

Program EVALPLOT
(Version 2017-1)

by

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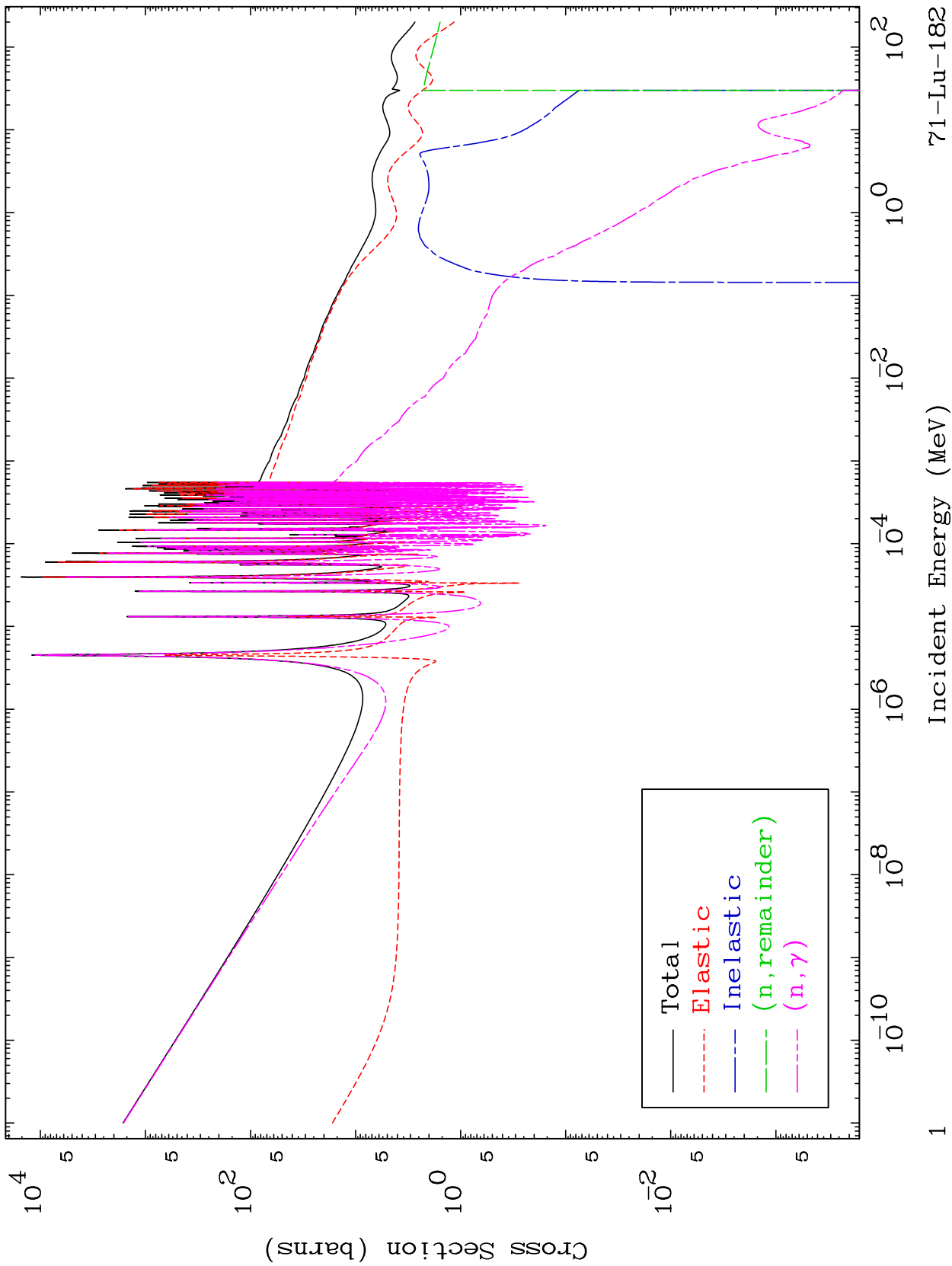
Press Mouse Button to Start

MAT 7146

Major

293 Kelvin Cross Sections

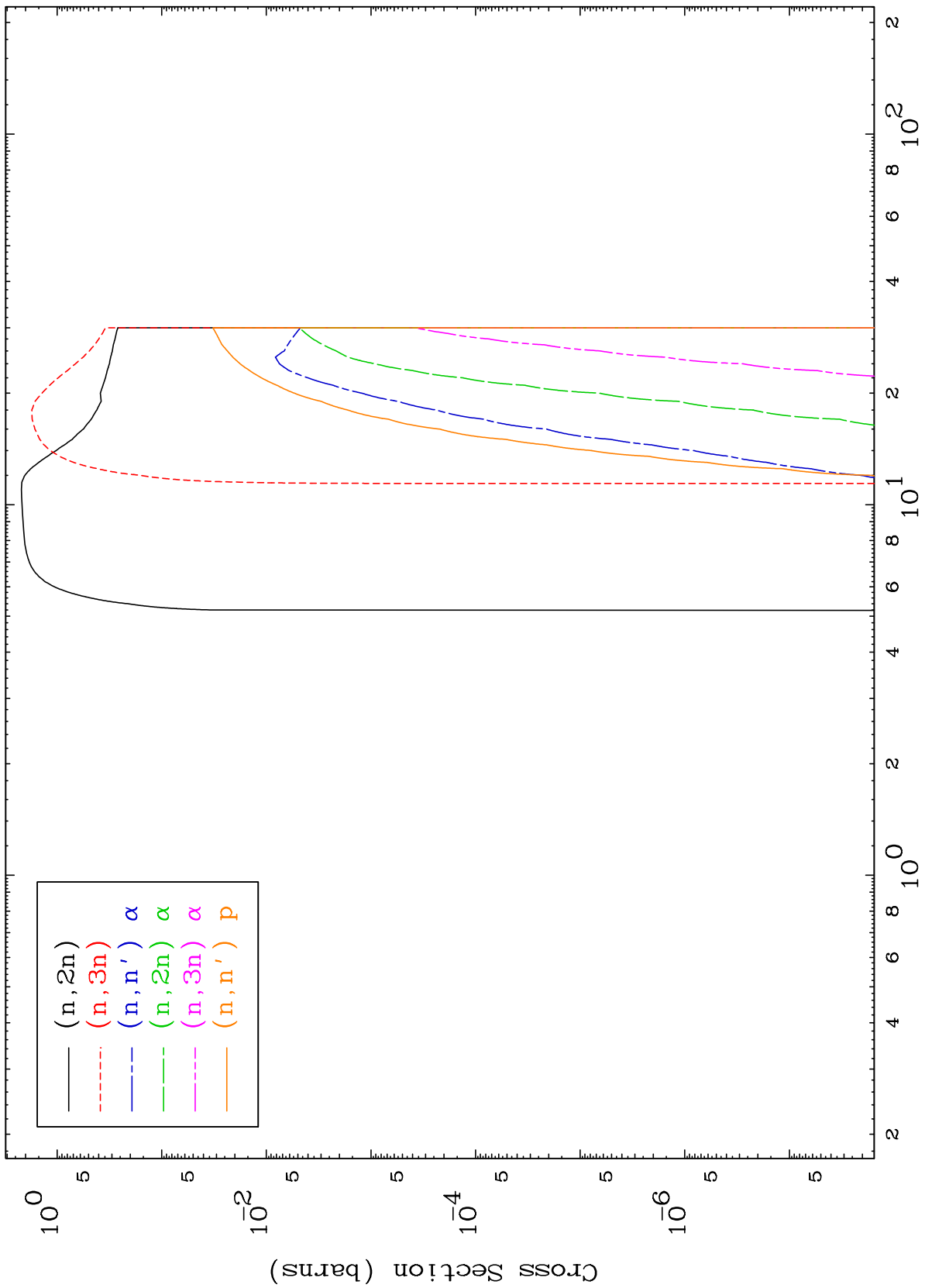
71-Lu-182



MAT 7146

Neutron Production
293 Kelvin Cross Sections

71-Lu-182



Incident Energy (MeV)

