

Program EVALPLOT  
(Version 2017-1)

by

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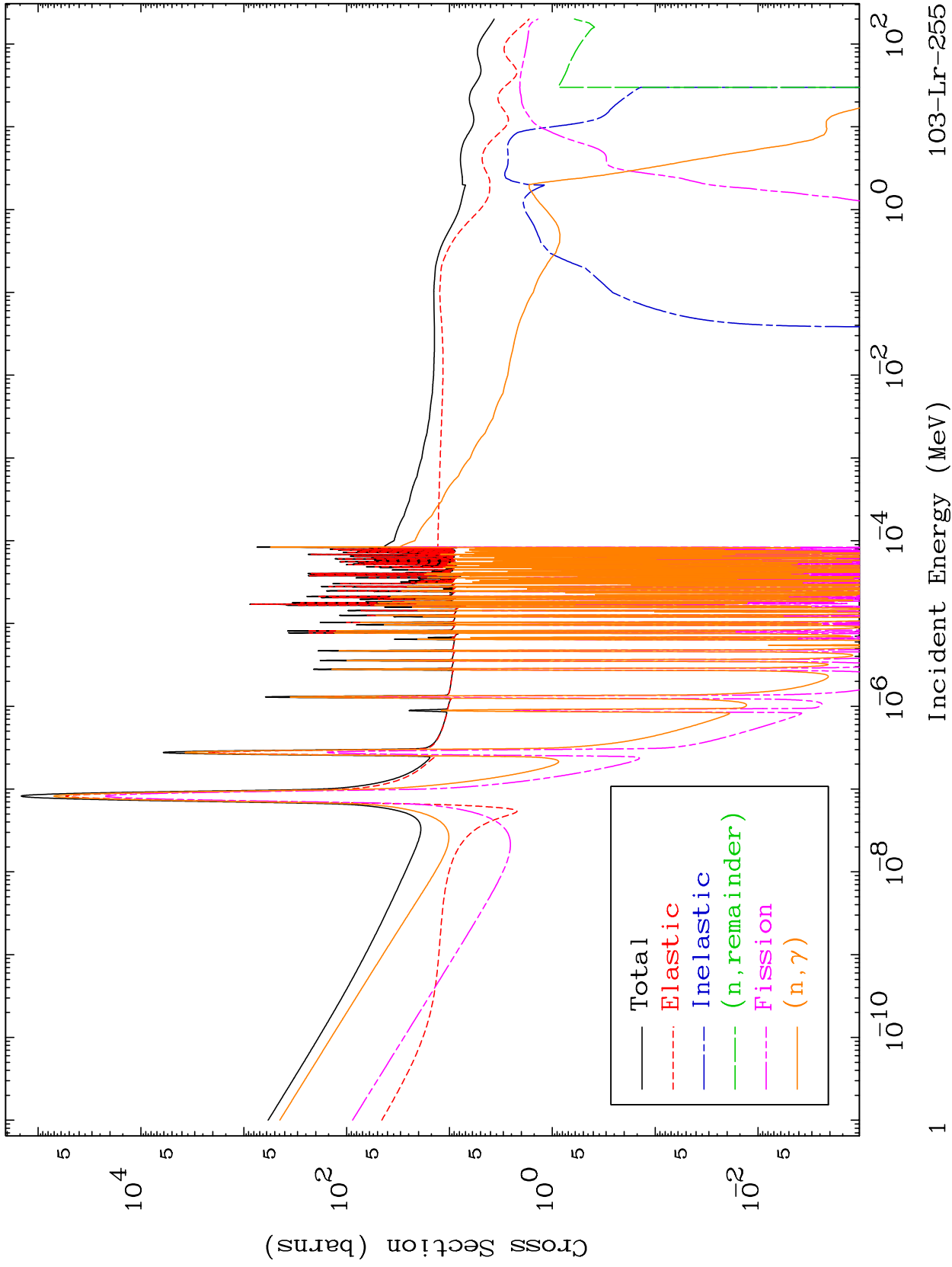
Press Mouse Button to Start

