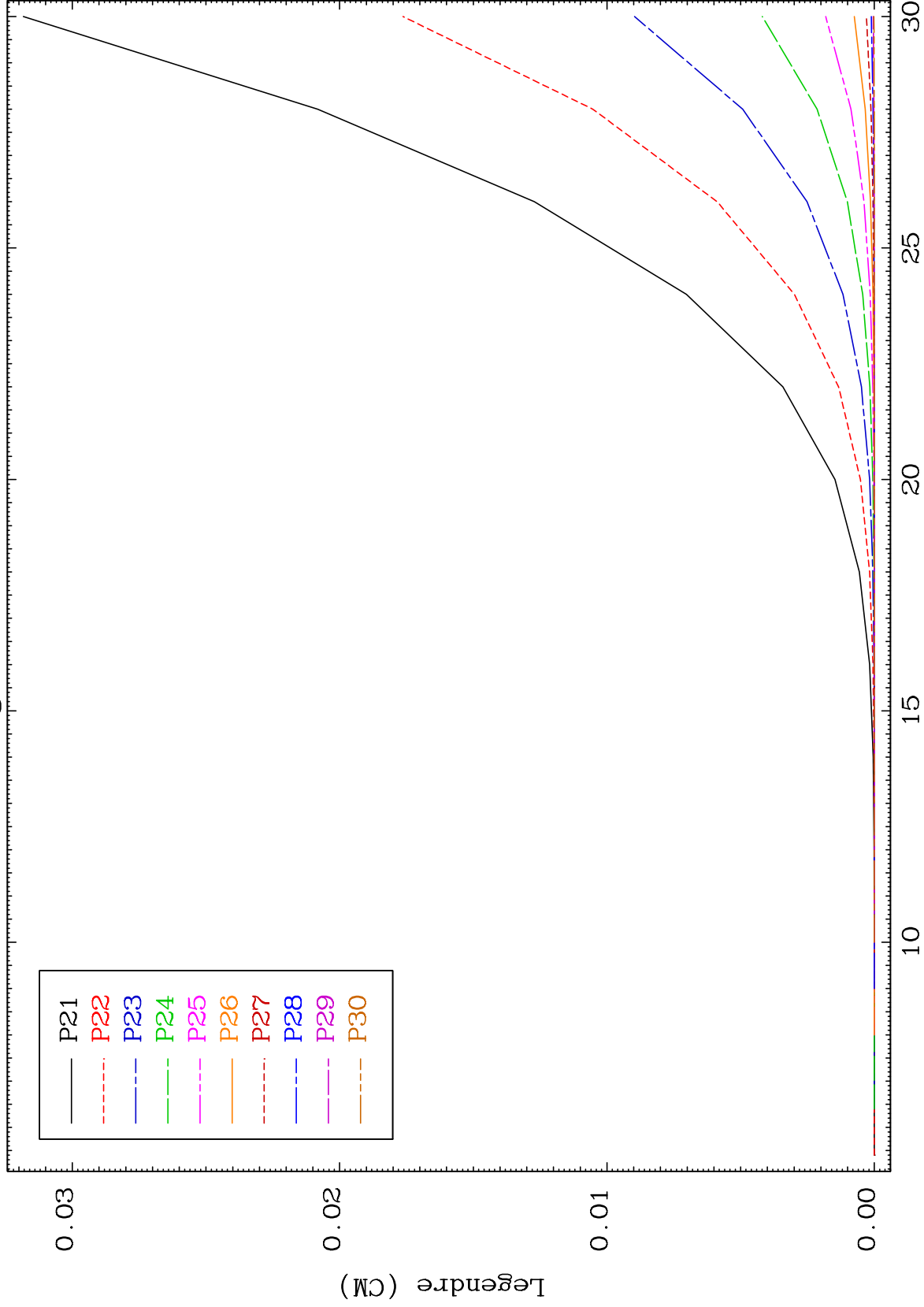




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Elastic Legendre Coefficients

