

Program EVALPLOT  
(Version 2017-1)

by

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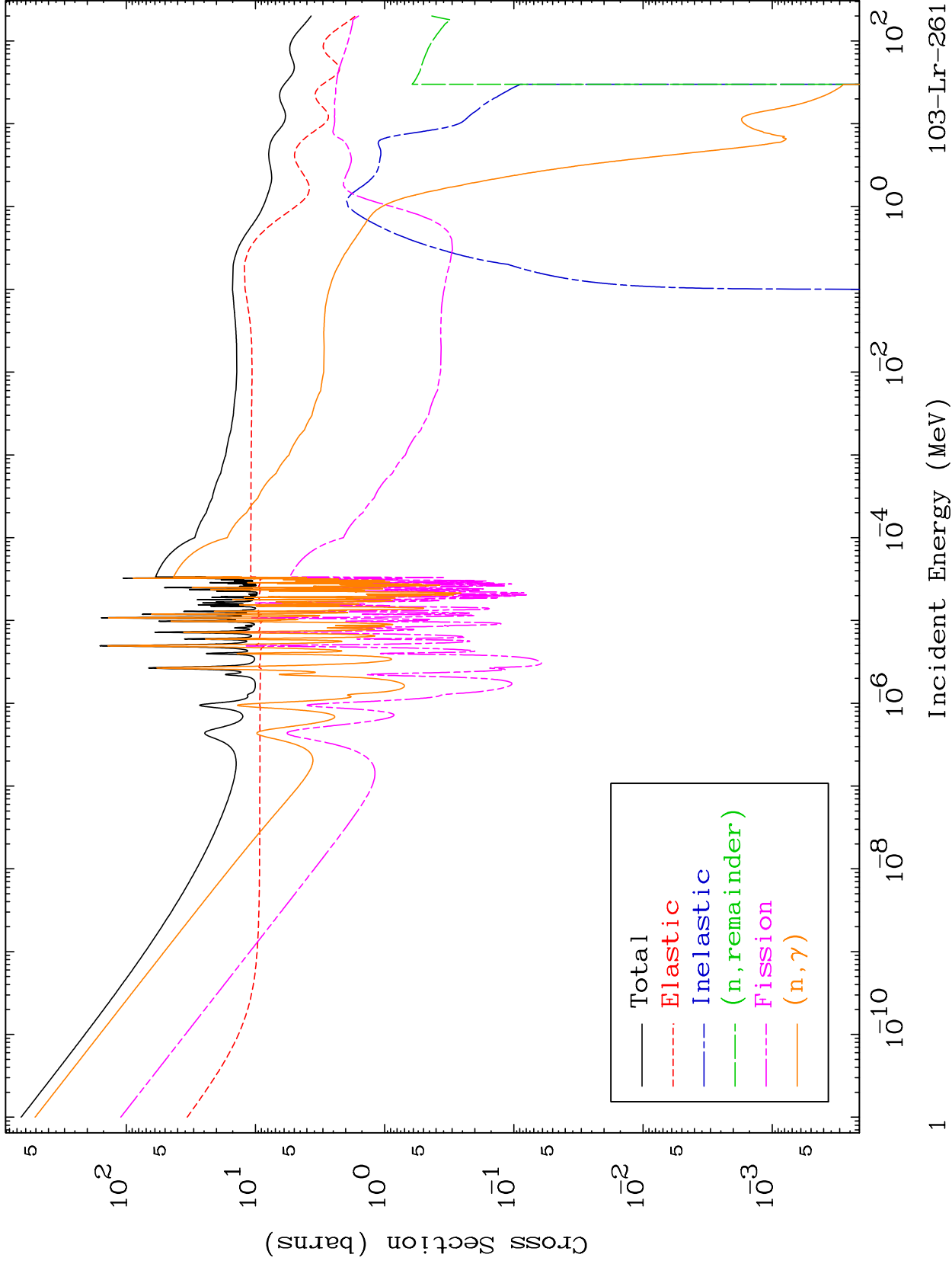
Press Mouse Button to Start

