

Program EVALPLOT
(Version 2017-1)

by

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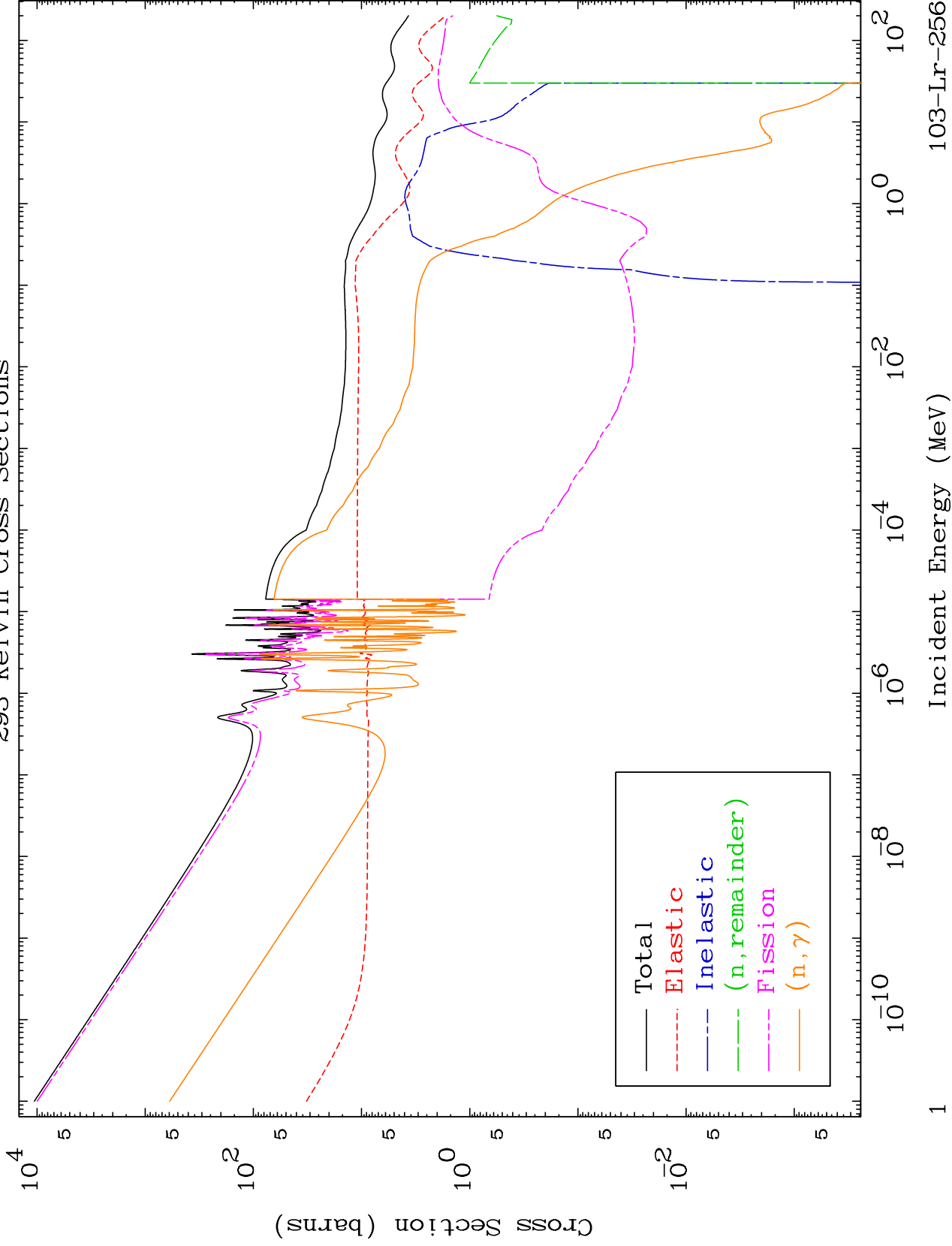
Press Mouse Button to Start

