

Program EVALPLOT  
(Version 2017-1)

by

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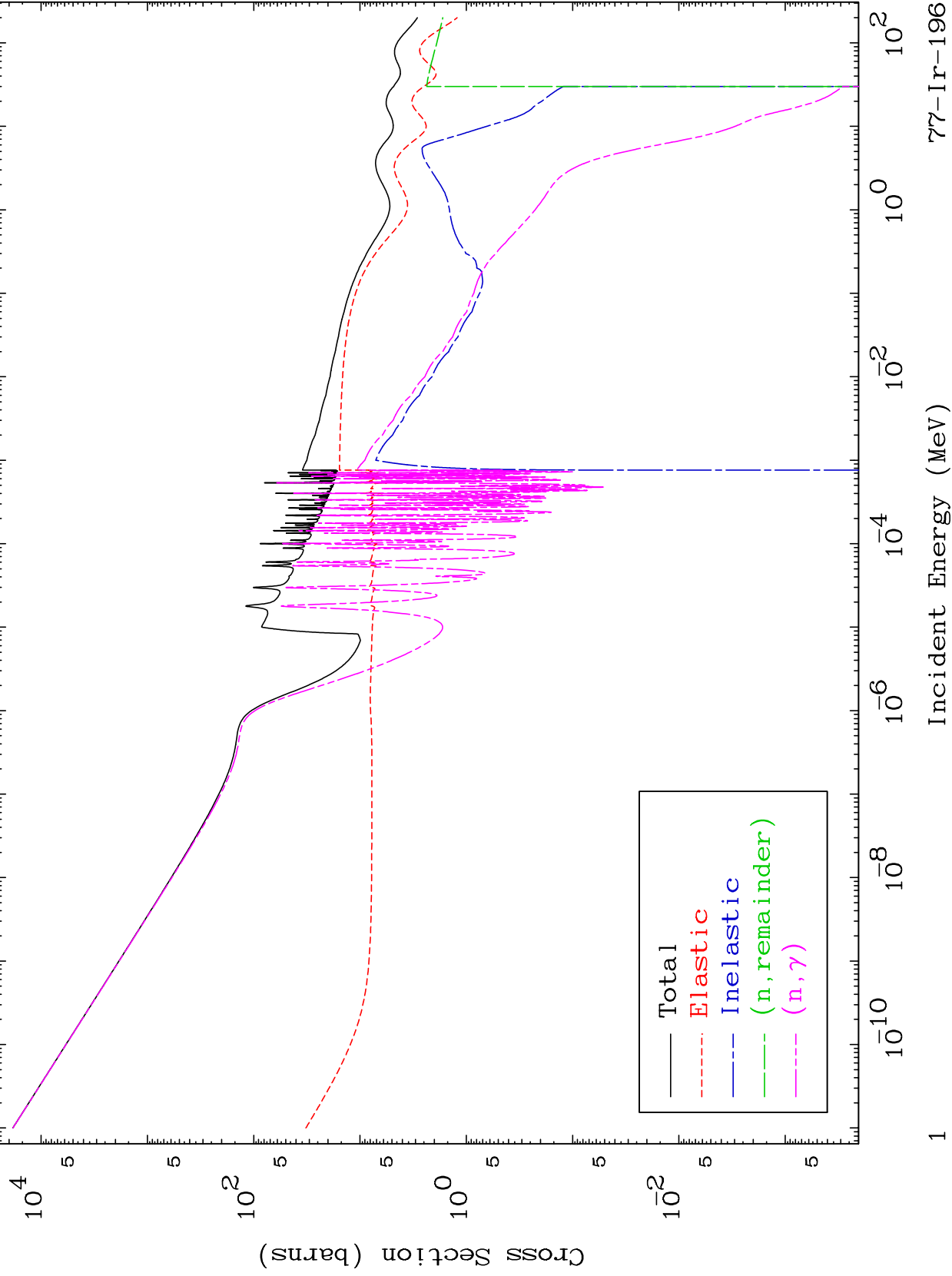
Press Mouse Button to Start