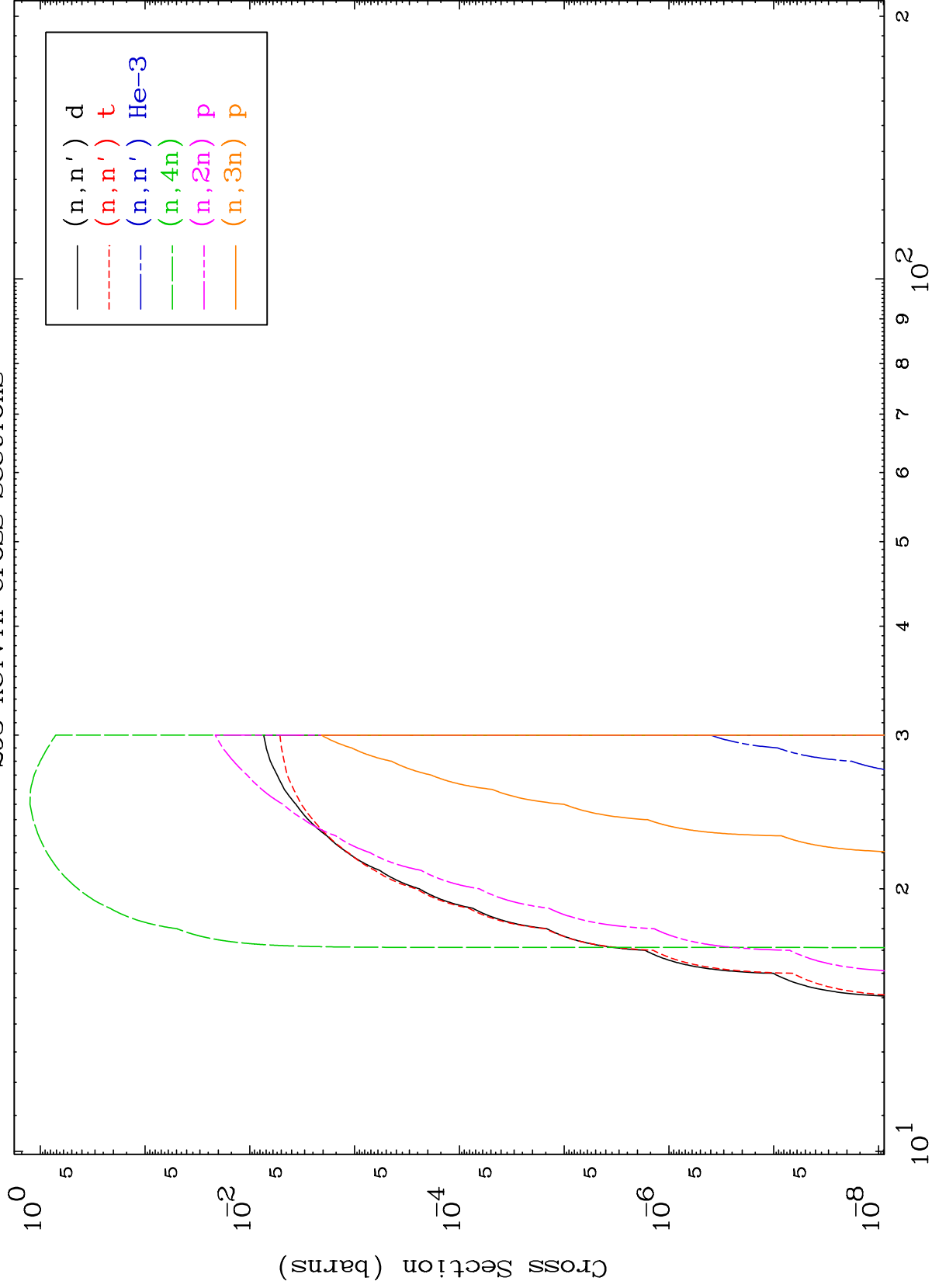
71-Lu-182

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Neutron Production
293 Kelvin Cross Sections

71-Lu-182



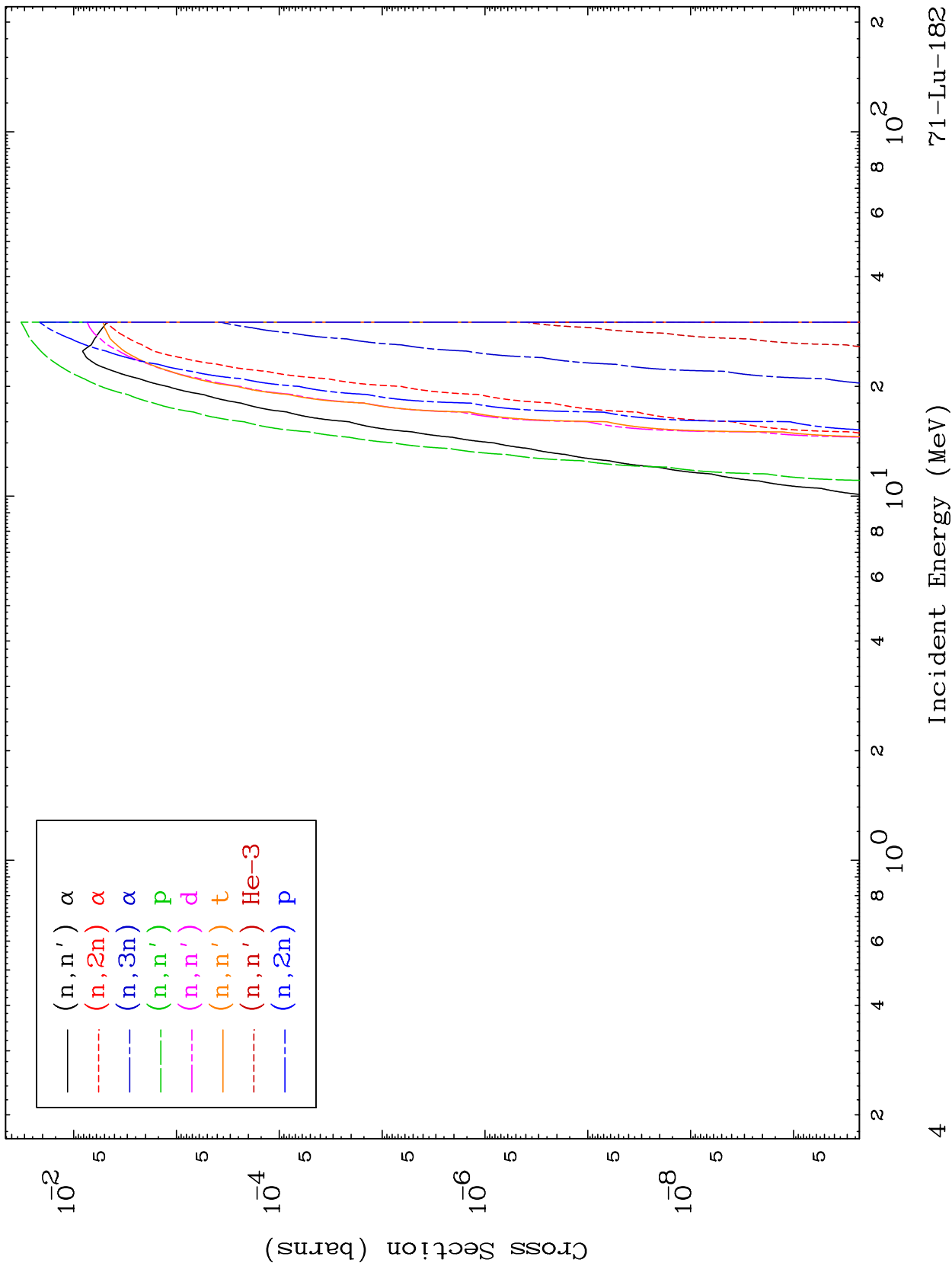
Incident Energy (MeV)

71-Lu-182

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Charged Particle
293 Kelvin Cross Sections

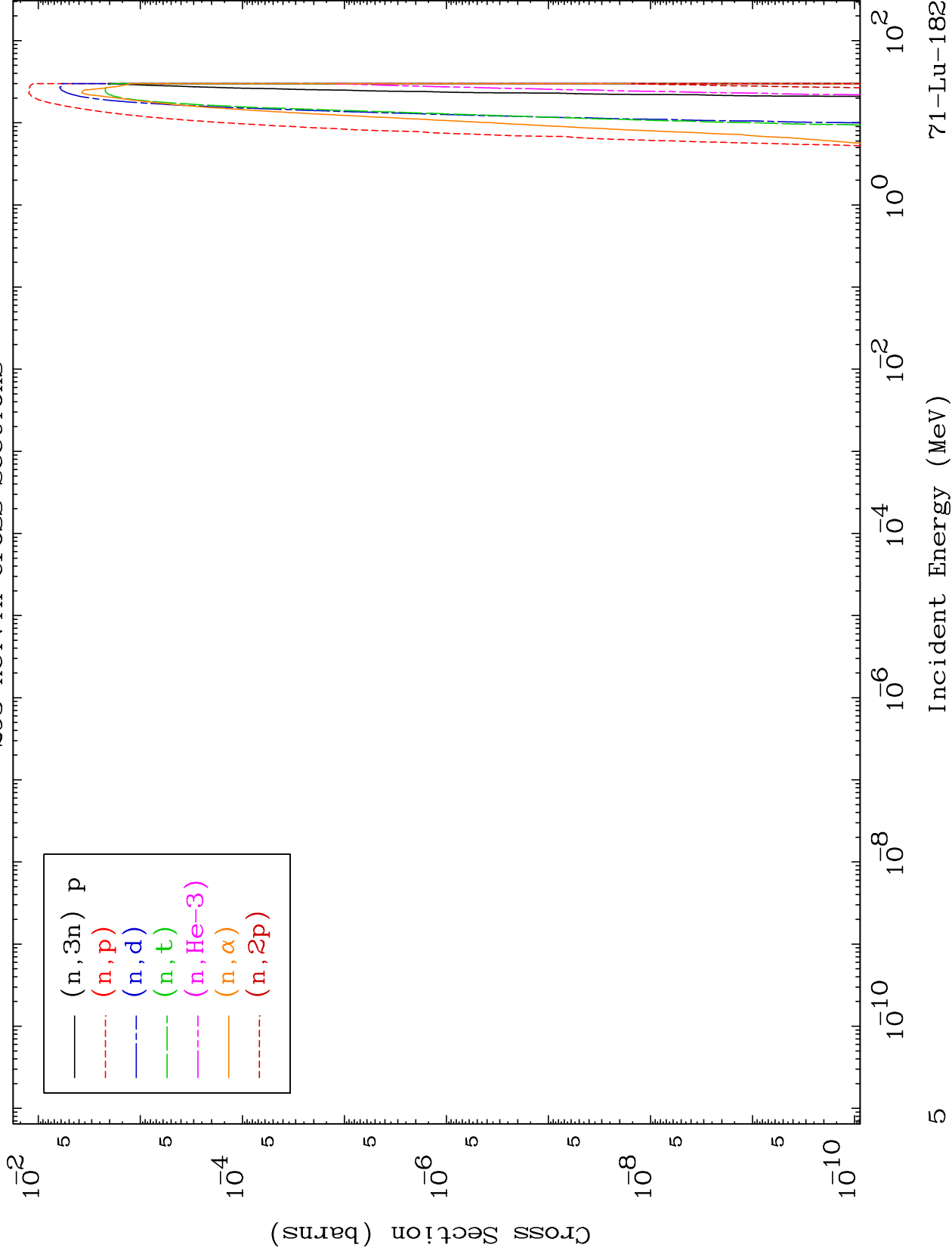
71-Lu-182



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Charged Particle
293 Kelvin Cross Sections