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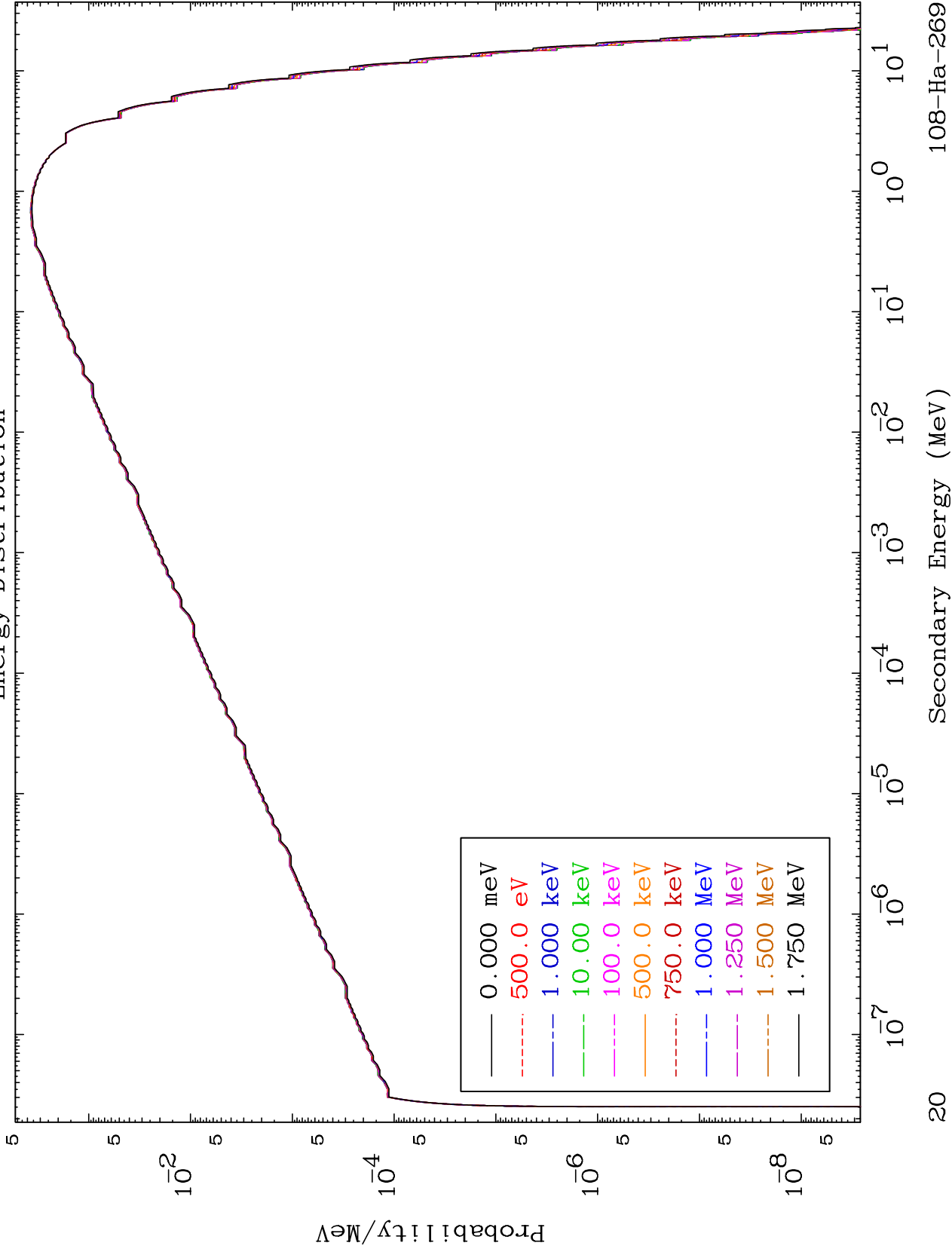
Incident Energy (MeV)

