

Program EVALPLOT  
(Version 2017-1)

by

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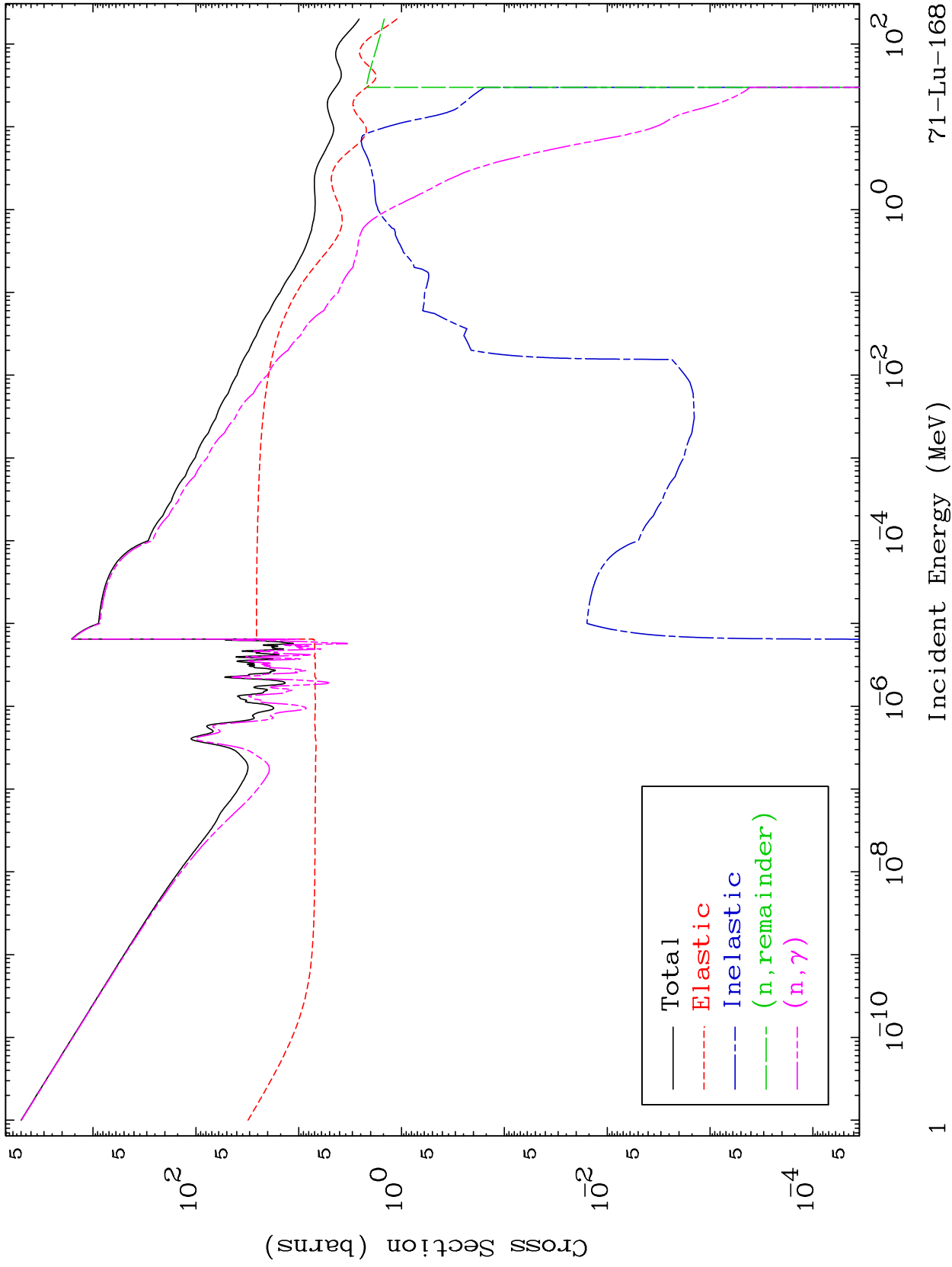
Press Mouse Button to Start