MAT 7741

Major

293 Kelvin Cross Sections

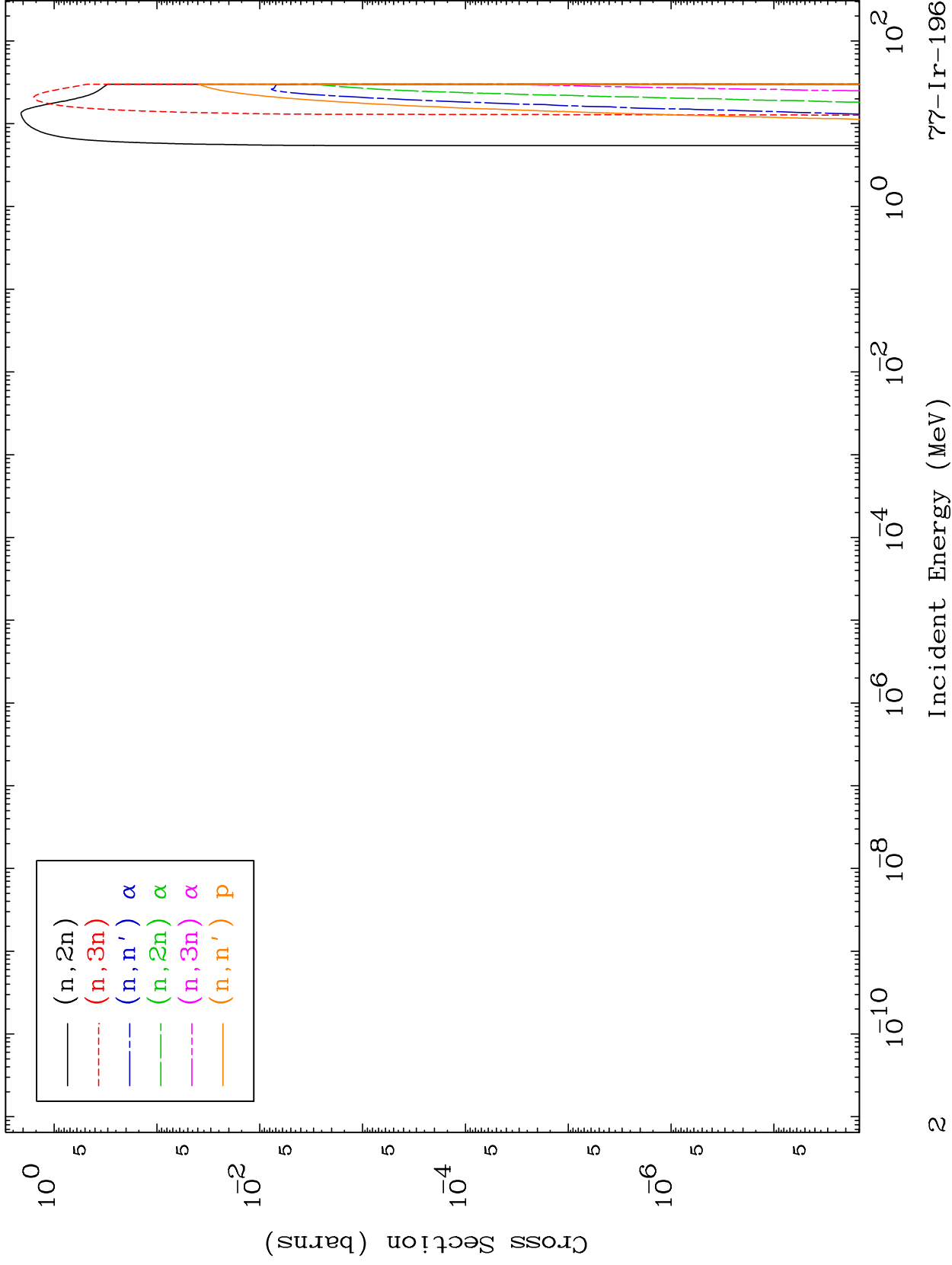
77-Ir-196



MAT 7741

Neutron Production  
293 Kelvin Cross Sections

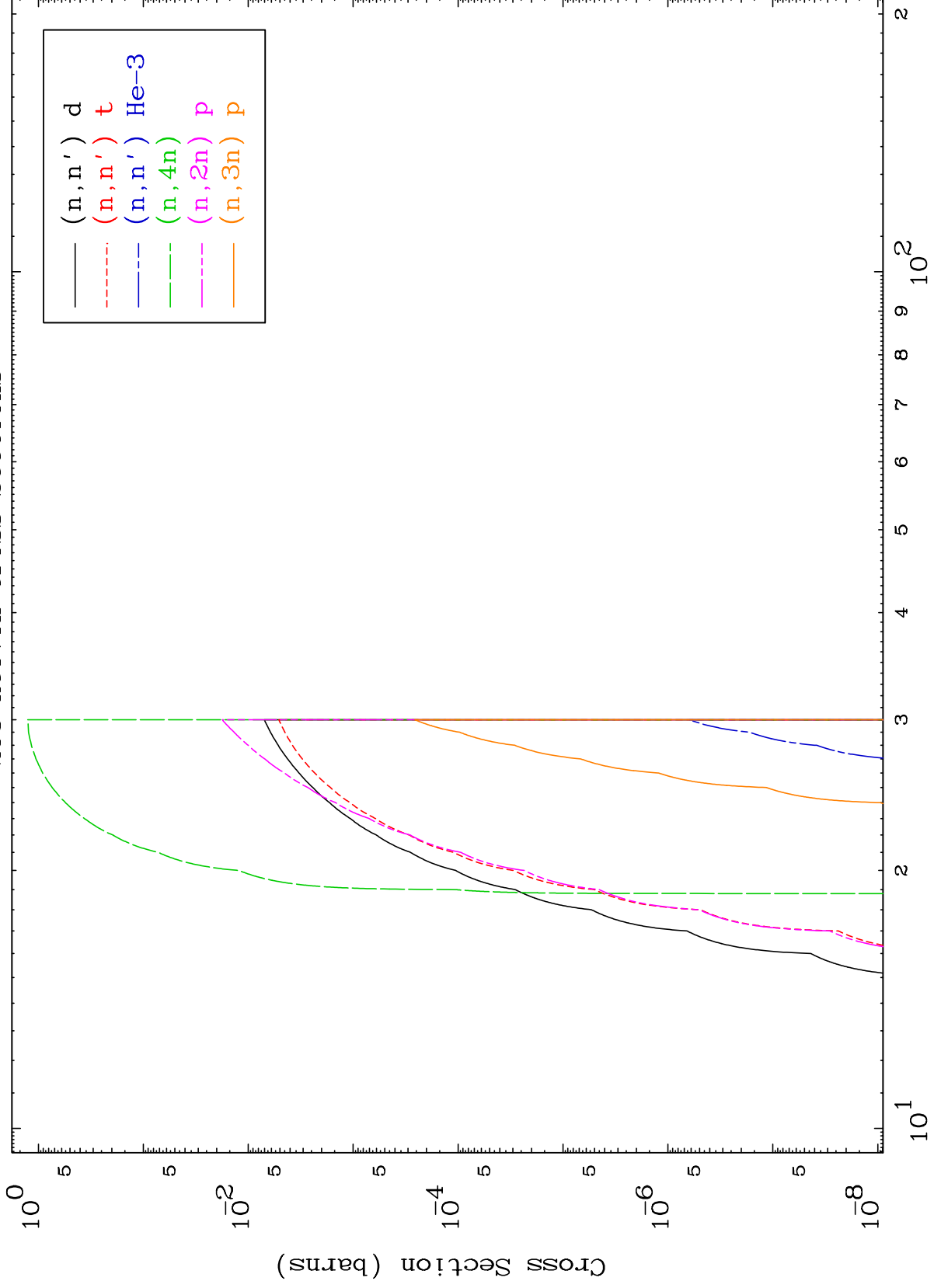
77-Ir-196



MAT 7741

Neutron Production  
293 Kelvin Cross Sections

77-Ir-196



77-Ir-196

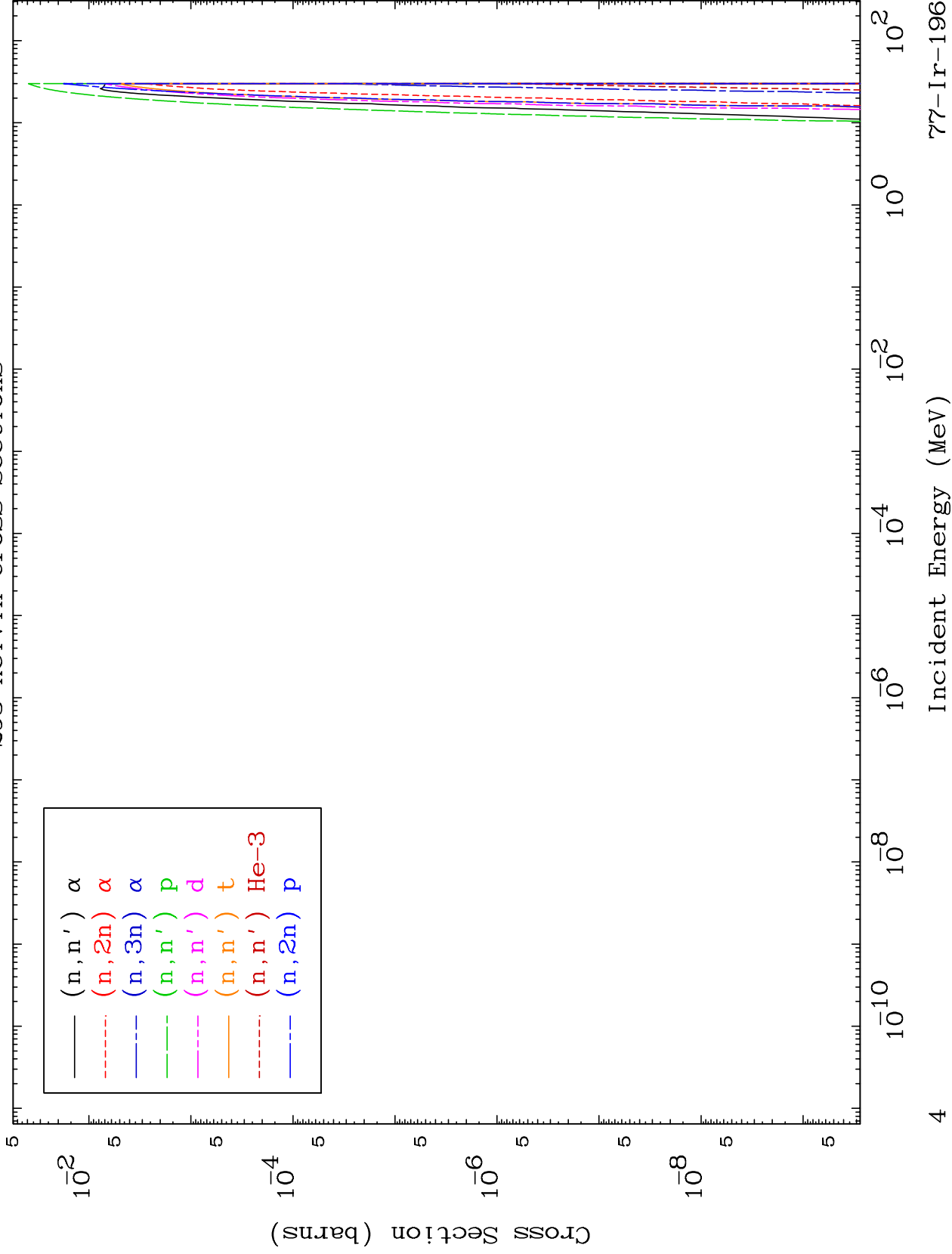
Incident Energy (MeV)