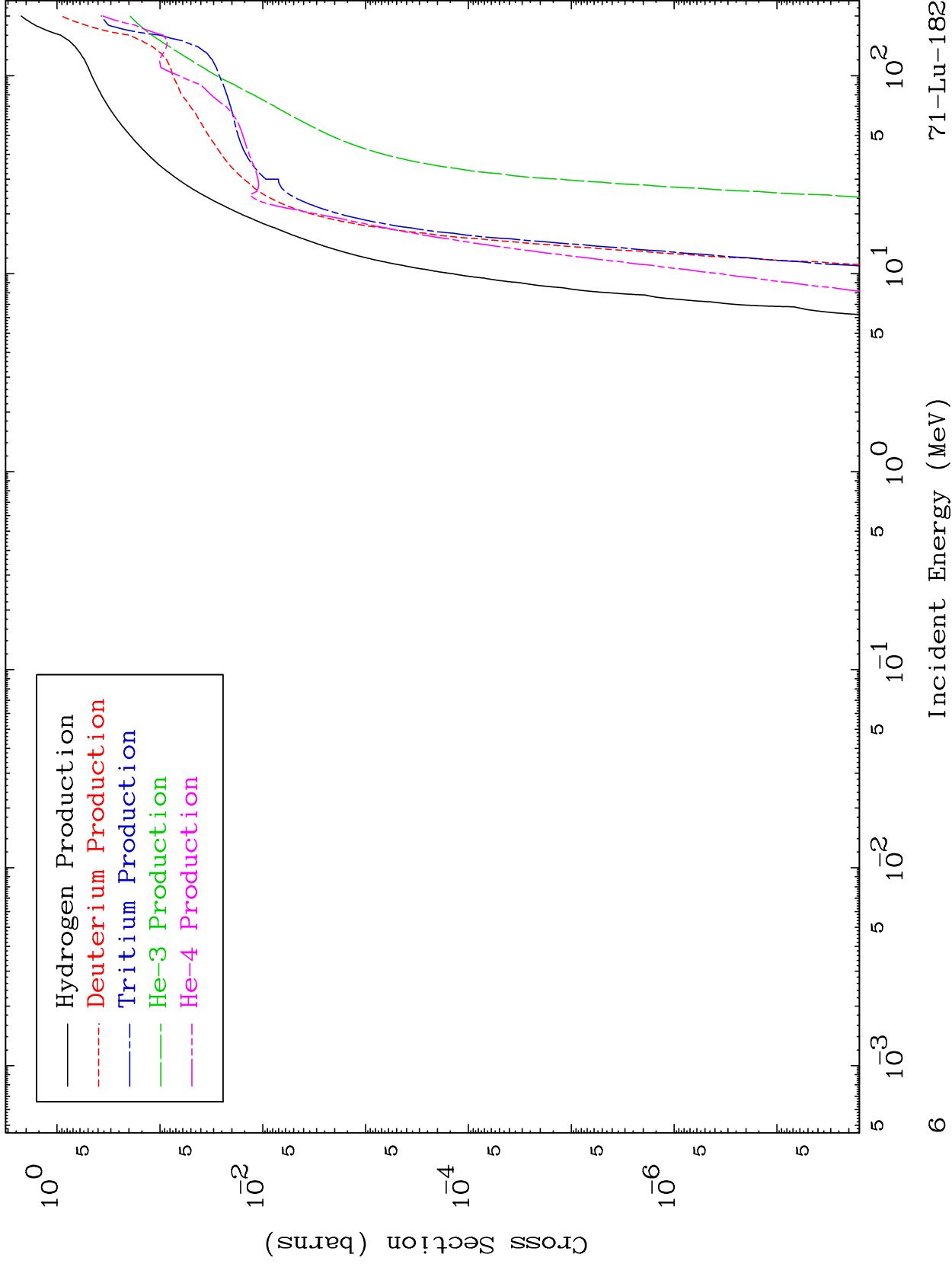
71-Lu-182



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Particle Production
293 Kelvin Cross Sections

71-Lu-182



71-Lu-182

Incident Energy (MeV)

