

Program EVALPLOT
(Version 2017-1)

by

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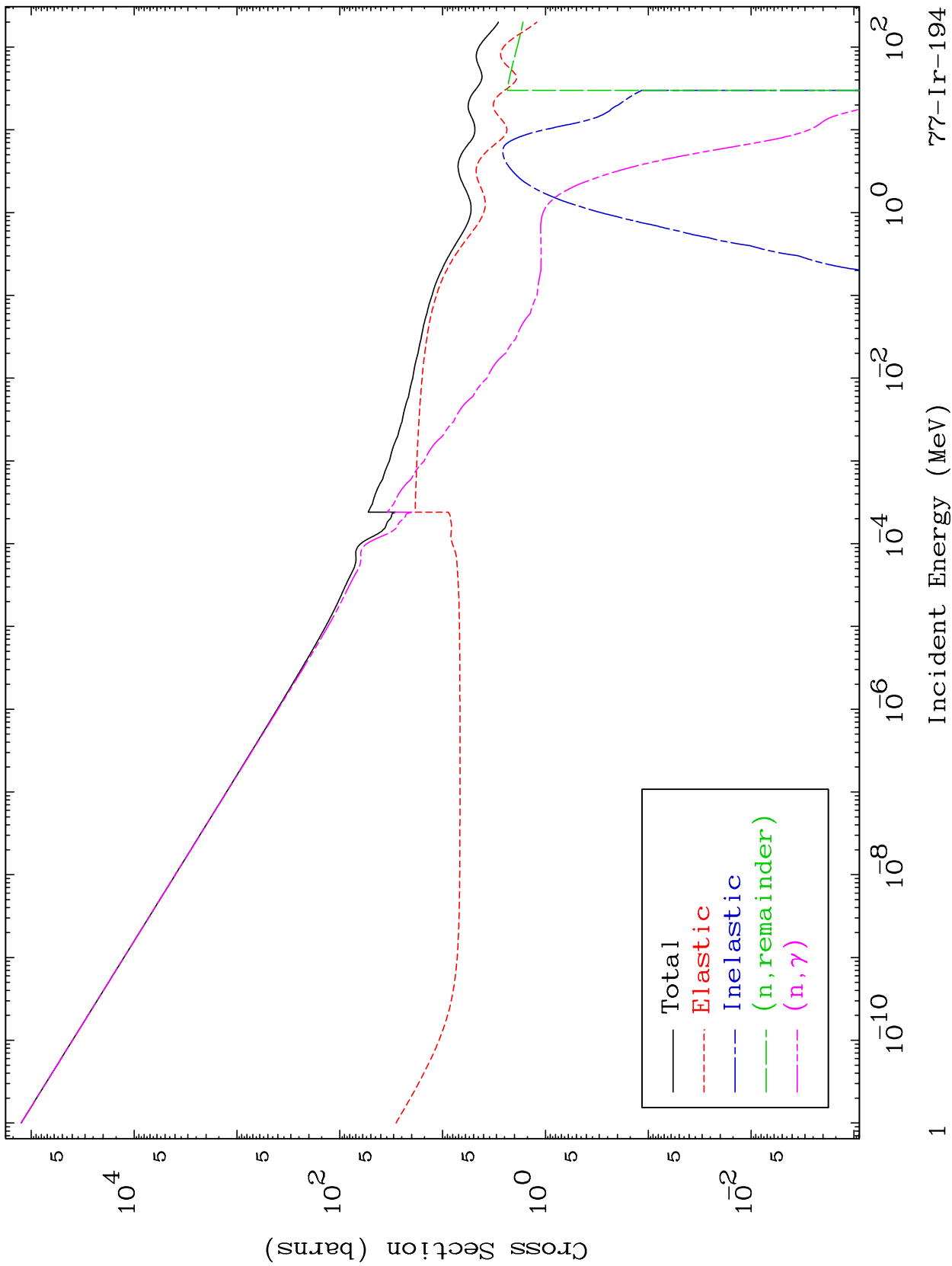
Press Mouse Button to Start

MAT 7735

Major

293 Kelvin Cross Sections

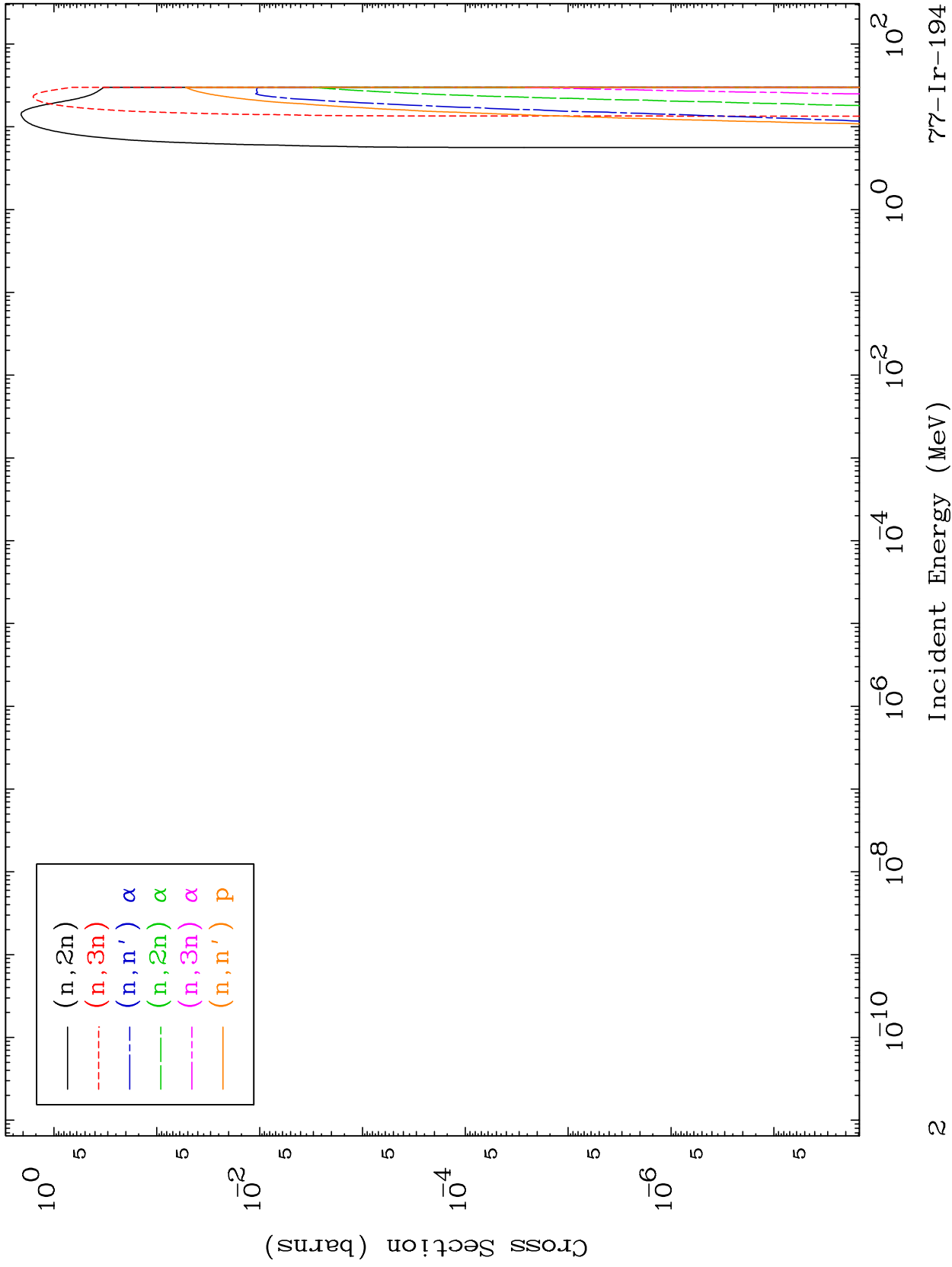
77-Ir-194



MAT 7735

Neutron Production
293 Kelvin Cross Sections

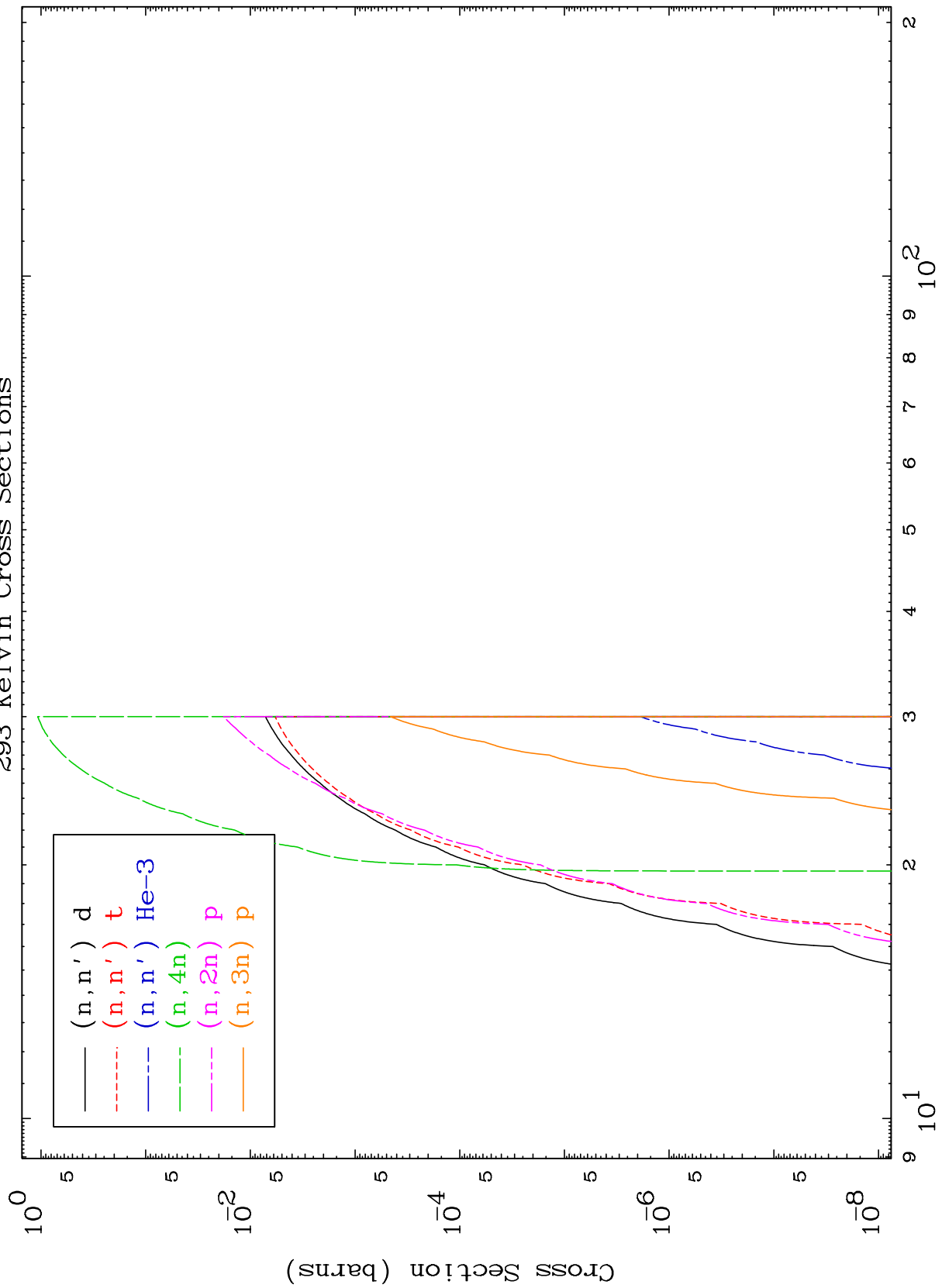
77-Ir-194



MAT 7735

Neutron Production
293 Kelvin Cross Sections

77-Ir-194



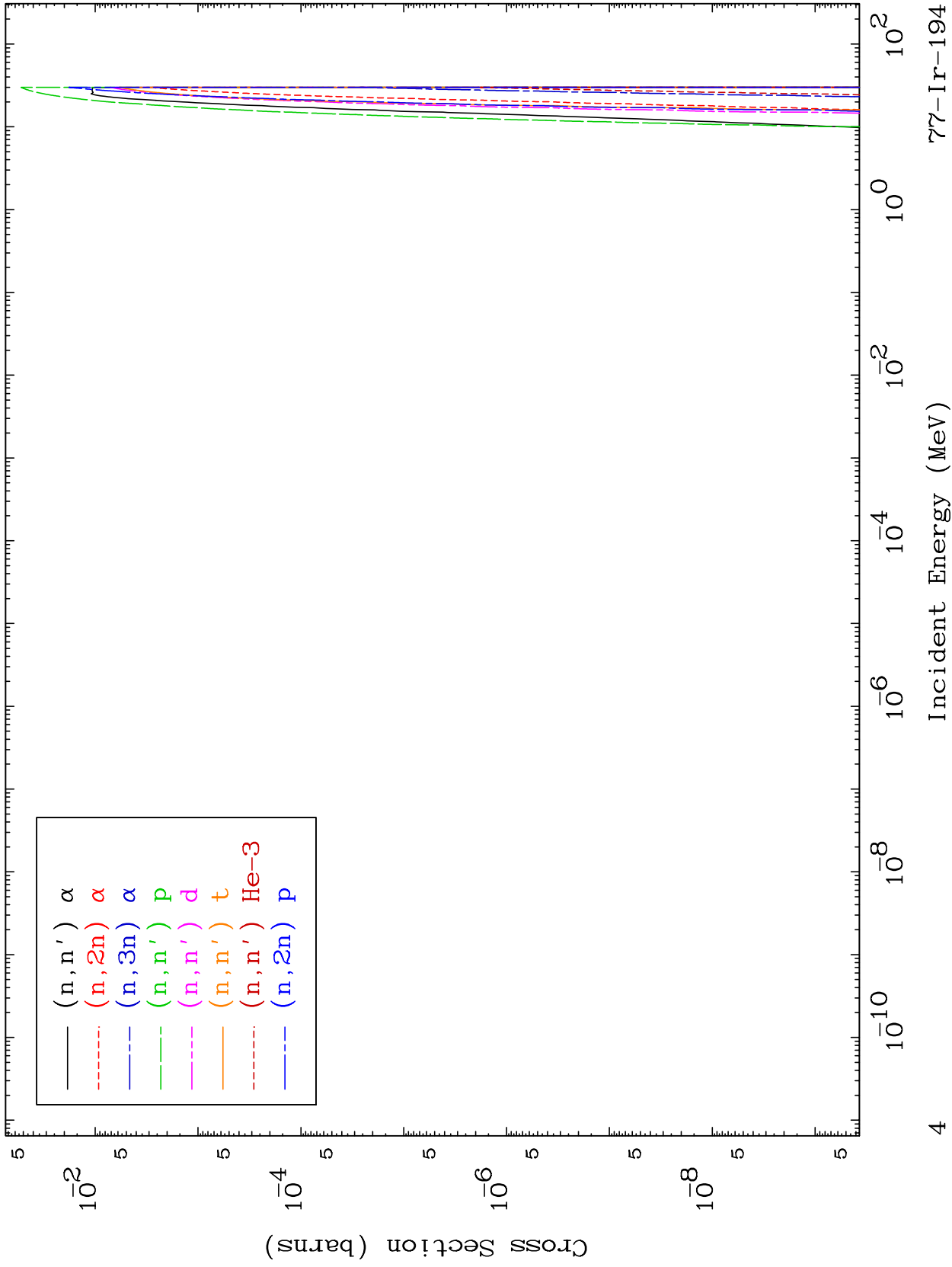
Incident Energy (MeV)