3

MAT 7741

Charged Particle  
293 Kelvin Cross Sections

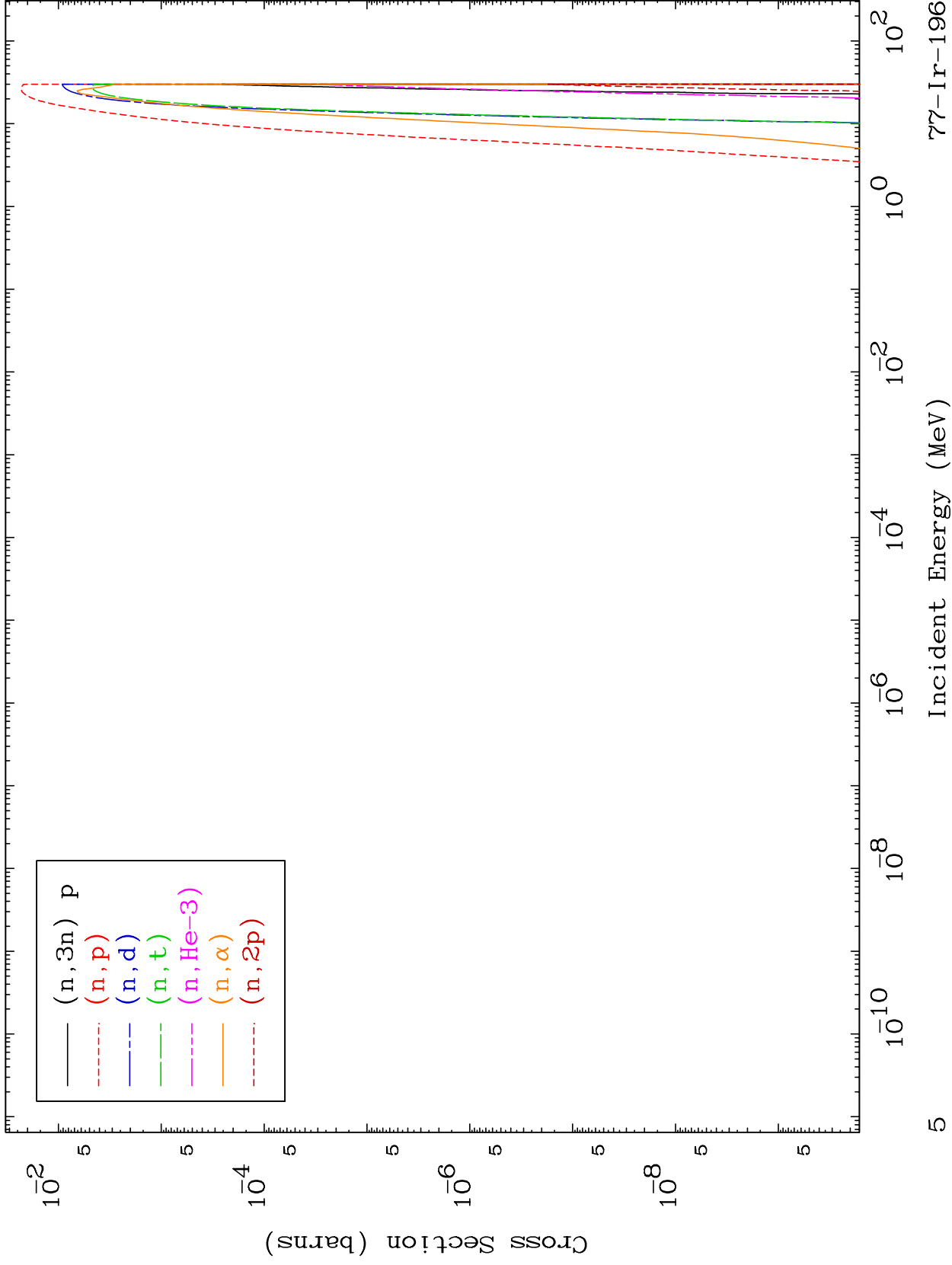
77-Ir-196



MAT 7741

Charged Particle  
293 Kelvin Cross Sections

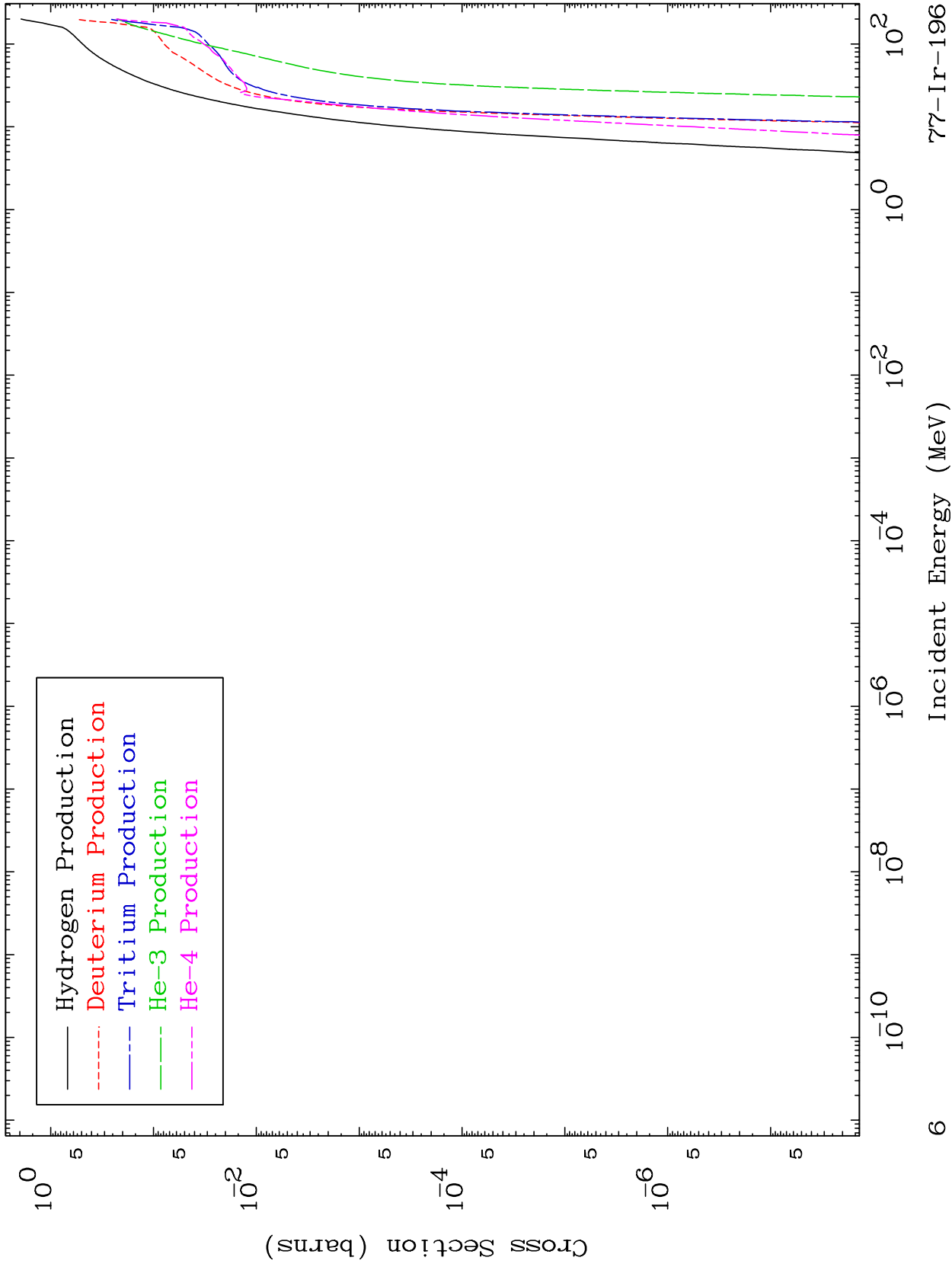
77-Ir-196



MAT 7741

Particle Production  
293 Kelvin Cross Sections

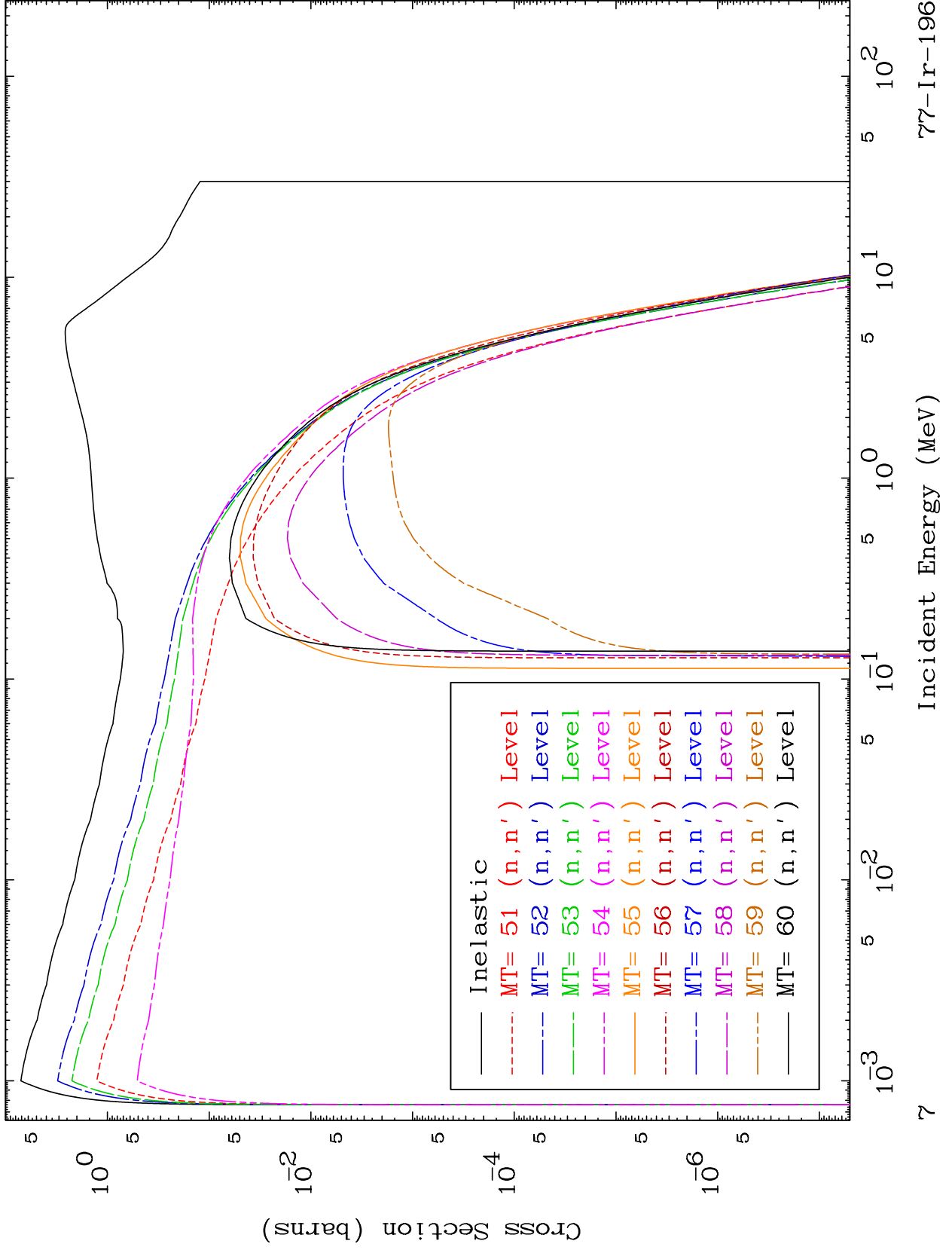
77-Ir-196



MAT 7741

(n,n') Level  
293 Kelvin Cross Sections