MAT 7105

Major

293 Kelvin Cross Sections

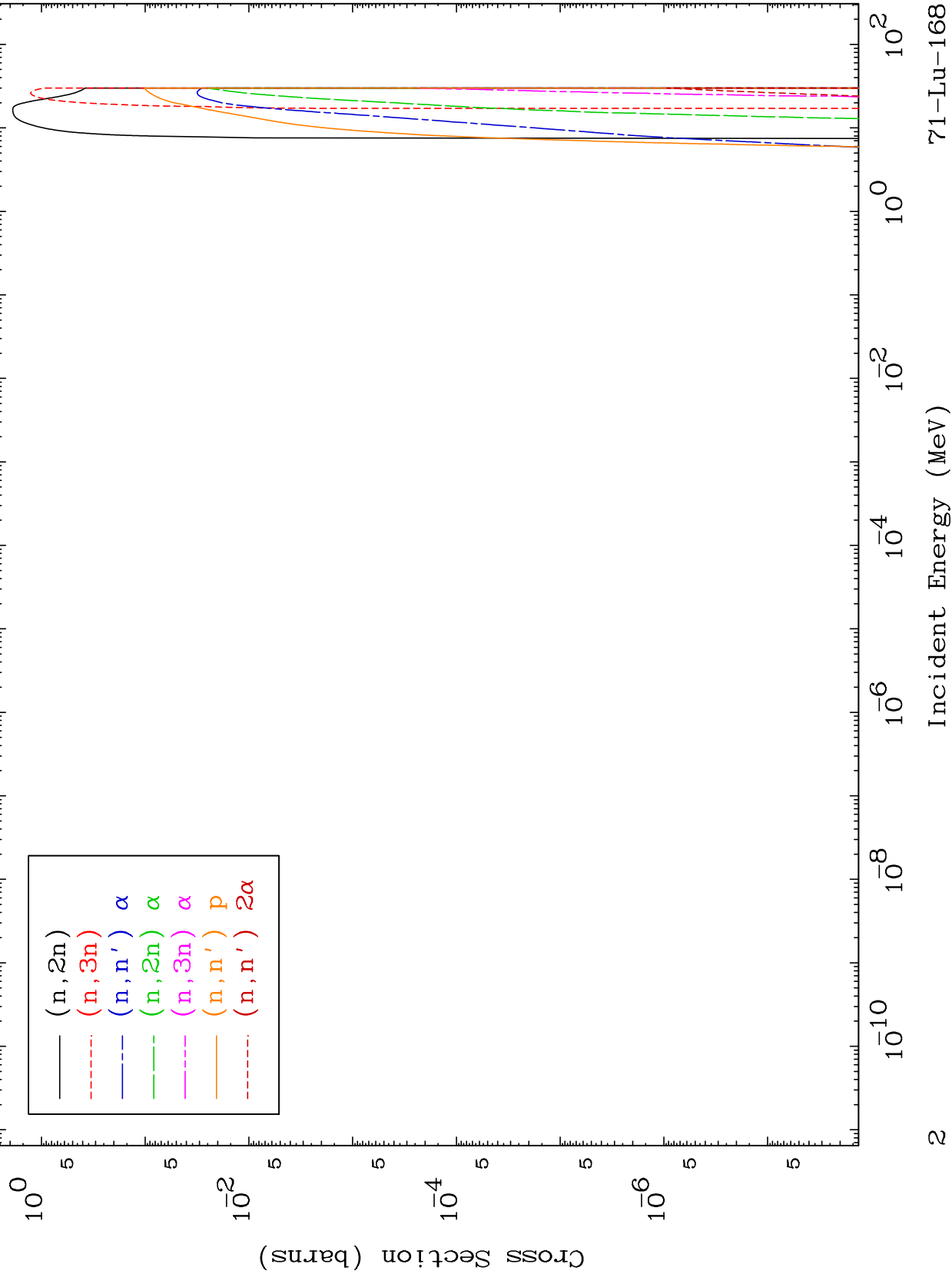
71-Lu-168

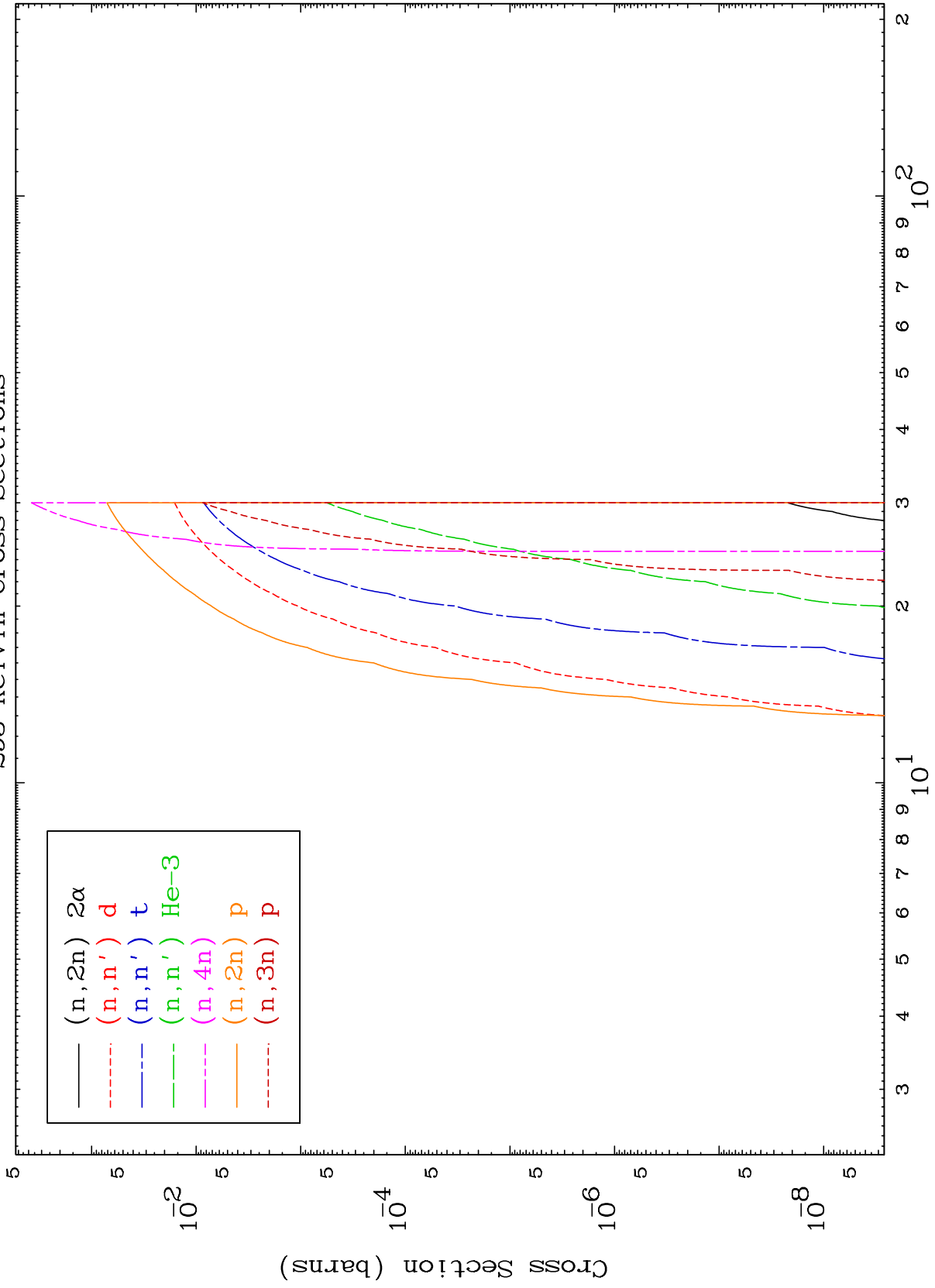


MAT 7105

Neutron Production  
293 Kelvin Cross Sections

71-Lu-168

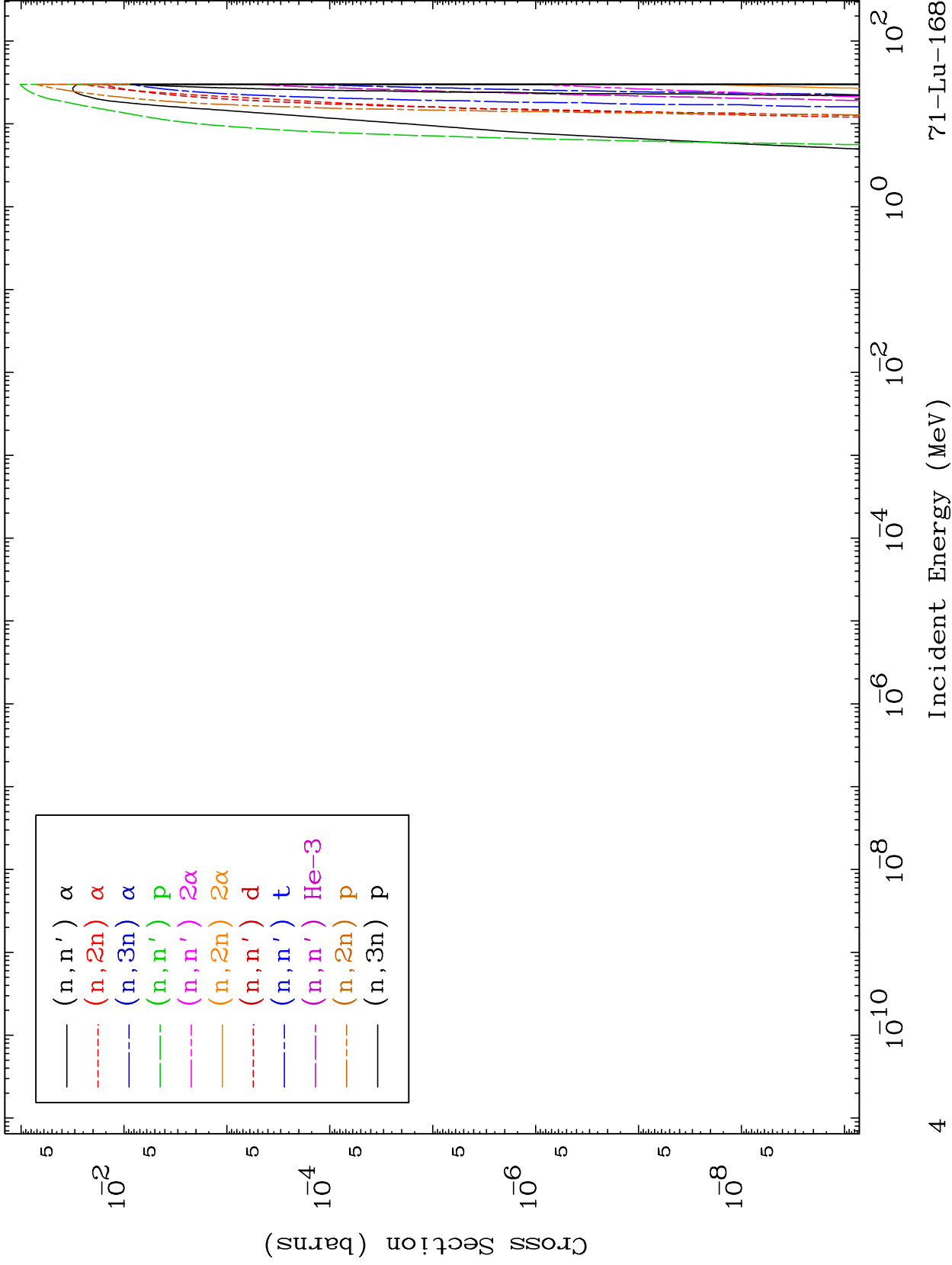


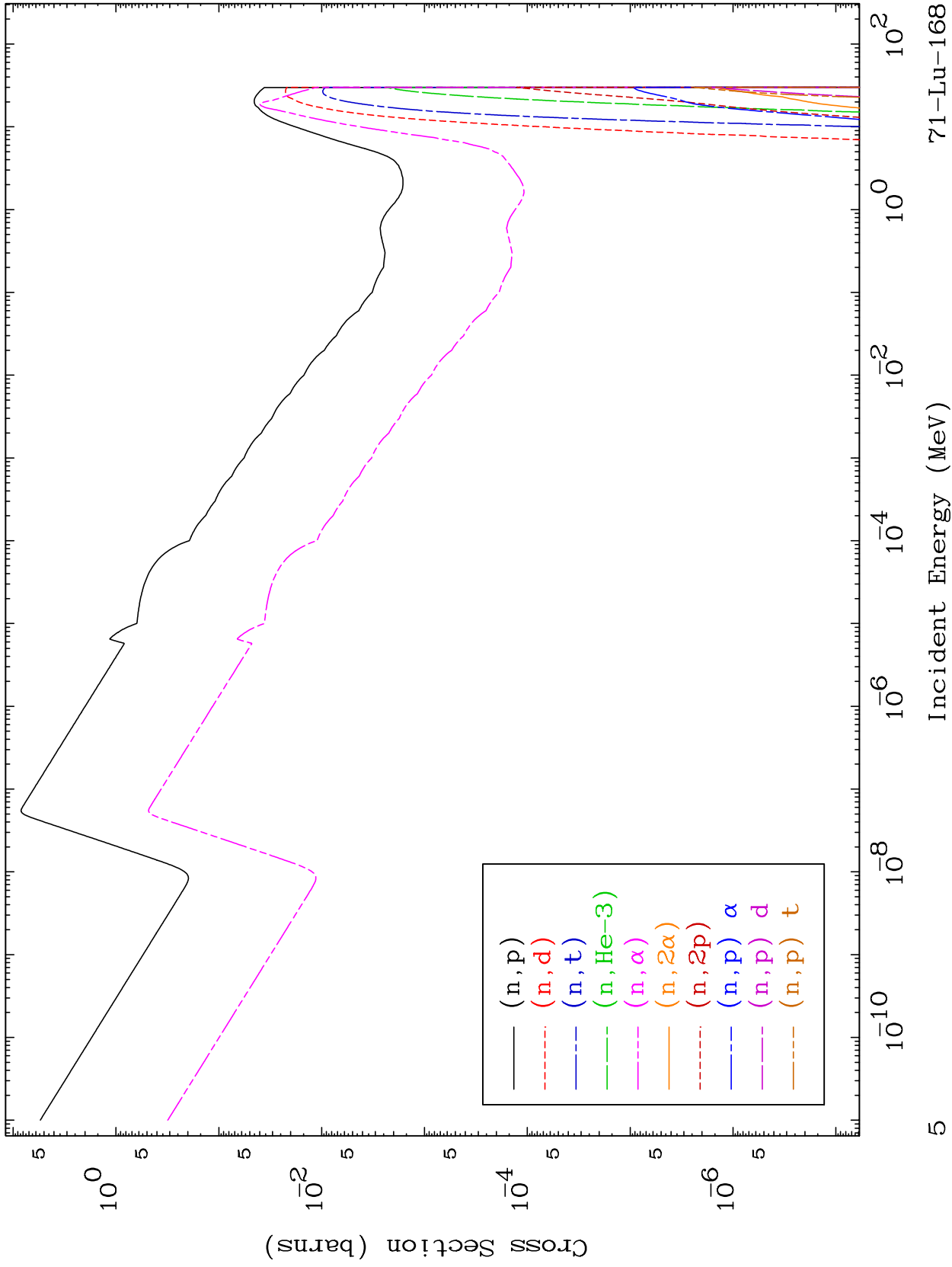


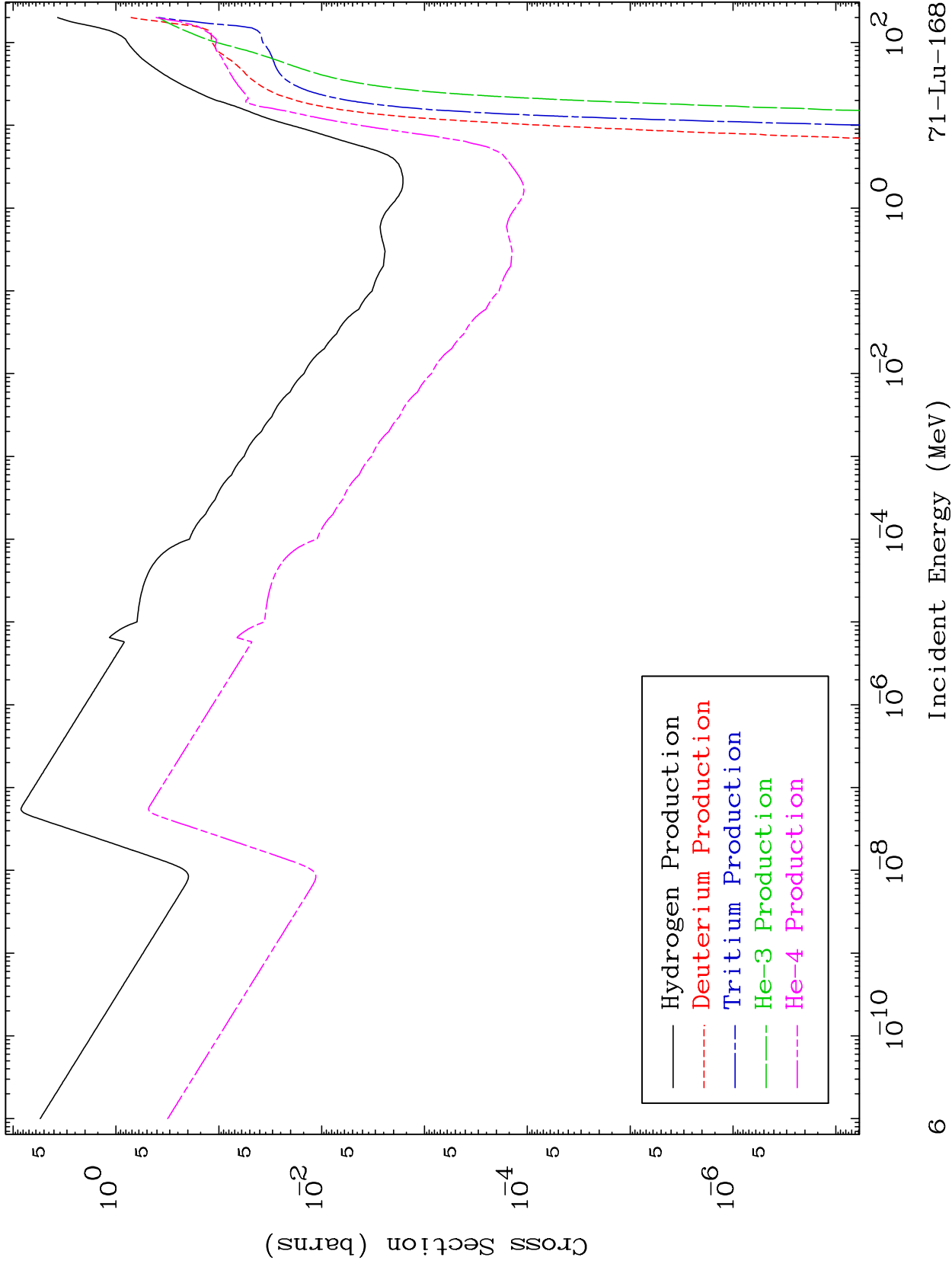
MAT 7105