108-Ha-269

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Fission Energy Distribution

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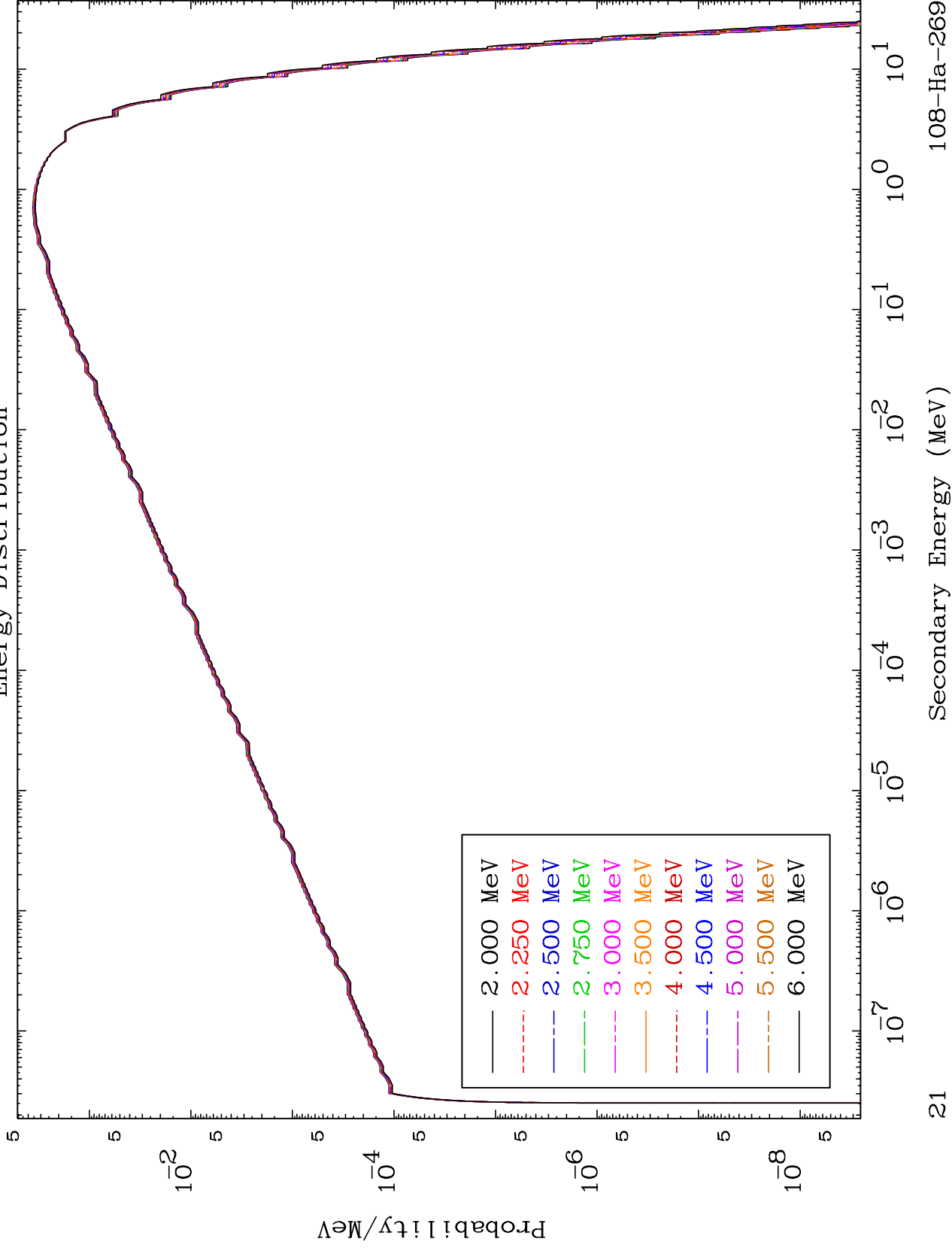