MAT 9990

Major

293 Kelvin Cross Sections

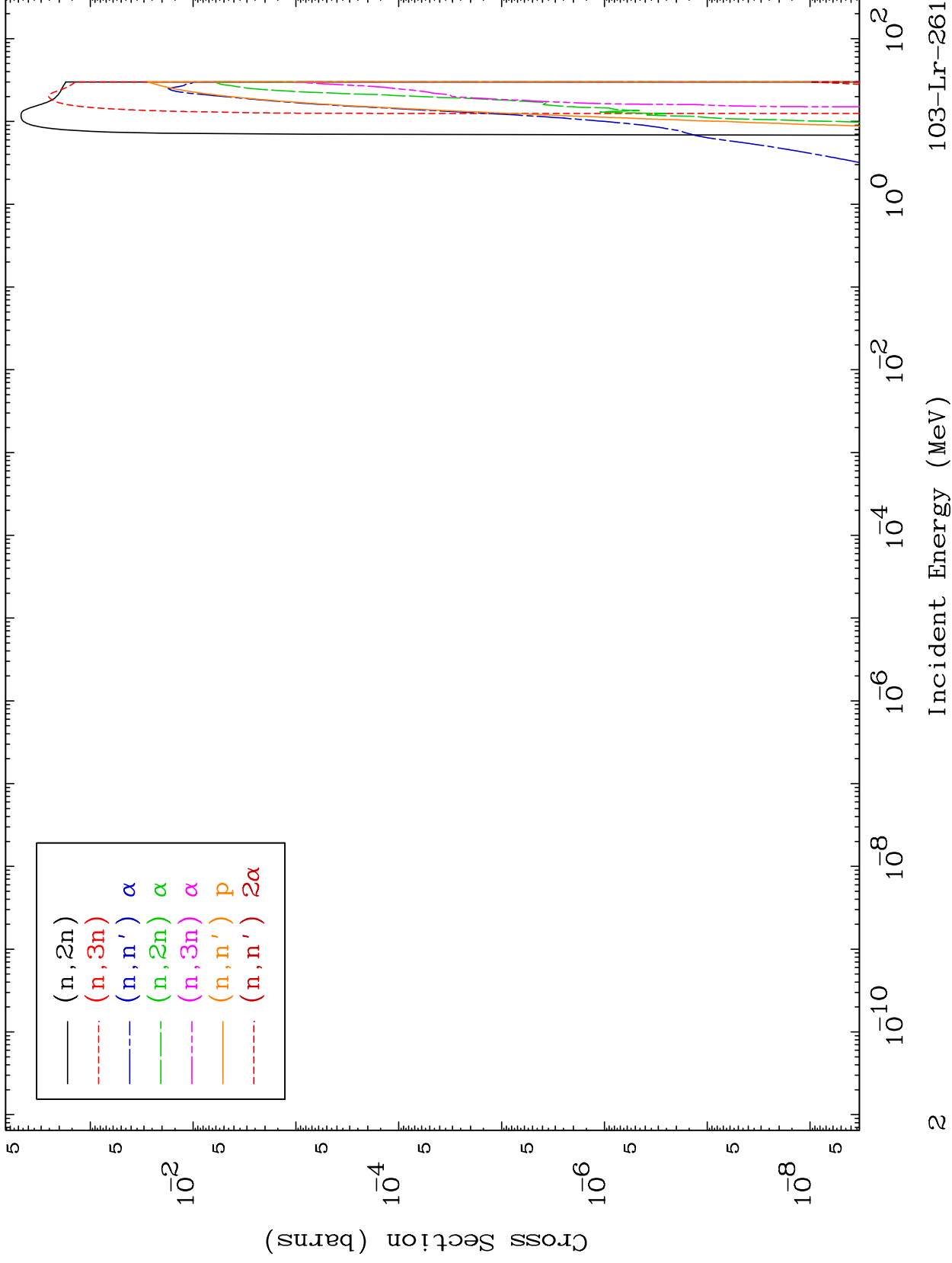
103-Lr-261

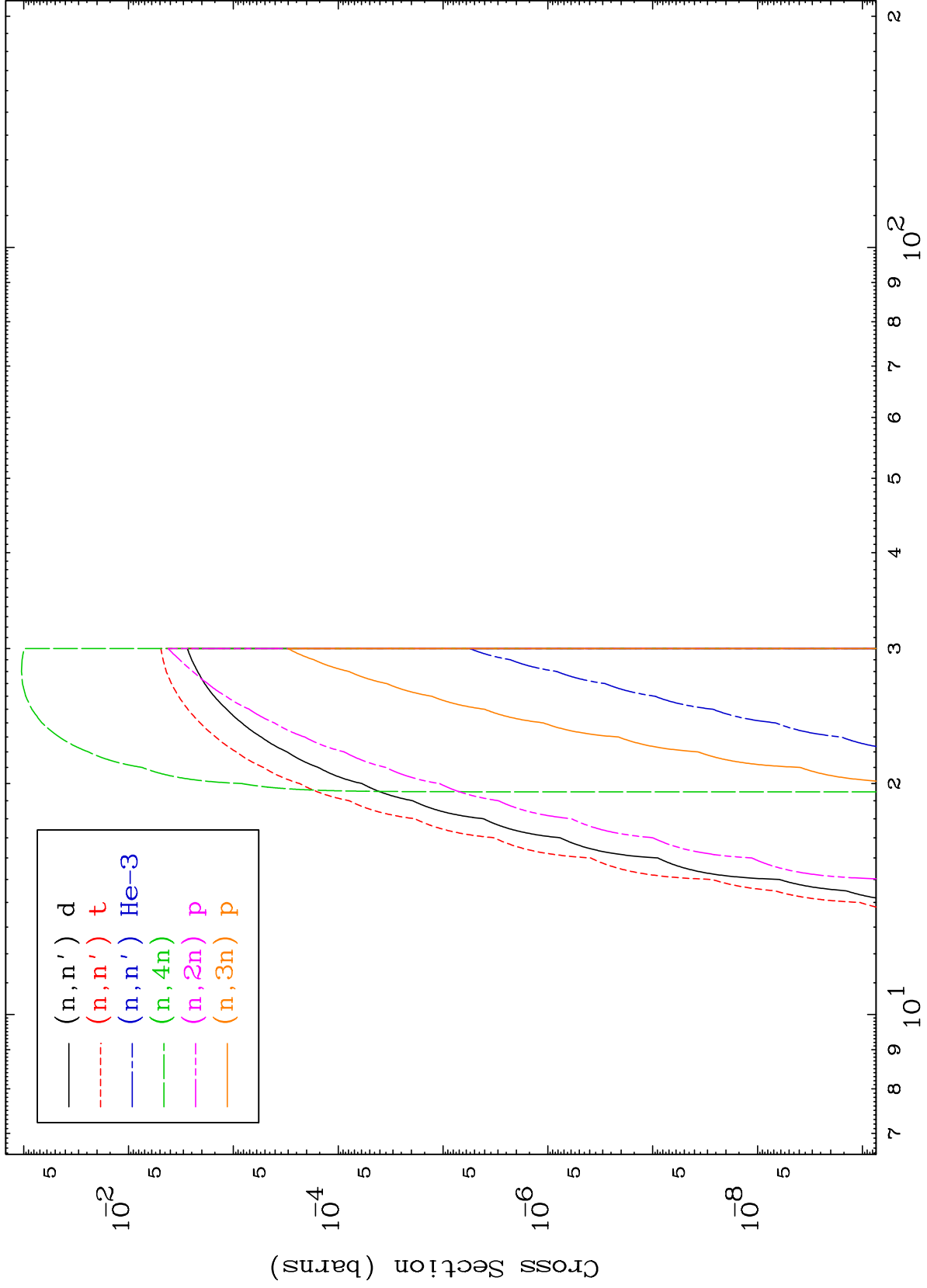


MAT 9990

Neutron Production  
293 Kelvin Cross Sections

103-Lr-261

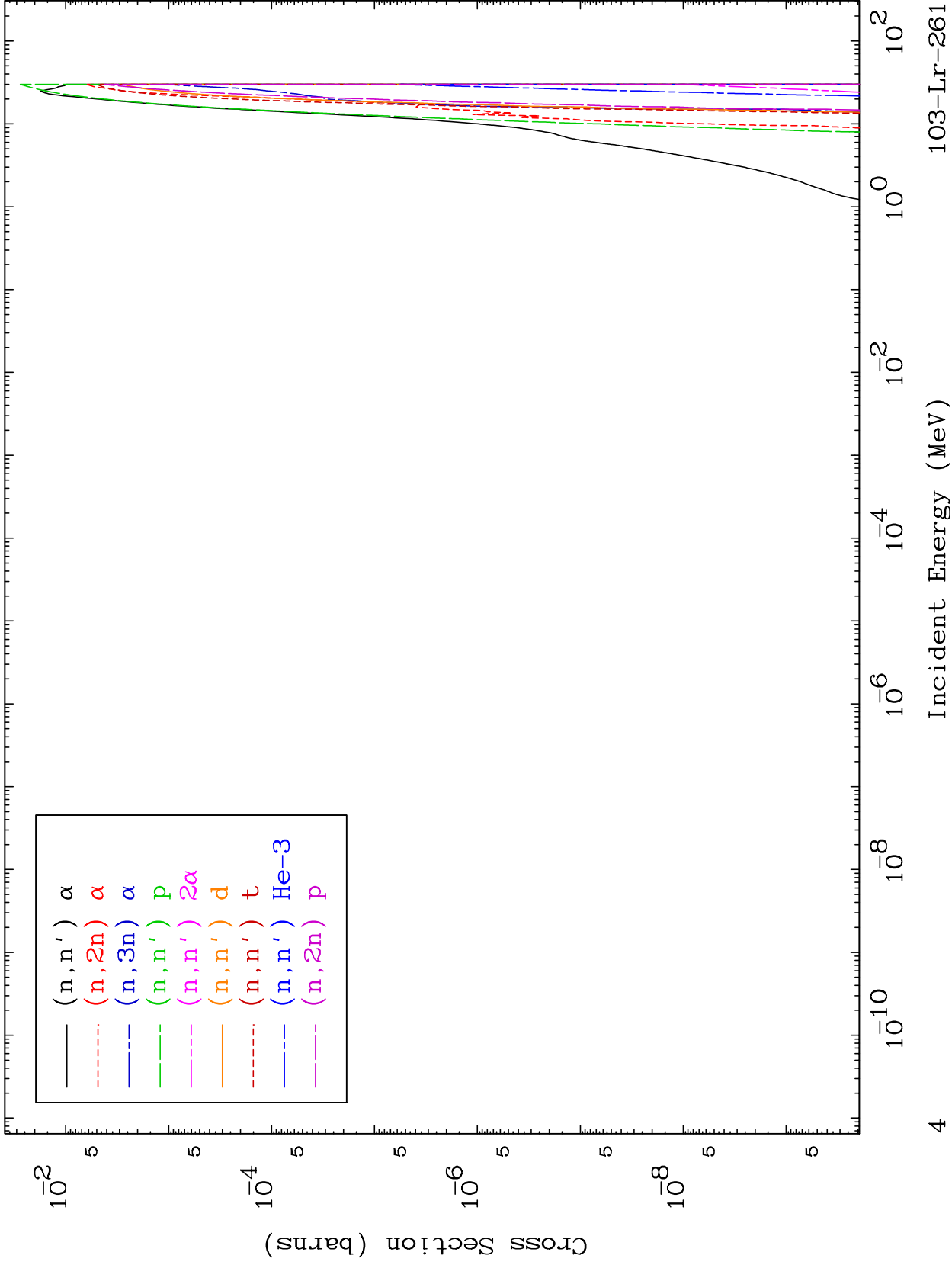




MAT 9990

Charged Particle  
293 Kelvin Cross Sections

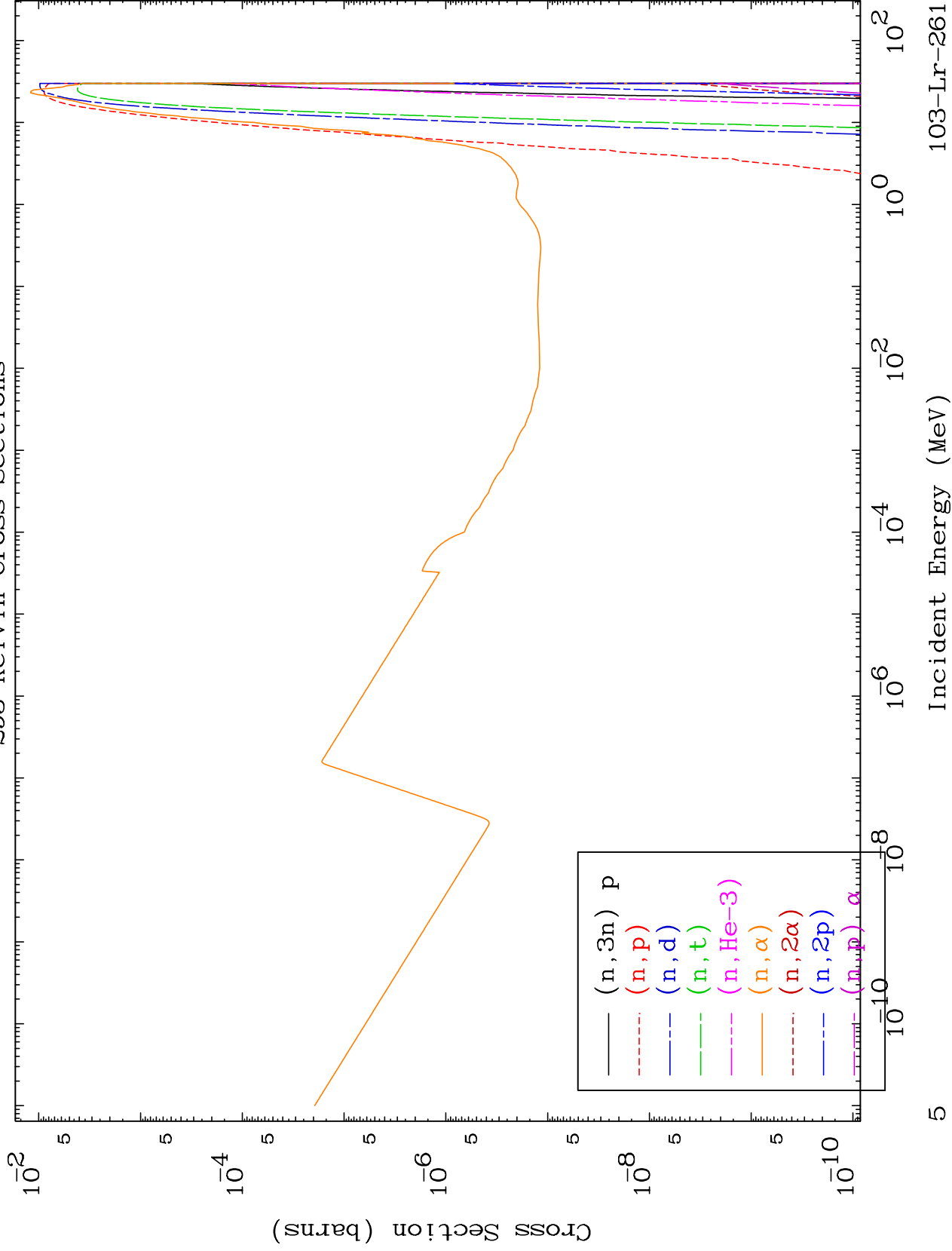
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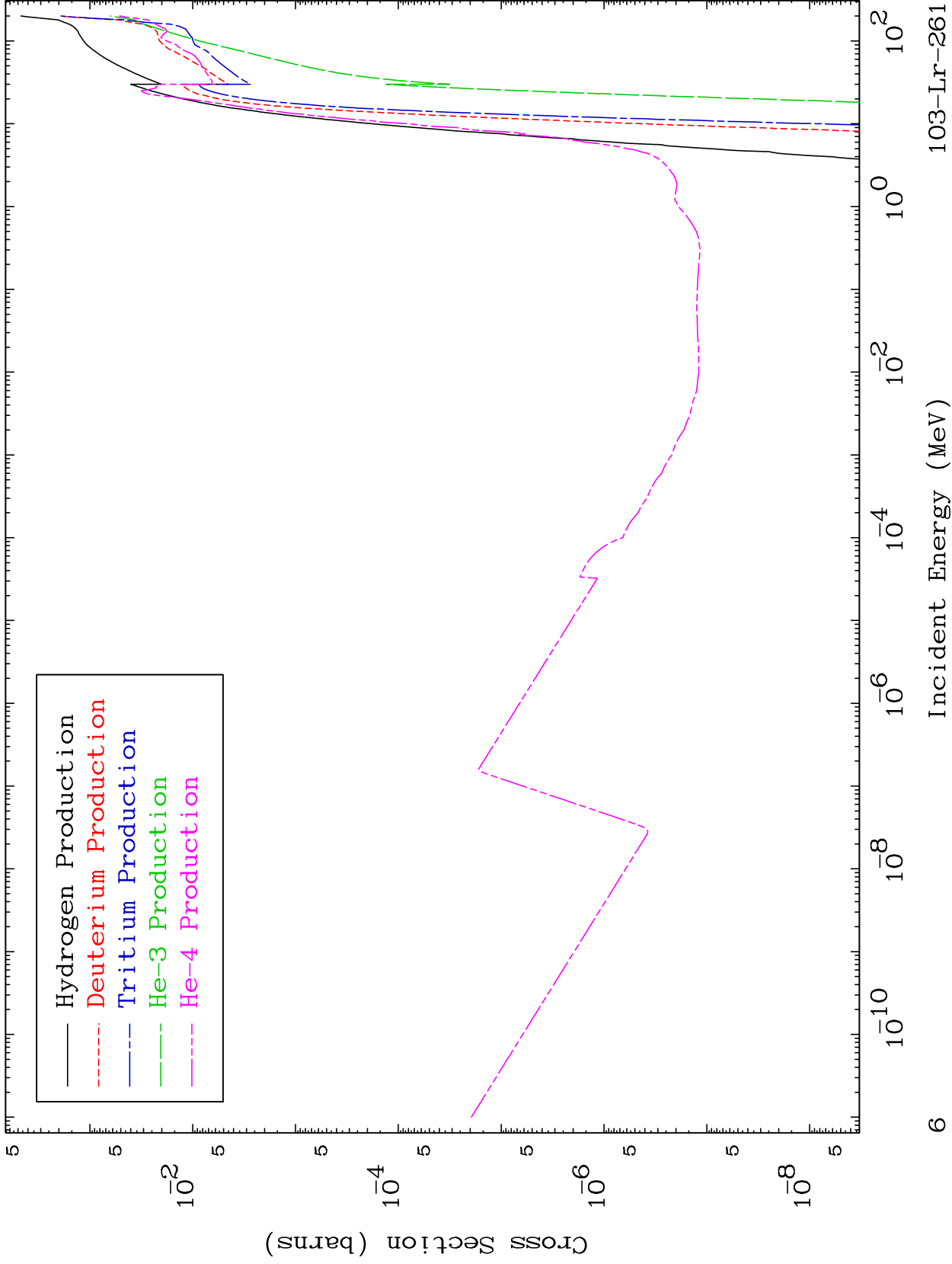


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Charged Particle  
293 Kelvin Cross Sections