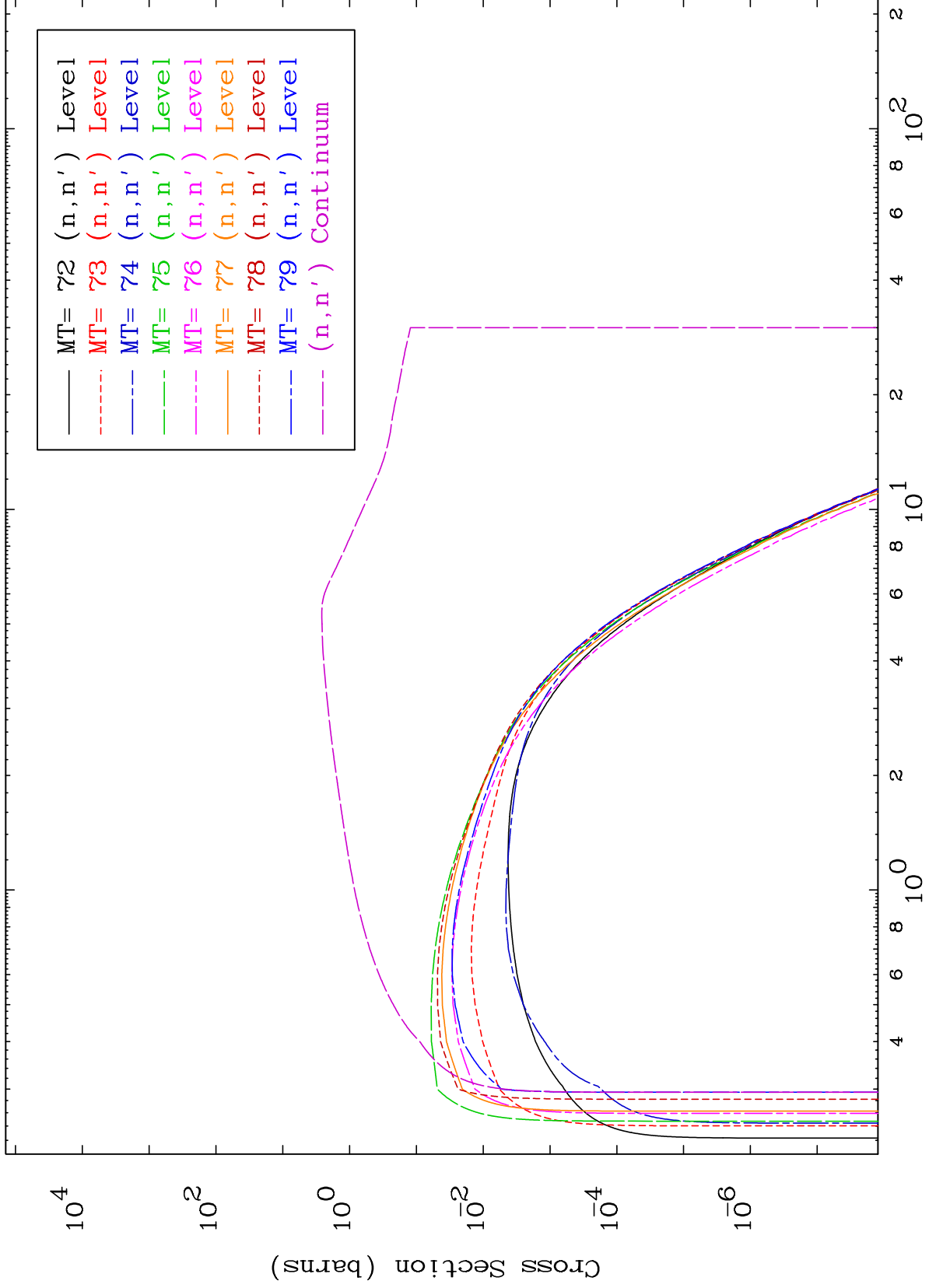
77-Ir-196







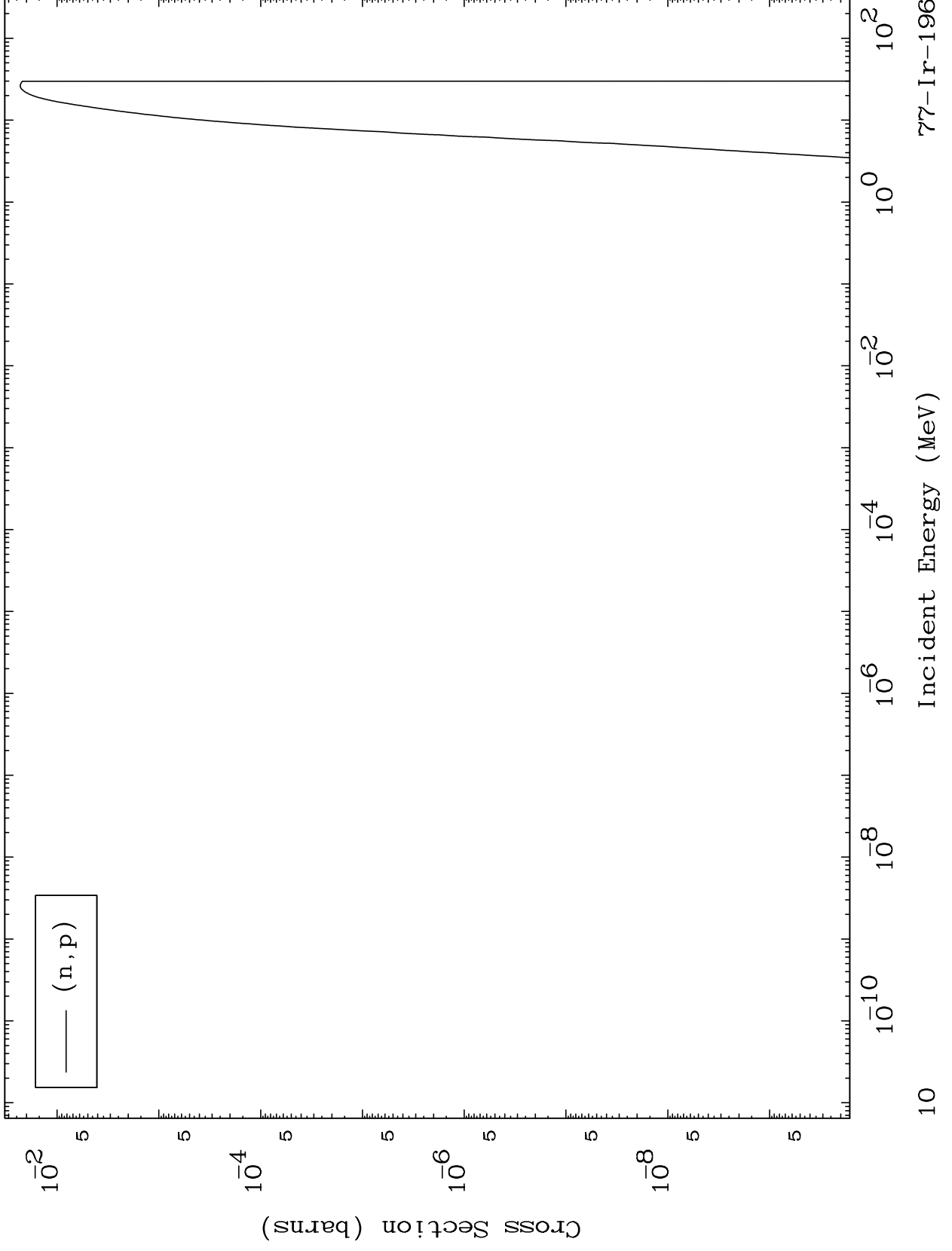
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

77-Ir-196



10

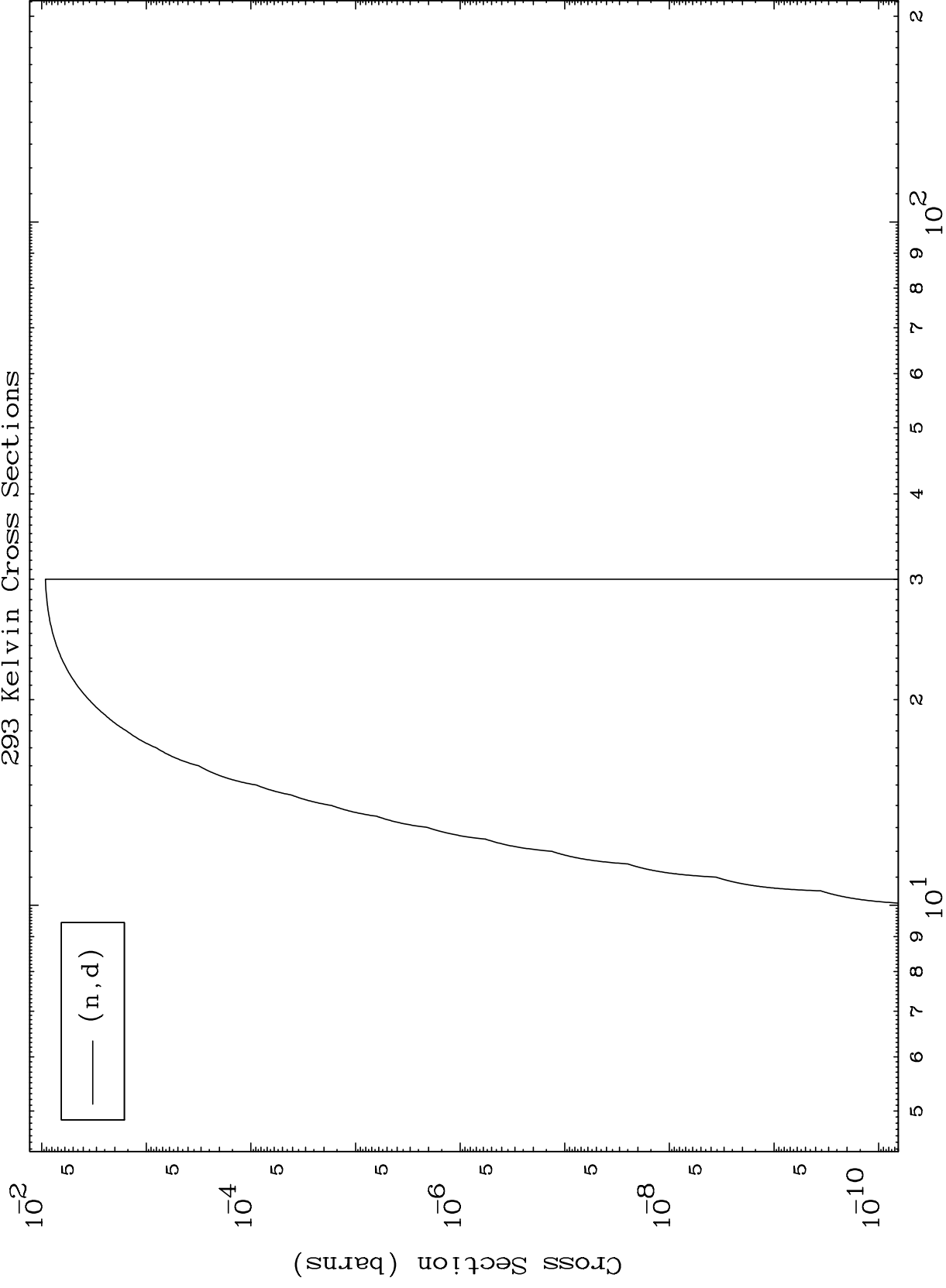
Incident Energy (MeV)

77-Ir-196

MAT 7741

(n,d) Levels  
293 Kelvin Cross Sections

77-Ir-196



11

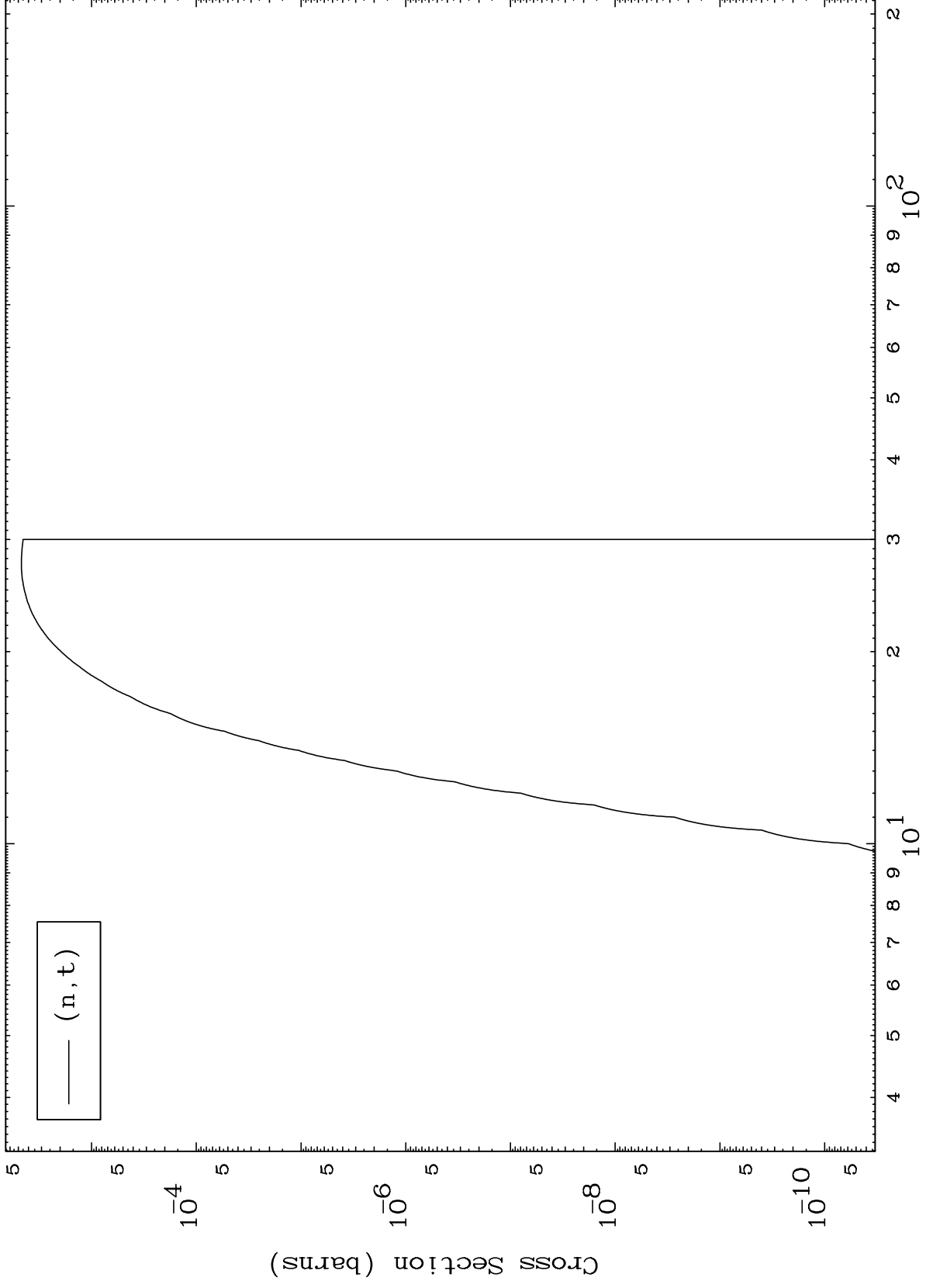
Incident Energy (MeV)