77-Ir-194

MAT 7735

Charged Particle
293 Kelvin Cross Sections

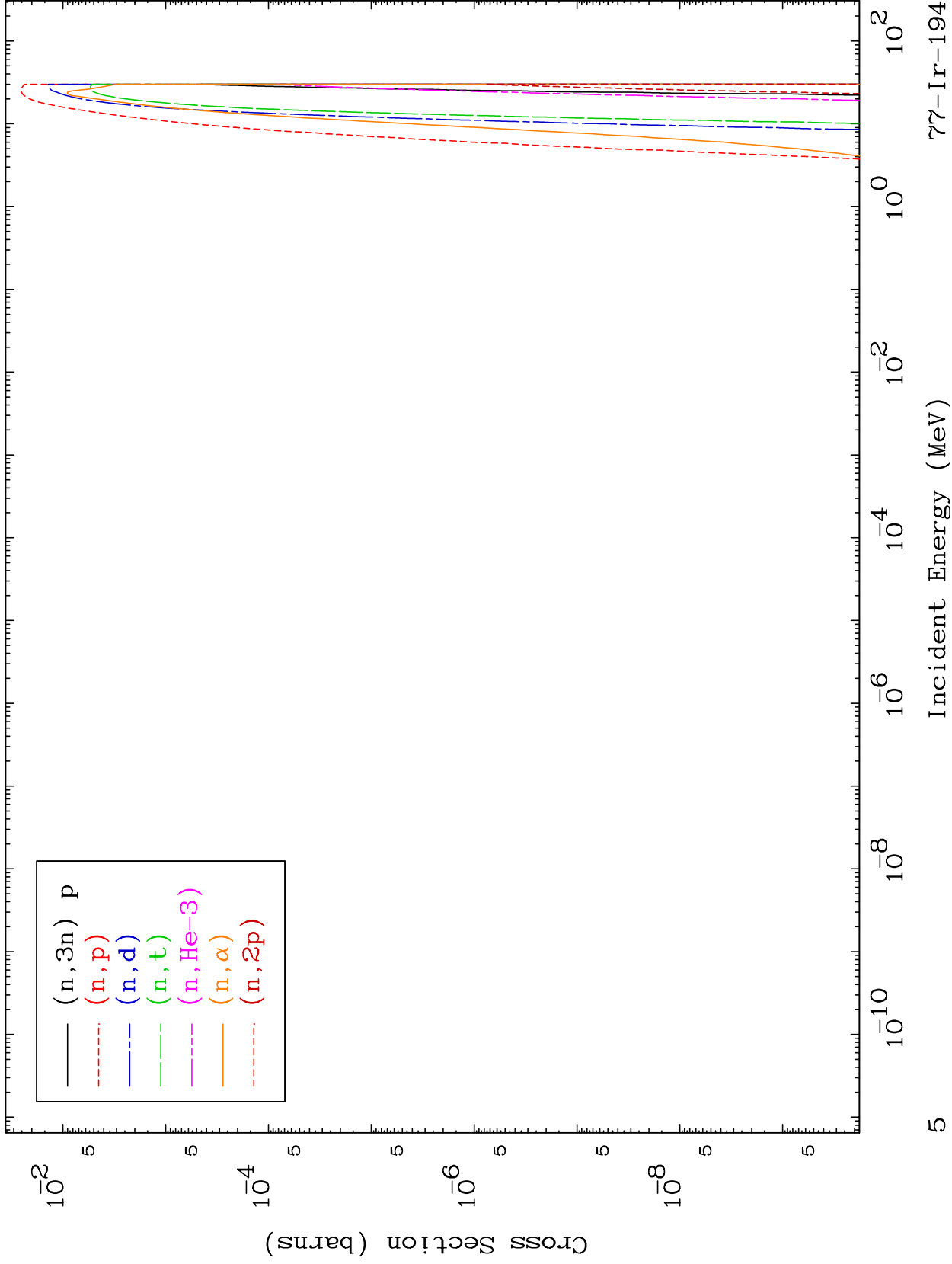
77-Ir-194



MAT 7735

Charged Particle
293 Kelvin Cross Sections

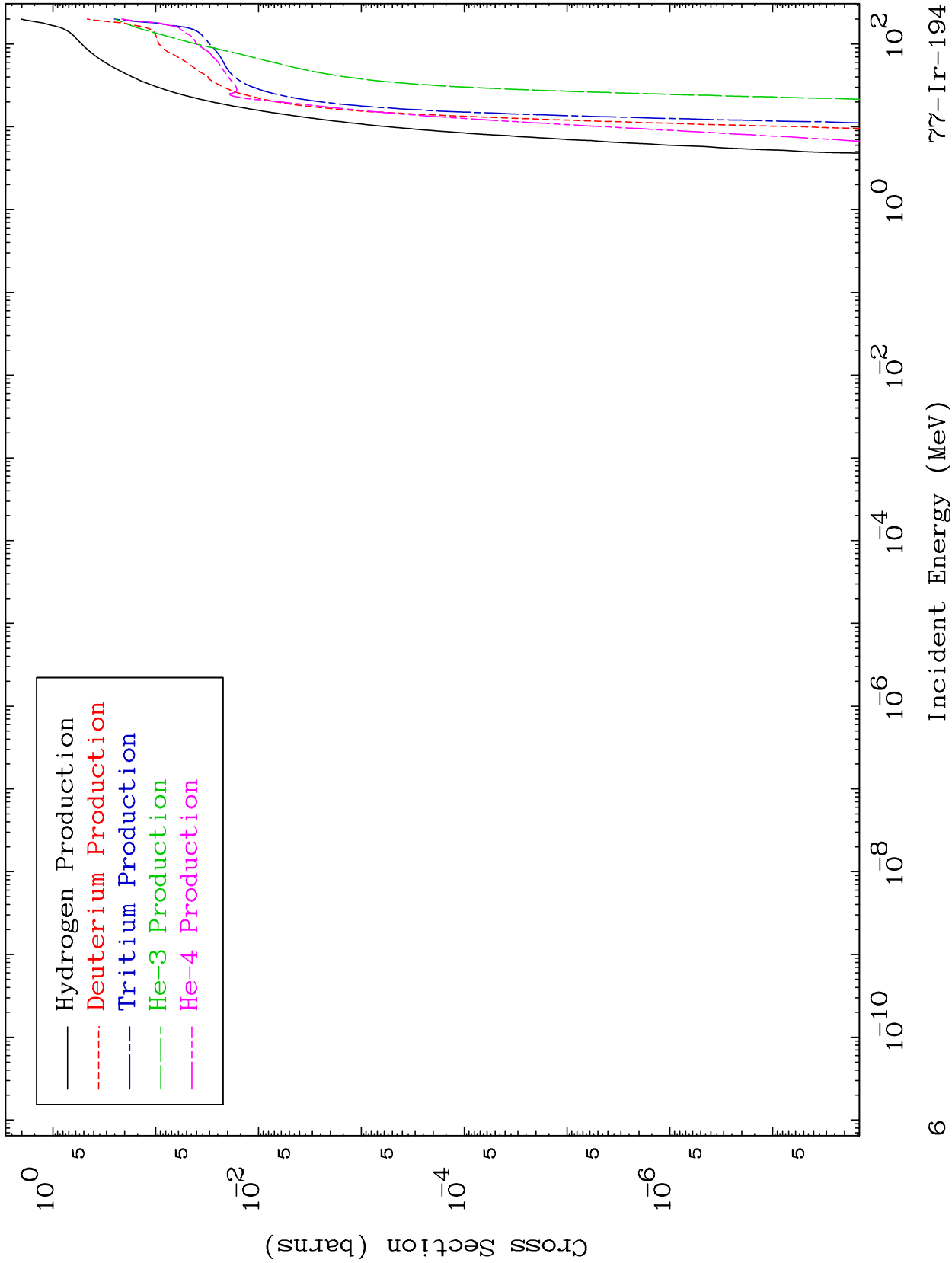
77-Ir-194



MAT 7735

Particle Production
293 Kelvin Cross Sections

77-Ir-194

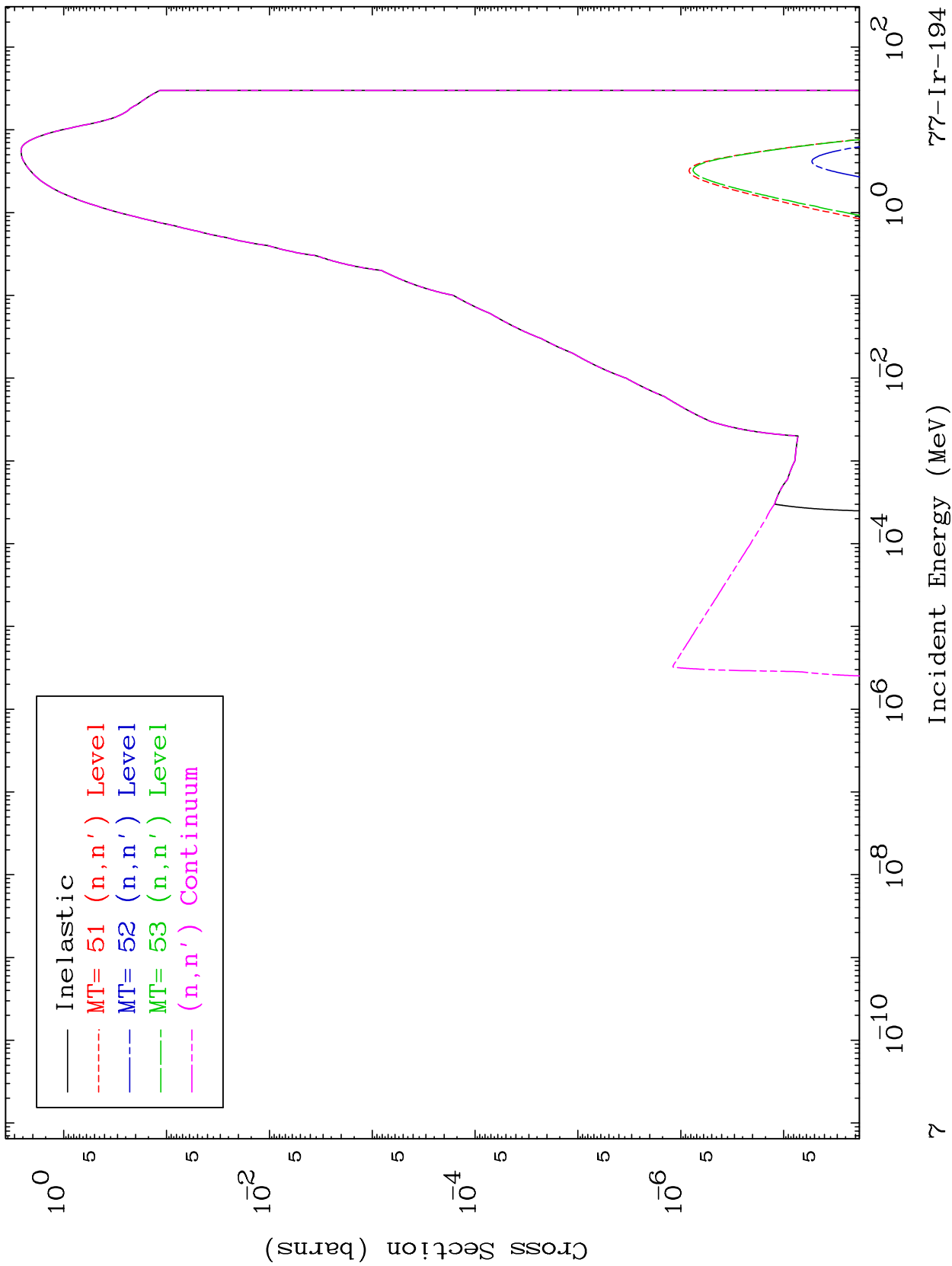


MAT 7735

(n,n') Level

77-Ir-194

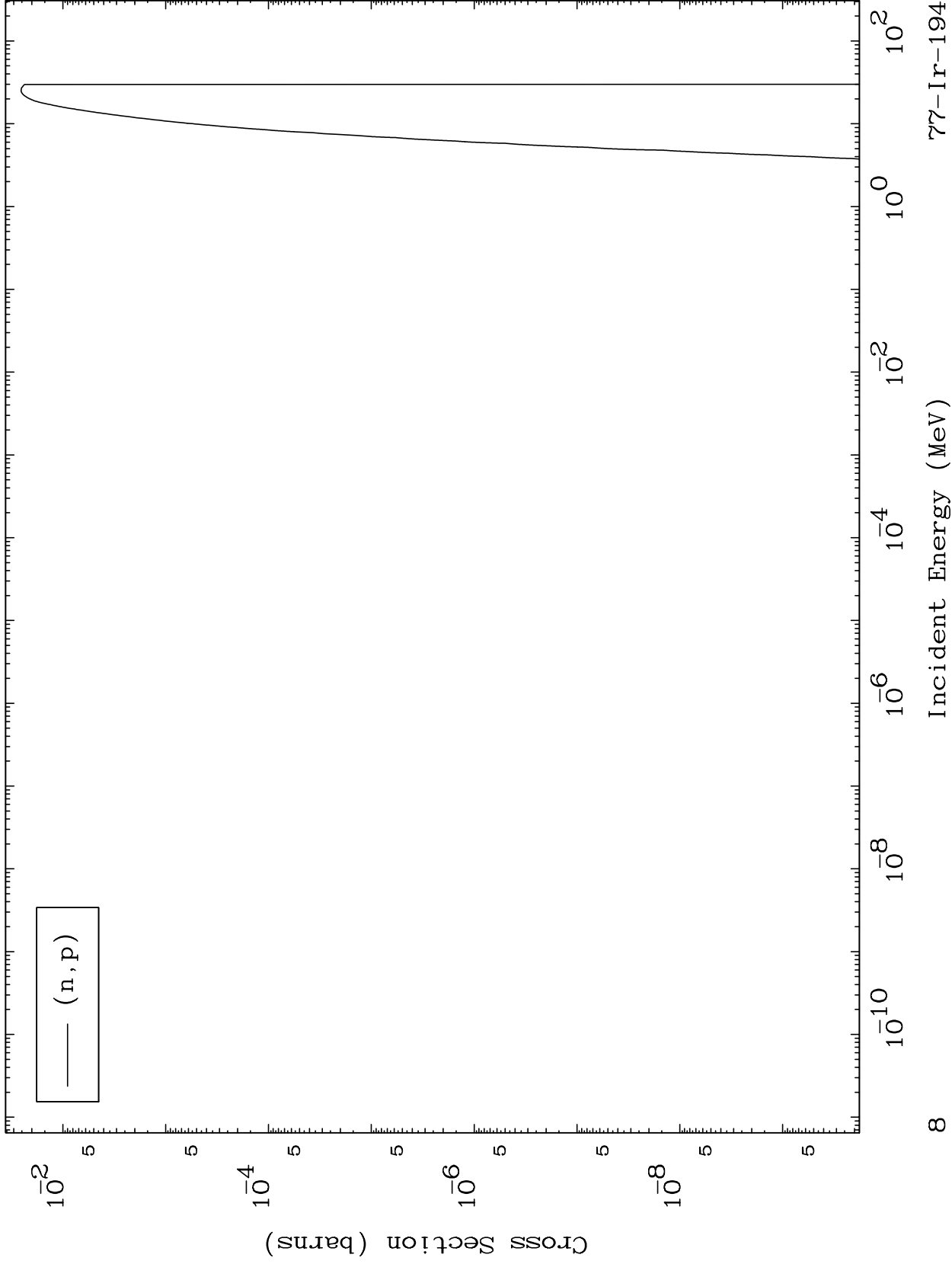
293 Kelvin Cross Sections



MAT 7735

(n,p) Levels
293 Kelvin Cross Sections

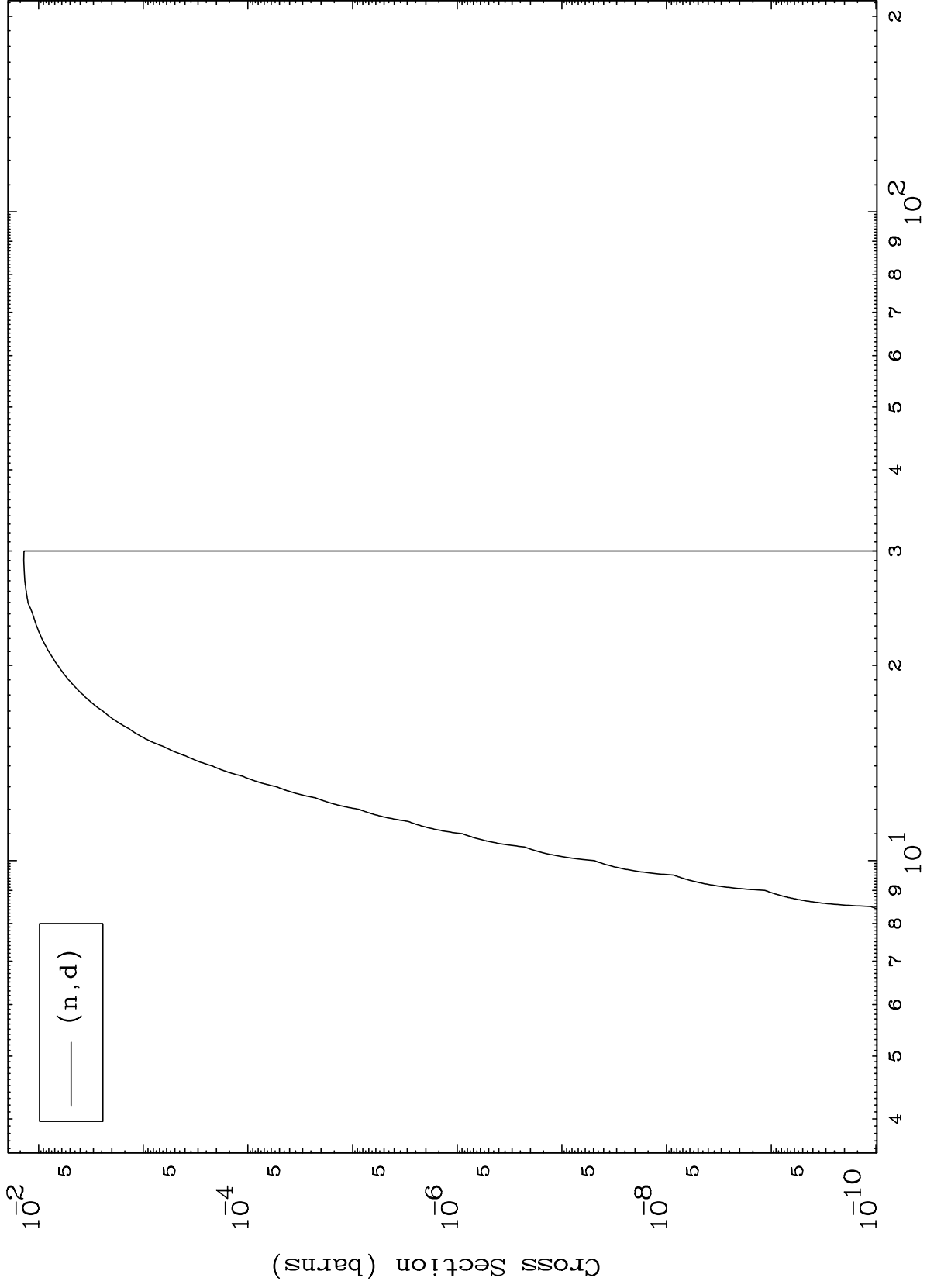
77-Ir-194



MAT 7735

(n,d) Levels
293 Kelvin Cross Sections

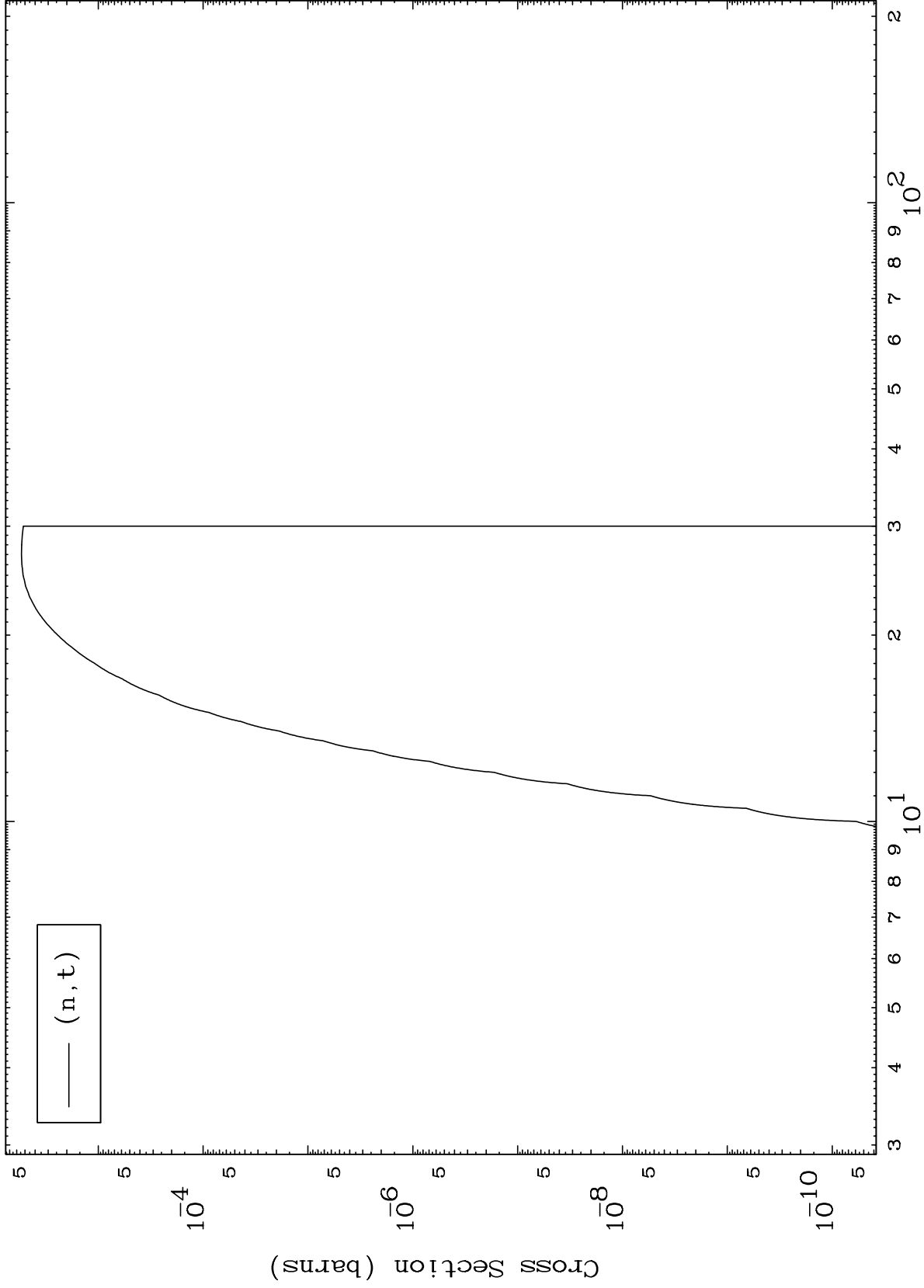
77-Ir-194



MAT 7735

(n,t) Levels
293 Kelvin Cross Sections

77-Ir-194



Incident Energy (MeV)