103-Lr-261

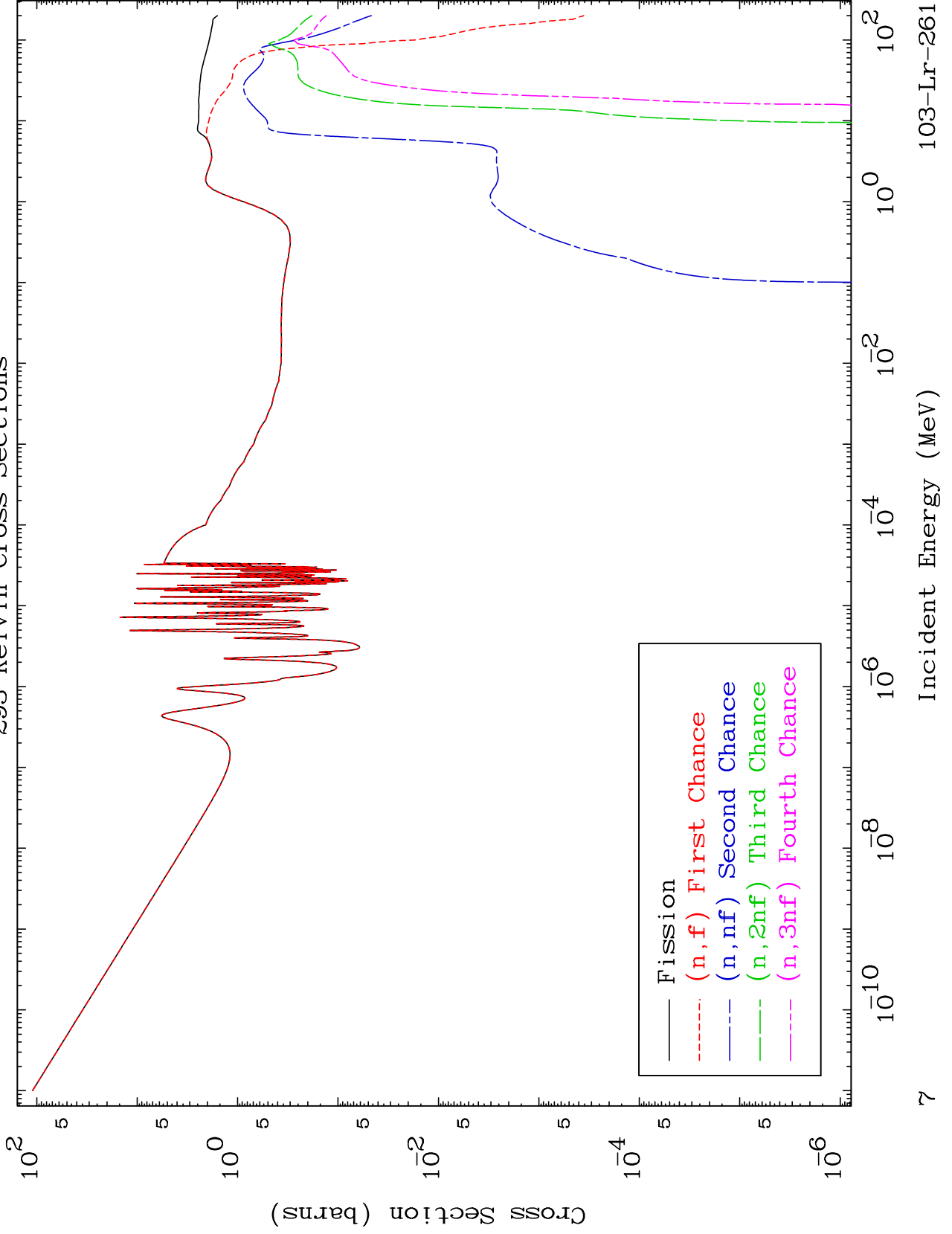




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Fission  
293 Kelvin Cross Sections

103-Lr-261

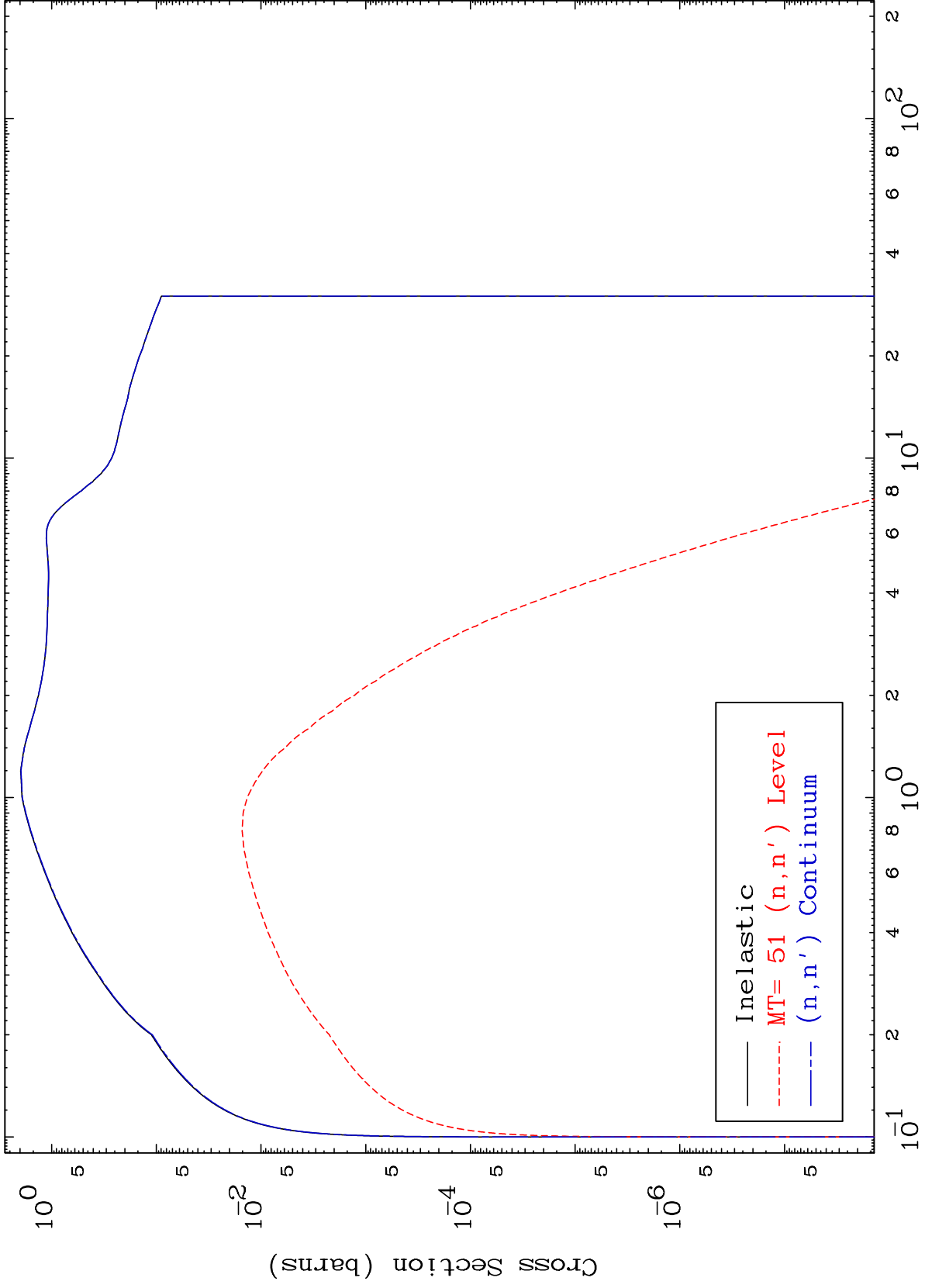




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(n,n') Level  
293 Kelvin Cross Sections

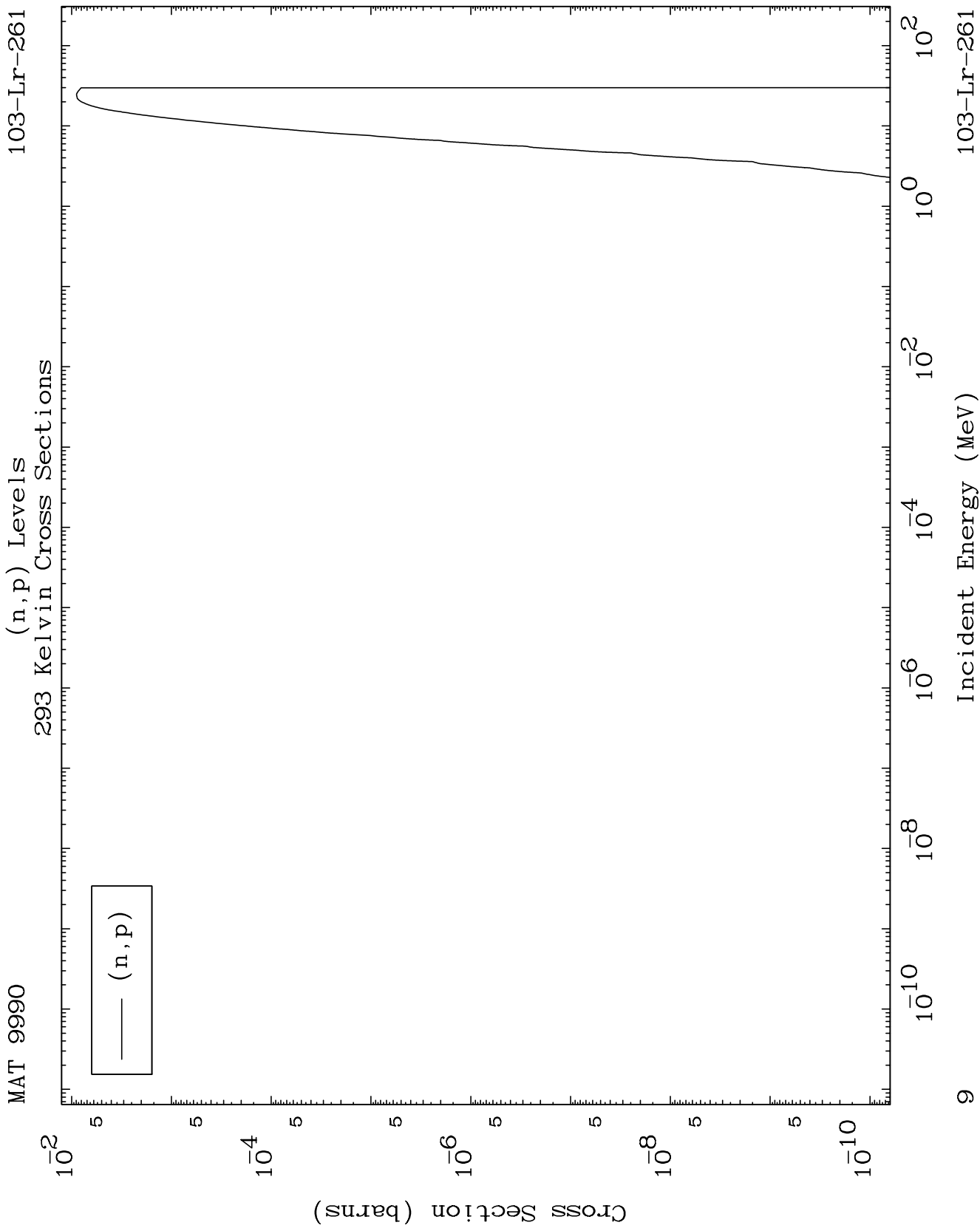
103-Lr-261



Incident Energy (MeV)

