

Program EVALPLOT
(Version 2017-1)

by

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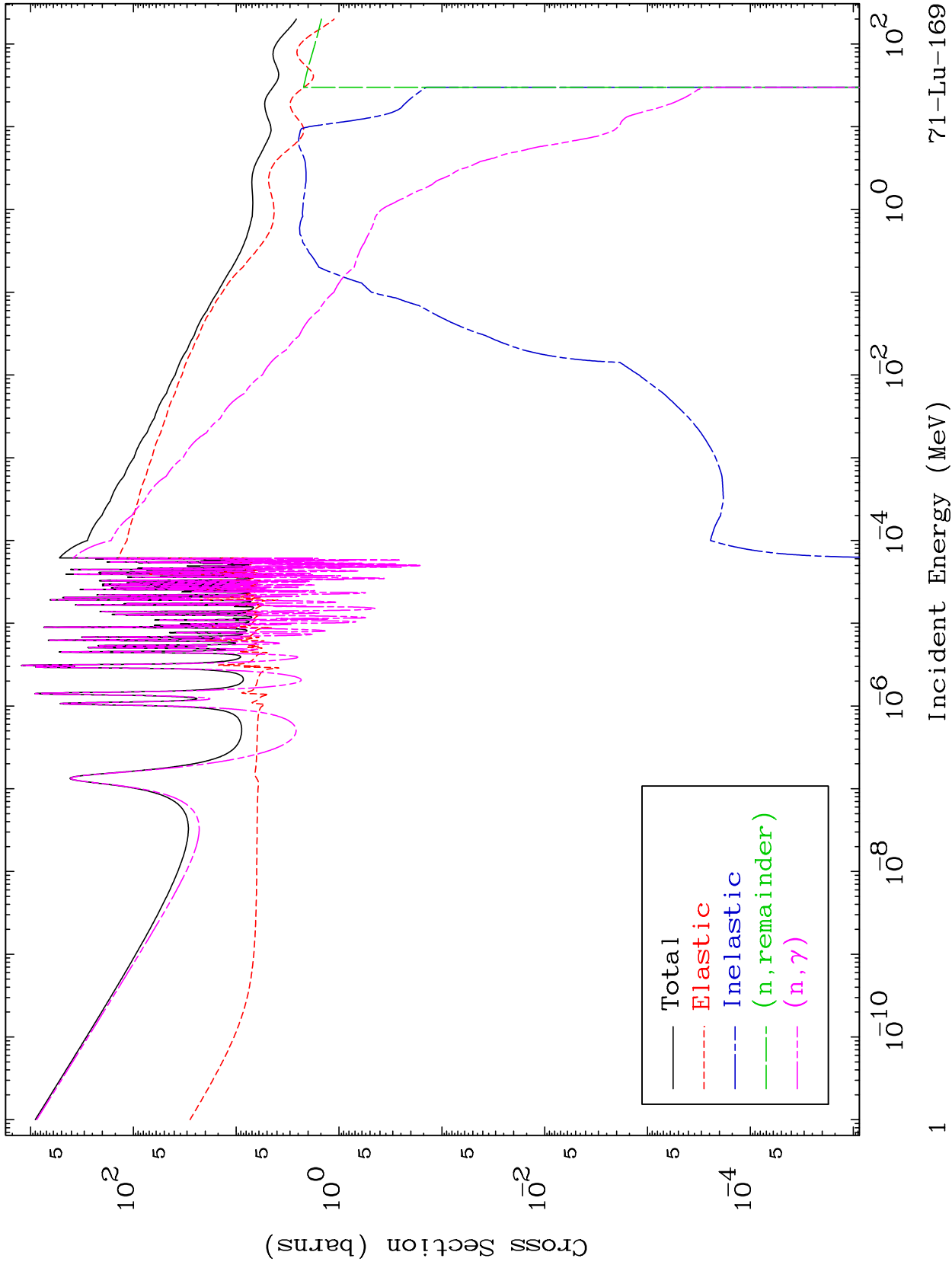
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7108