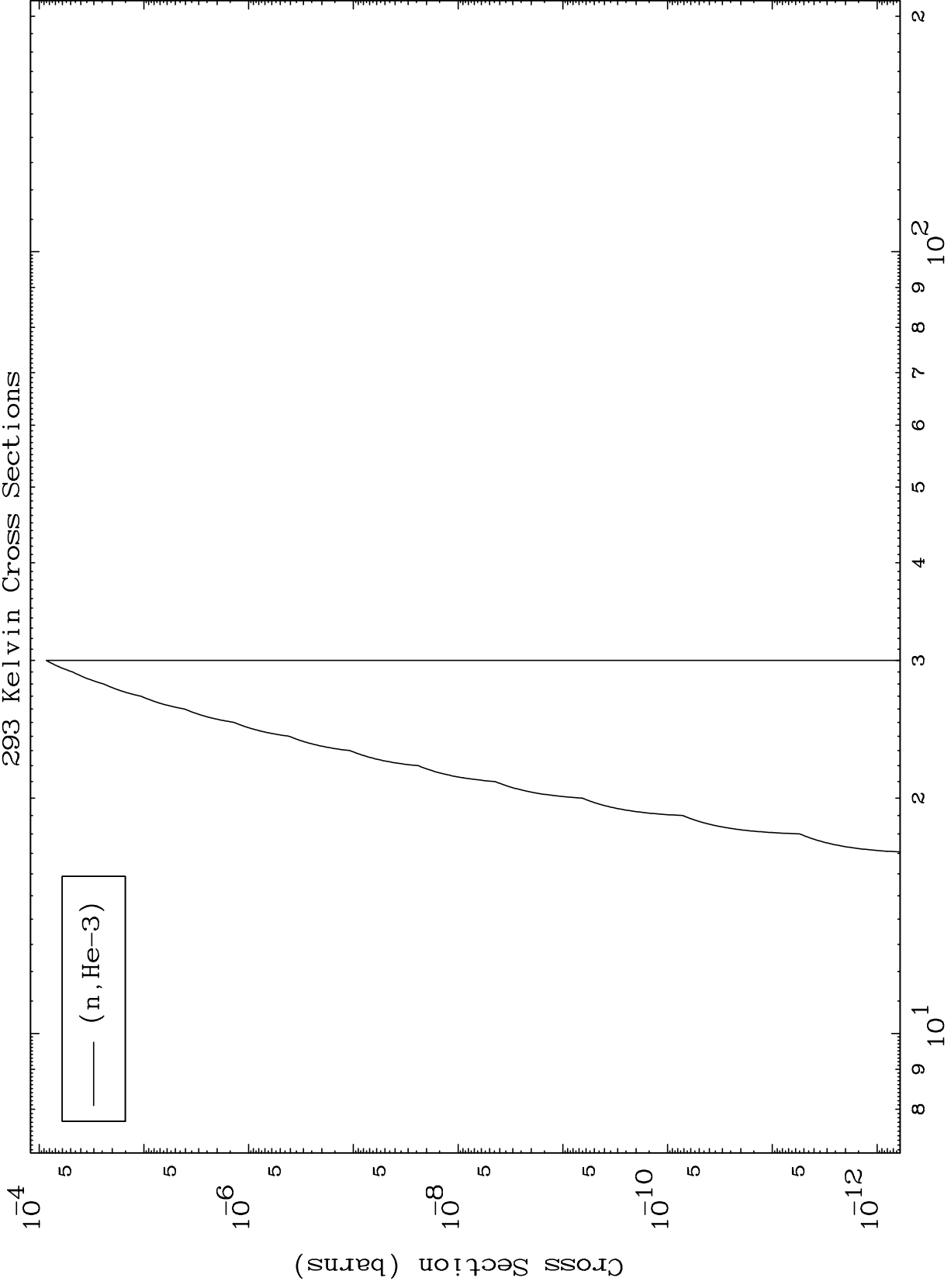
77-Ir-194

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MAT 7735

(n,He3) Levels
293 Kelvin Cross Sections

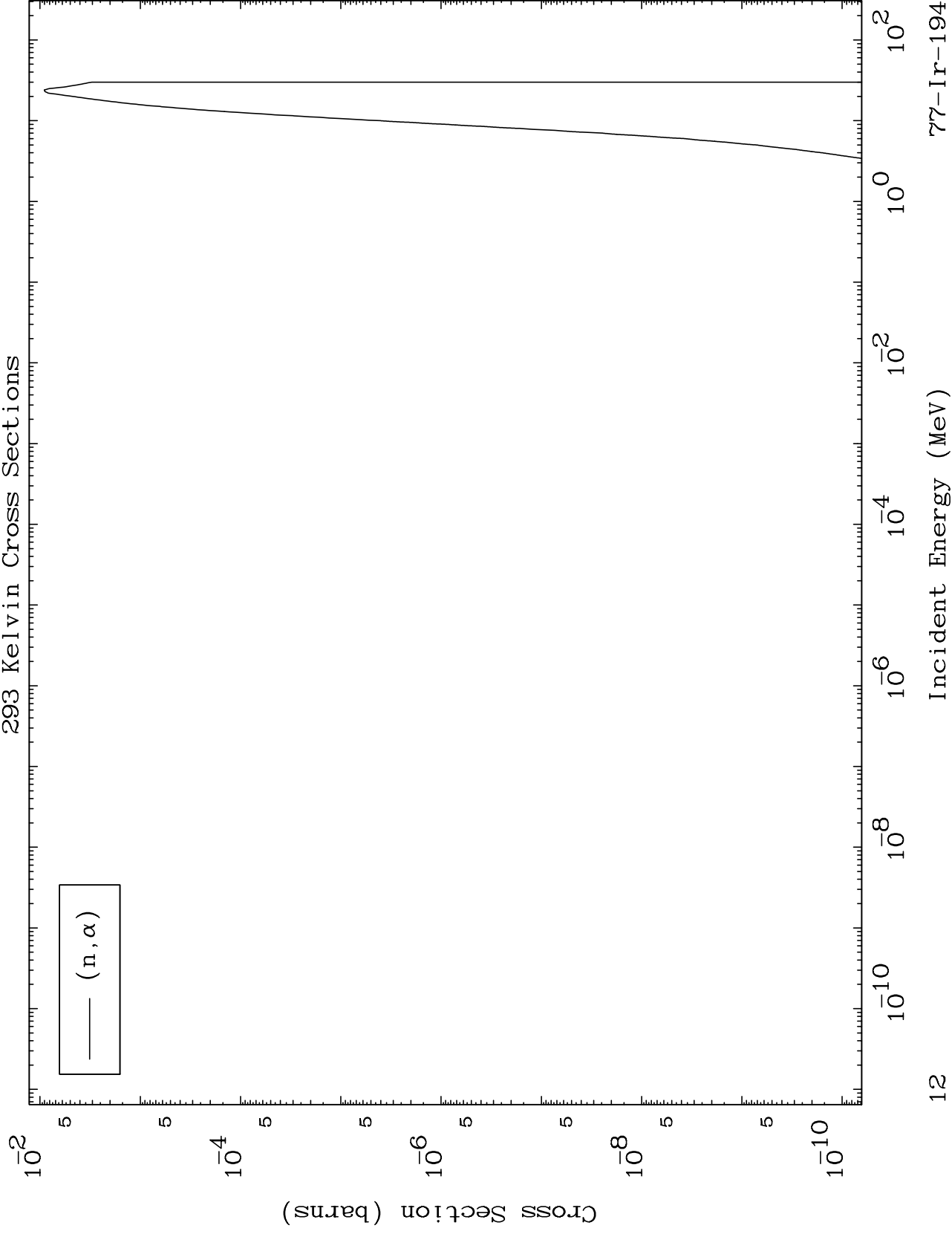
77-Ir-194



MAT 7735

(n, α) Levels
293 Kelvin Cross Sections

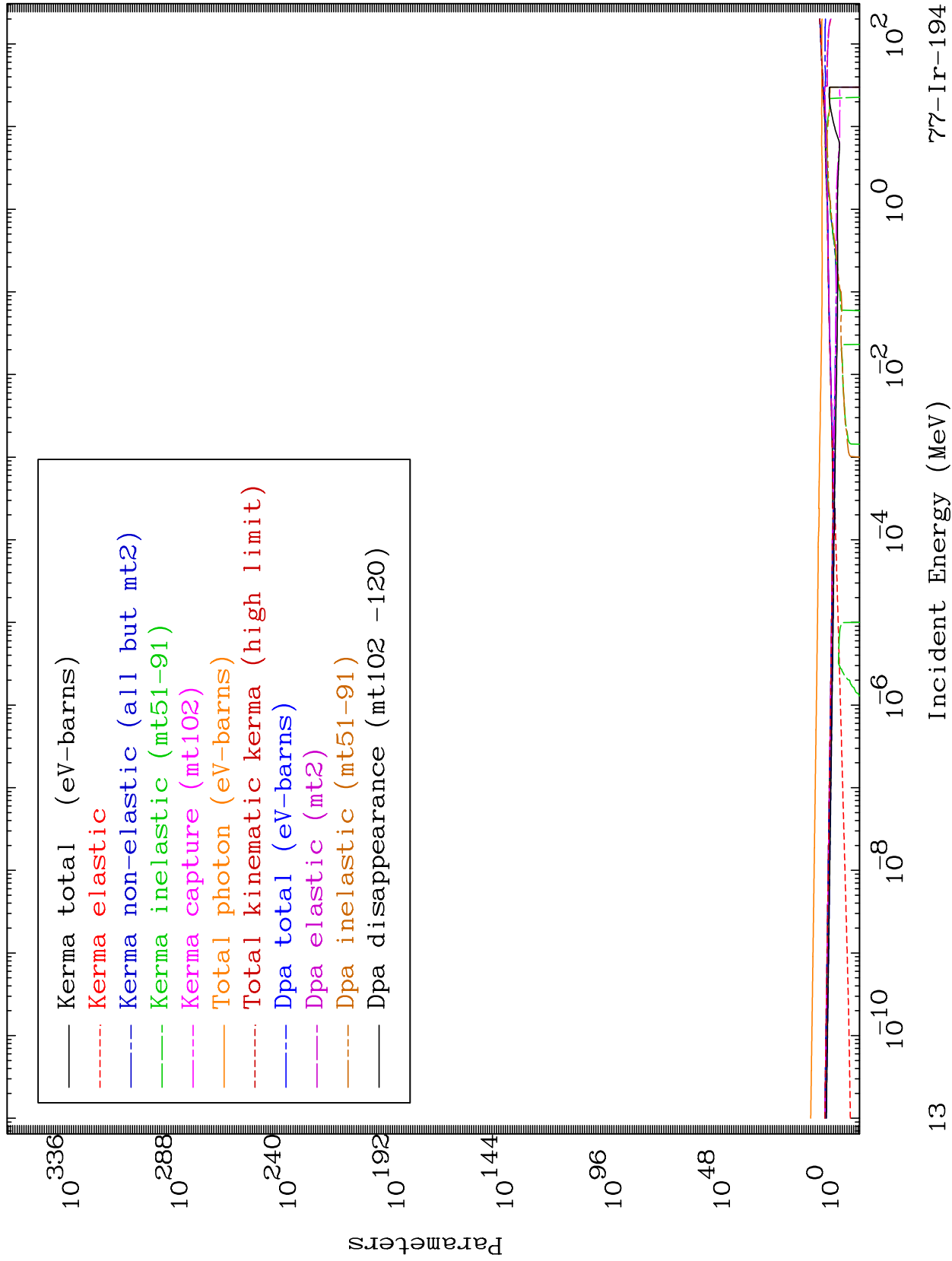
77-Ir-194

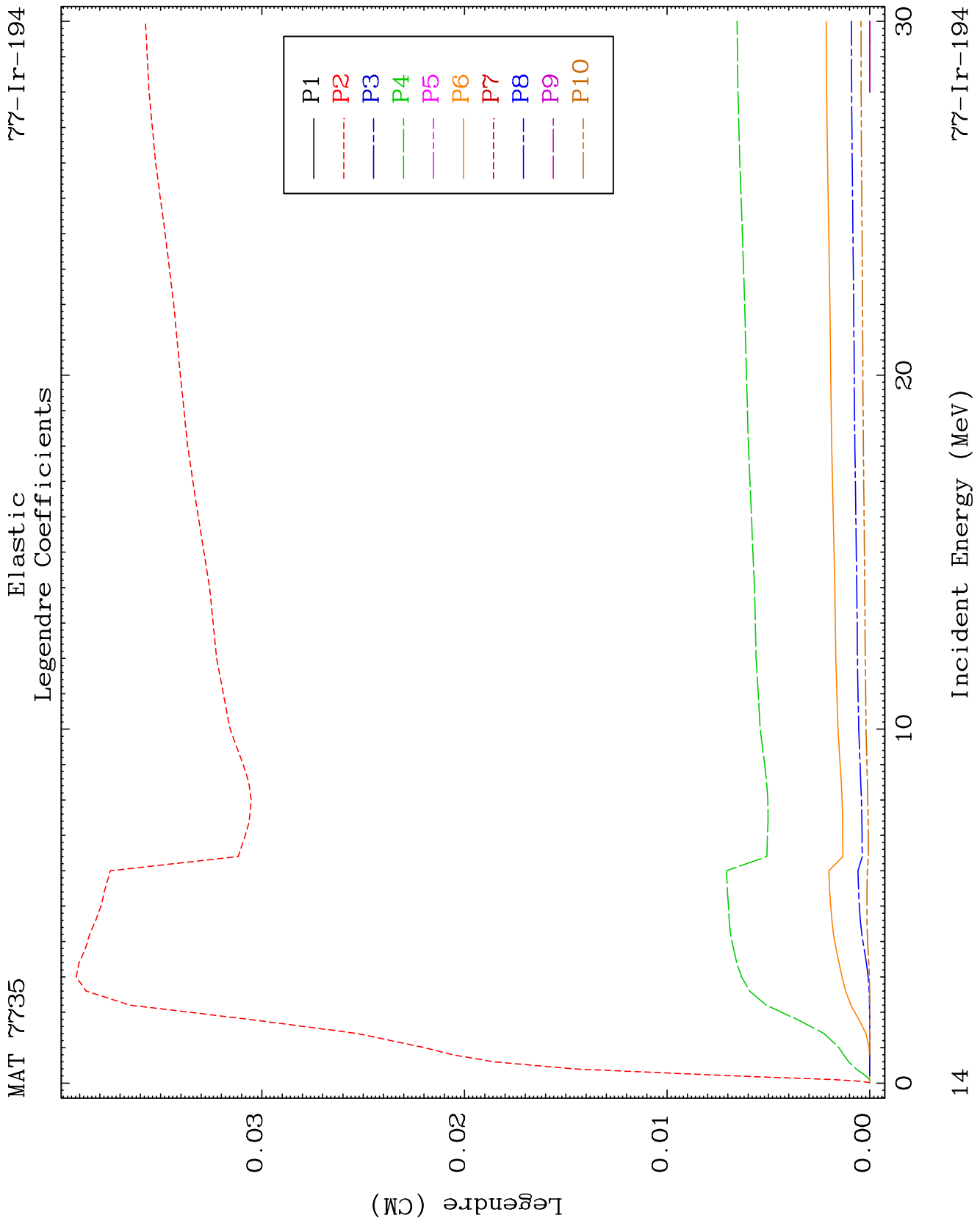


12

Incident Energy (MeV)