103-Lr-261

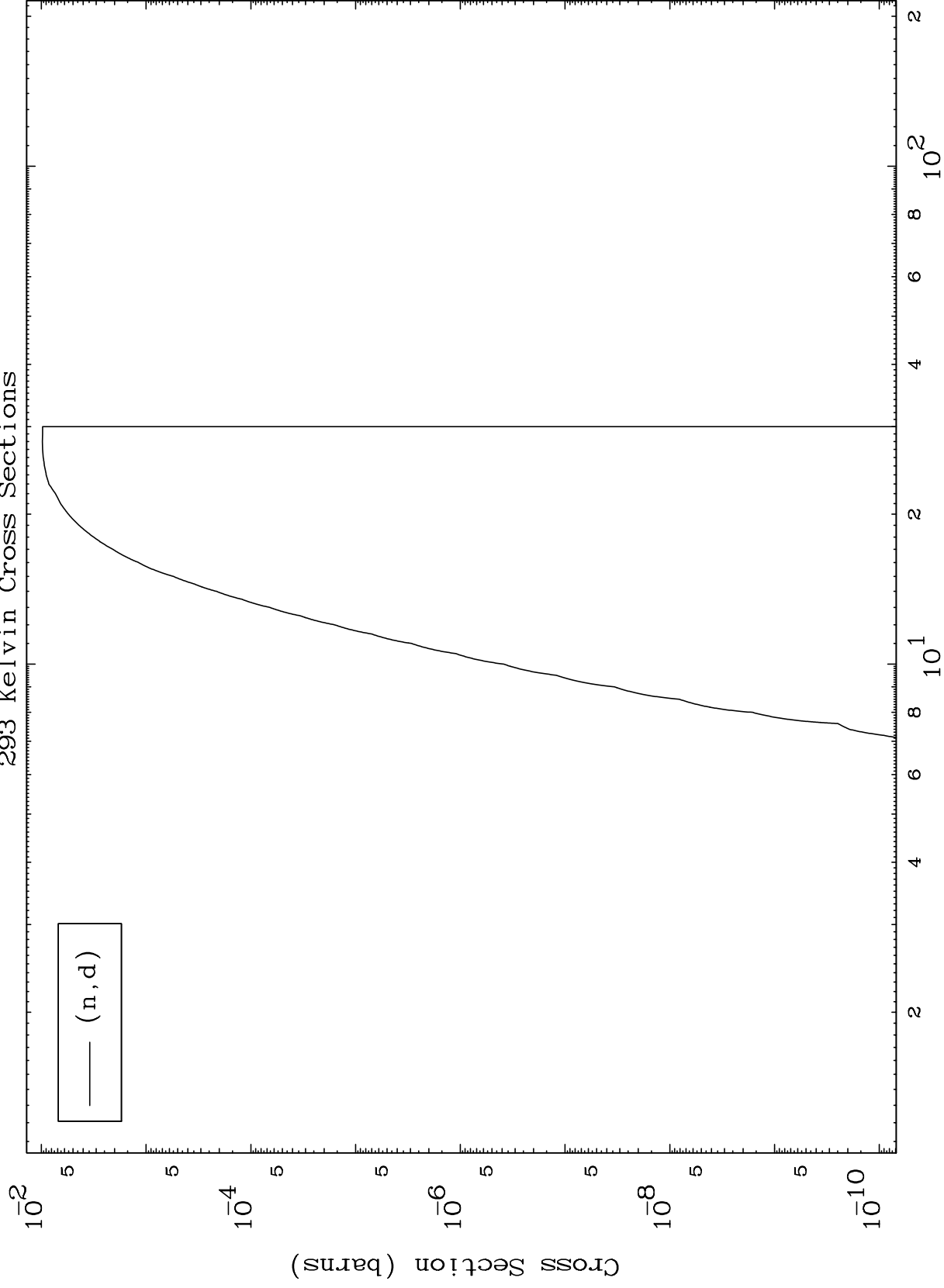
8



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(n,d) Levels  
293 Kelvin Cross Sections

103-Lr-261



10

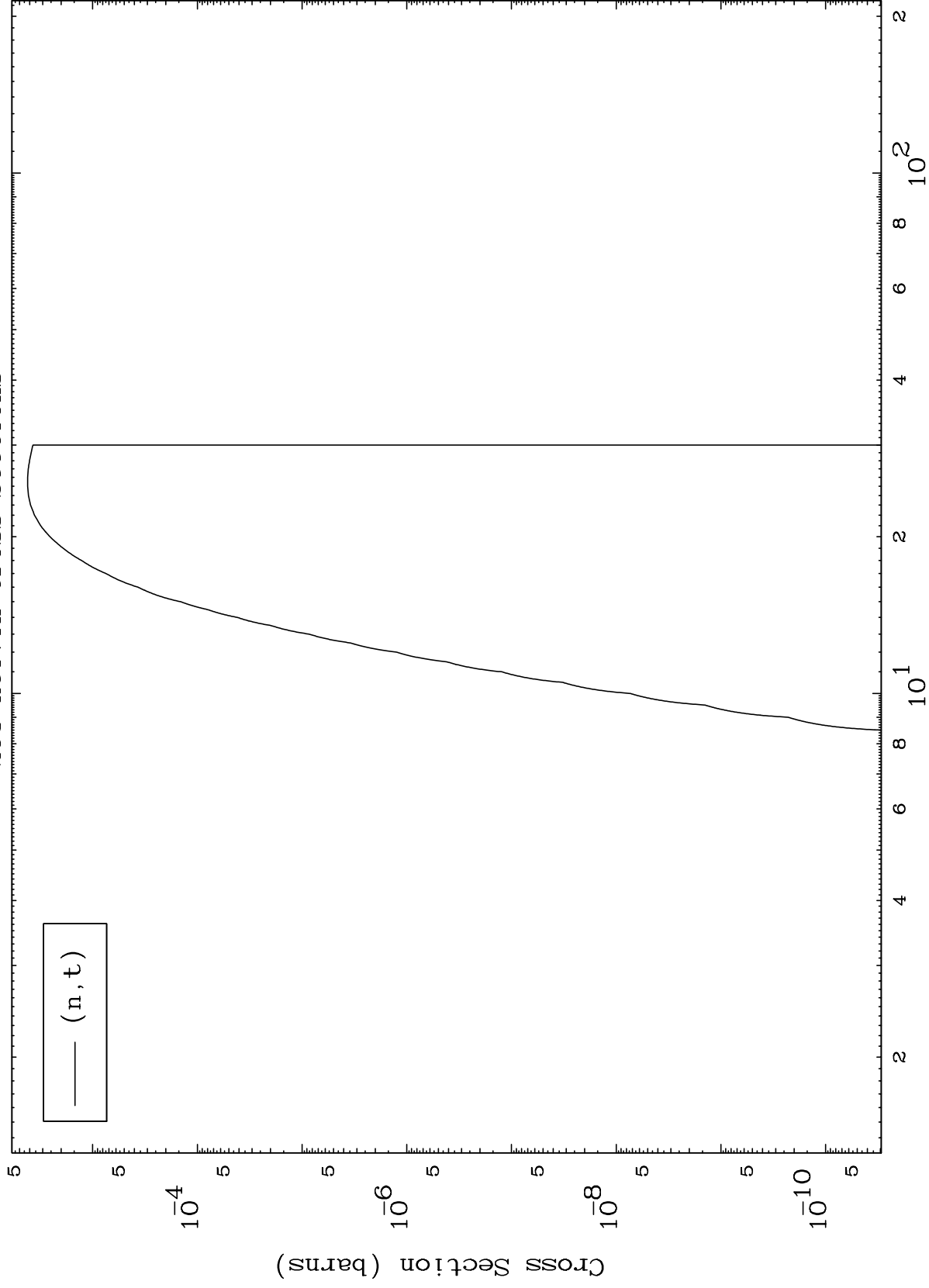
Incident Energy (MeV)

103-Lr-261

MAT 9990

(n,t) Levels  
293 Kelvin Cross Sections

103-Lr-261



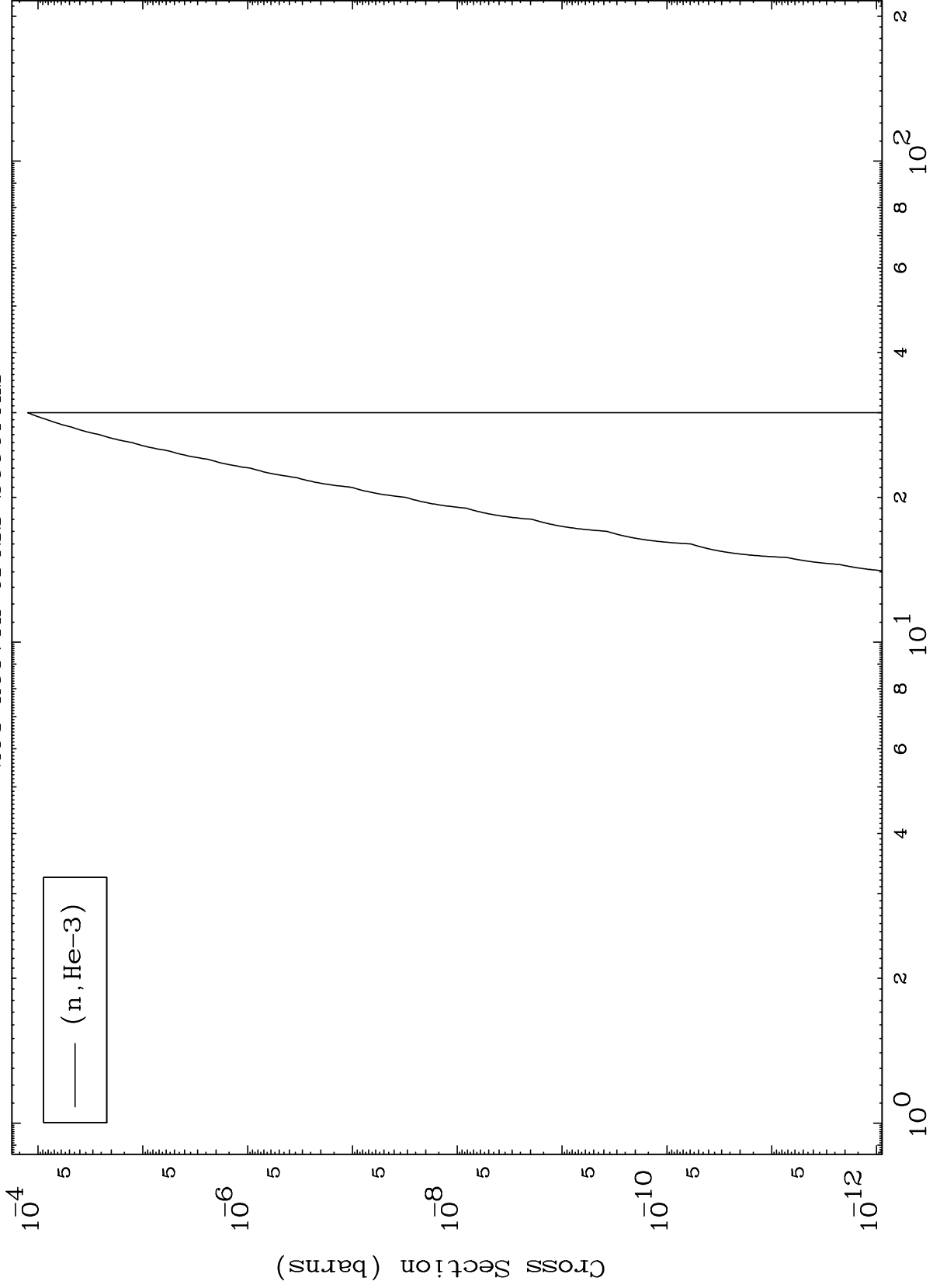
11

103-Lr-261

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(n,He3) Levels  
293 Kelvin Cross Sections