Charged Particle  
293 Kelvin Cross Sections

71-Lu-168



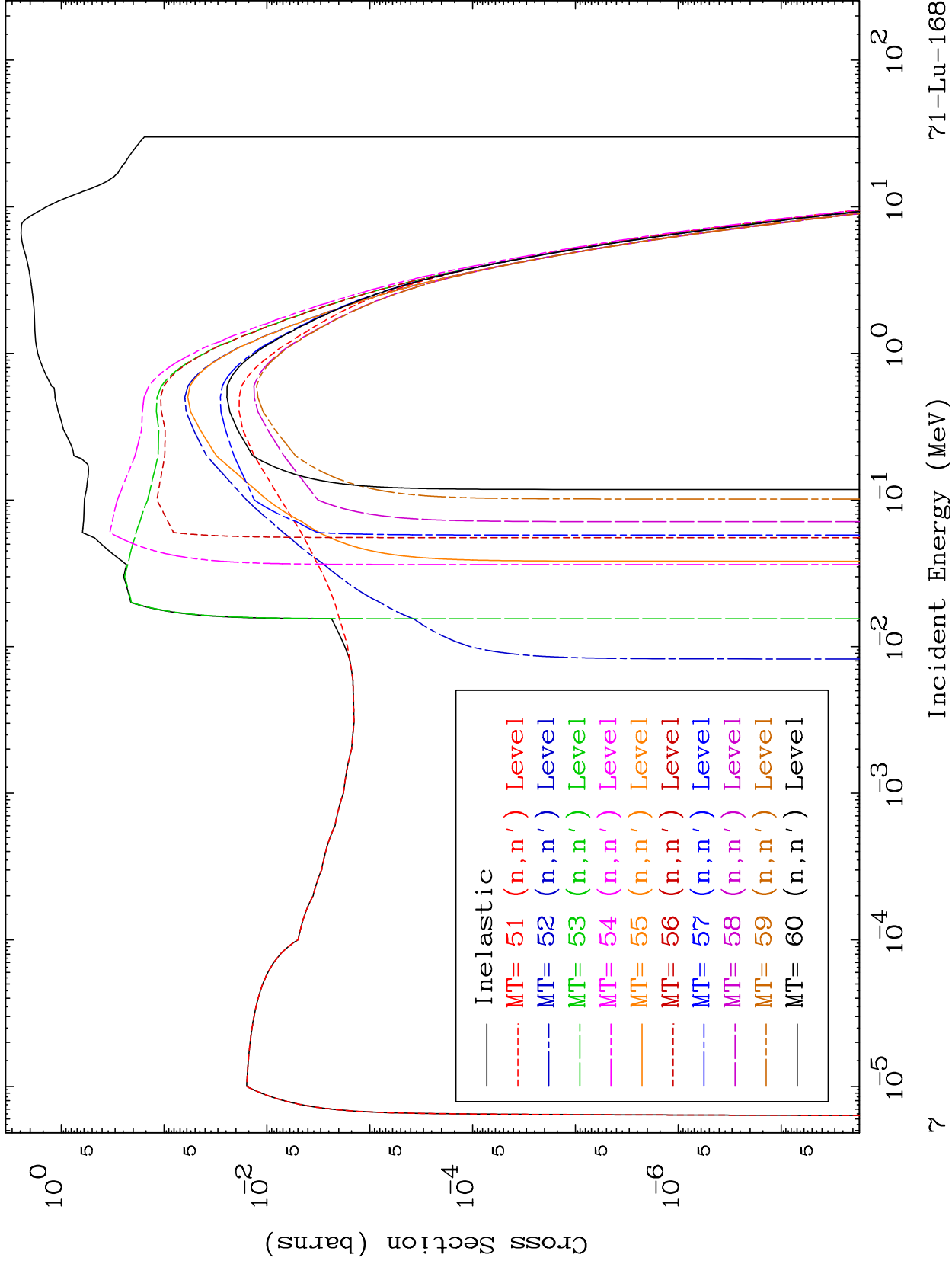




MAT 7105

293 Kelvin Cross Sections  
(n,n') Level

71-Lu-168



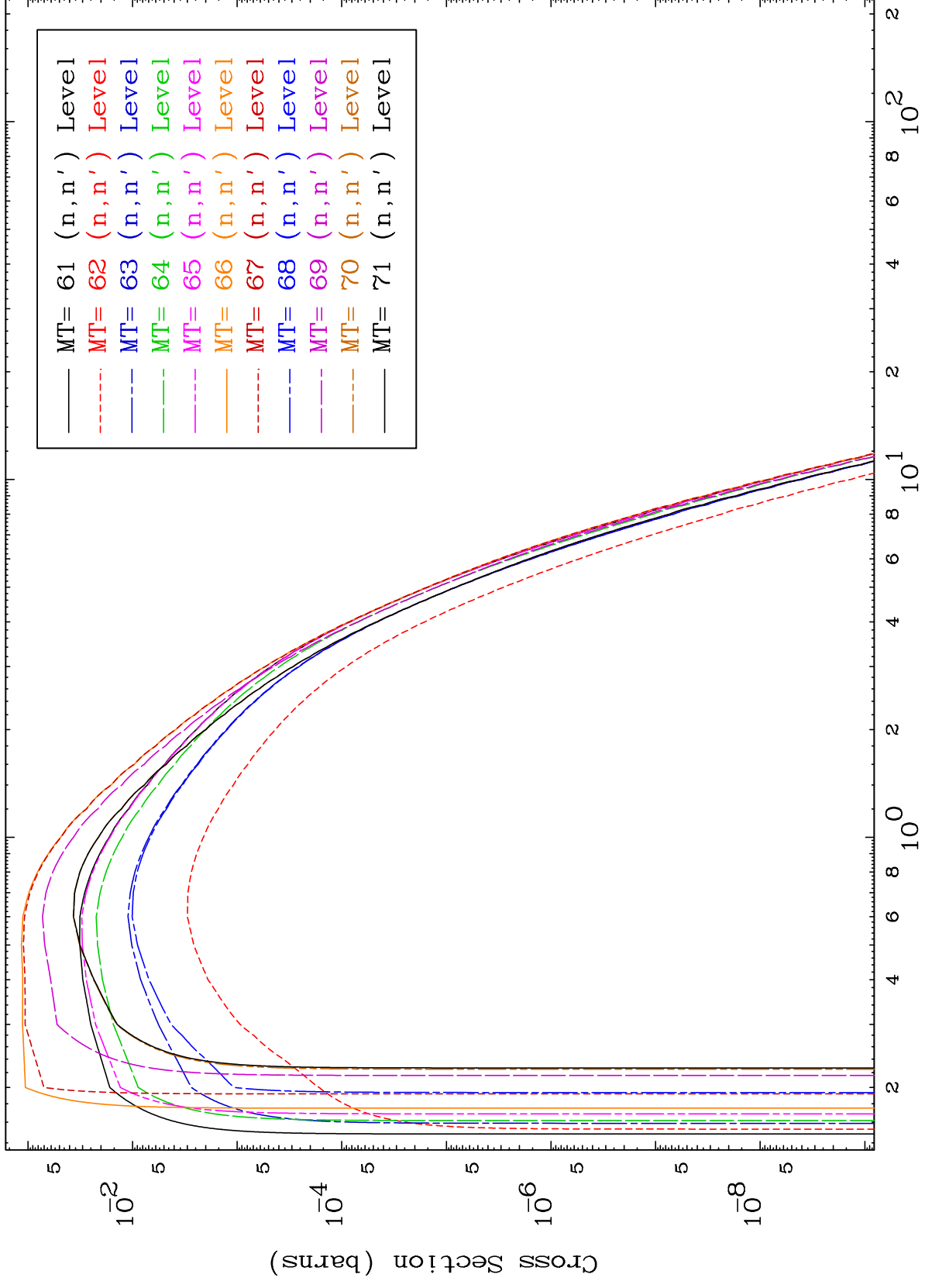


MAT 7105

(n,n') Level

71-Lu-168

293 Kelvin Cross Sections

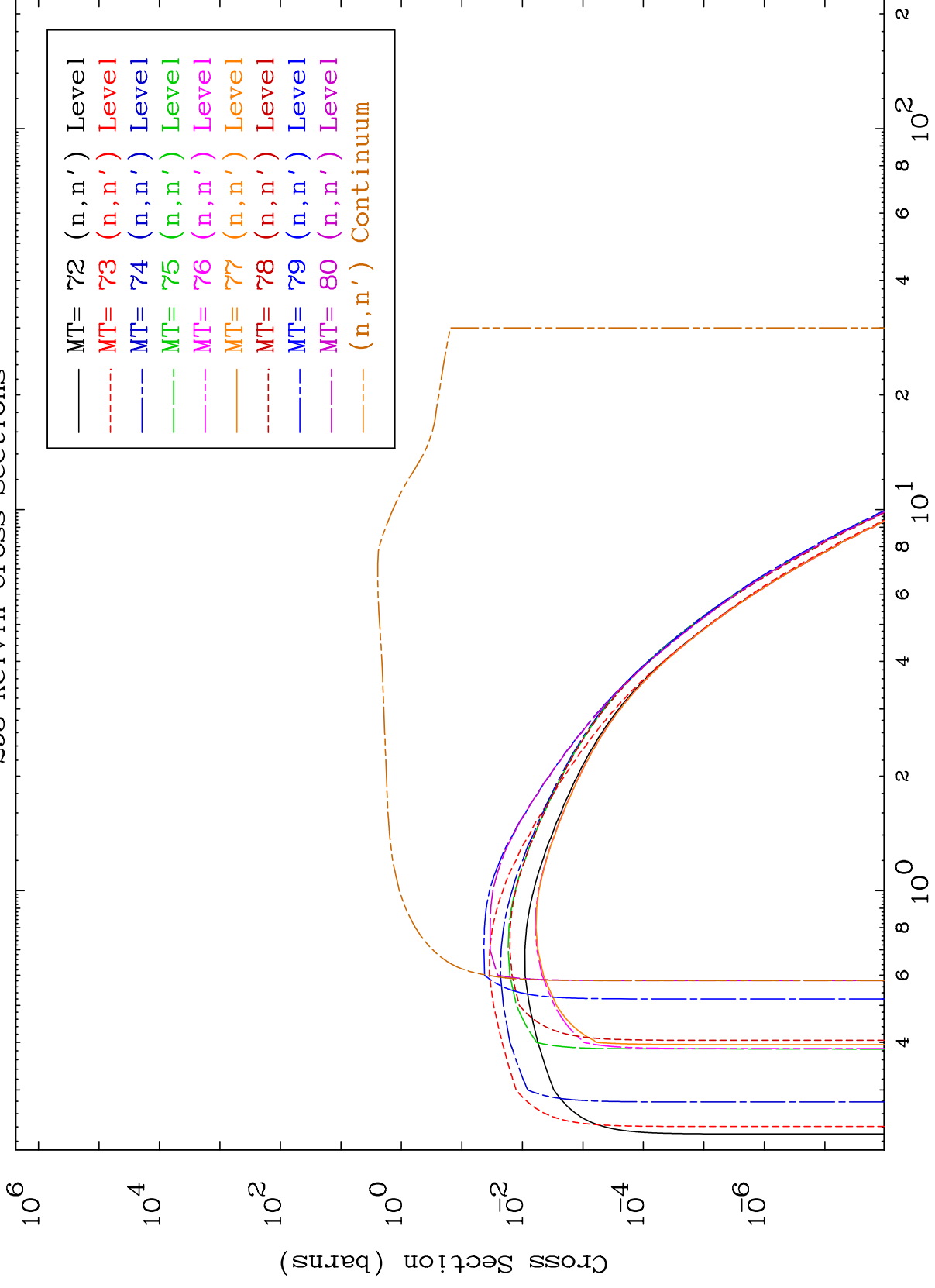


Incident Energy (MeV)