MAT 9985

Major

293 Kelvin Cross Sections

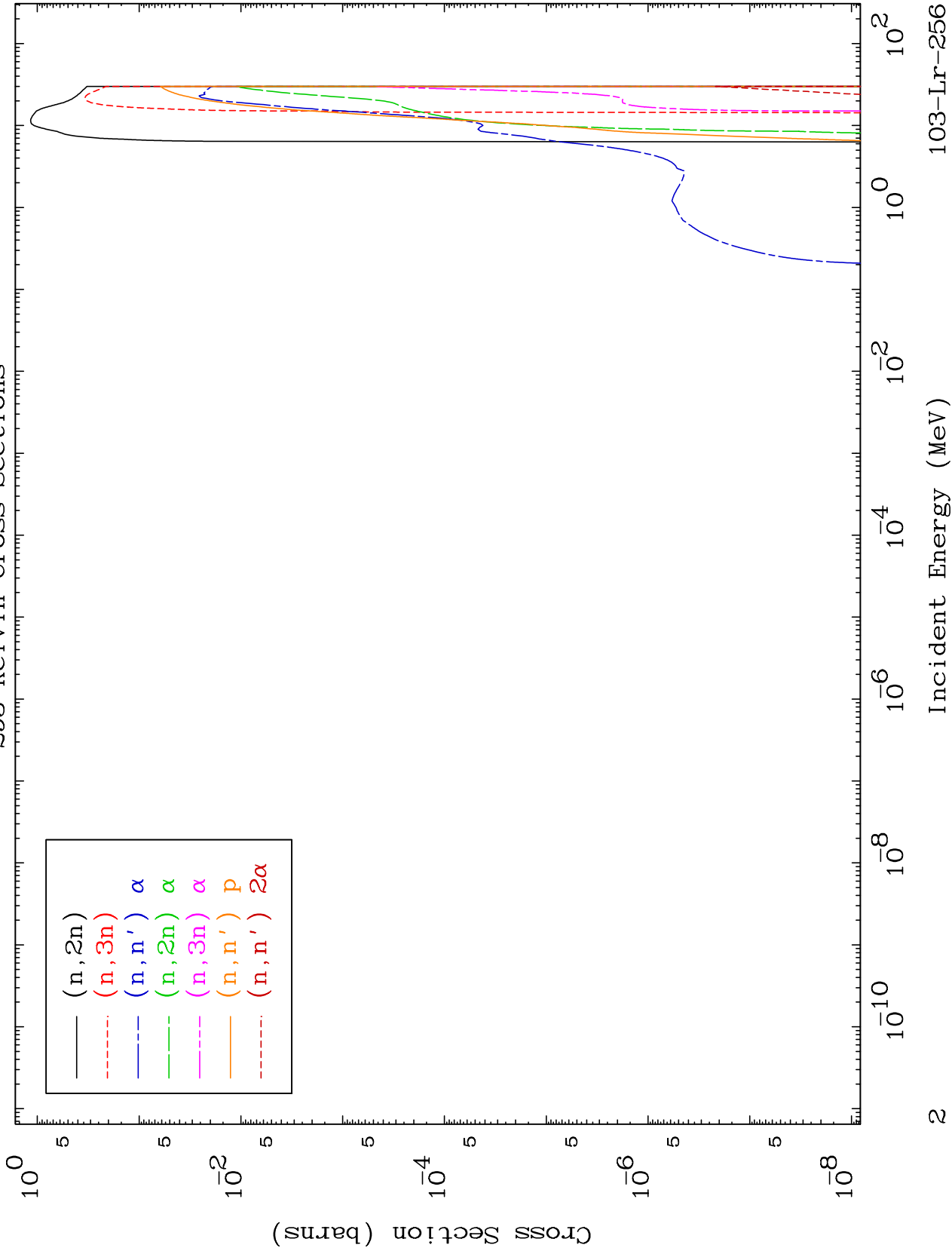
103-Lr-256



MAT 9985

Neutron Production
293 Kelvin Cross Sections

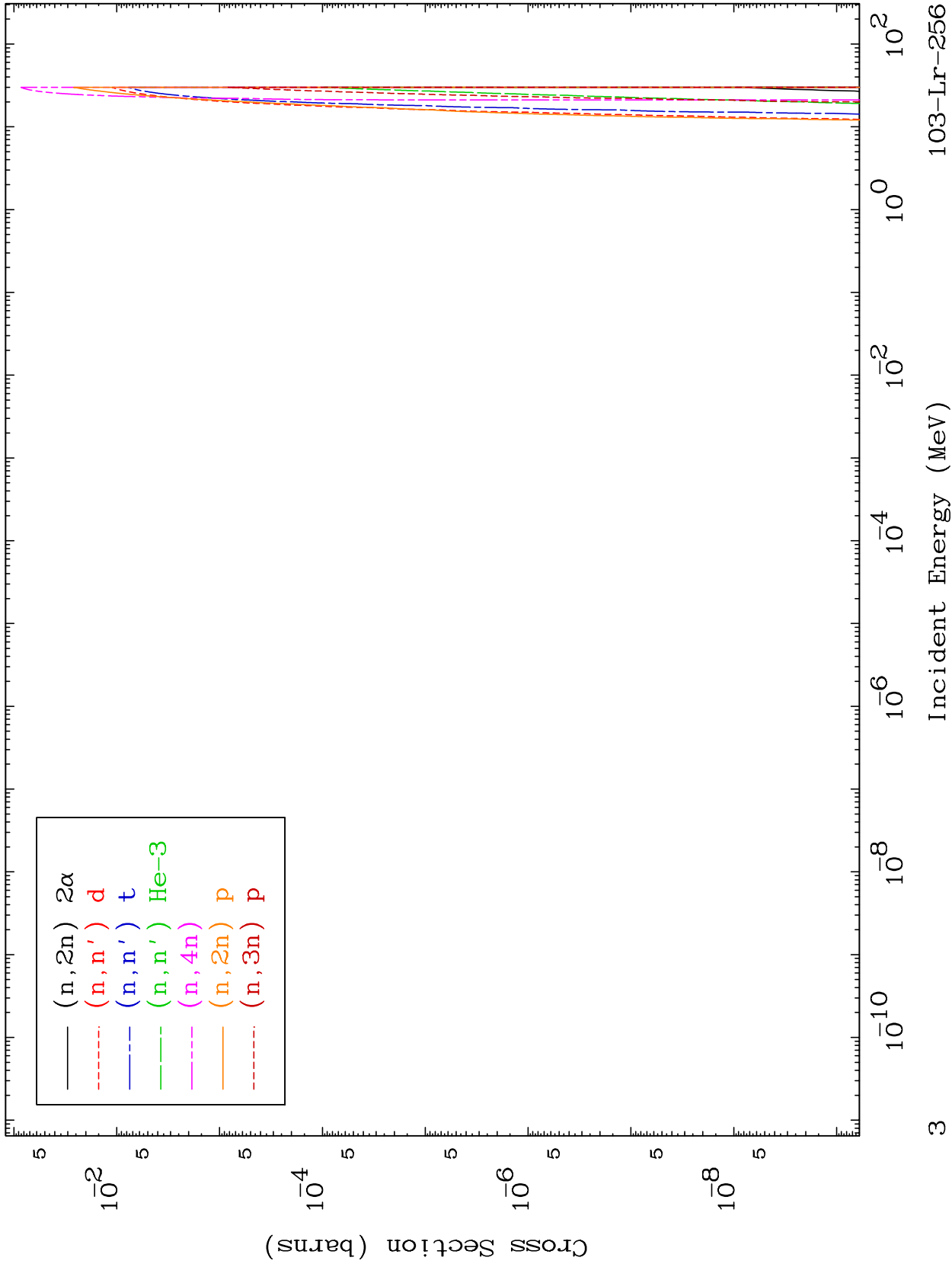
103-Lr-256



MAT 9985

Neutron Production
293 Kelvin Cross Sections

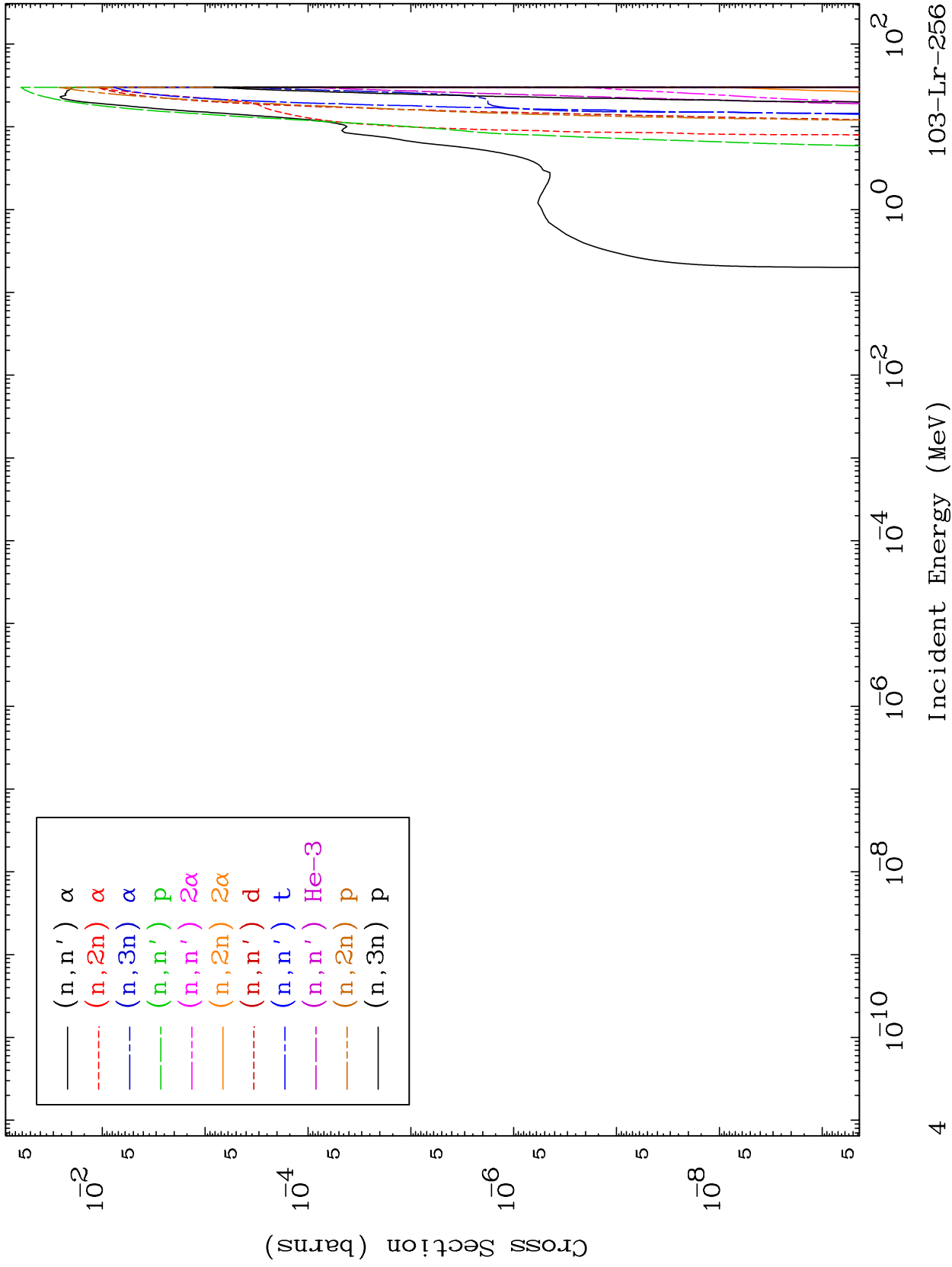
103-Lr-256



MAT 9985

Charged Particle
293 Kelvin Cross Sections

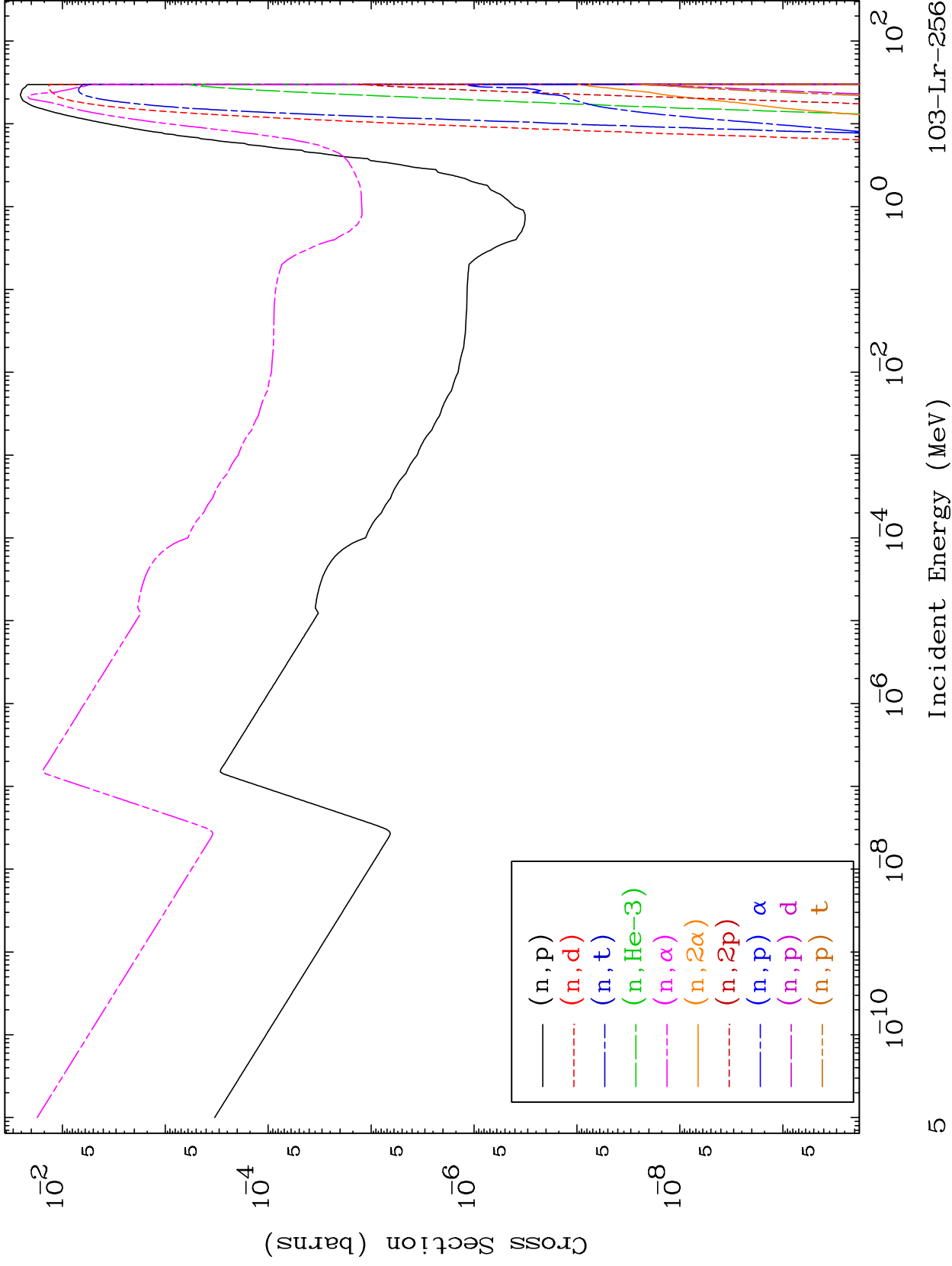
103-Lr-256



MAT 9985

Charged Particle
293 Kelvin Cross Sections

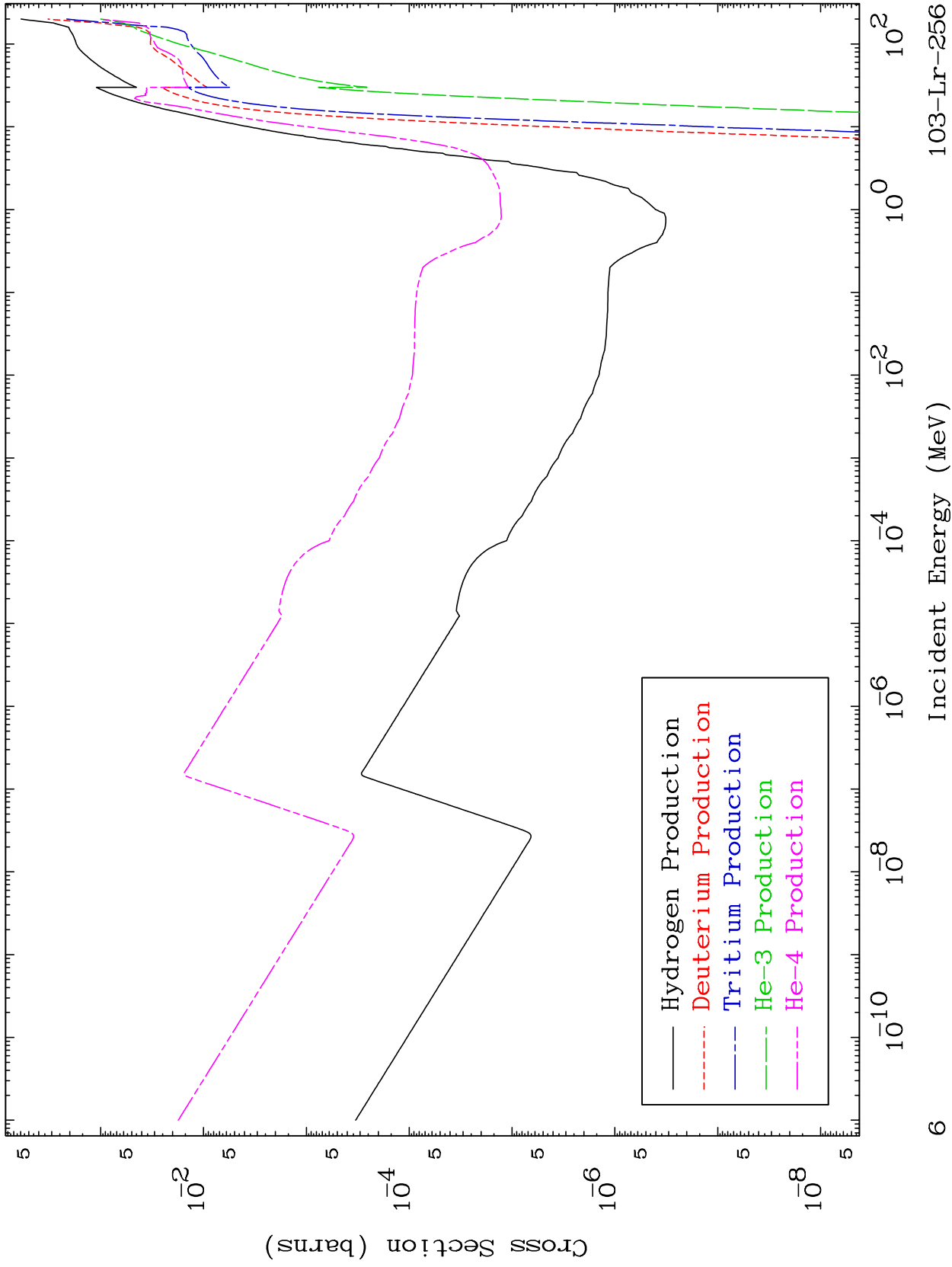
103-Lr-256



MAT 9985

Particle Production
293 Kelvin Cross Sections

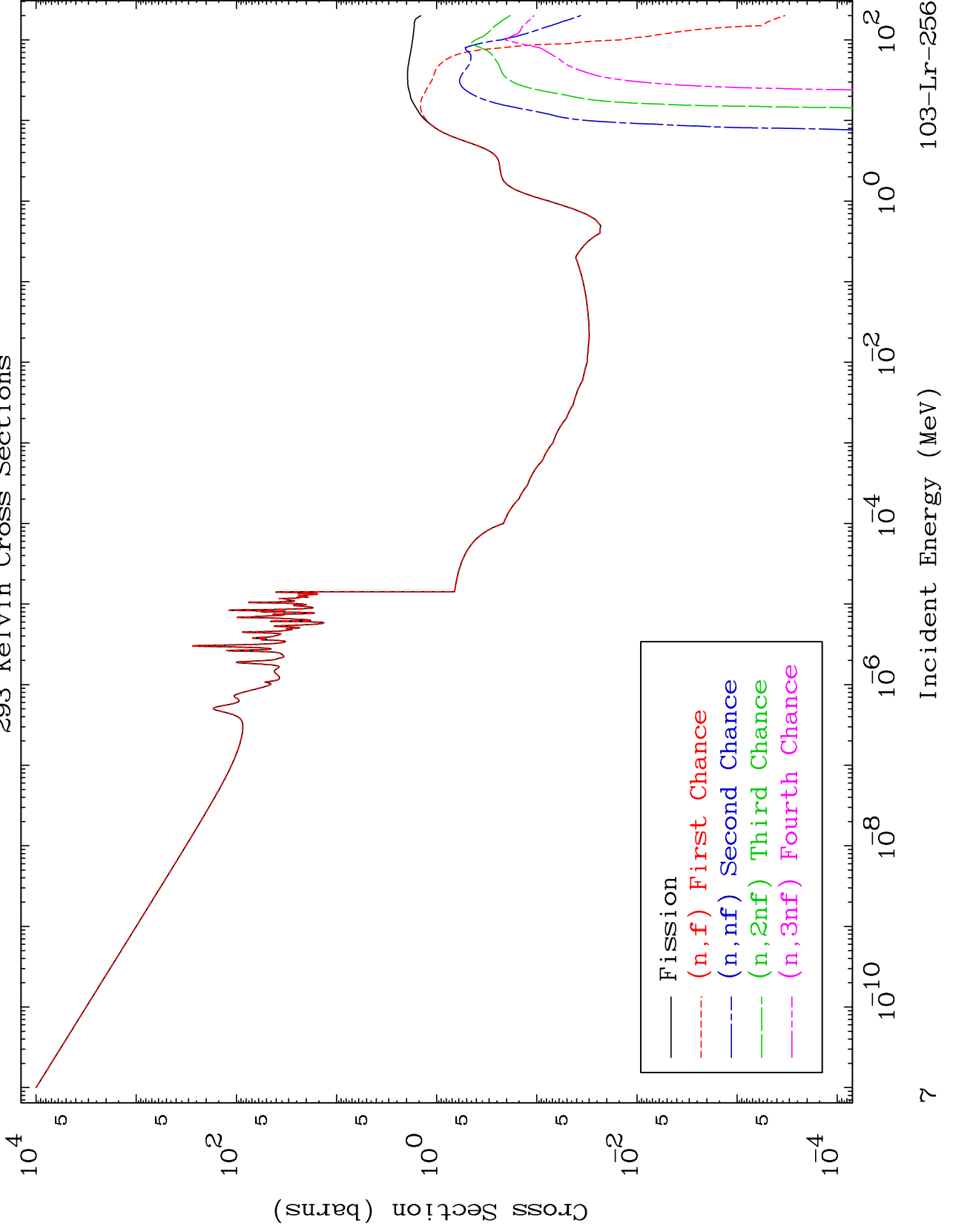
103-Lr-256



MAT 9985

Fission
293 Kelvin Cross Sections

103-Lr-256

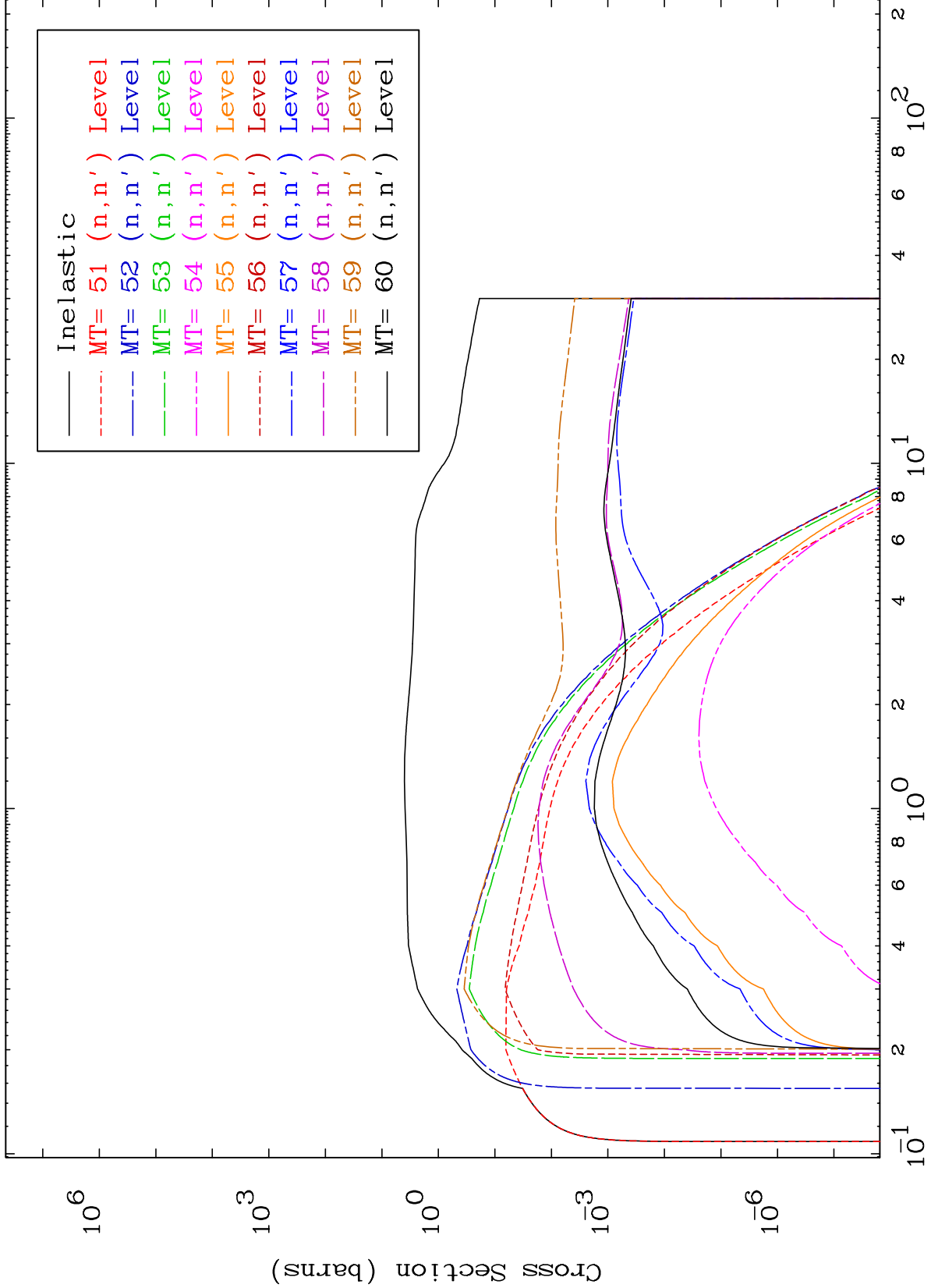


MAT 9985

(n,n') Level

103-Lr-256

293 Kelvin Cross Sections



Incident Energy (MeV)