103-Lr-261



12

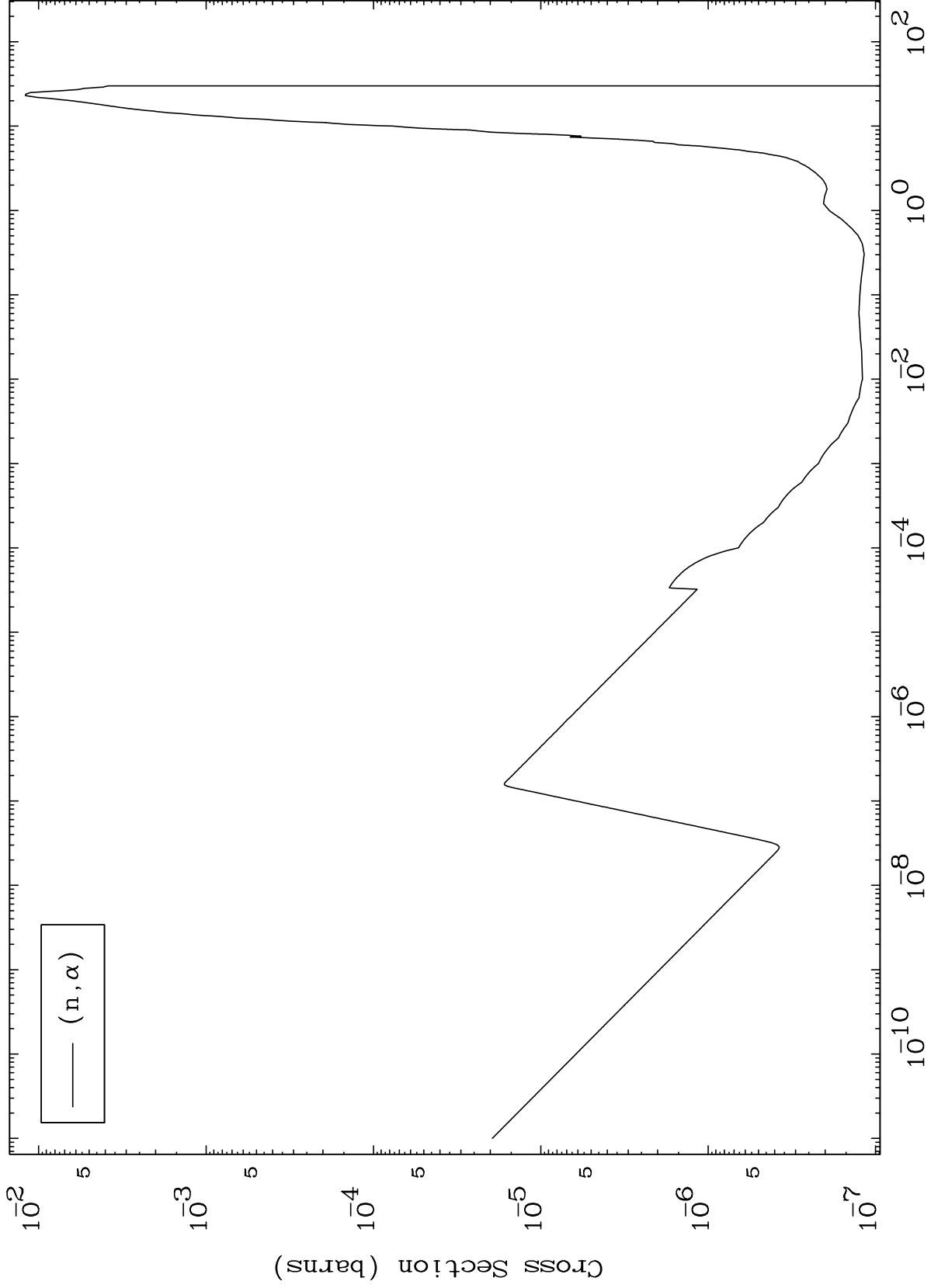
Incident Energy (MeV)

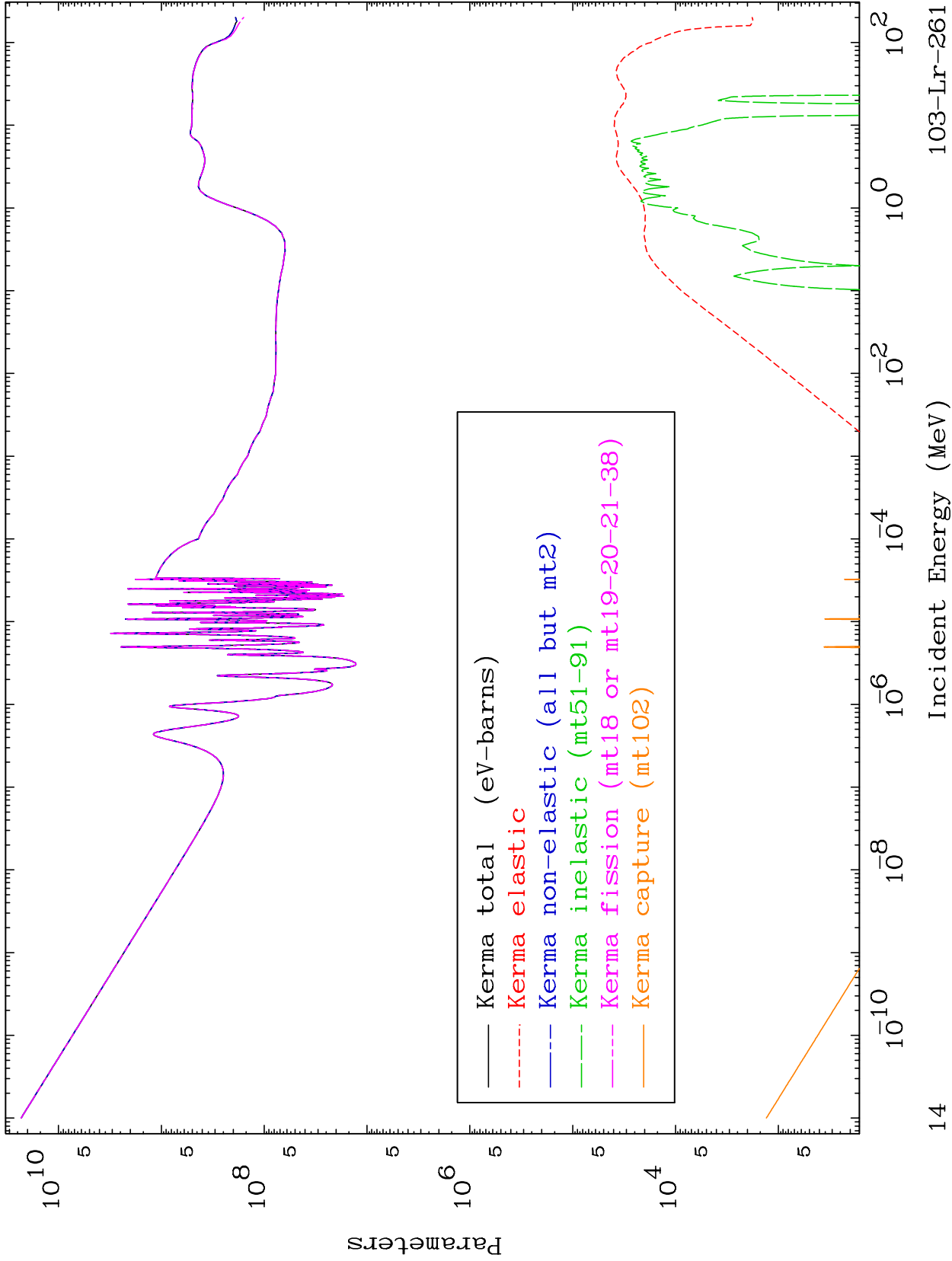
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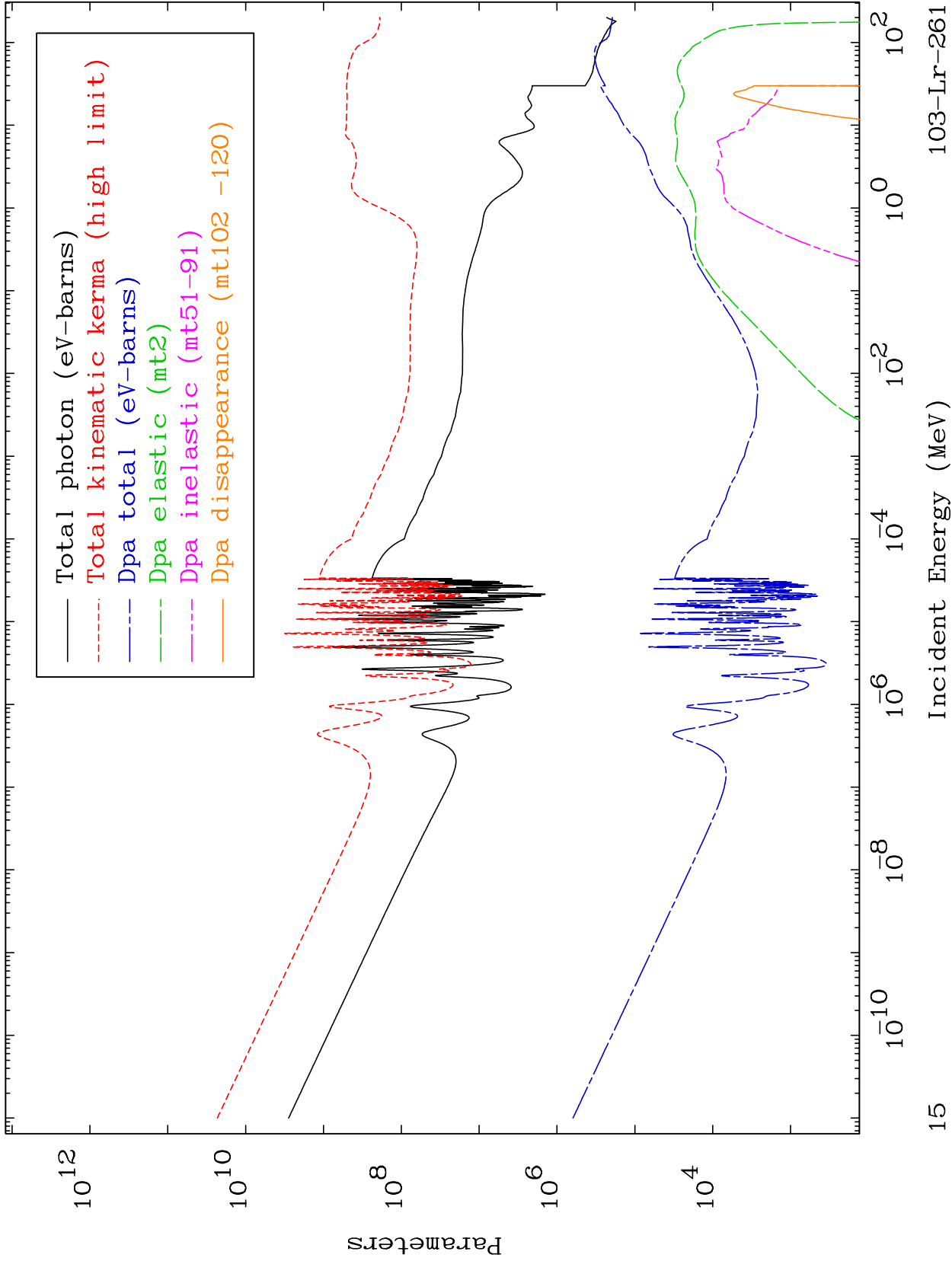
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