77-Ir-196

MAT 7741

(n,t) Levels  
293 Kelvin Cross Sections

77-Ir-196



12

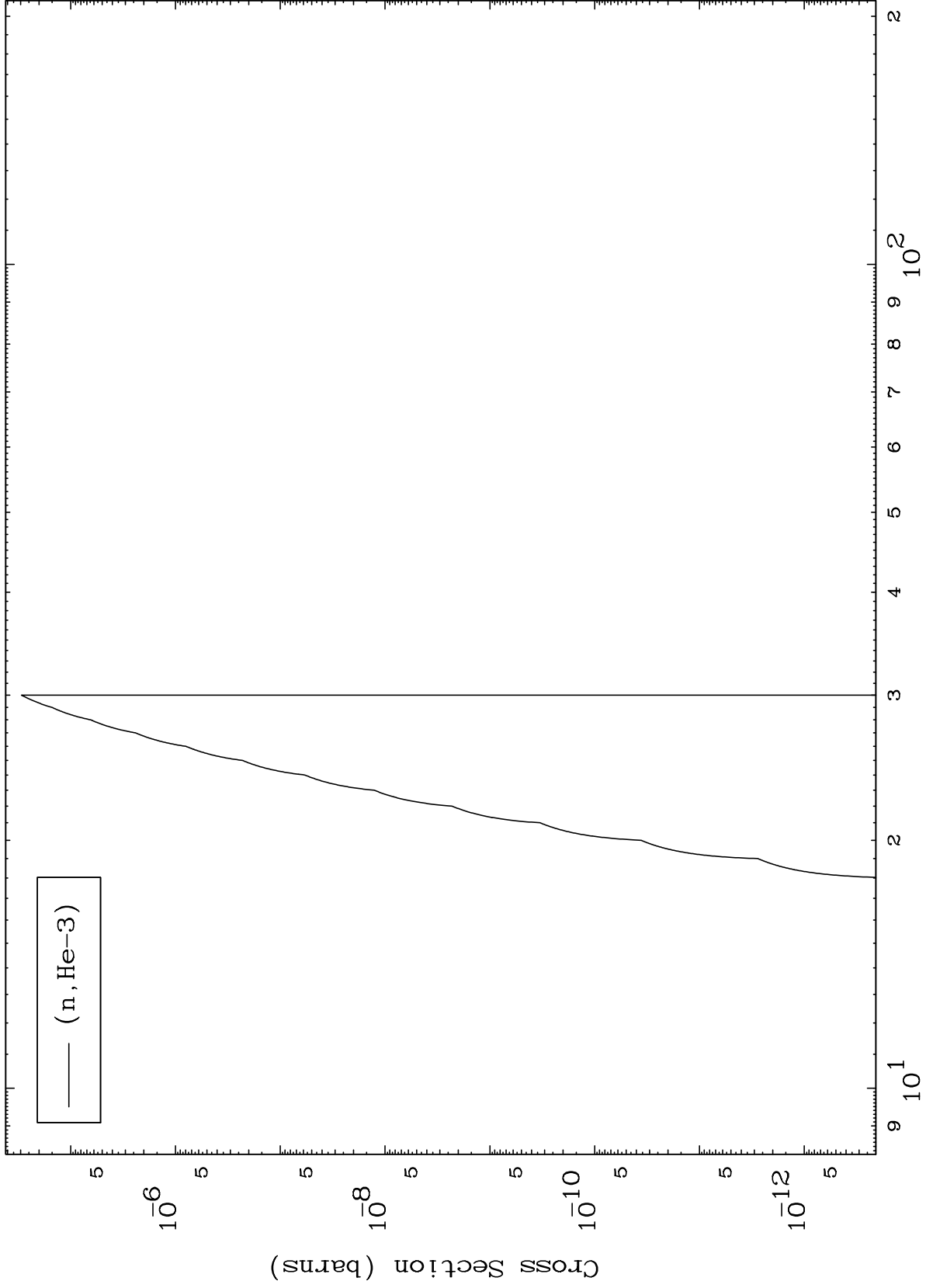
Incident Energy (MeV)

77-Ir-196

MAT 7741

(n,He3) Levels  
293 Kelvin Cross Sections

77-Ir-196



13

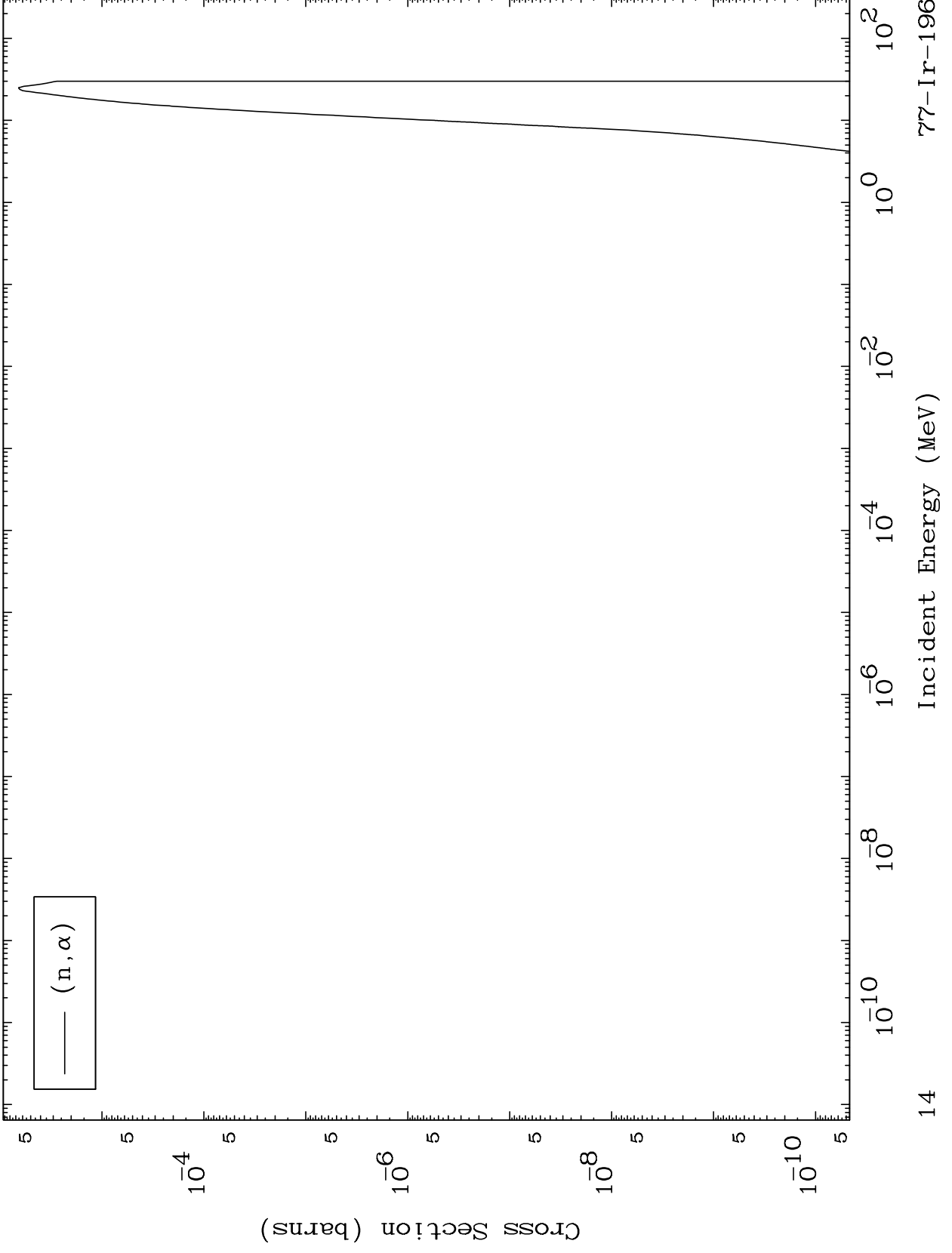
Incident Energy (MeV)