77-Ir-194

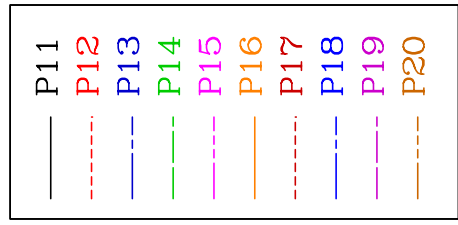




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Elastic
Legendre Coefficients

77-Ir-194



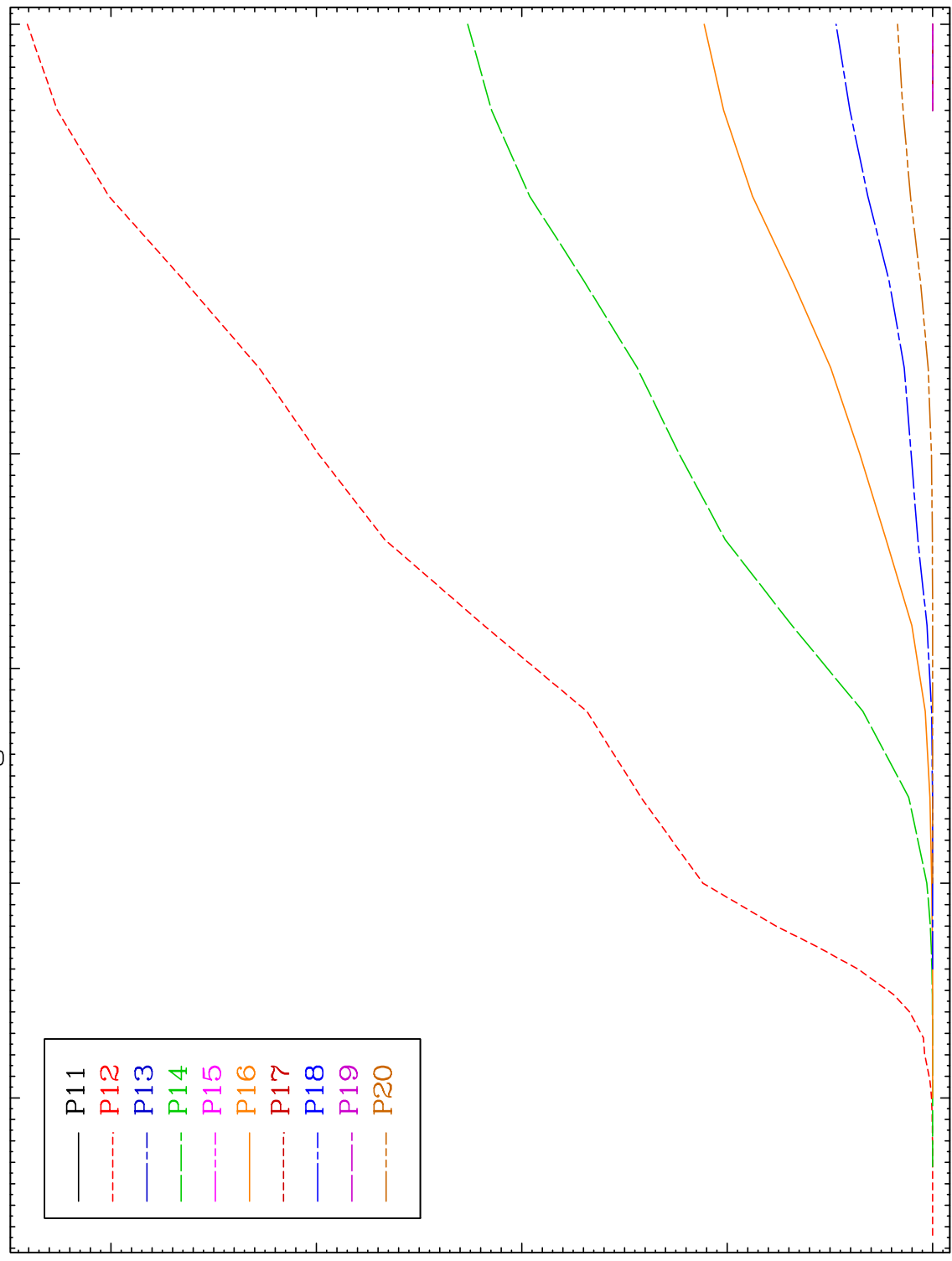
Legendre (CM) $\times 10^{-4}$

2.0
1.5
1.0
0.5
0.0

Incident Energy (MeV)

15 5 10 15 20 25 30

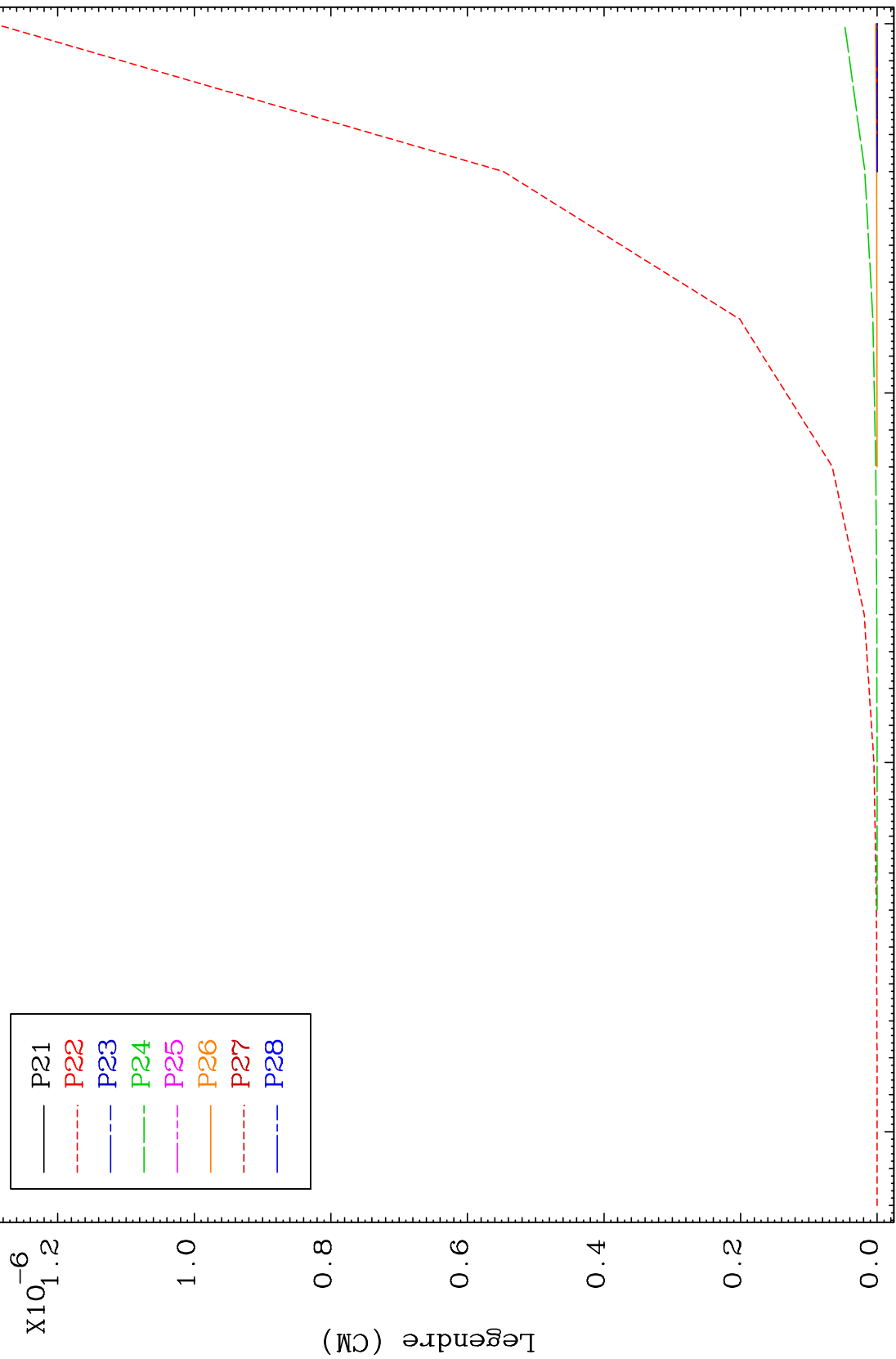
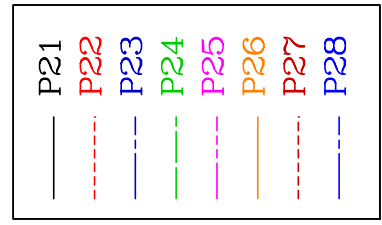
77-Ir-194