108-Ha-269

MAT 869

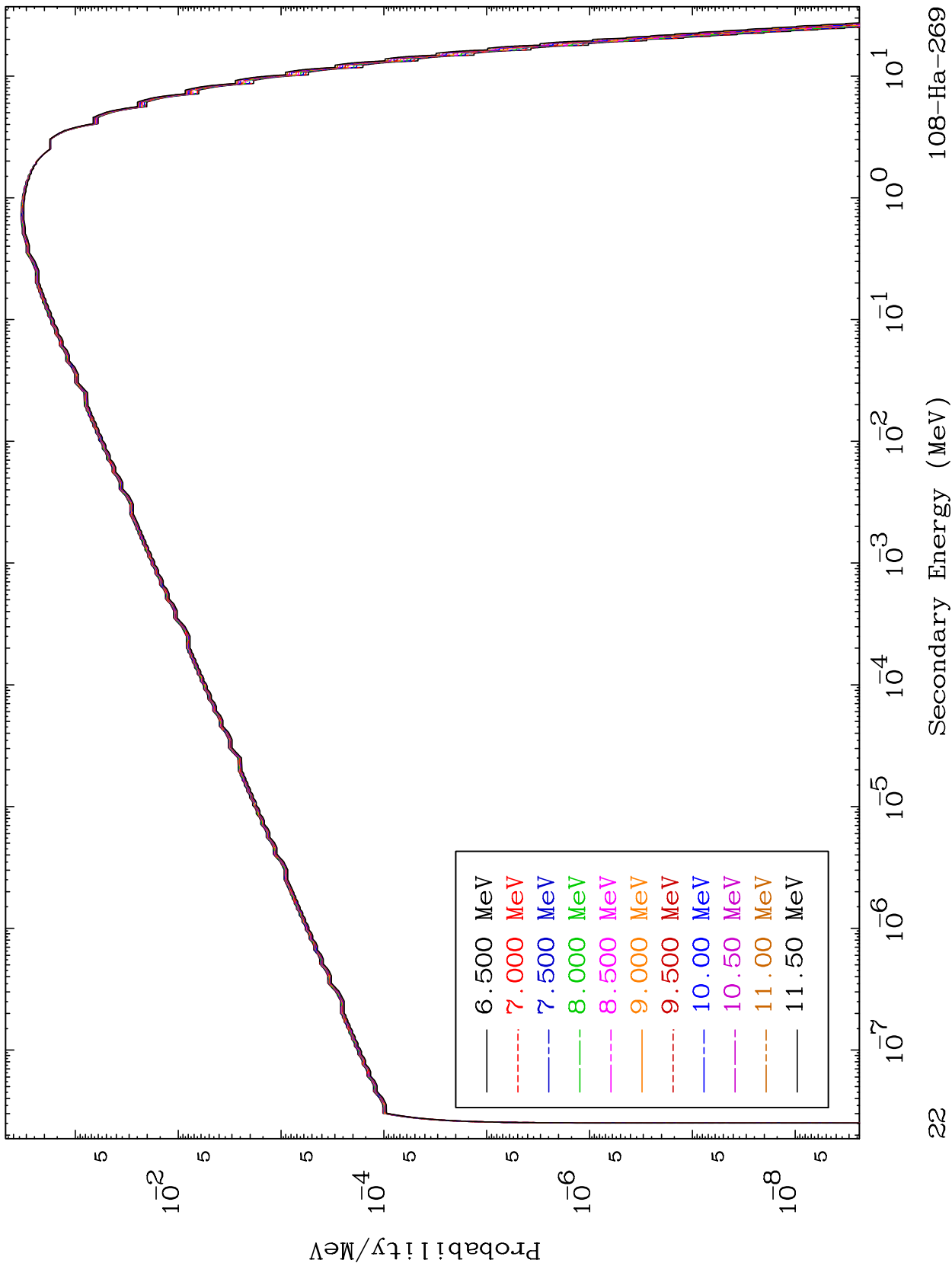
Fission Energy Distribution

