

Program EVALPLOT  
(Version 2017-1)

by

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(Present Contact Information)

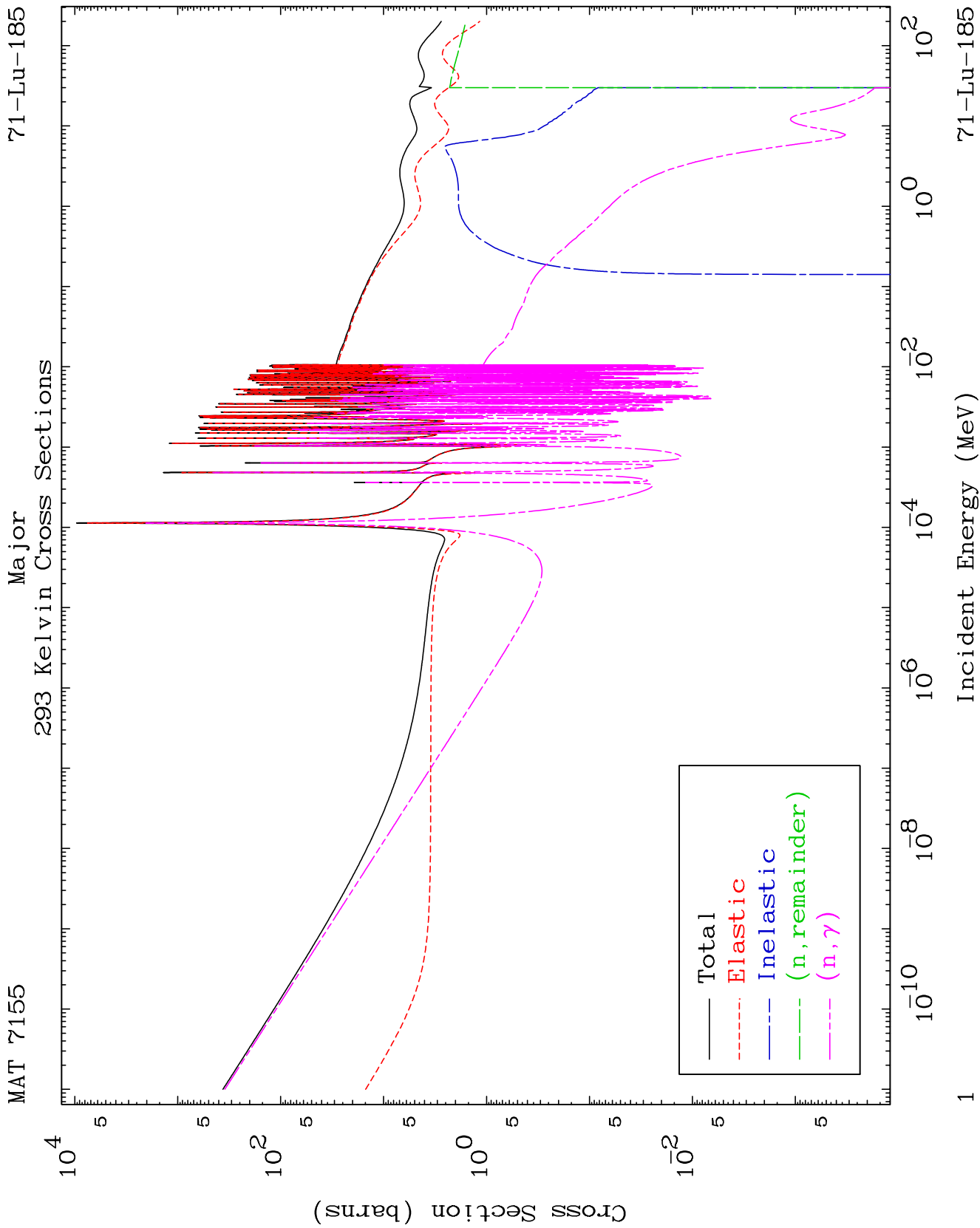
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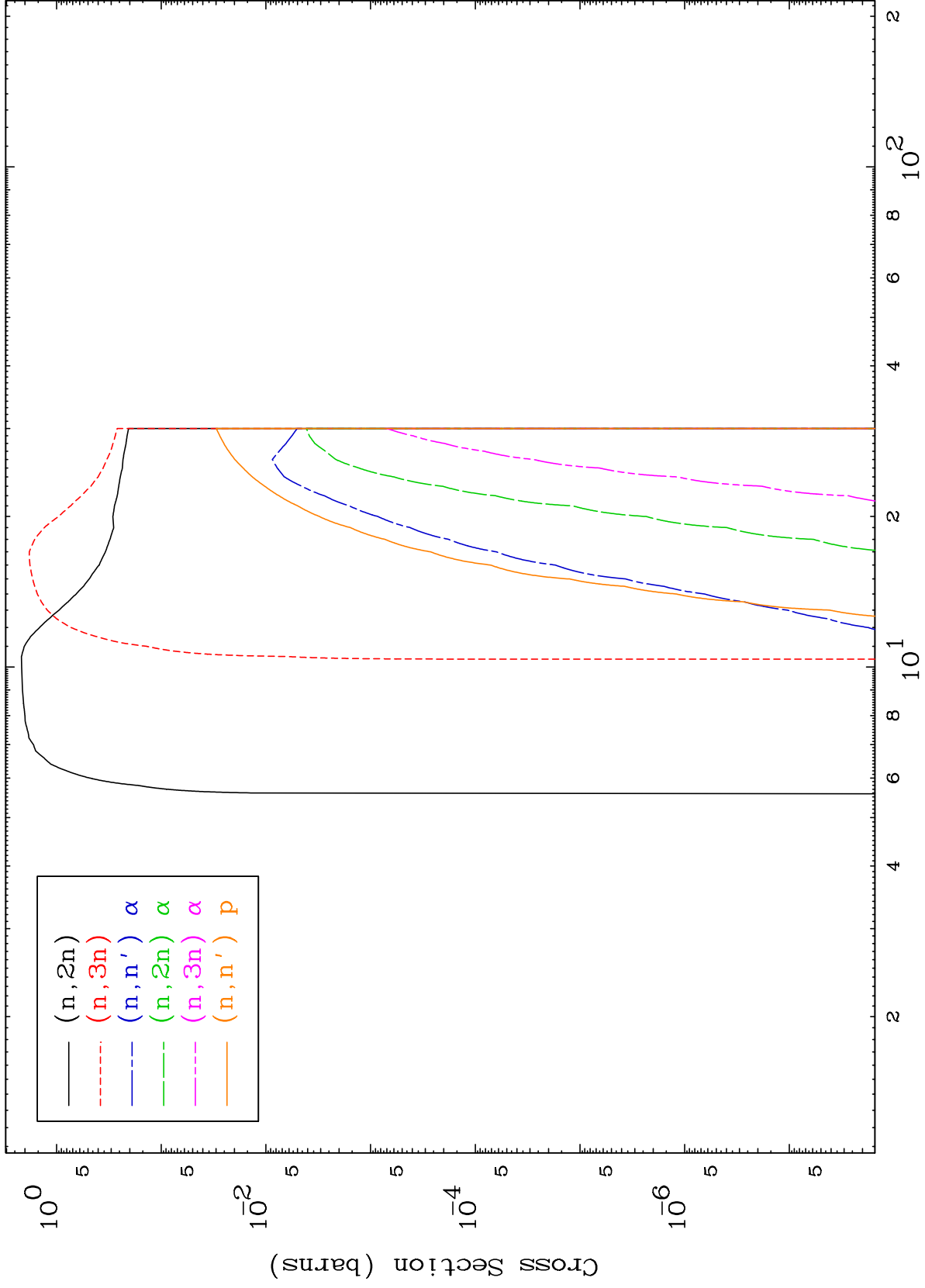
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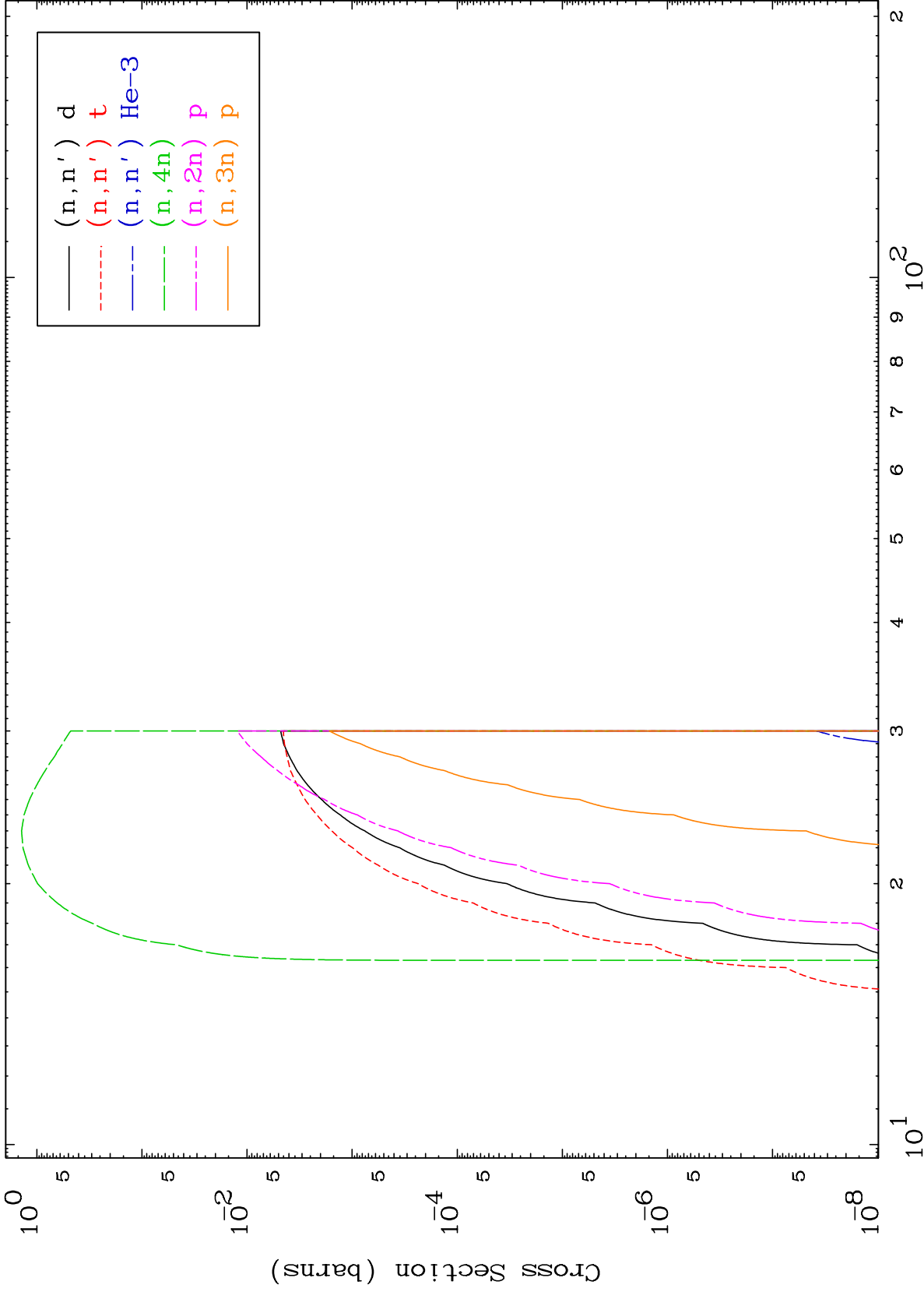