Major
293 Kelvin Cross Sections

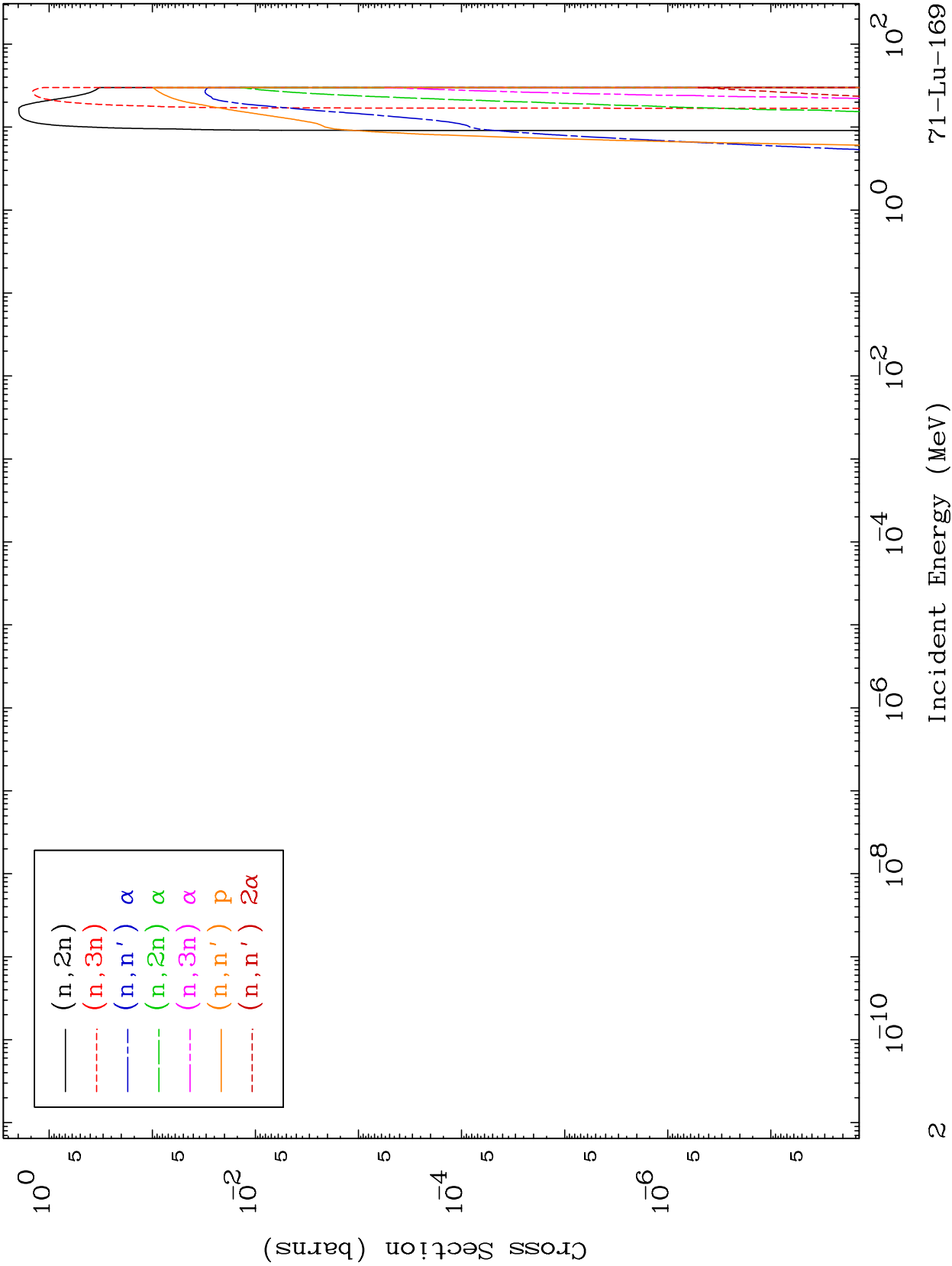
71-Lu-169

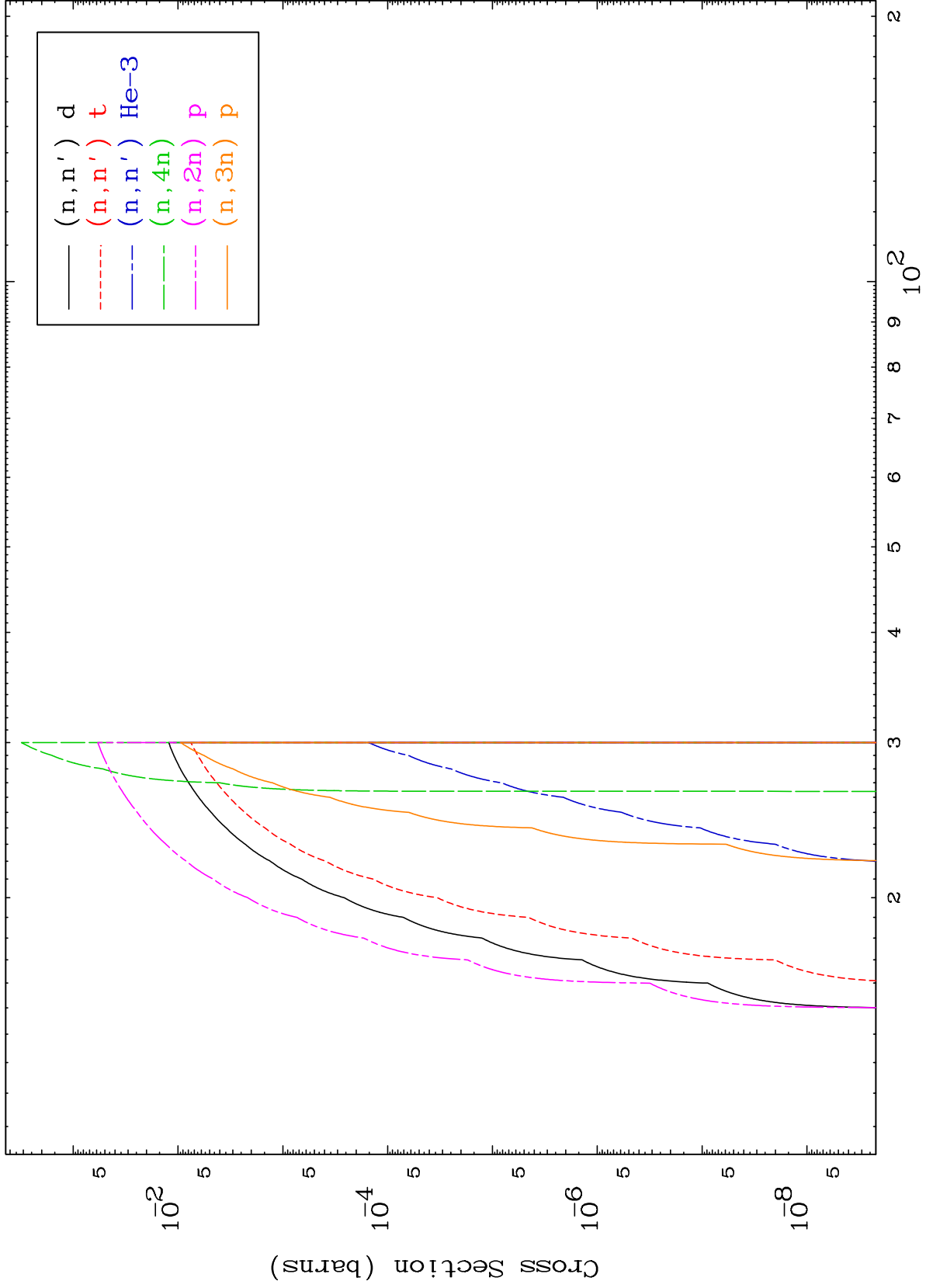


MAT 7108

Neutron Production
293 Kelvin Cross Sections

71-Lu-169

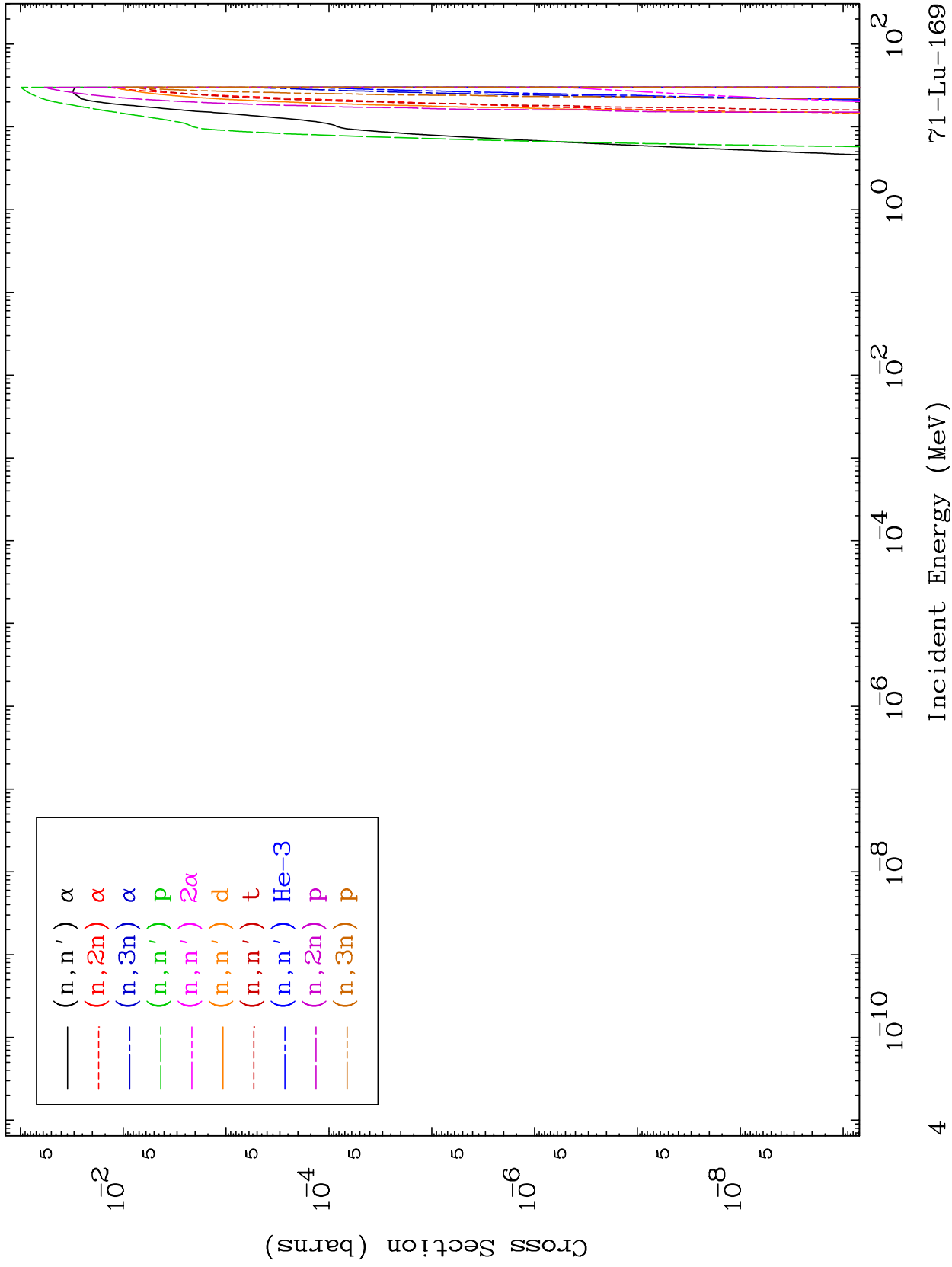


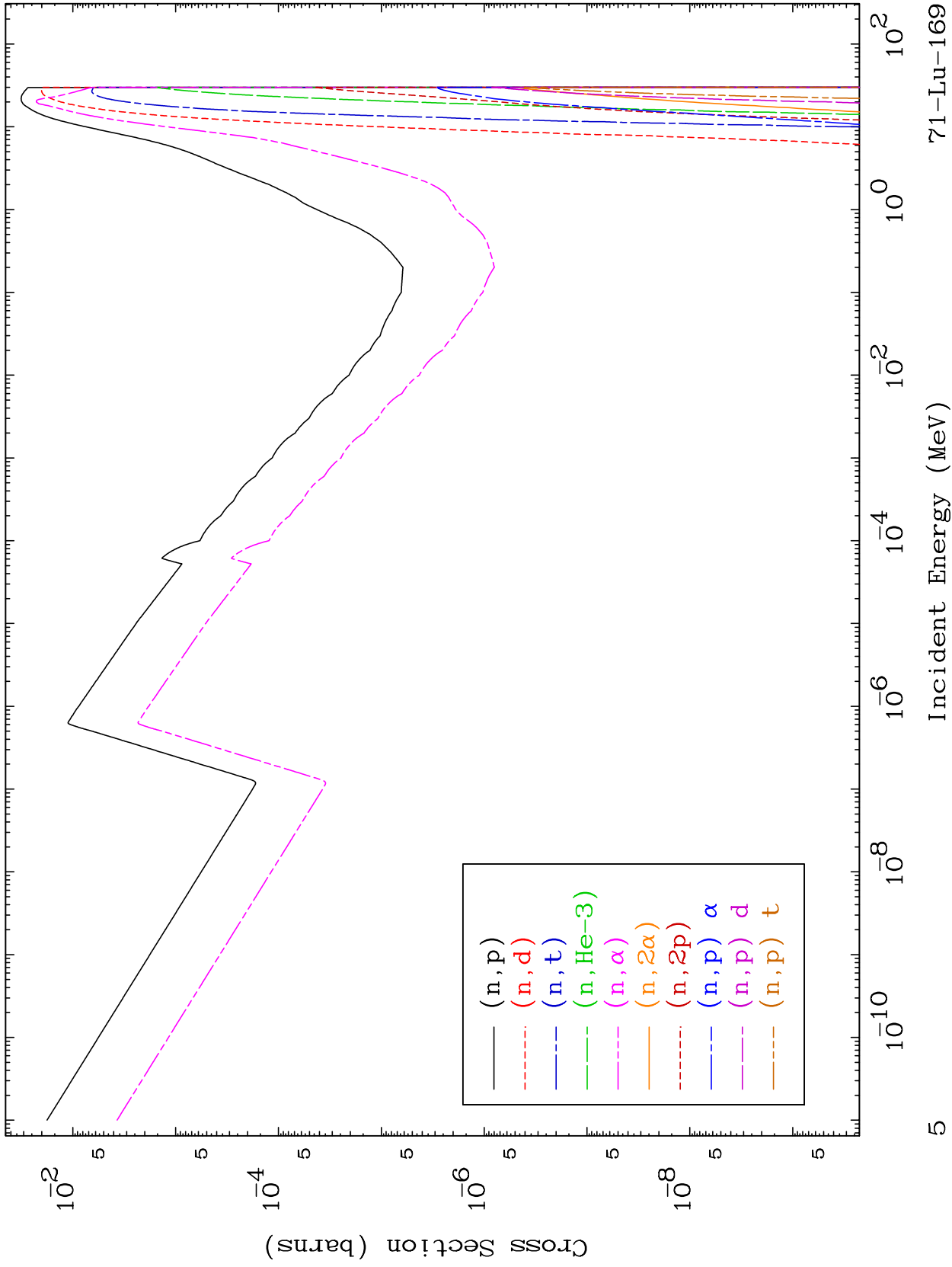


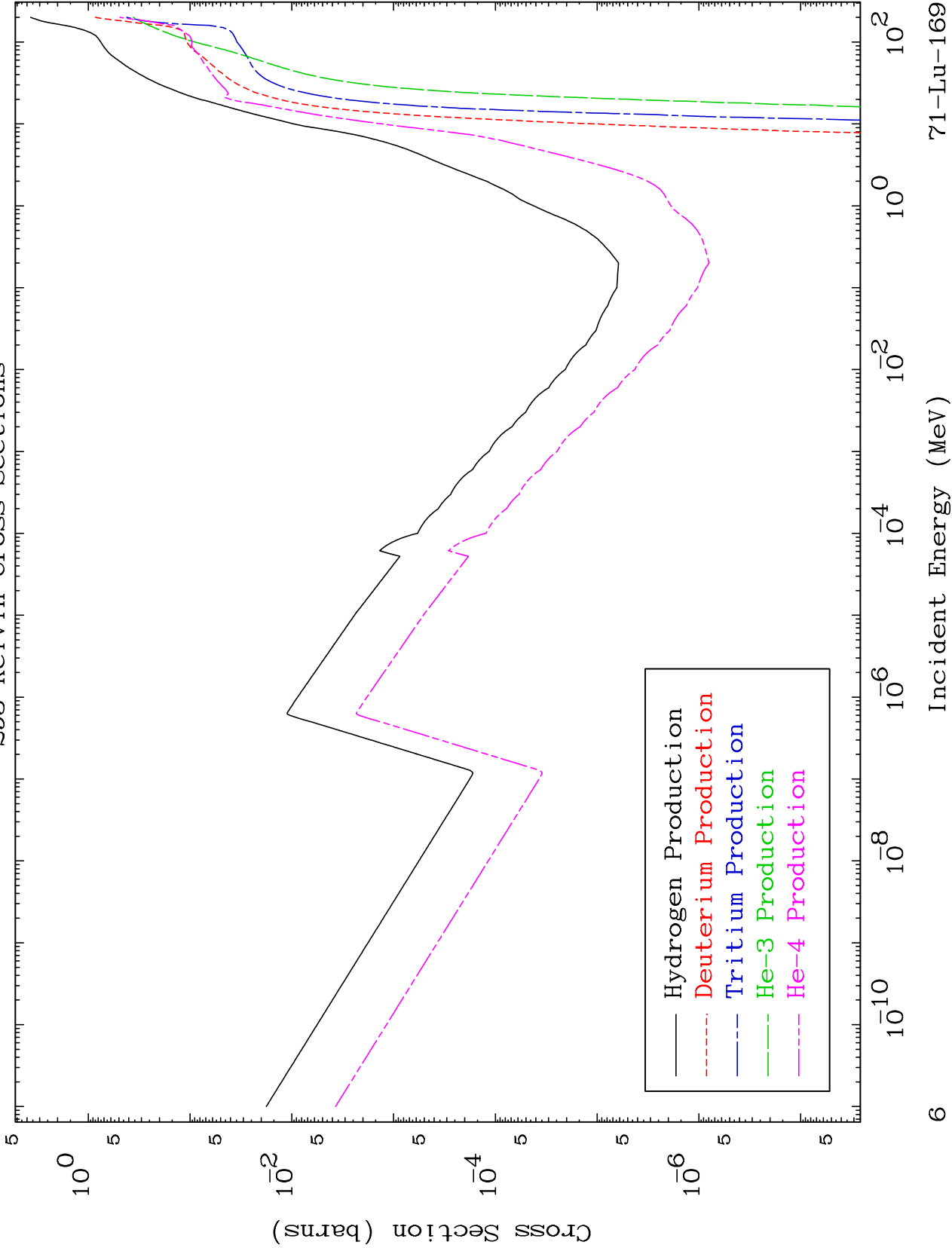
MAT 7108