MAT 9984

Major

293 Kelvin Cross Sections

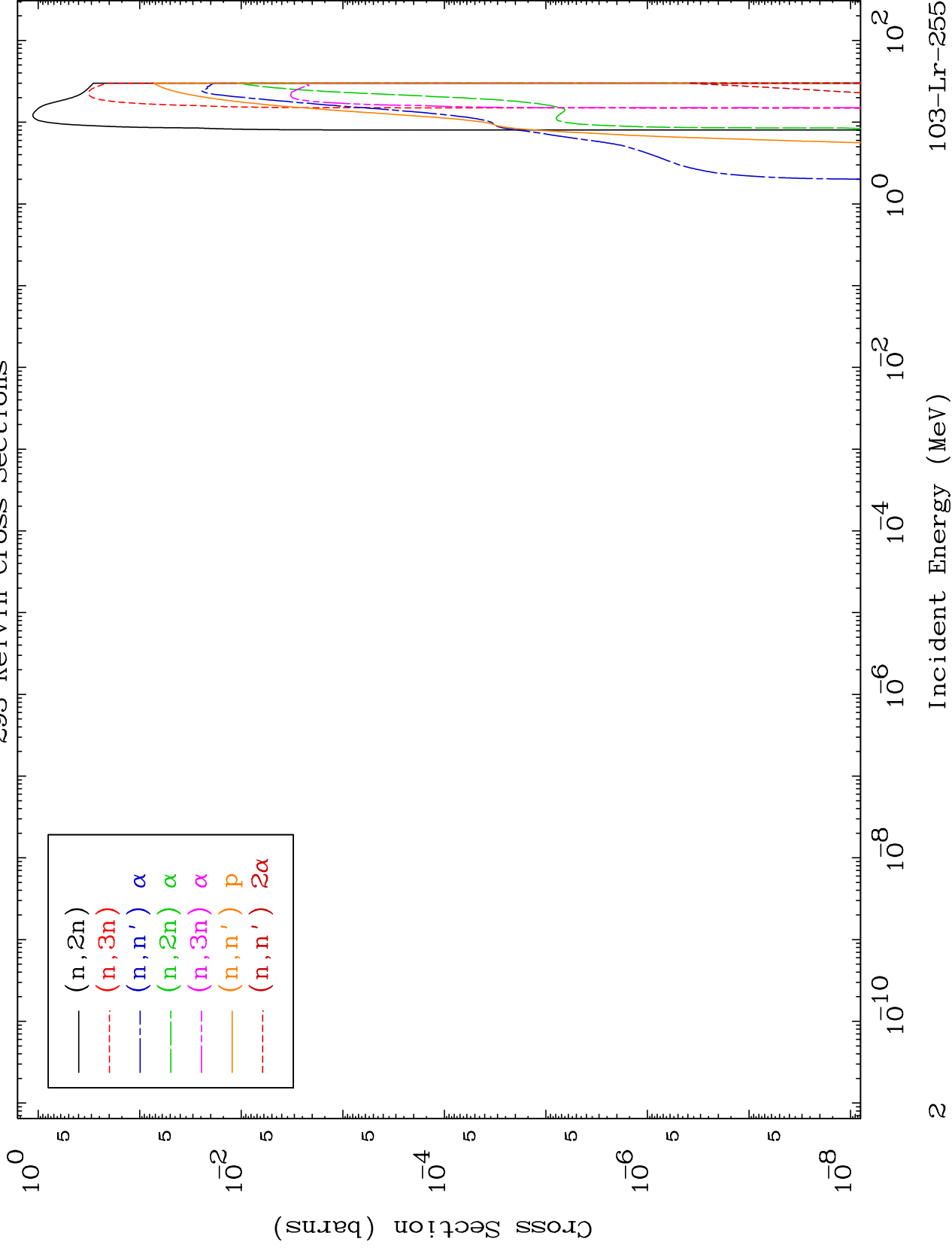
103-Lr-255



MAT 9984

Neutron Production  
293 Kelvin Cross Sections

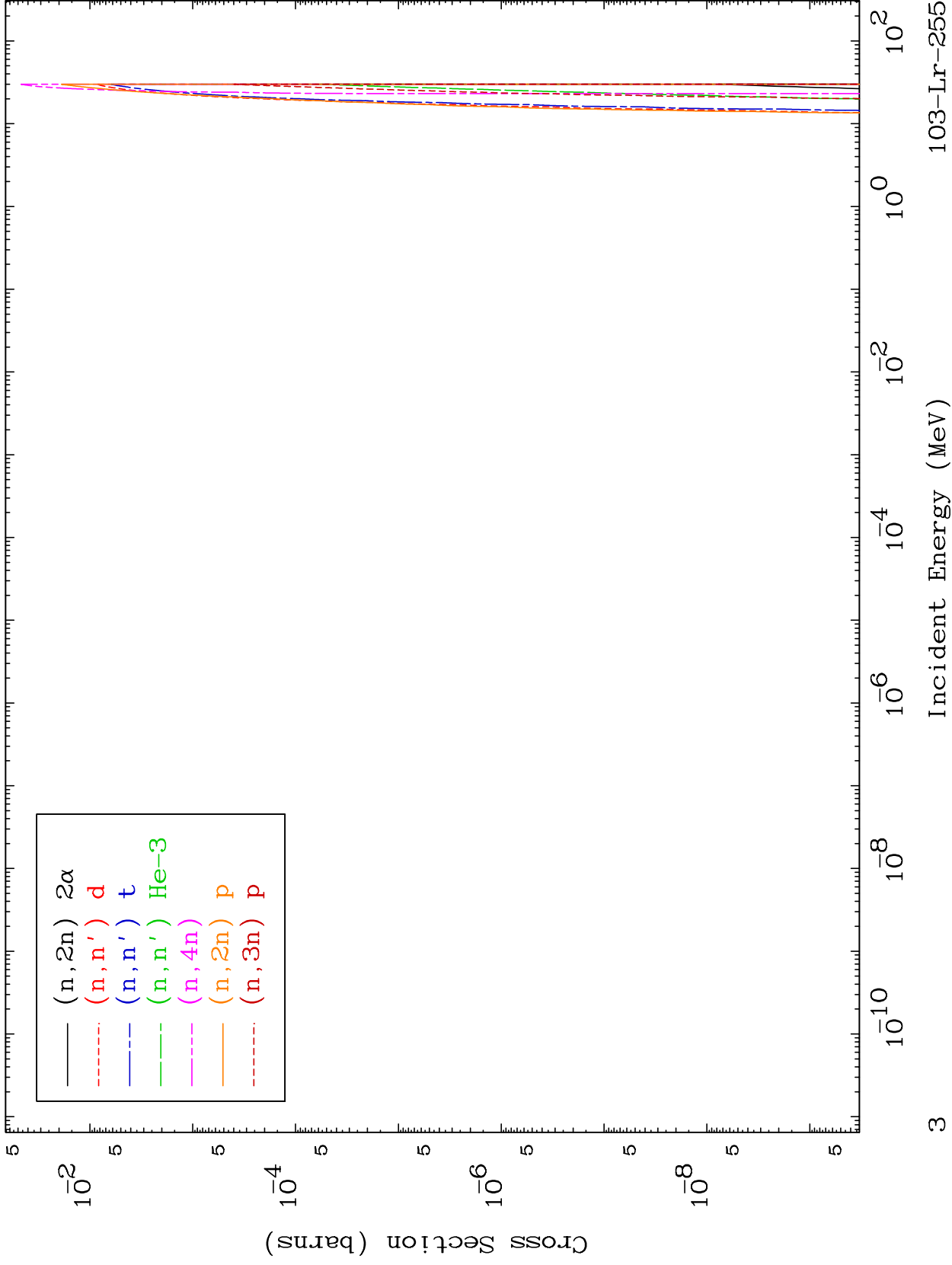
103-Lr-255



MAT 9984

Neutron Production  
293 Kelvin Cross Sections

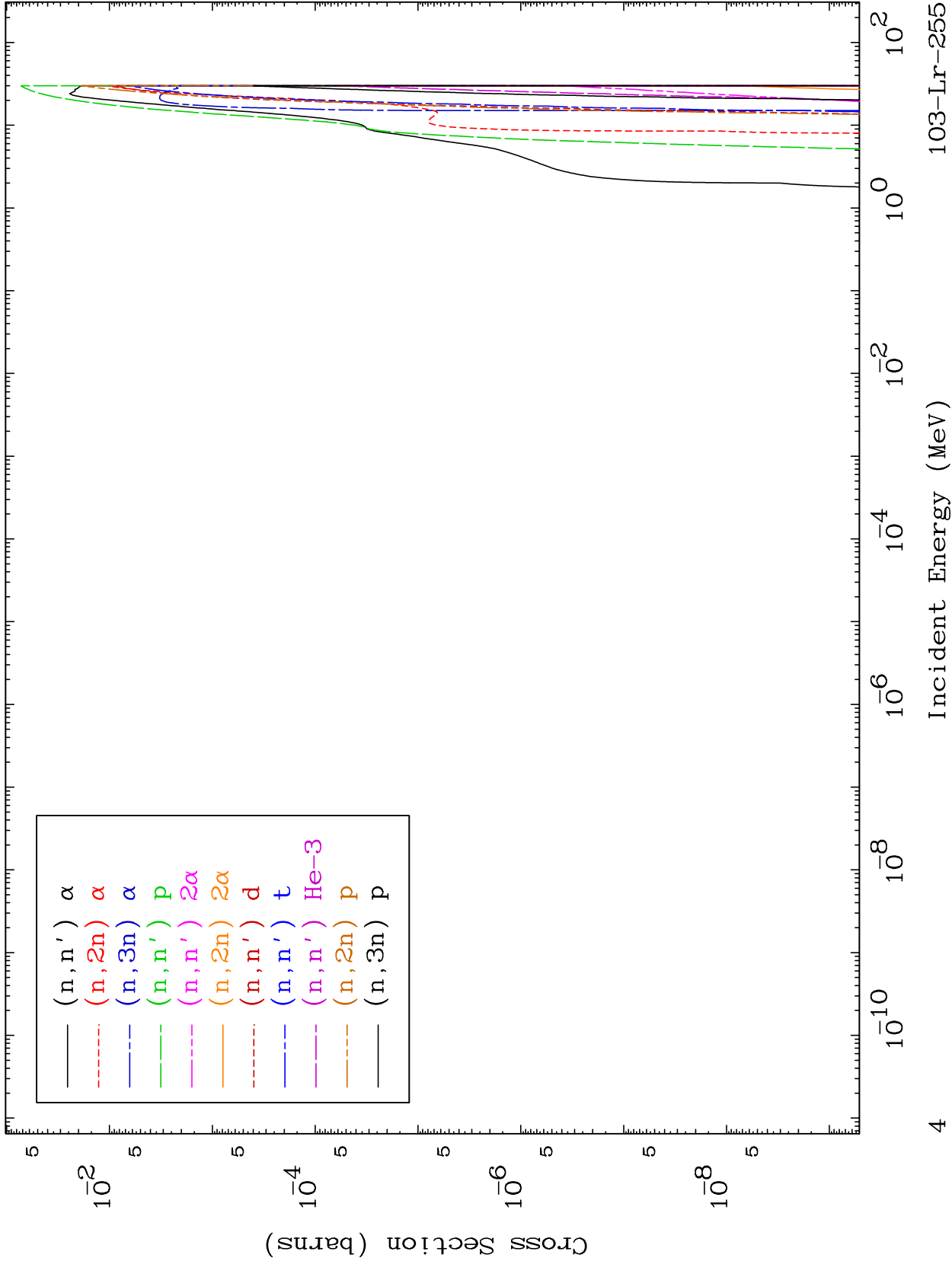
103-Lr-255



MAT 9984

Charged Particle  
293 Kelvin Cross Sections

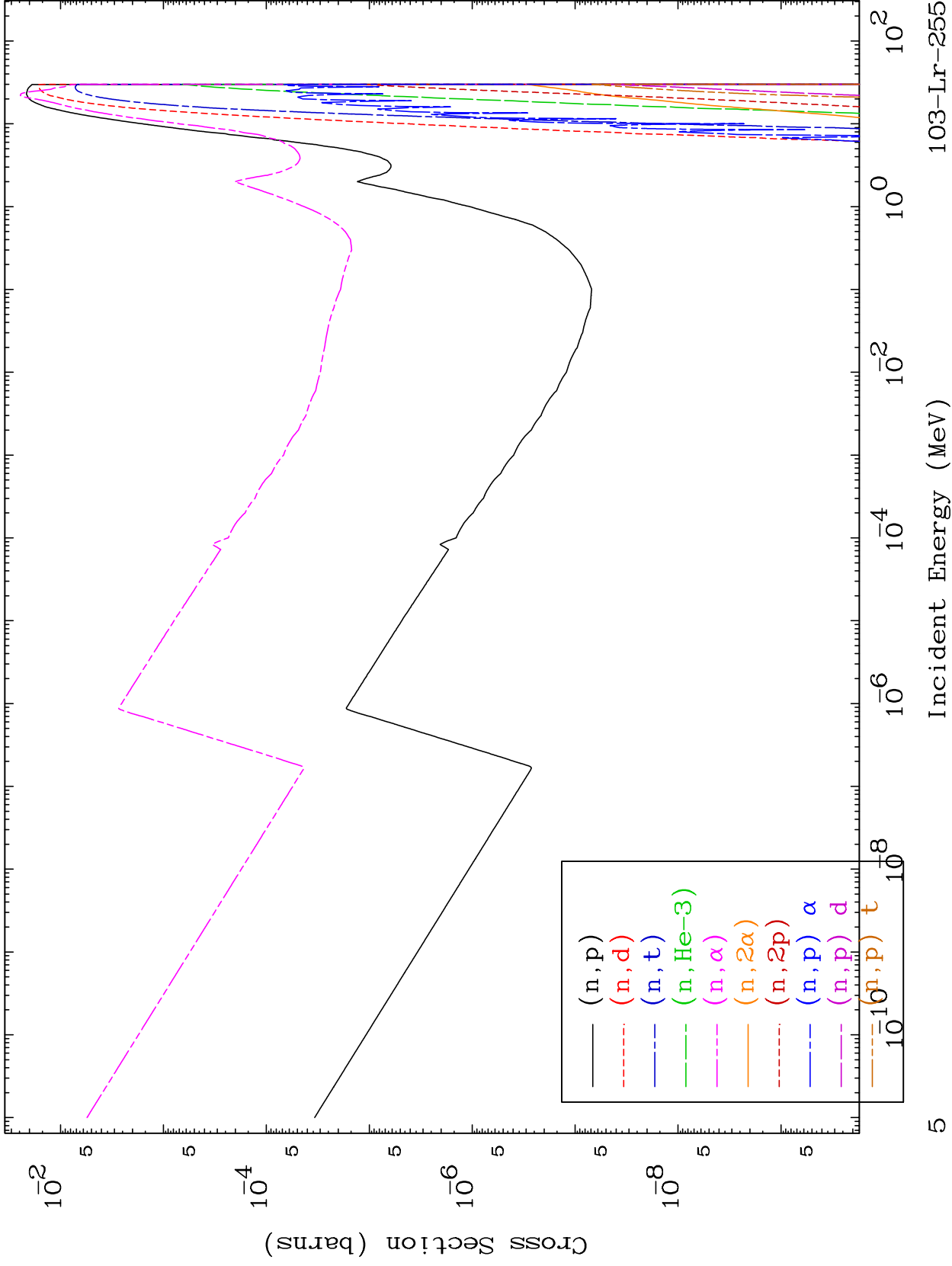
103-Lr-255



MAT 9984

Charged Particle  
293 Kelvin Cross Sections

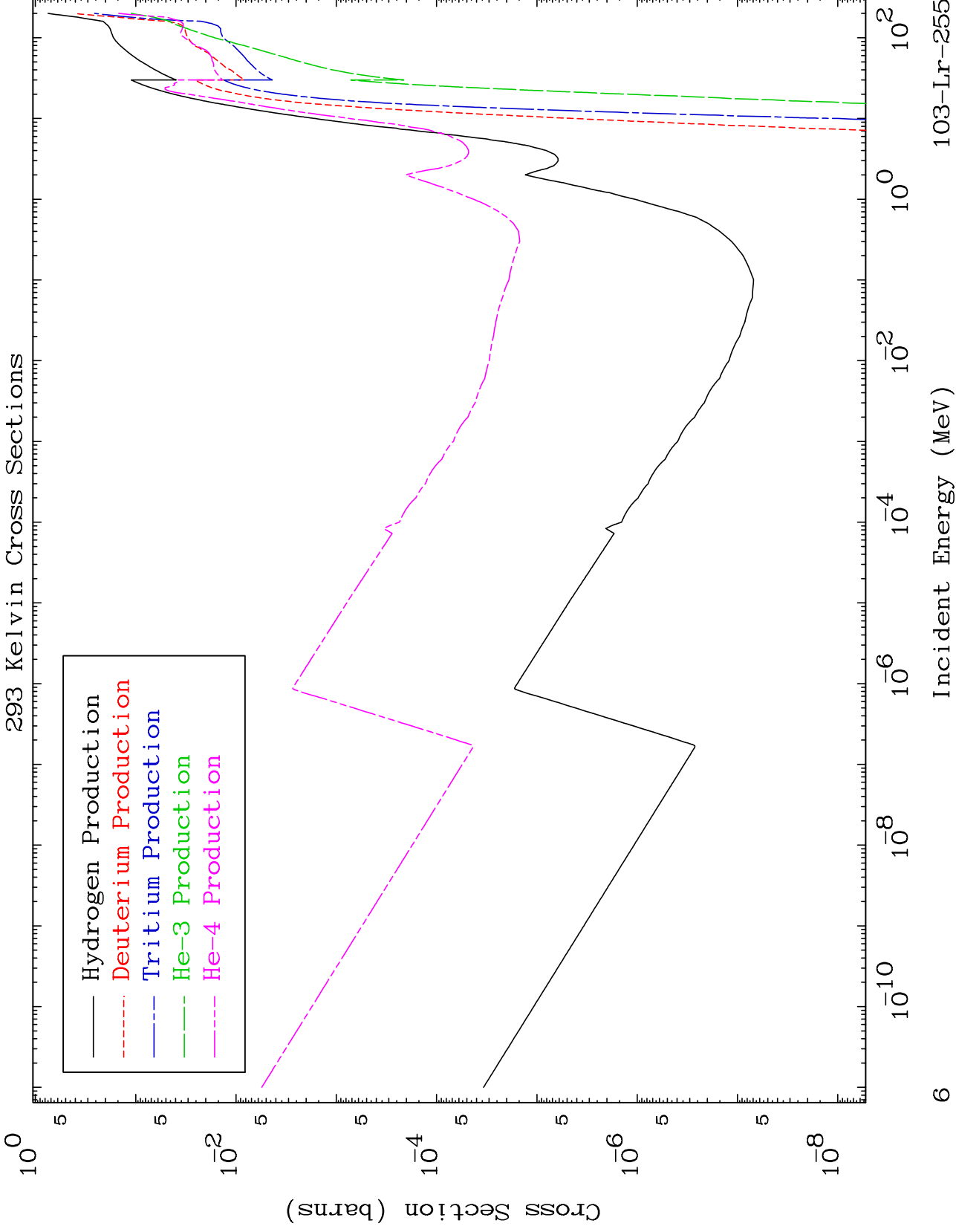
103-Lr-255



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Particle Production  
293 Kelvin Cross Sections