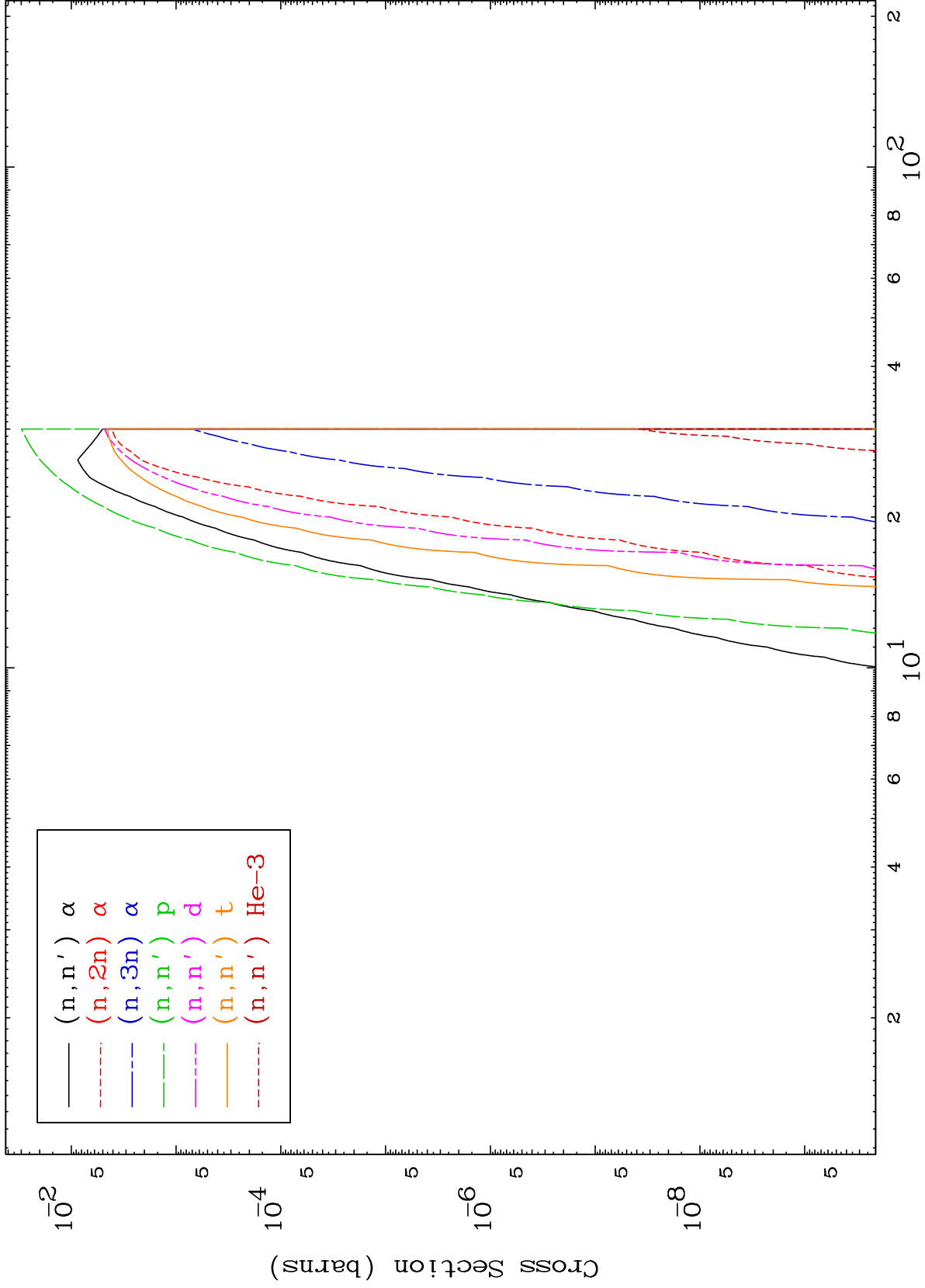
MAT 7155

Neutron Production  
293 Kelvin Cross Sections

71-Lu-185



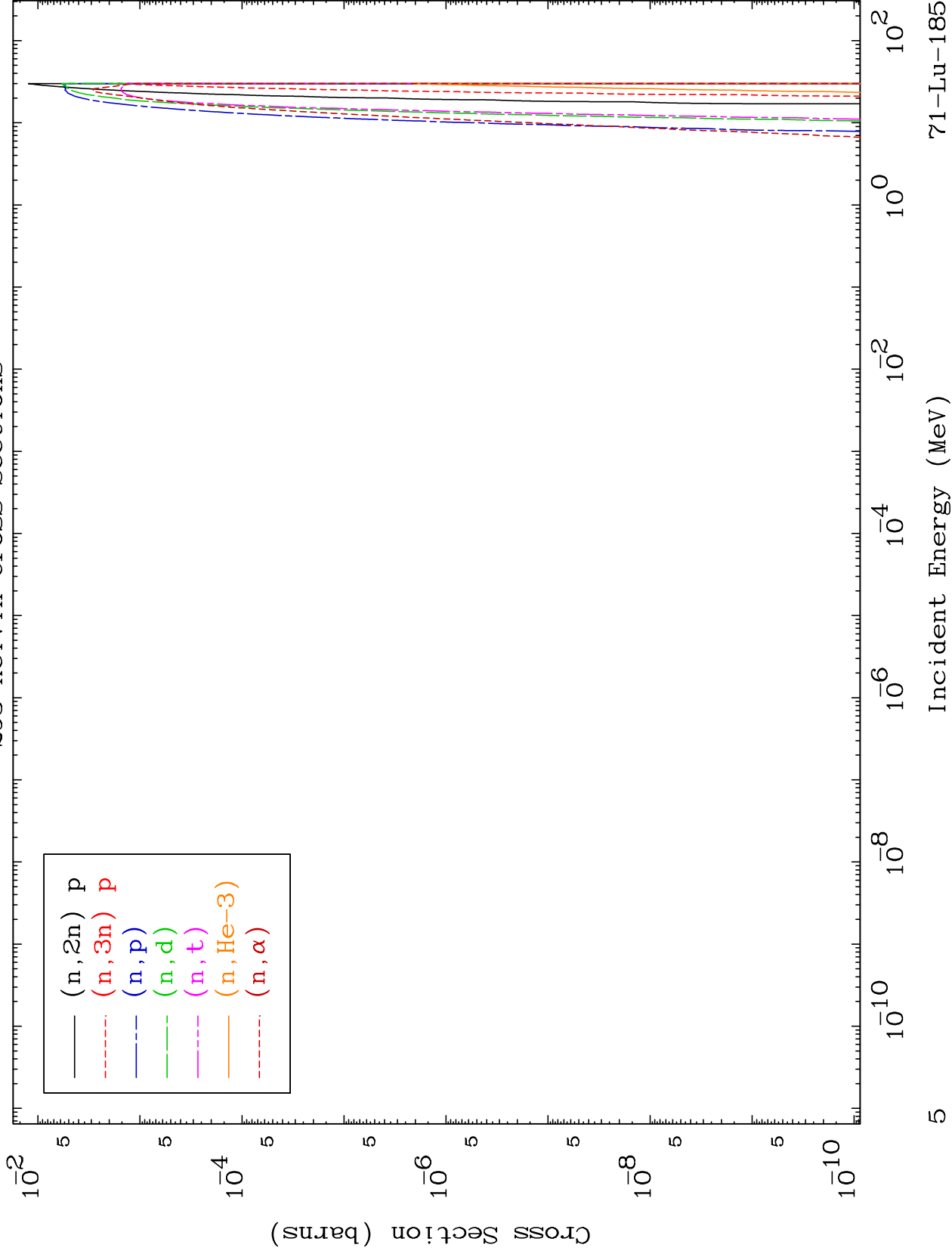




MAT 7155

Charged Particle  
293 Kelvin Cross Sections

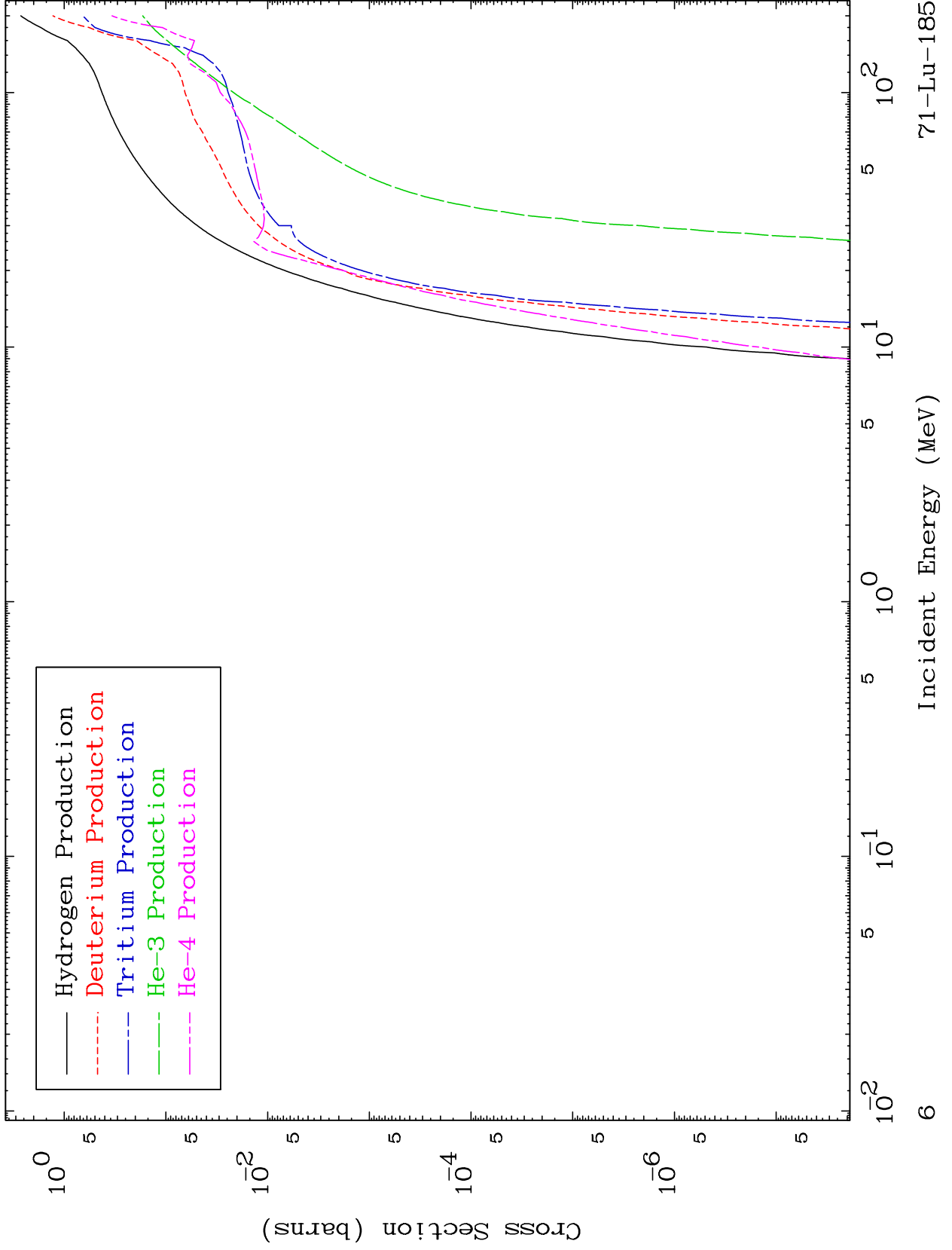
