

Program EVALPLOT
(Version 2017-1)

by

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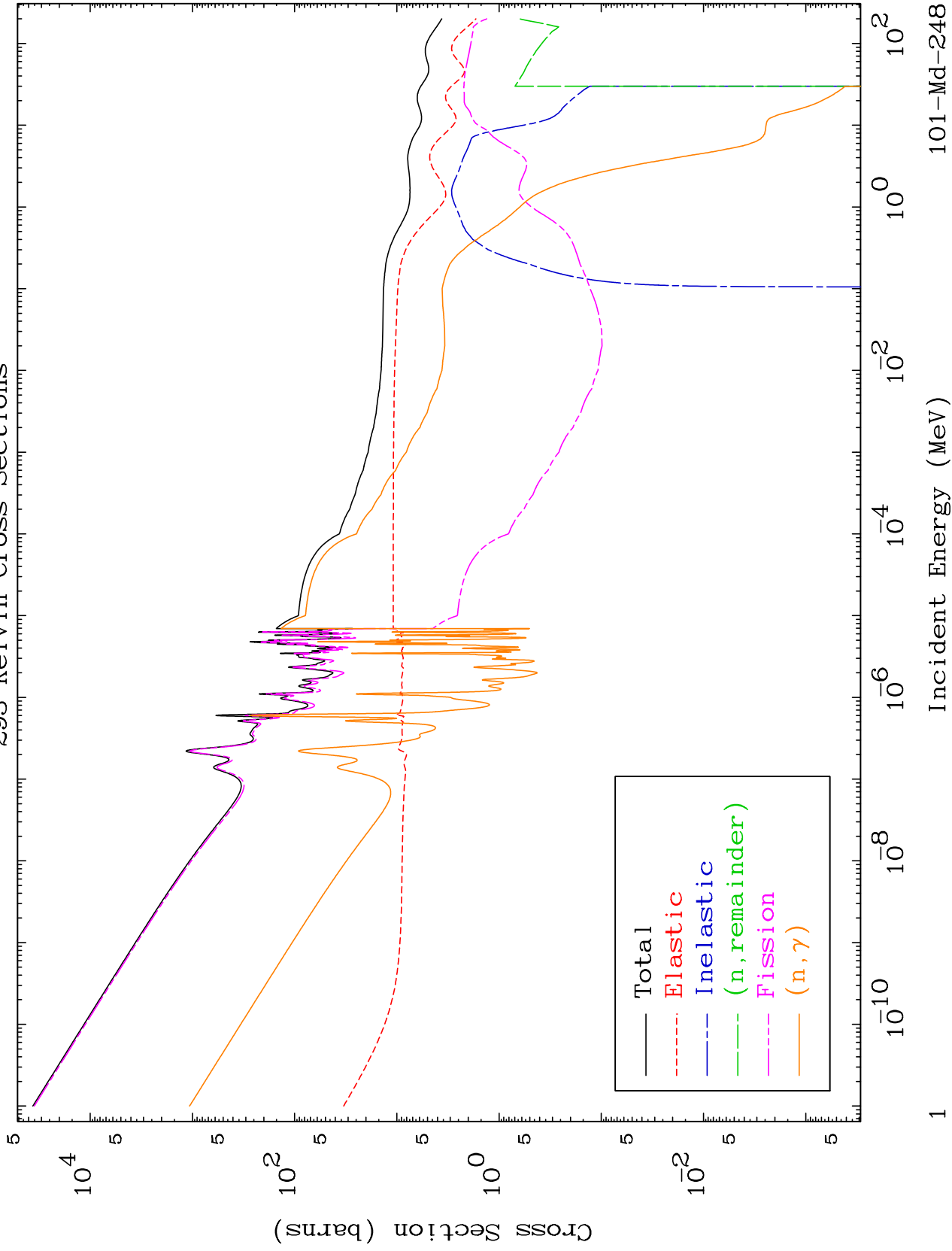
Press Mouse Button to Start