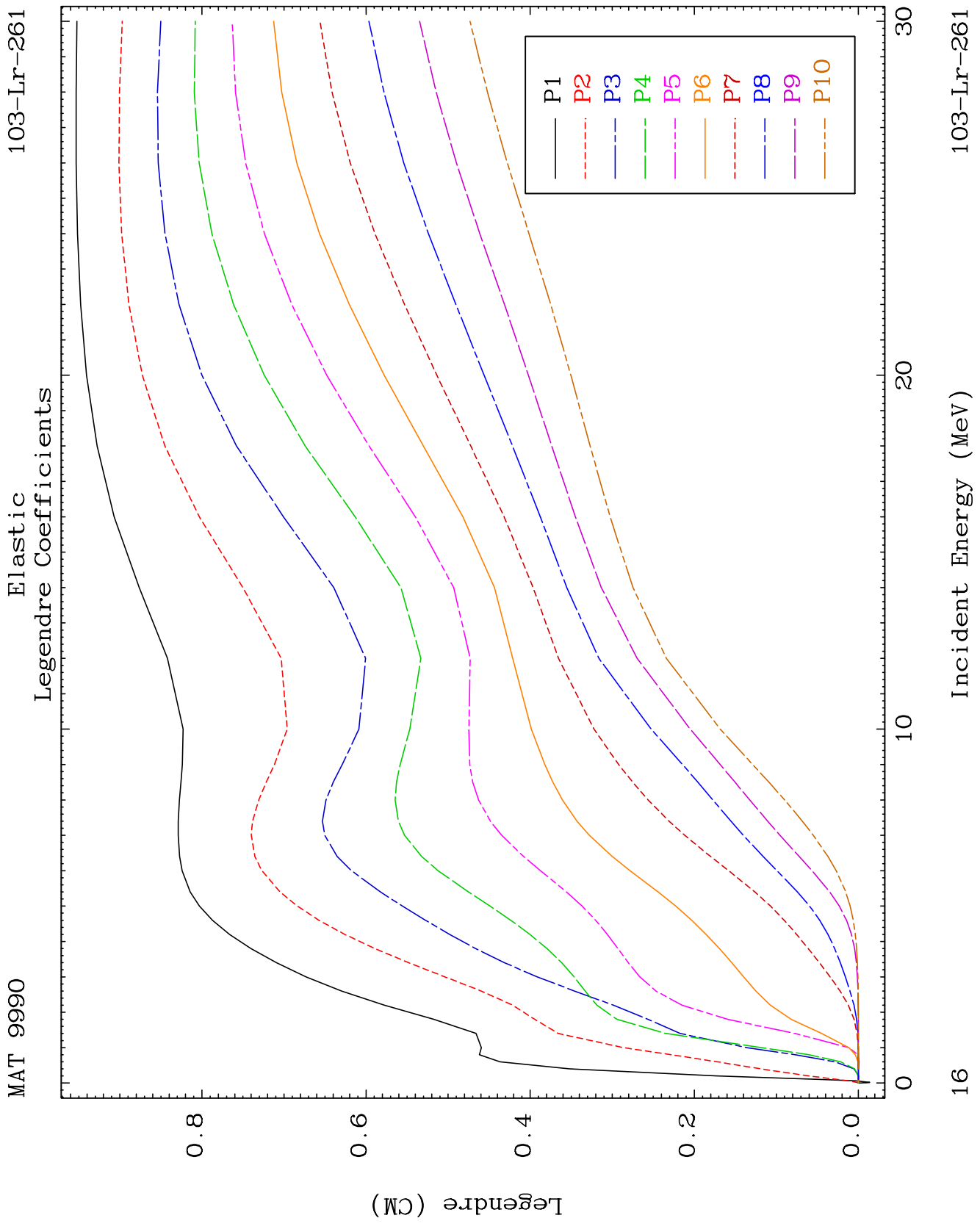
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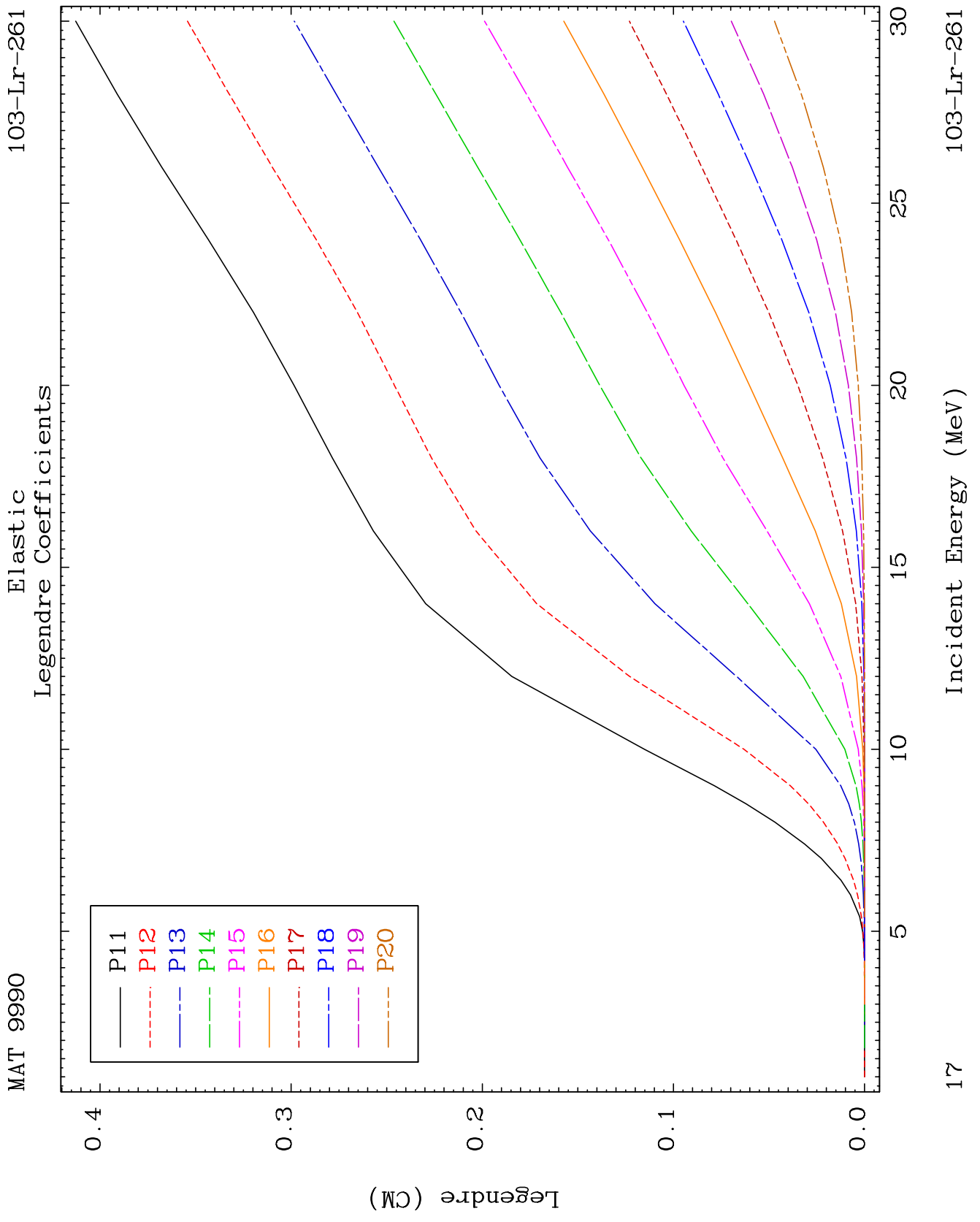








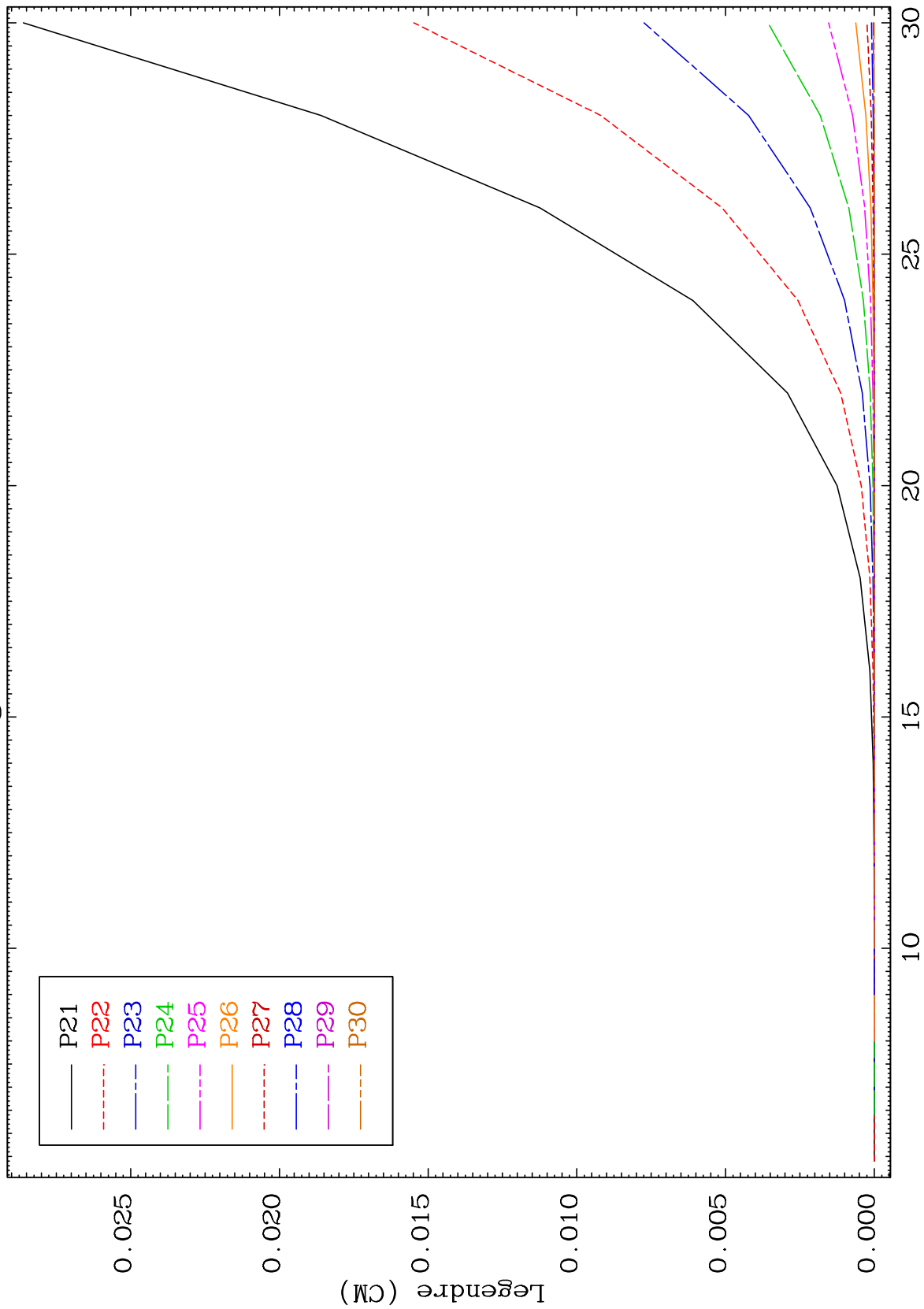




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Elastic  
Legendre Coefficients

103-Lr-261



18

Incident Energy (MeV)

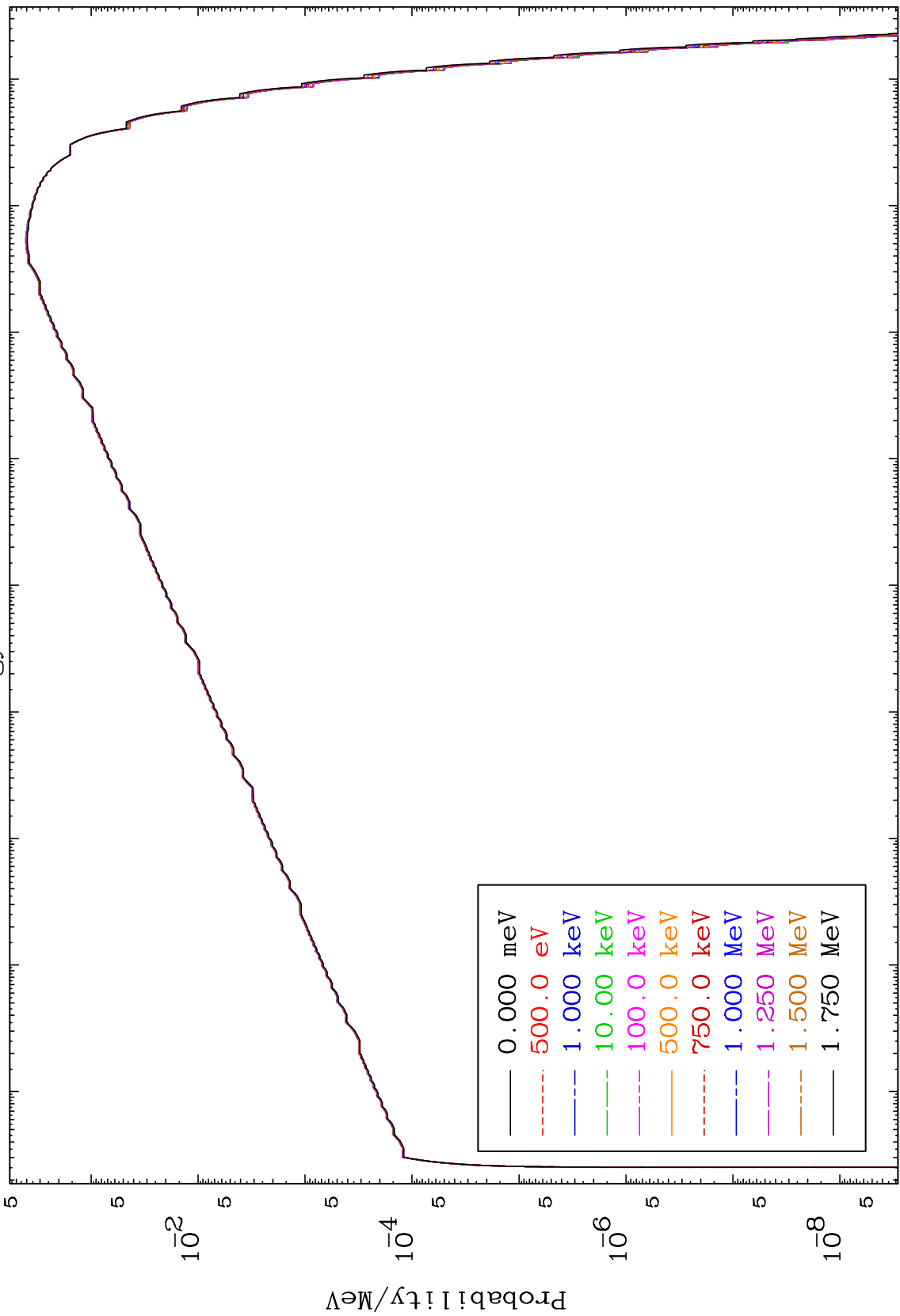
103-Lr-261



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### Fission Energy Distribution