71-Lu-185



MAT 7155

Particle Production  
293 Kelvin Cross Sections

71-Lu-185



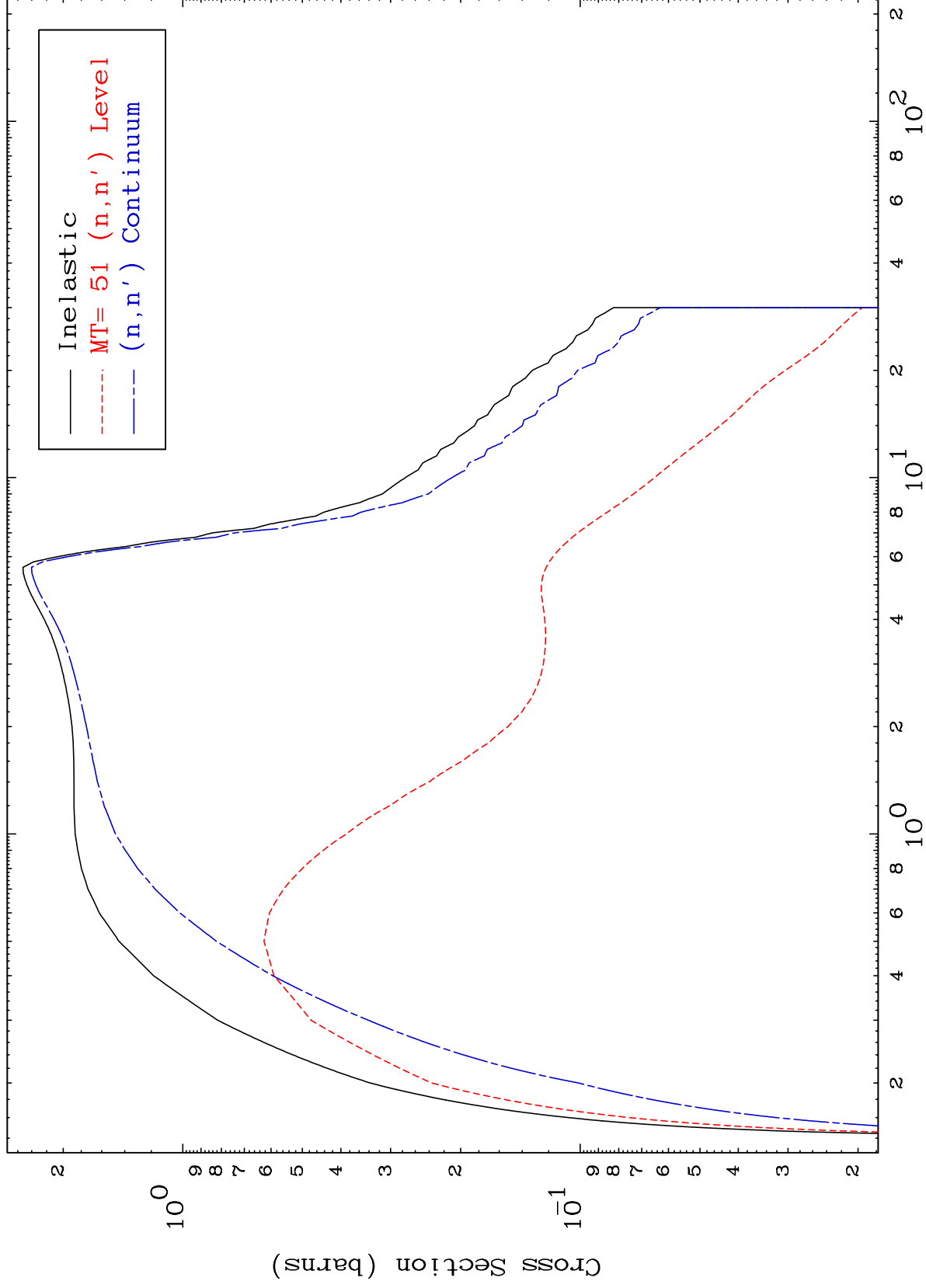
71-Lu-185

MAT 7155

(n,n') Level

71-Lu-185

293 Kelvin Cross Sections



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Incident Energy (MeV)