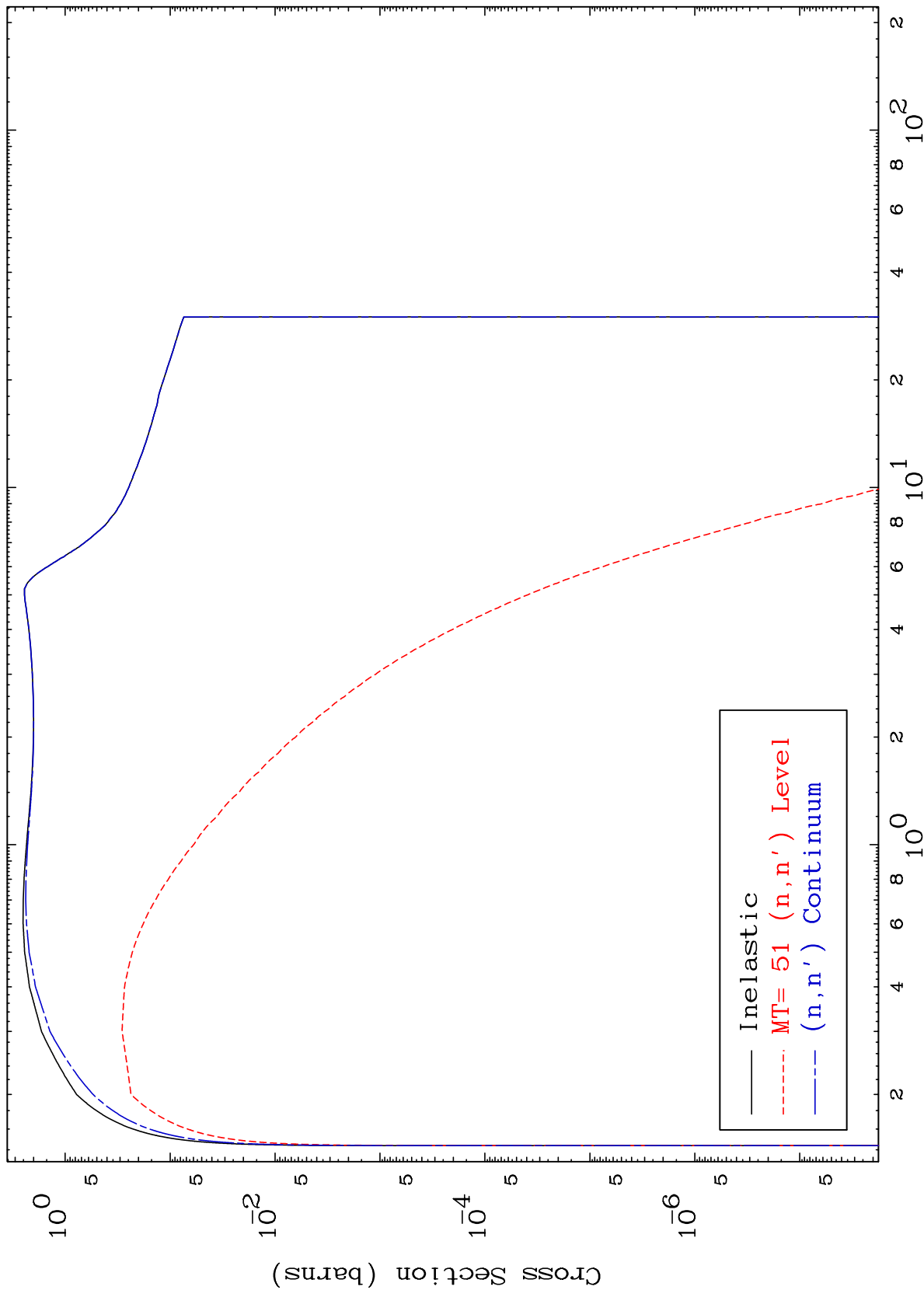
6

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(n,n') Level

71-Lu-182

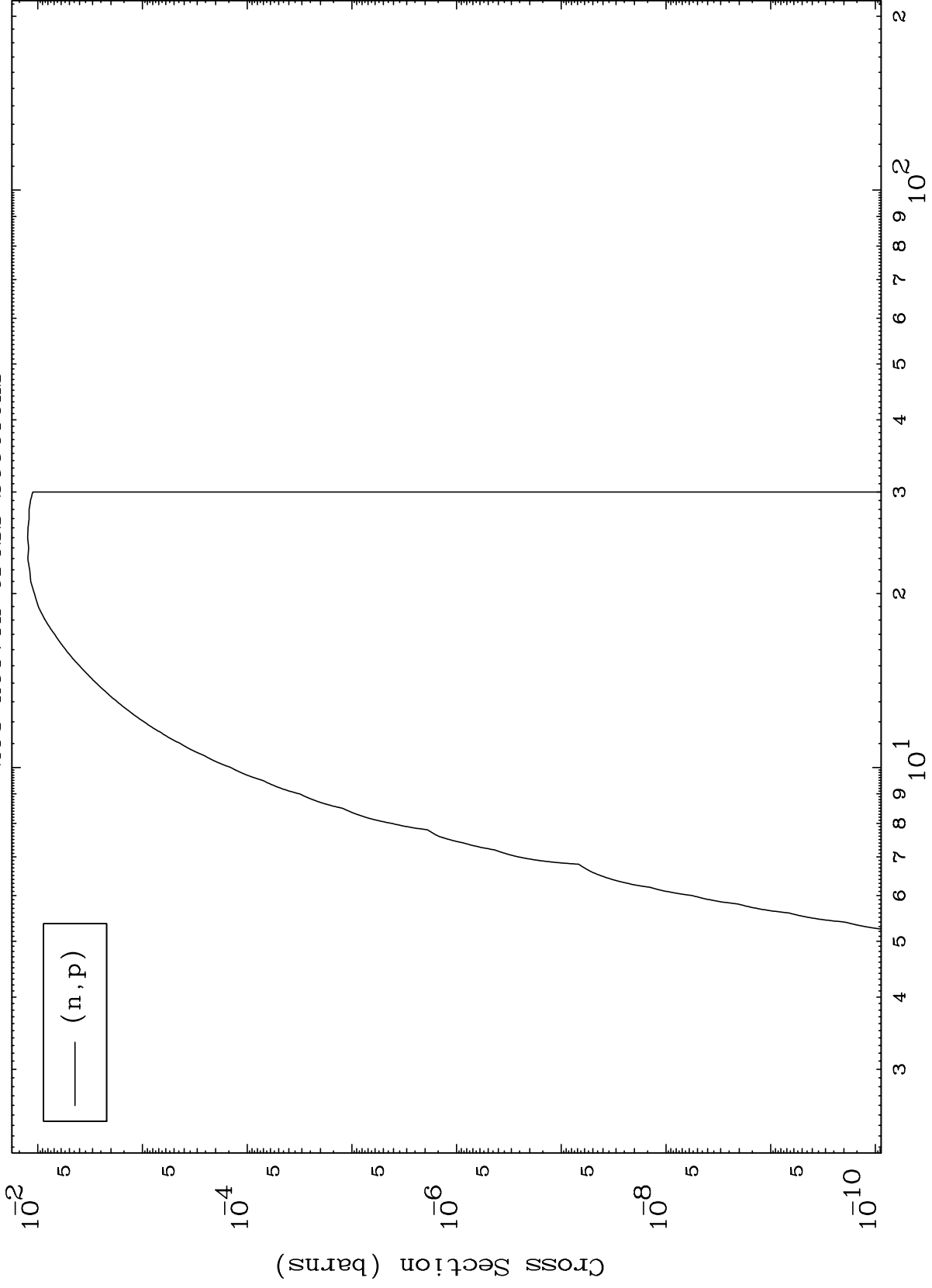
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

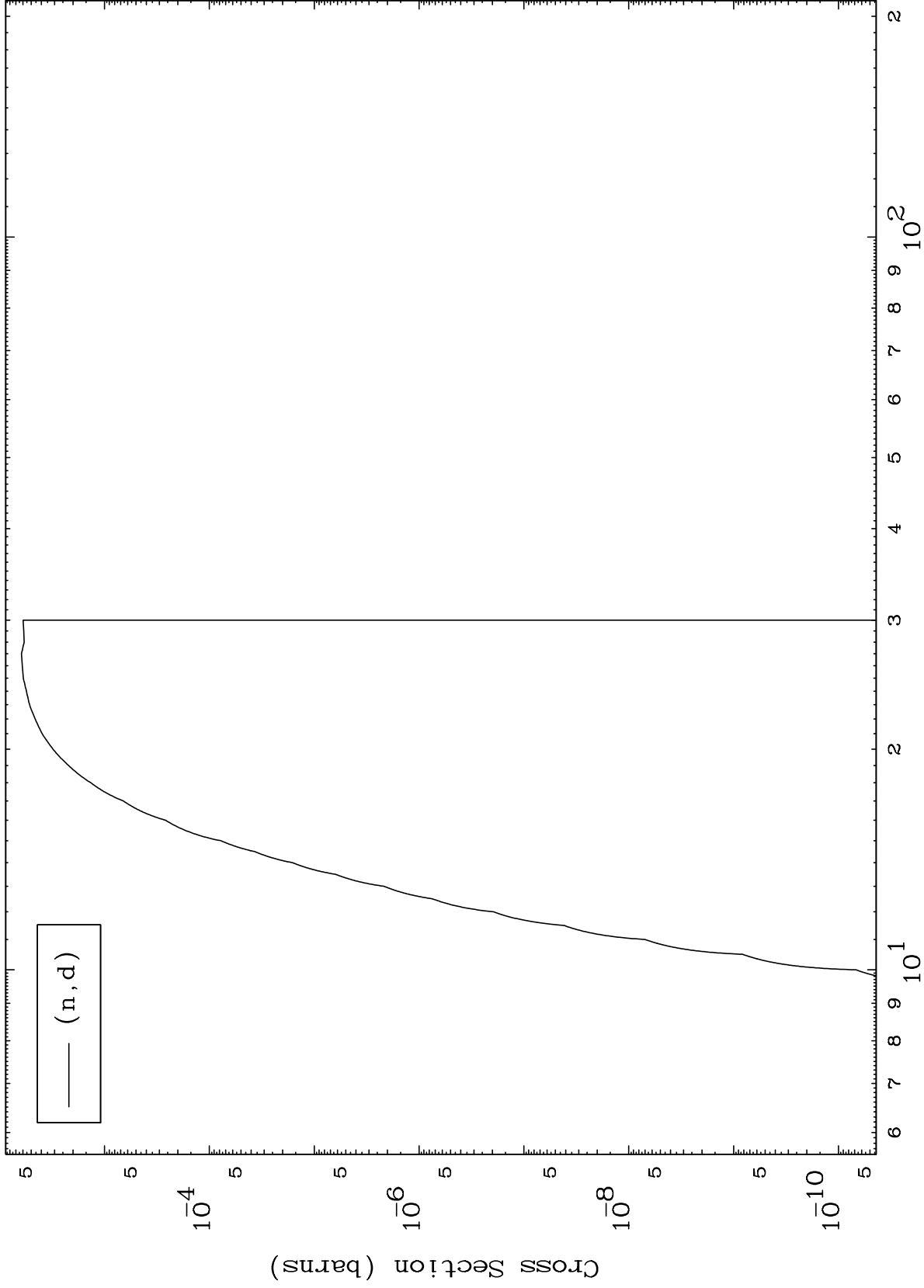
71-Lu-182



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(n,d) Levels
293 Kelvin Cross Sections