103-Lr-261



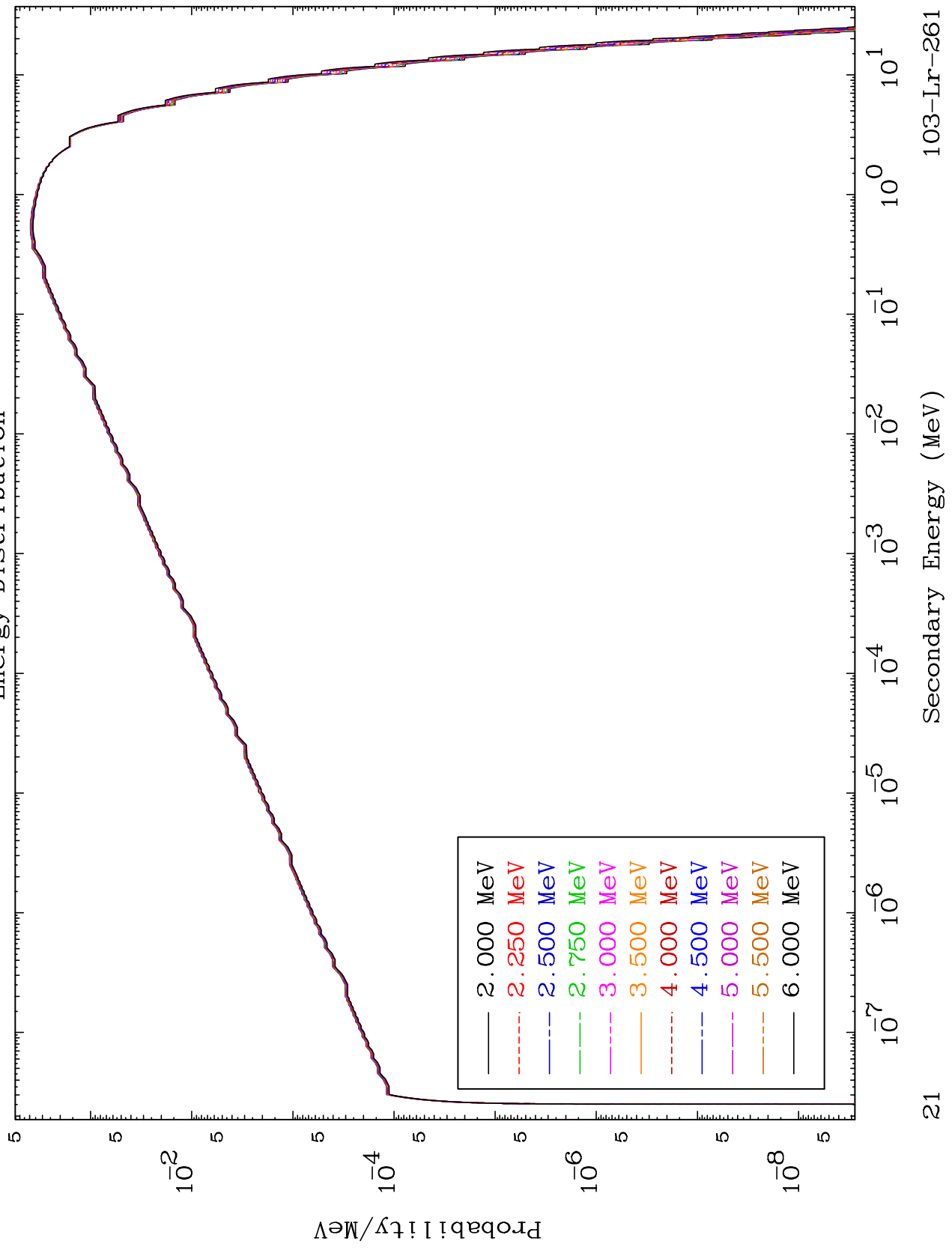
103-Lr-261

Secondary Energy (MeV)

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### Fission Energy Distribution

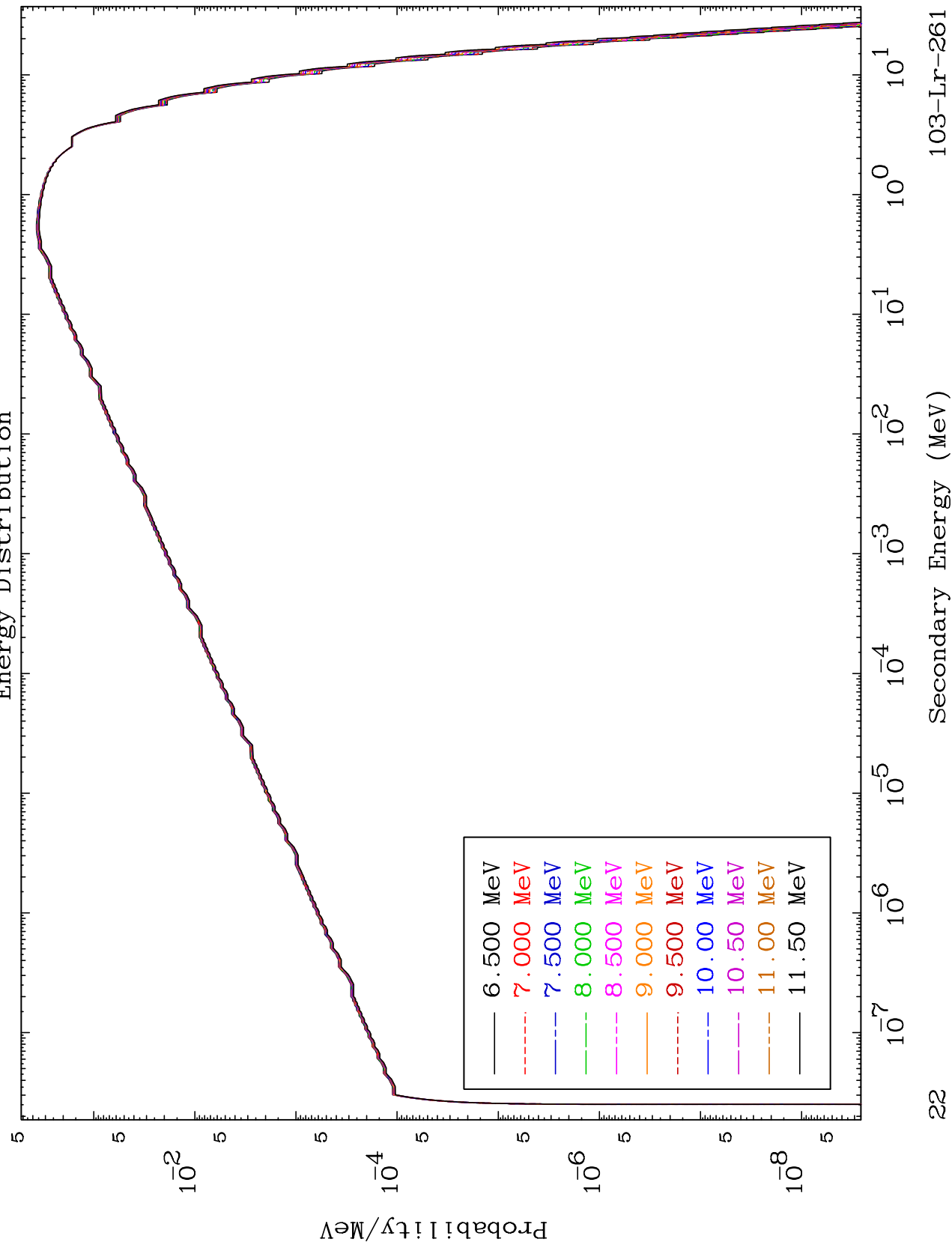
103-Lr-261



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### Fission Energy Distribution