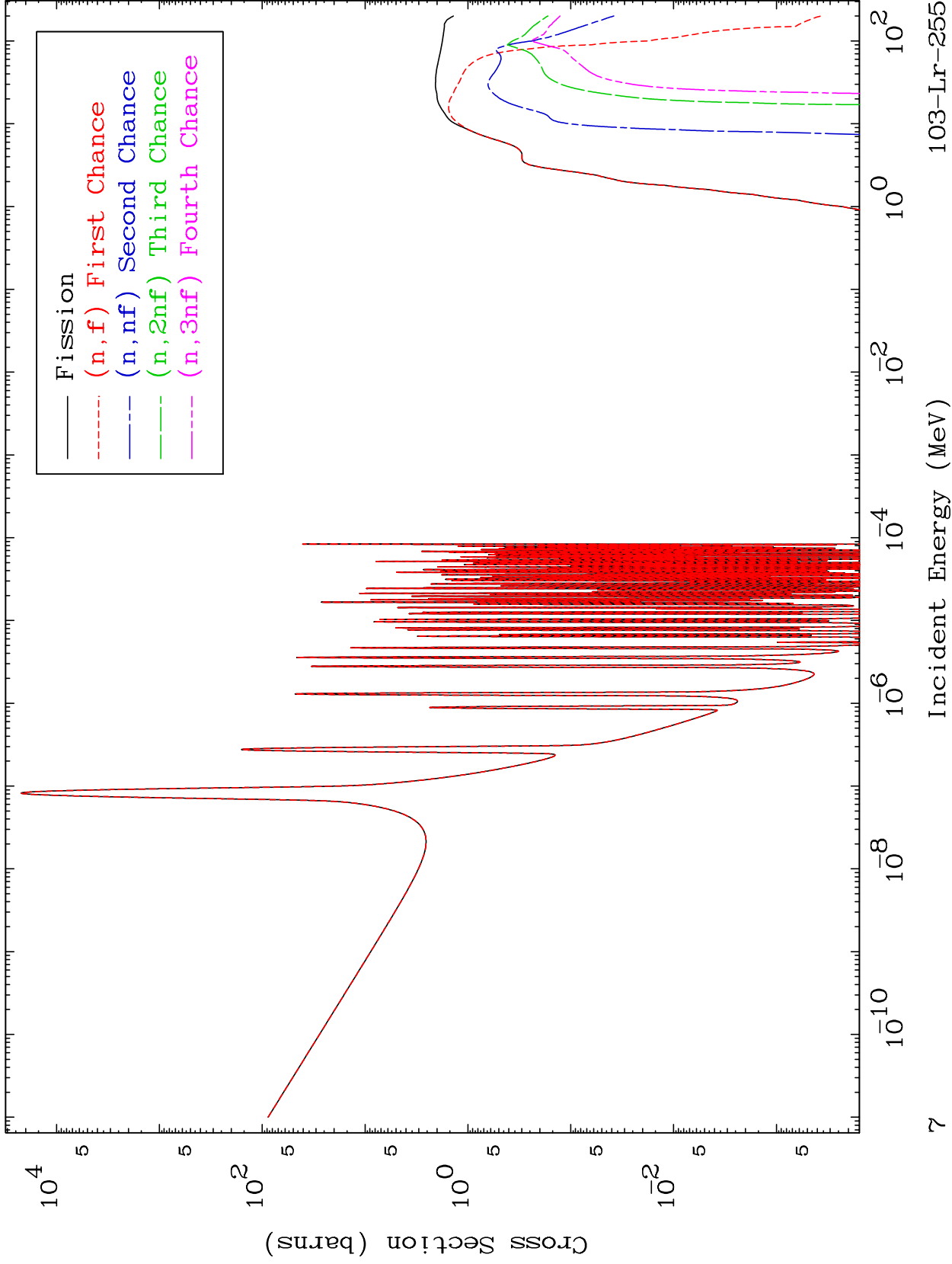
103-Lr-255



MAT 9984

Fission  
293 Kelvin Cross Sections

103-Lr-255

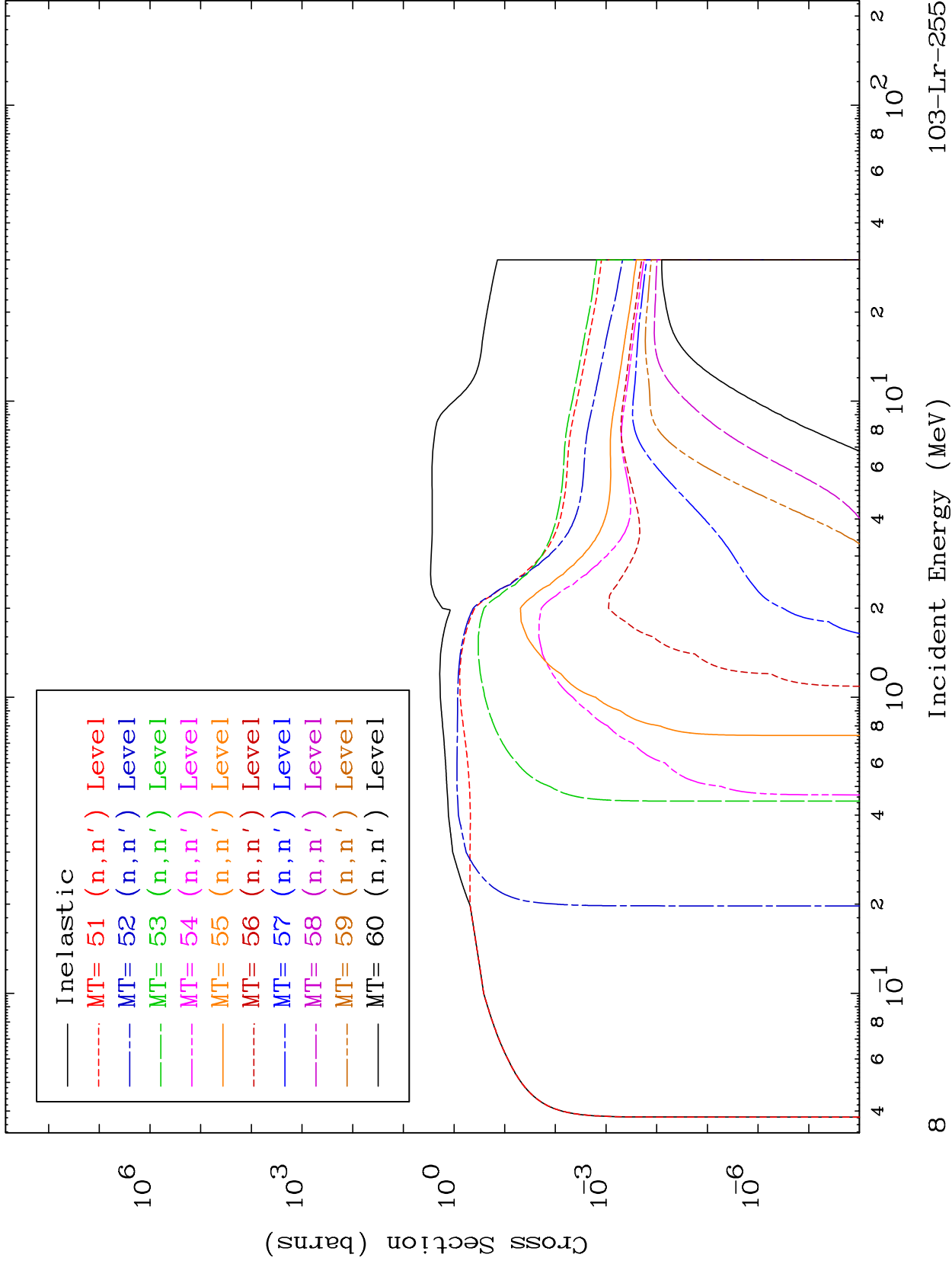




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(n,n') Level  
293 Kelvin Cross Sections

103-Lr-255

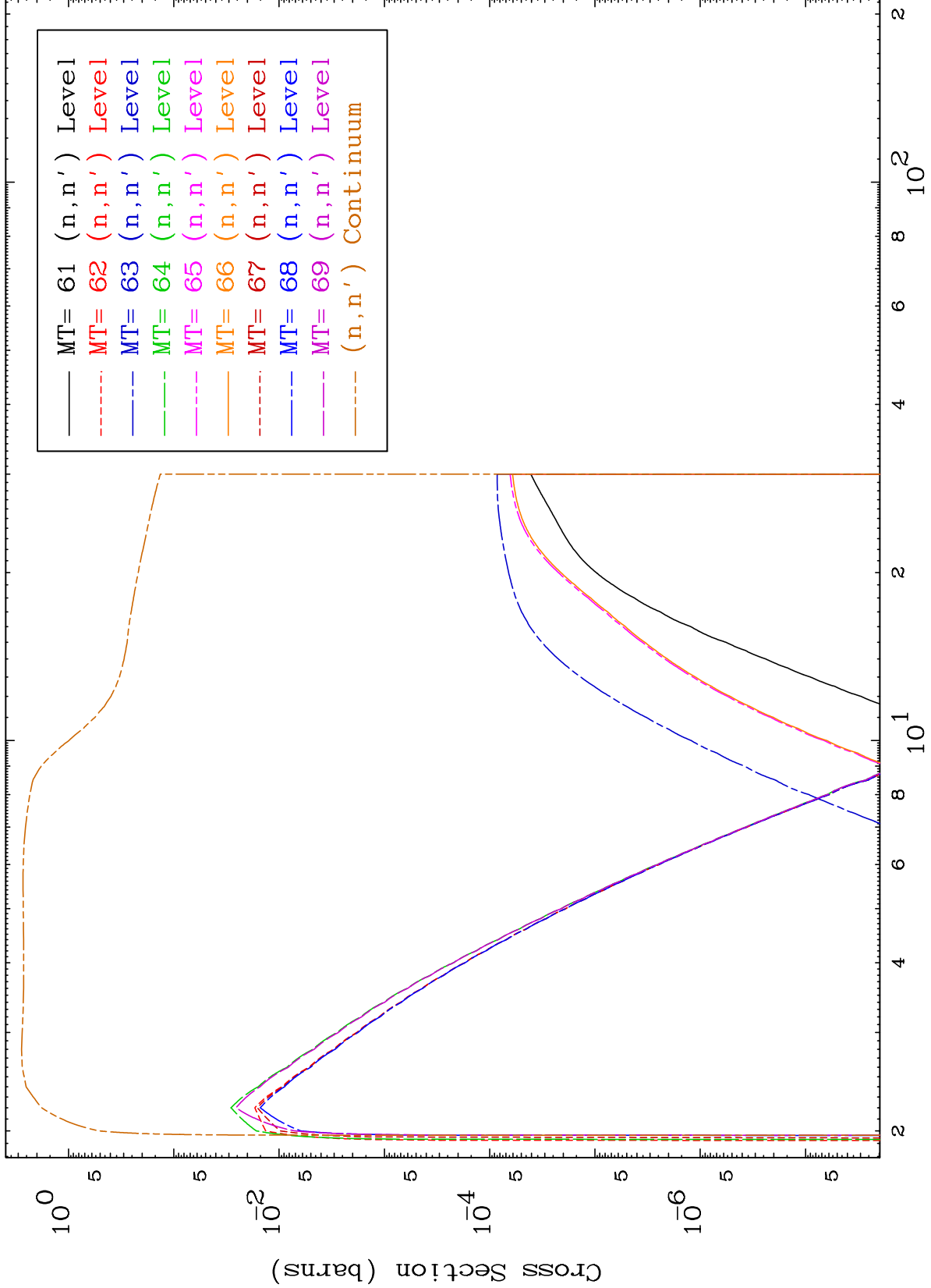


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(n,n') Level

103-Lr-255

293 Kelvin Cross Sections



9

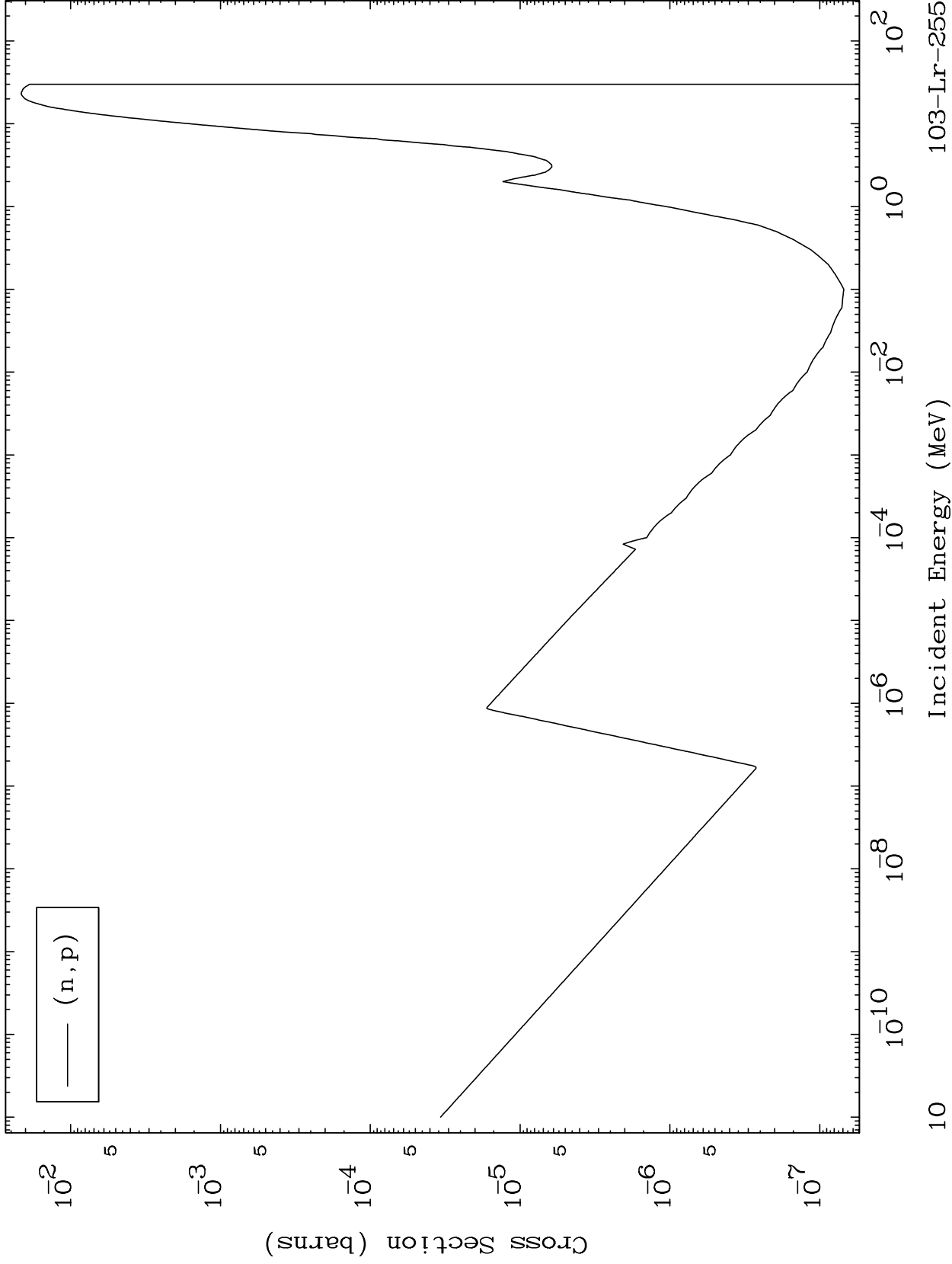
Incident Energy (MeV)

103-Lr-255

MAT 9984

(n,p) Levels  
293 Kelvin Cross Sections

103-Lr-255



10

Incident Energy (MeV)

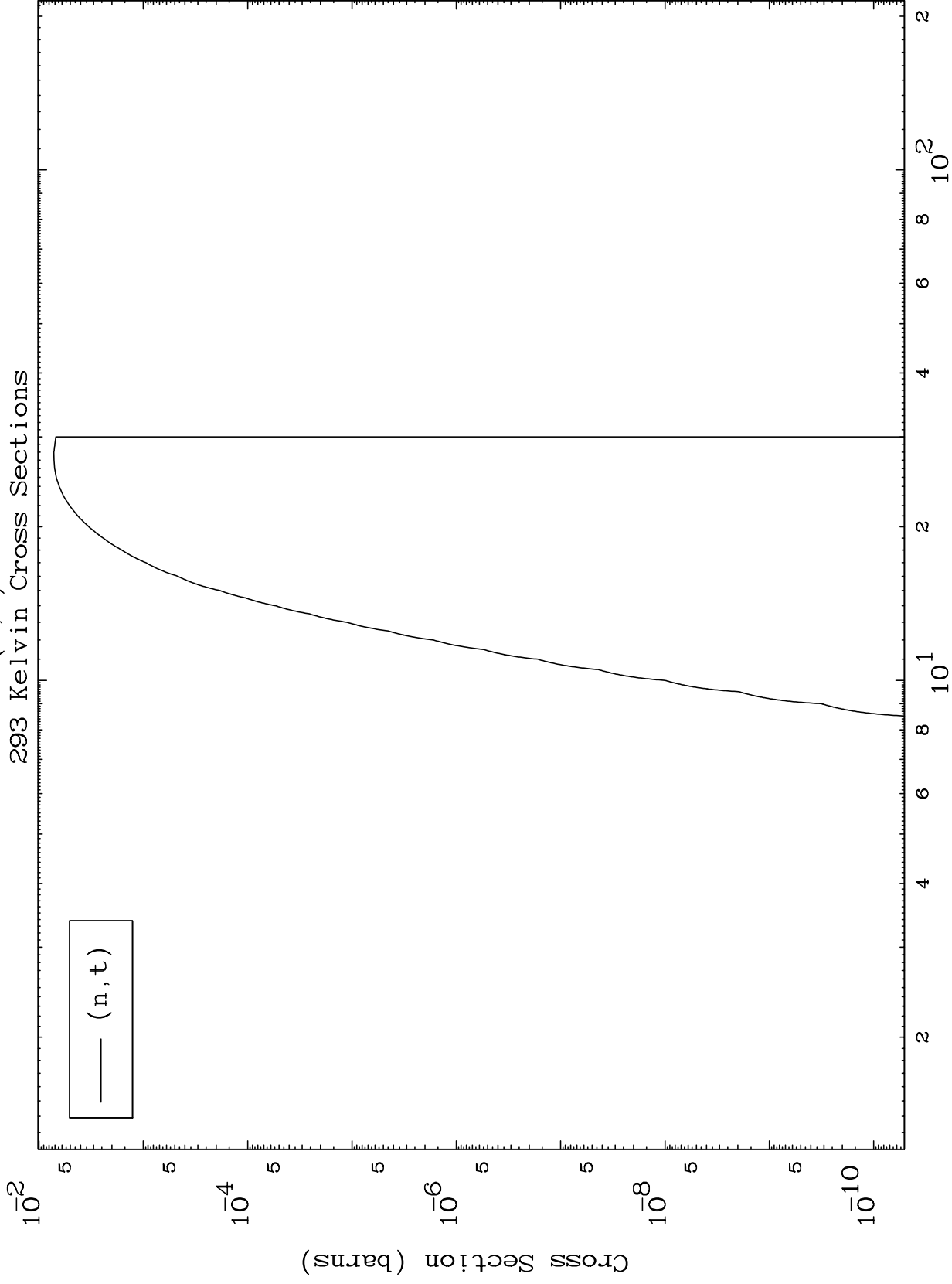
103-Lr-255



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(n,t) Levels  
293 Kelvin Cross Sections