MAT 7735

Elastic
Legendre Coefficients

77-Ir-194



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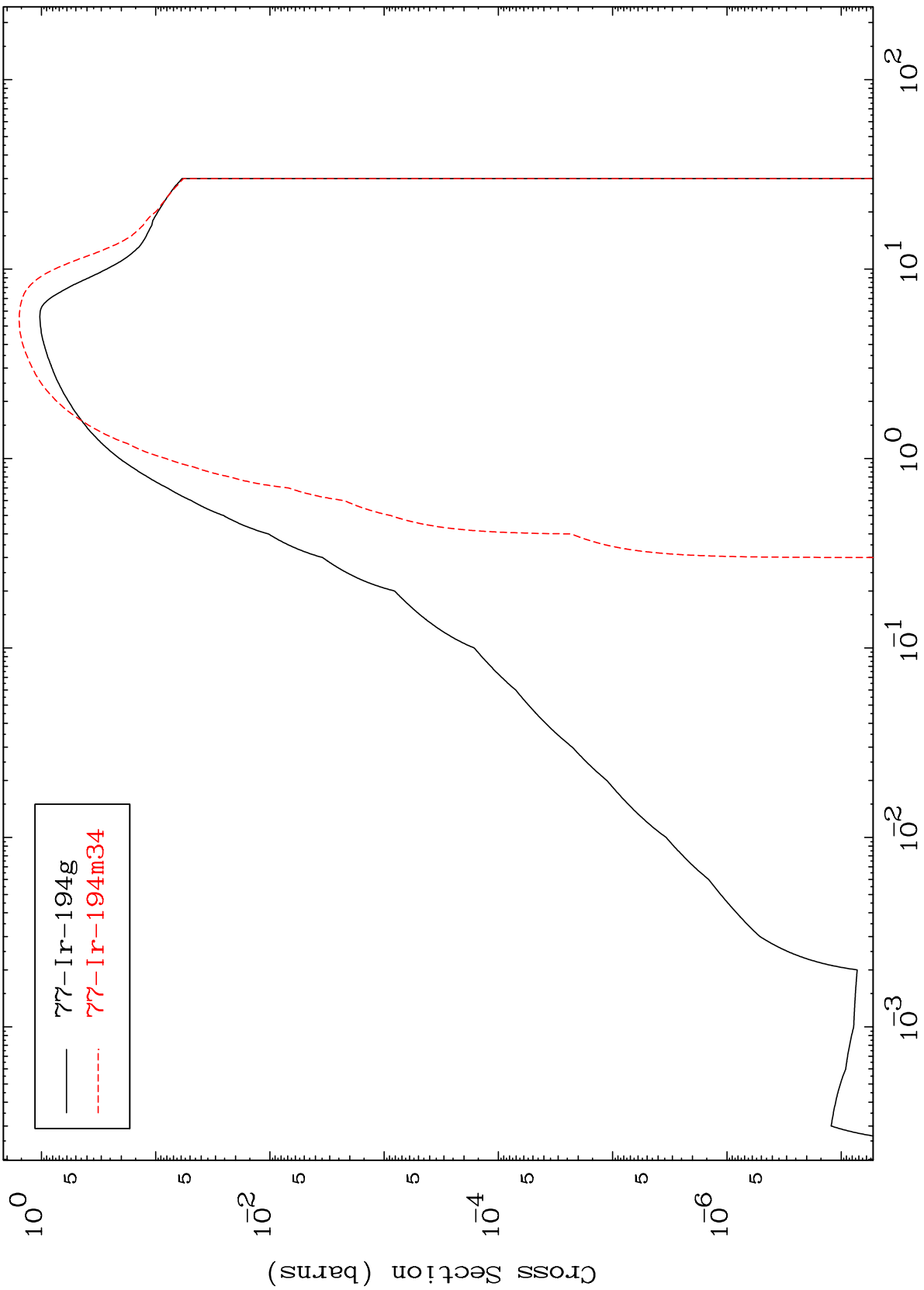
Incident Energy (MeV)

77-Ir-194

MAT 7735

77-Ir-194

Inelastic
Radionuclide Production Cross Section



77-Ir-194g
77-Ir-194m34

77-Ir-194

Incident Energy (MeV)

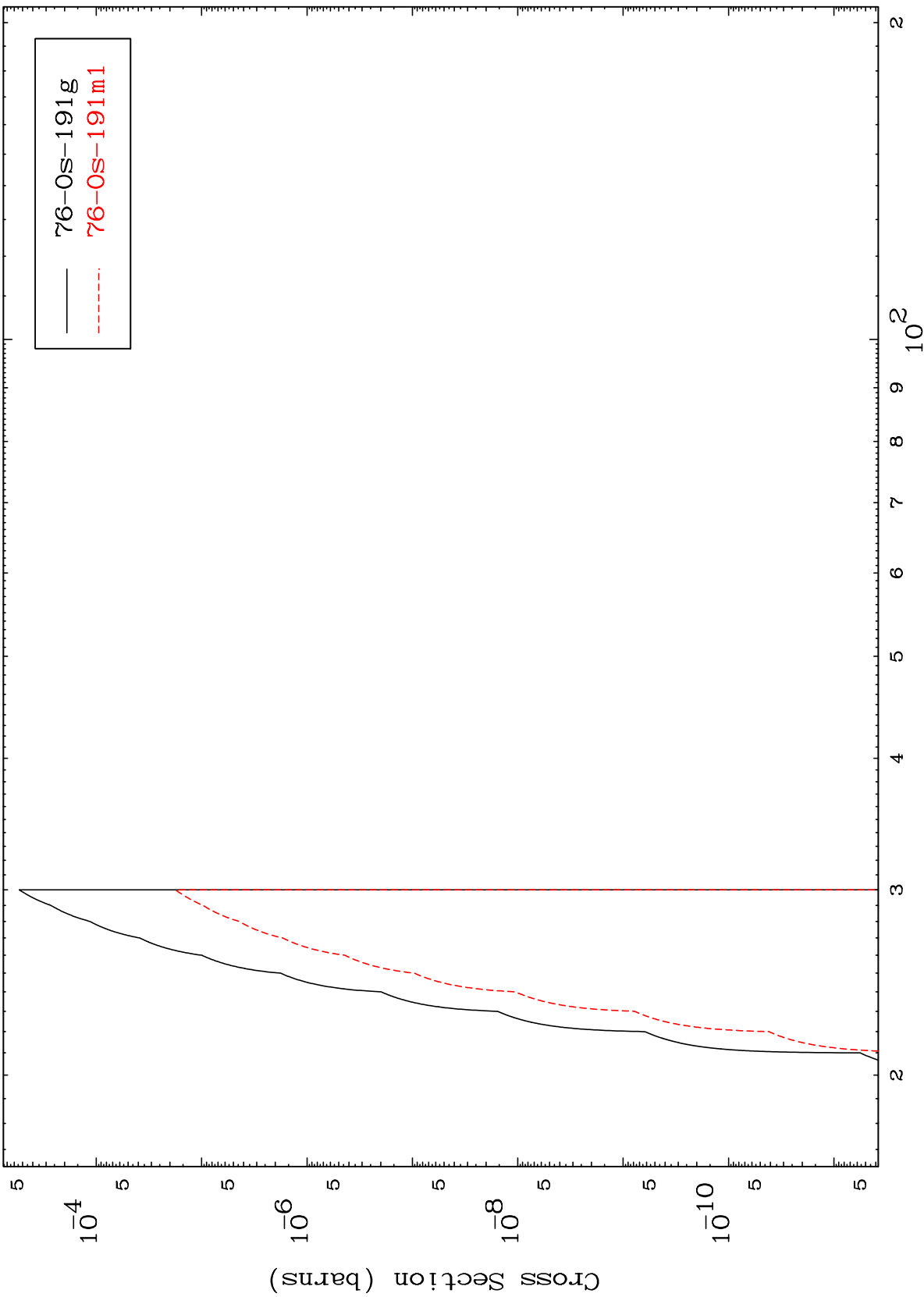
17

MAT 7735

(n,2n) d

77-Ir-194

Radionuclide Production Cross Section



76-Os-191g
76-Os-191m1

18

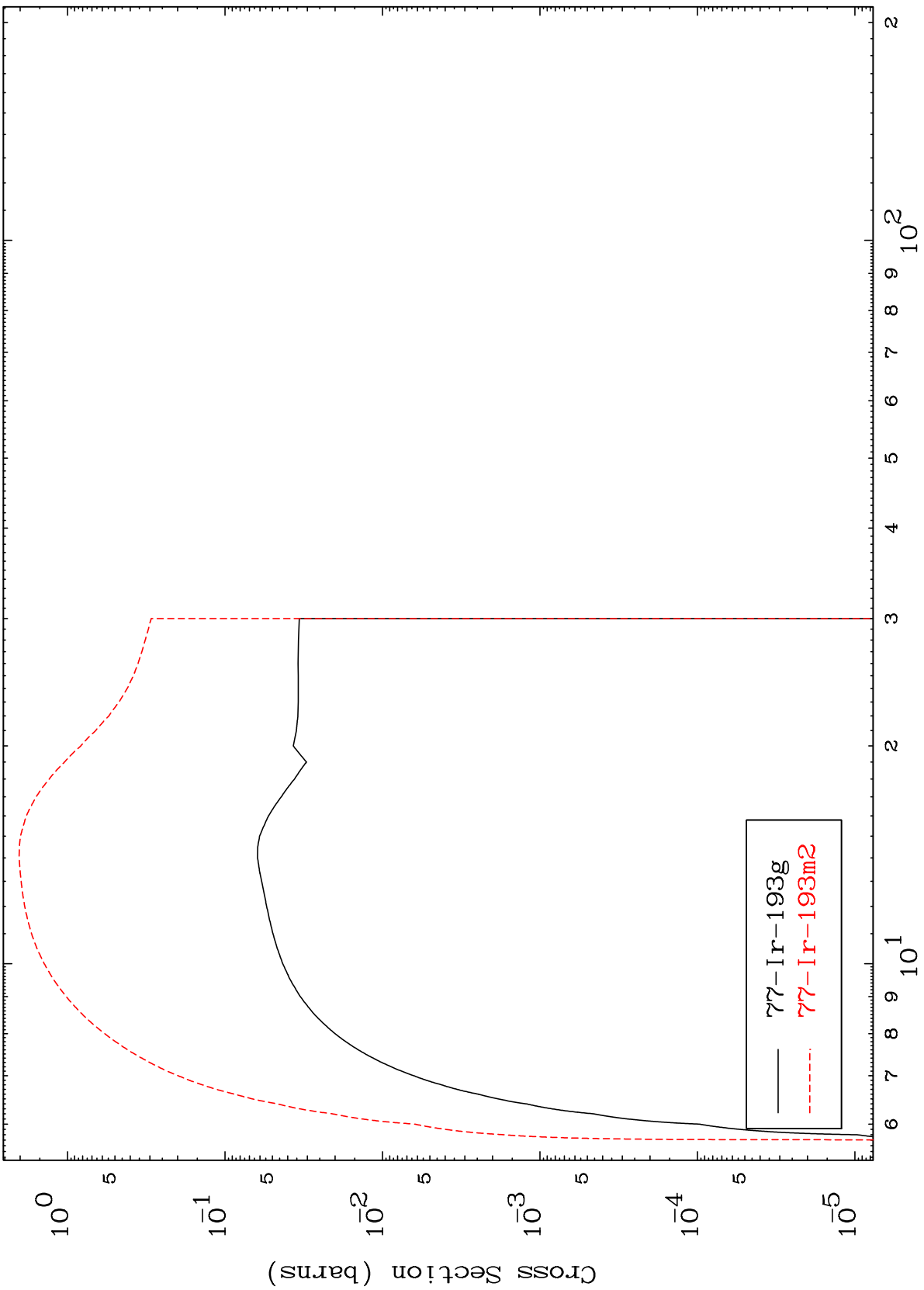
Incident Energy (MeV)

77-Ir-194

MAT 7735

⁷⁷Ir-194

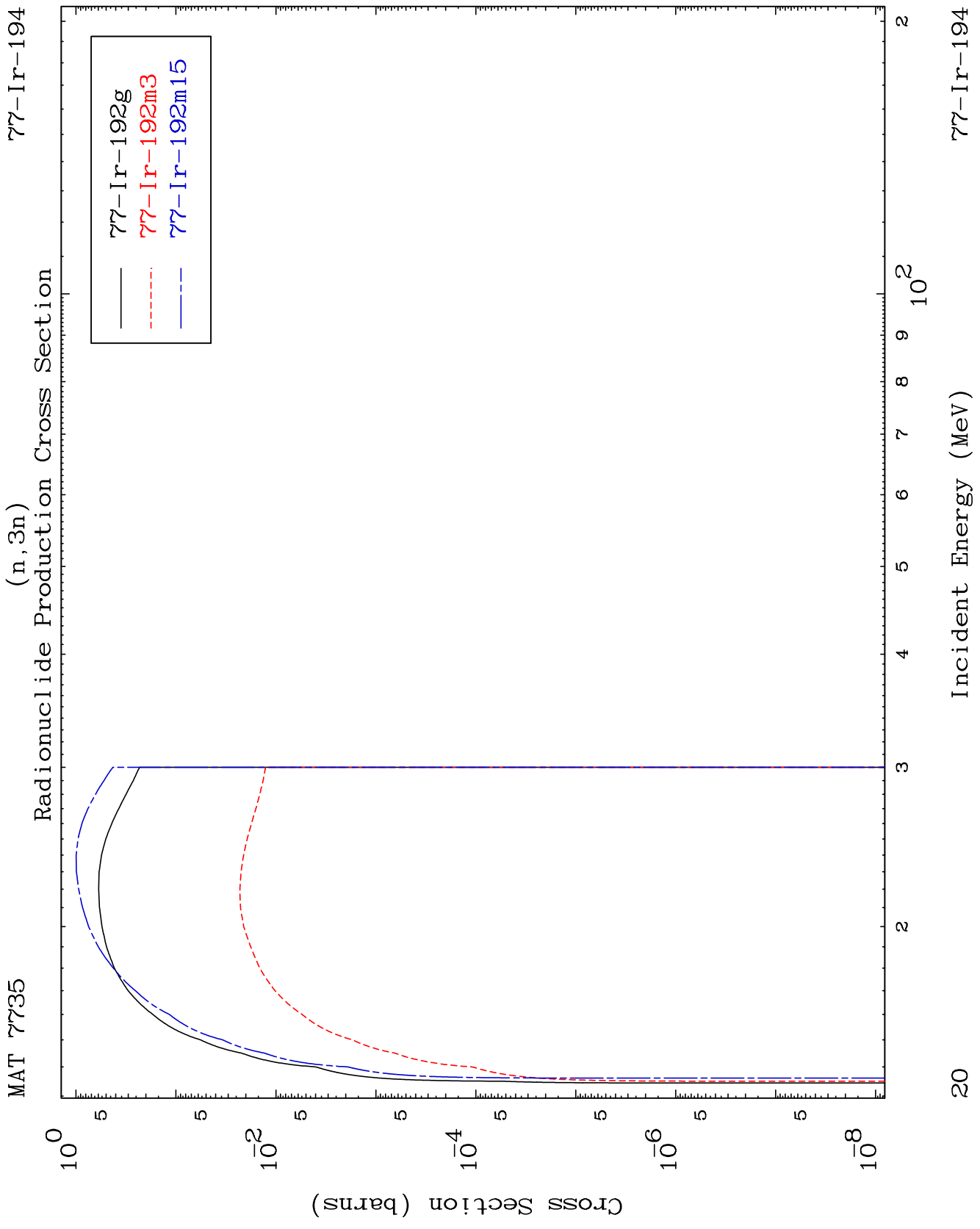
Radionuclide Production Cross Section
(n,2n)