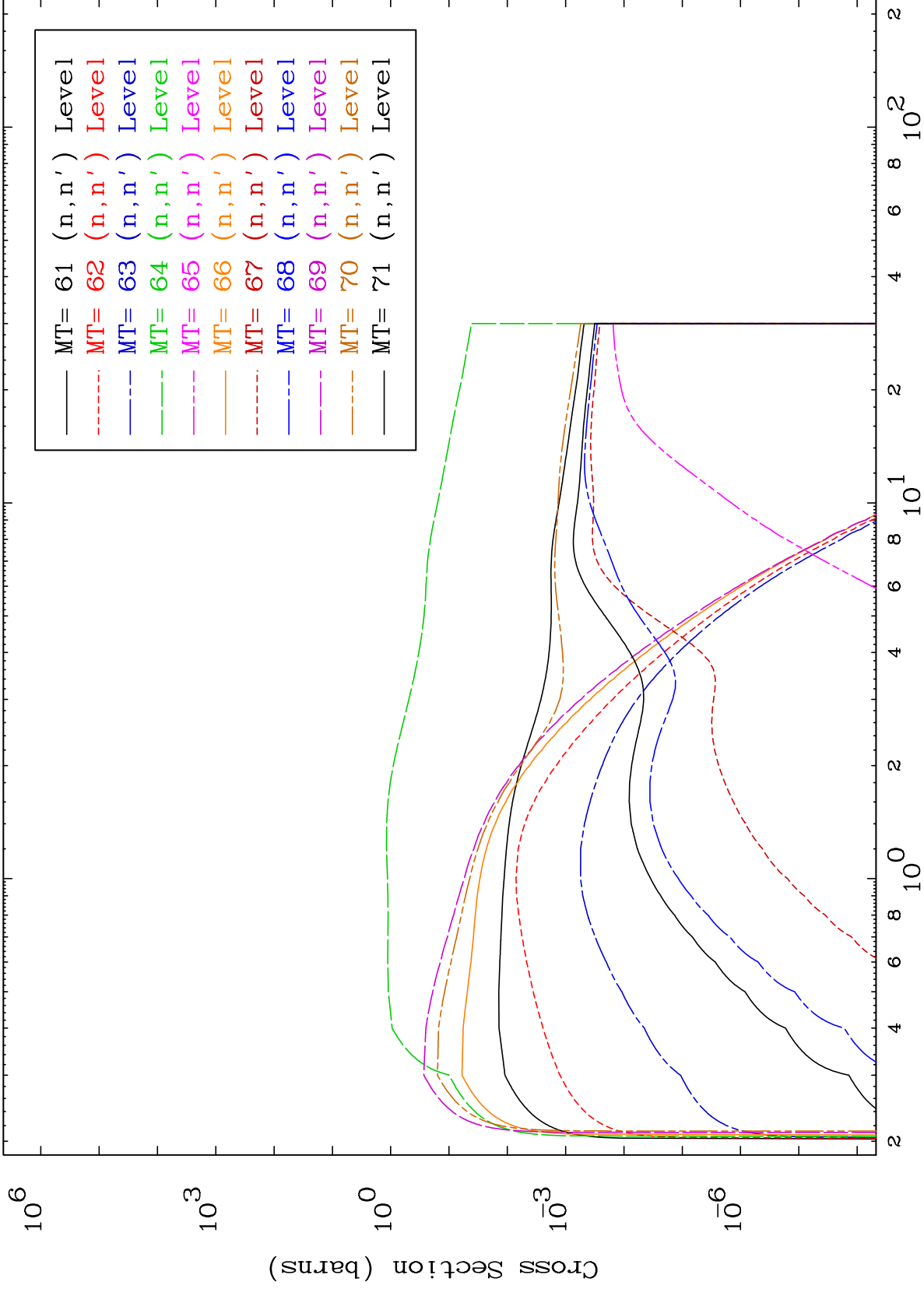
103-Lr-256

MAT 9985

(n,n') Level

293 Kelvin Cross Sections

103-Lr-256



9

Incident Energy (MeV)

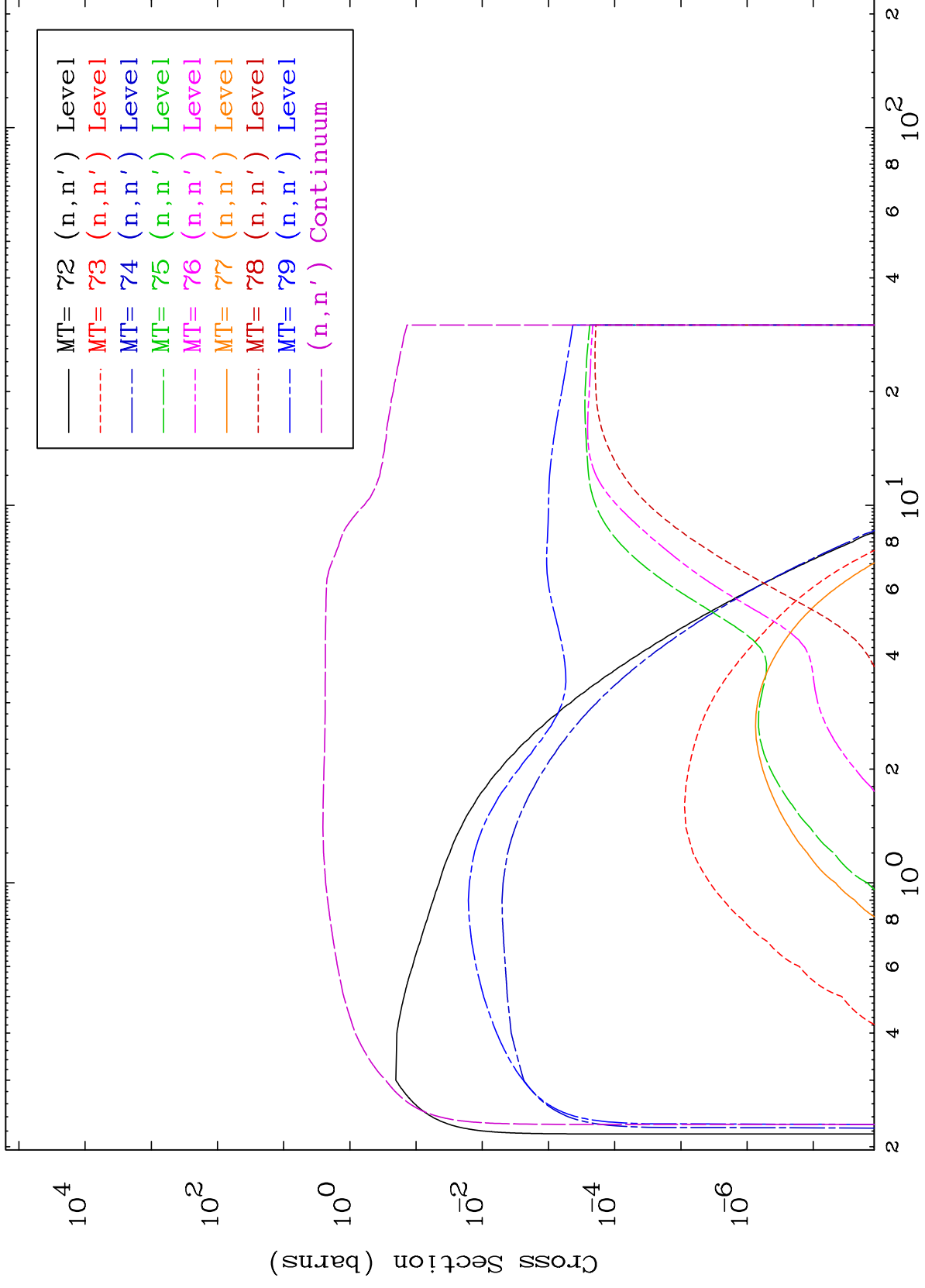
103-Lr-256

MAT 9985

(n,n') Level

103-Lr-256

293 Kelvin Cross Sections



10

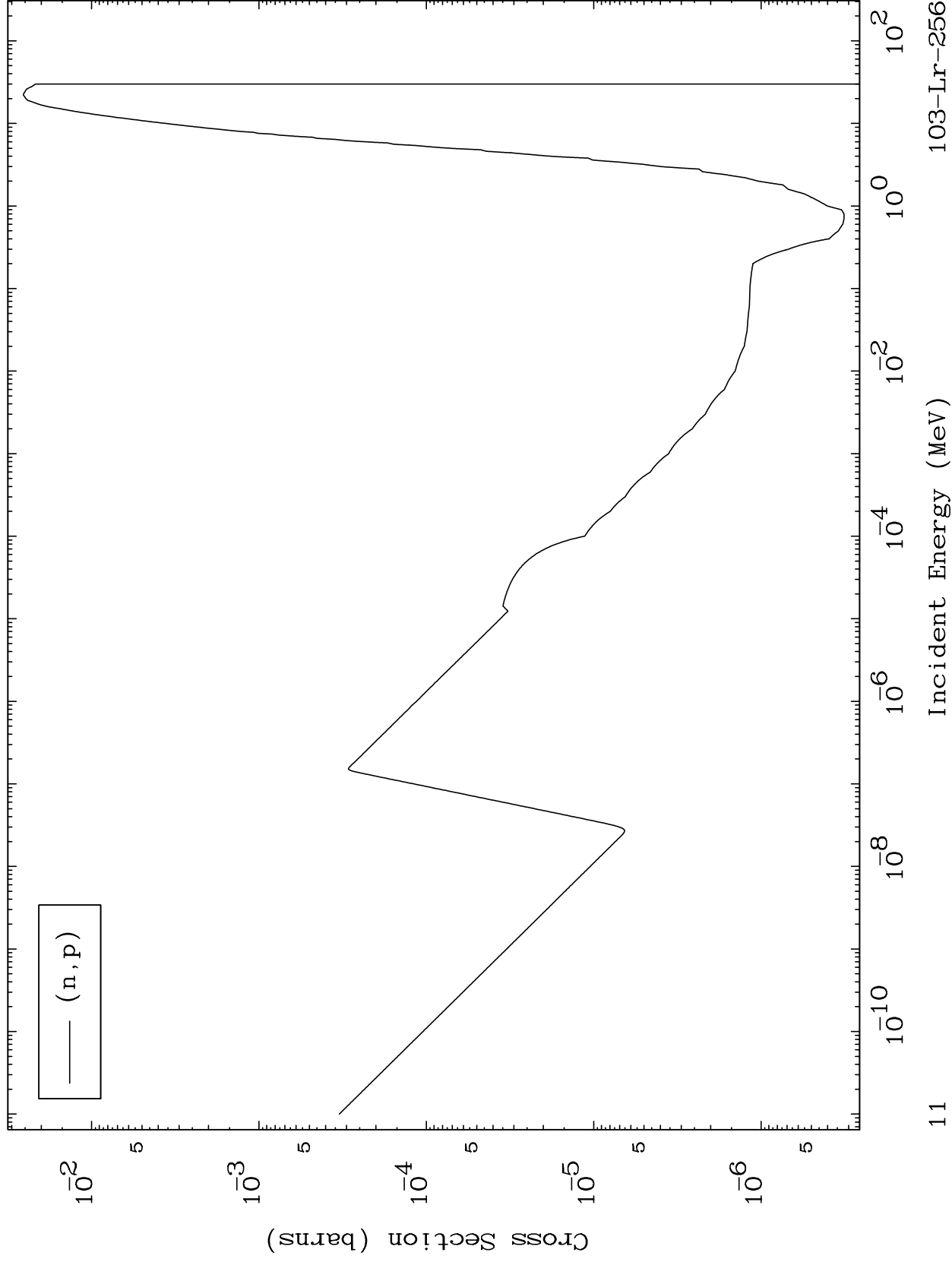
Incident Energy (MeV)