MAT 9949

Major

293 Kelvin Cross Sections

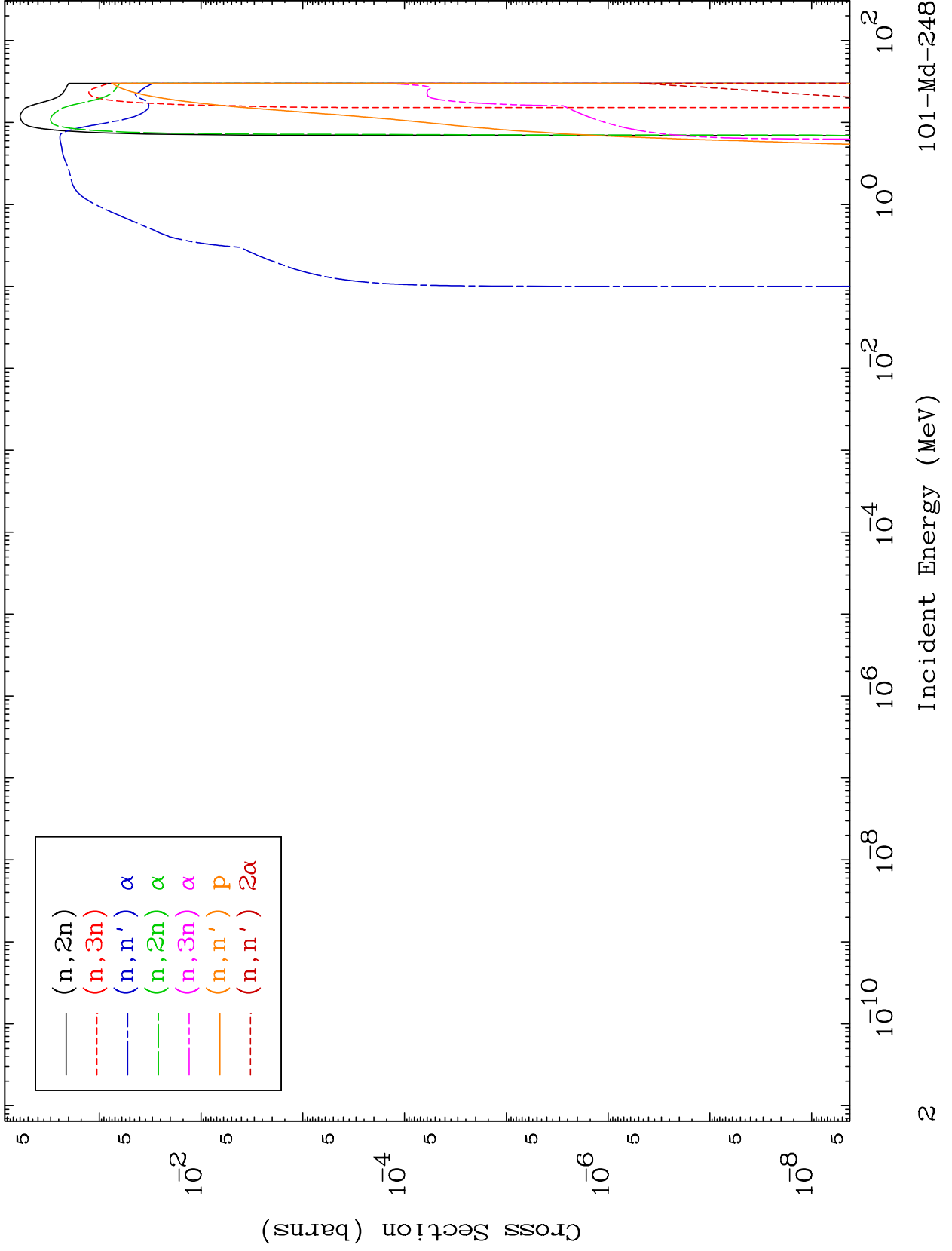
101-Md-248



MAT 9949

Neutron Production