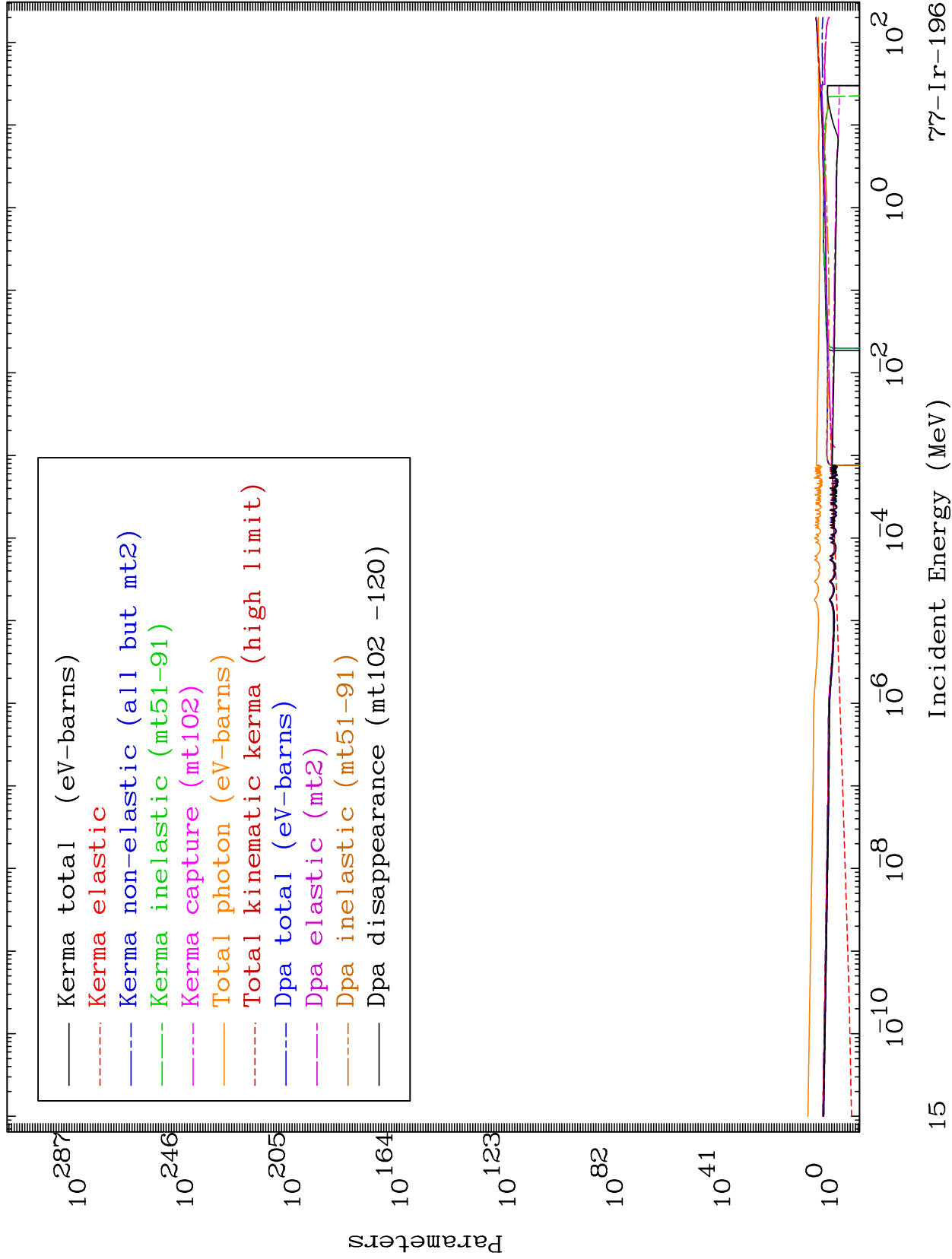
77-Ir-196

MAT 7741

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

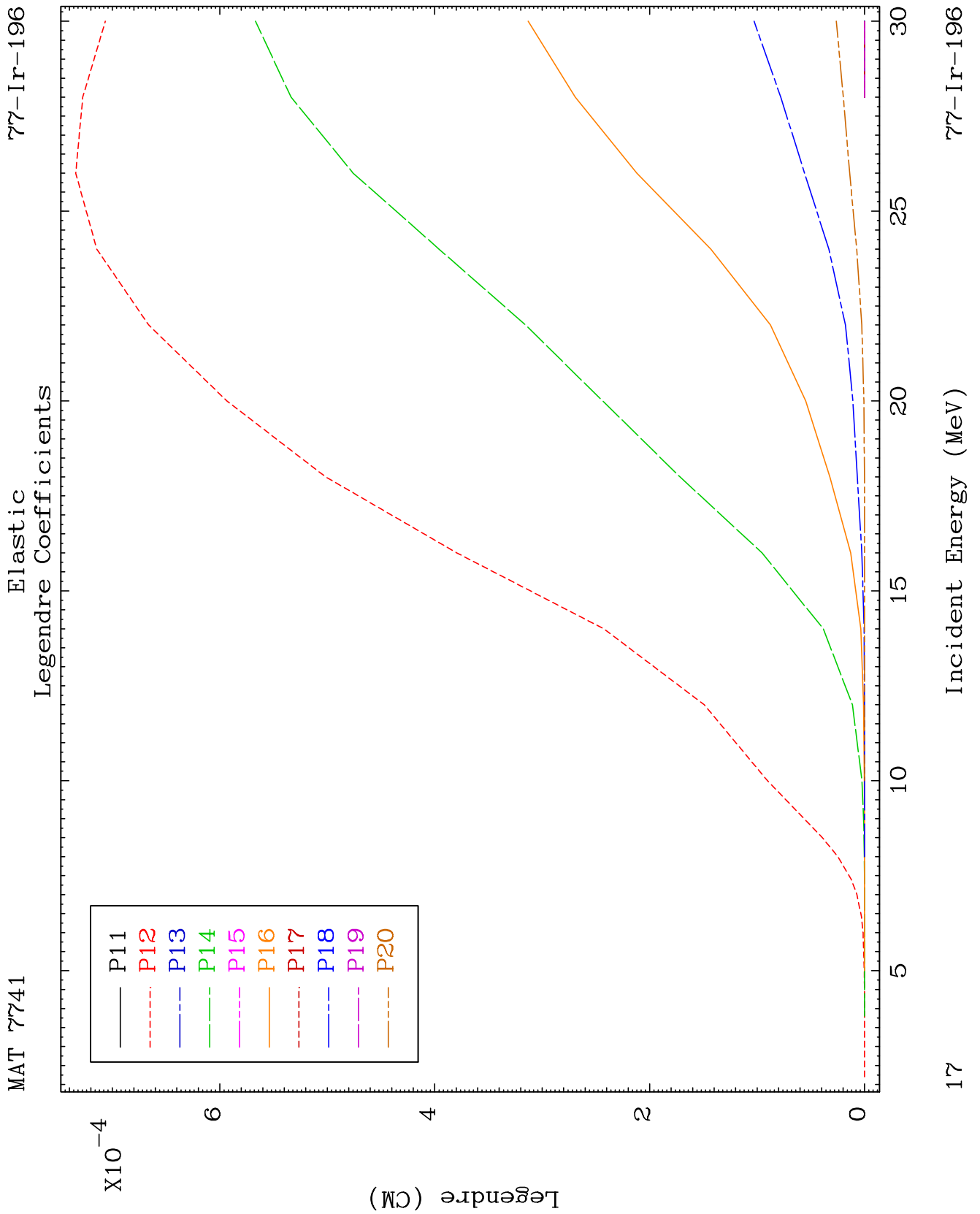
77-Ir-196







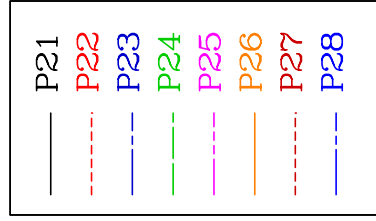




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Elastic  
Legendre Coefficients

77-Ir-196



$\times 10^{-6}$

Legendre (CM)



18

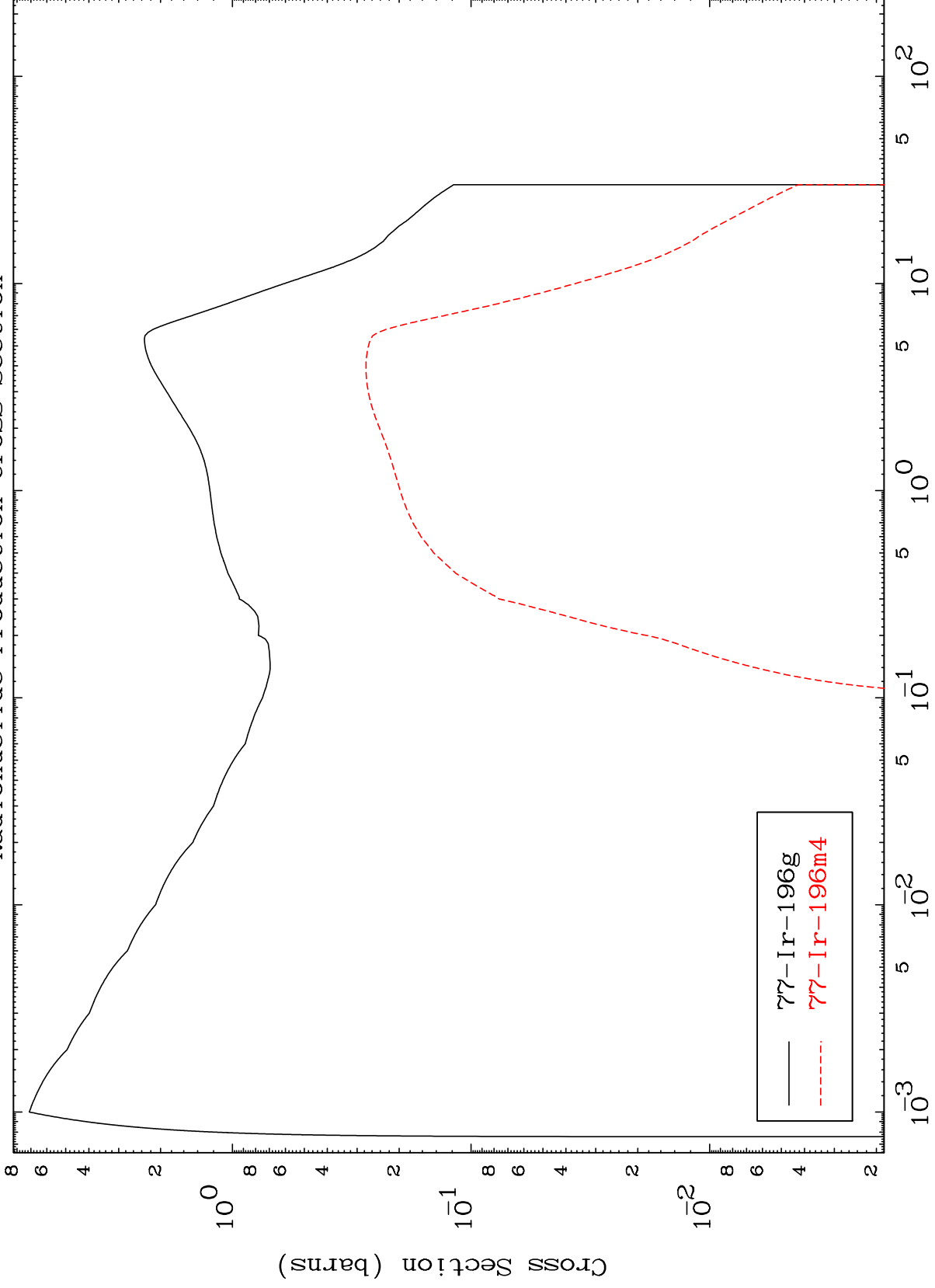
Incident Energy (MeV)

77-Ir-196

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<sup>77</sup>Ir-196

Inelastic  
Radionuclide Production Cross Section



<sup>77</sup>Ir-196

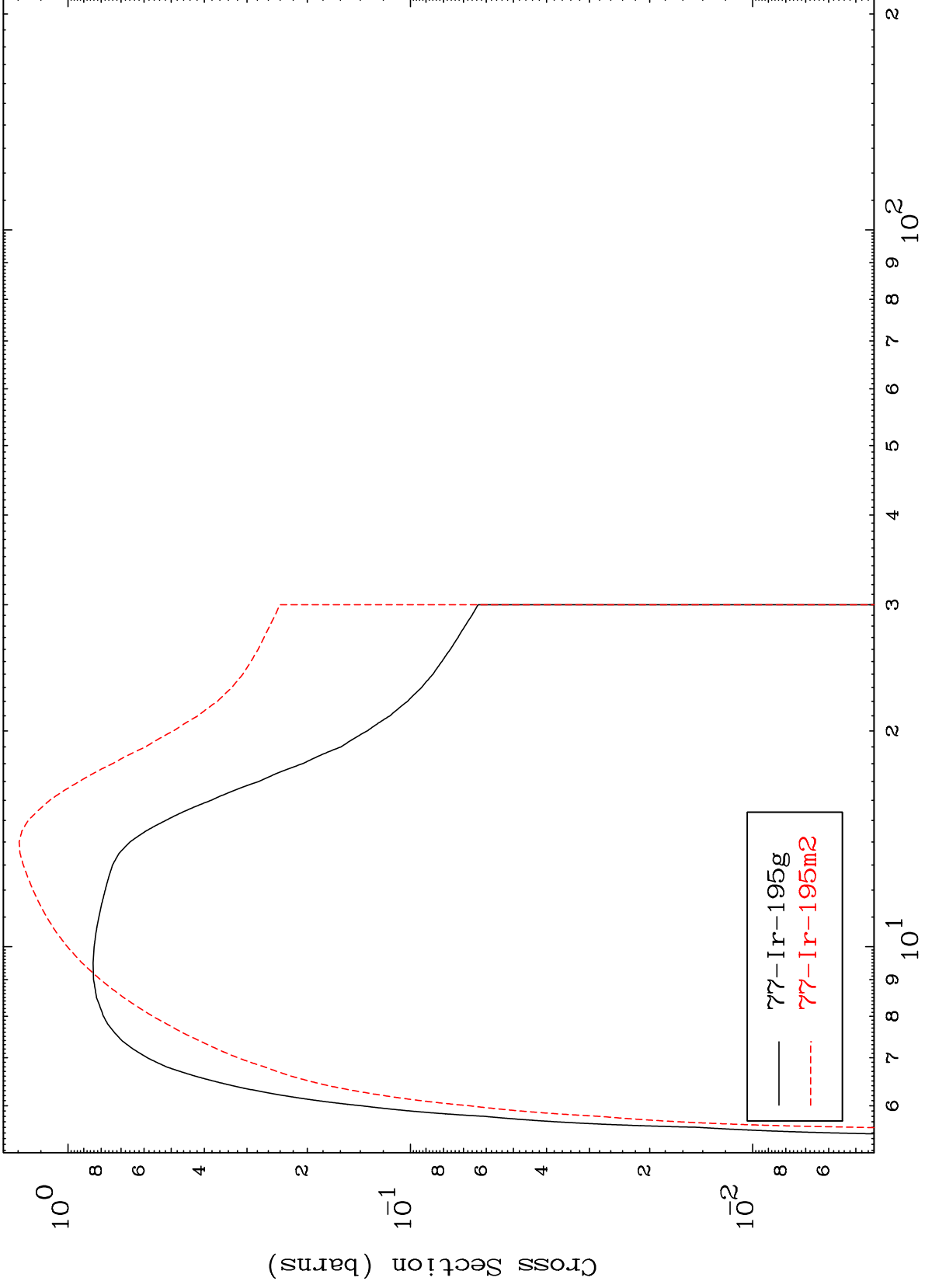
Incident Energy (MeV)