103-Lr-255



12

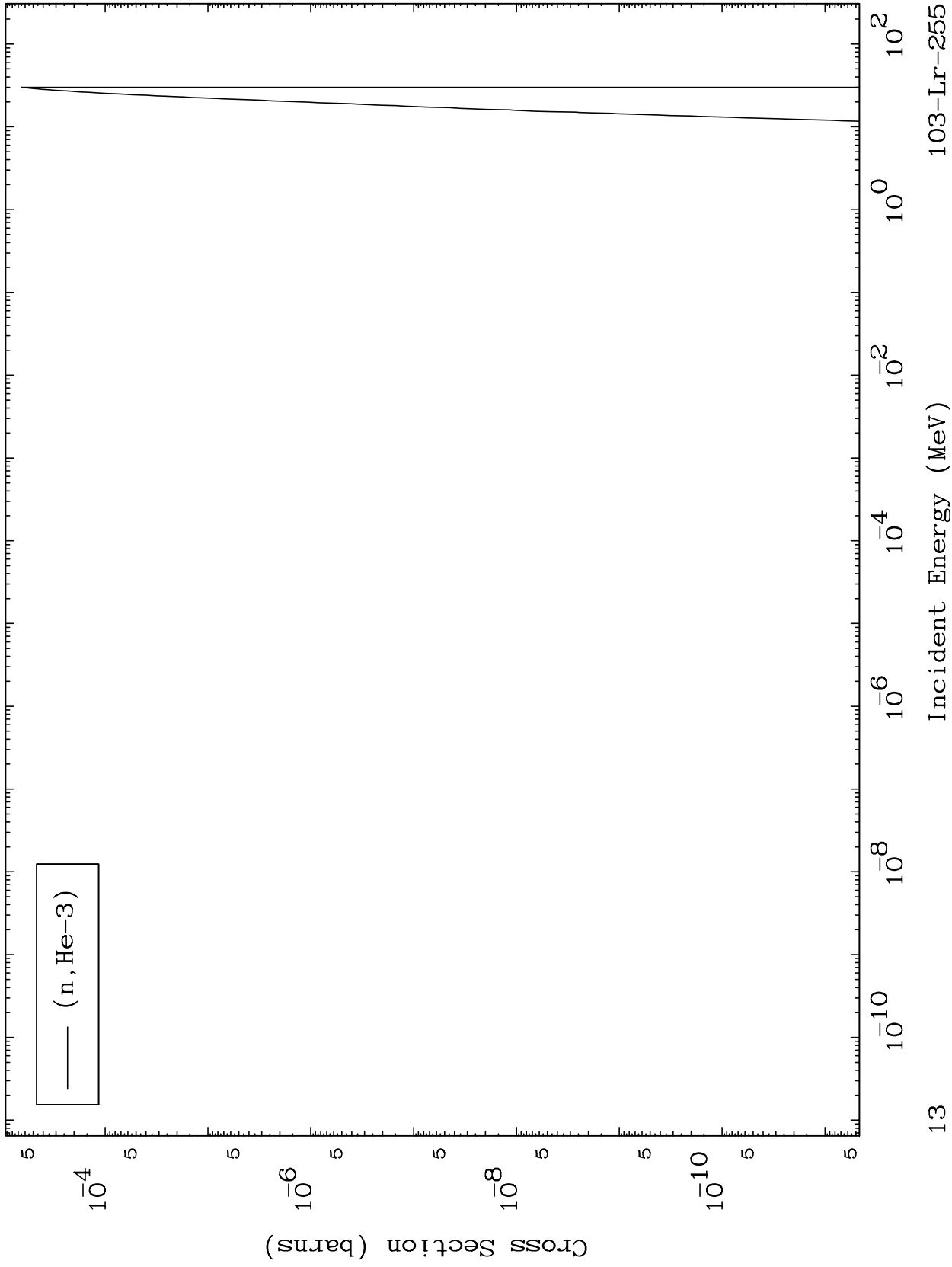
Incident Energy (MeV)

103-Lr-255

MAT 9984

(n,He3) Levels  
293 Kelvin Cross Sections

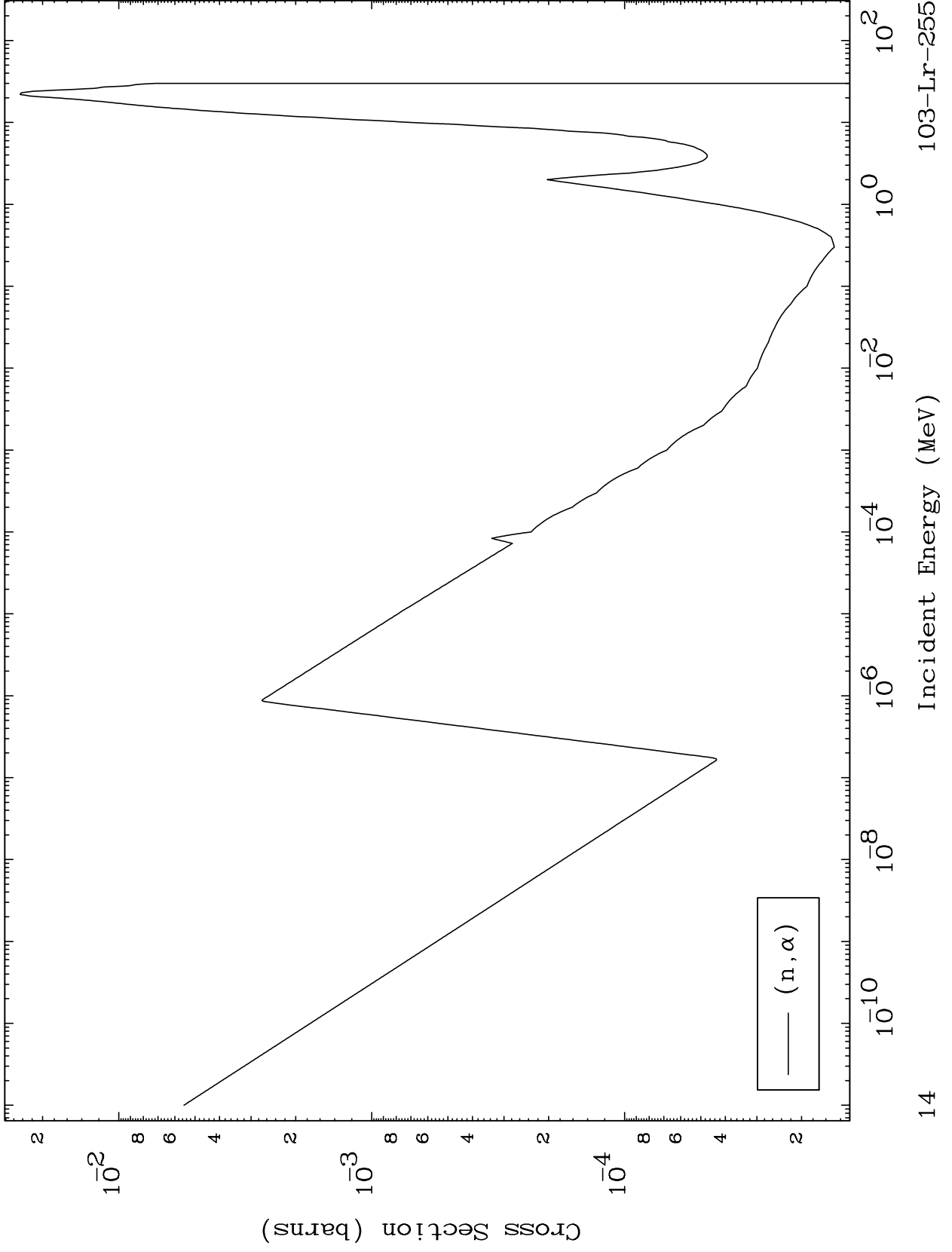
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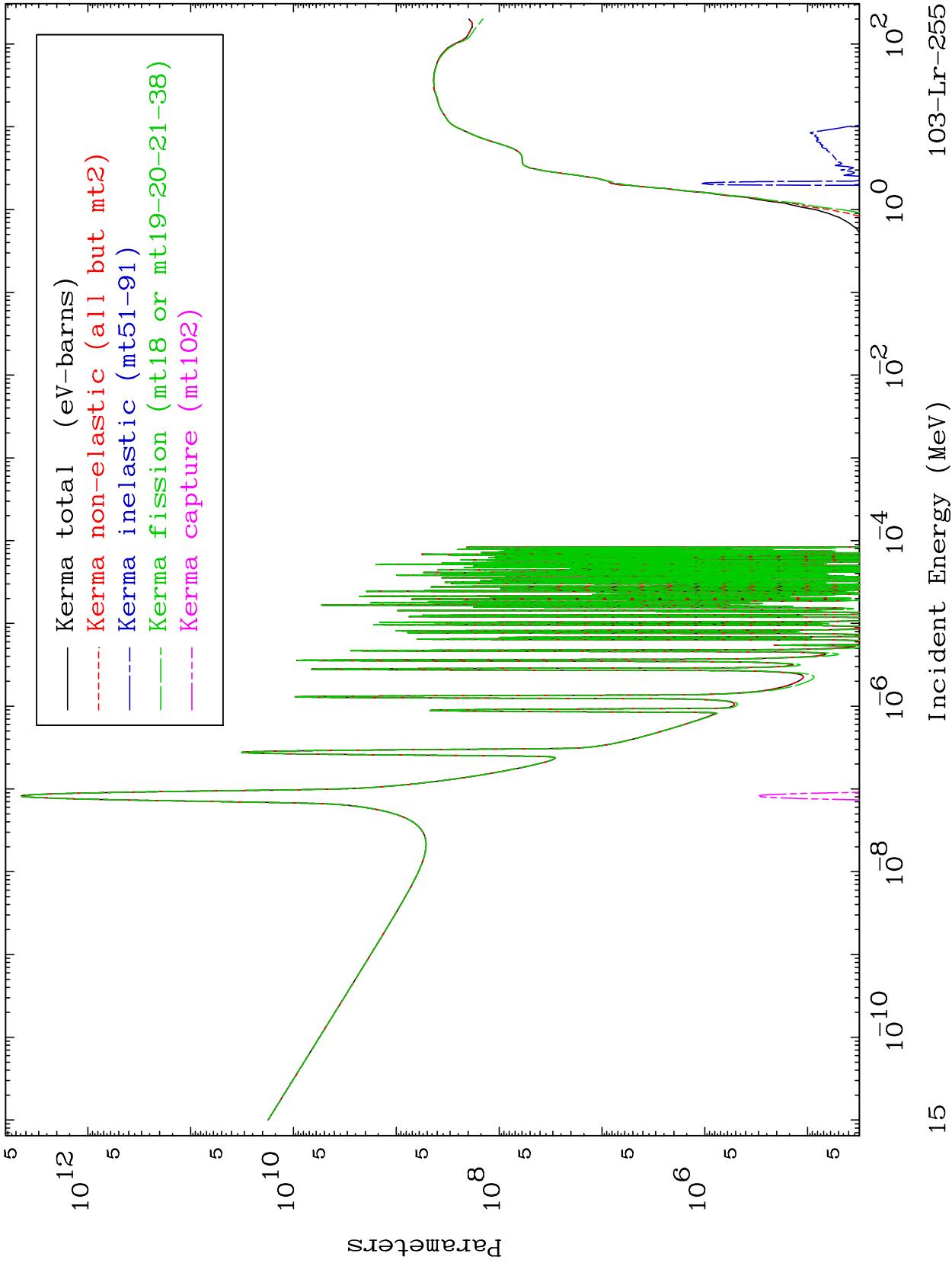


MAT 9984

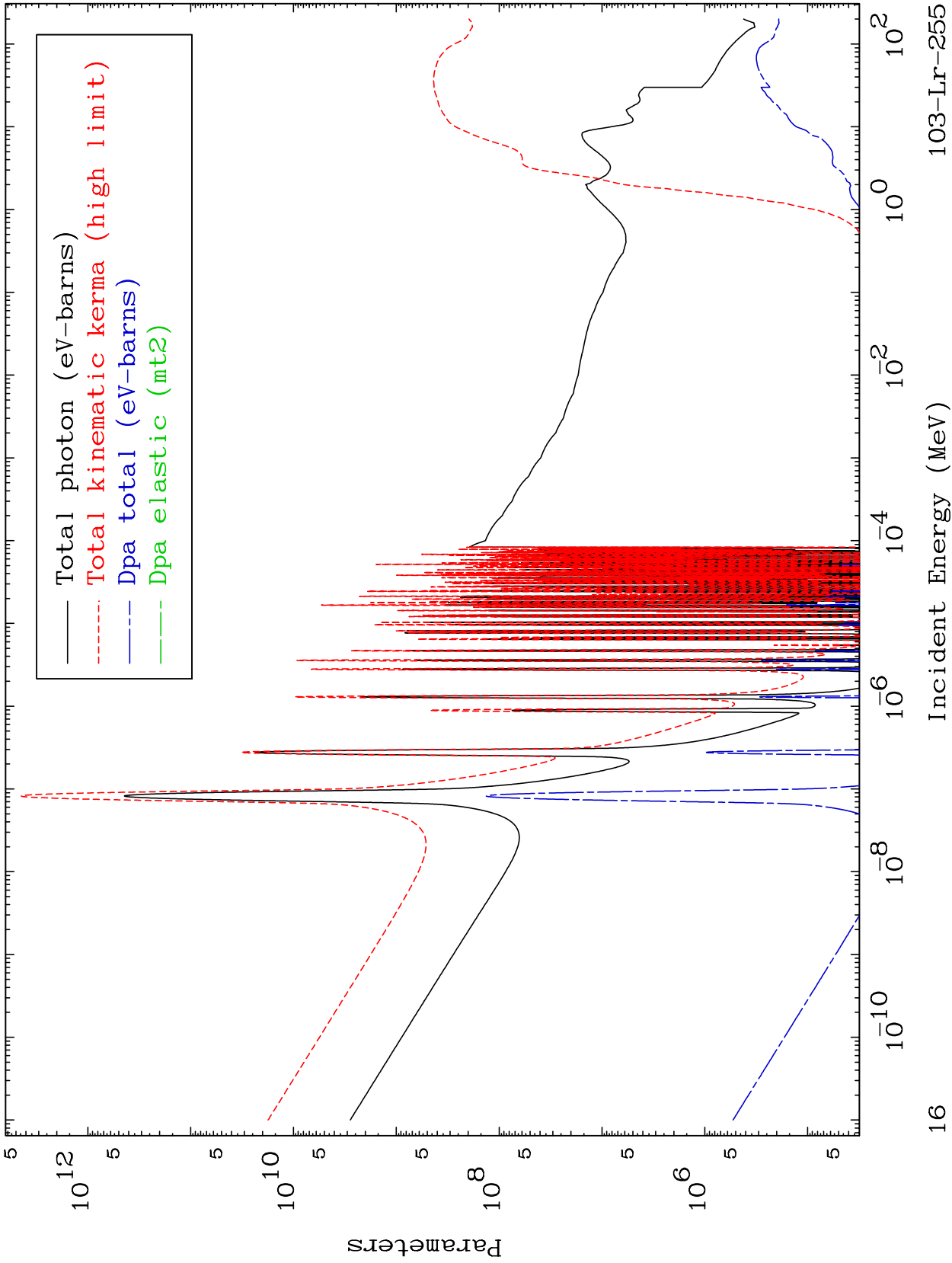
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