103-Lr-256

MAT 9985

(n,p) Levels
293 Kelvin Cross Sections

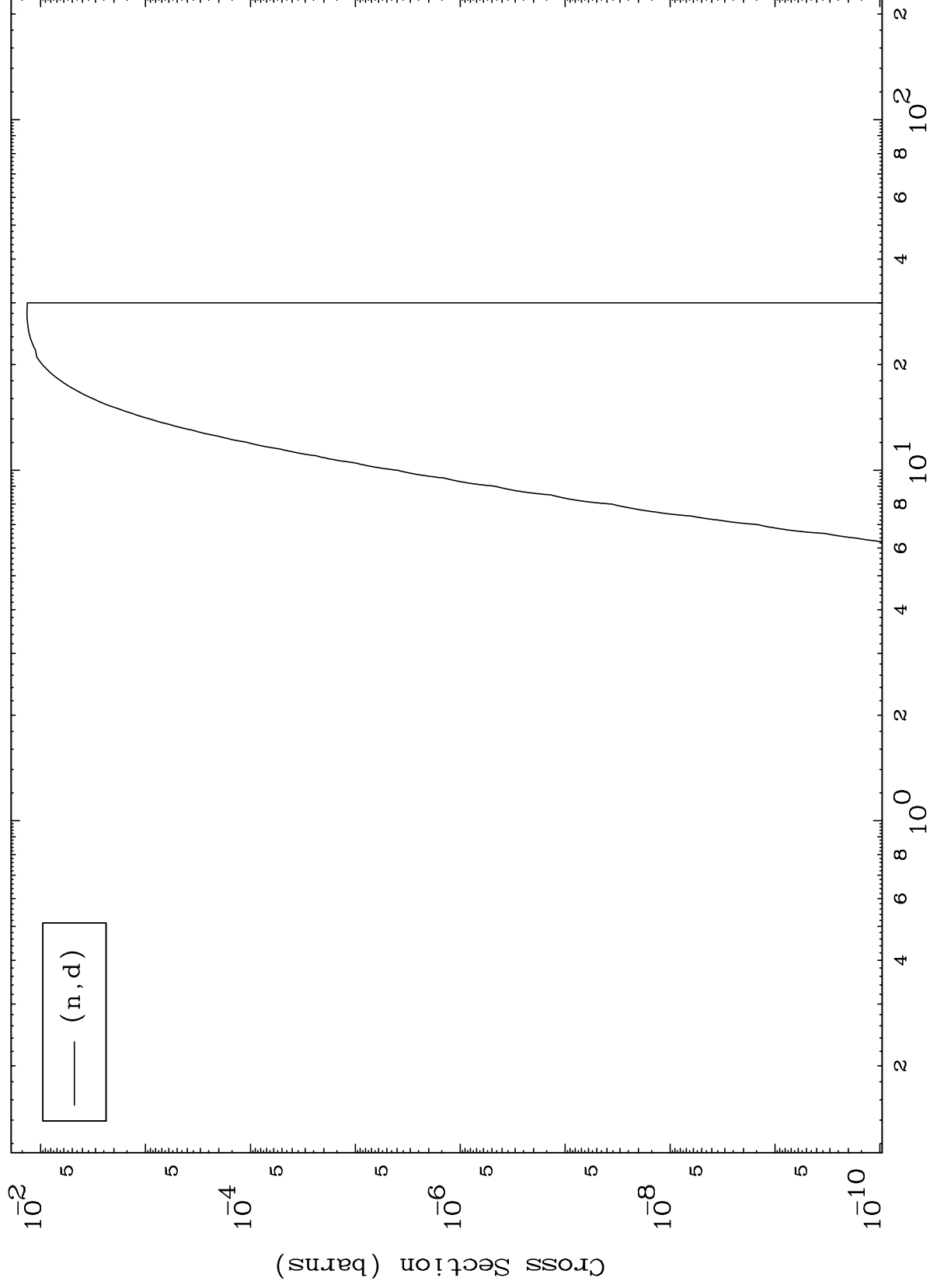
103-Lr-256



MAT 9985

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-256



12

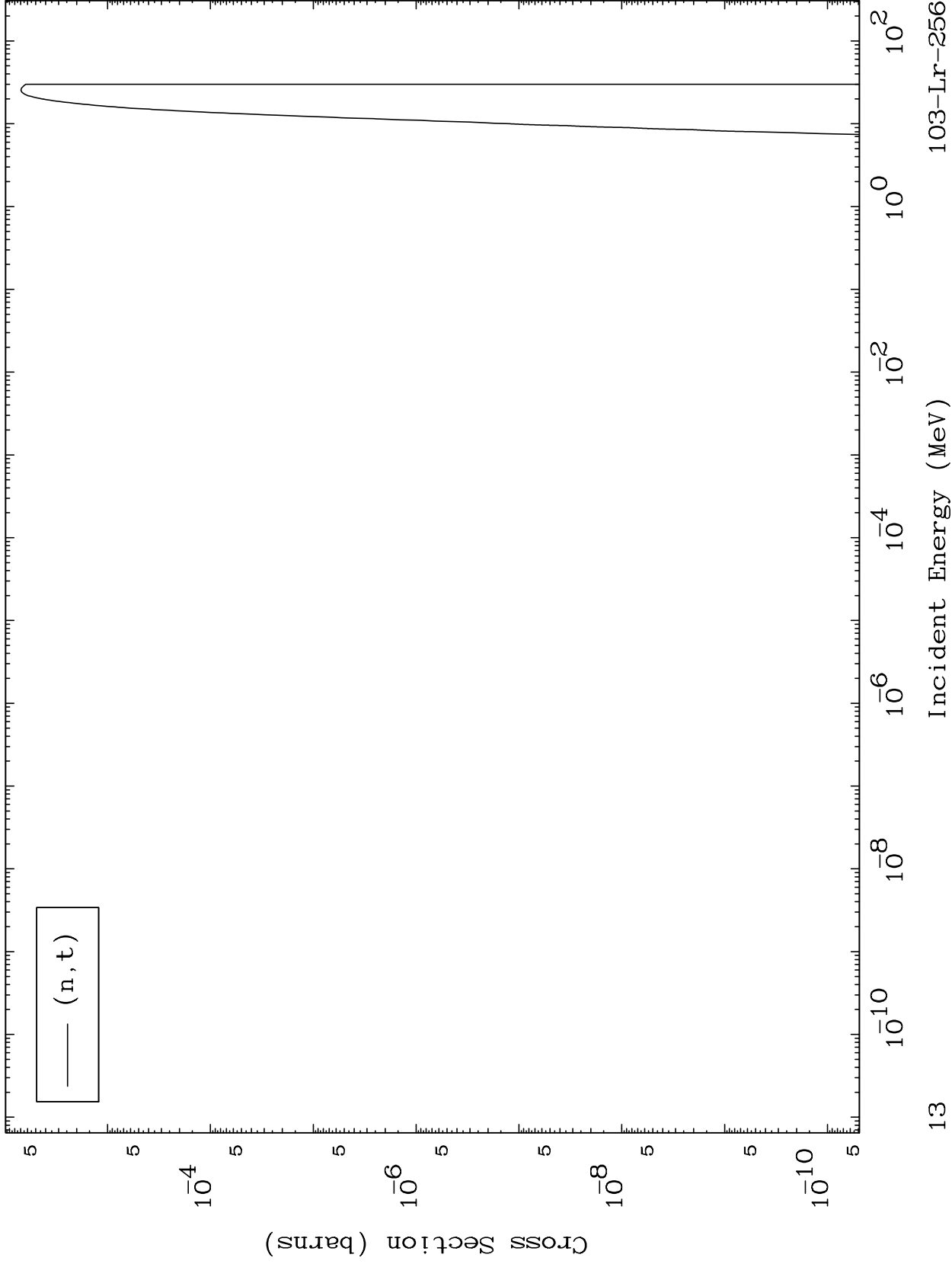
Incident Energy (MeV)