71-Lu-168

8

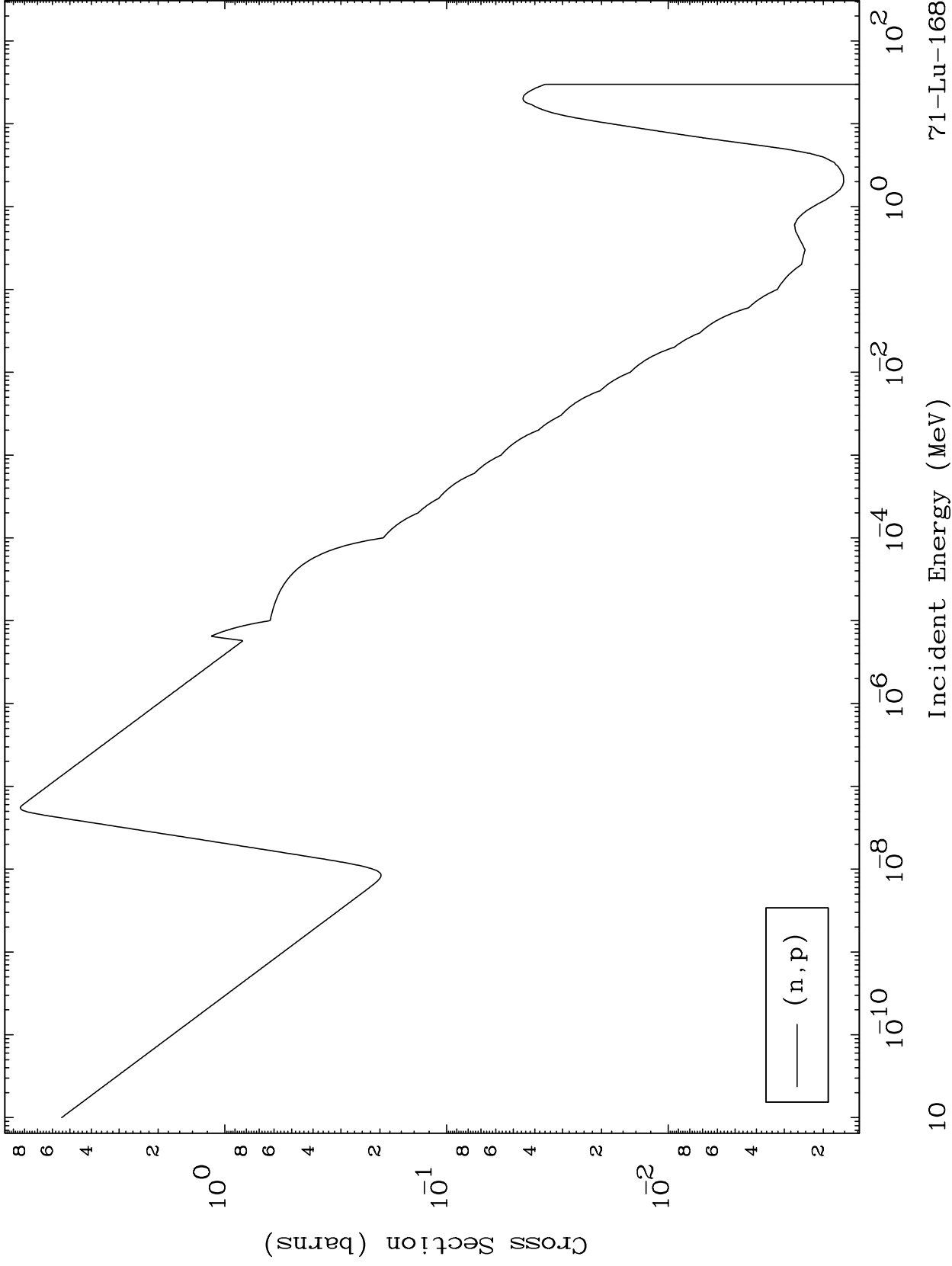
293 Kelvin Cross Sections



MAT 7105

(n,p) Levels  
293 Kelvin Cross Sections

71-Lu-168



10

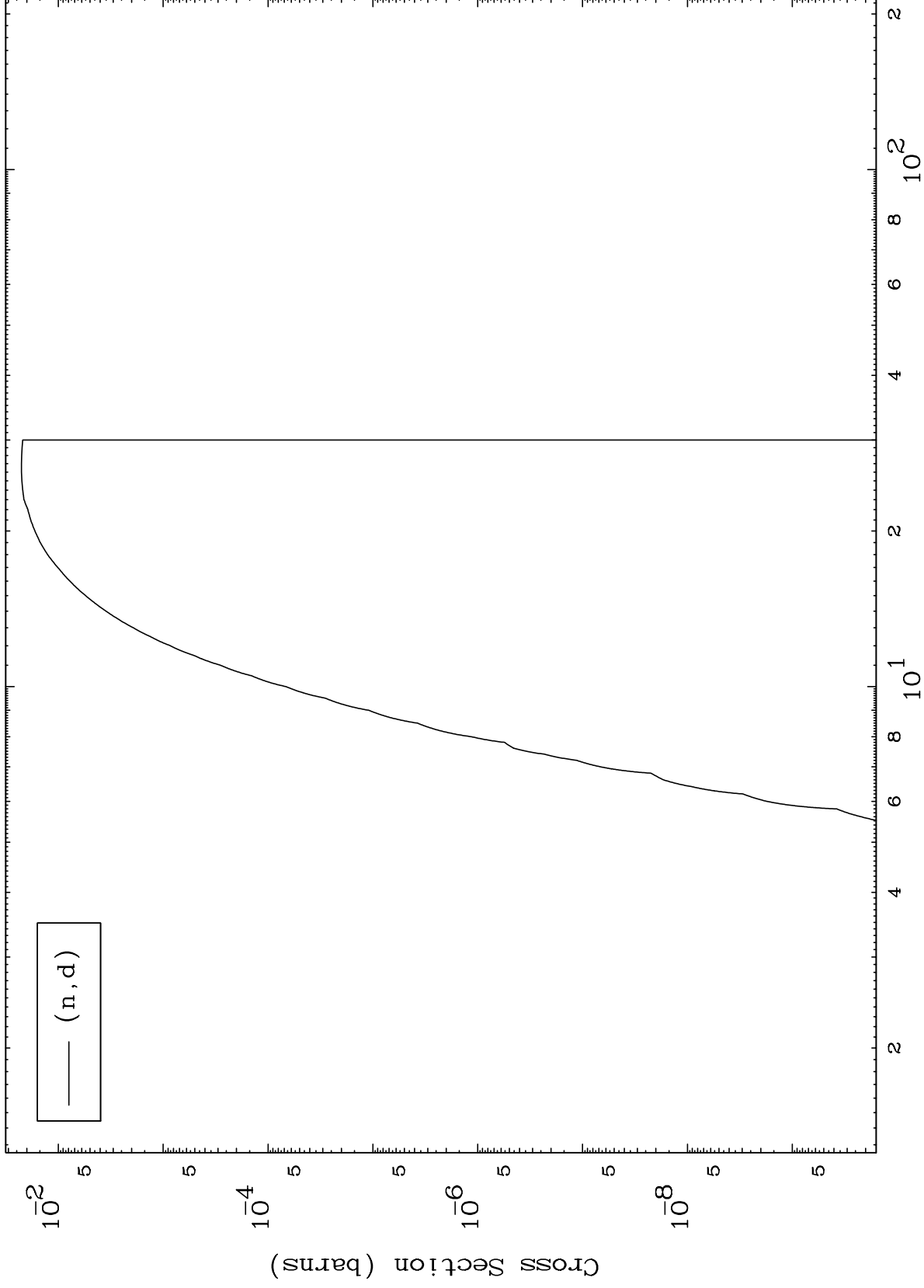
Incident Energy (MeV)

71-Lu-168

MAT 7105

(n,d) Levels  
293 Kelvin Cross Sections

71-Lu-168



11

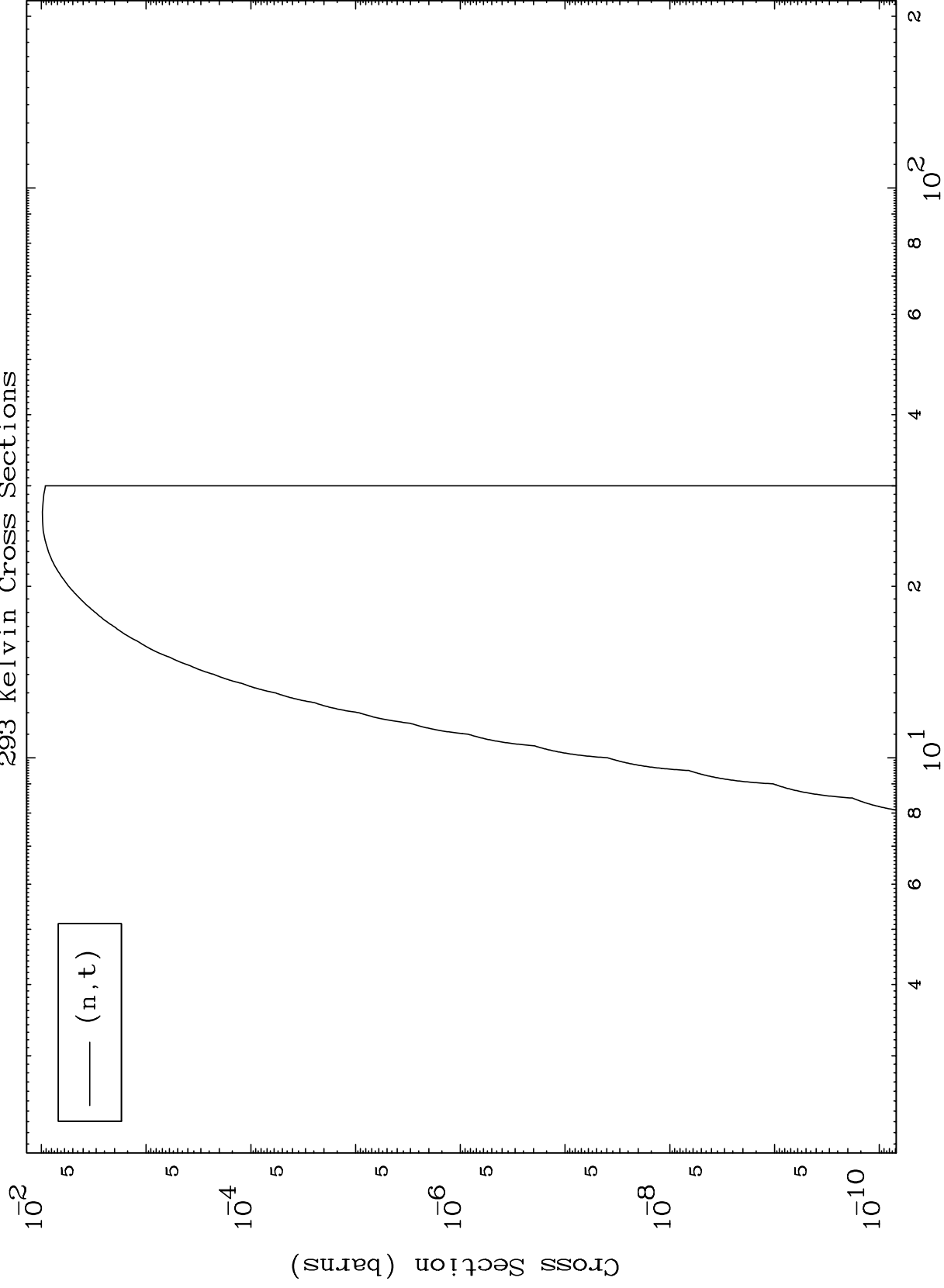
Incident Energy (MeV)

71-Lu-168

MAT 7105

(n,t) Levels  
293 Kelvin Cross Sections

71-Lu-168



12

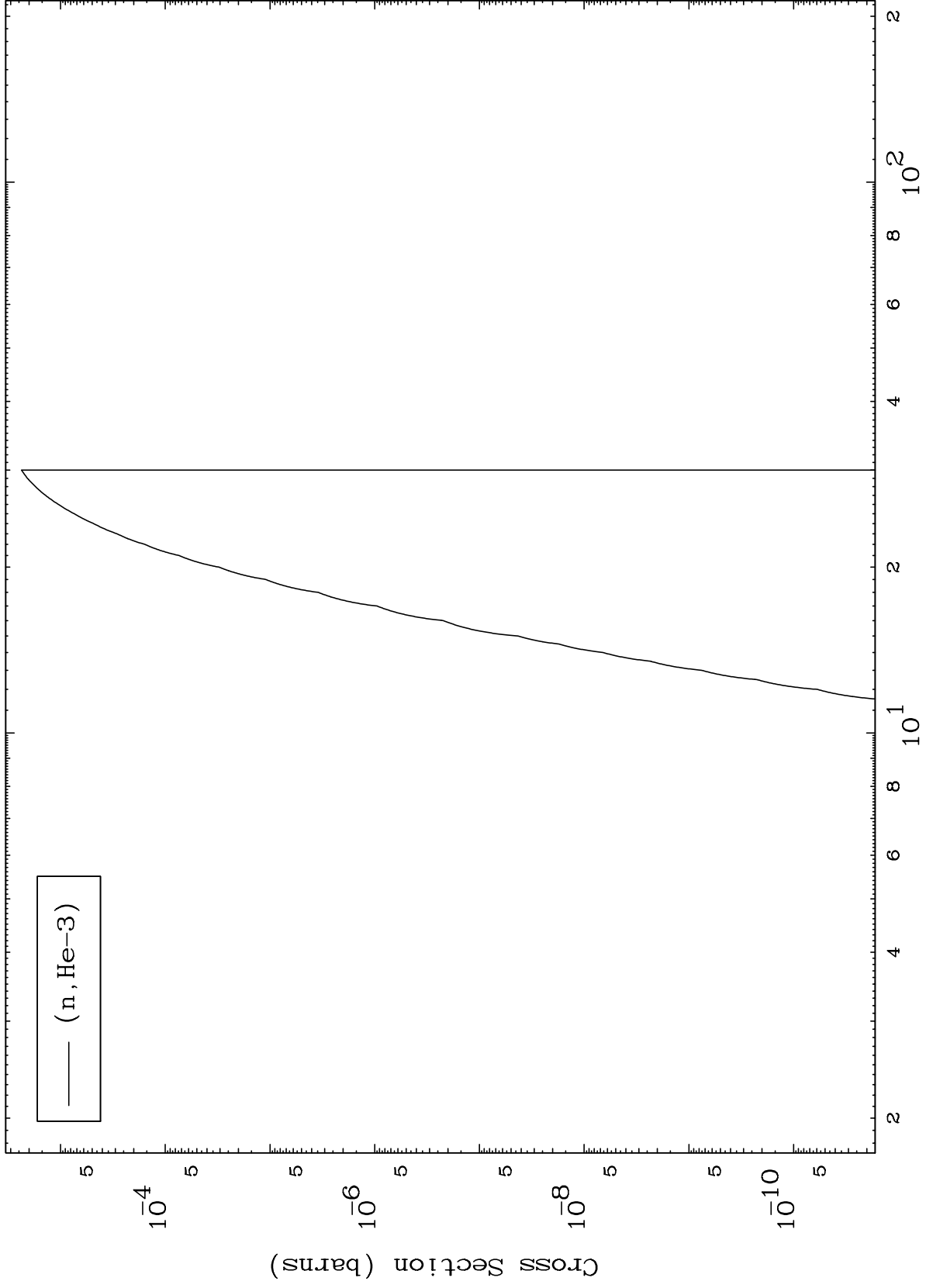
Incident Energy (MeV)