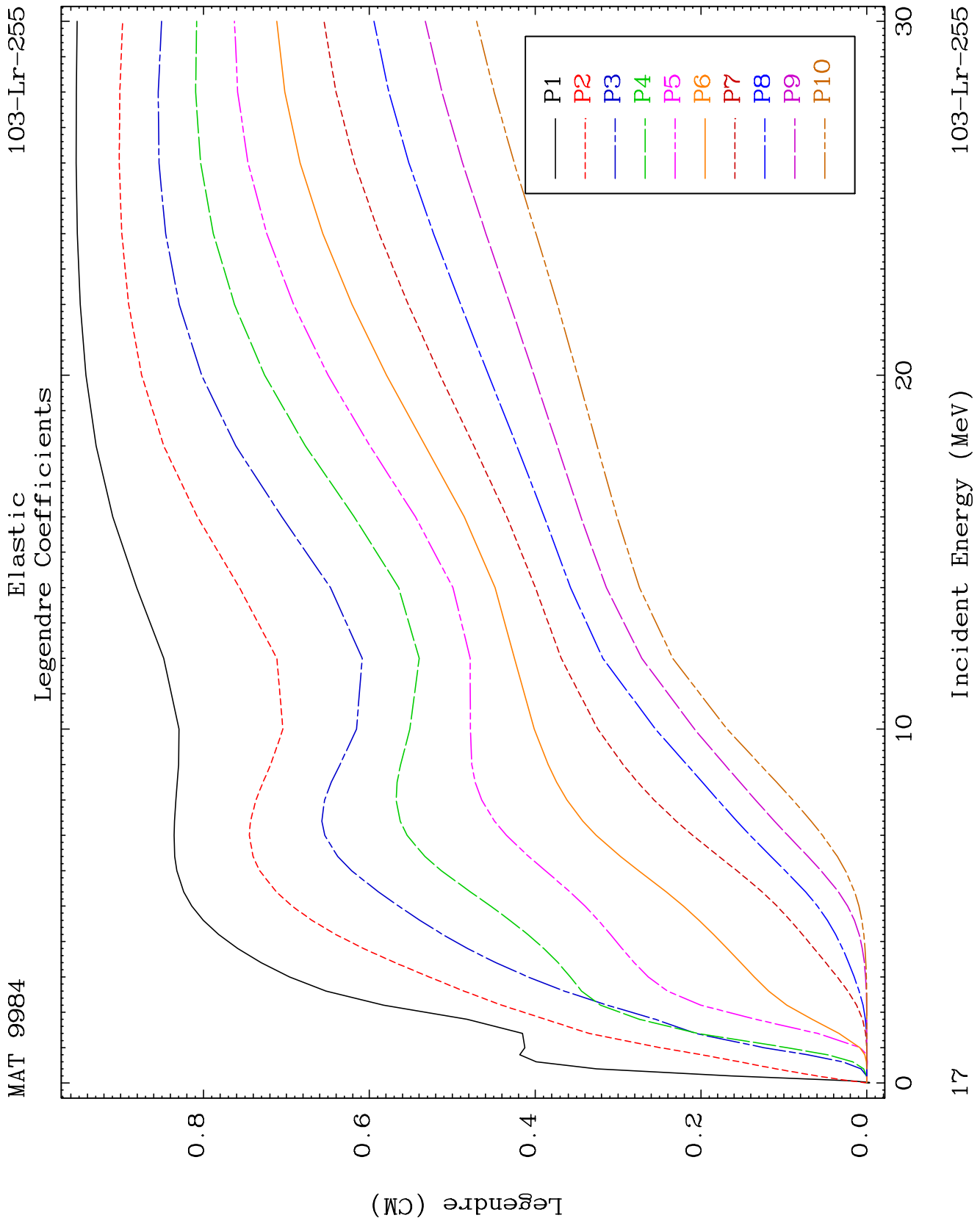
103-Lr-255

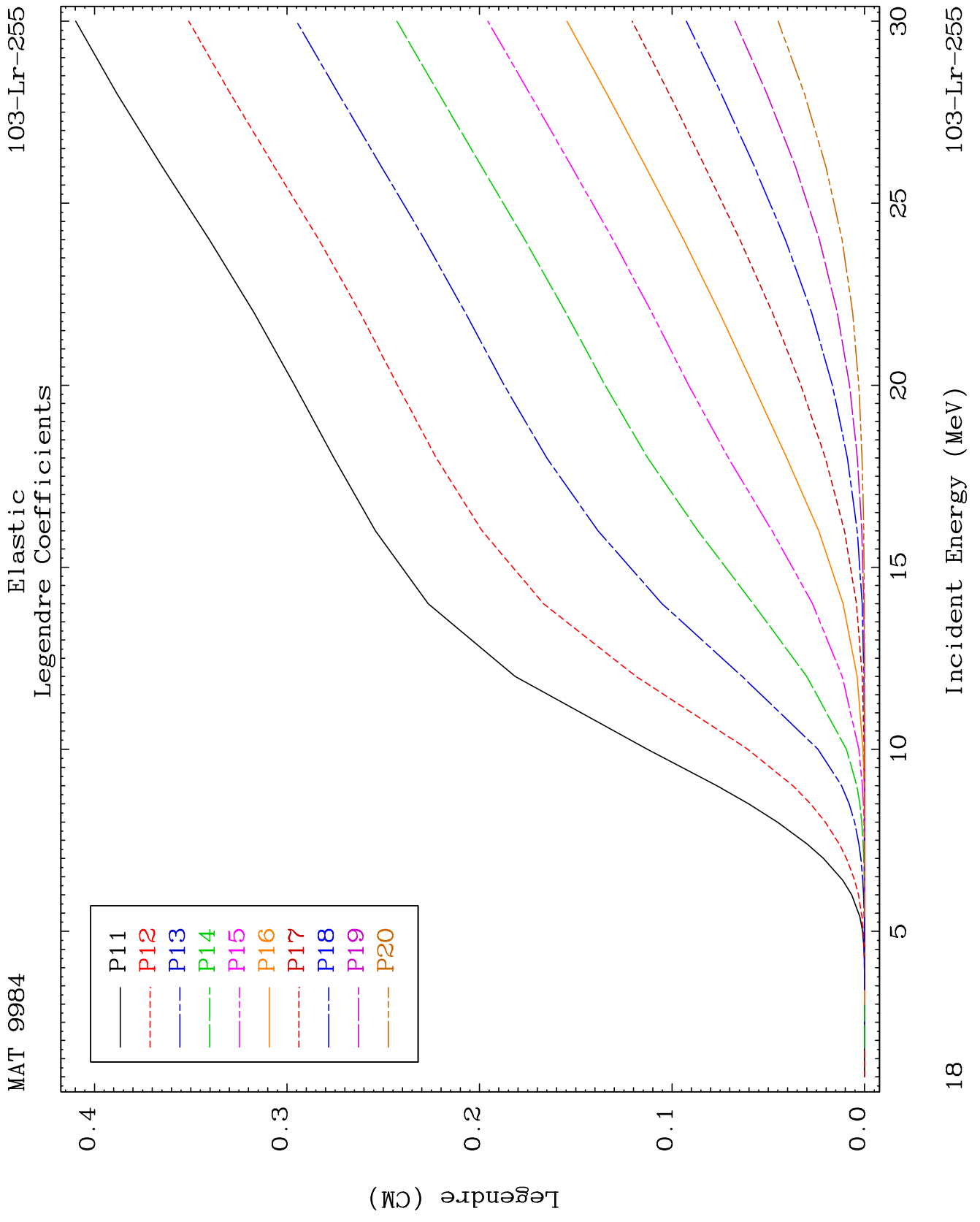








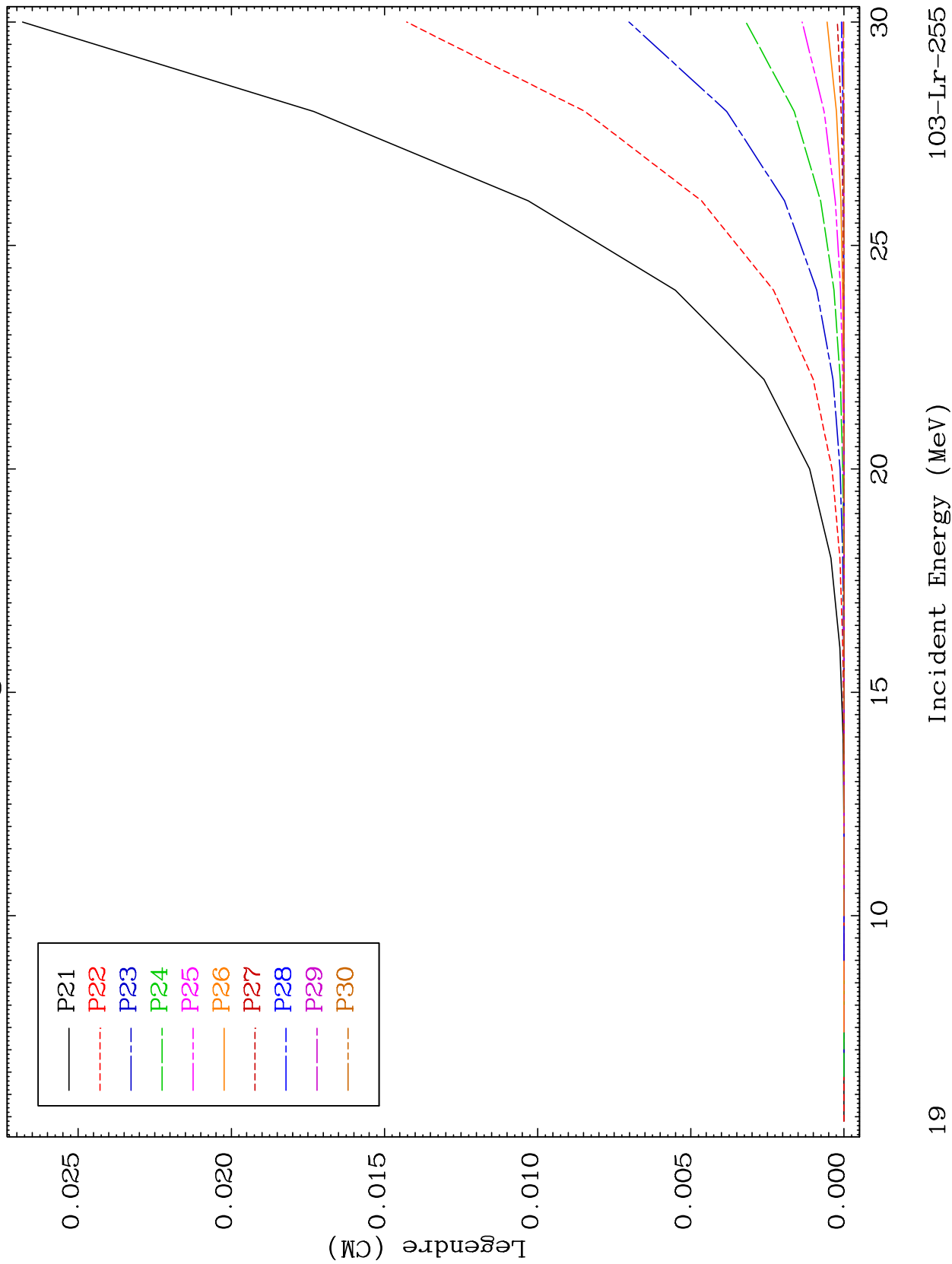




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### Elastic Legendre Coefficients