103-Lr-256

MAT 9985

(n,t) Levels
293 Kelvin Cross Sections

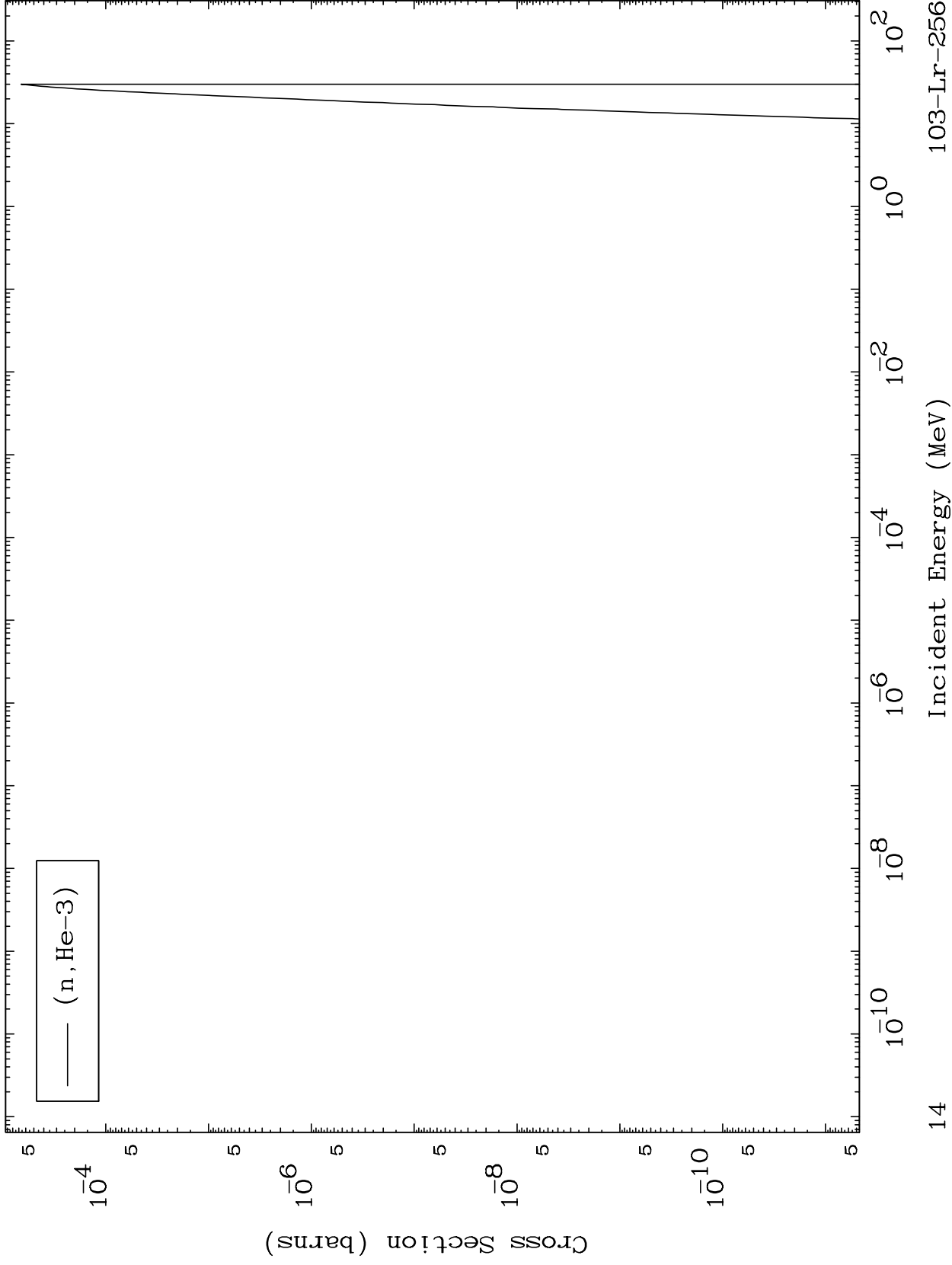
103-Lr-256



MAT 9985

(n,He3) Levels
293 Kelvin Cross Sections

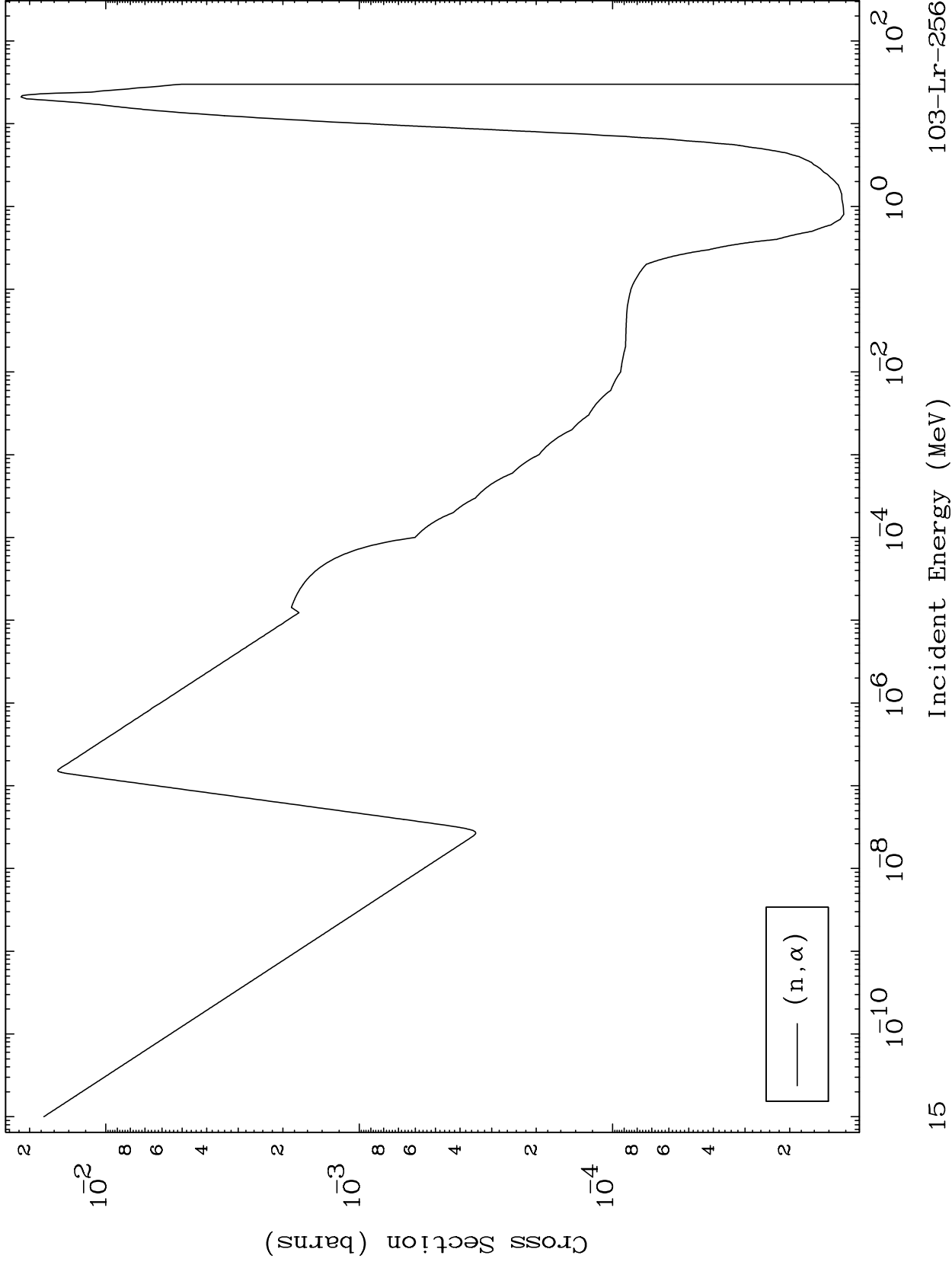
103-Lr-256



MAT 9985

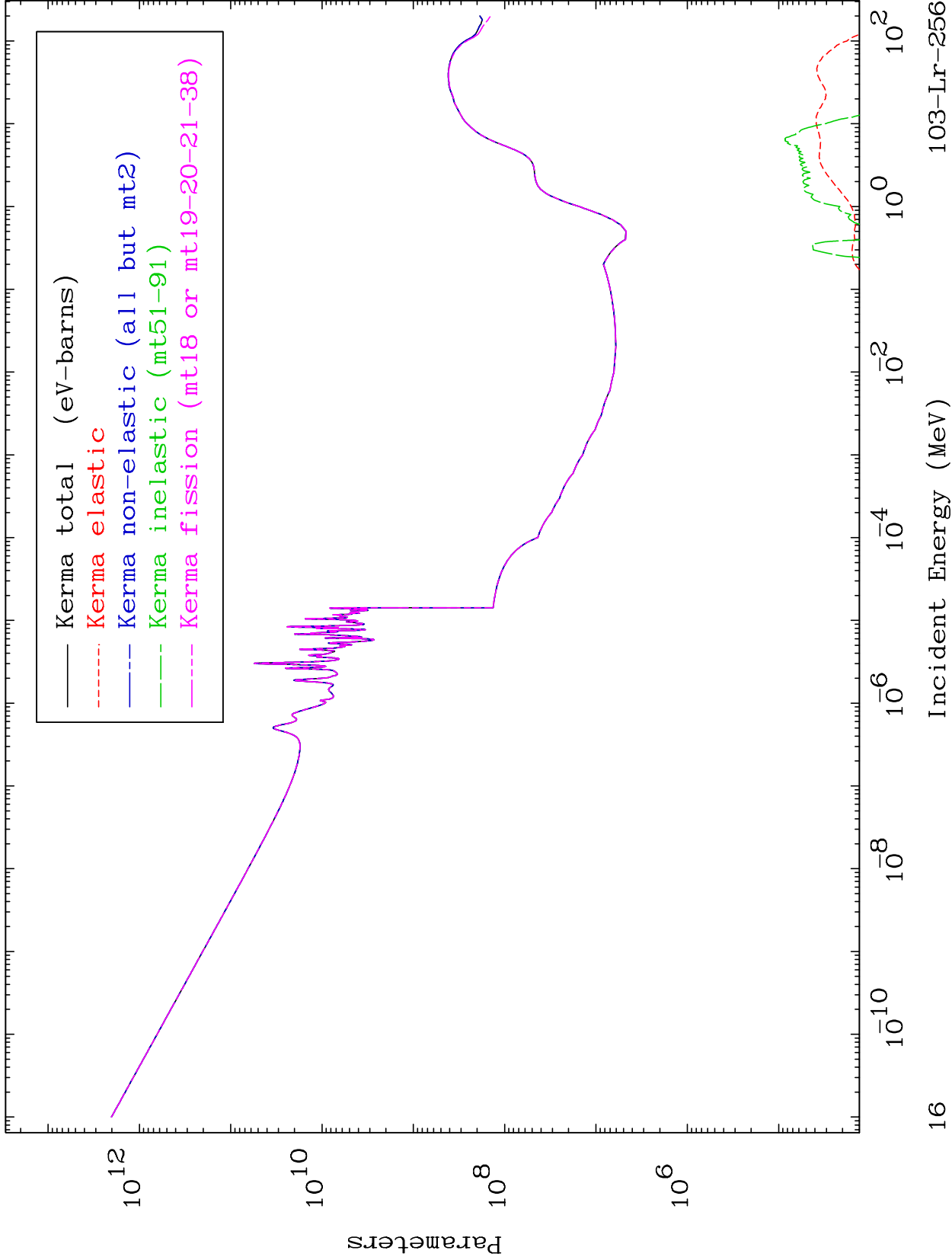
(n,α) Levels
293 Kelvin Cross Sections

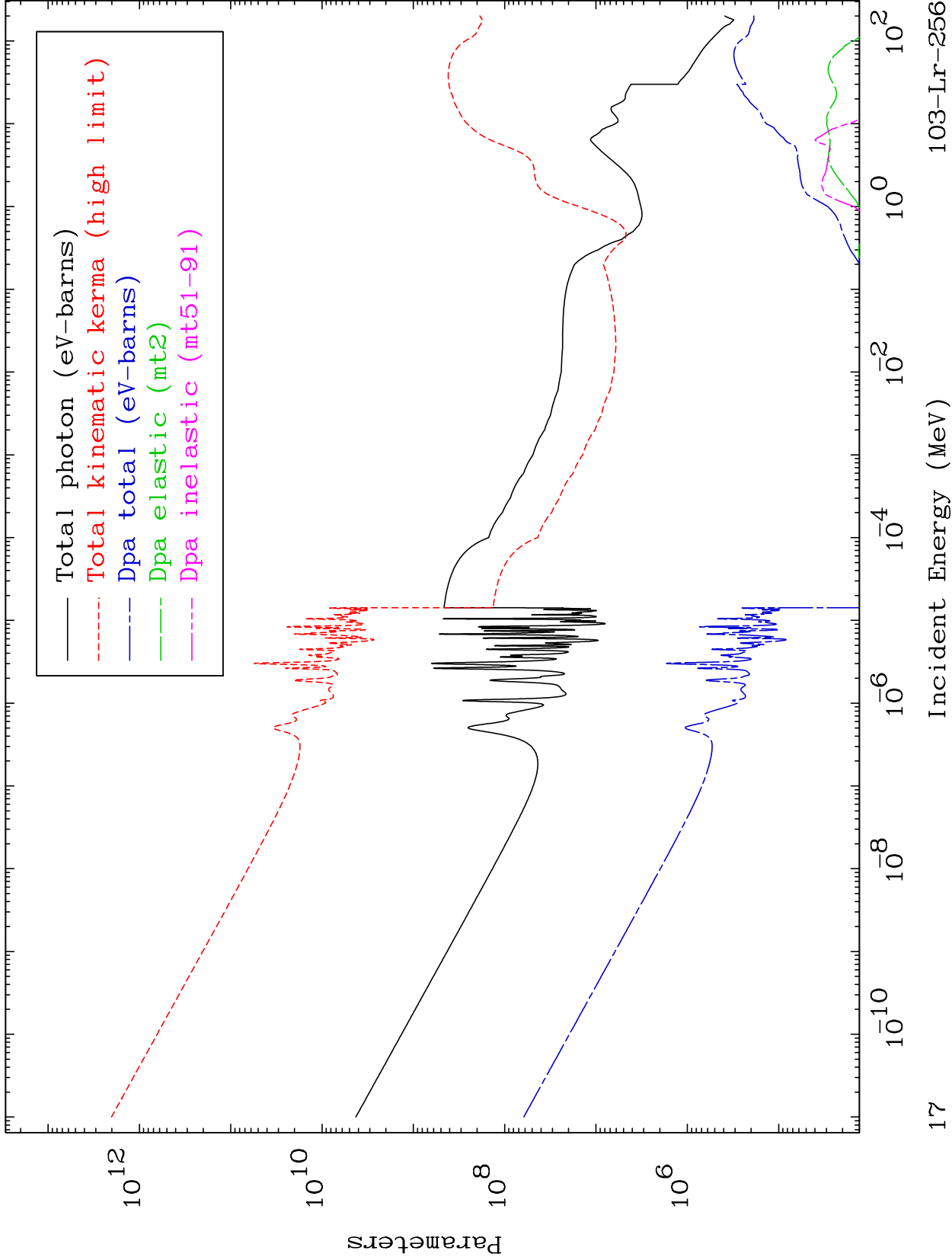
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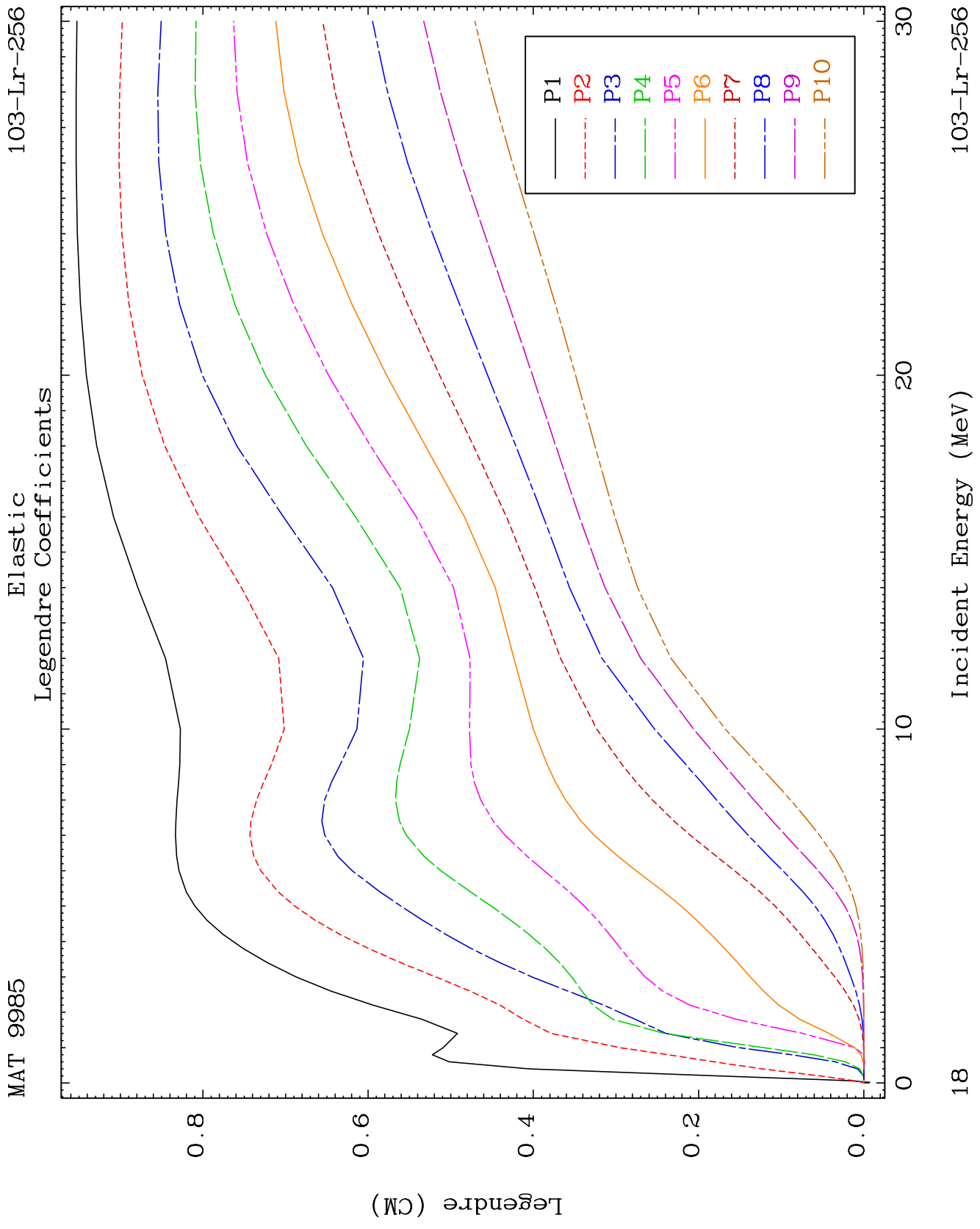


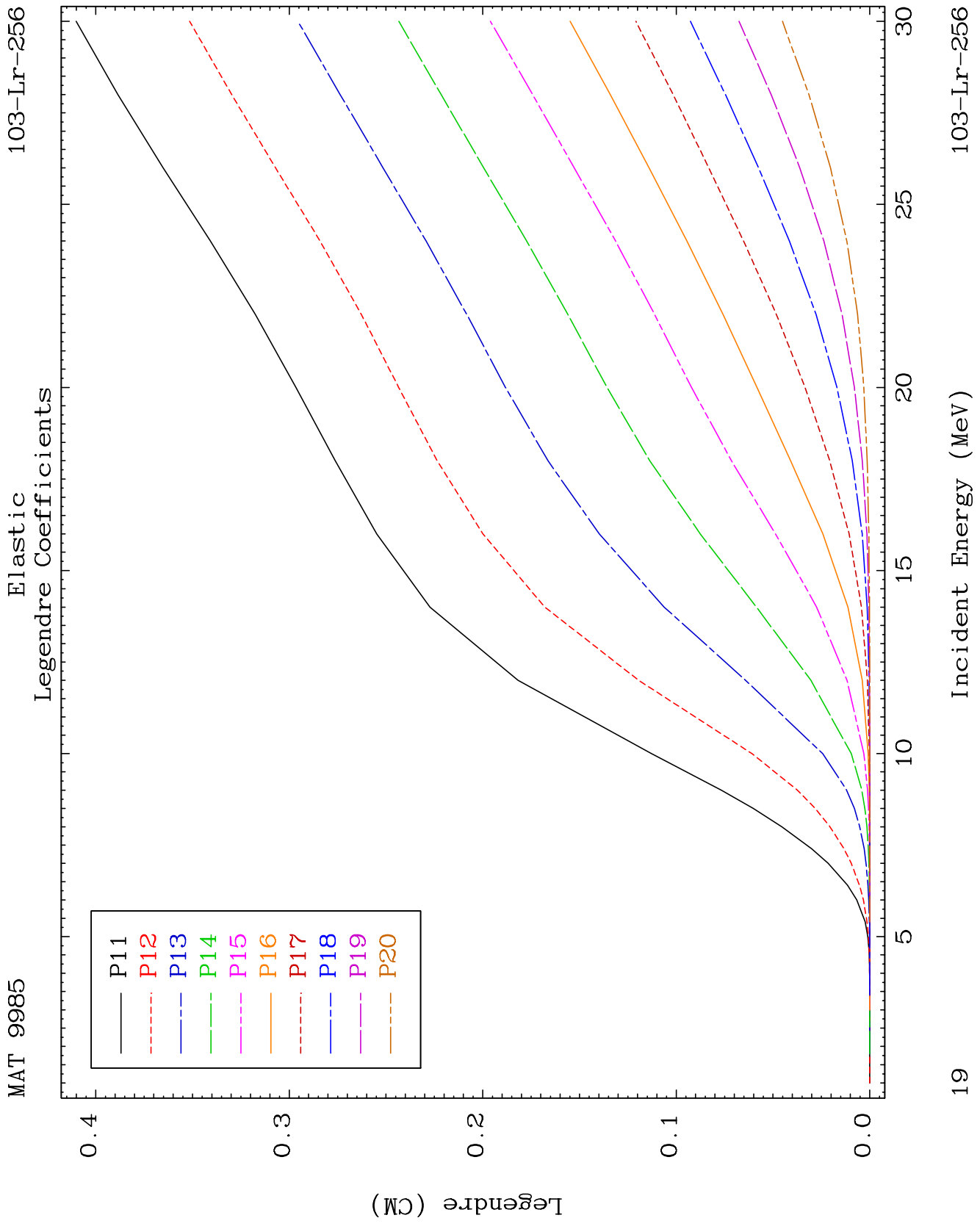
15

103-Lr-256





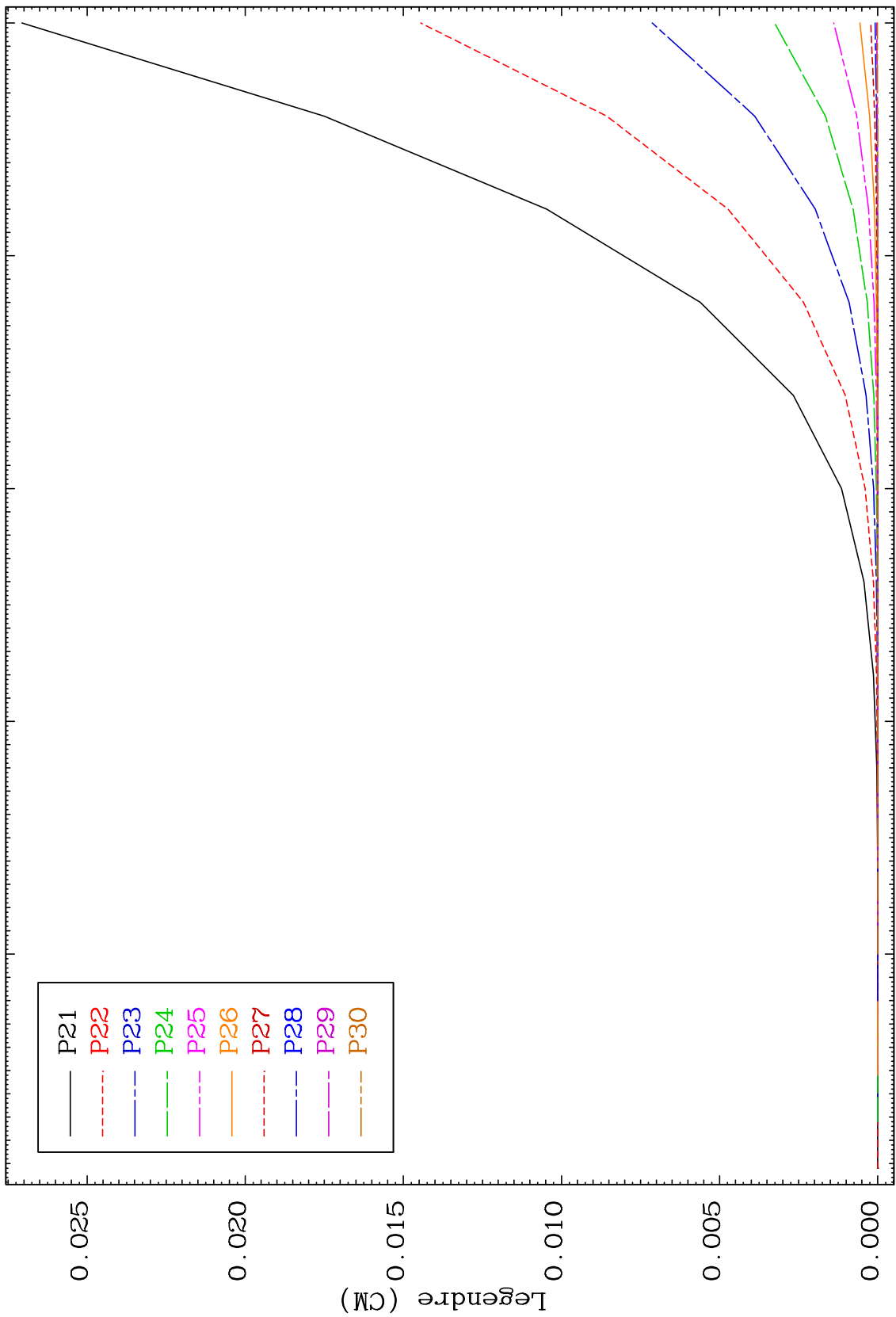




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Elastic Legendre Coefficients