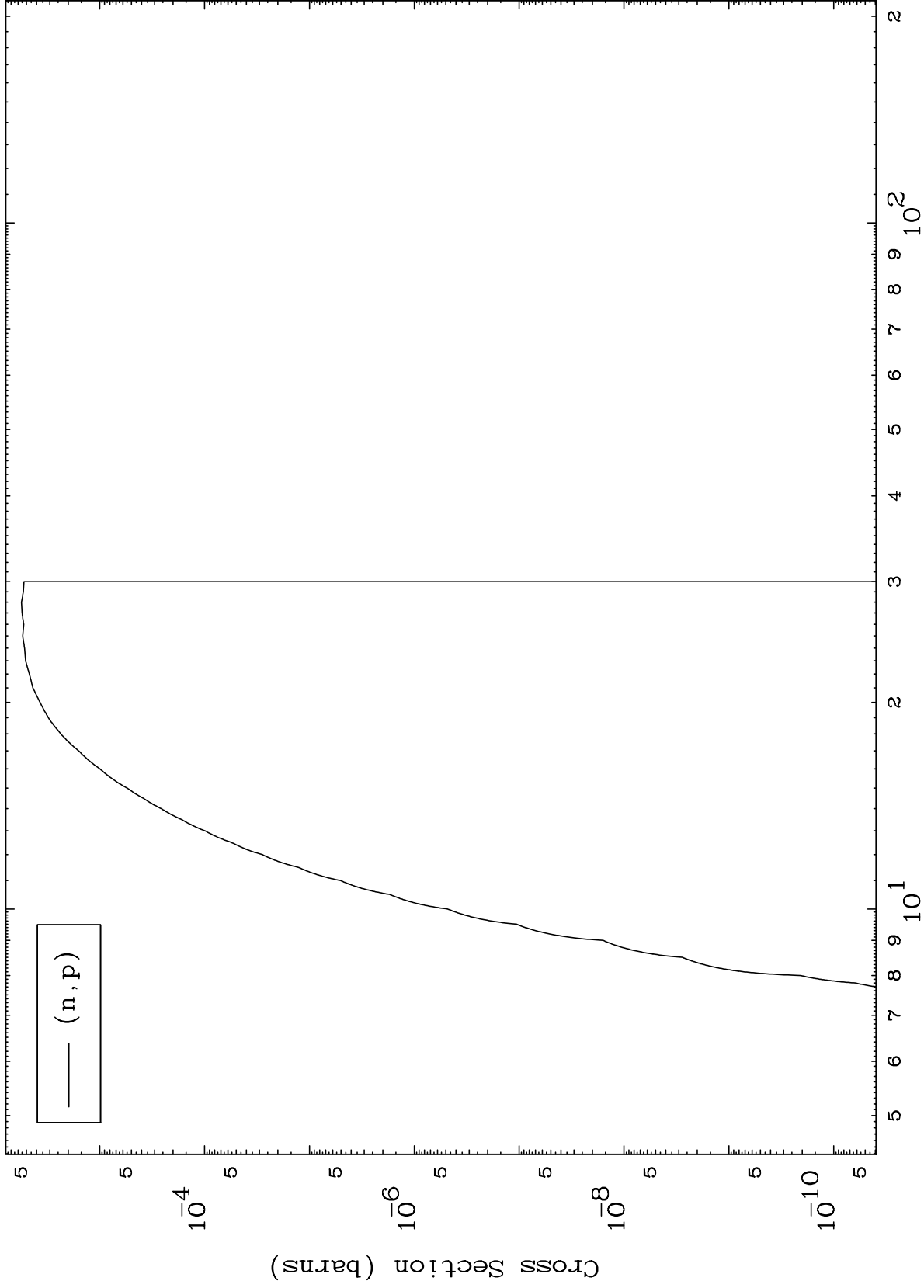
71-Lu-185



MAT 7155

(n,p) Levels  
293 Kelvin Cross Sections

71-Lu-185



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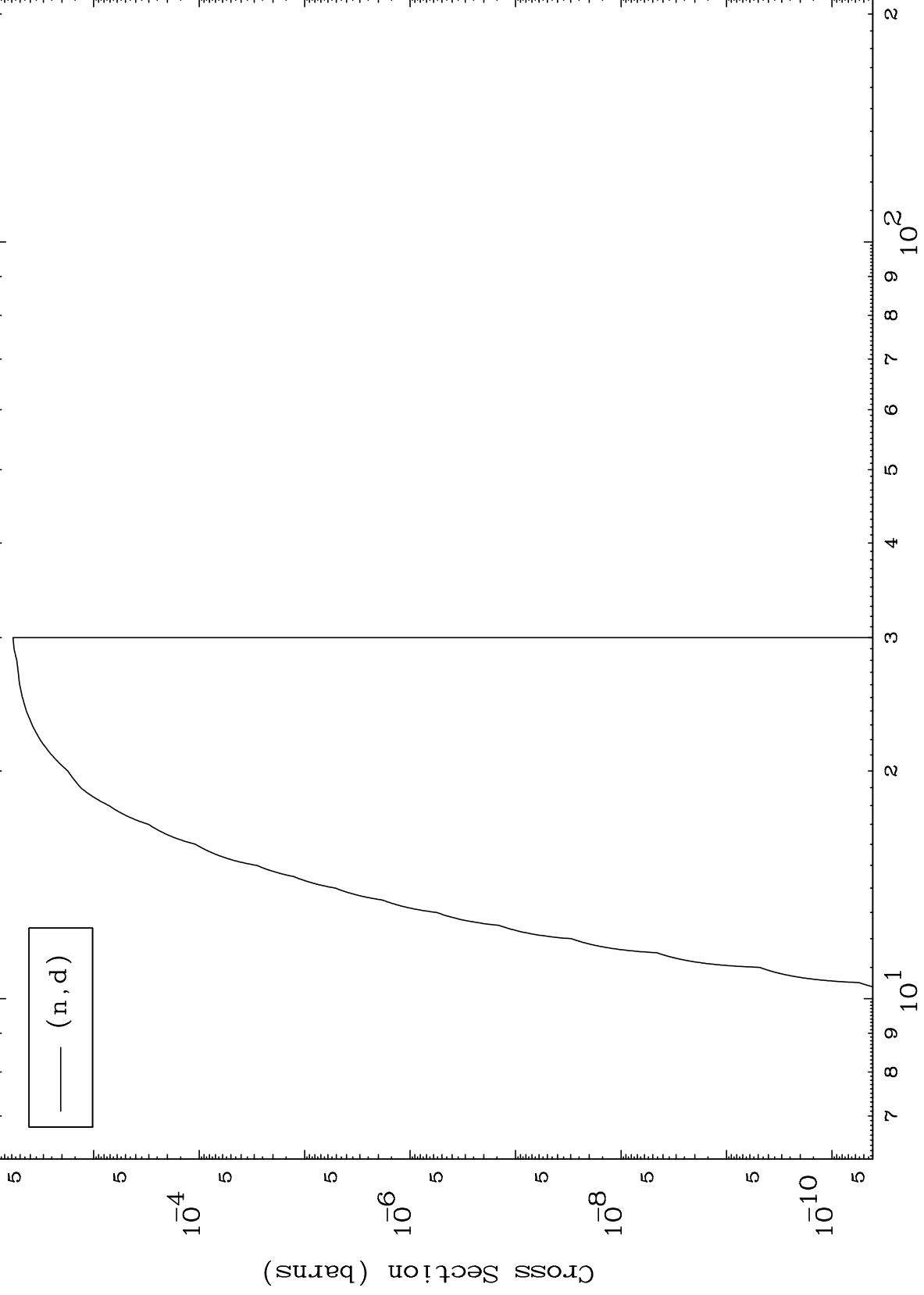
Incident Energy (MeV)