103-Lr-255



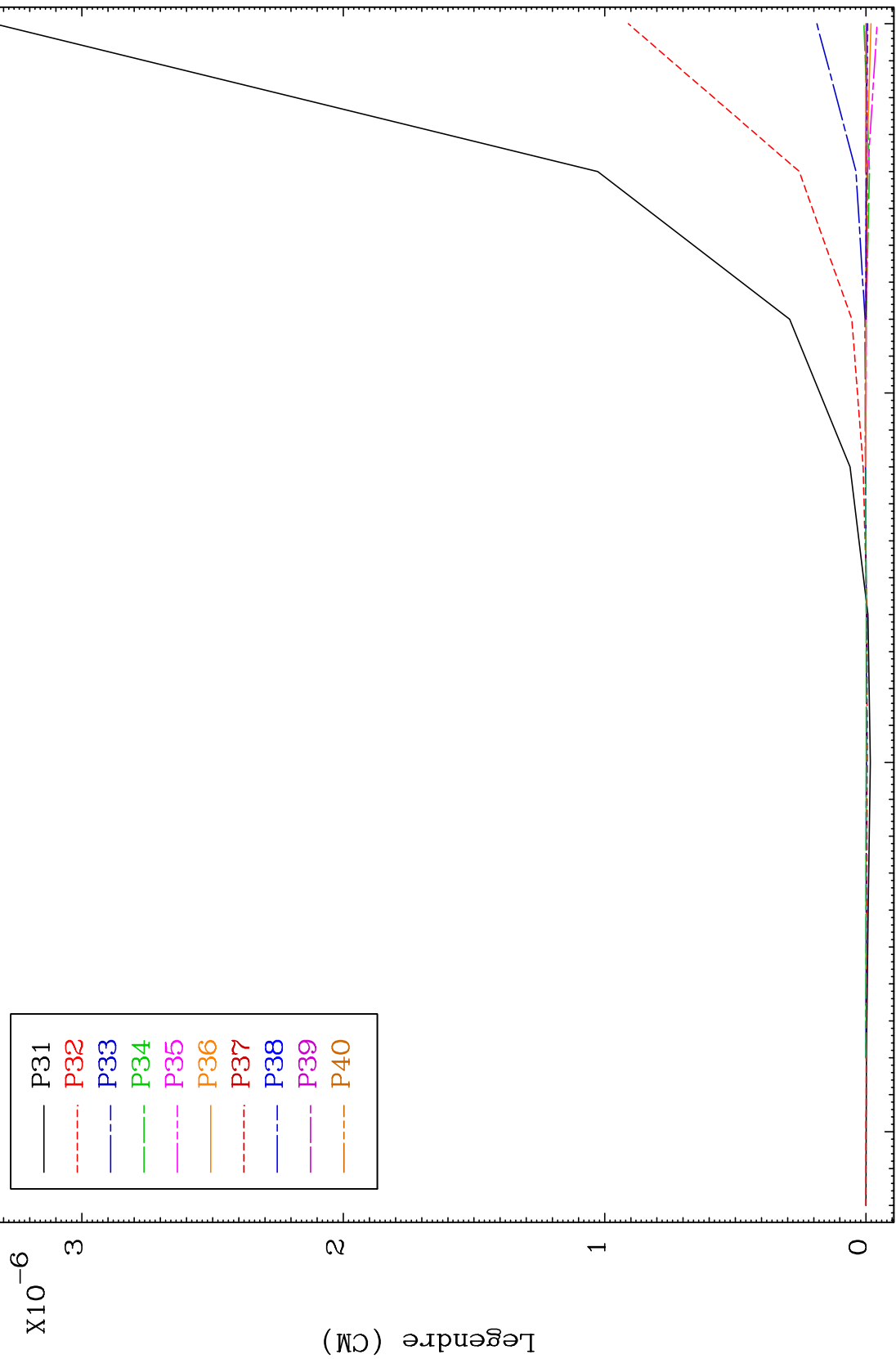
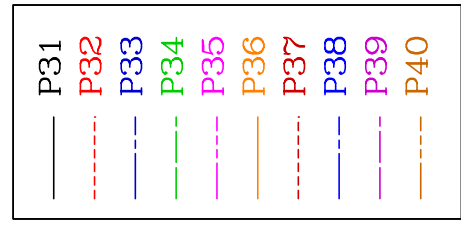
19

103-Lr-255

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Elastic Legendre Coefficients

103-Lr-255



103-Lr-255

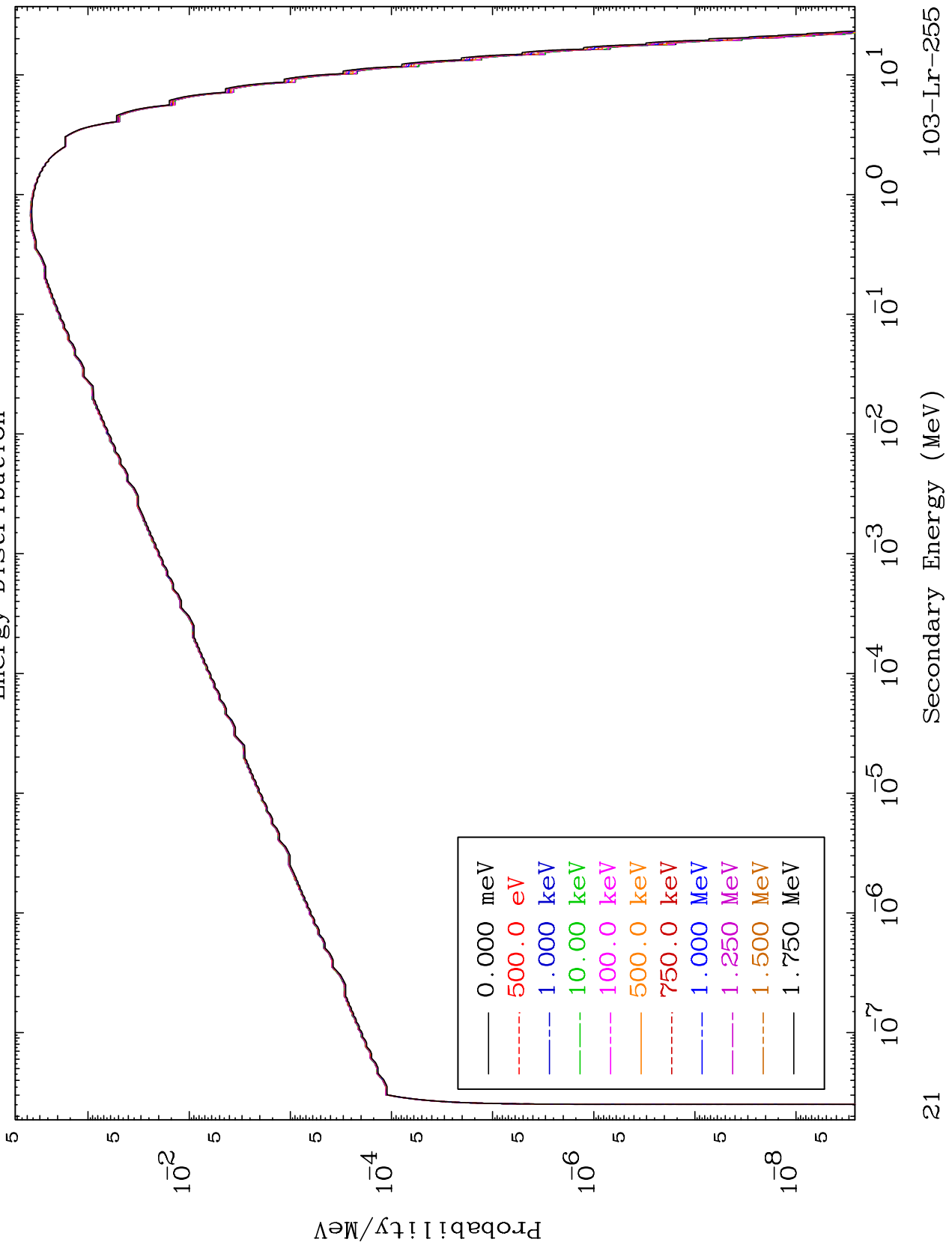
Incident Energy (MeV)

20

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### Fission Energy Distribution

103-Lr-255

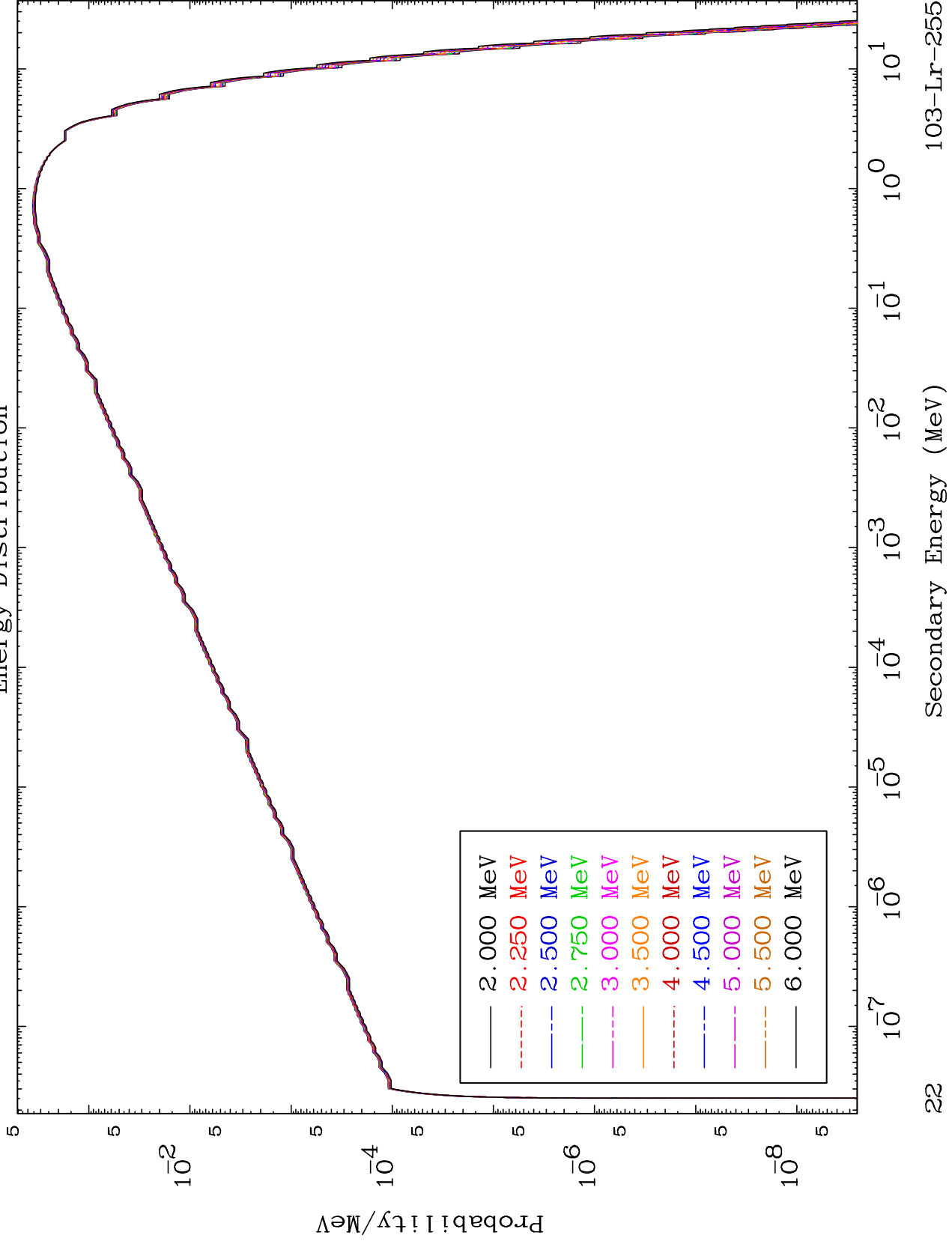


103-Lr-255

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Fission  
Energy Distribution