71-Lu-168

MAT 7105

(n,He3) Levels  
293 Kelvin Cross Sections

71-Lu-168



13

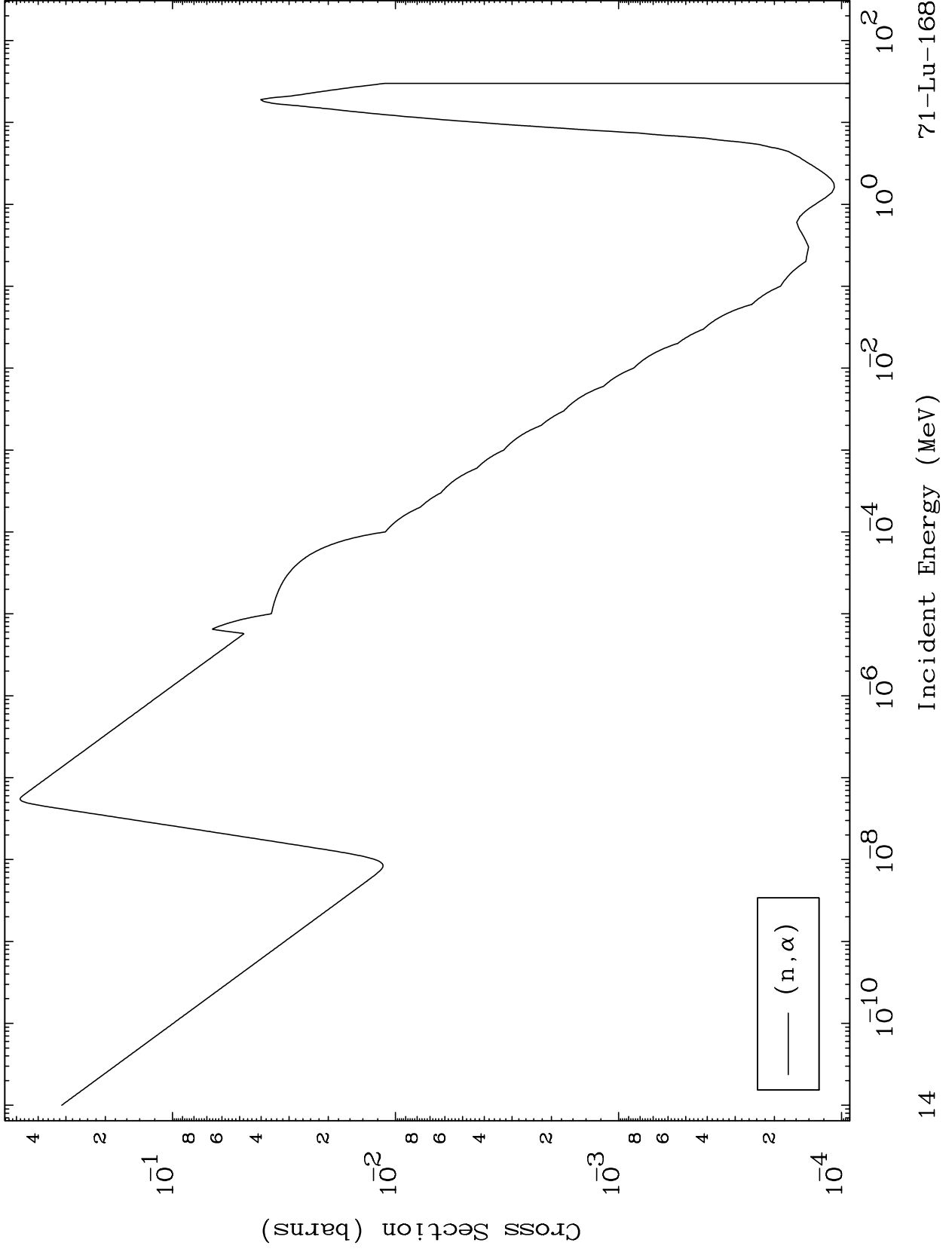
Incident Energy (MeV)