Charged Particle
293 Kelvin Cross Sections

71-Lu-169





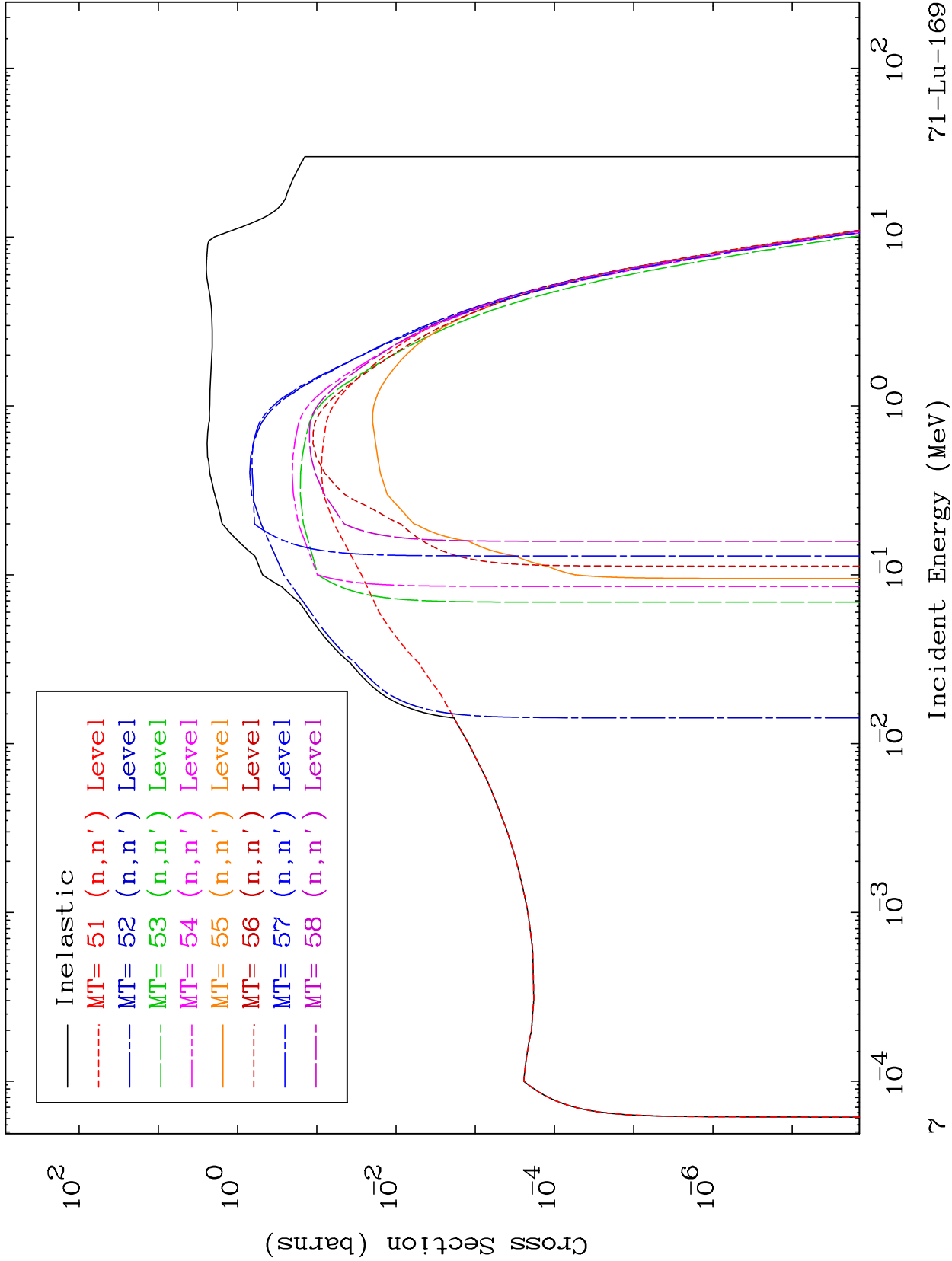


MAT 7108

293 Kelvin Cross Sections

(n,n') Level

71-Lu-169

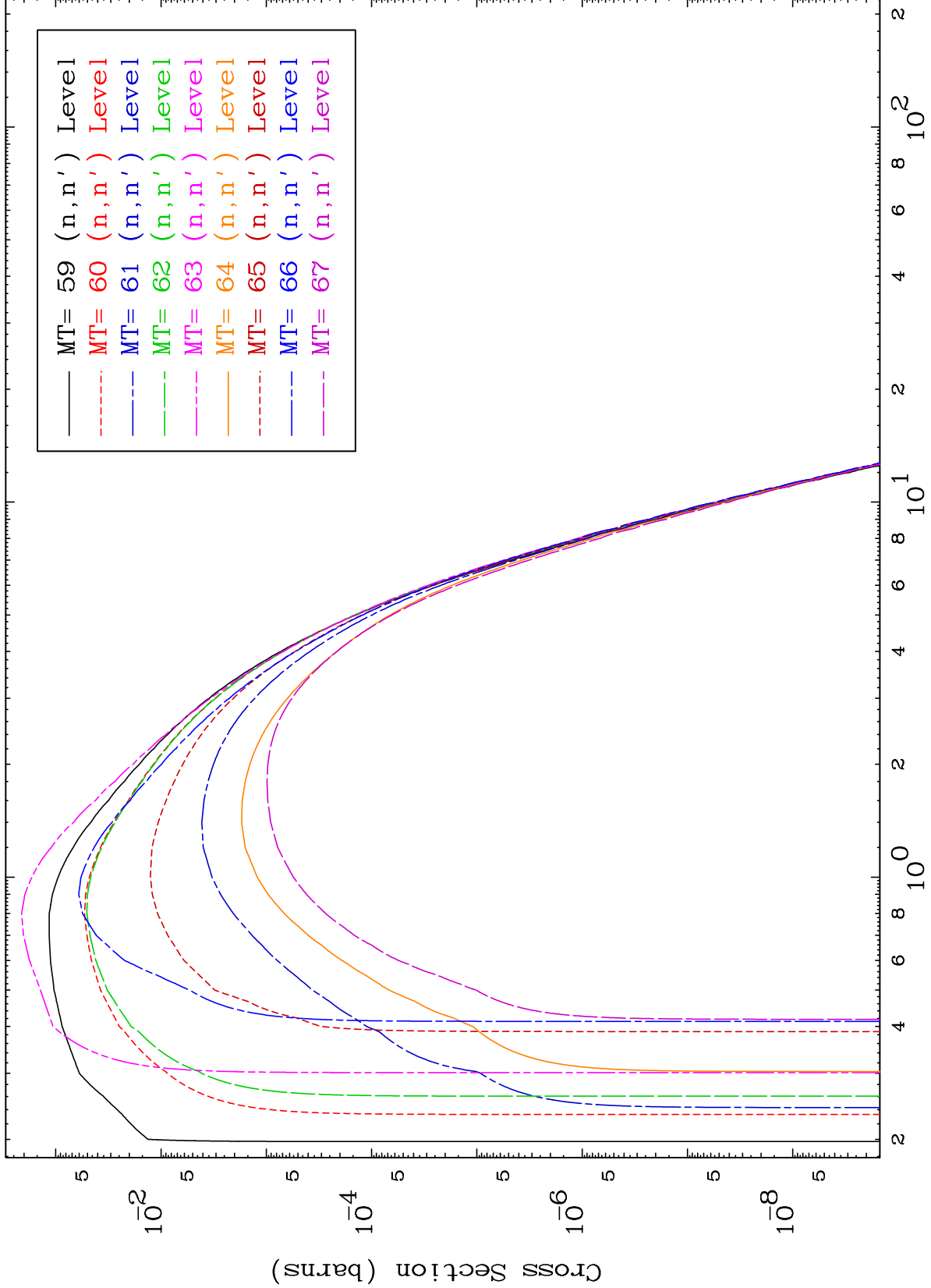


MAT 7108

(n,n') Level

293 Kelvin Cross Sections

71-Lu-169

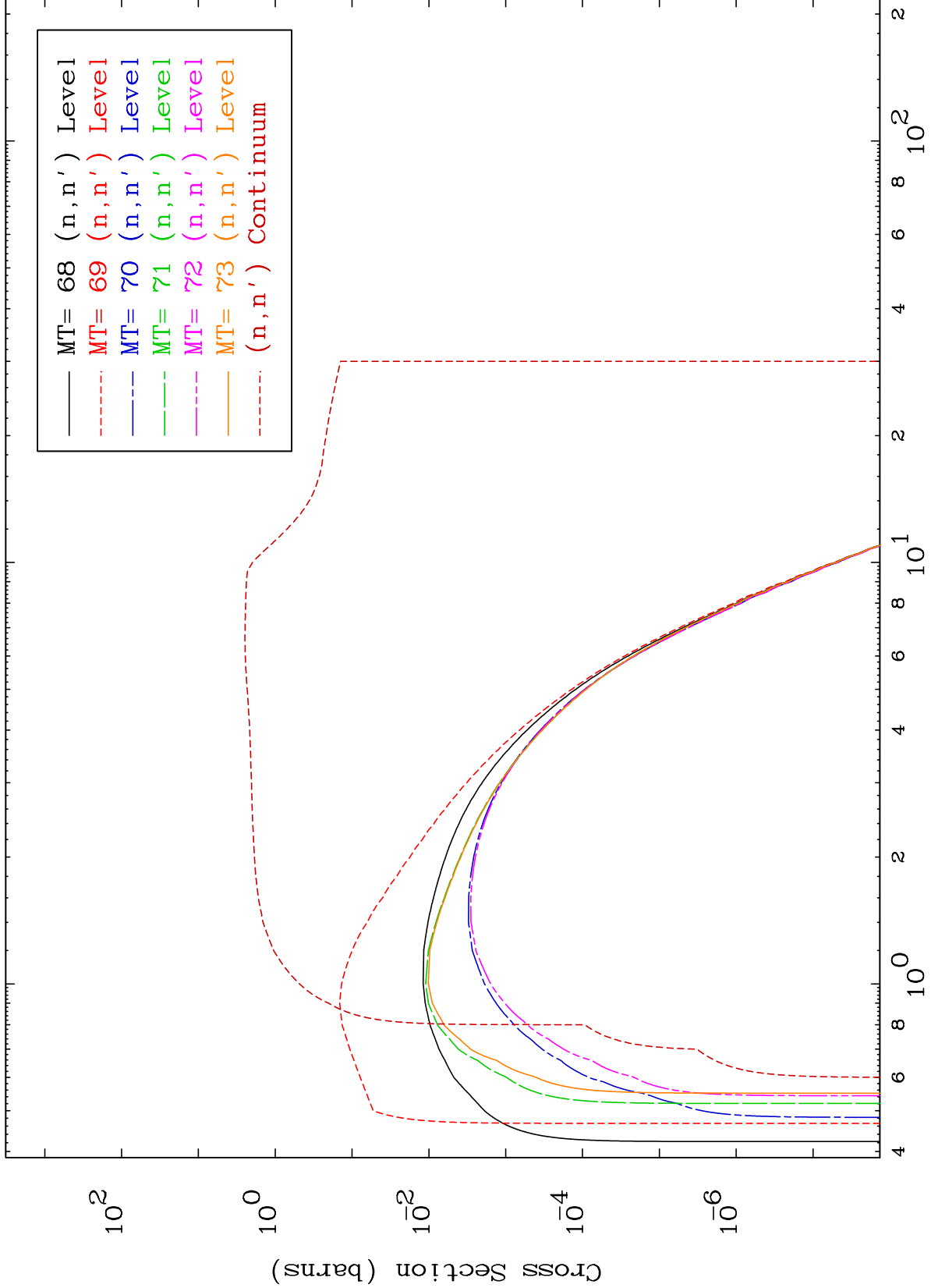


Incident Energy (MeV)