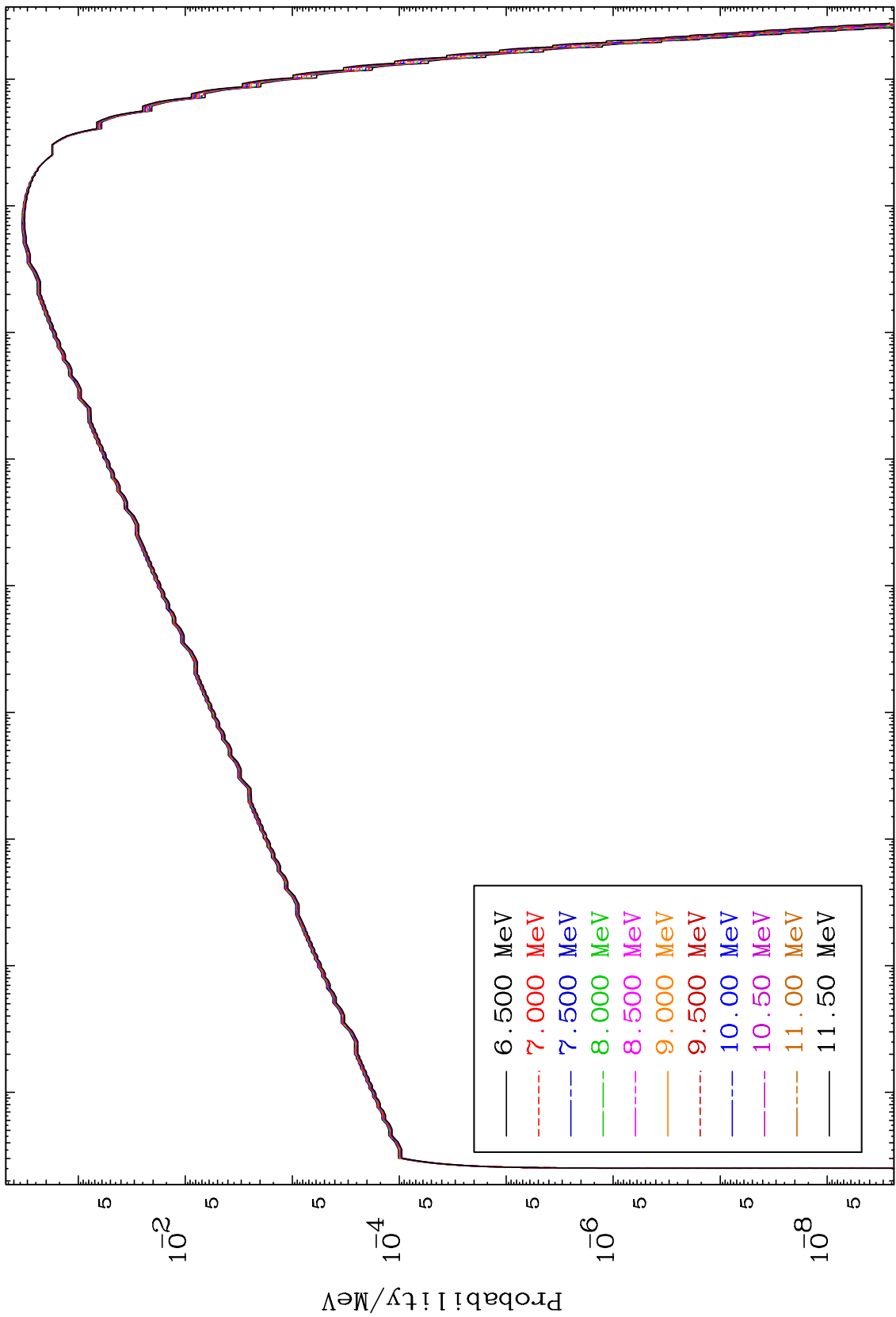
103-Lr-255



MAT 9984

Fission  
Energy Distribution

103-Lr-255



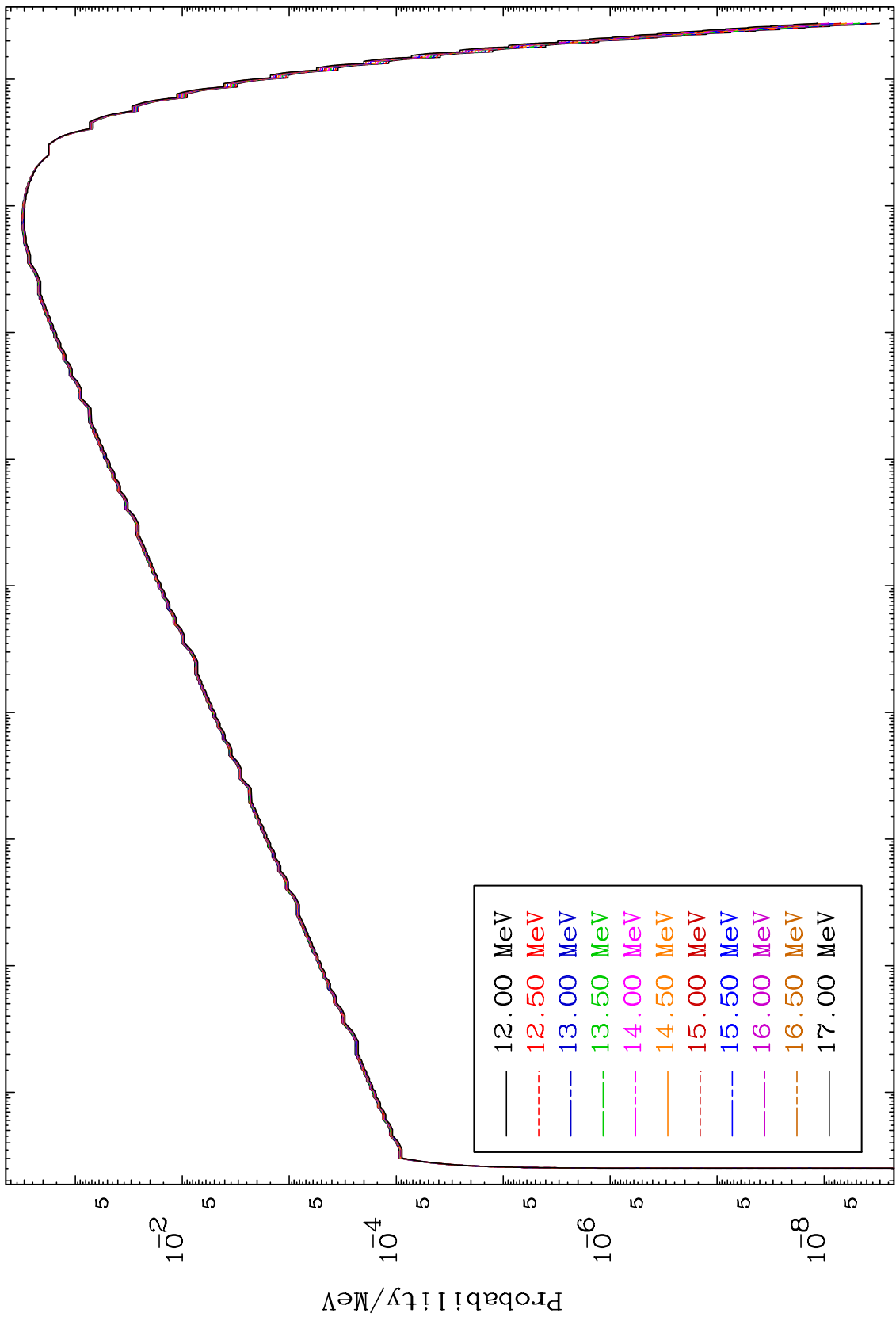
103-Lr-255



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### Fission Energy Distribution

103-Lr-255

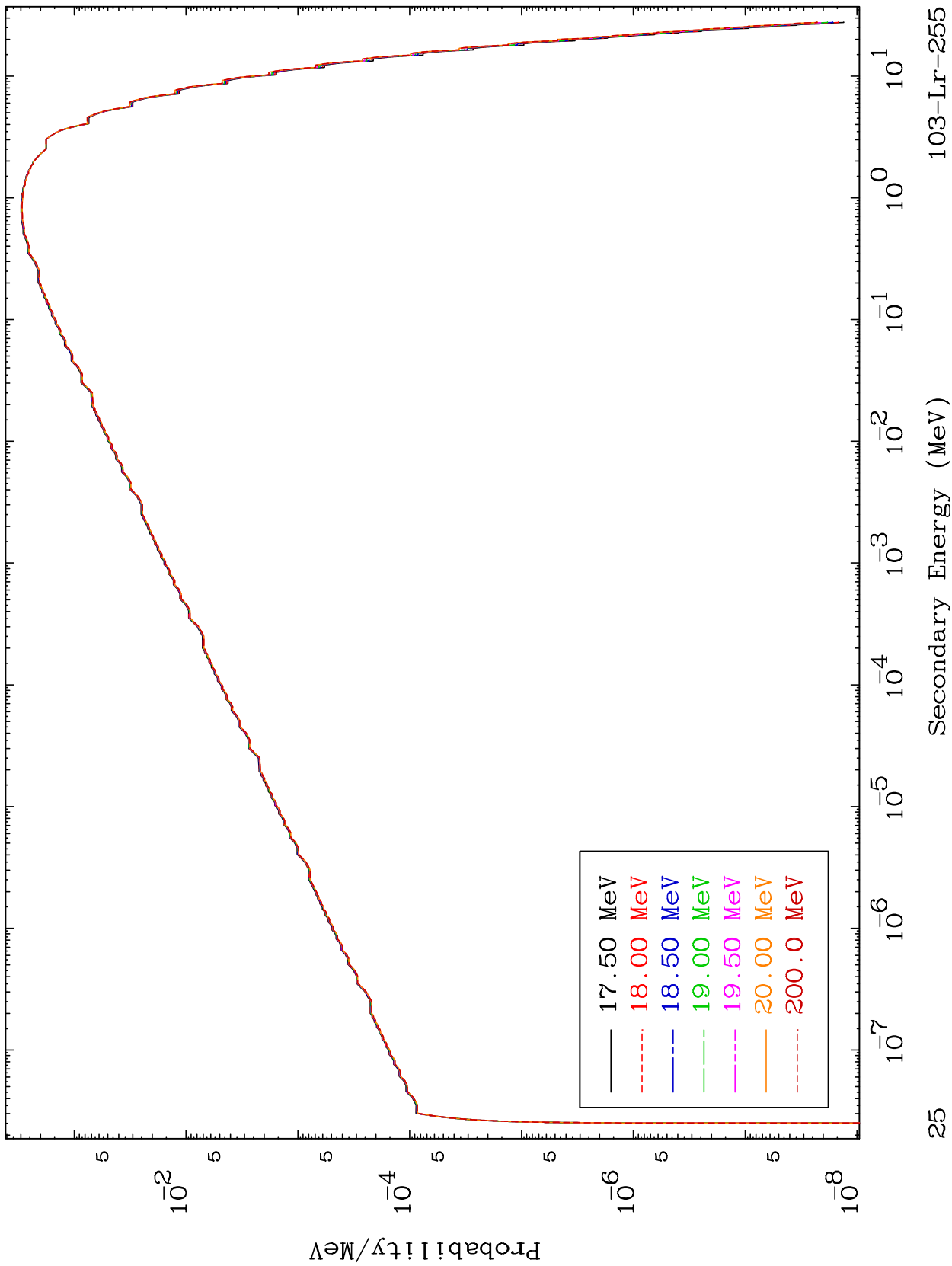


103-Lr-255

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### Fission Energy Distribution