103-Lr-256



20

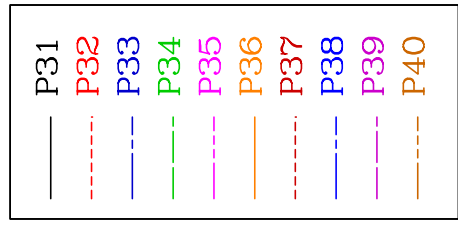
Incident Energy (MeV)

103-Lr-256

MAT 9985

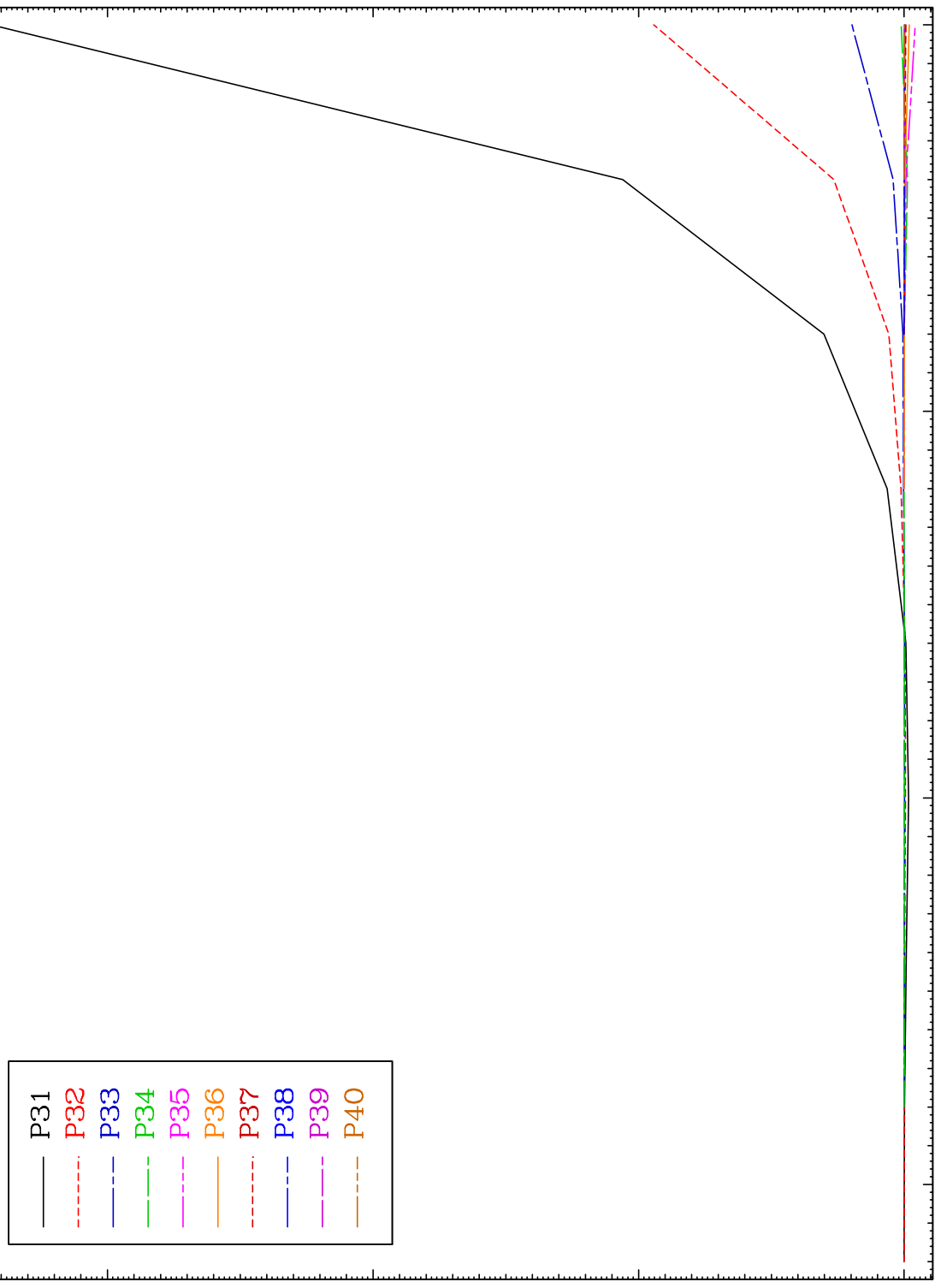
Elastic Legendre Coefficients

103-Lr-256



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

21

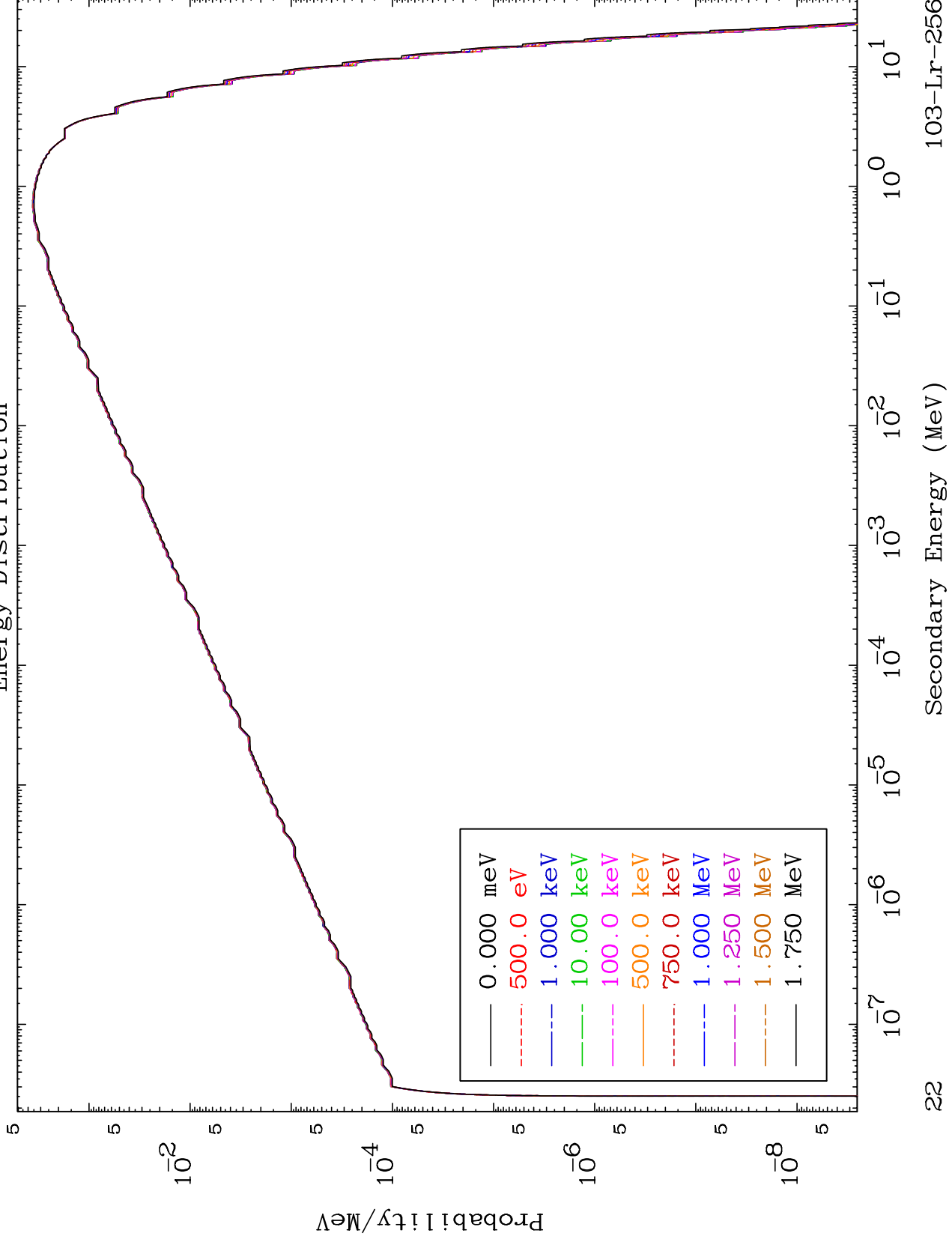
Incident Energy (MeV)