71-Lu-169

8

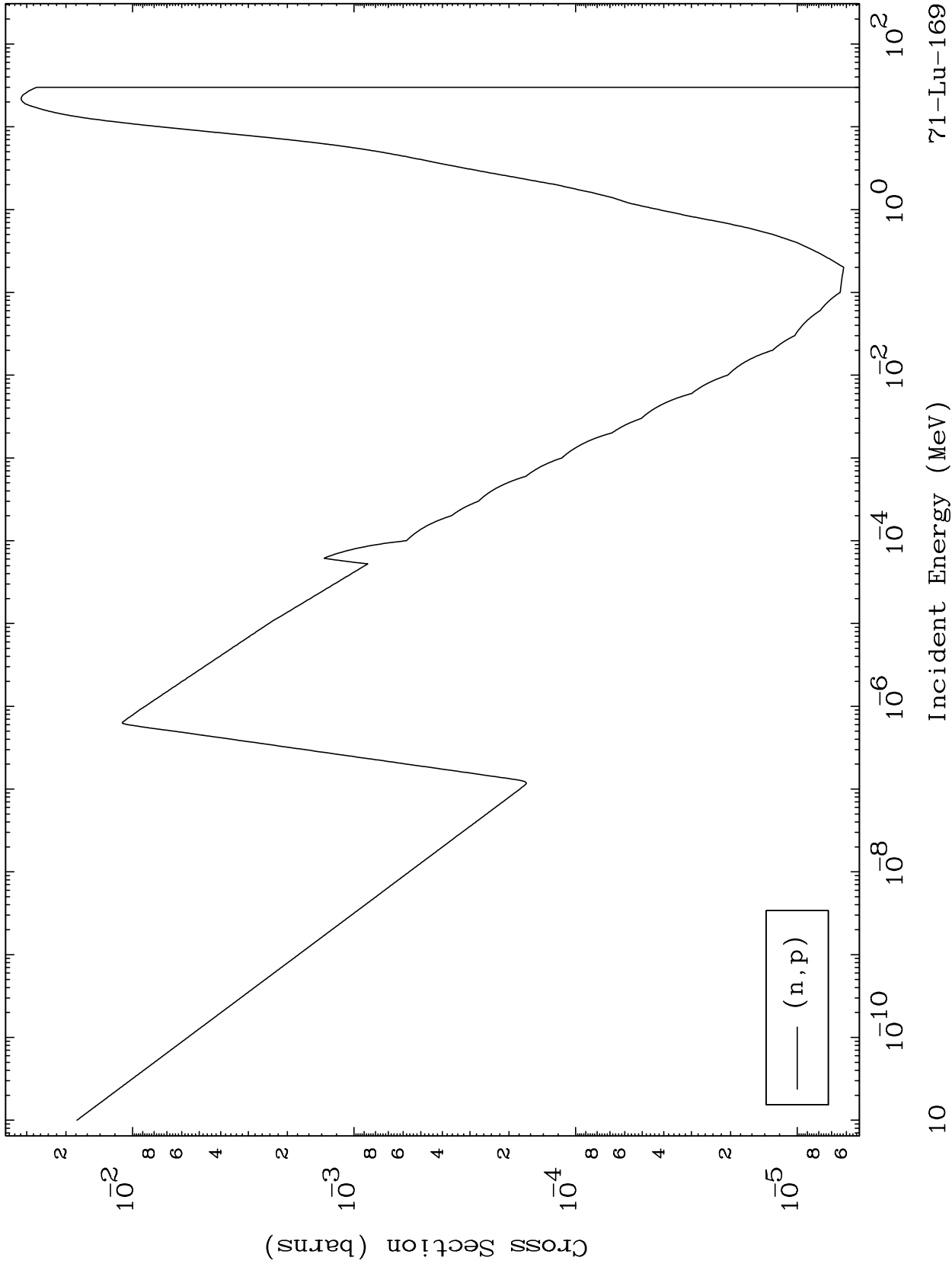
293 Kelvin Cross Sections



MAT 7108

(n,p) Levels
293 Kelvin Cross Sections

71-Lu-169



10

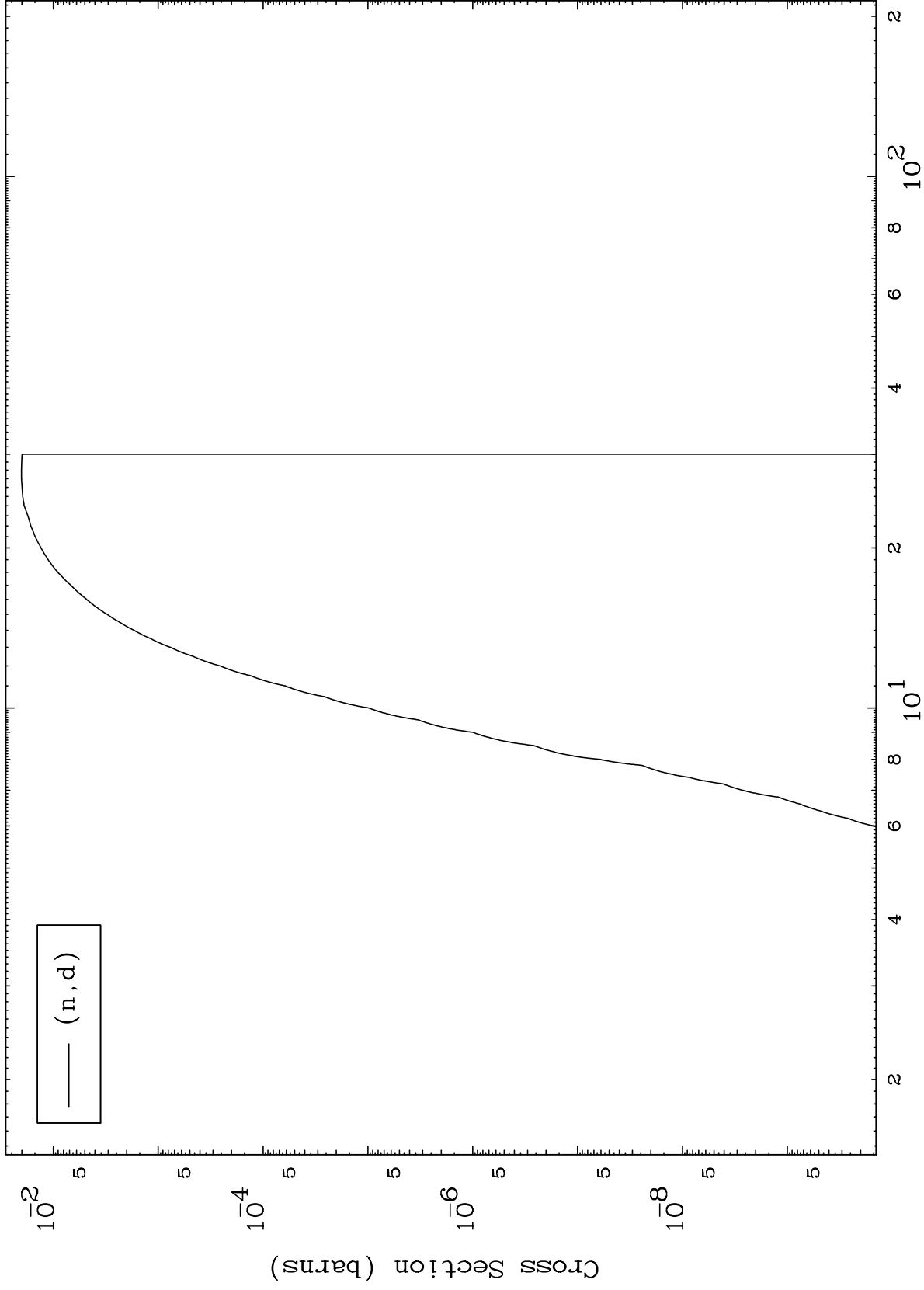
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,d) Levels
293 Kelvin Cross Sections

71-Lu-169



11

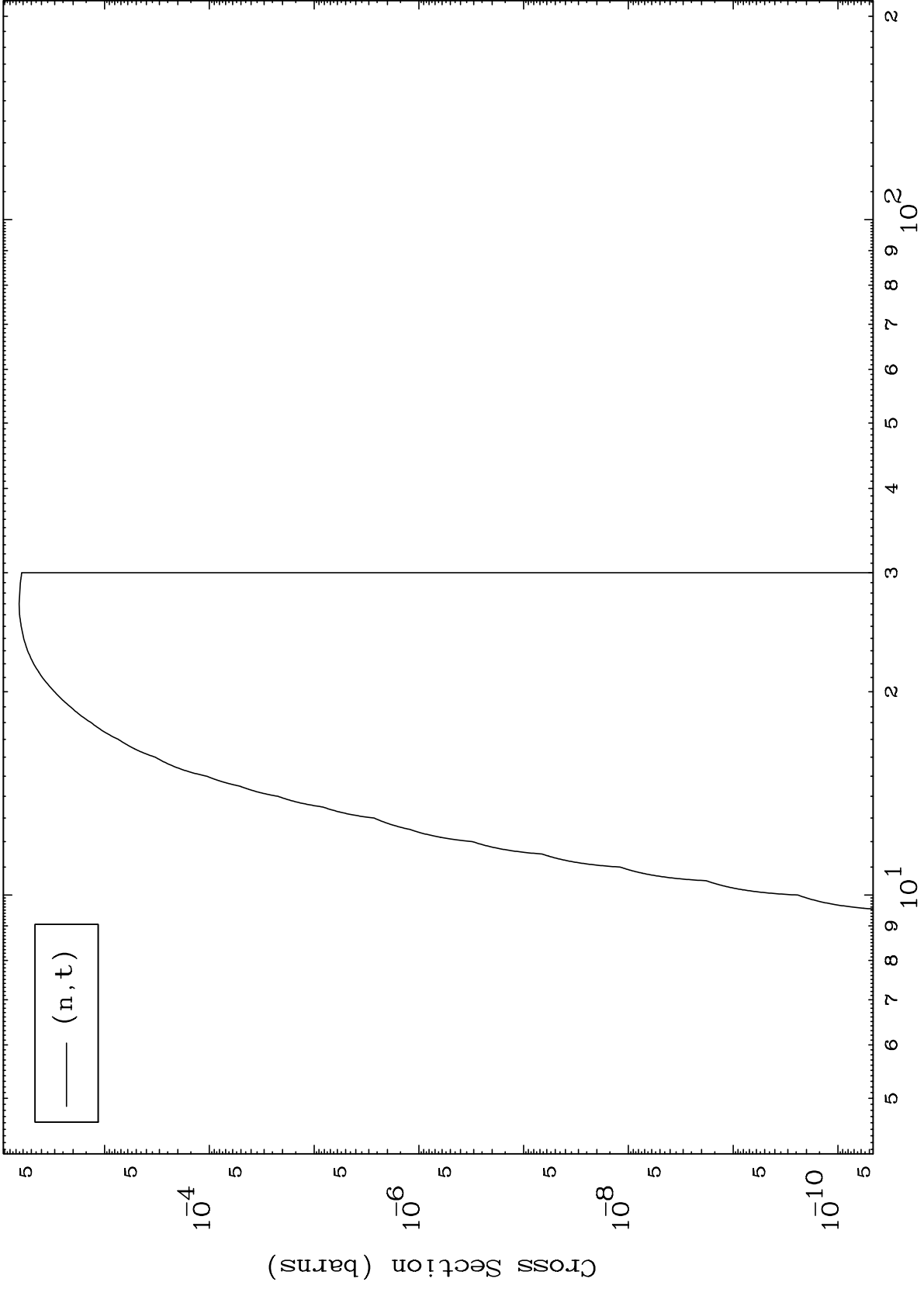
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,t) Levels
293 Kelvin Cross Sections

71-Lu-169



12

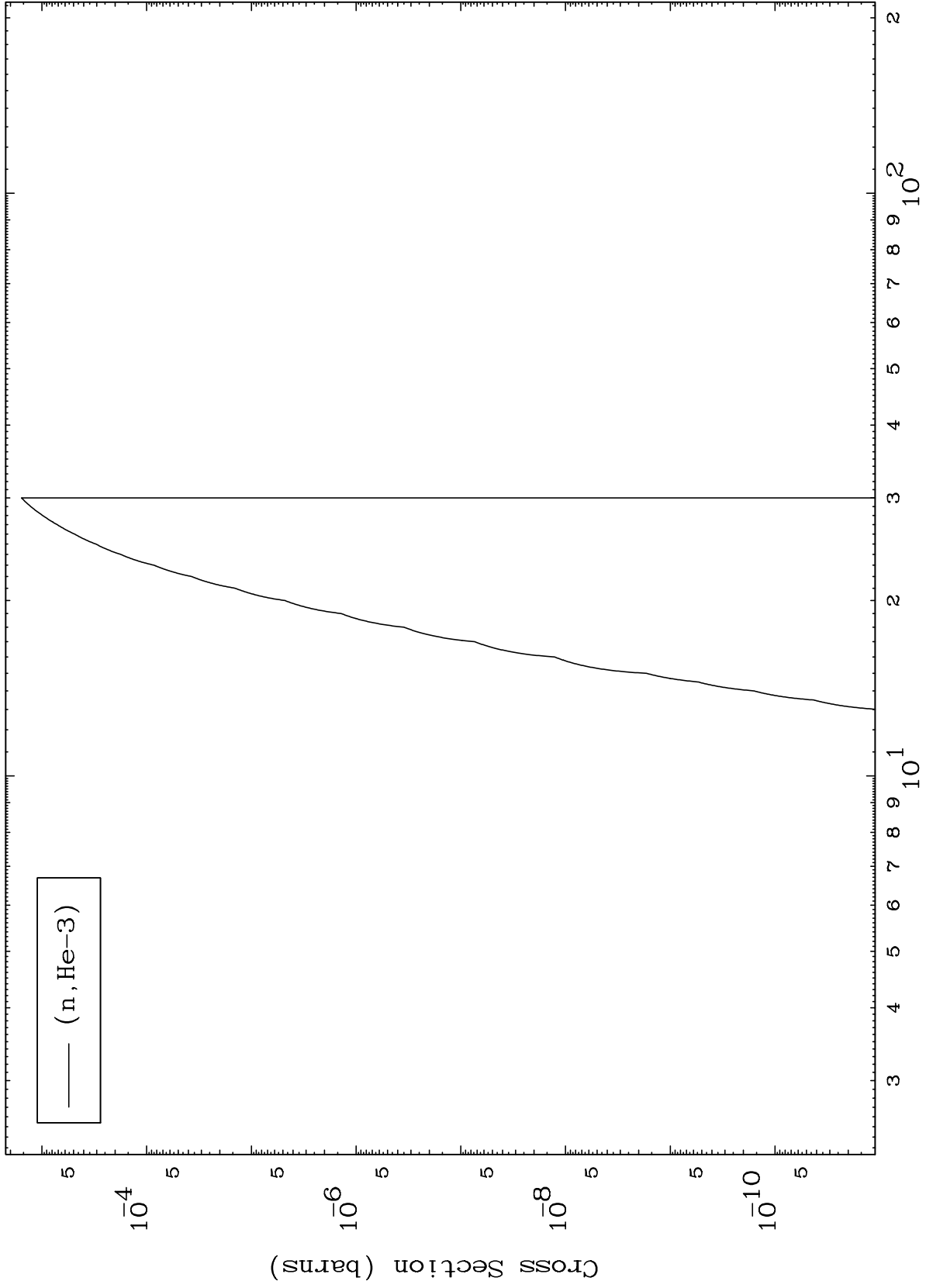
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,He3) Levels
293 Kelvin Cross Sections