71-Lu-182



9

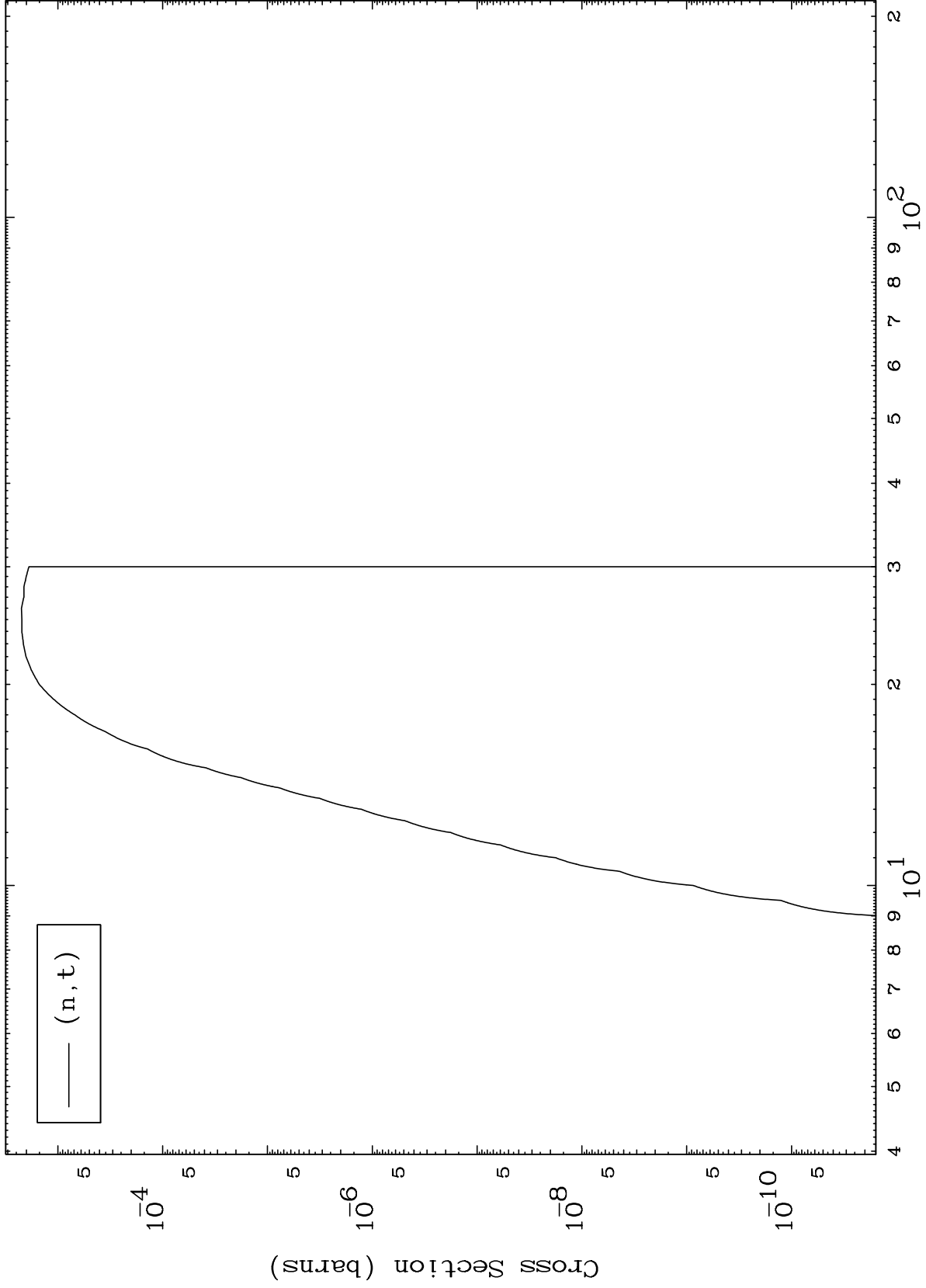
Incident Energy (MeV)

71-Lu-182

MAT 7146

(n,t) Levels
293 Kelvin Cross Sections

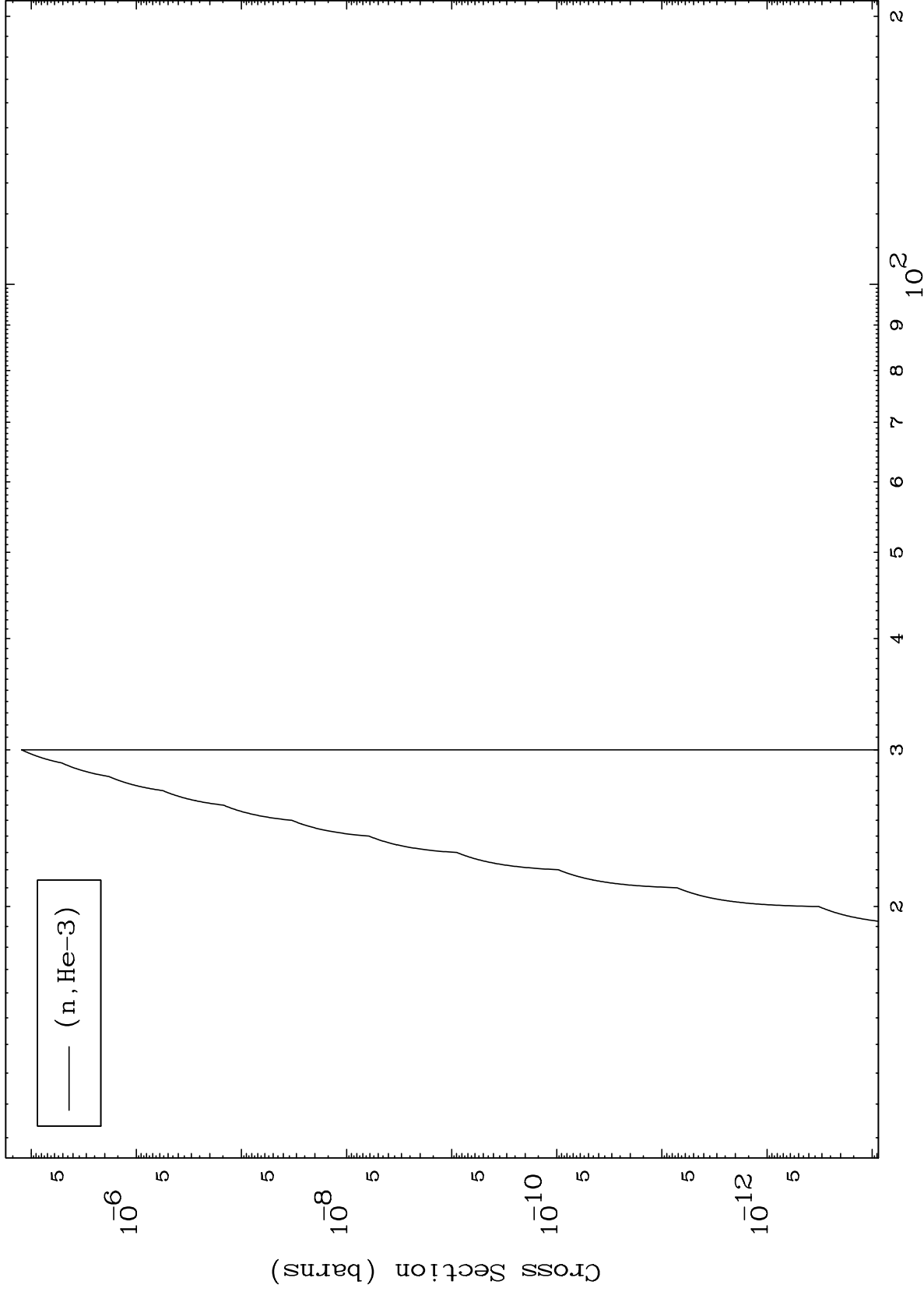
71-Lu-182



10

Incident Energy (MeV)