108-Ha-269

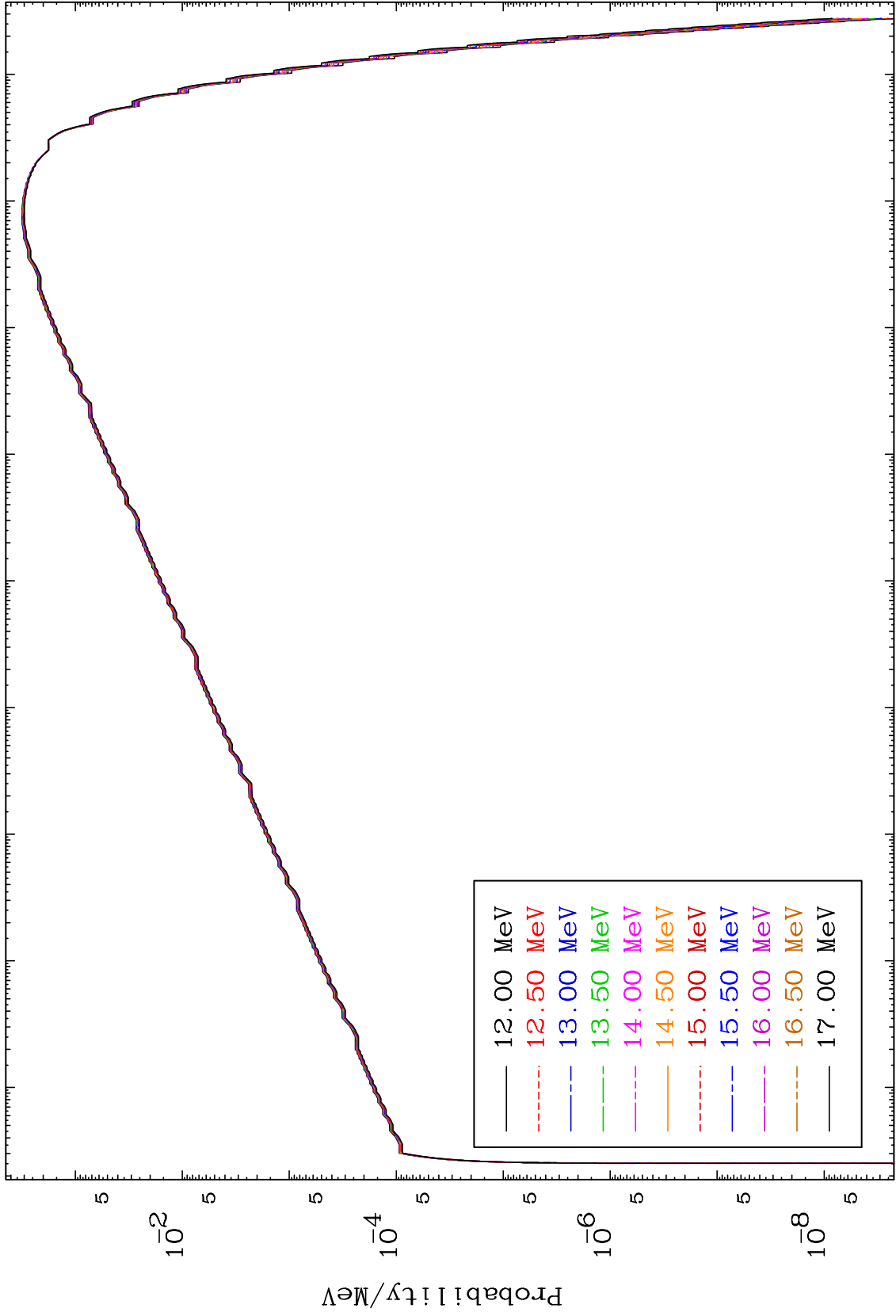


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