71-Lu-168

MAT 7105

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

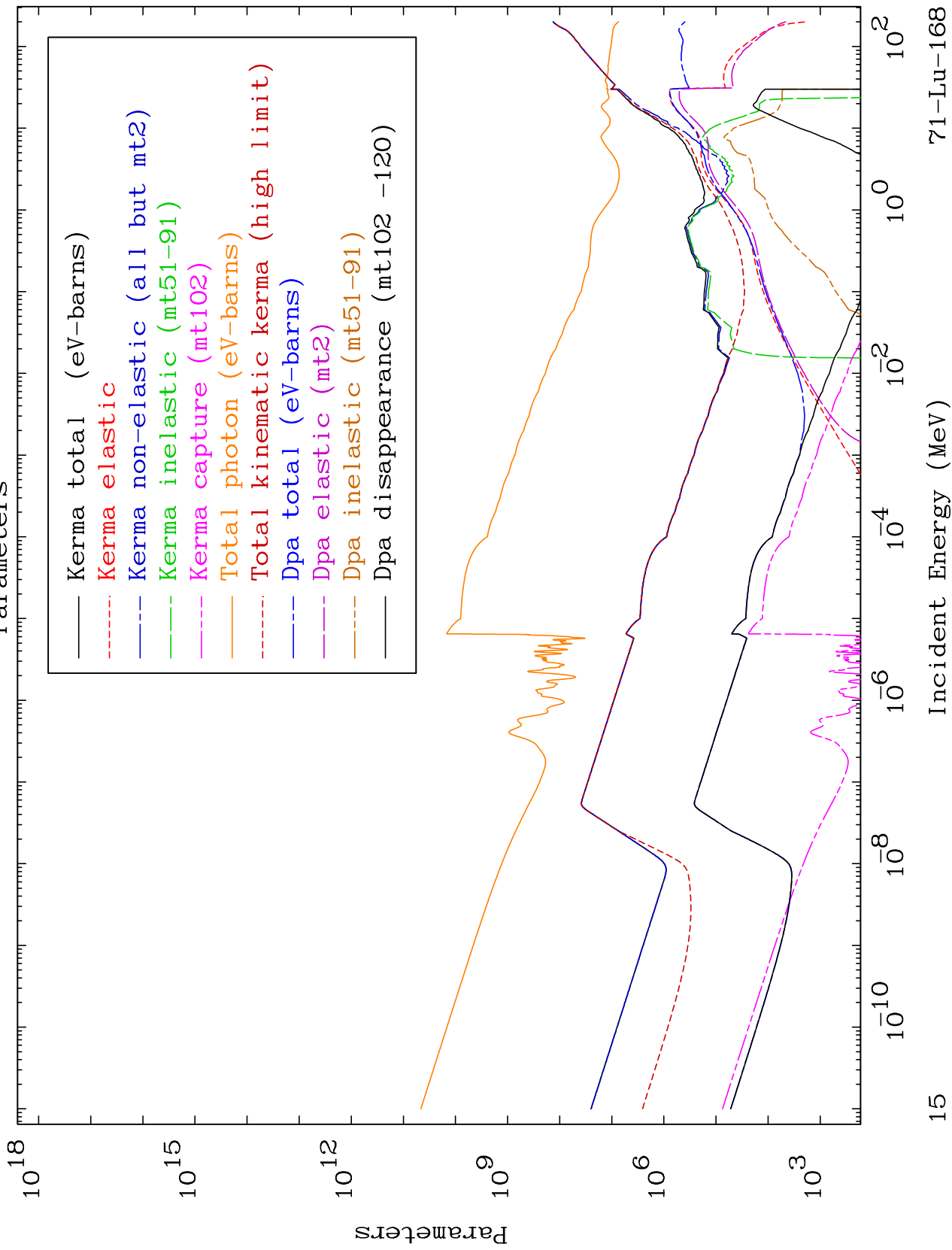
71-Lu-168



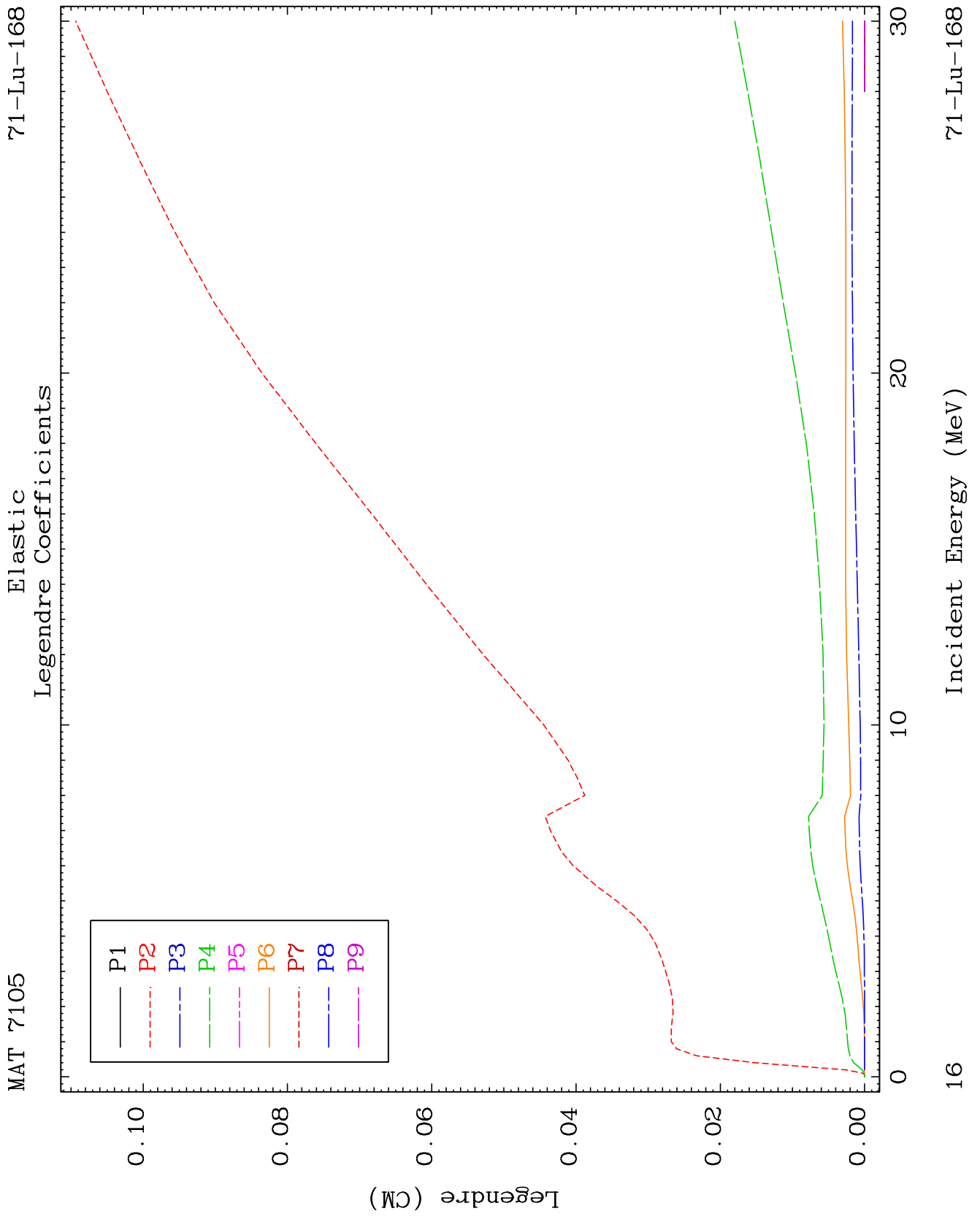
14

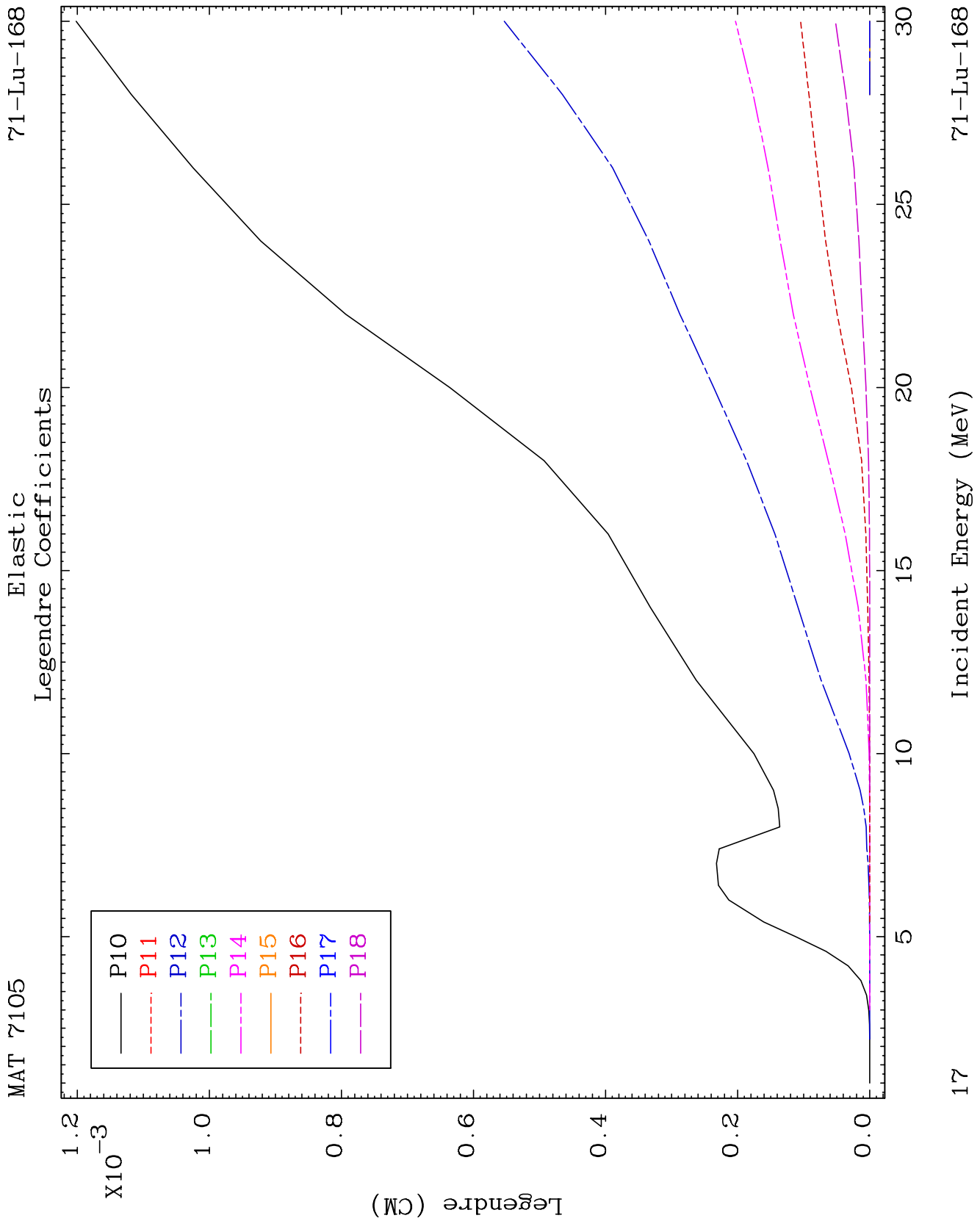
Incident Energy (MeV)

71-Lu-168







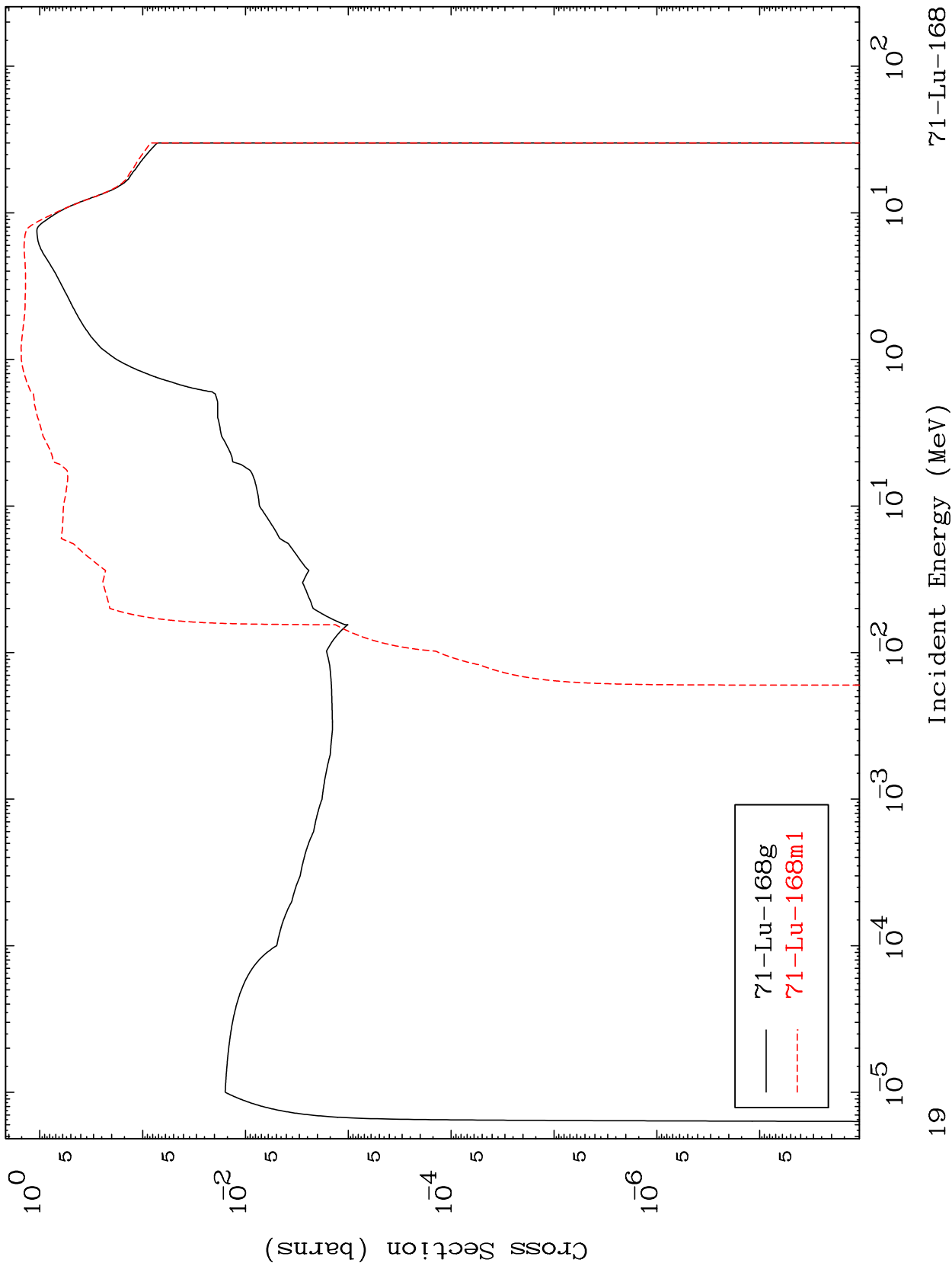




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71-Lu-168

Inelastic  
Radionuclide Production Cross Section

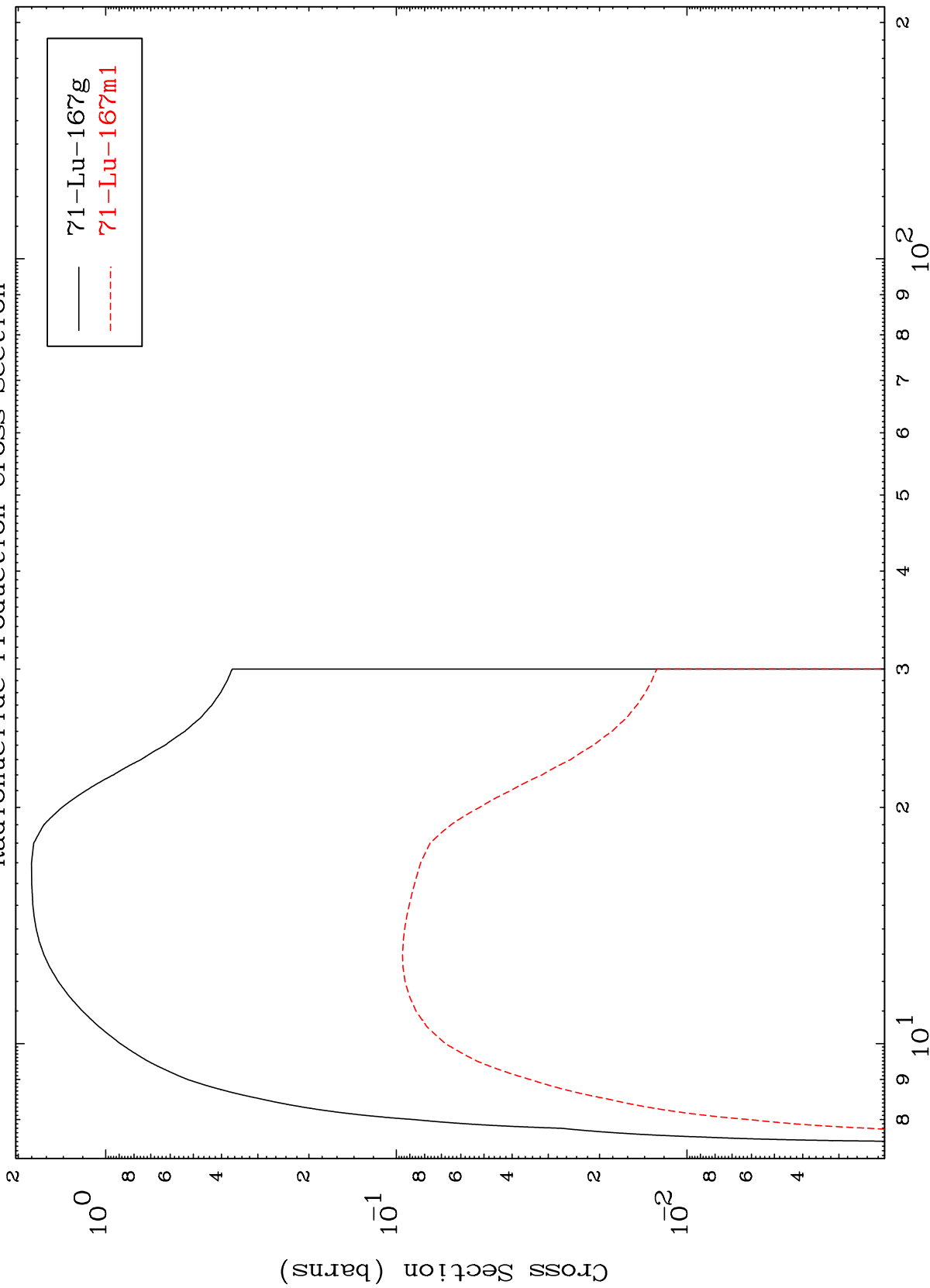


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71-Lu-168

(n,2n)

Radionuclide Production Cross Section



71-Lu-168

Incident Energy (MeV)

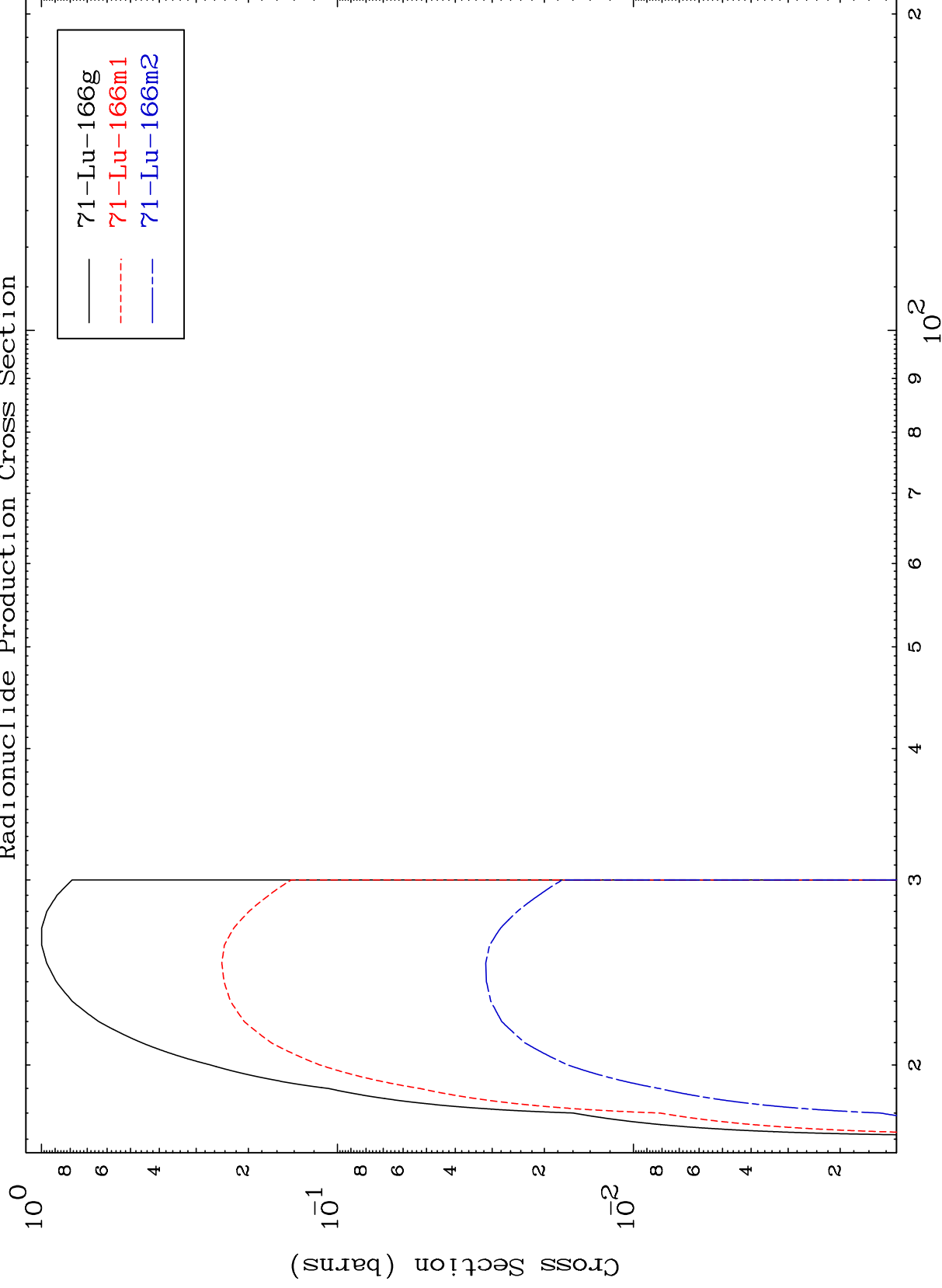
20

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(n,3n)