71-Lu-169



13

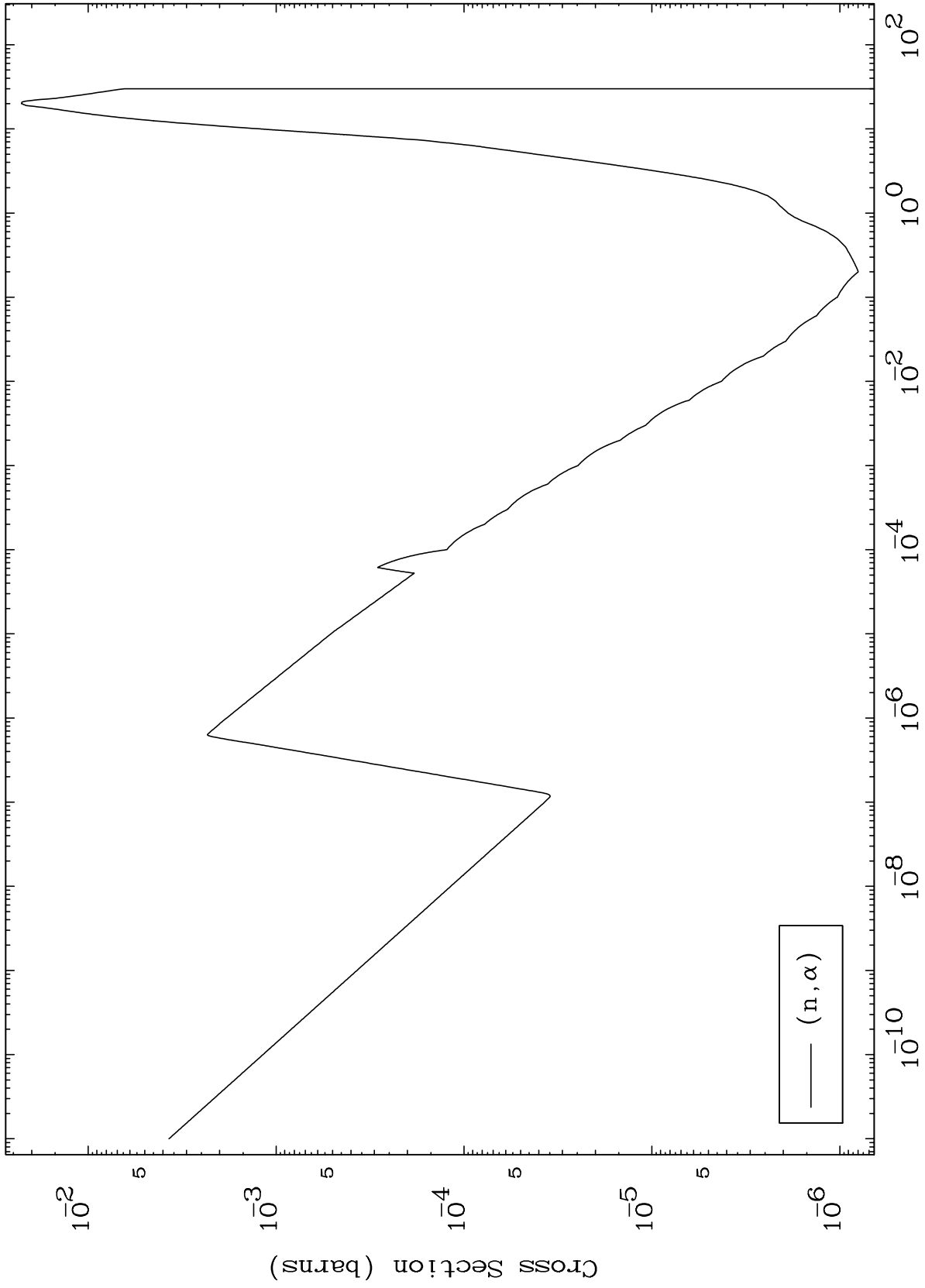
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,α) Levels
293 Kelvin Cross Sections

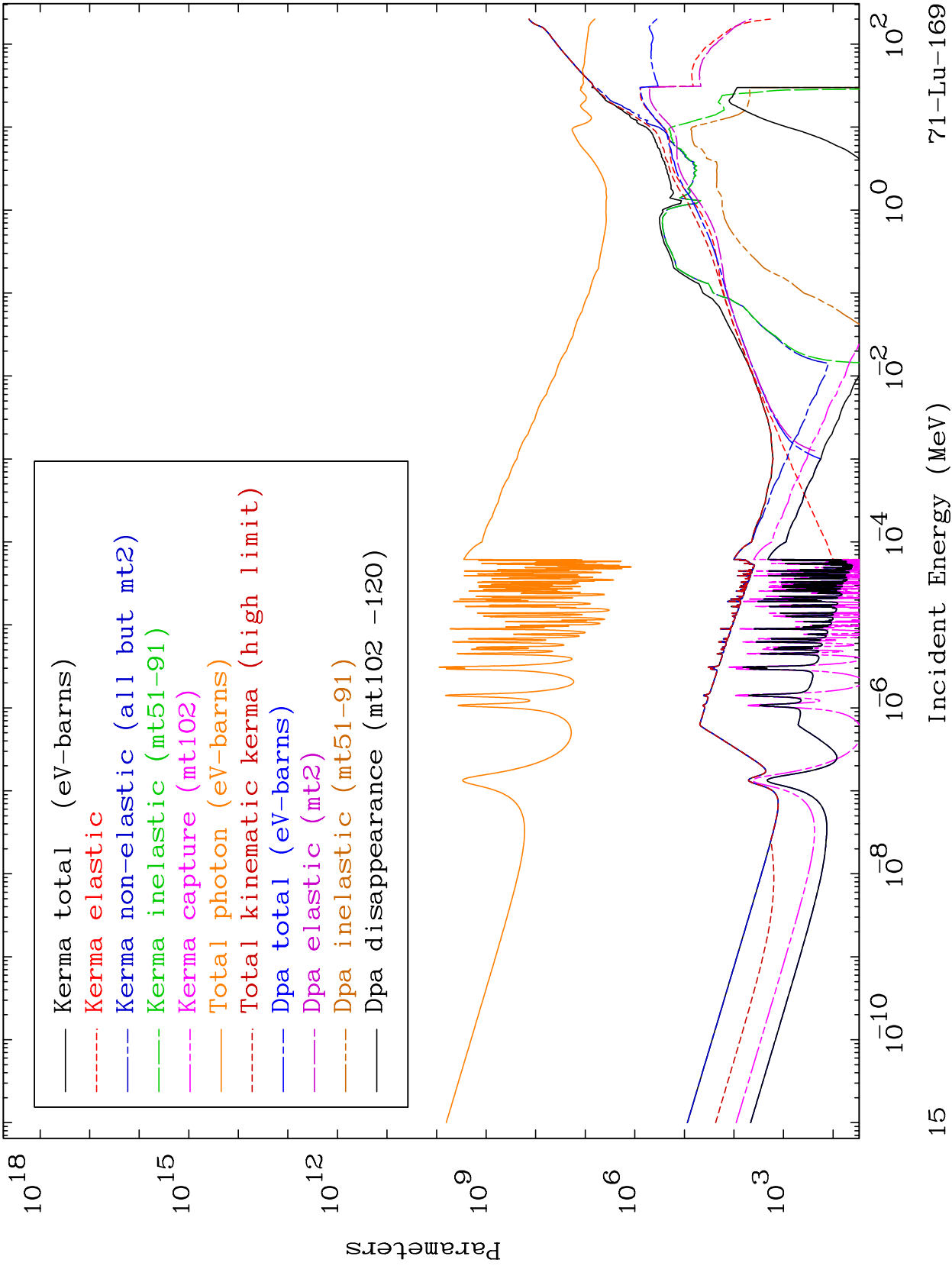
71-Lu-169

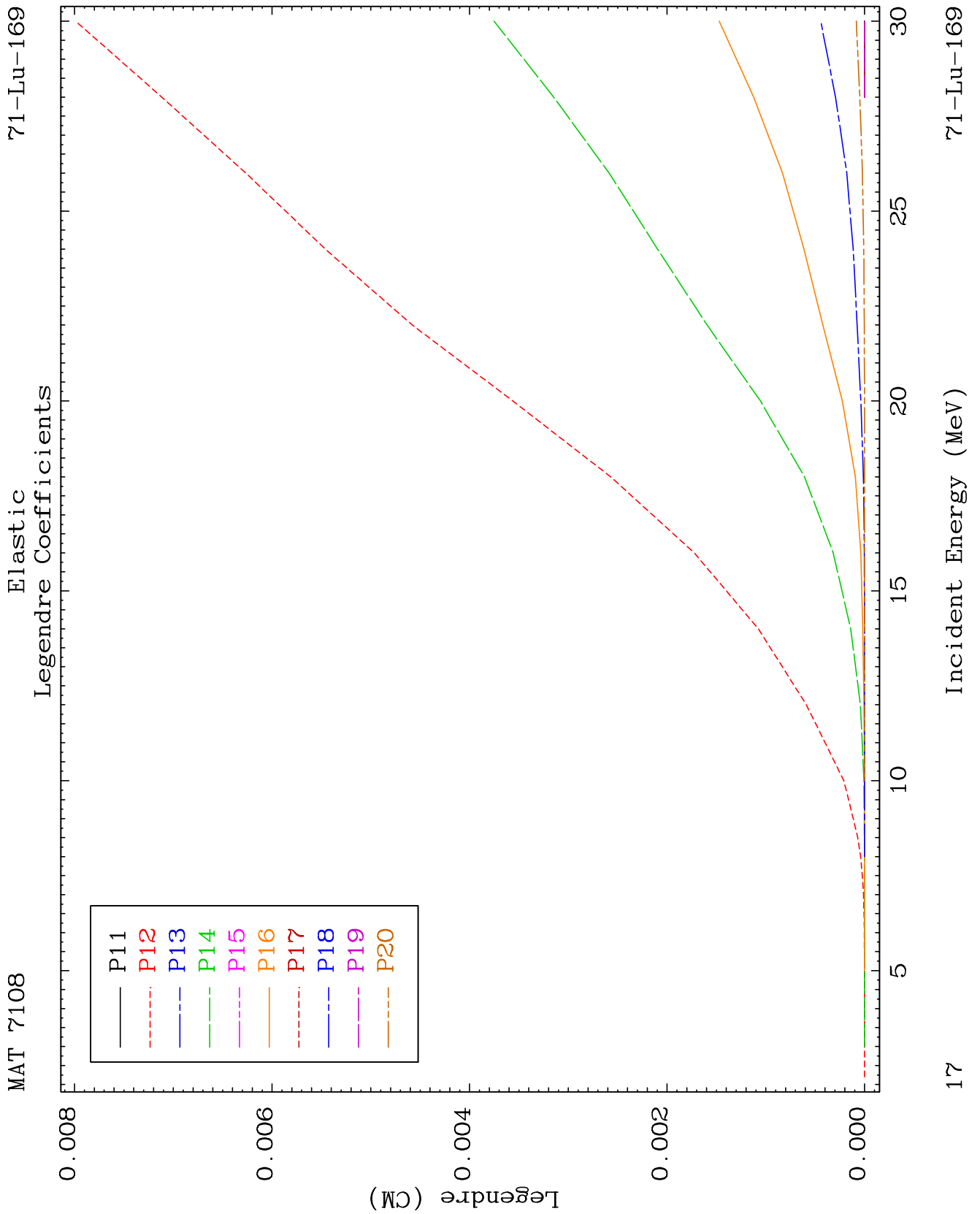


14

Incident Energy (MeV)