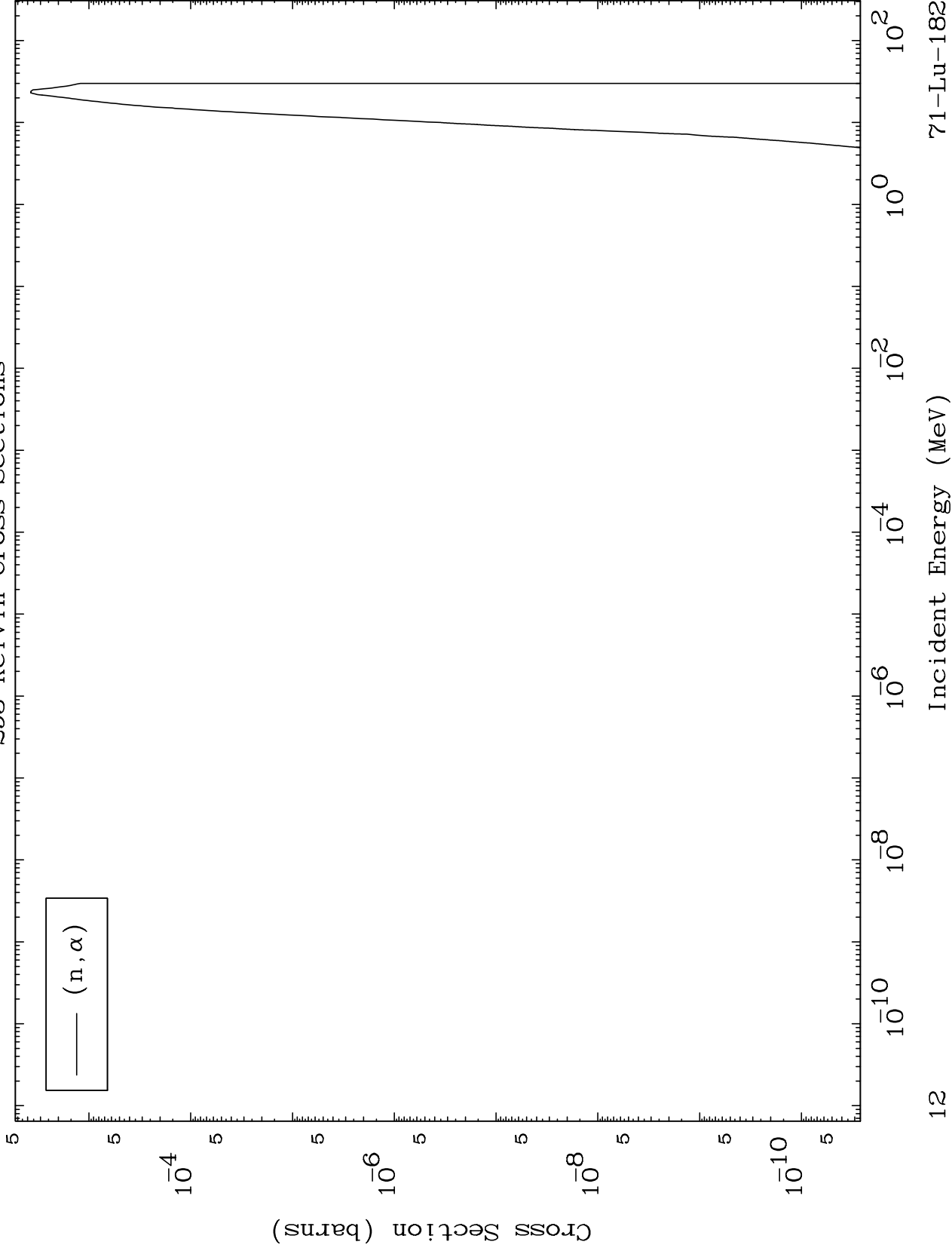
71-Lu-182



MAT 7146

(n, α) Levels
293 Kelvin Cross Sections

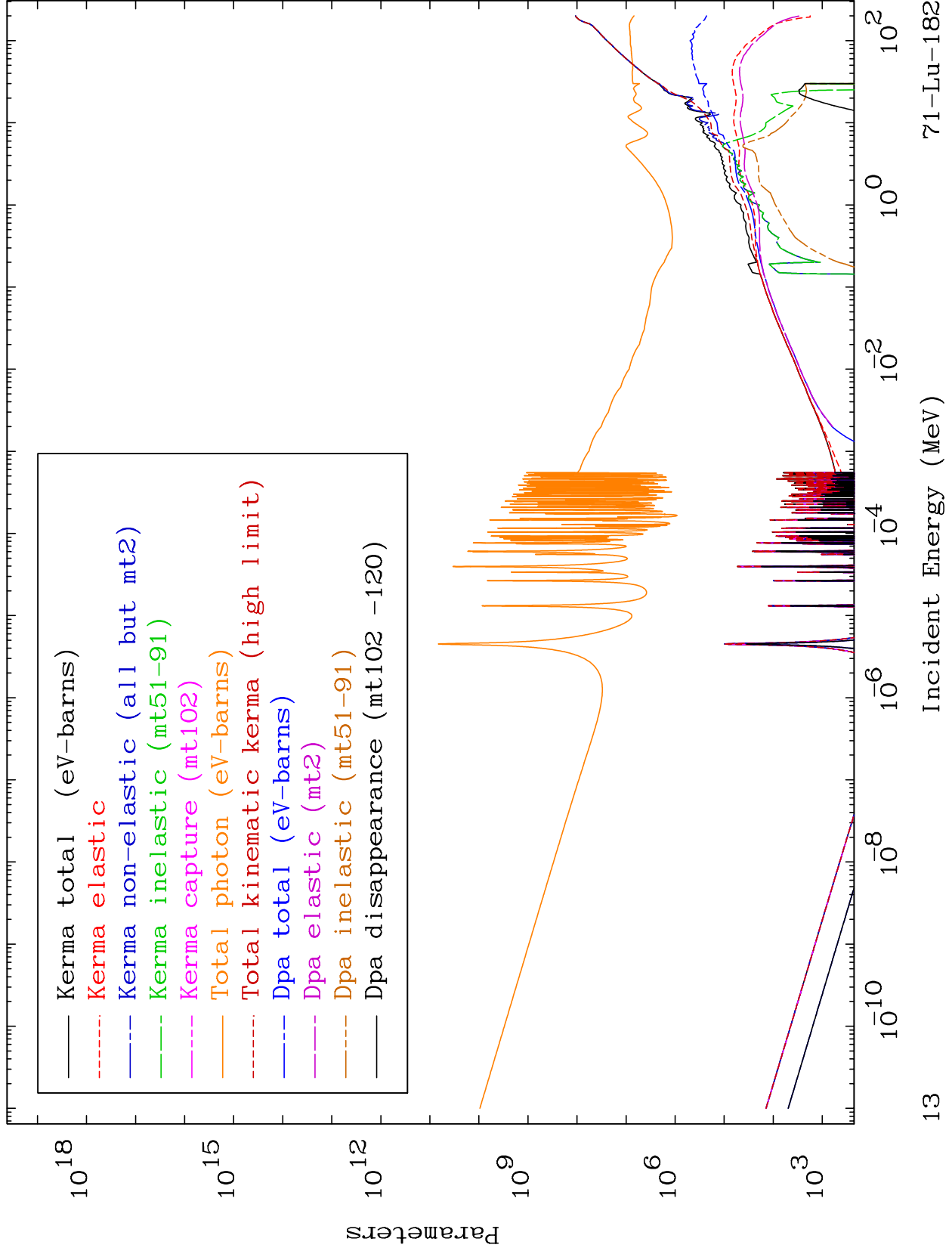
71-Lu-182



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Incident Energy (MeV)