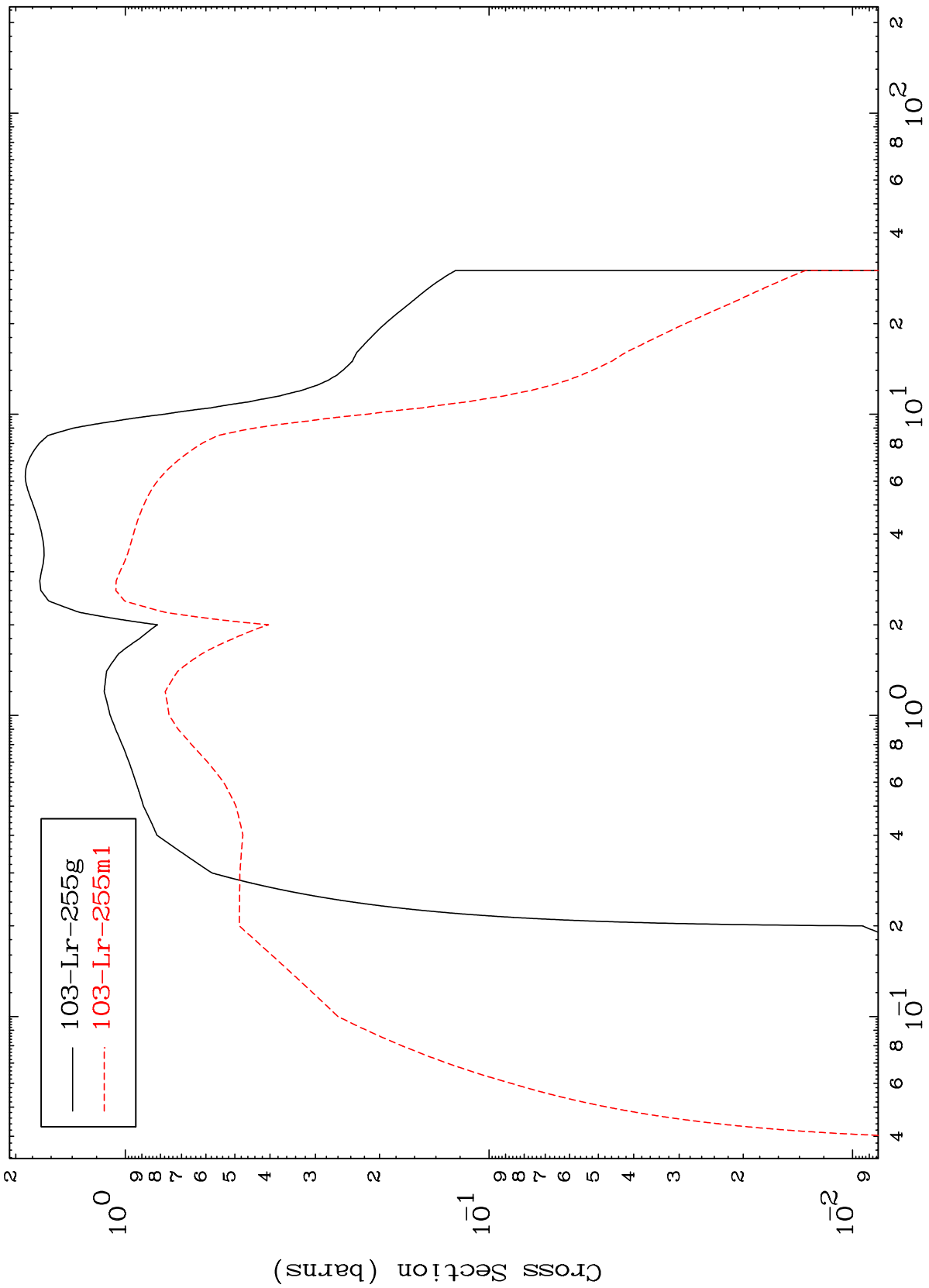
103-Lr-255



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103-Lr-255

Inelastic  
Radionuclide Production Cross Section



26

103-Lr-255

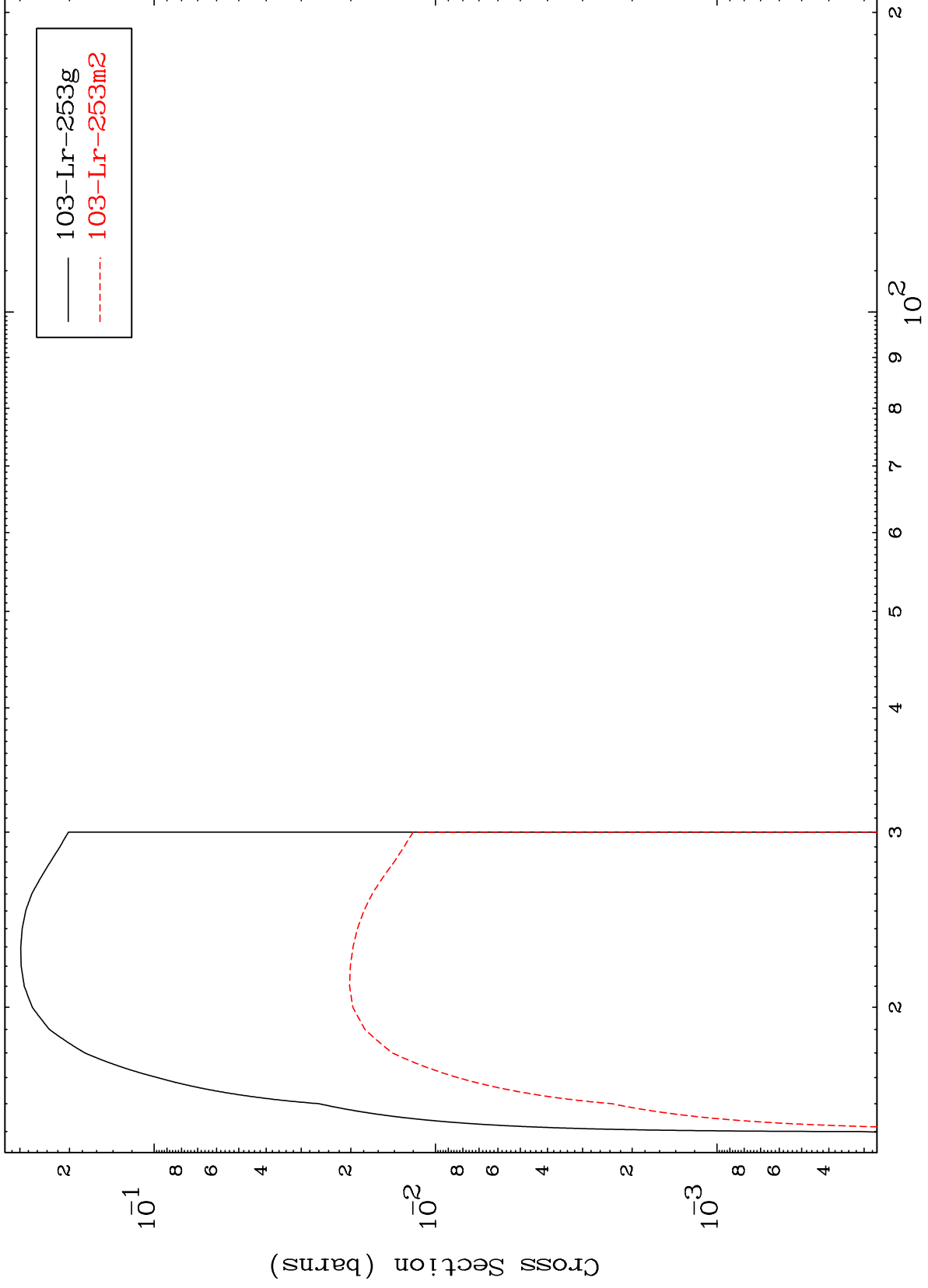
Incident Energy (MeV)

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(n,3n)

103-Lr-255

Radionuclide Production Cross Section



27

Incident Energy (MeV)

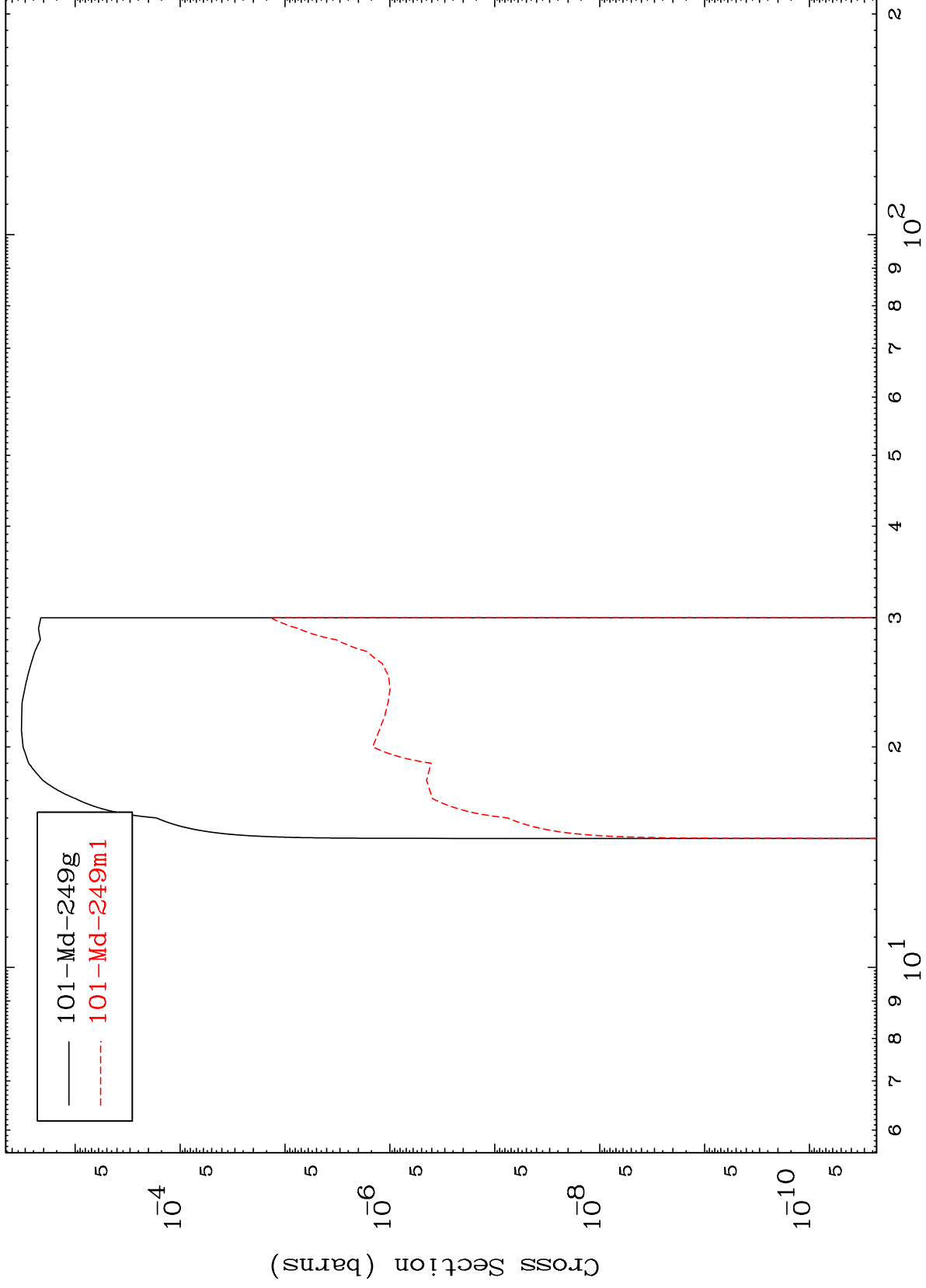
103-Lr-255

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(n,3n)  $\alpha$

103-Lr-255

Radionuclide Production Cross Section



28

Incident Energy (MeV)

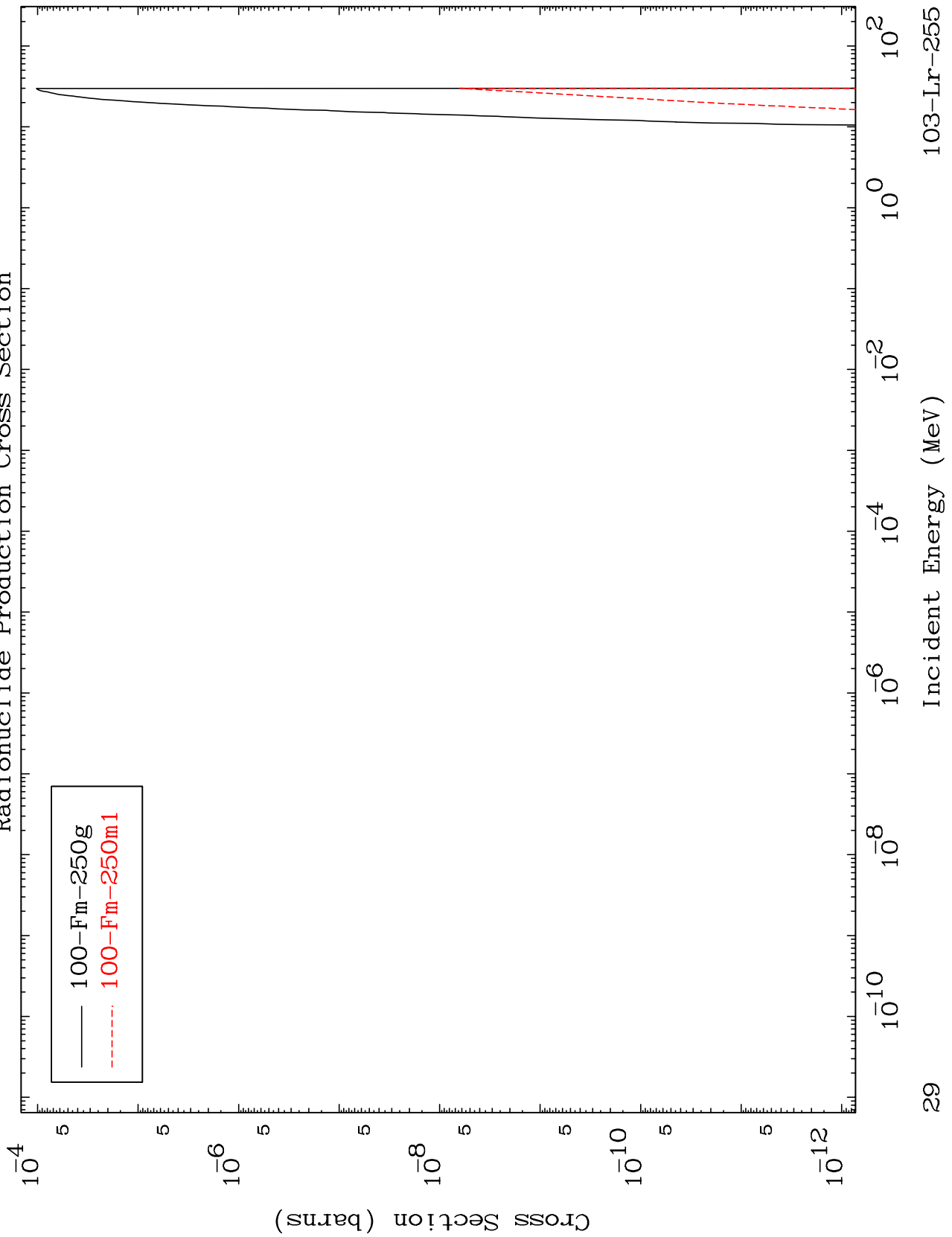
103-Lr-255

MAT 9984

(n,n') p  $\alpha$

103-Lr-255

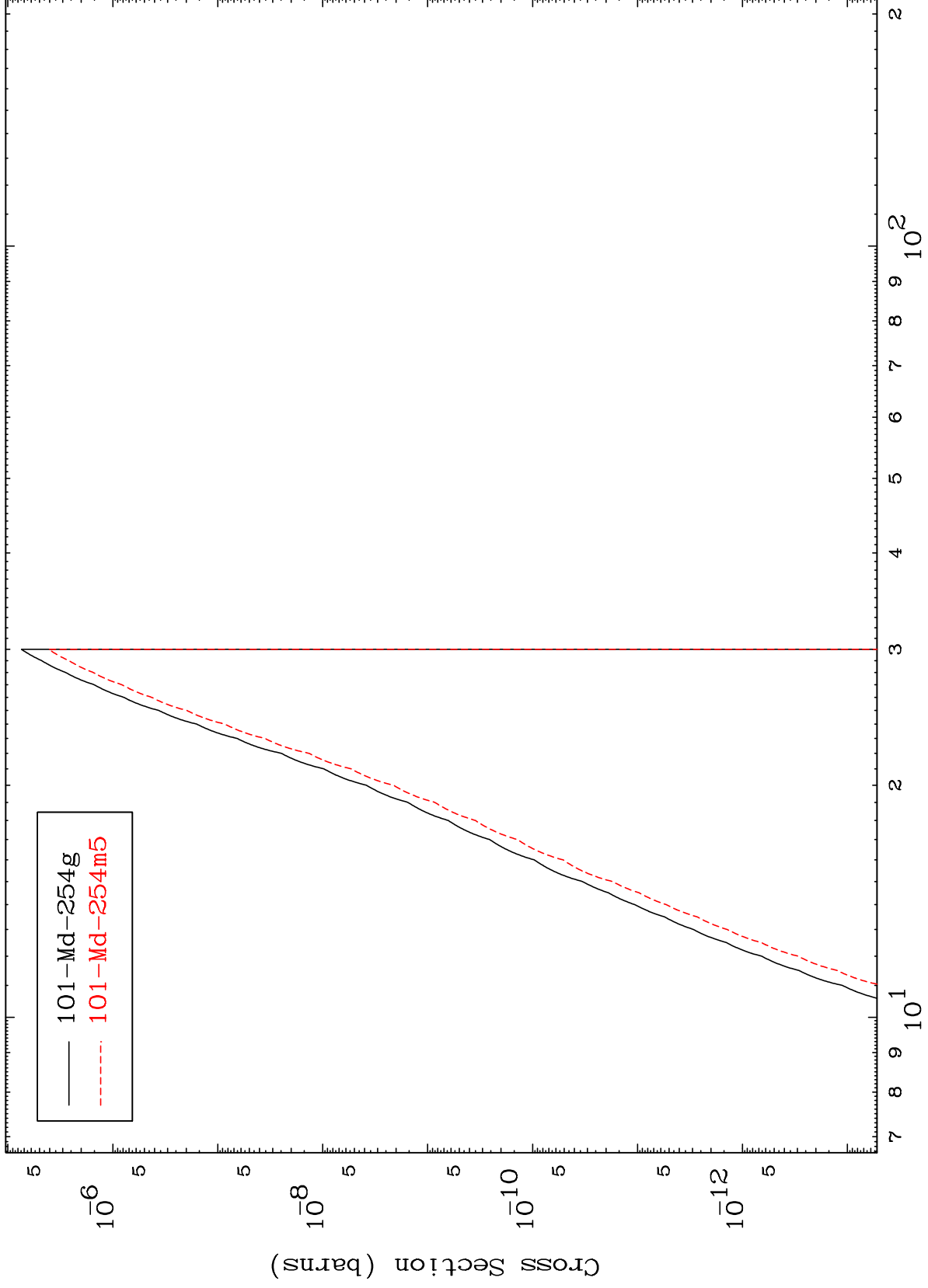
Radionuclide Production Cross Section



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103-Lr-255

(n,2p)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

103-Lr-255

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(n,d)  $\alpha$

103-Lr-255

Radionuclide Production Cross Section