71-Lu-185

MAT 7155

(n,d) Levels  
293 Kelvin Cross Sections

71-Lu-185



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Incident Energy (MeV)

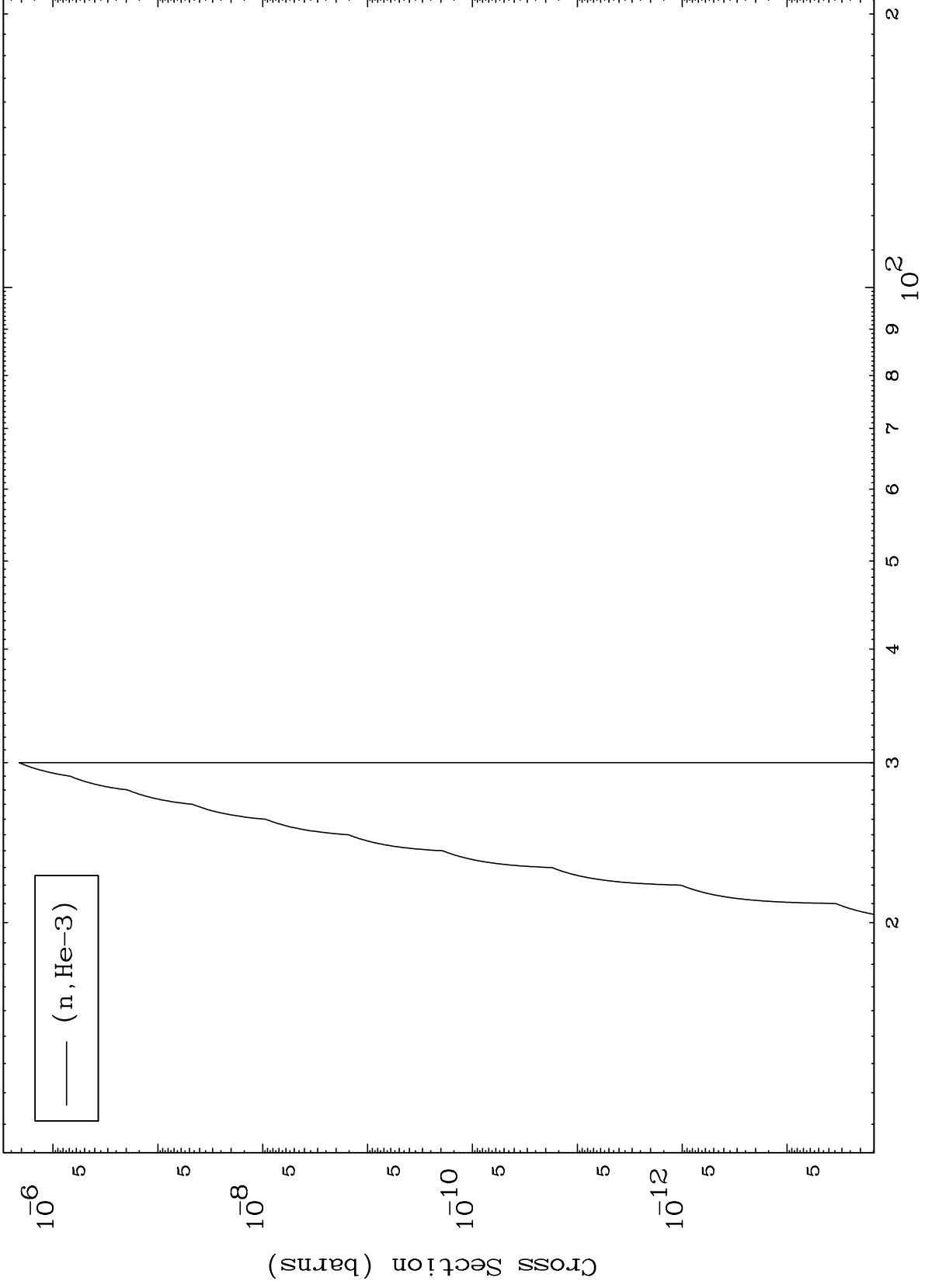
71-Lu-185



MAT 7155

(n,He3) Levels  
293 Kelvin Cross Sections