71-Lu-168

Radionuclide Production Cross Section



21

Incident Energy (MeV)

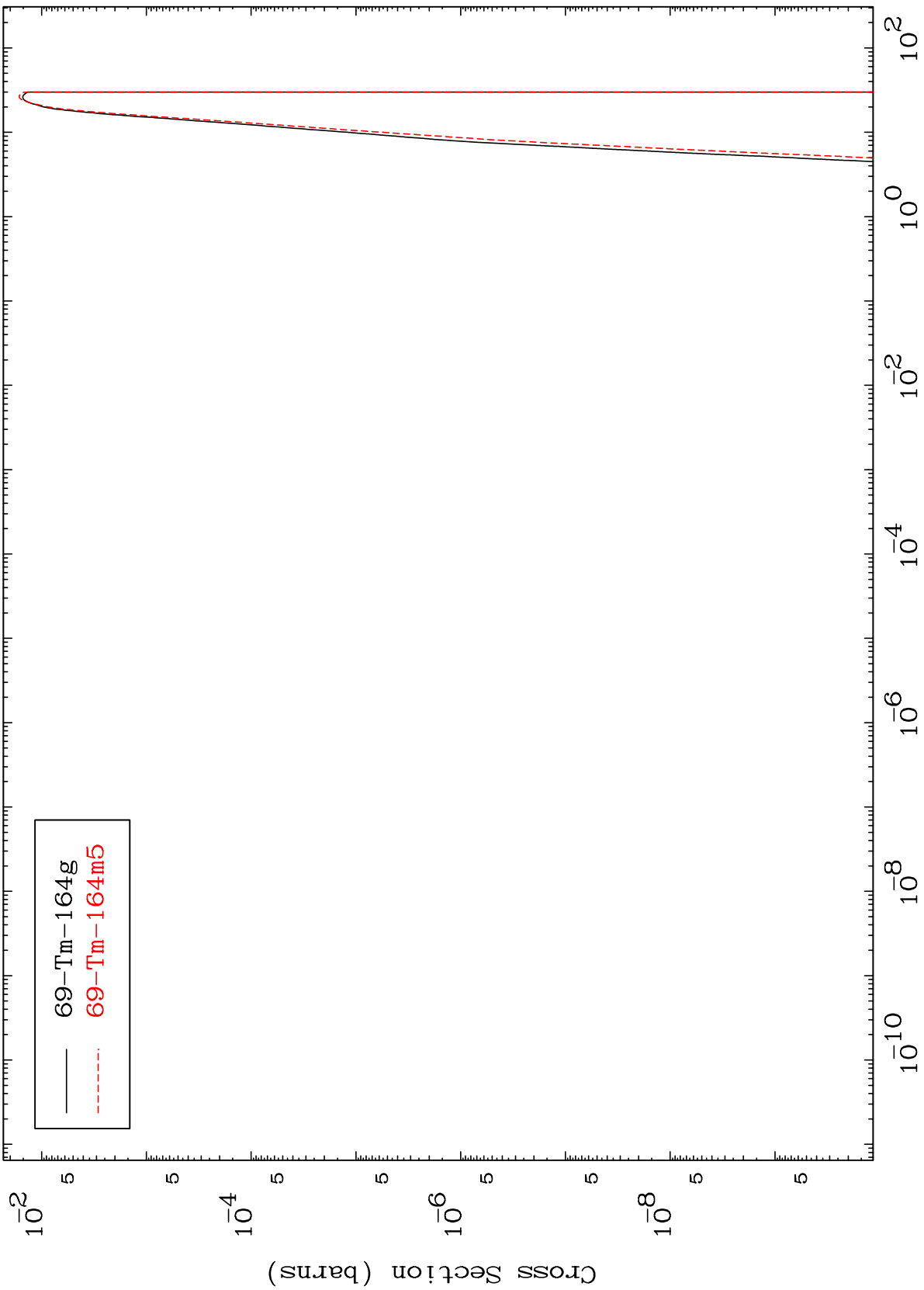
71-Lu-168

MAT 7105

$(n, n') \alpha$

$^{71}\text{Lu-168}$

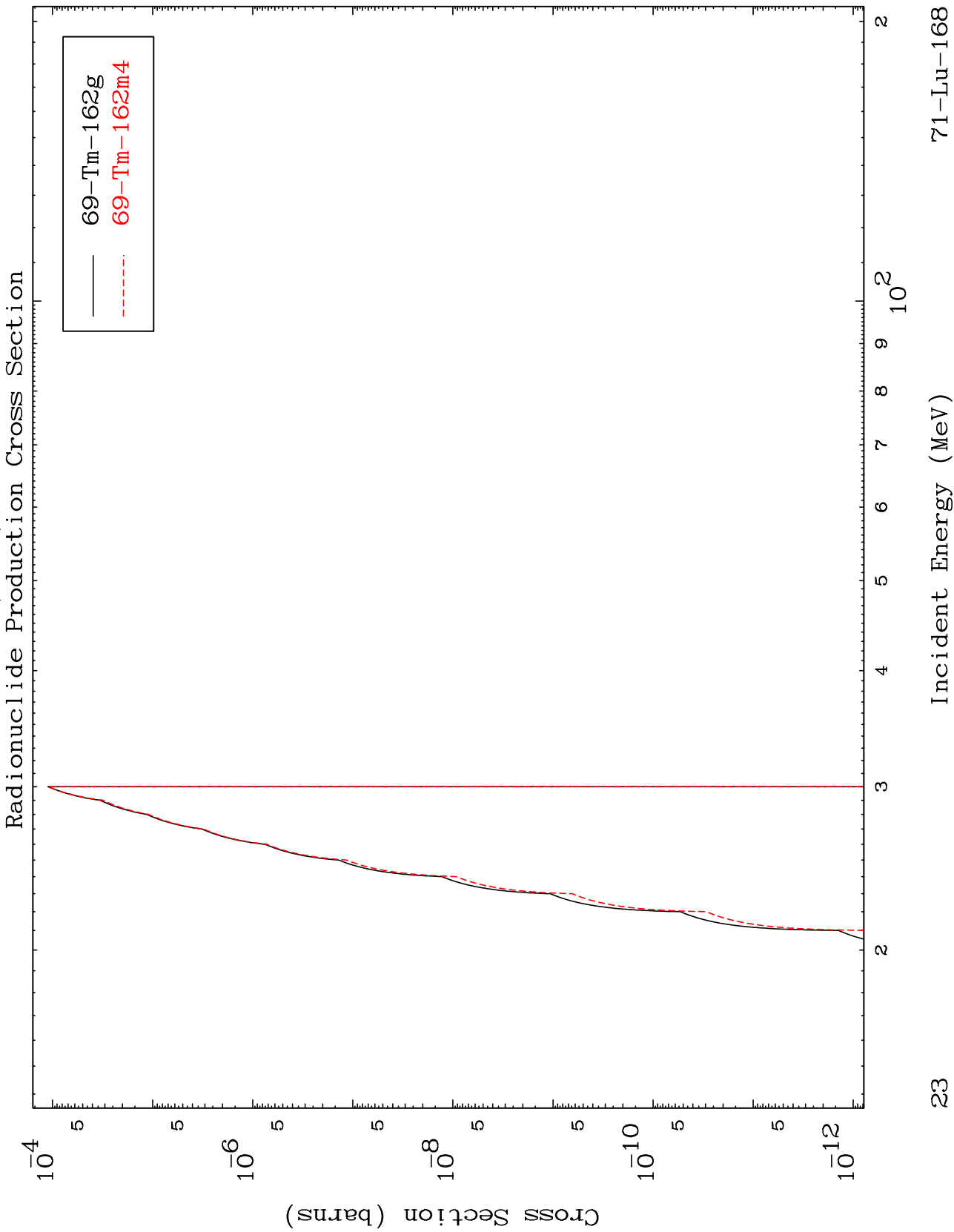
Radionuclide Production Cross Section



22

Incident Energy (MeV)