19

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77-Ir-196

(n,2n)  
Radionuclide Production Cross Section



20

Incident Energy (MeV)

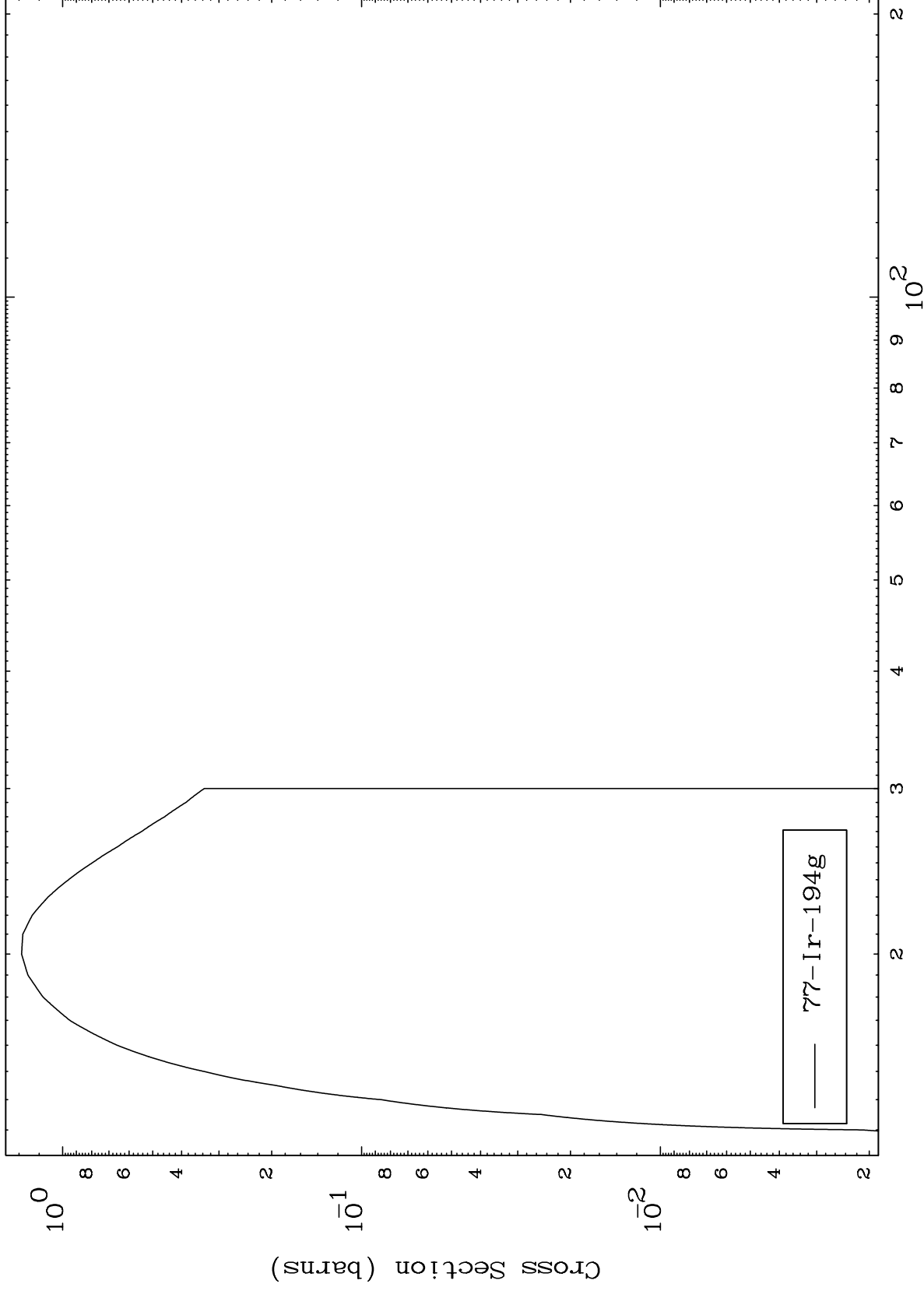
77-Ir-196

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(n,3n)

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



21

Incident Energy (MeV)