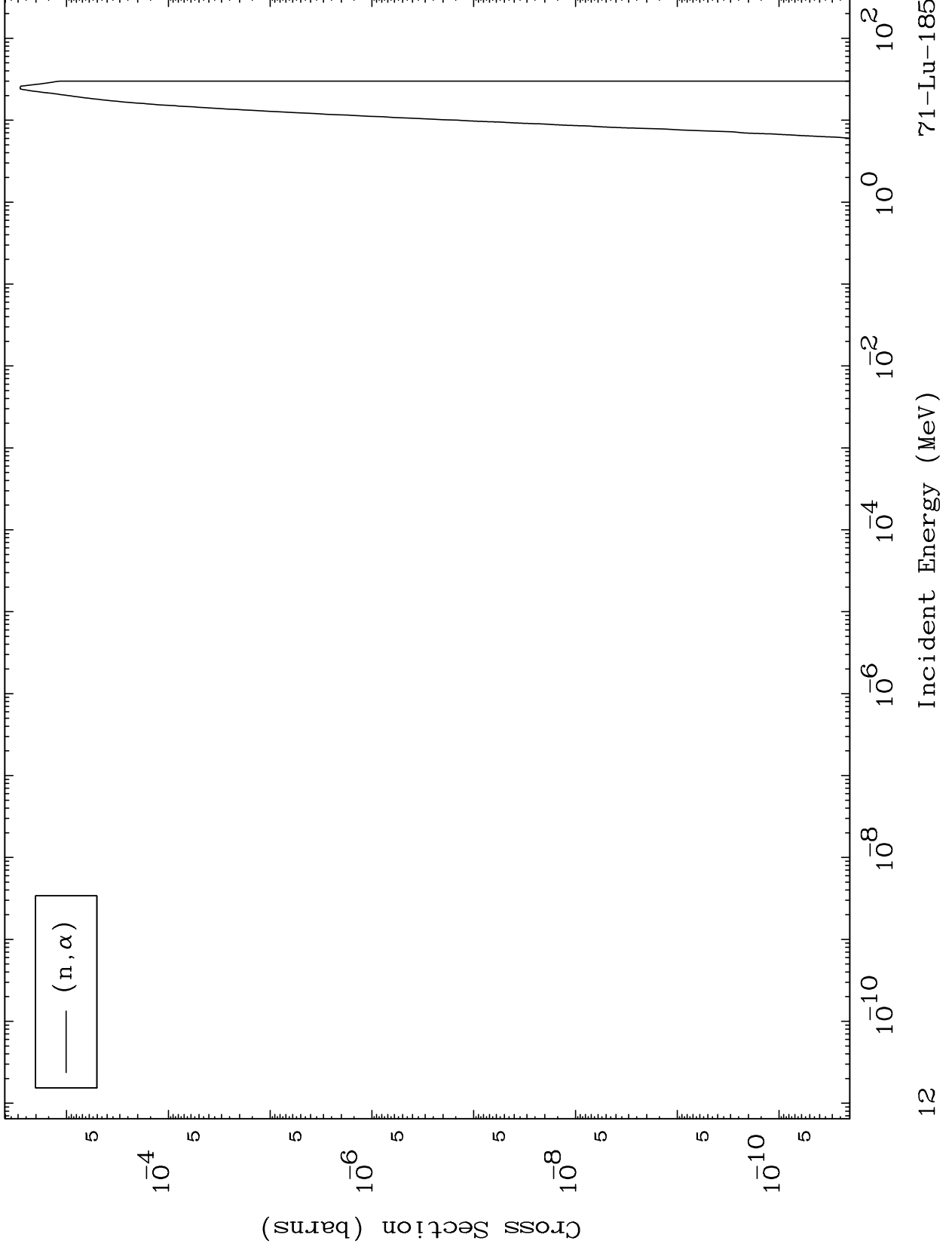
71-Lu-185

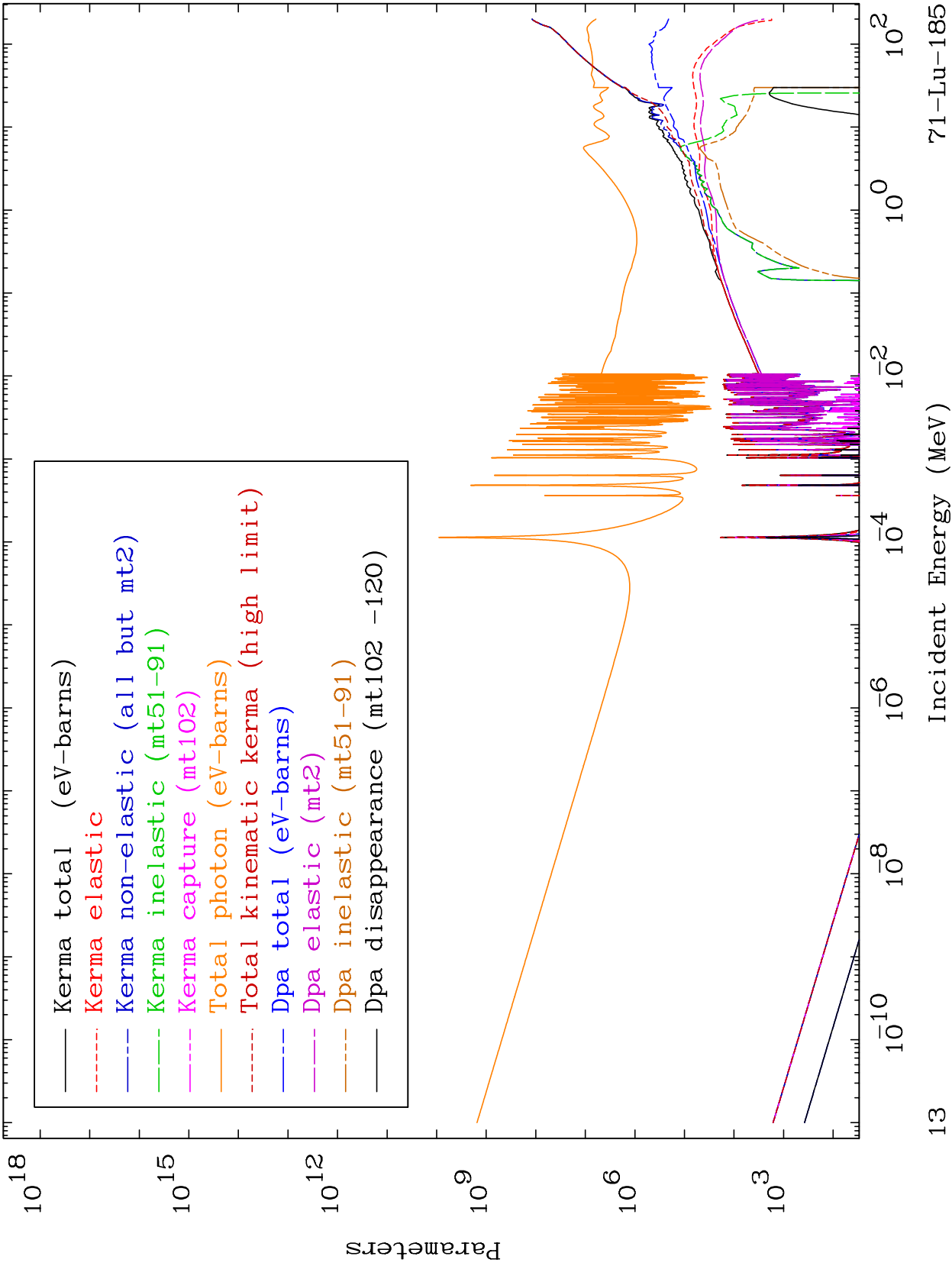


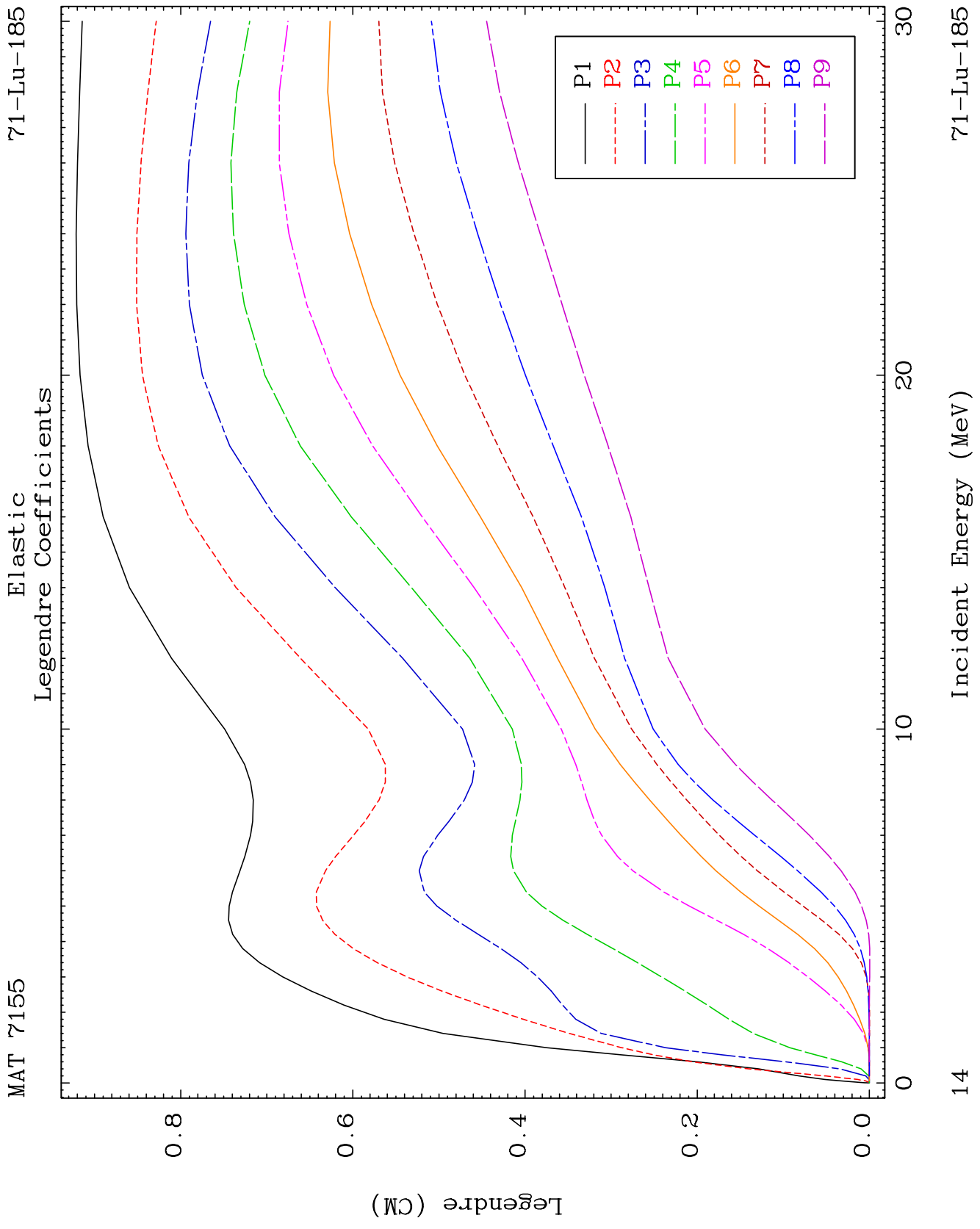
MAT 7155

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

71-Lu-185



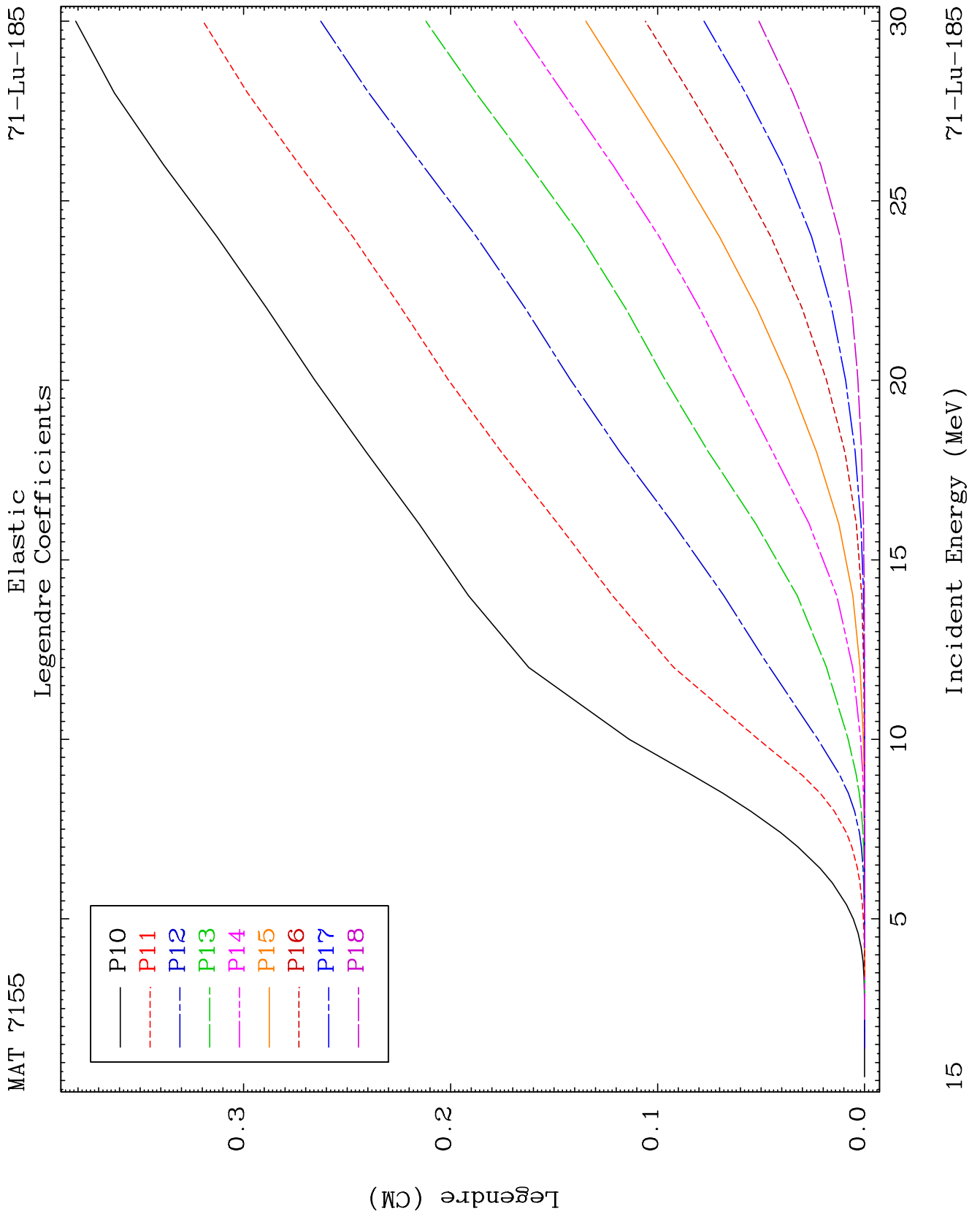




71-Lu-185

Incident Energy (MeV)