103-Lr-261

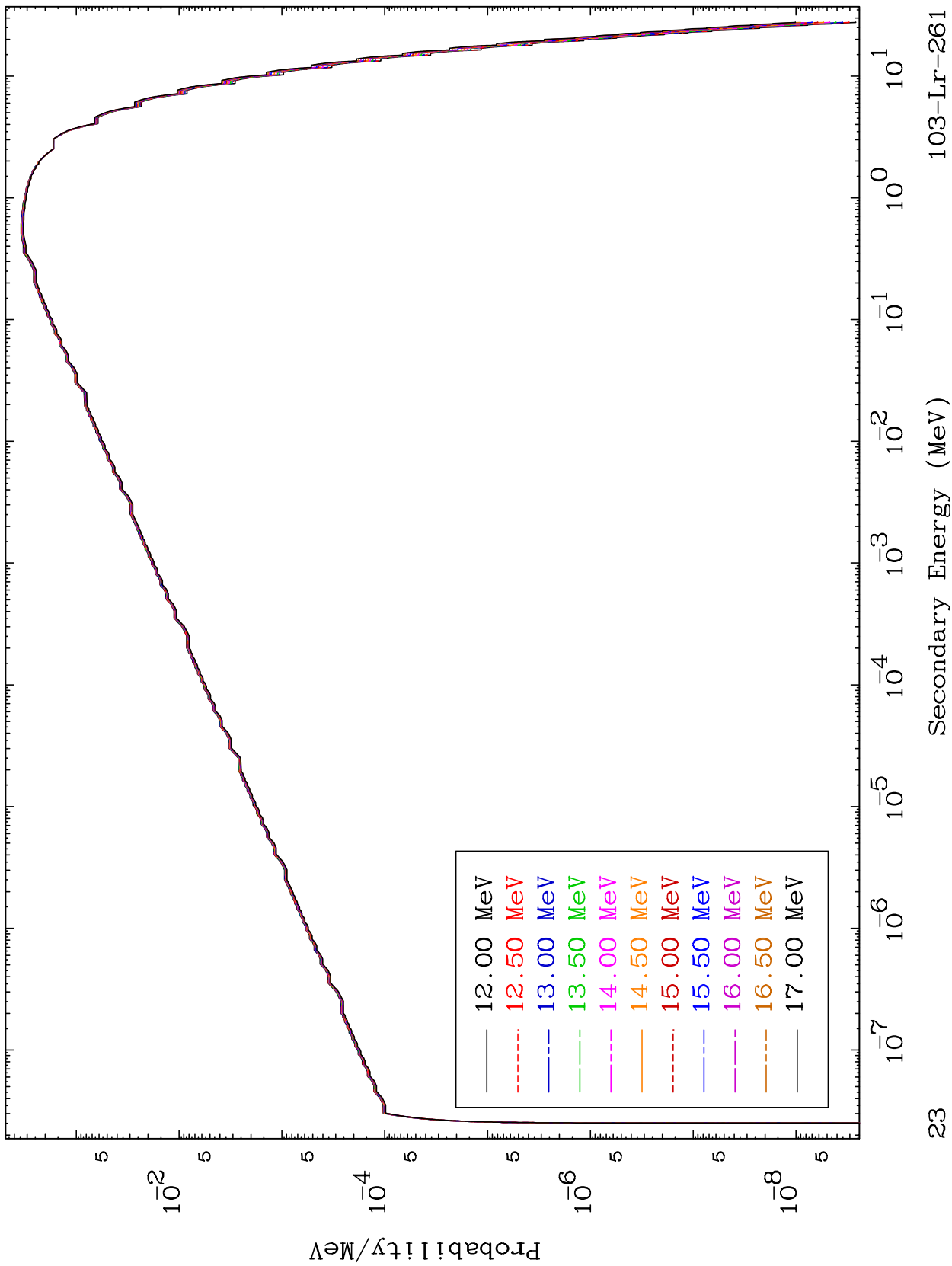


103-Lr-261

MAT 9990

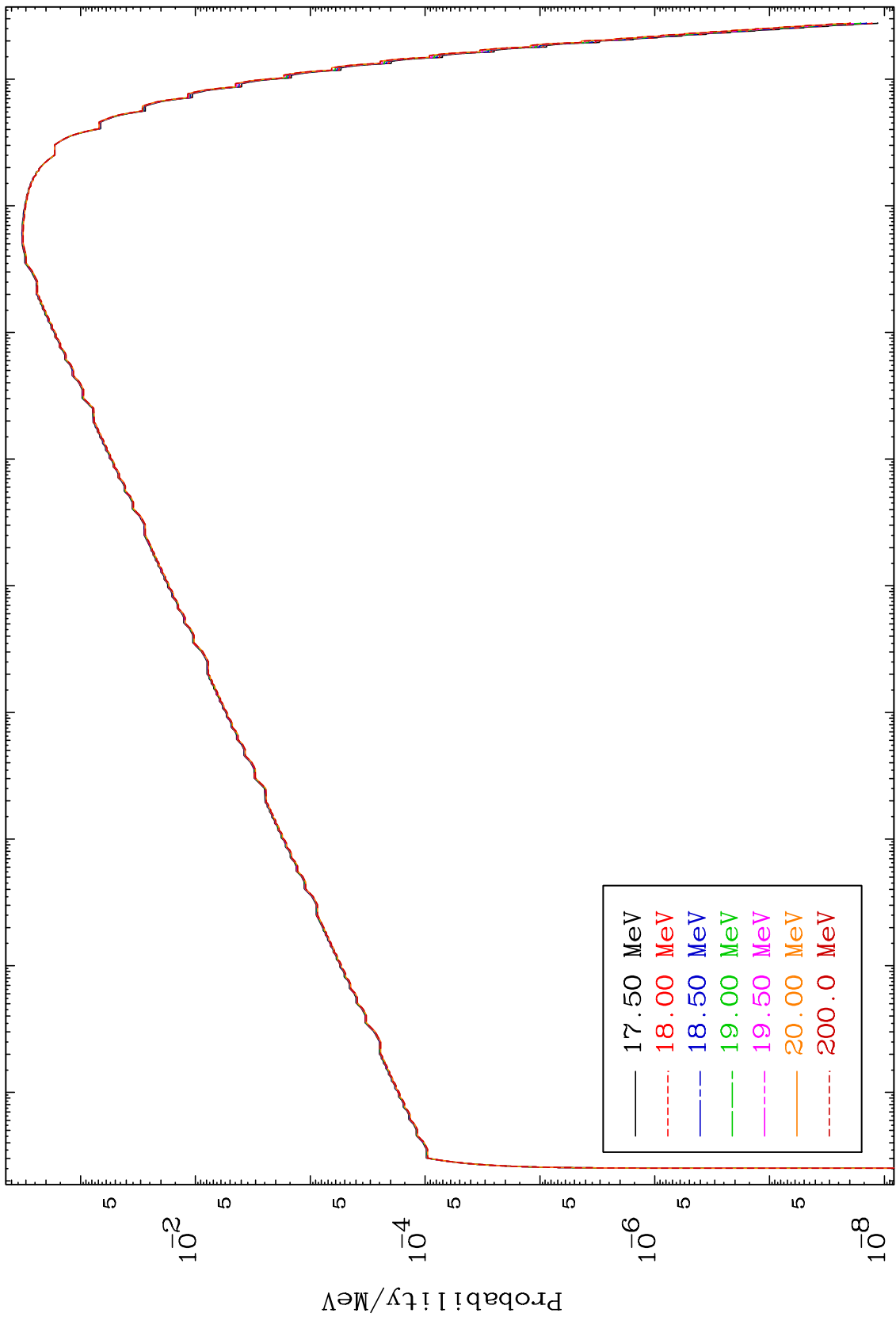
### Fission Energy Distribution

103-Lr-261





Fission  
Energy Distribution

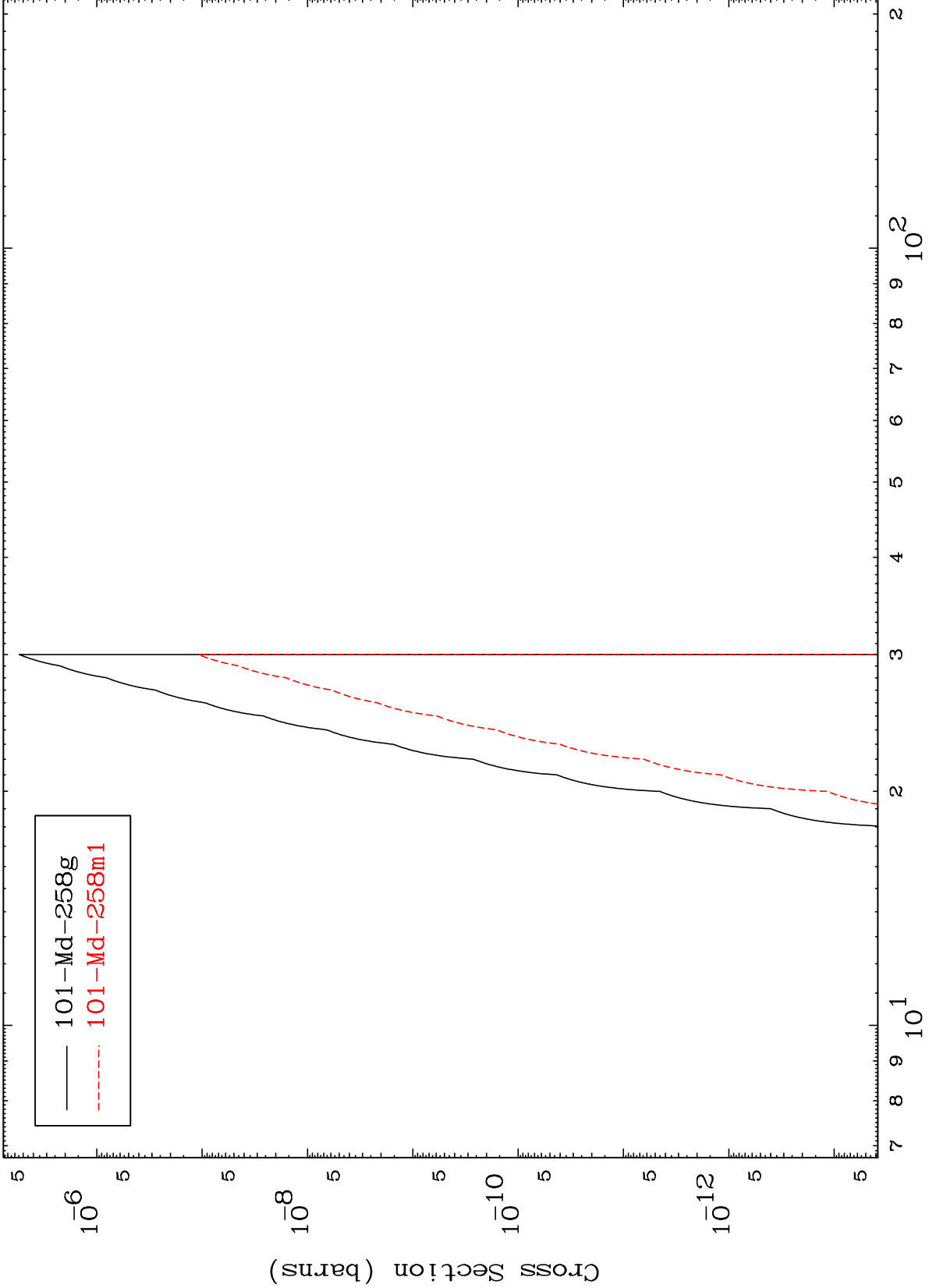


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(n,n') He-3

103-Lr-261

Radionuclide Production Cross Section



101-Md-258g  
101-Md-258m1

25

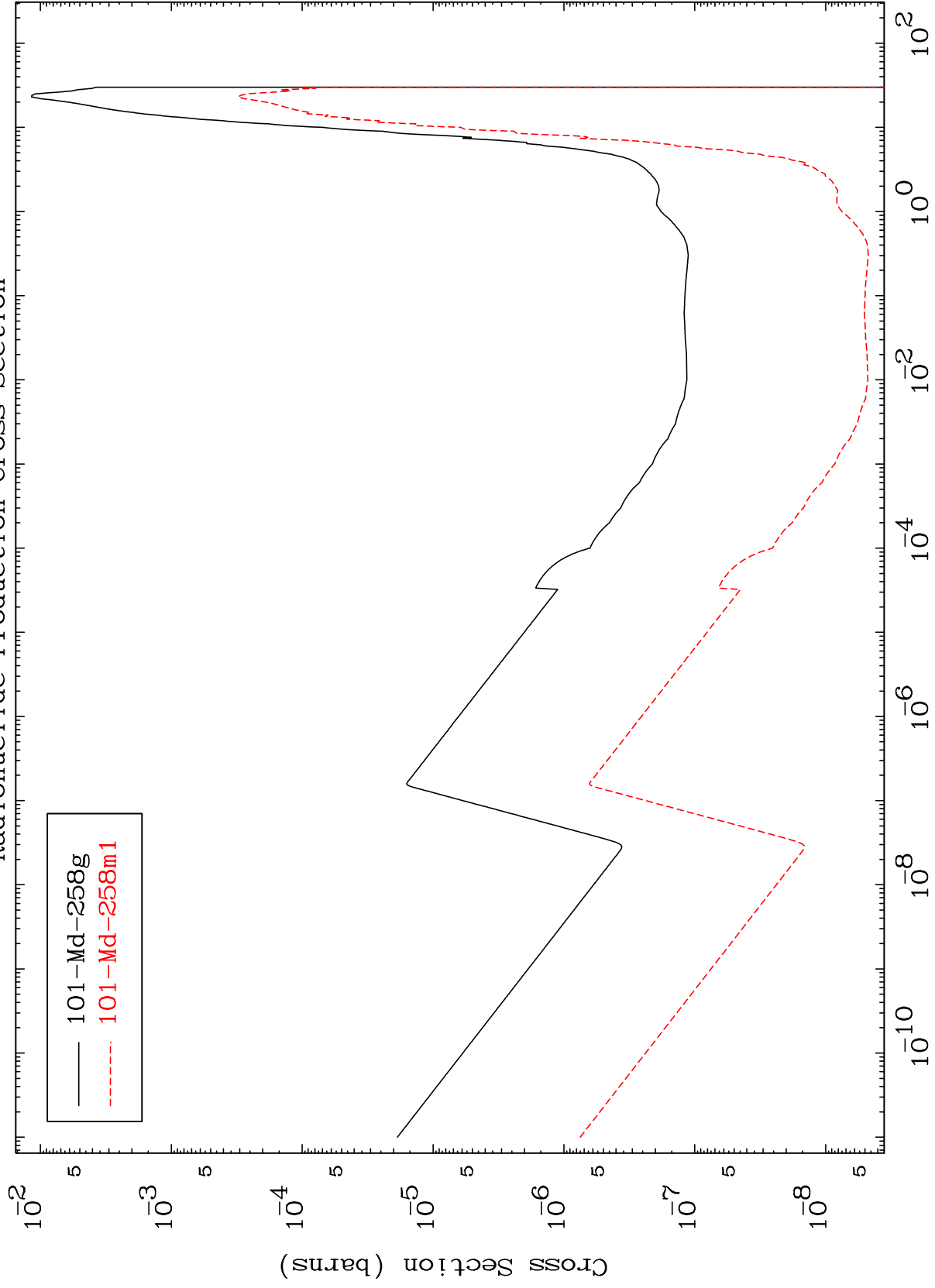
Incident Energy (MeV)

103-Lr-261

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103-Lr-261

Radionuclide Production Cross Section  
(n,  $\alpha$ )



103-Lr-261

Incident Energy (MeV)

26

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(n,2α)

103-Lr-261

Radionuclide Production Cross Section