19

Incident Energy (MeV)

⁷⁷Ir-194

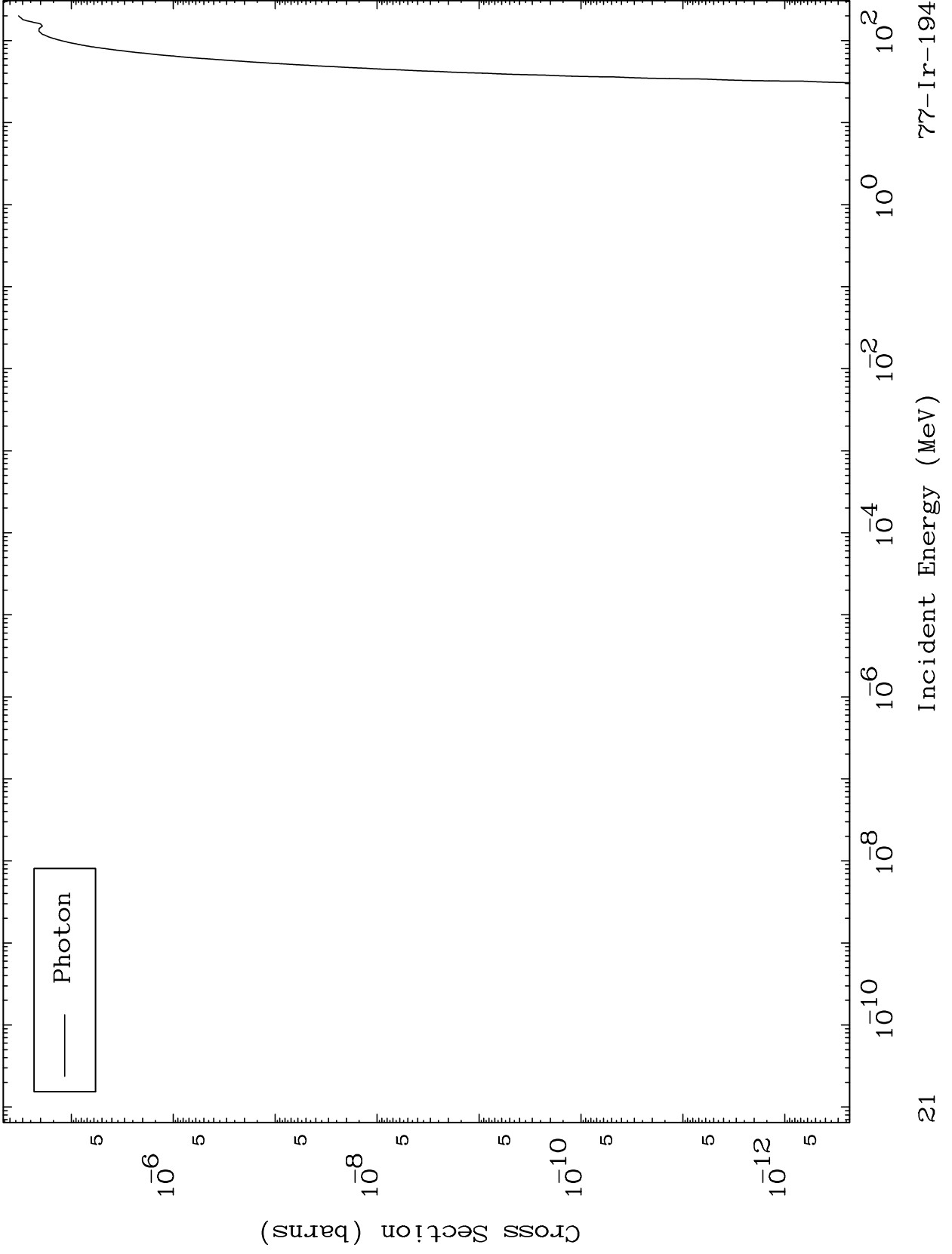


MAT 7735

Fission

⁷⁷Ir-194

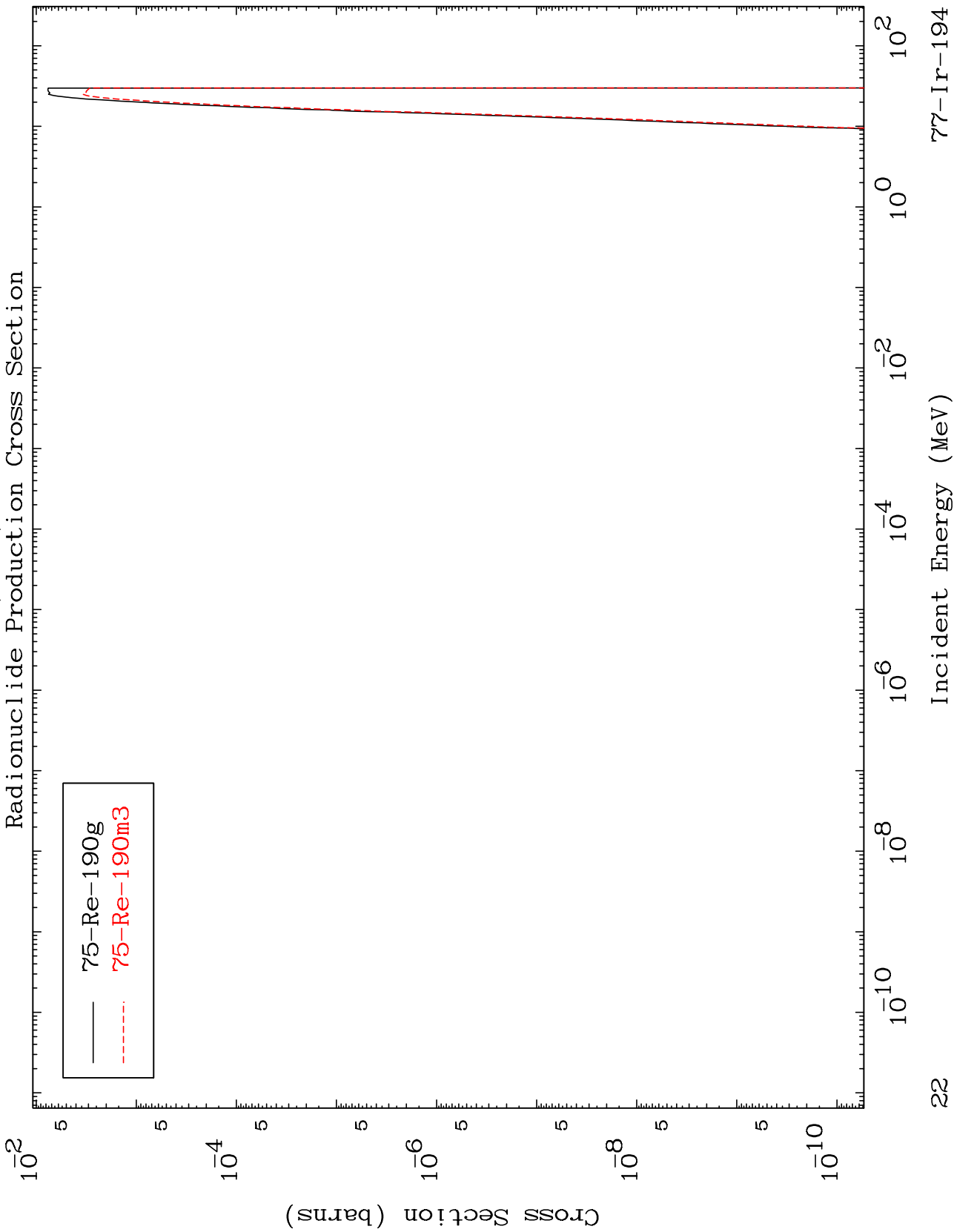
Radionuclide Production Cross Section



MAT 7735

$(n, n') \alpha$

$^{77}\text{Ir-194}$



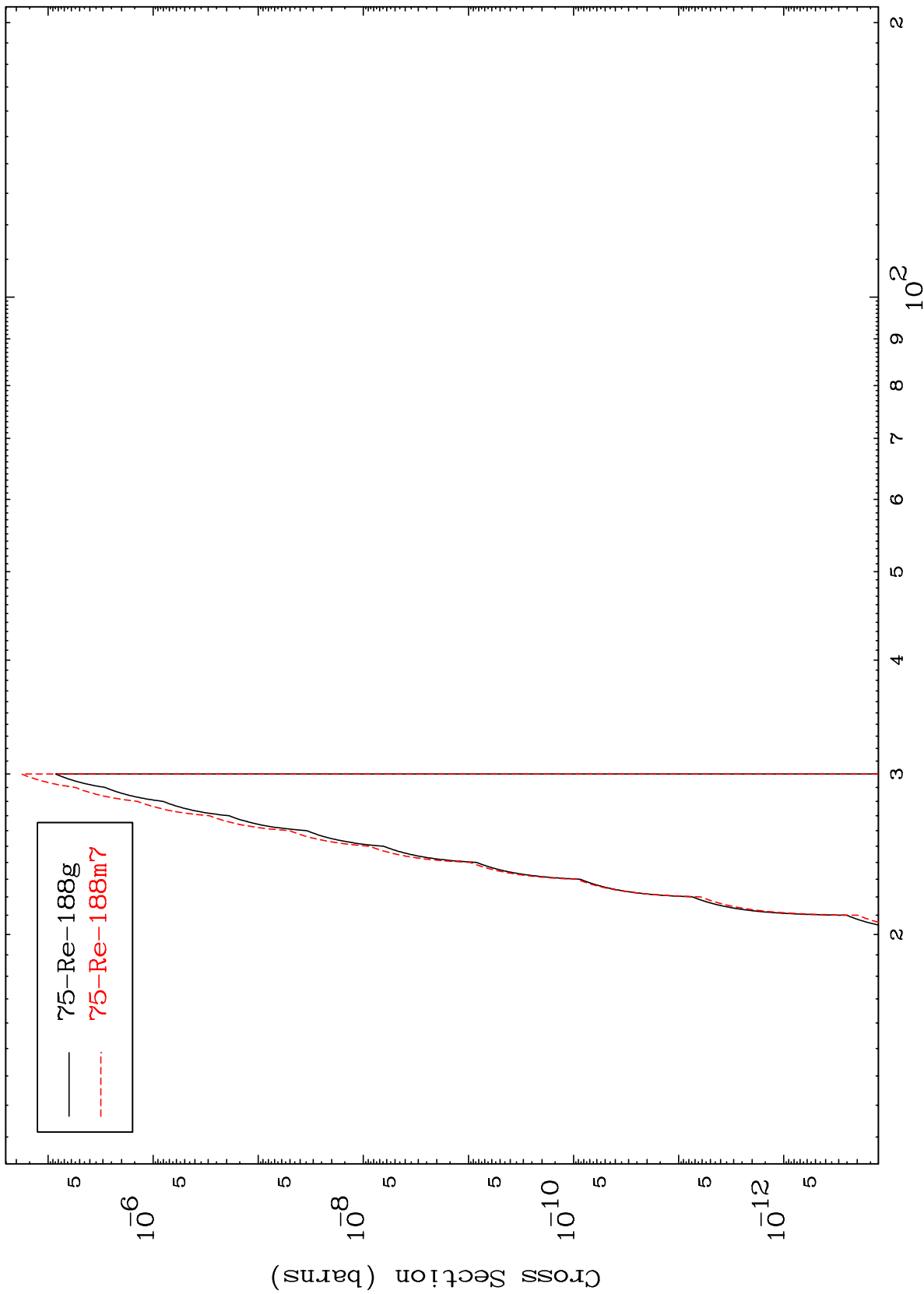
75-Re-190g
75-Re-190m3

MAT 7735

(n,3n) α

⁷⁷Ir-194

Radionuclide Production Cross Section

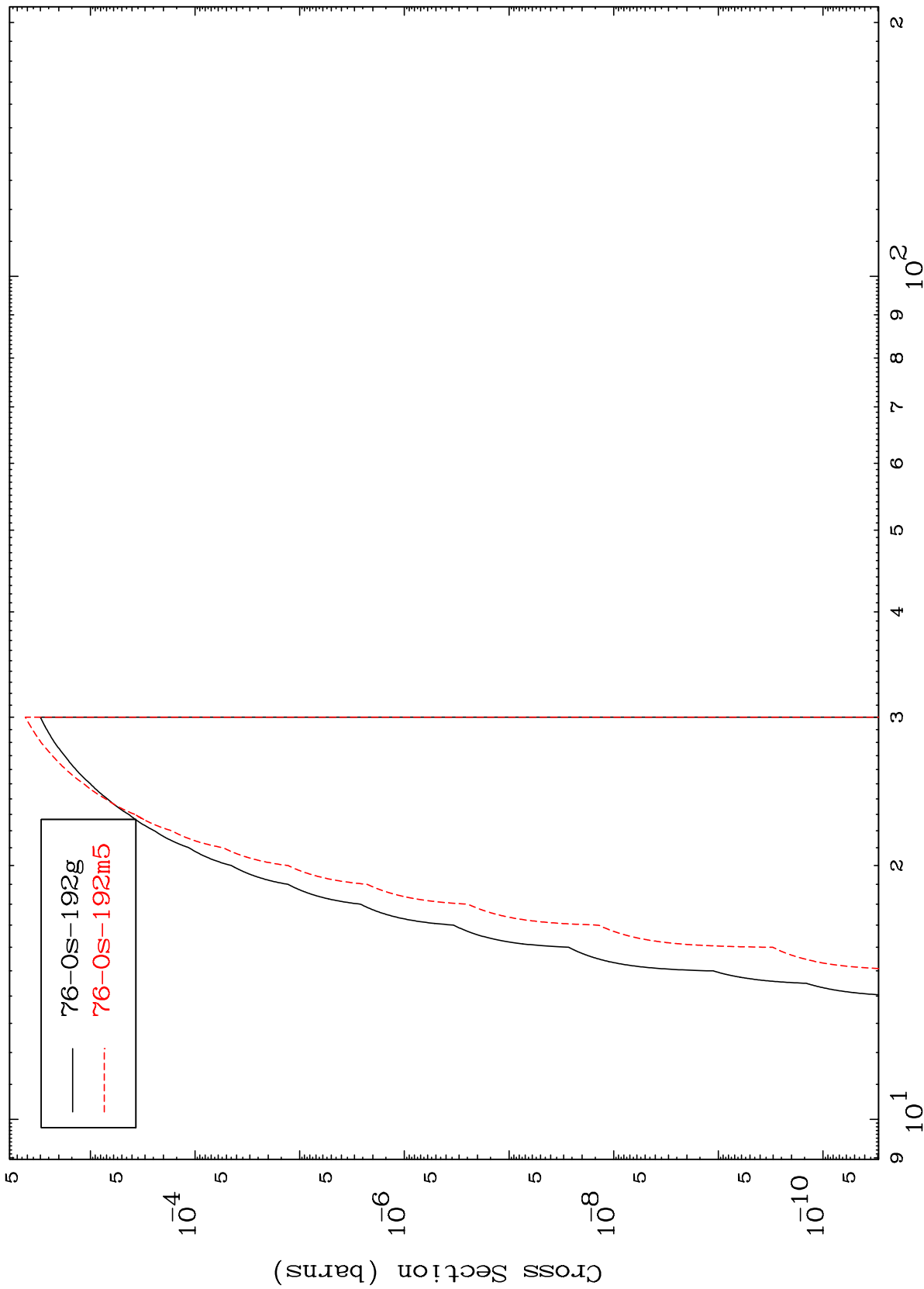


MAT 7735

(n,n') d

77-Ir-194

Radionuclide Production Cross Section



24

Incident Energy (MeV)