14





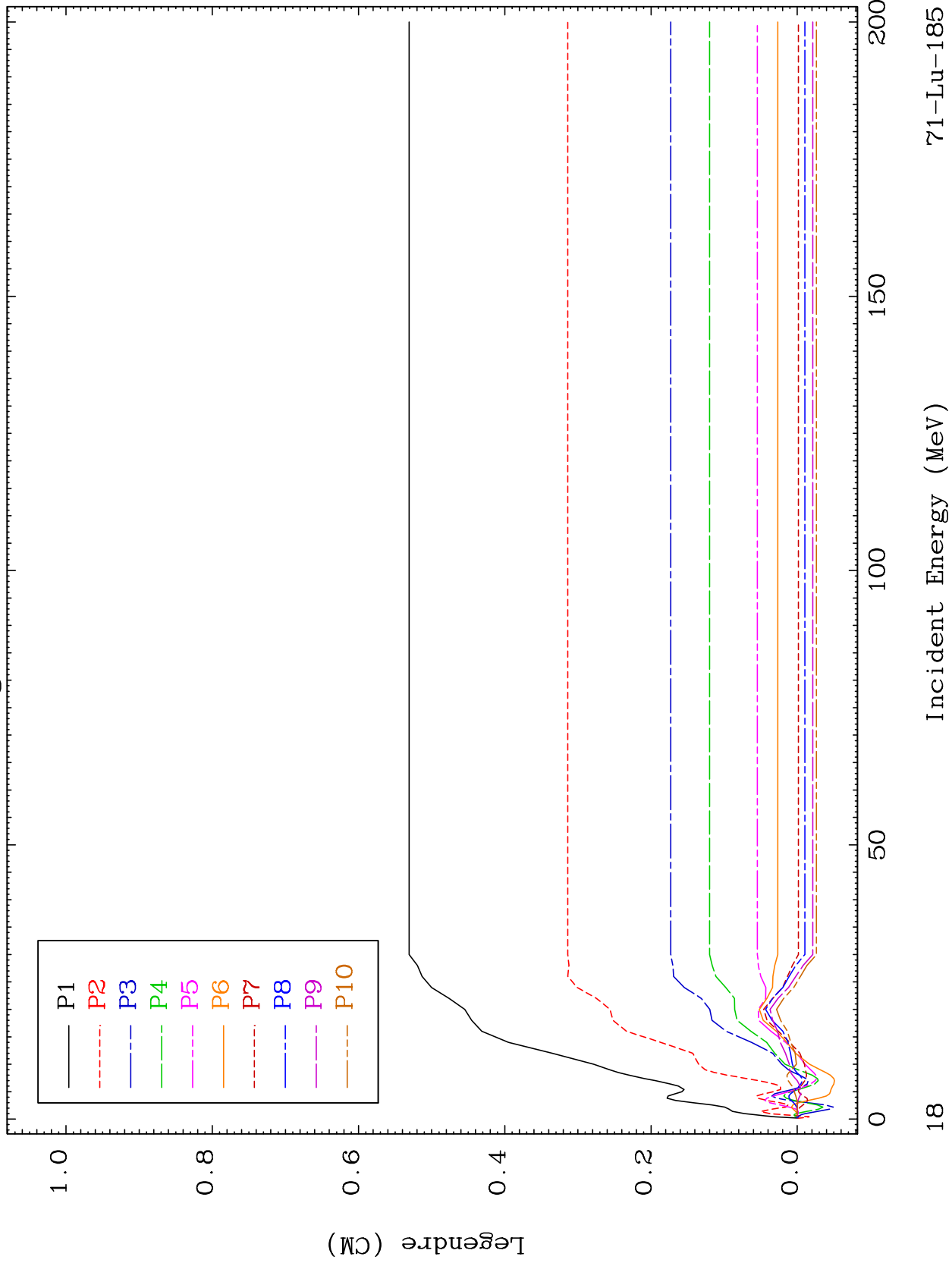




MAT 7155

MT= 51 (n,n') Level  
Legendre Coefficients

71-Lu-185



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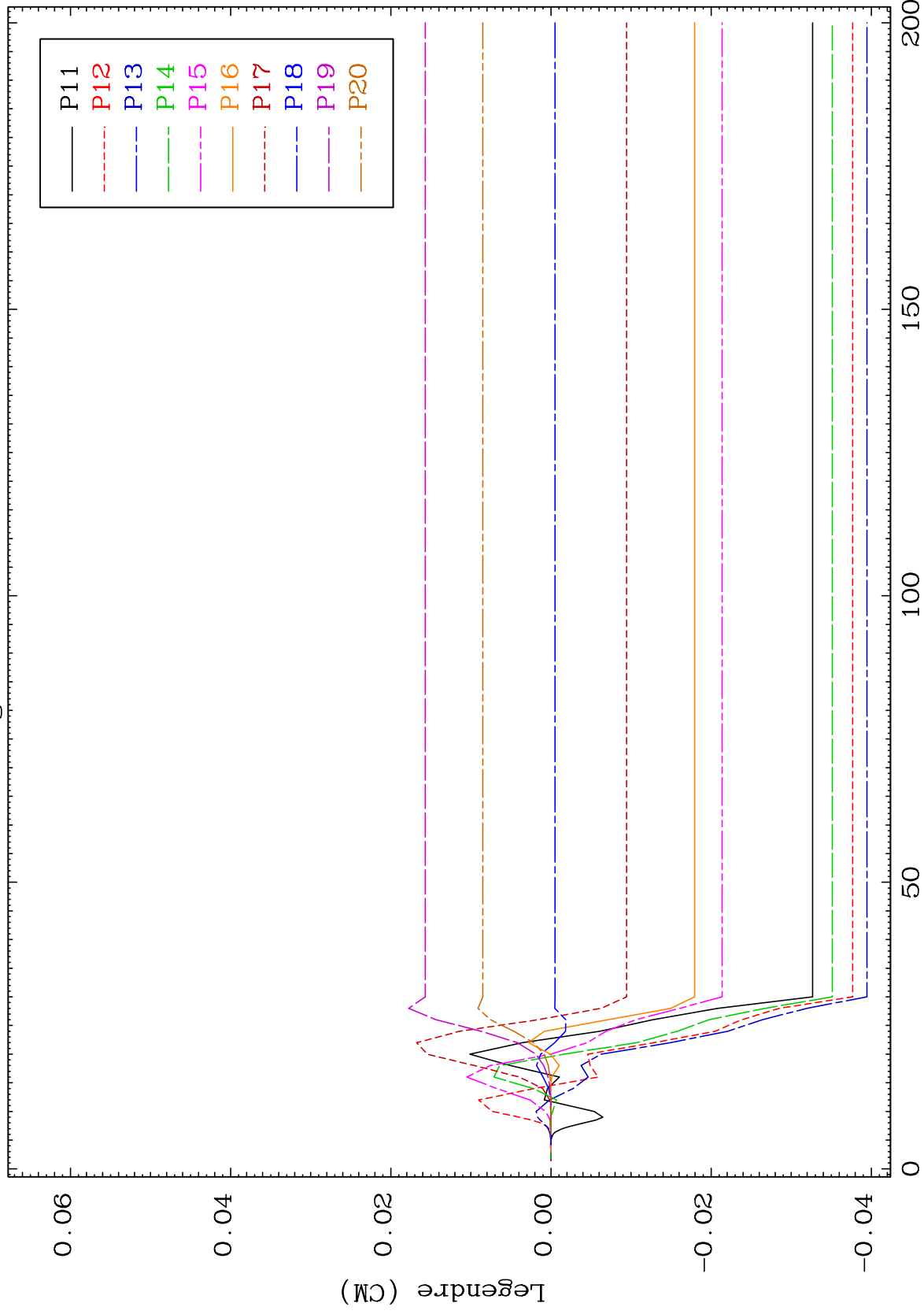
Incident Energy (MeV)

71-Lu-185

MAT 7155

MT= 51 (n,n') Level  
Legendre Coefficients

71-Lu-185



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Incident Energy (MeV)