71-Lu-169





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Elastic Legendre Coefficients

71-Lu-169

17

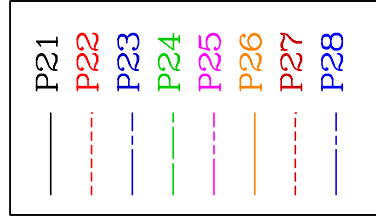
Incident Energy (MeV)

71-Lu-169

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Elastic
Legendre Coefficients

71-Lu-169



$\times 10^{-6}$

Legendre (CM)



18

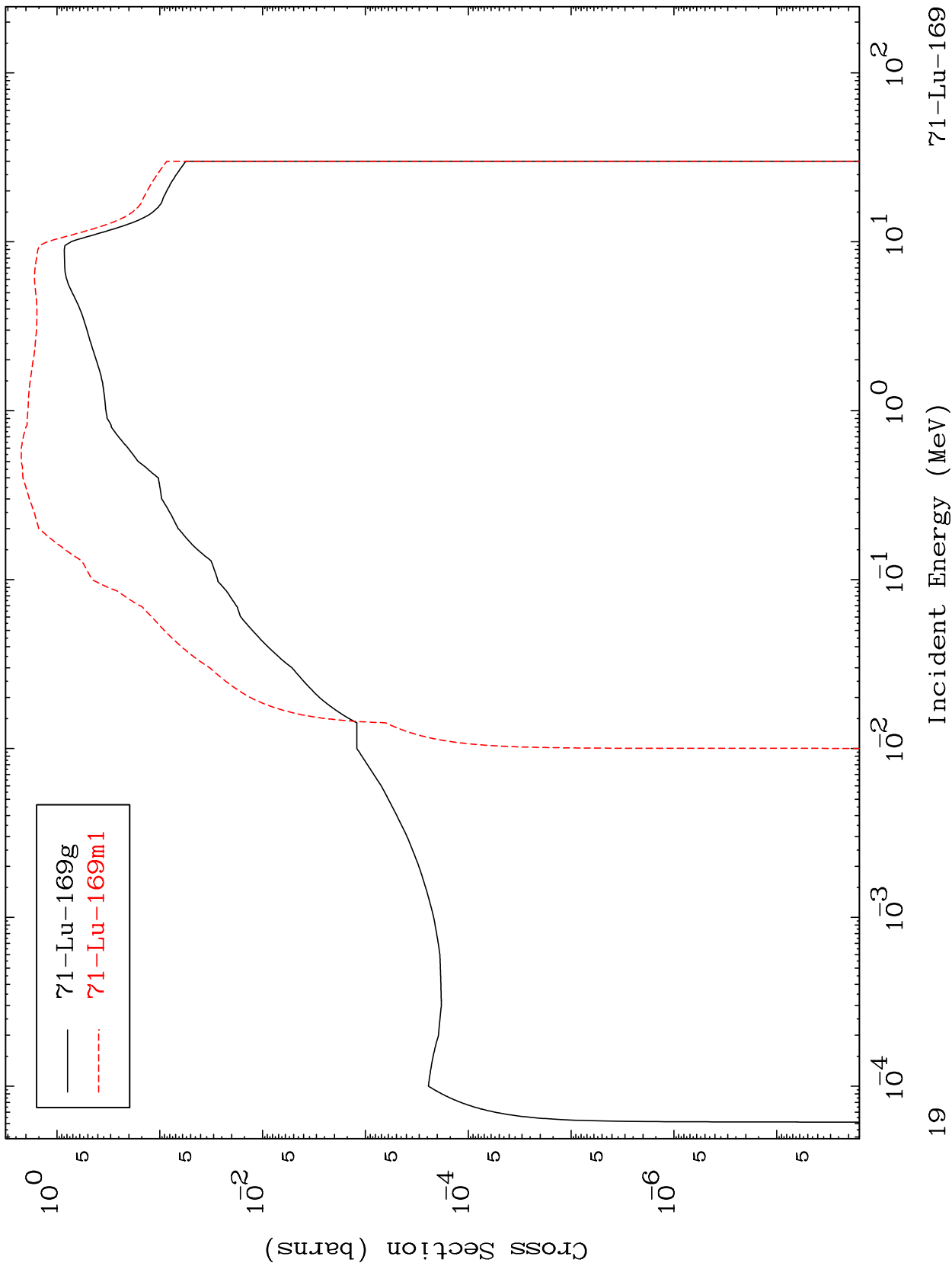
Incident Energy (MeV)

71-Lu-169

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71-Lu-169

Radionuclide Production Cross Section



19

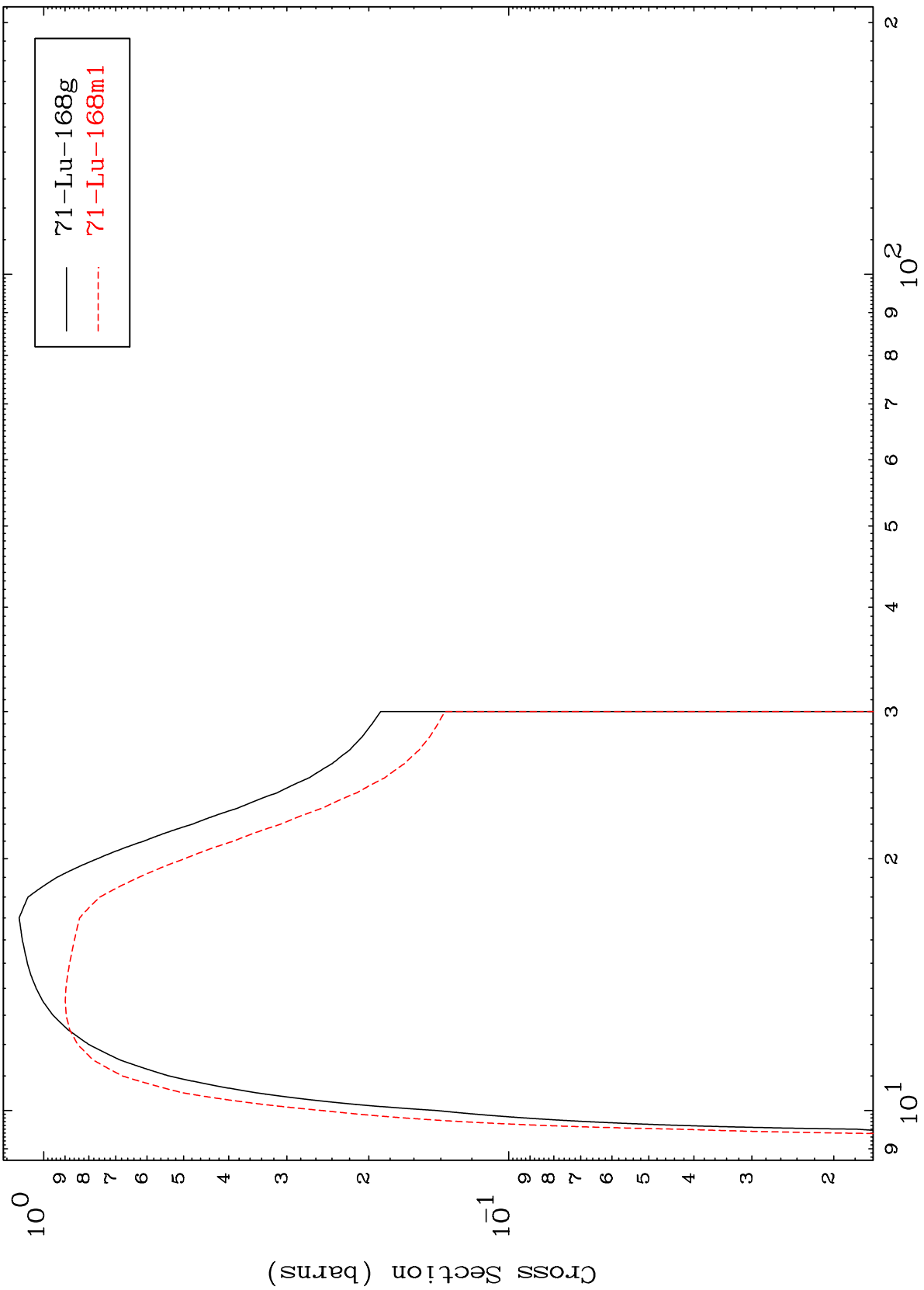
71-Lu-169

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71-Lu-169

(n,2n)

Radionuclide Production Cross Section



71-Lu-168g
71-Lu-168m1

71-Lu-169

Incident Energy (MeV)