$^{71}\text{Lu-168}$



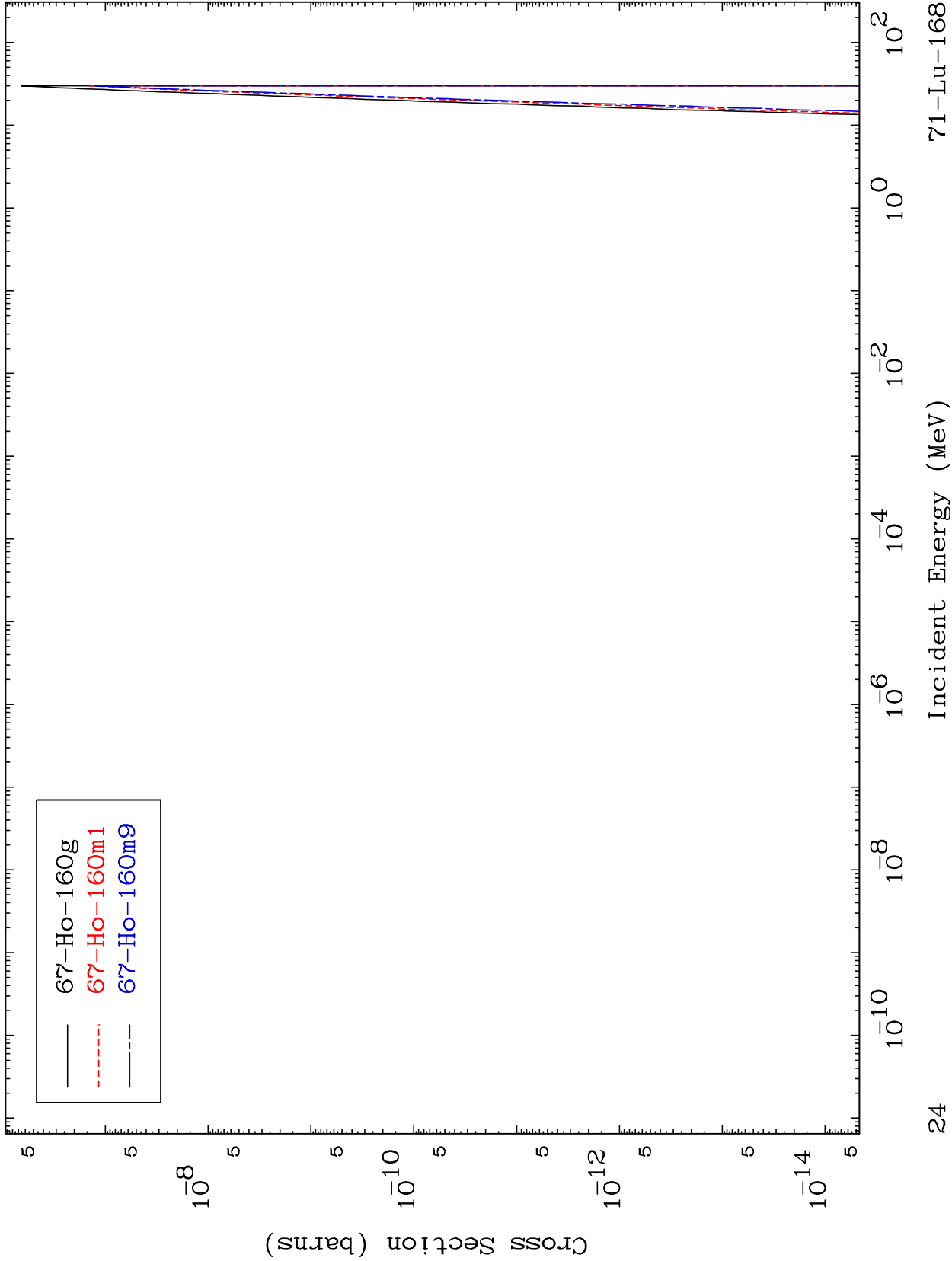


MAT 7105

(n,n') 2 $\alpha$

<sup>71</sup>Lu-168

Radionuclide Production Cross Section

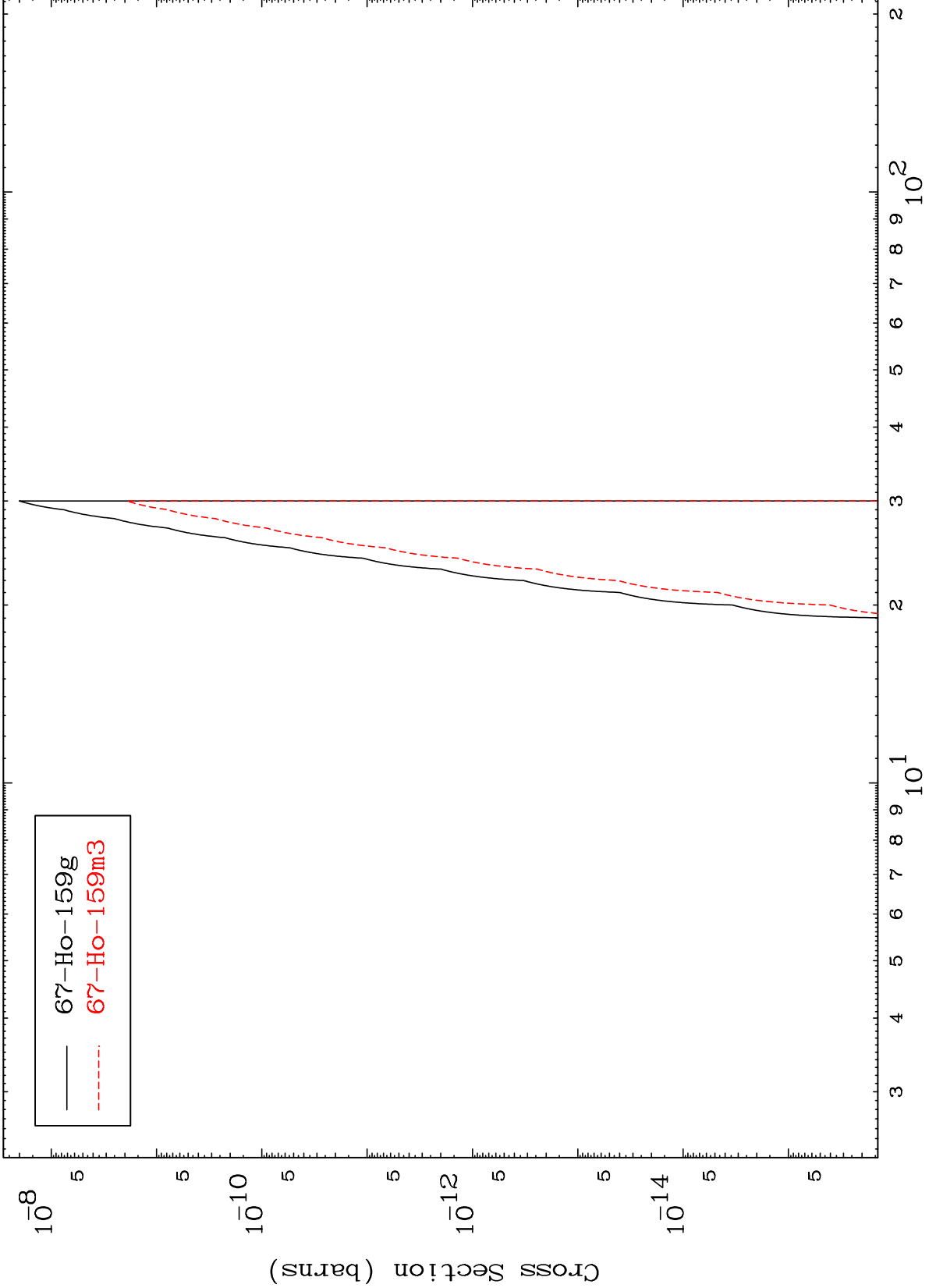


MAT 7105

$(n,2n) 2\alpha$

$^{71}\text{Lu-168}$

Radionuclide Production Cross Section



— 67-Ho-159g  
- - - 67-Ho-159m3

25

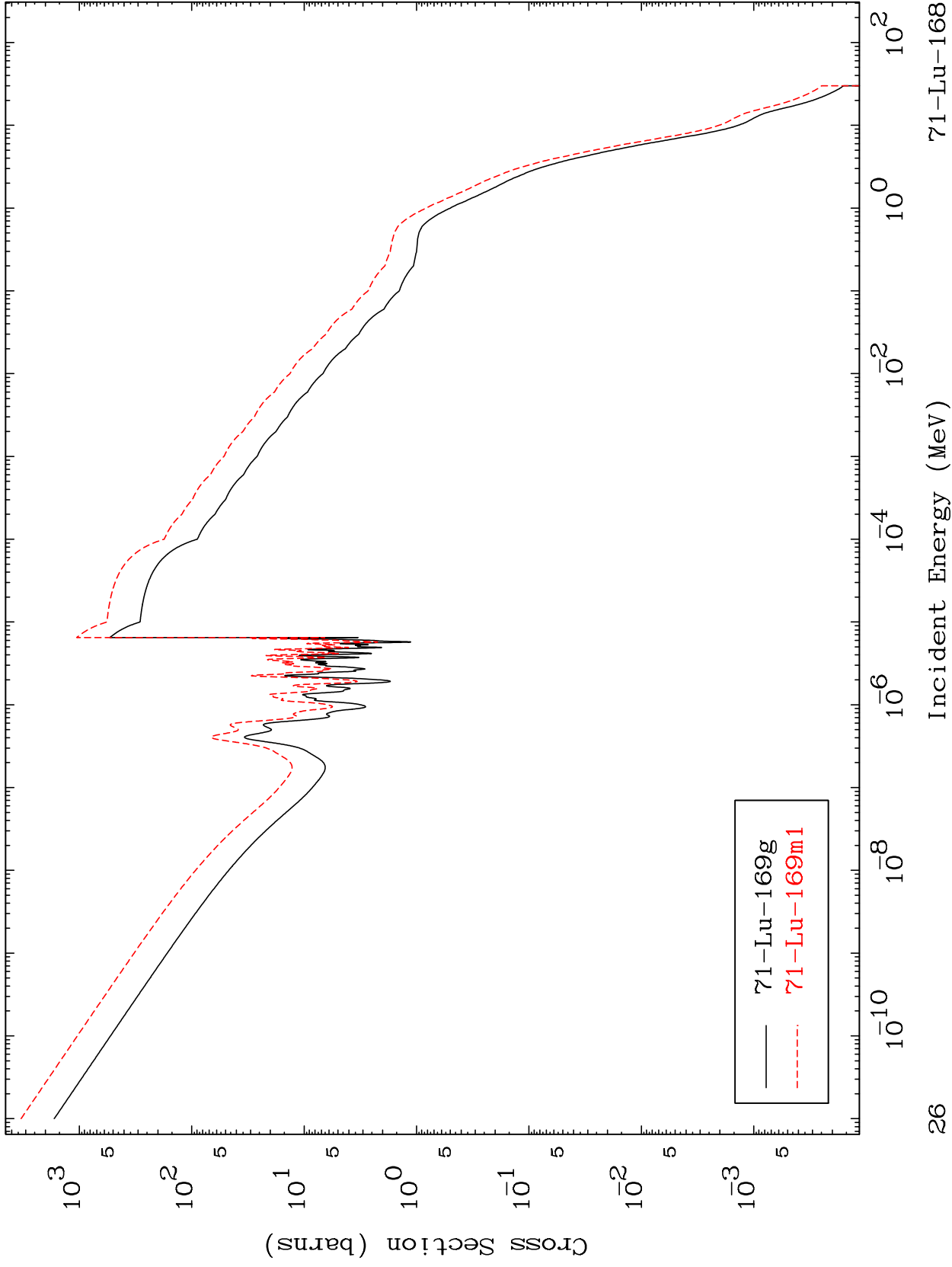
Incident Energy (MeV)

$^{71}\text{Lu-168}$

MAT 7105

<sup>71</sup>Lu-168

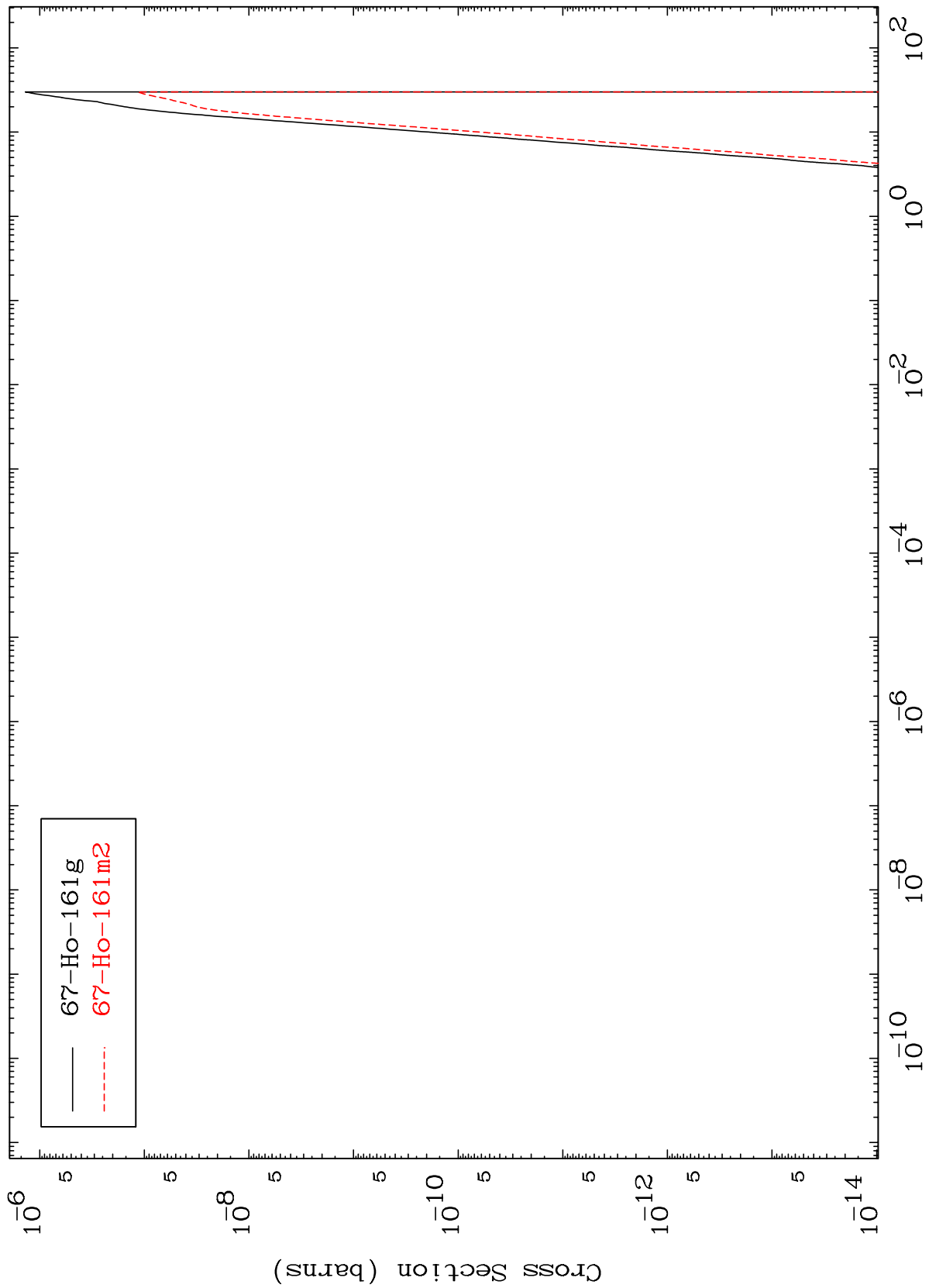
(n,γ)  
Radionuclide Production Cross Section



MAT 7105

<sup>71</sup>Lu-168

Radionuclide Production Cross Section



27

<sup>71</sup>Lu-168

Incident Energy (MeV)