103-Lr-256

MAT 9985

Fission Energy Distribution

103-Lr-256

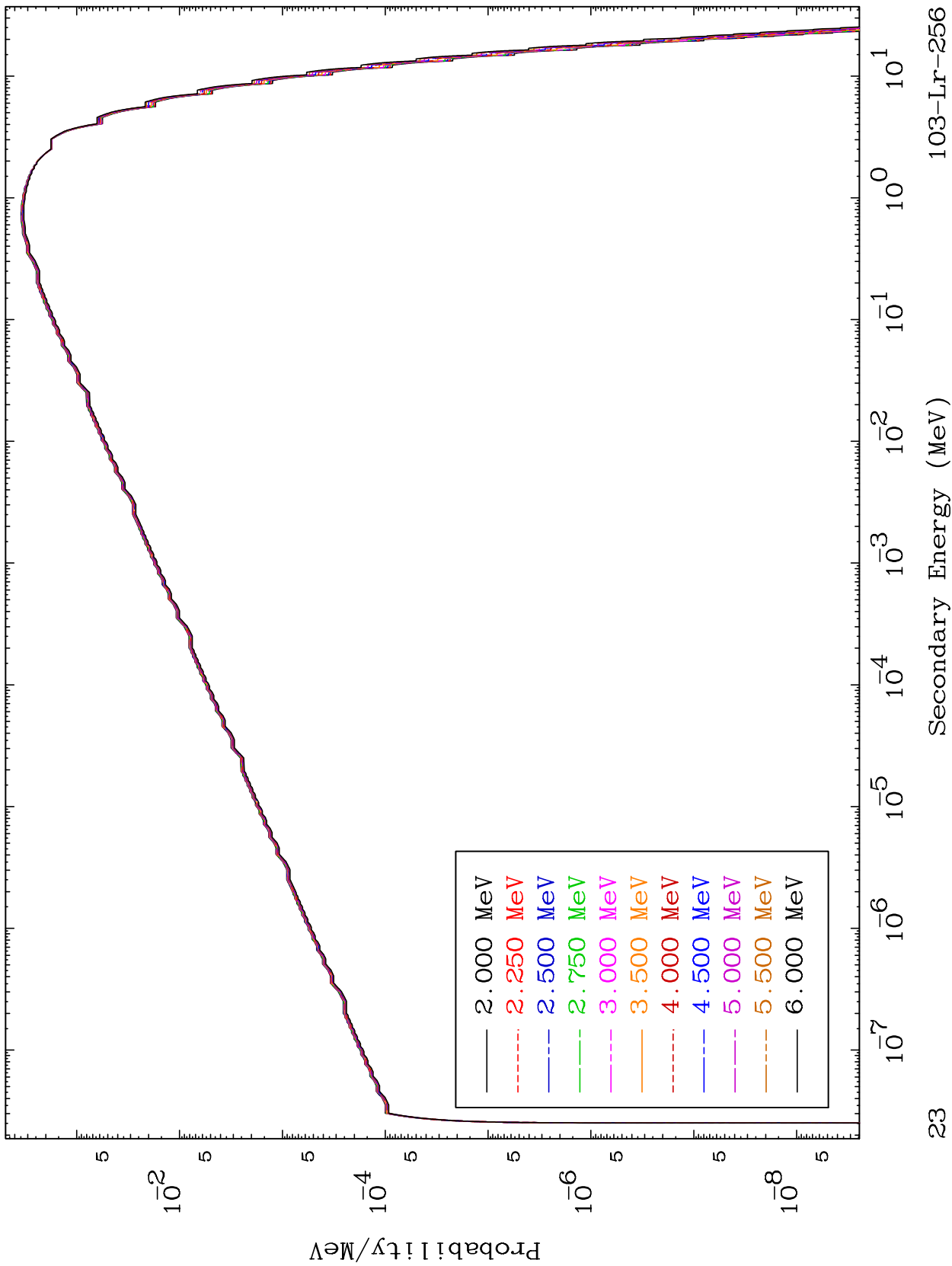


103-Lr-256

MAT 9985

Fission Energy Distribution

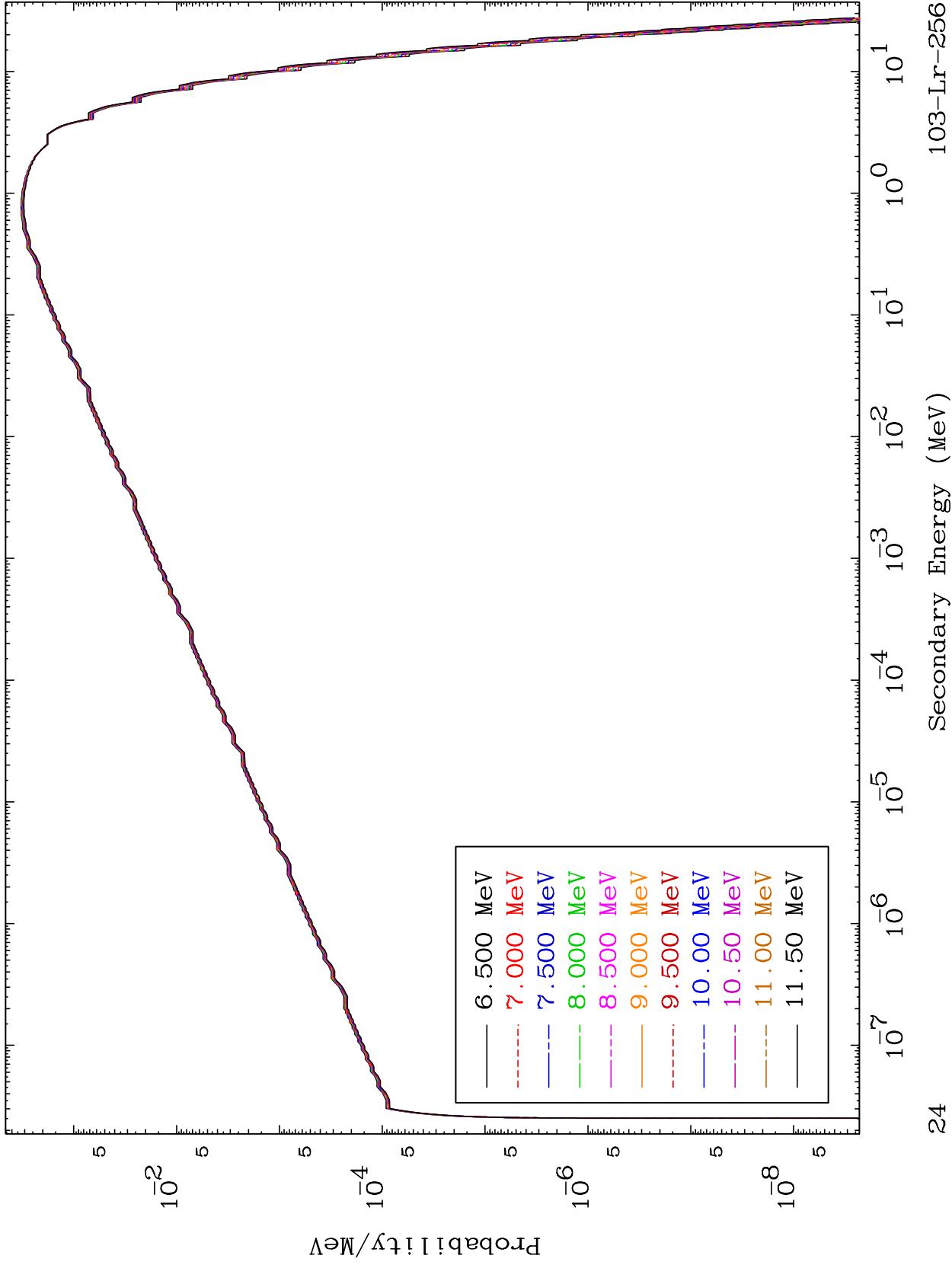
103-Lr-256



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Fission
Energy Distribution

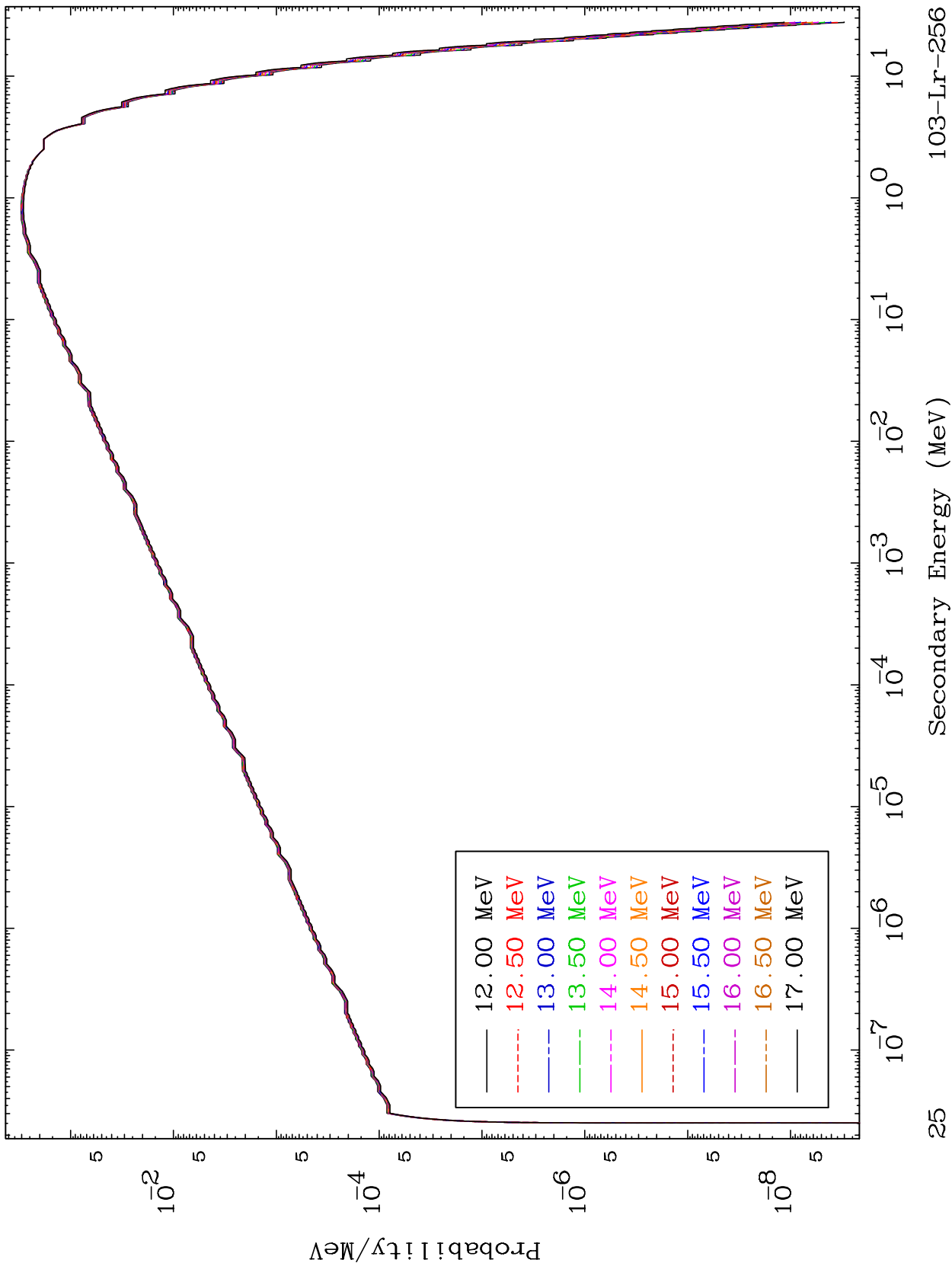
103-Lr-256



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Fission Energy Distribution

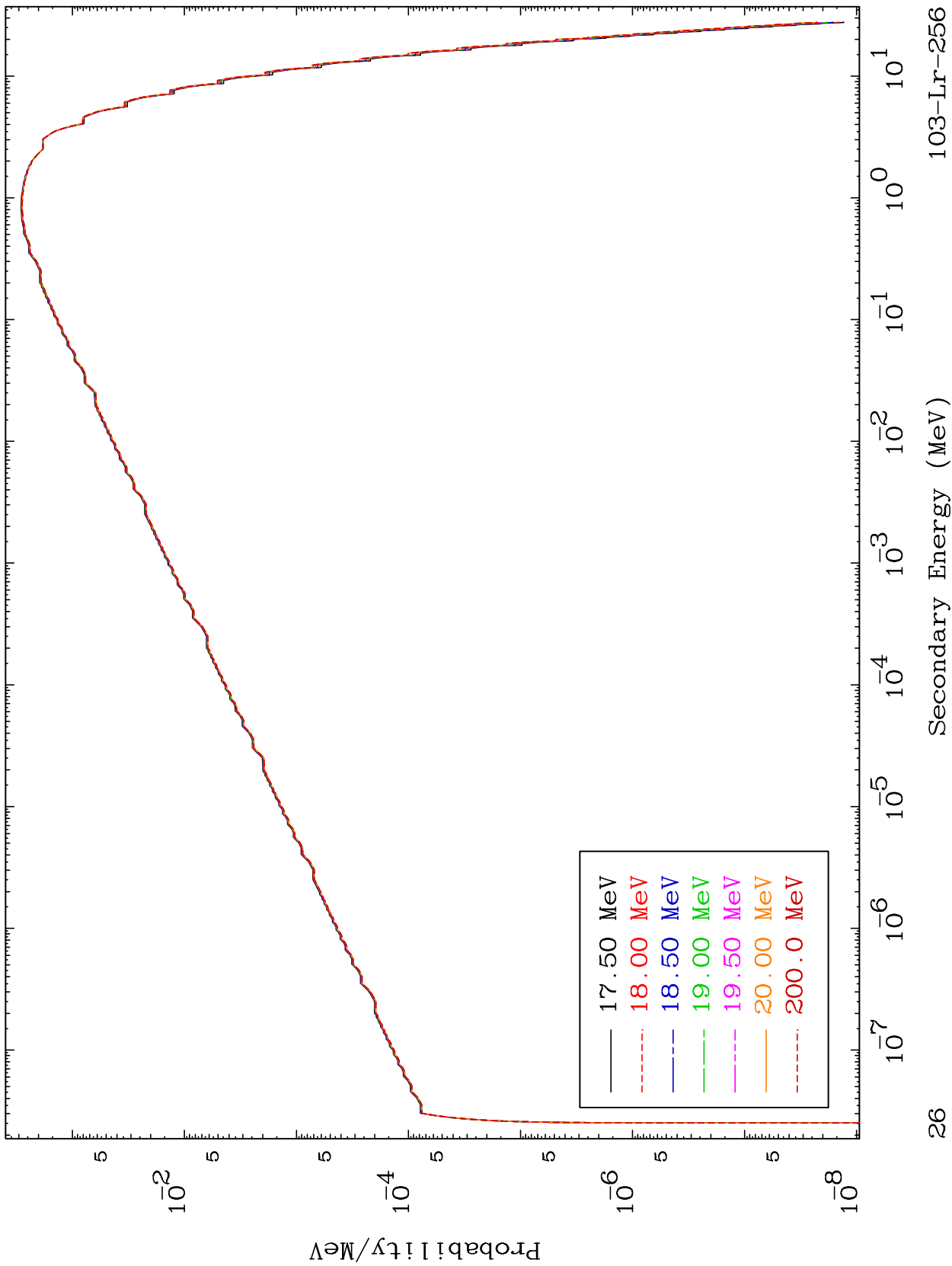
103-Lr-256



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Fission Energy Distribution

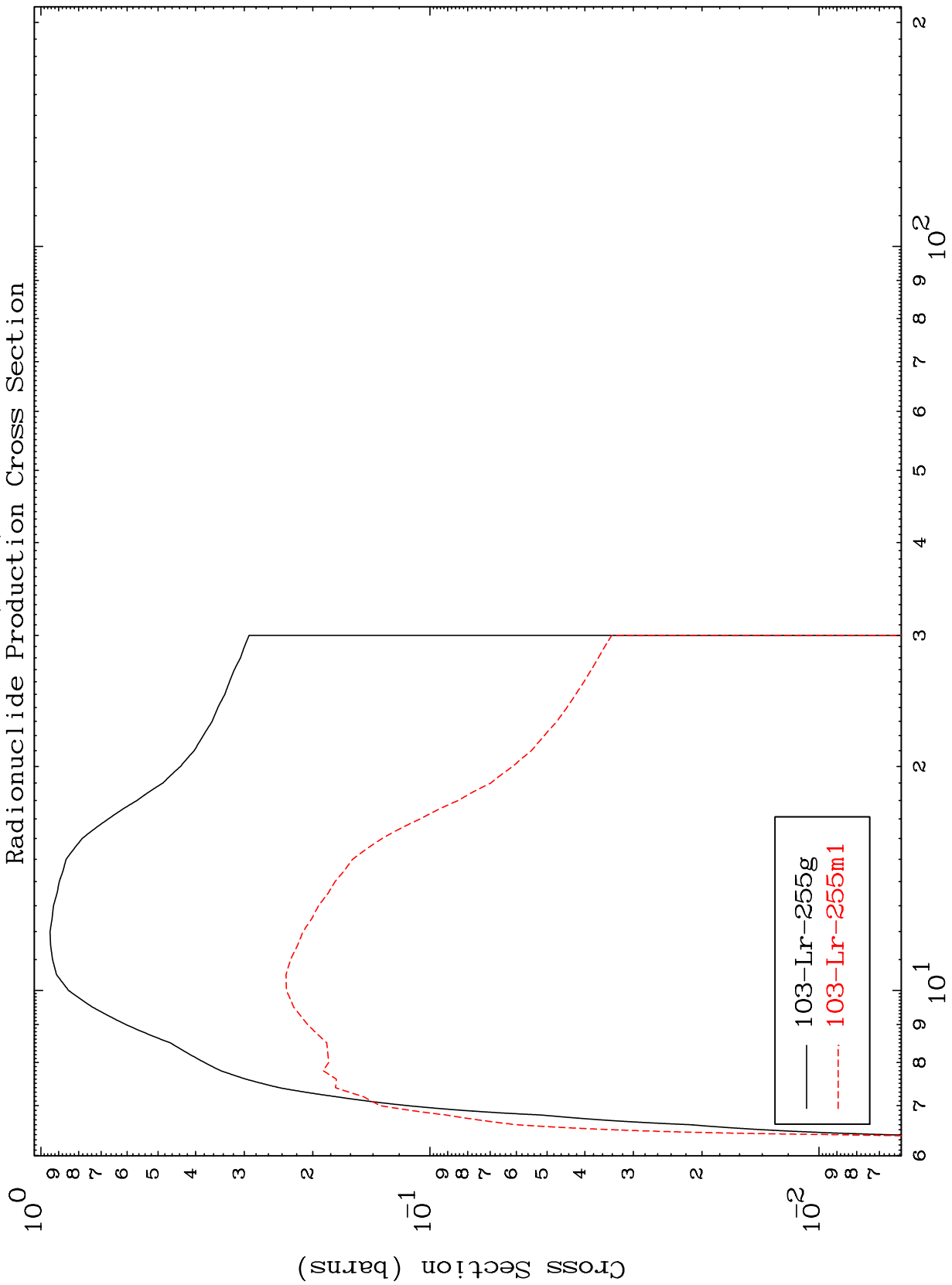
103-Lr-256