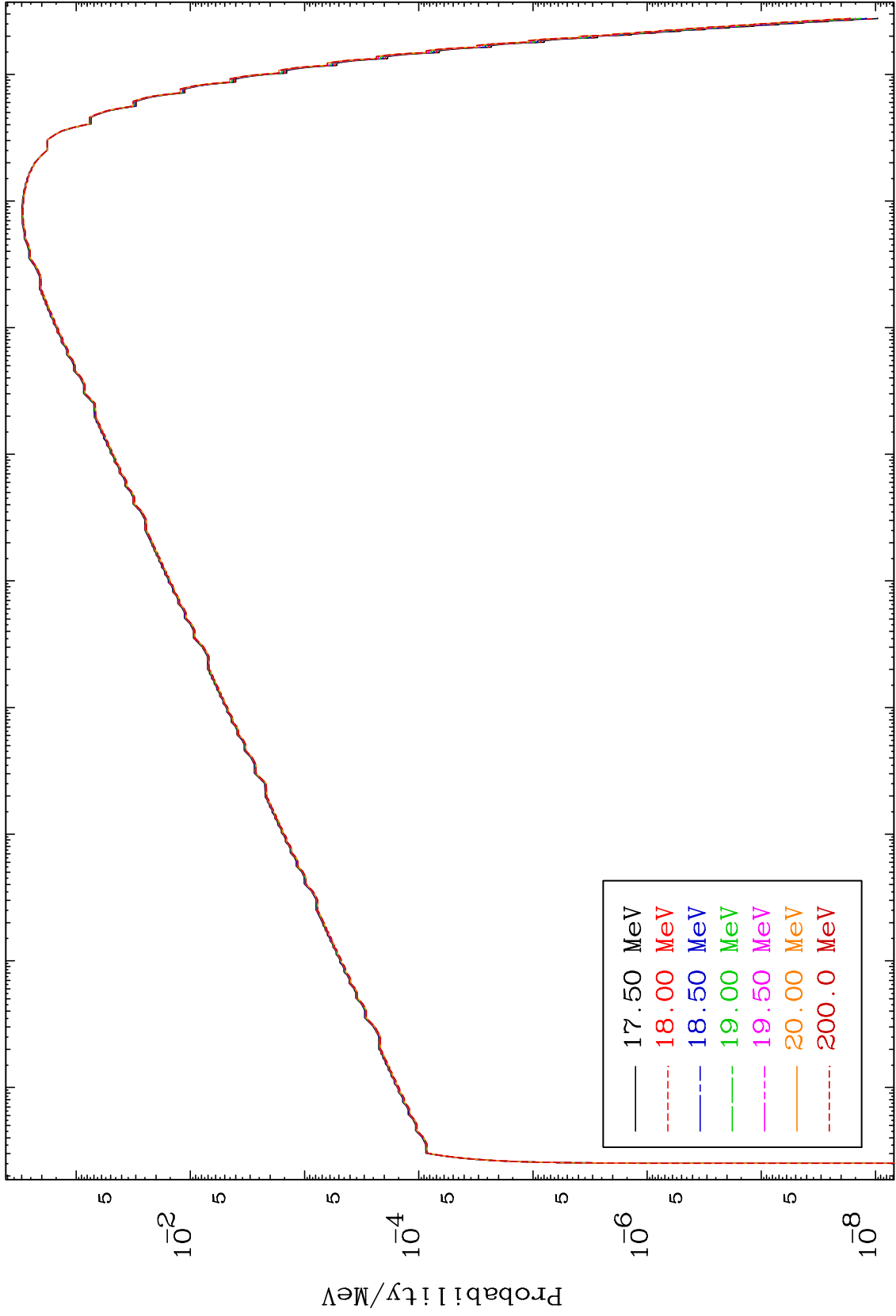
Fission
Energy Distribution