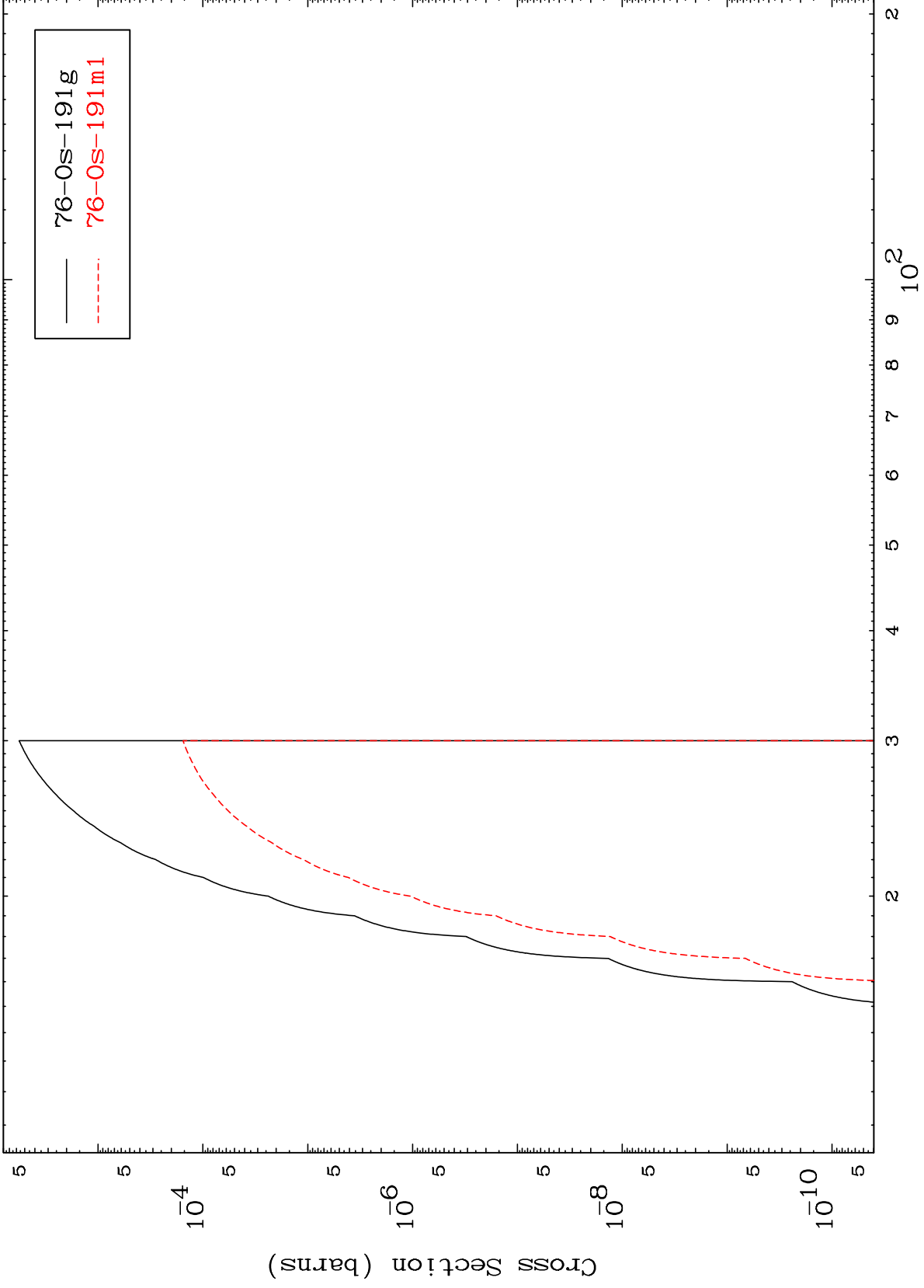
77-Ir-194

MAT 7735

(n,n') t

77-Ir-194

Radionuclide Production Cross Section

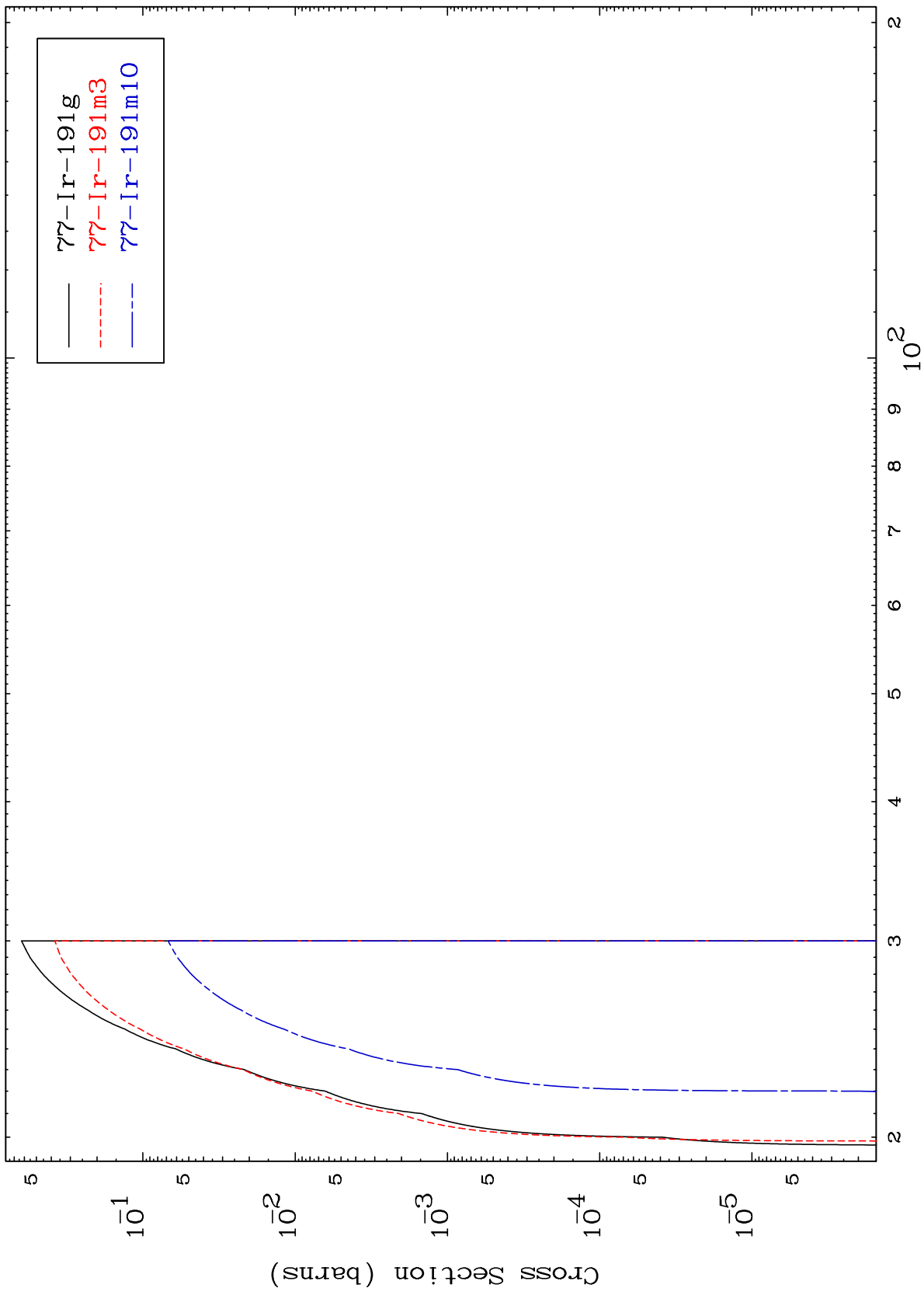


25

Incident Energy (MeV)