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103-Lr-256

(n,2n)
Radionuclide Production Cross Section



27

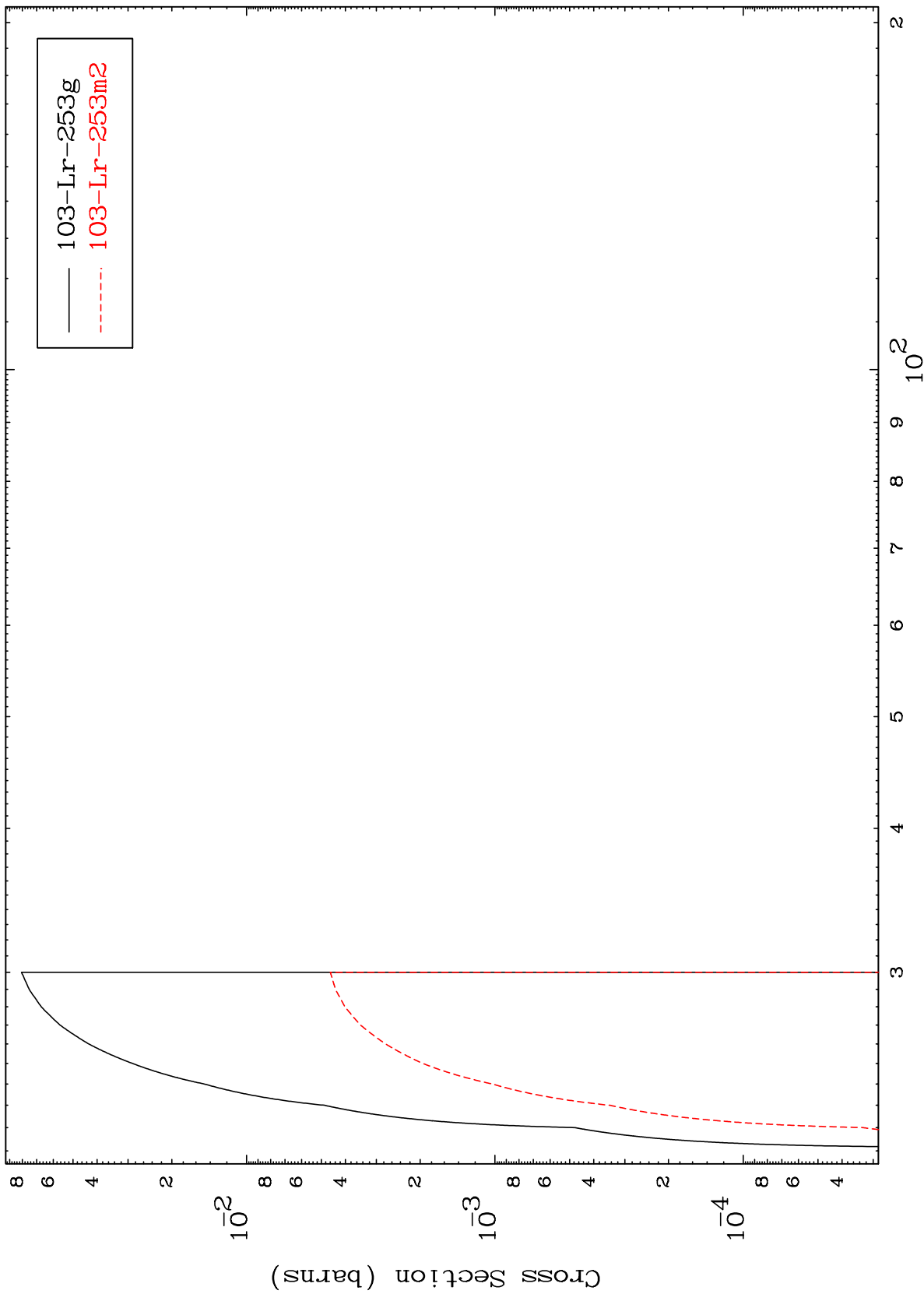
Incident Energy (MeV)

103-Lr-256

MAT 9985

103-Lr-256

(n,4n)
Radionuclide Production Cross Section



28

103-Lr-256

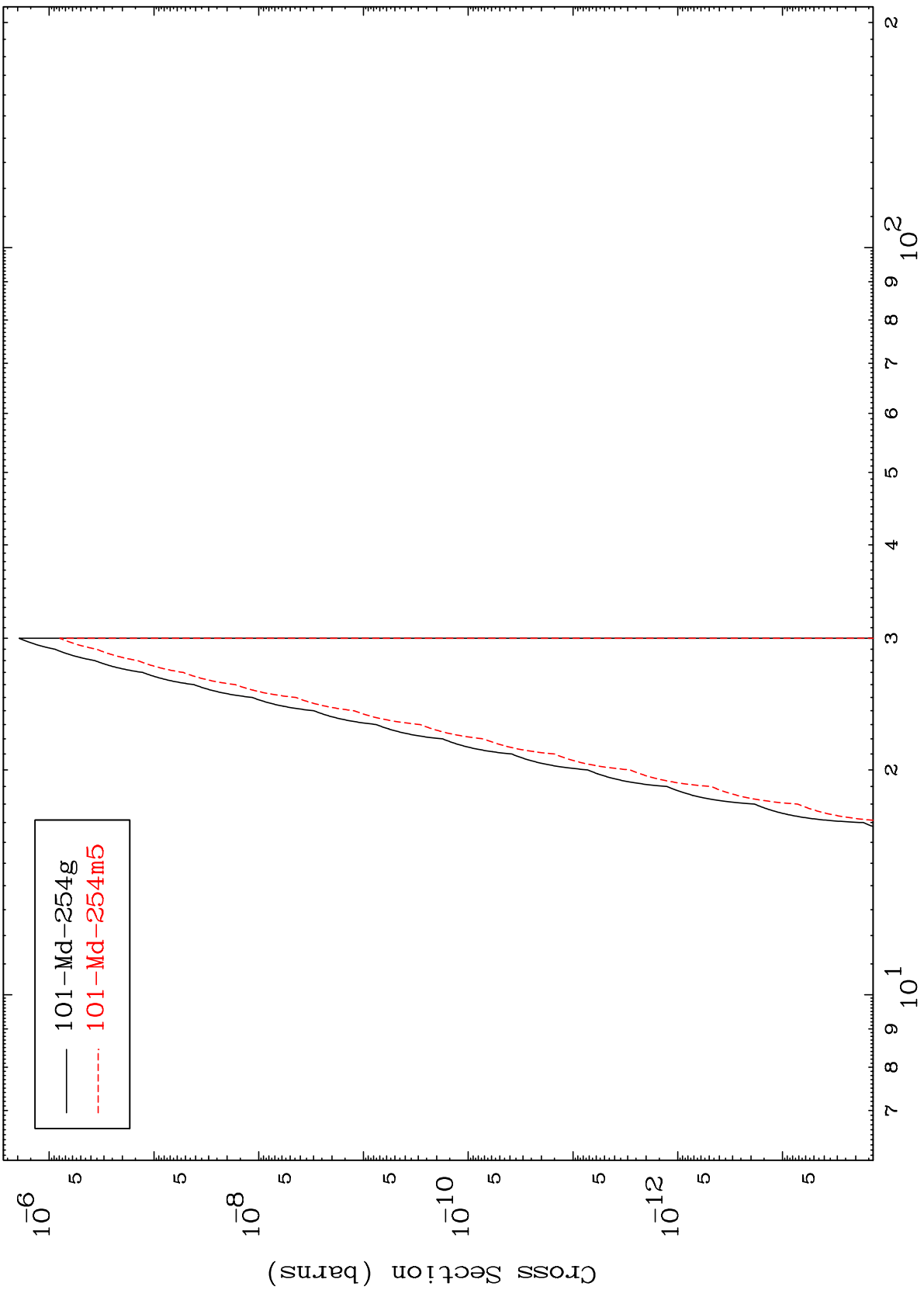
Incident Energy (MeV)

MAT 9985

(n,2n) p

103-Lr-256

Radionuclide Production Cross Section



29

Incident Energy (MeV)

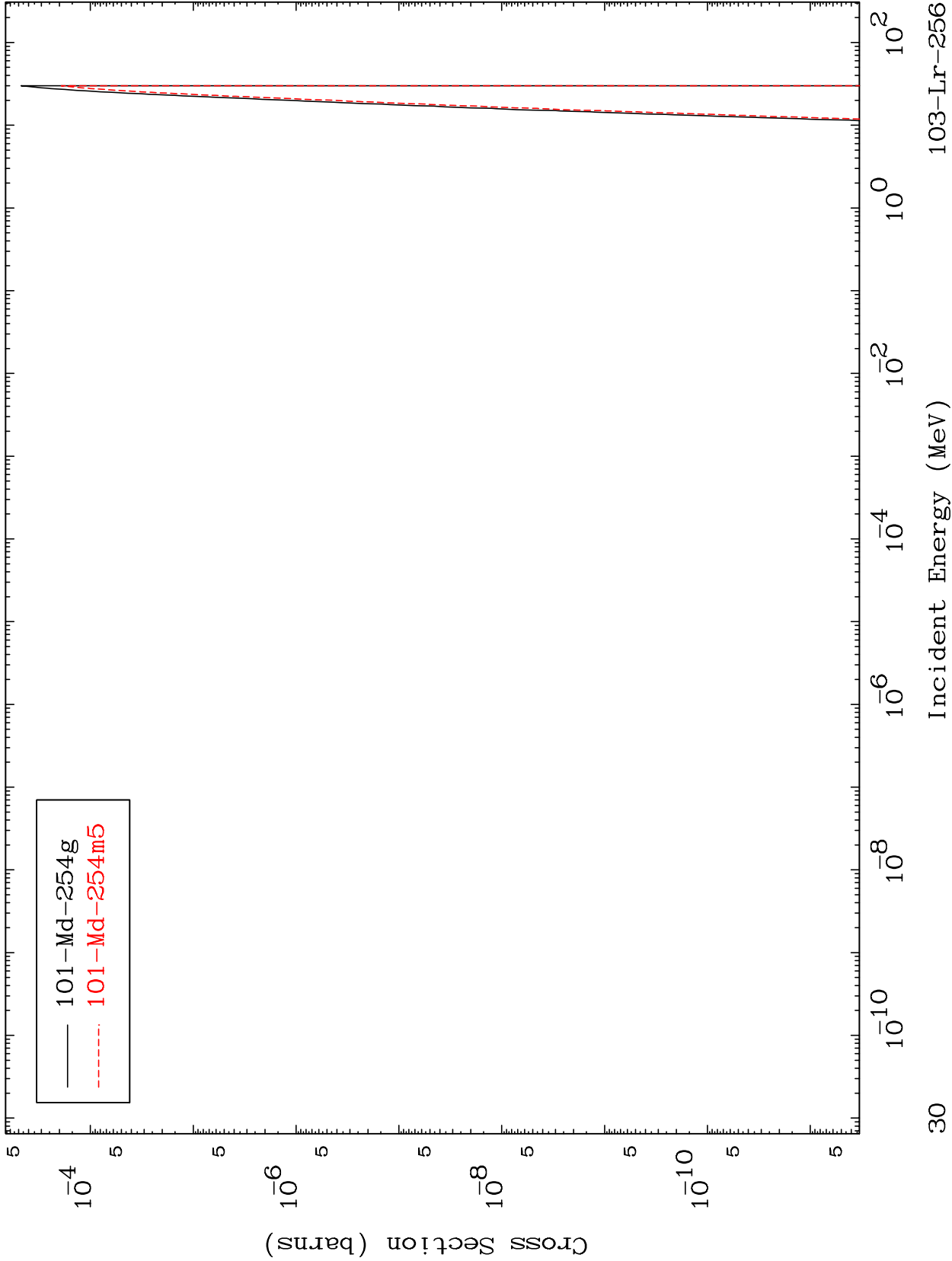
103-Lr-256

MAT 9985

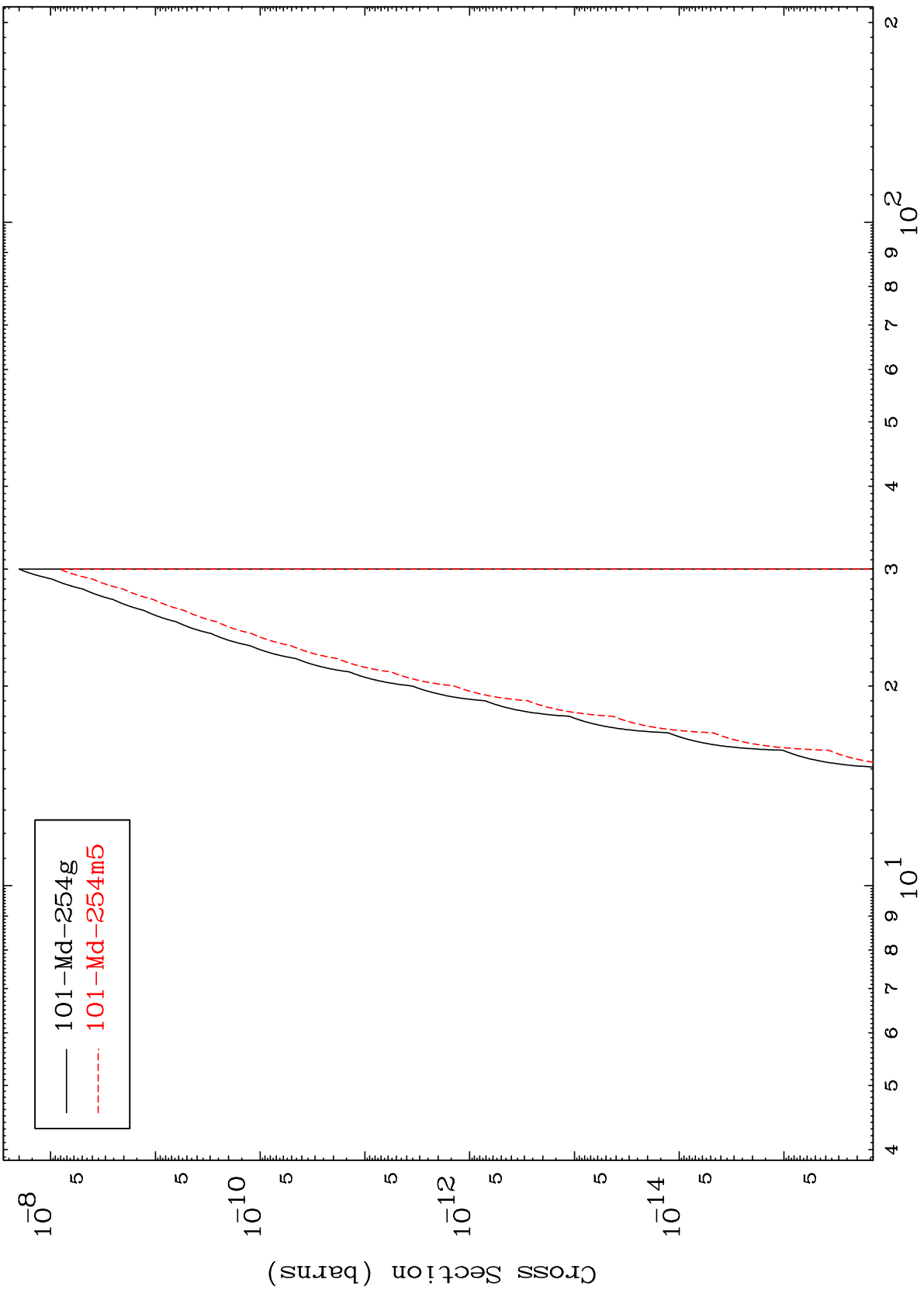
(n,He-3)

103-Lr-256

Radionuclide Production Cross Section



Radionuclide Production Cross Section



— 101-Md-254g
- - - 101-Md-254m5