20

10¹

2

4

5

6

7

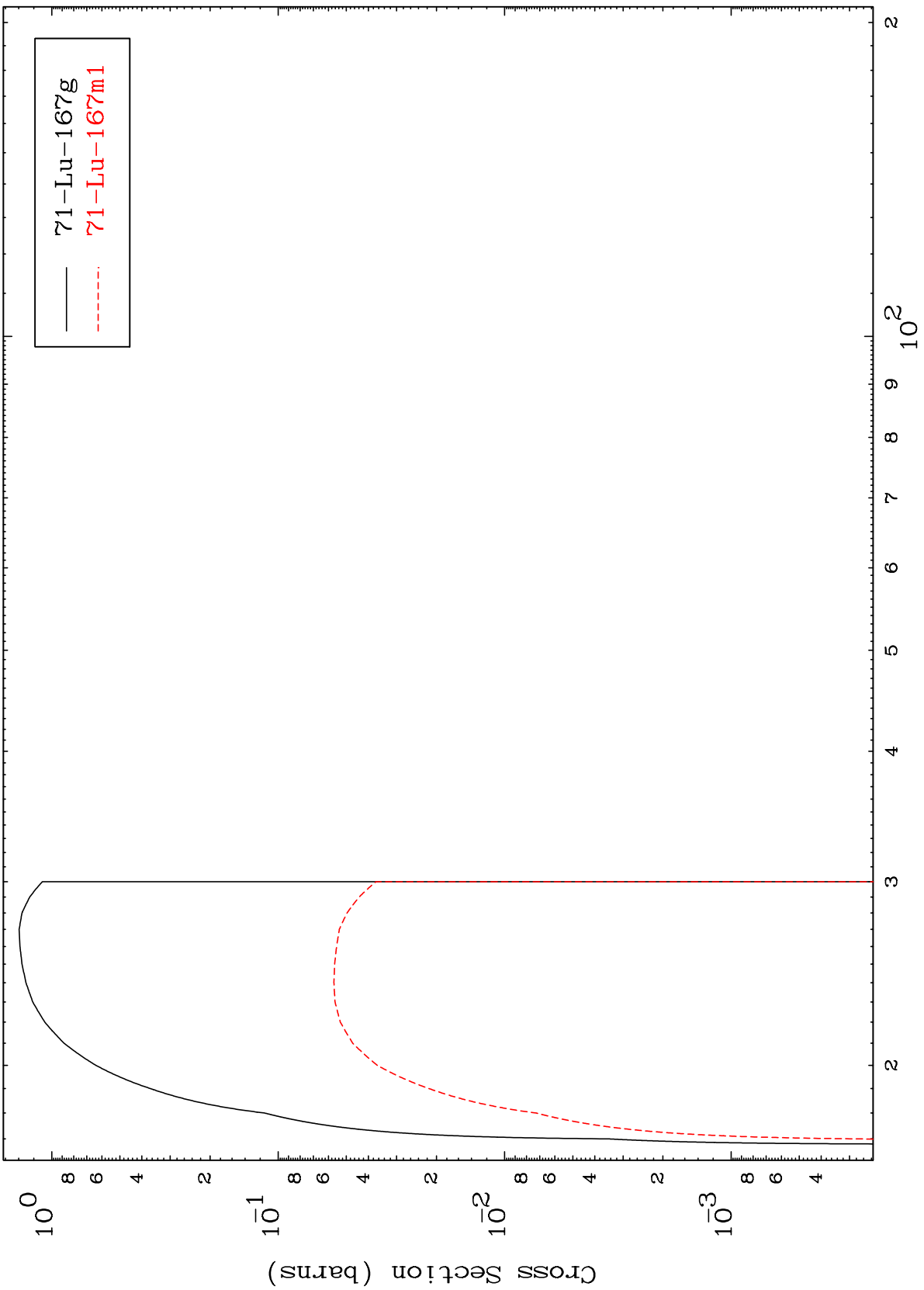
8

9

10²

2

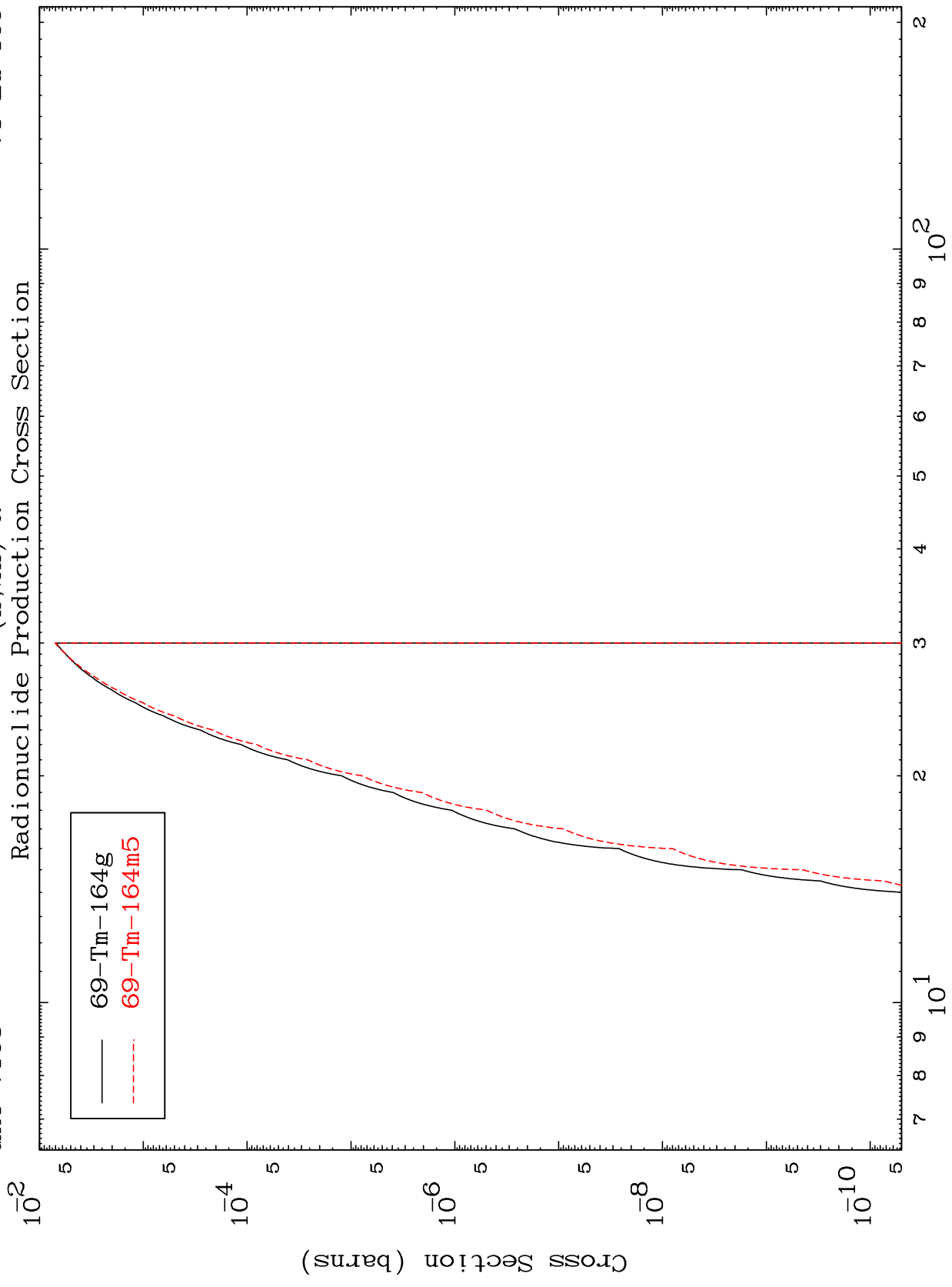
(n,3n)
Radionuclide Production Cross Section



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$(n,2n) \alpha$

71-Lu-169



22

Incident Energy (MeV)

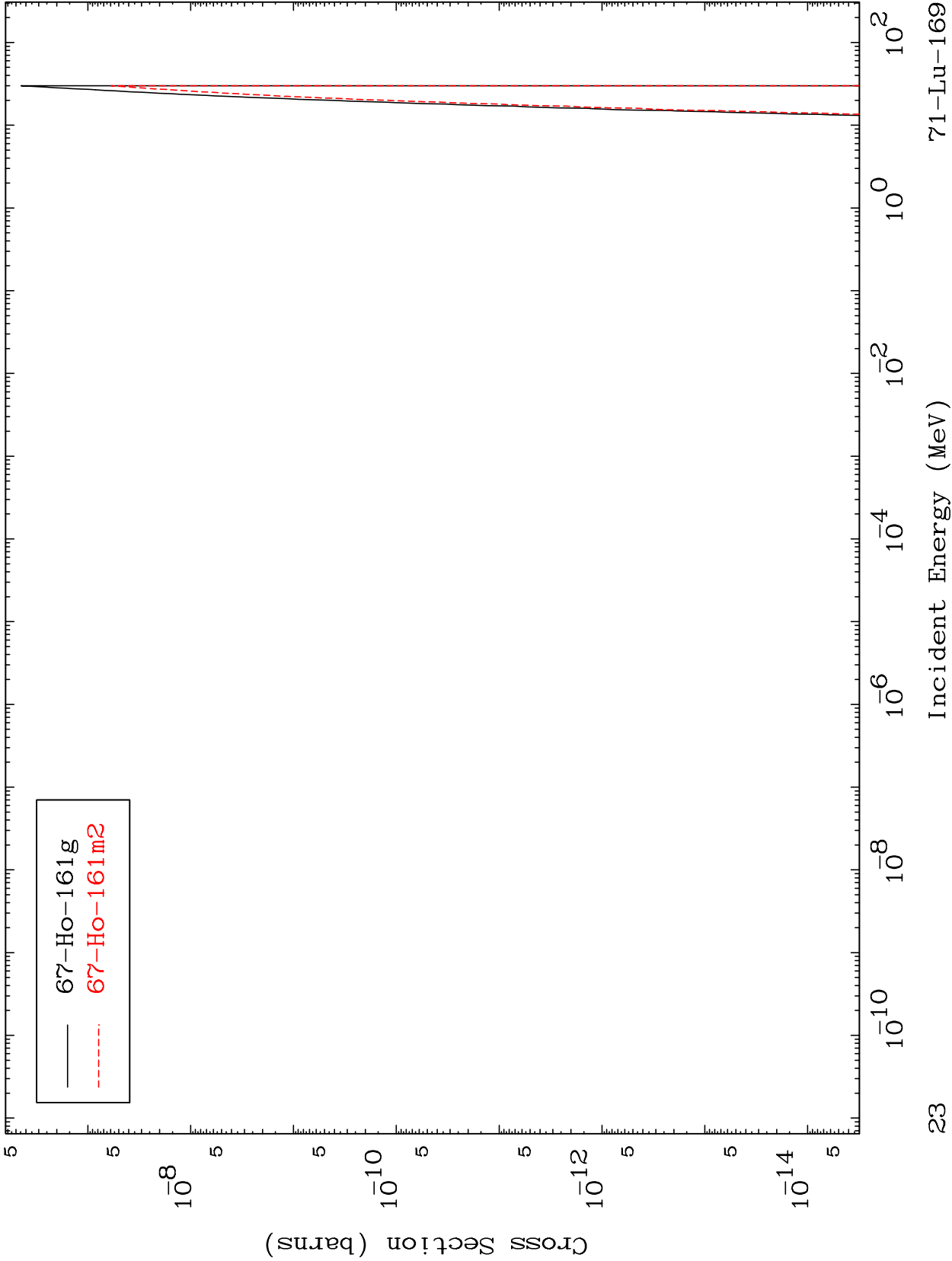
71-Lu-169

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(n, n') 2α

$^{71}\text{Lu-169}$

Radionuclide Production Cross Section

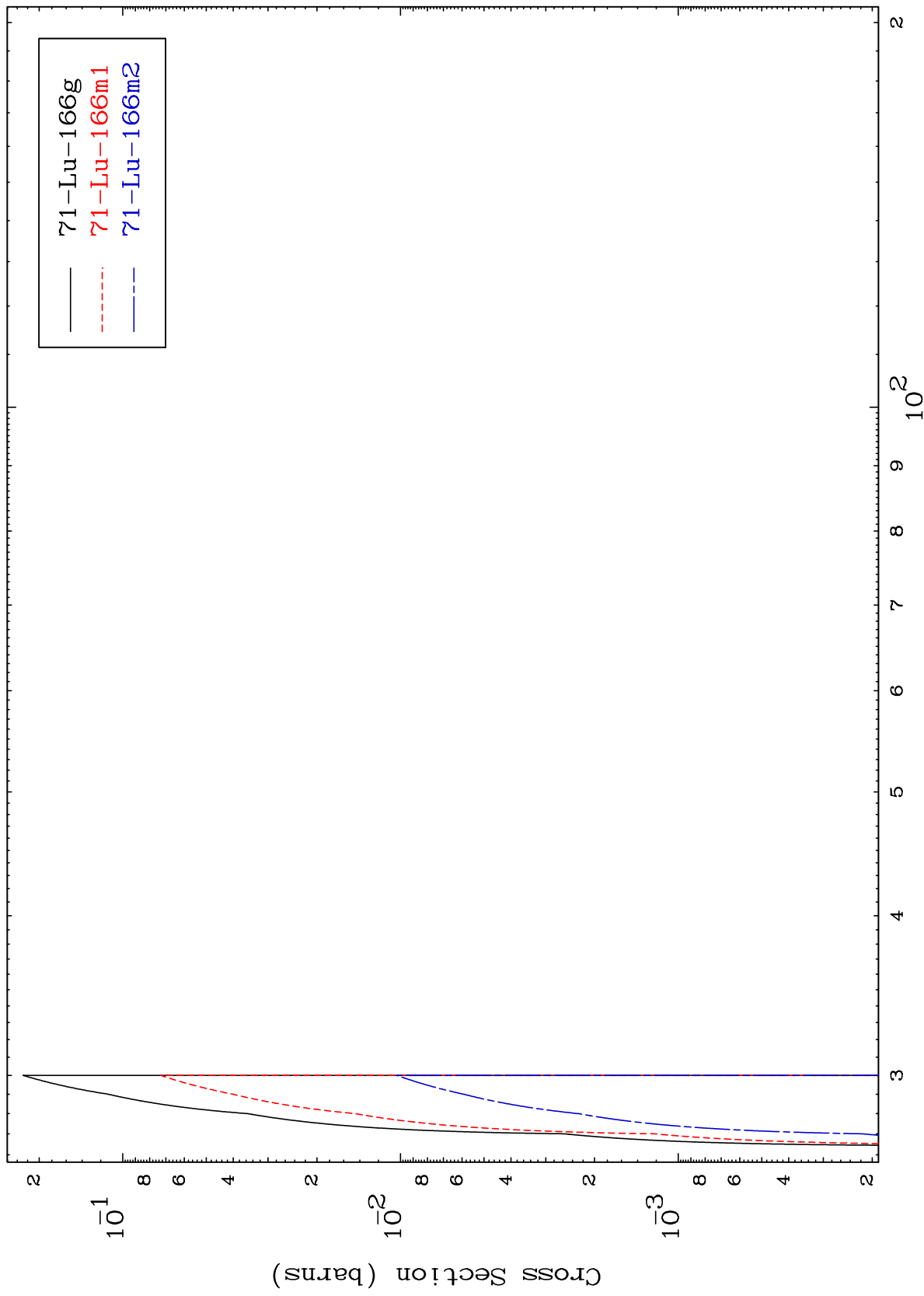


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⁷¹Lu-169

(n,4n)