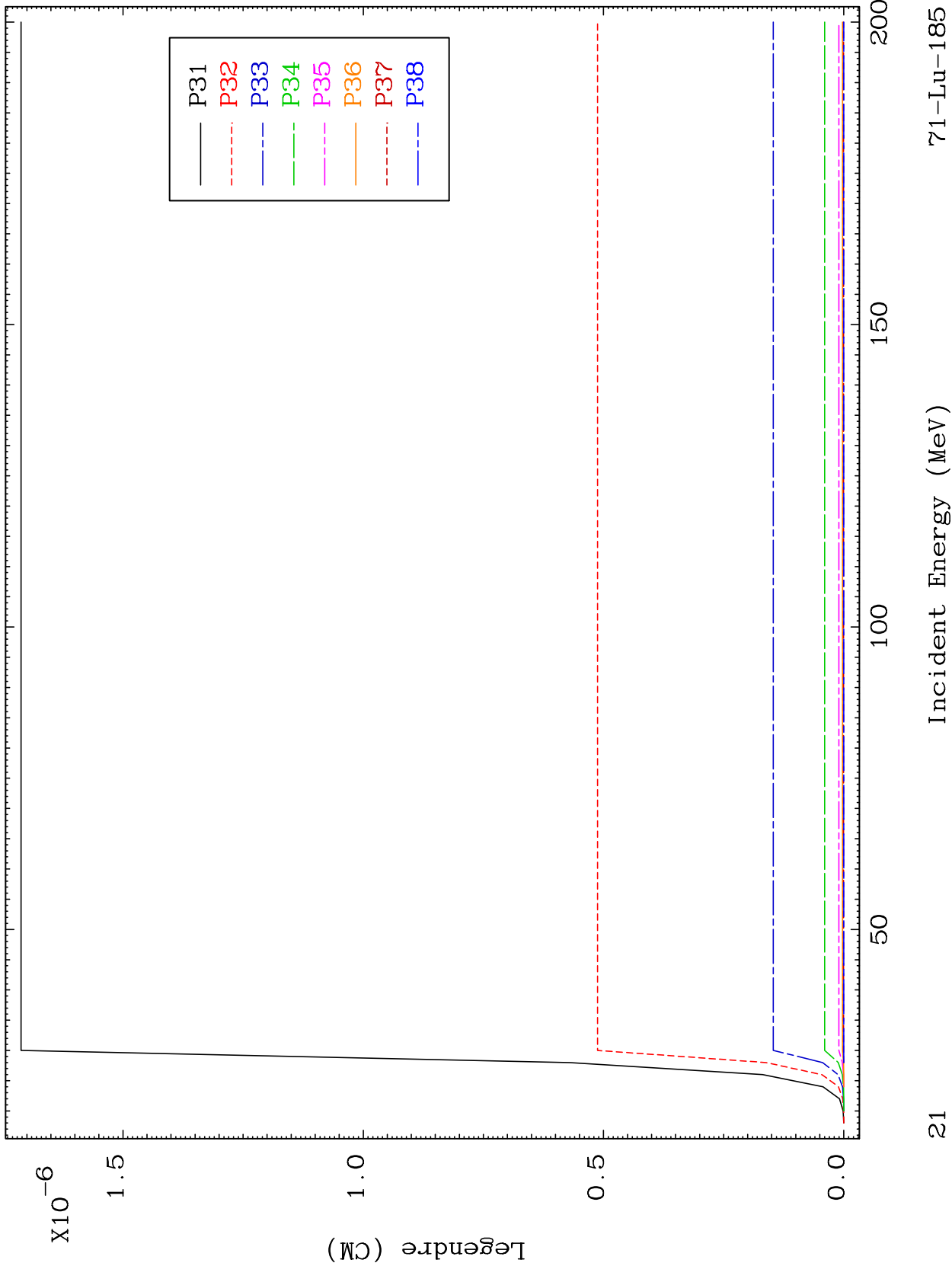
71-Lu-185



MAT 7155

MT= 51 (n,n') Level  
Legendre Coefficients

71-Lu-185



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Incident Energy (MeV)

71-Lu-185

MAT 7155

Fission

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Radionuclide Production Cross Section