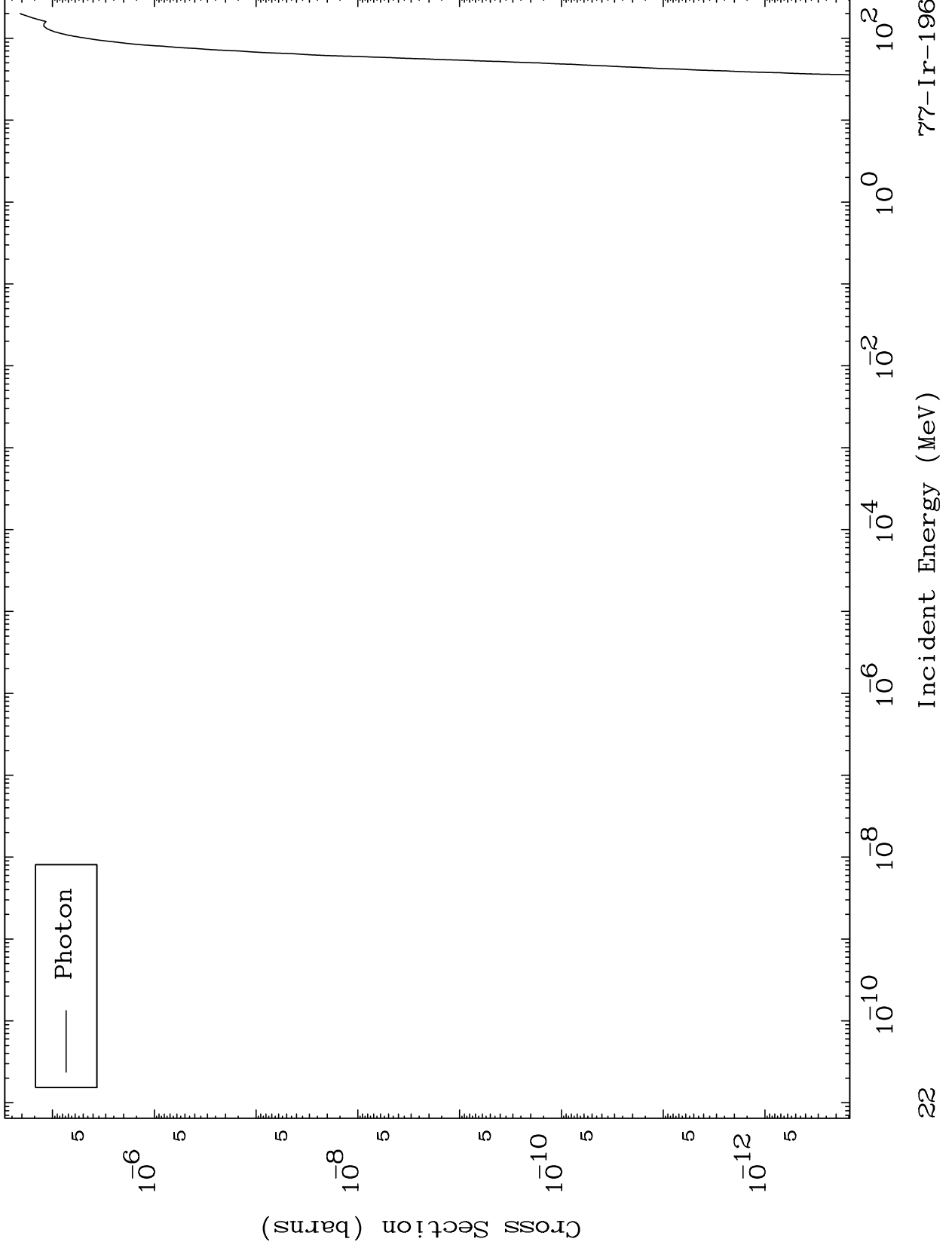
<sup>77</sup>Ir-196

MAT 7741

Fission

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

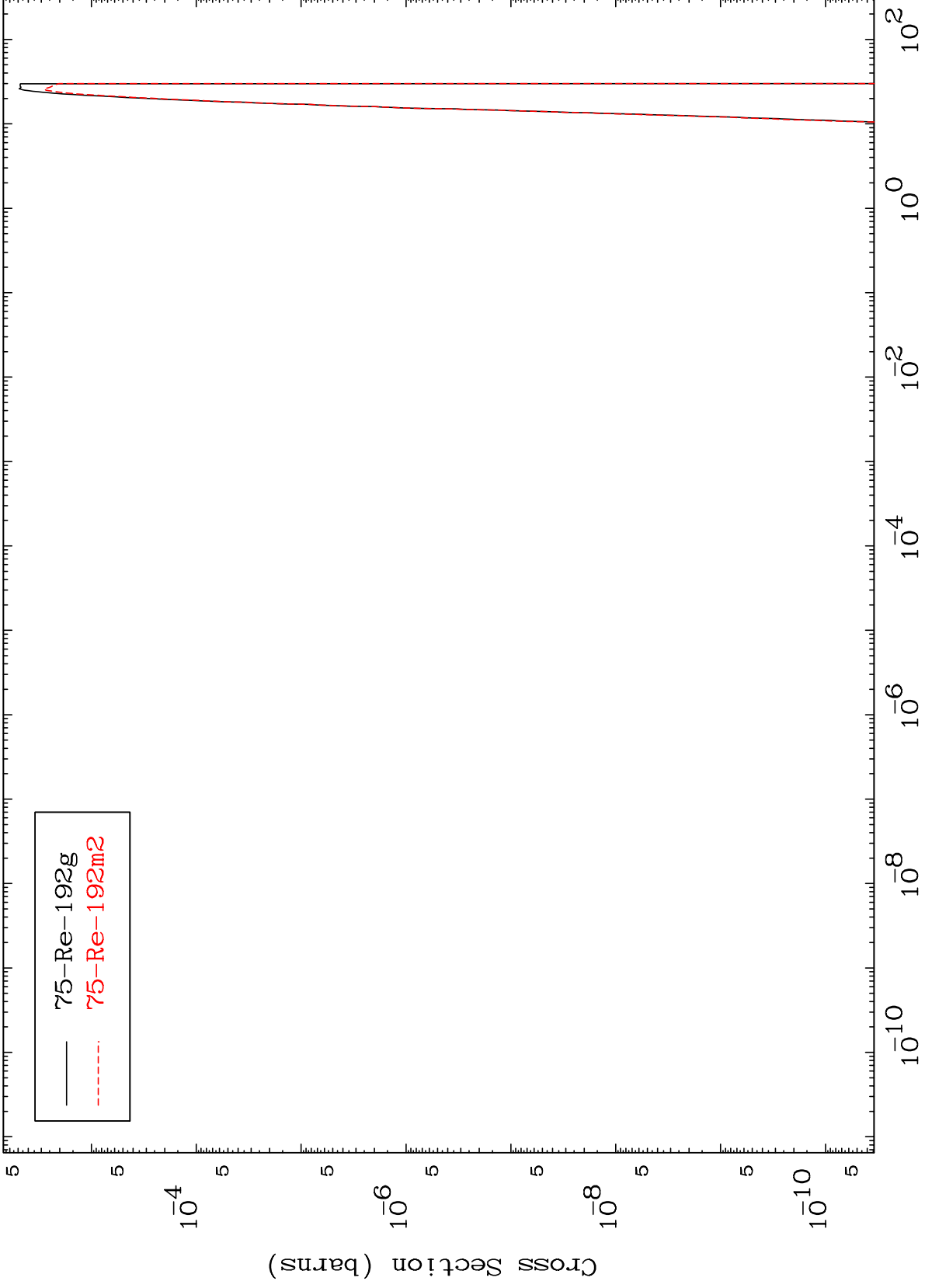


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$(n, n') \alpha$

$^{77}\text{Ir-196}$

Radionuclide Production Cross Section



23

Incident Energy (MeV)

$^{77}\text{Ir-196}$

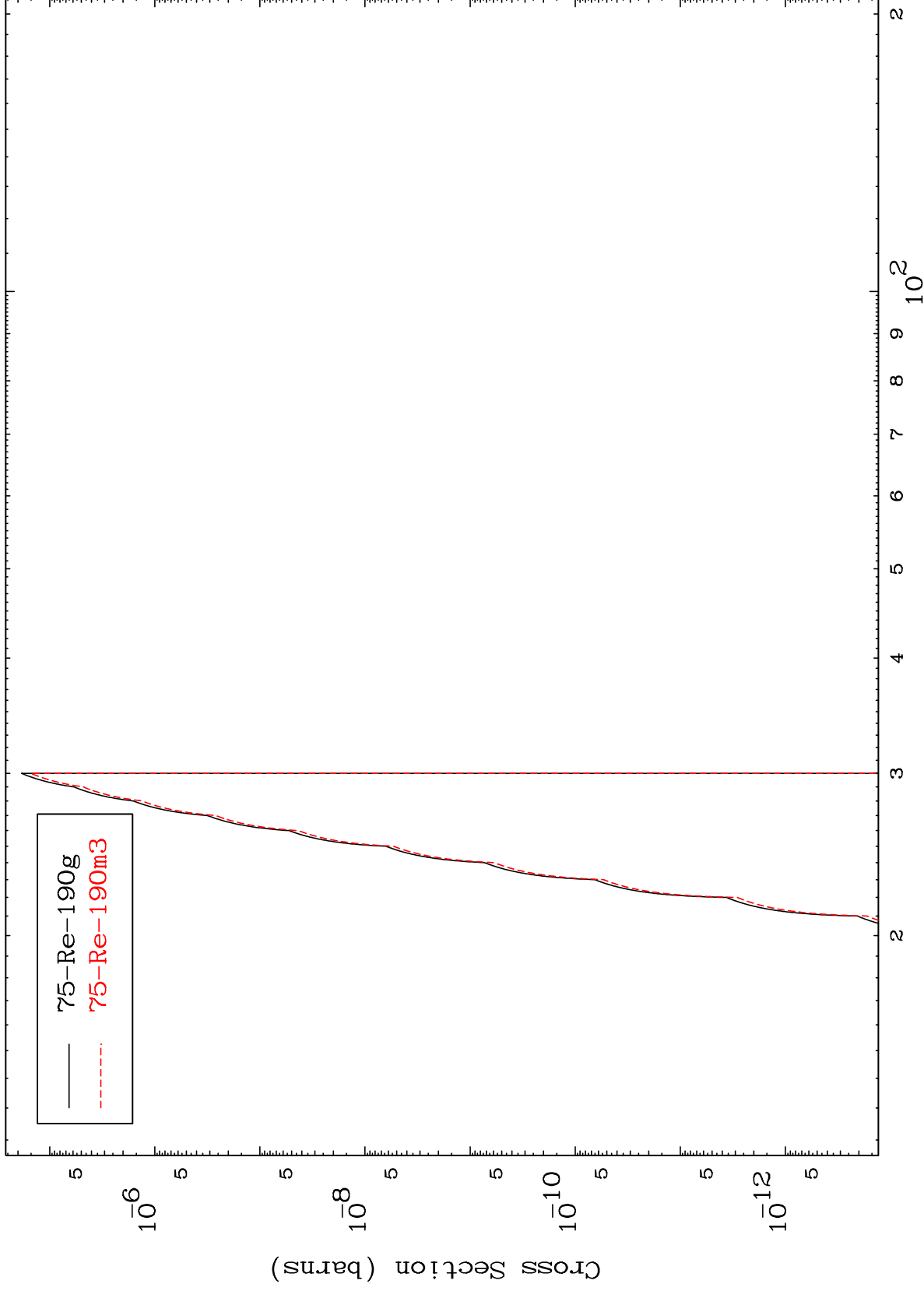


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(n,3n)  $\alpha$

77-Ir-196

Radionuclide Production Cross Section



24

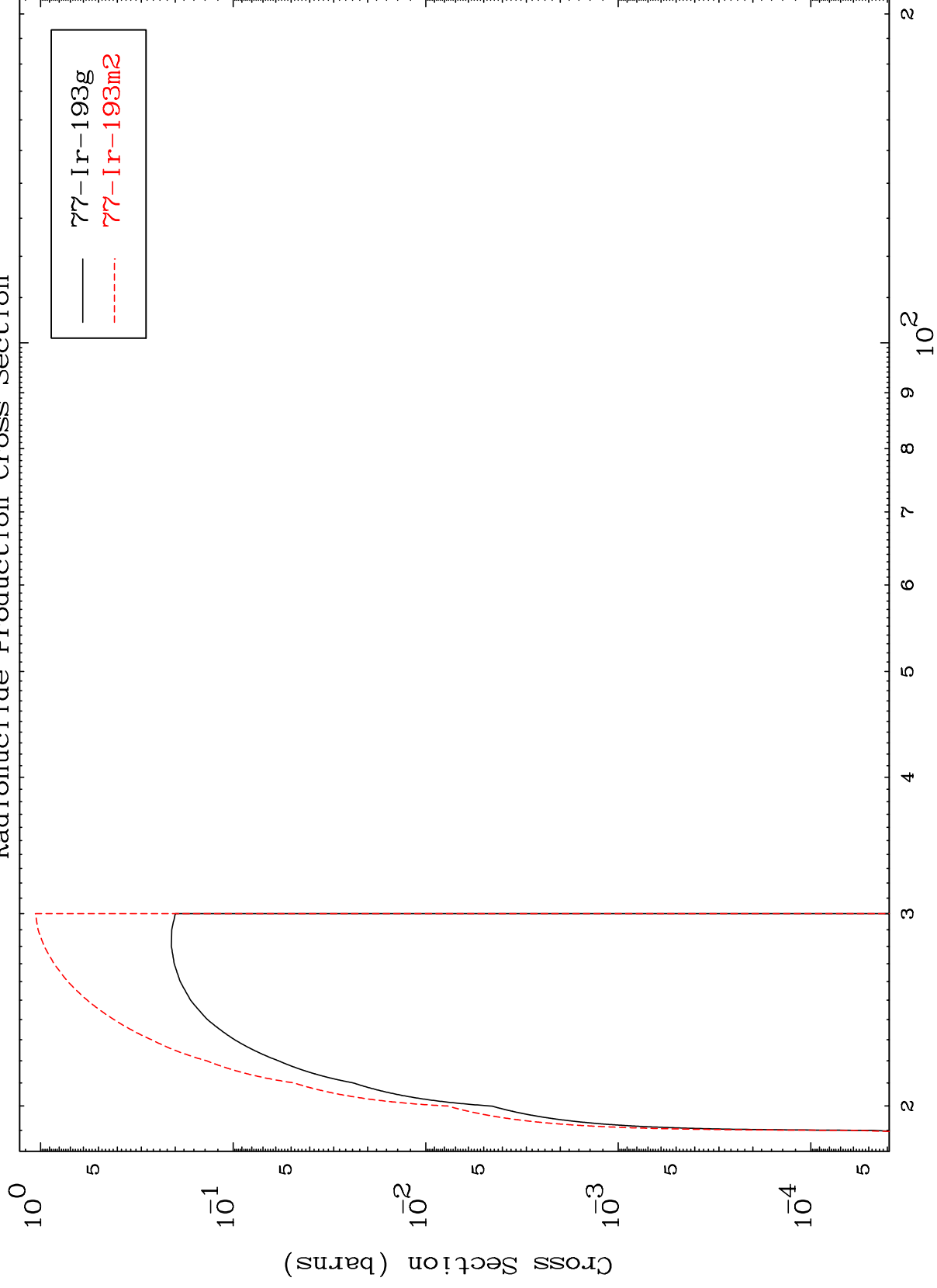
Incident Energy (MeV)

77-Ir-196

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77-Ir-196

(n,4n)  
Radionuclide Production Cross Section



25

Incident Energy (MeV)

77-Ir-196

MAT 7741

77-Ir-196

(n,γ)  
Radionuclide Production Cross Section