Radionuclide Production Cross Section



24

⁷¹Lu-169

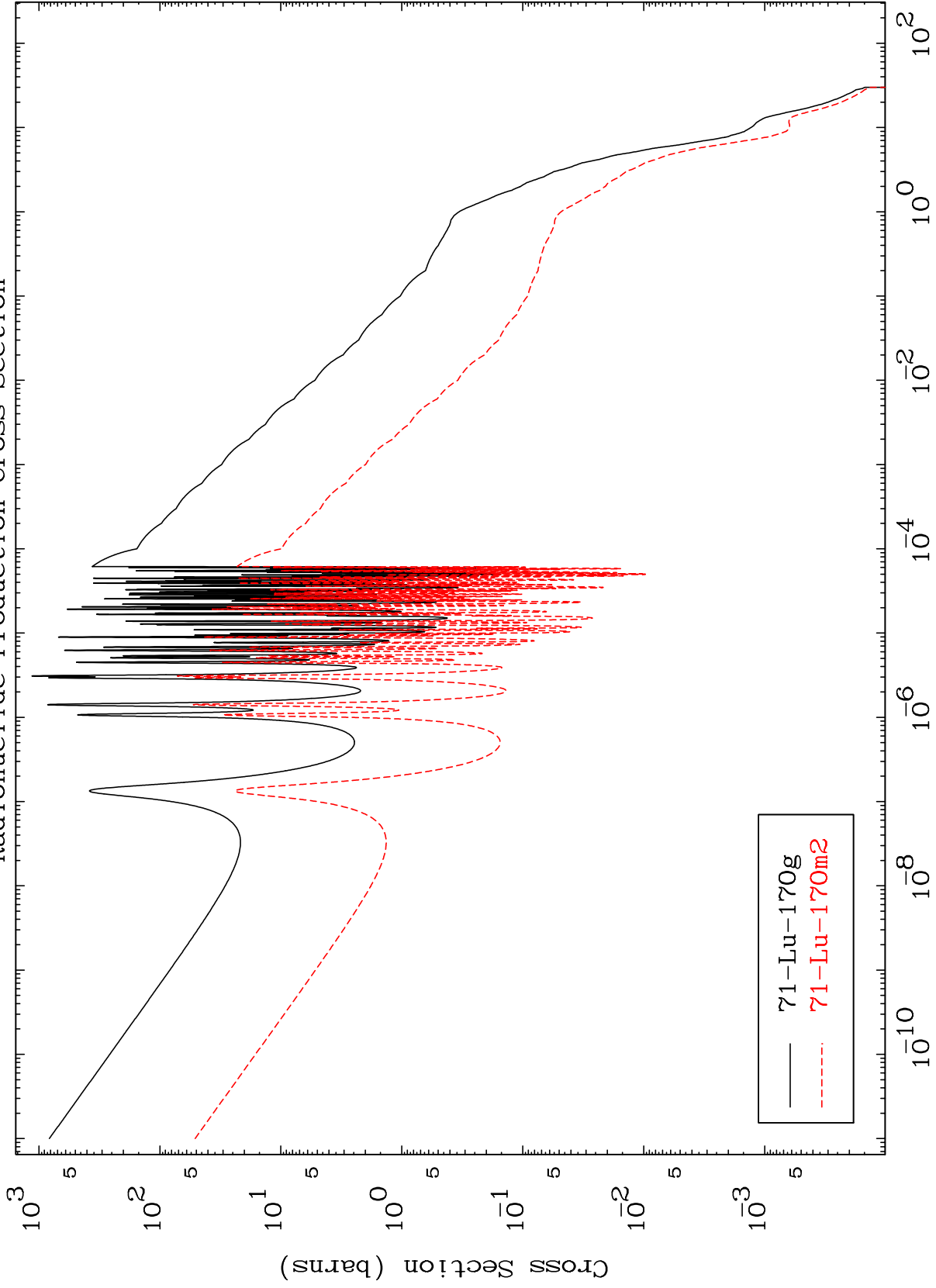
Incident Energy (MeV)

MAT 7108

71-Lu-169

Radionuclide Production Cross Section

(n, γ)

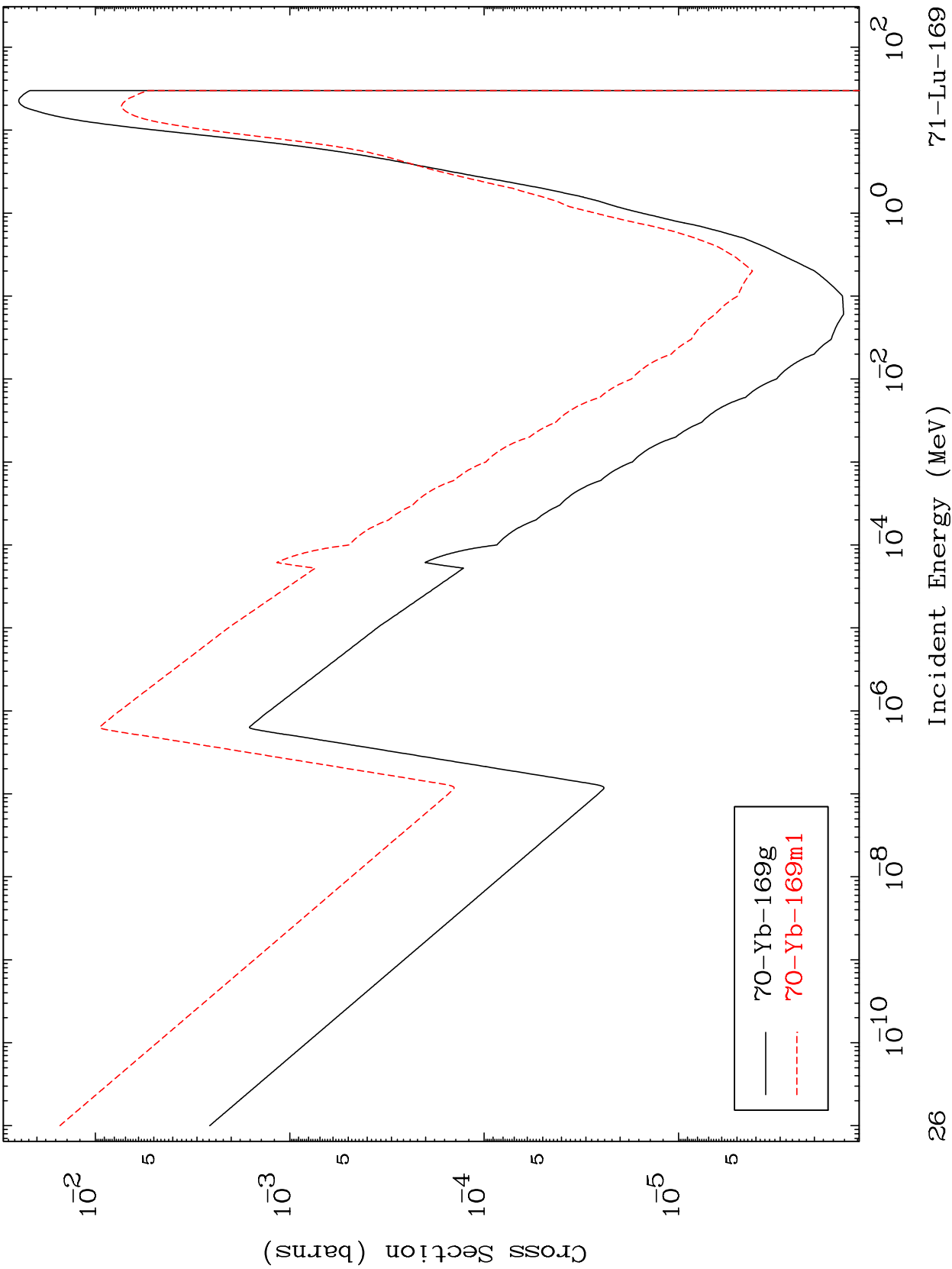


71-Lu-169

Incident Energy (MeV)

25

(n,p)
Radionuclide Production Cross Section

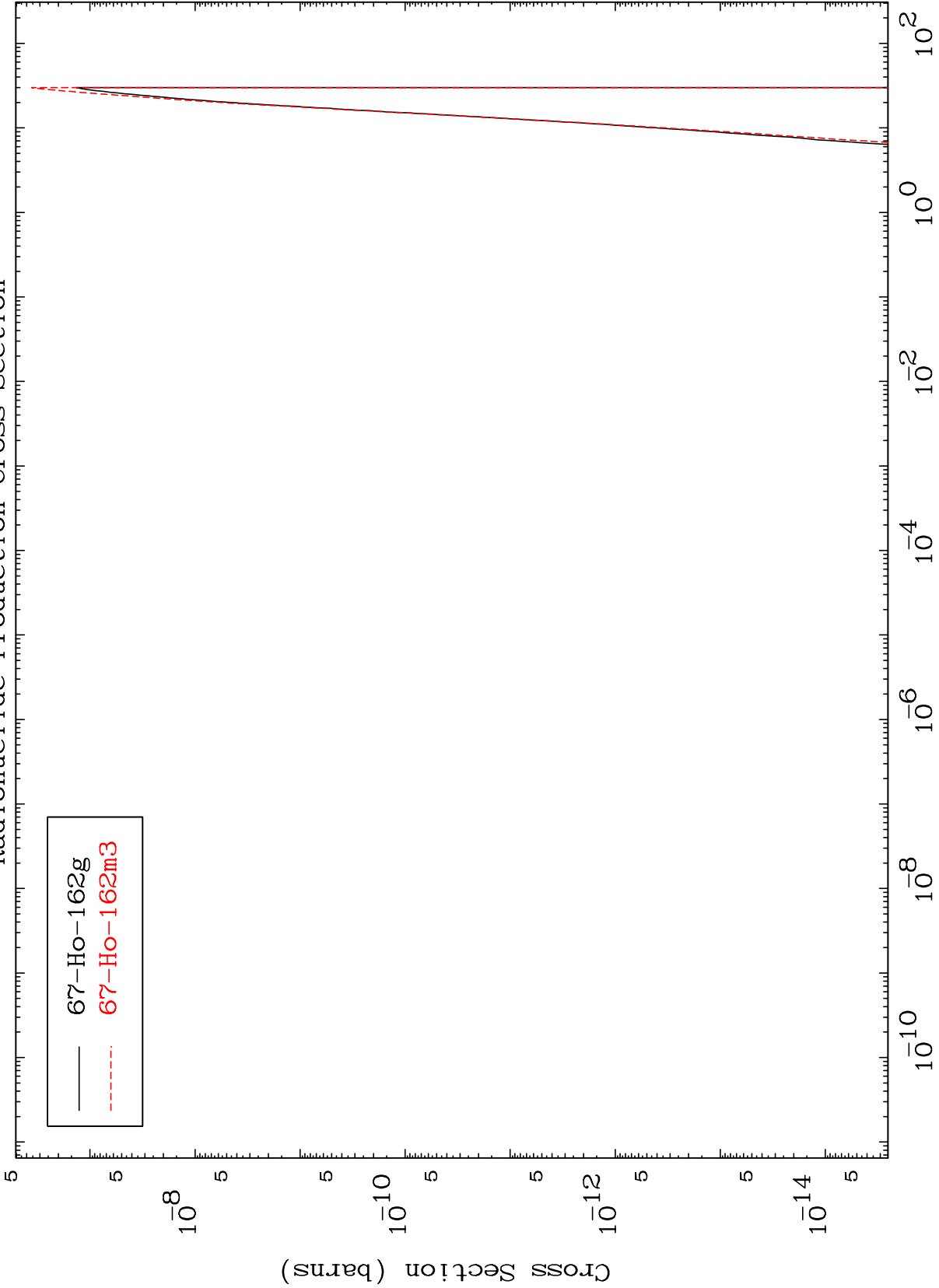


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(n,2α)

⁷¹Lu-169

Radionuclide Production Cross Section



27

Incident Energy (MeV)

⁷¹Lu-169