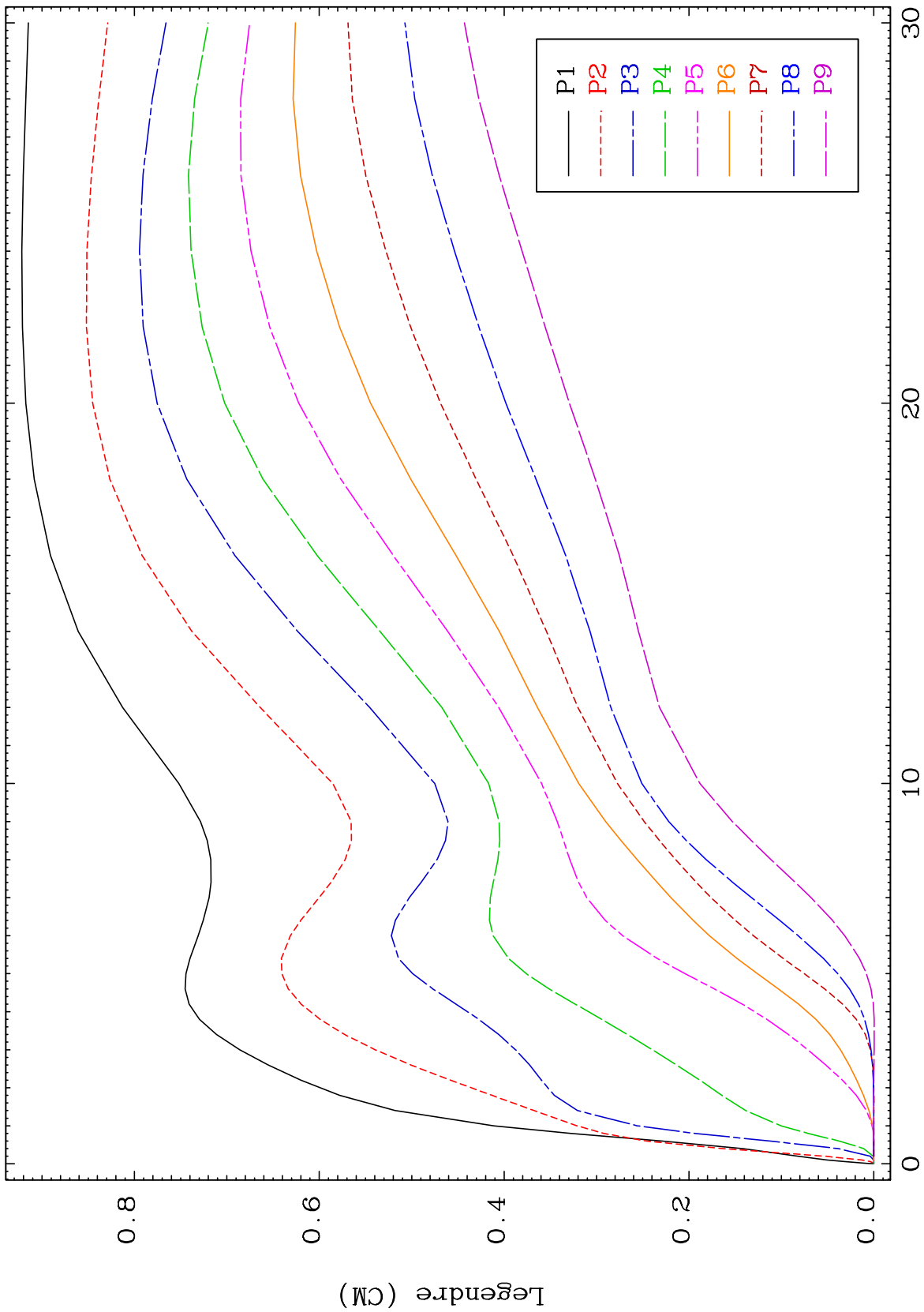
71-Lu-182



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71-Lu-182

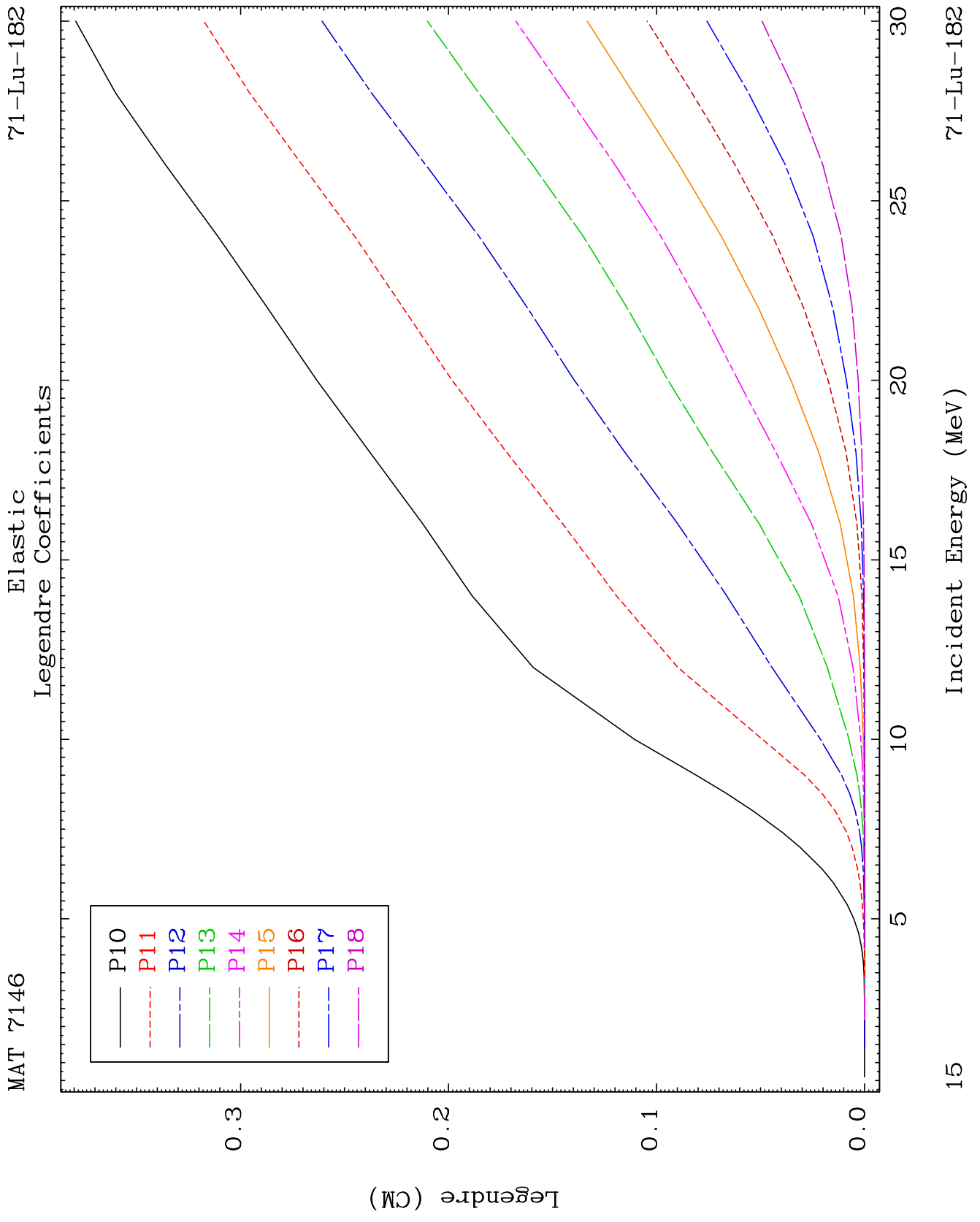
Elastic Legendre Coefficients



14

Incident Energy (MeV)

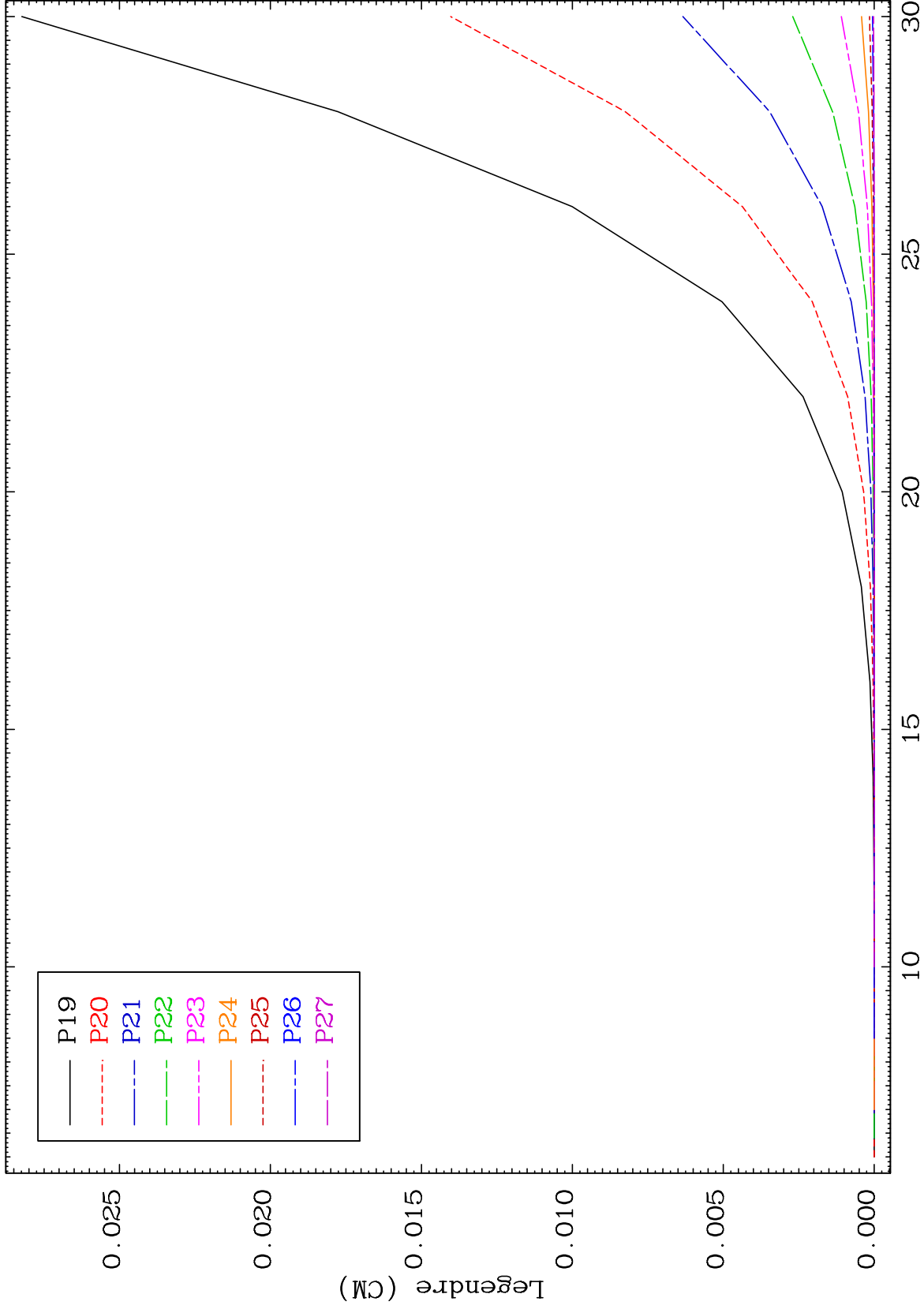
71-Lu-182



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Elastic Legendre Coefficients

71-Lu-182



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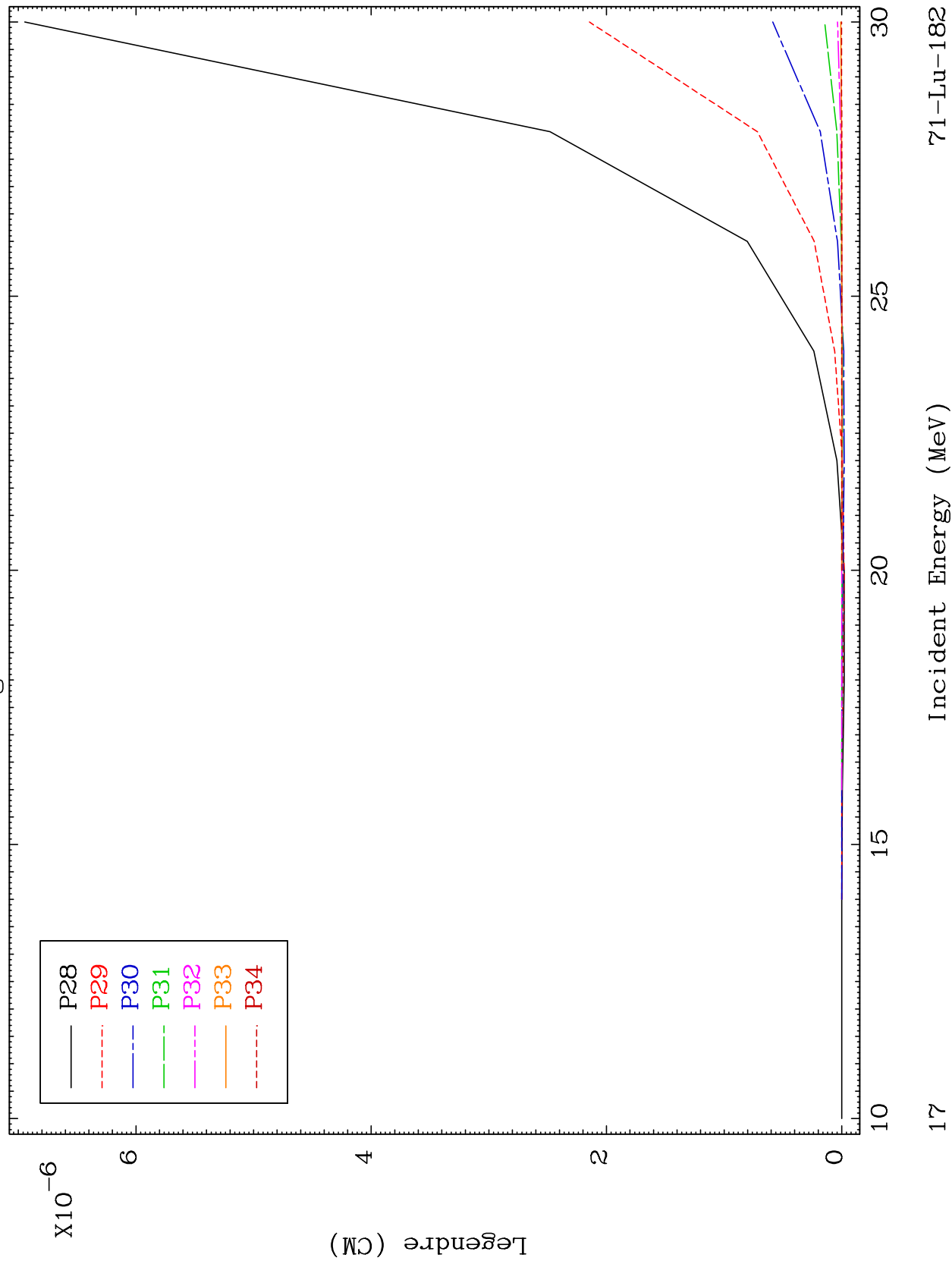
Incident Energy (MeV)