Fission
Energy Distribution



Fission Energy Distribution

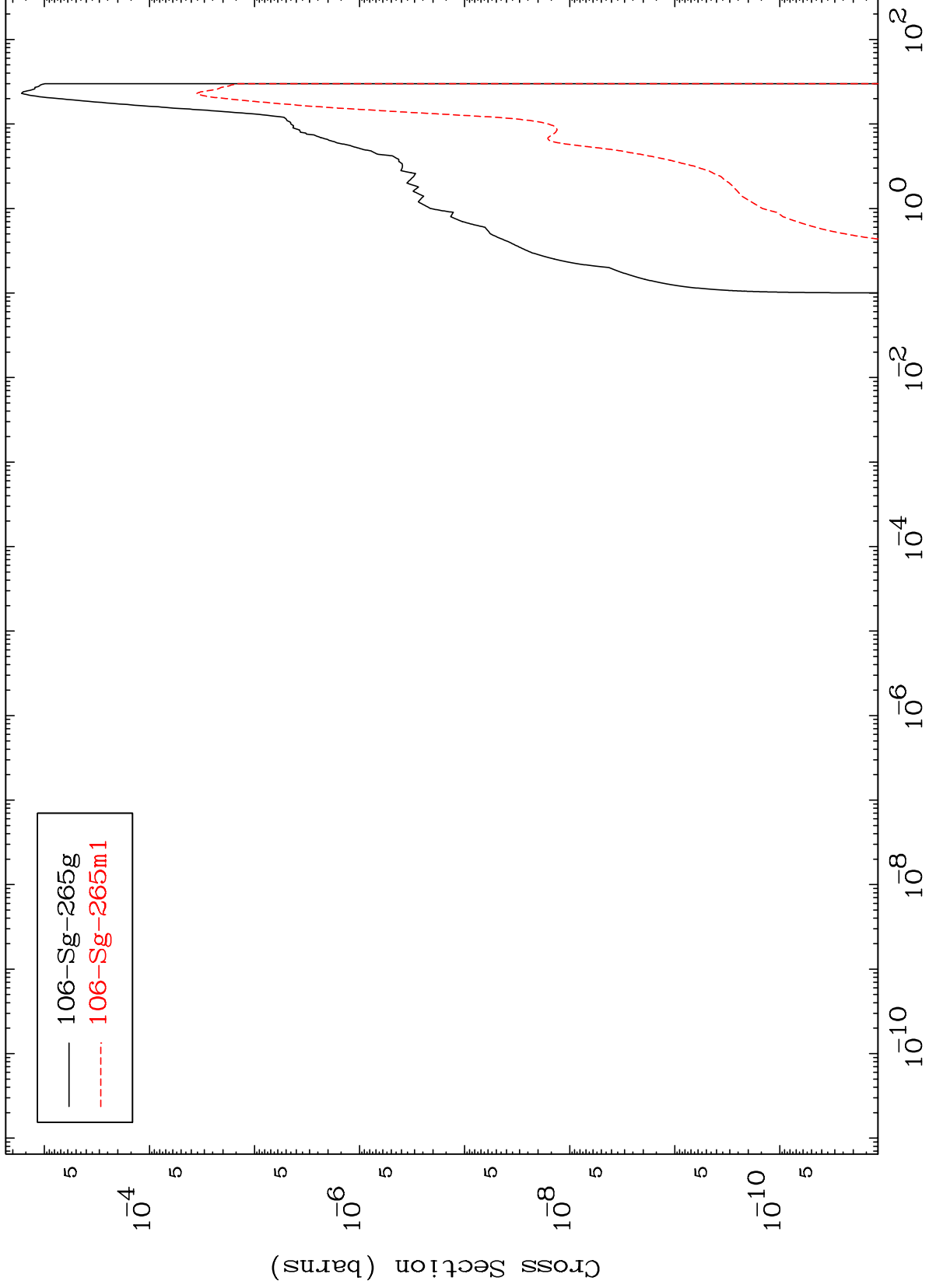


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$(n, n') \alpha$

108-Ha-269

Radionuclide Production Cross Section



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Incident Energy (MeV)

108-Ha-269

MAT 869

(n,n') 2 α

108-Ha-269

Radionuclide Production Cross Section