77-Ir-194

(n,4n)
Radionuclide Production Cross Section

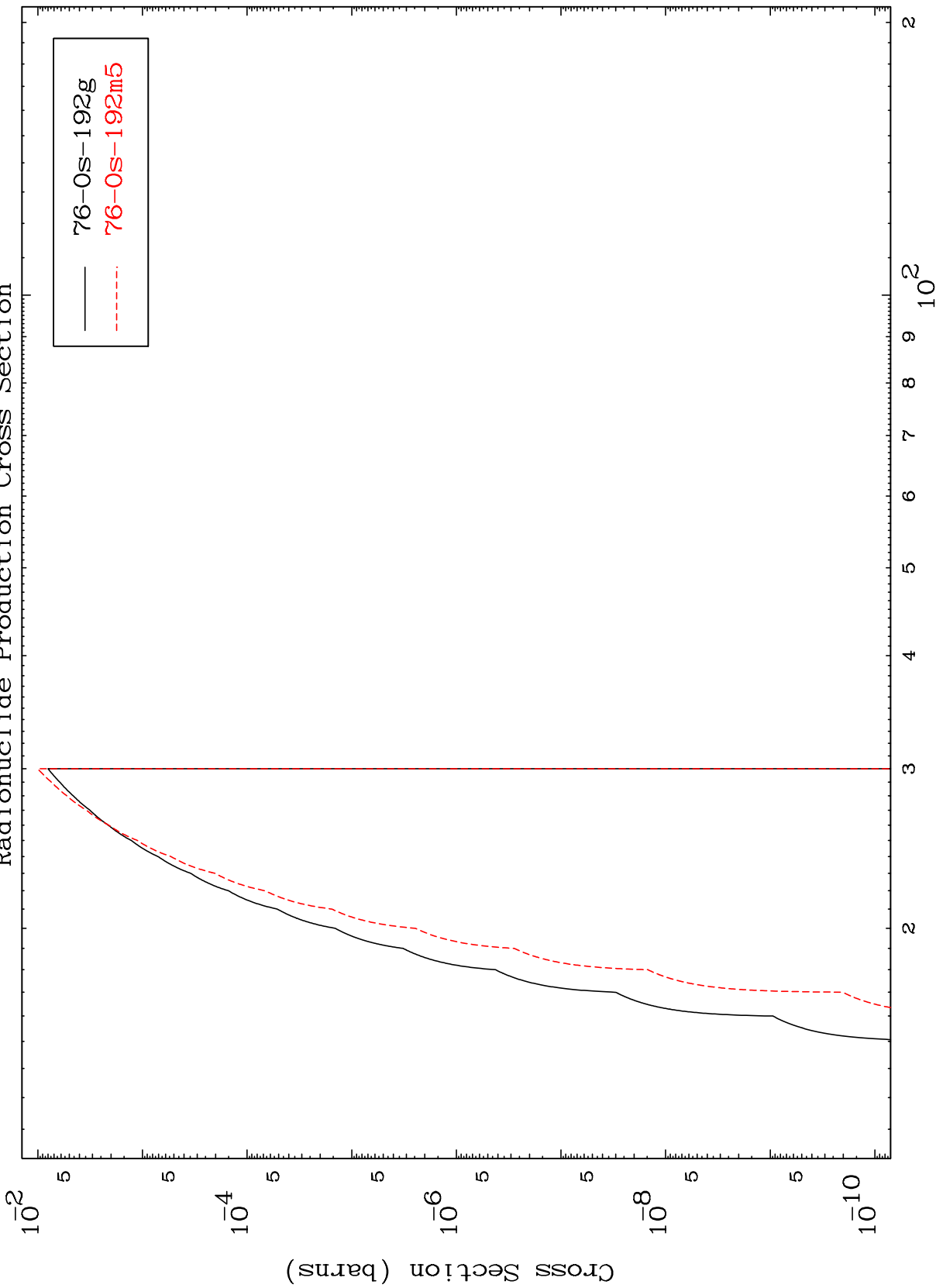


MAT 7735

(n,2n) p

77-Ir-194

Radionuclide Production Cross Section



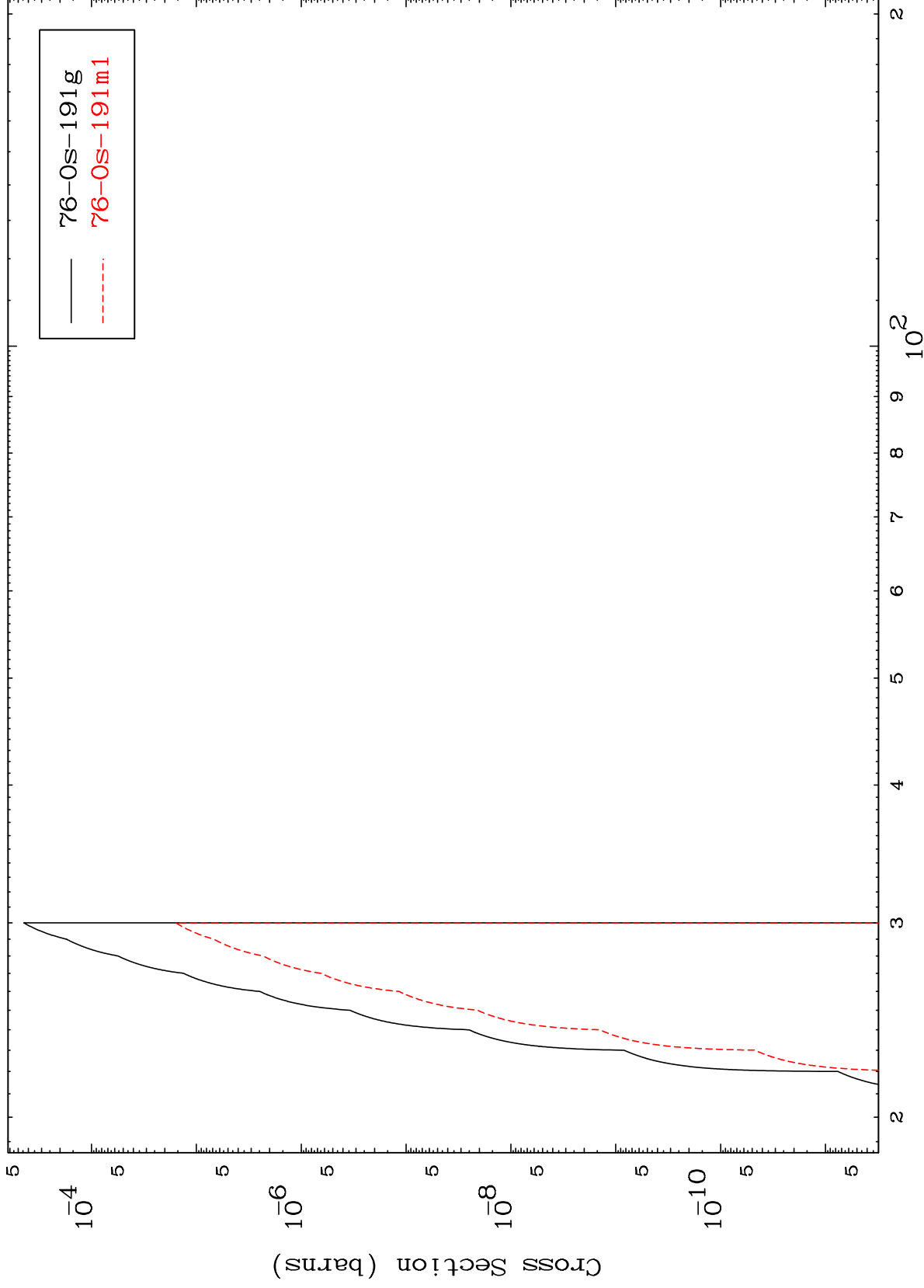
76-Os-192g
76-Os-192m5

MAT 7735

(n,3n) p

77-Ir-194

Radionuclide Production Cross Section



28

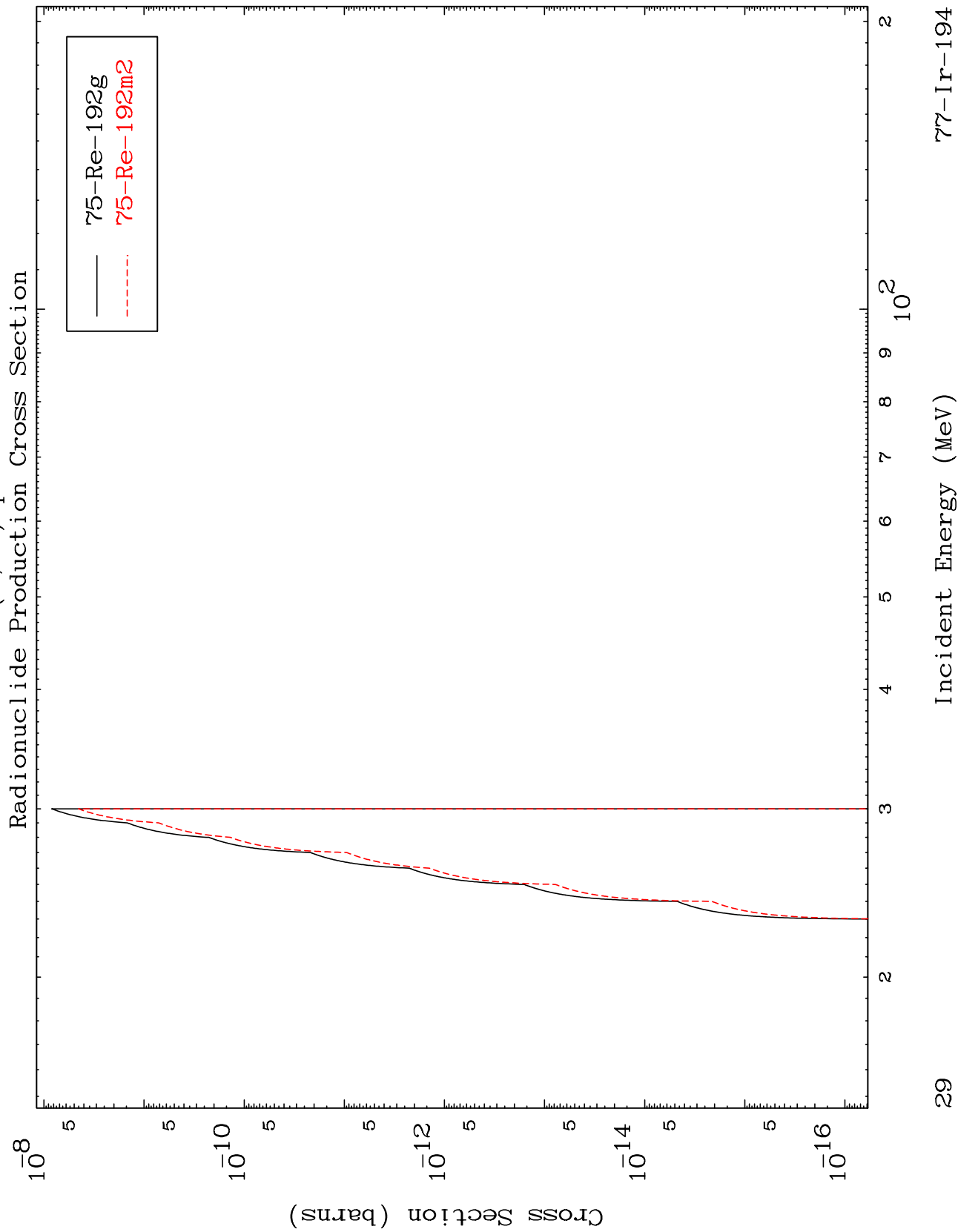
Incident Energy (MeV)

77-Ir-194

MAT 7735

(n,2n) p

77-Ir-194



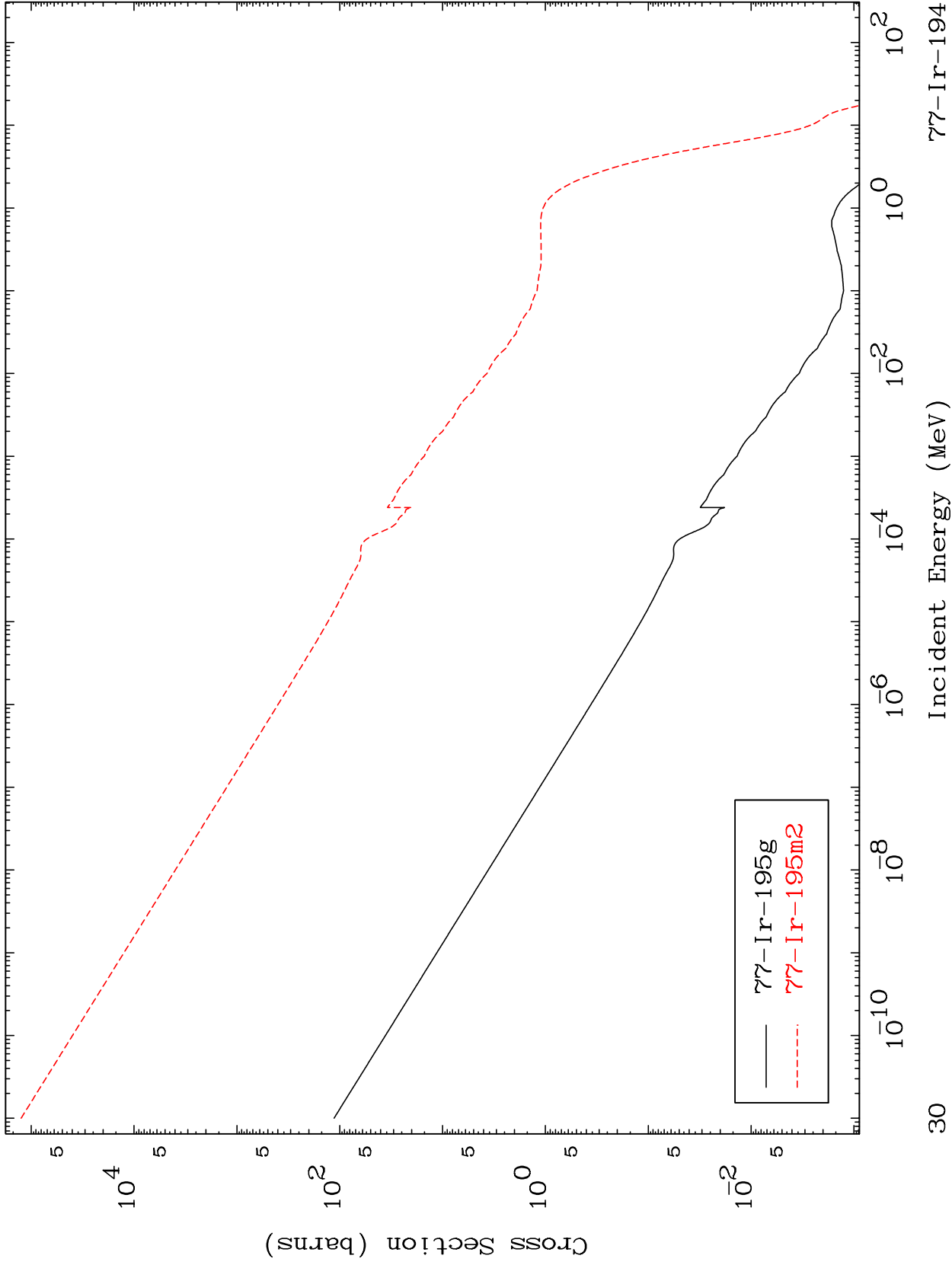
29

77-Ir-194

MAT 7735

77-Ir-194

(n,γ)
Radionuclide Production Cross Section



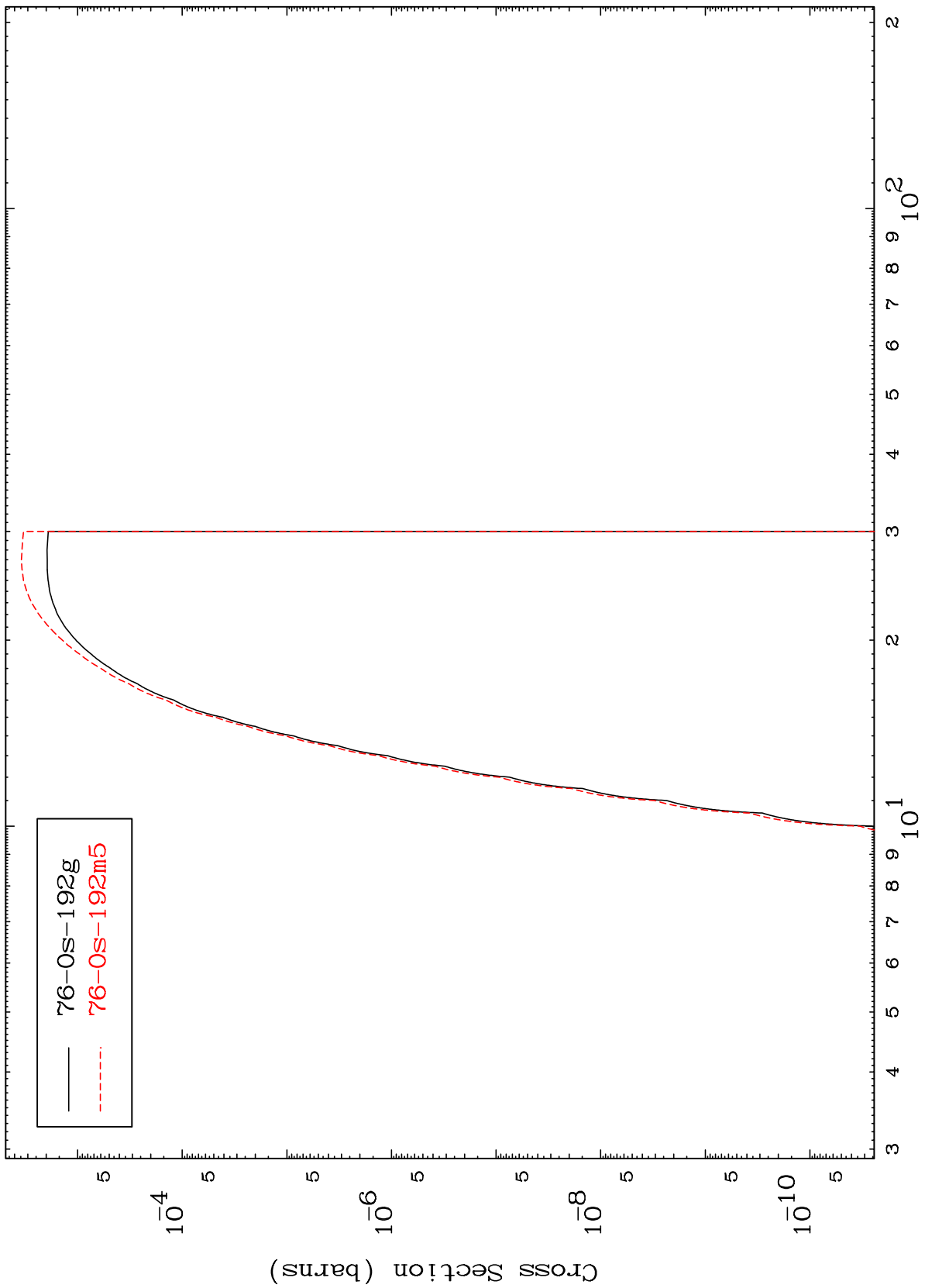
30

77-Ir-194

MAT 7735

77-Ir-194

(n, t)
Radionuclide Production Cross Section



31

Incident Energy (MeV)

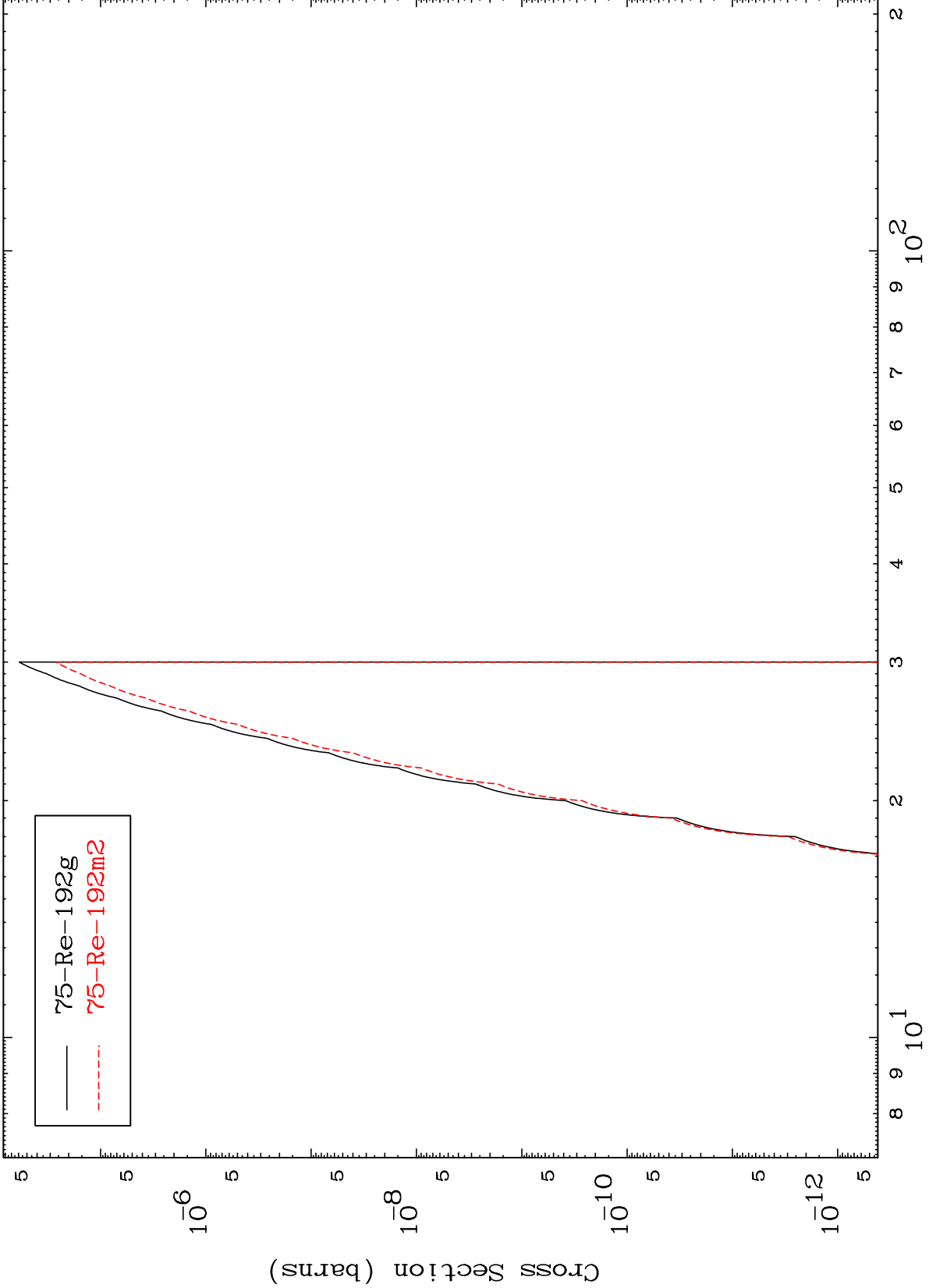
77-Ir-194

MAT 7735

(n,He-3)

77-Ir-194

Radionuclide Production Cross Section



32

Incident Energy (MeV)

77-Ir-194