71-Lu-182

MAT 7146

Elastic Legendre Coefficients

71-Lu-182



71-Lu-182

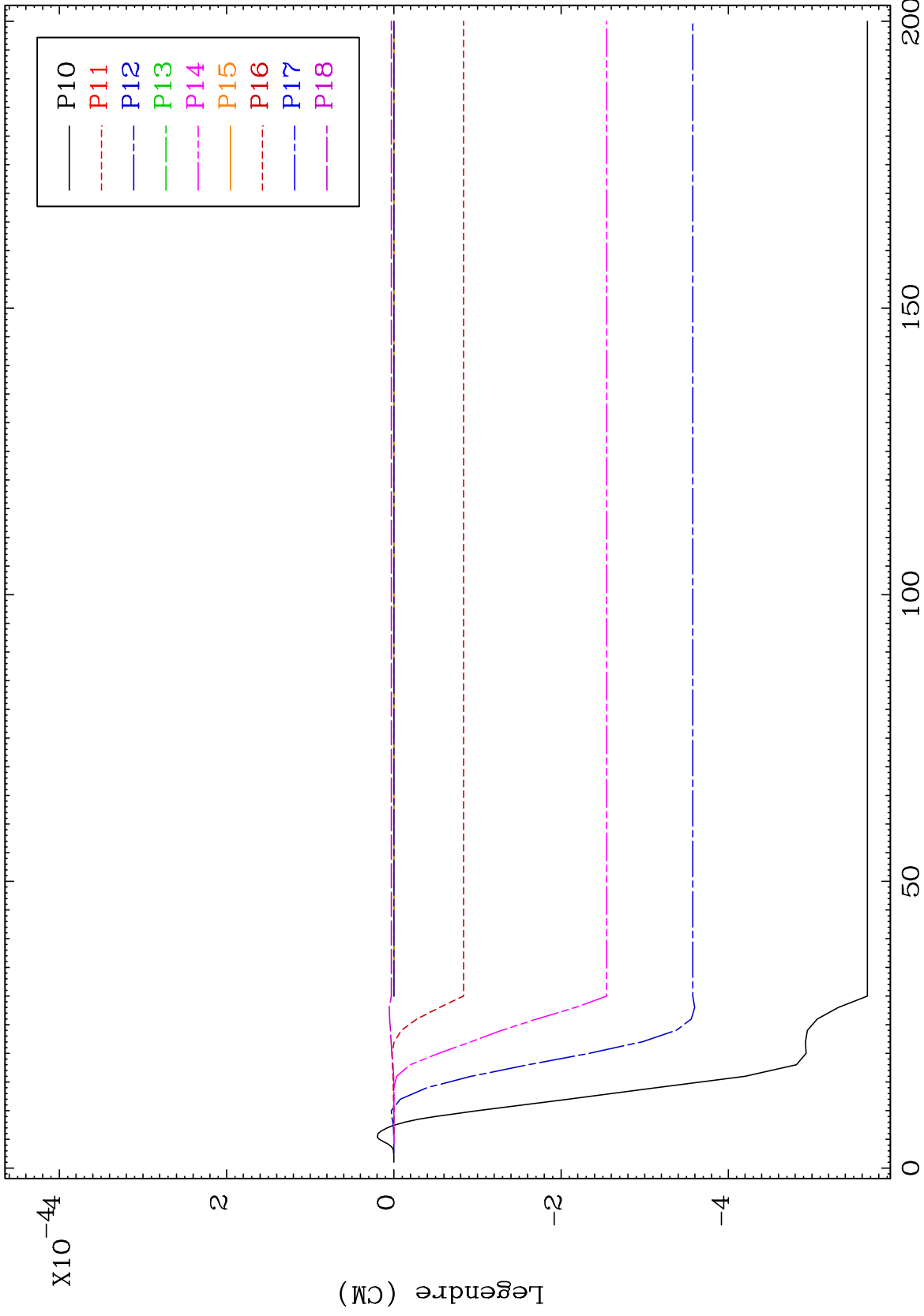
Incident Energy (MeV)

17

MAT 7146

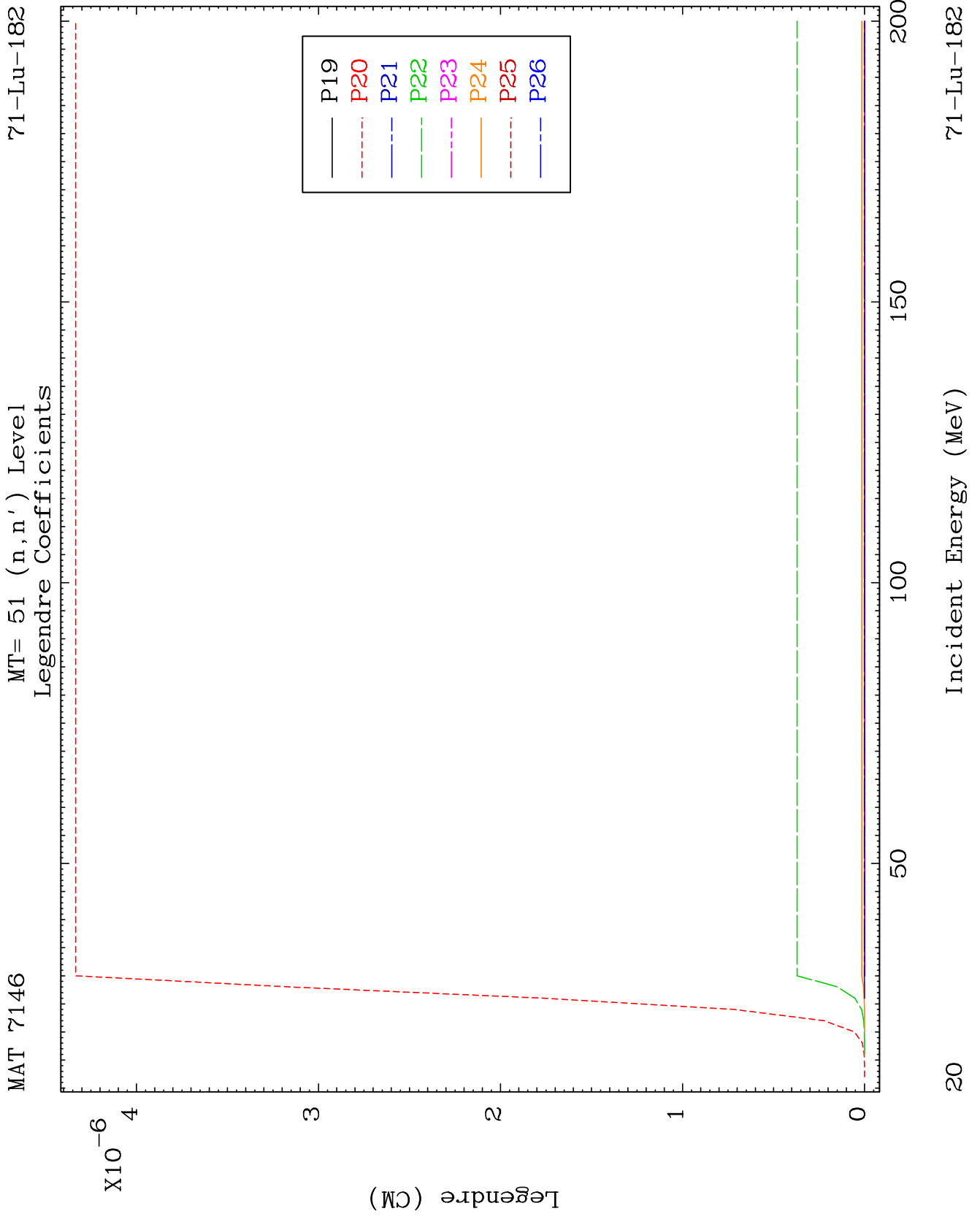
MT= 51 (n,n') Level
Legendre Coefficients

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71-Lu-182



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Fission

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Radionuclide Production Cross Section