293 Kelvin Cross Sections

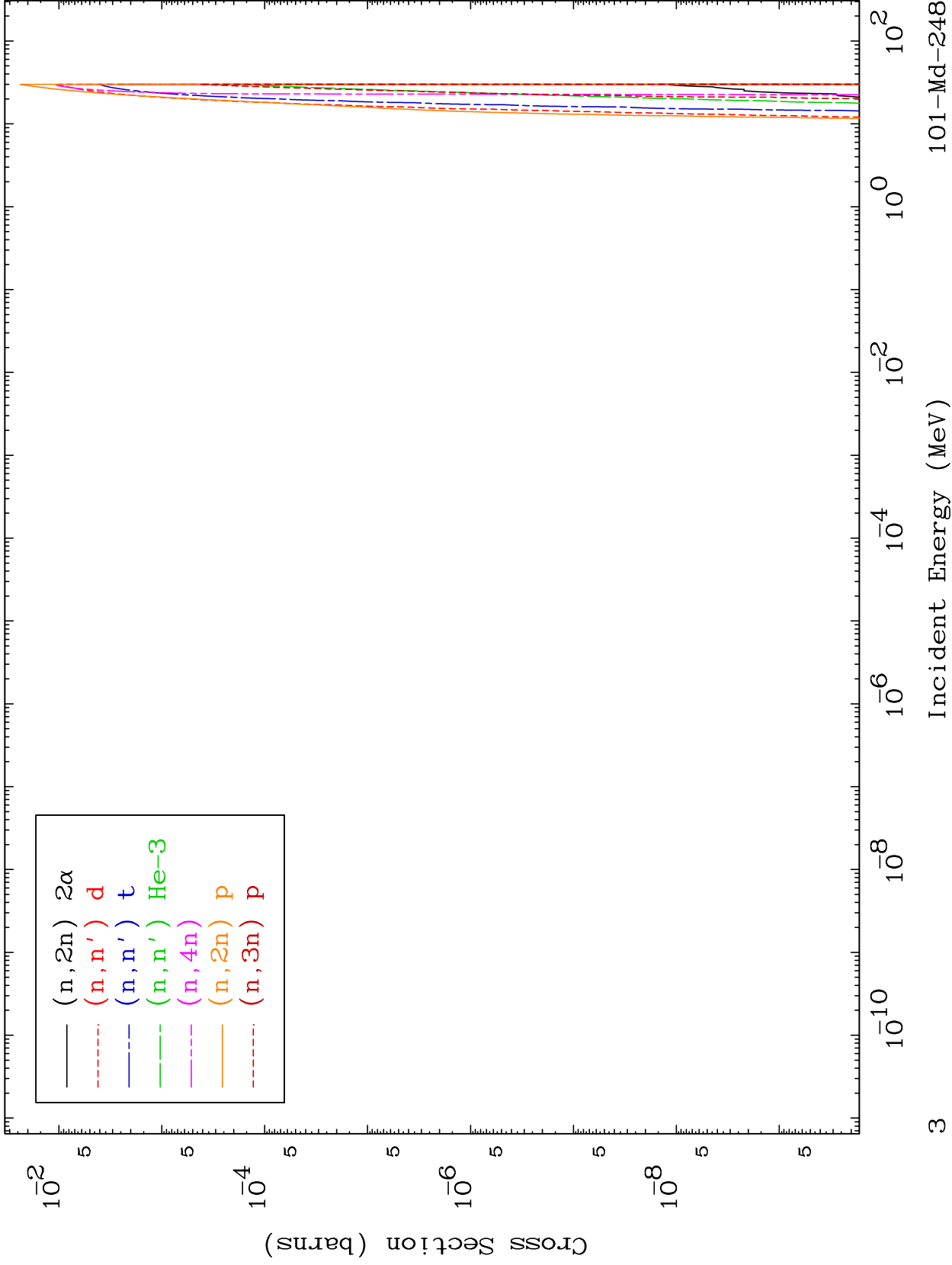
101-Md-248



MAT 9949

Neutron Production
293 Kelvin Cross Sections

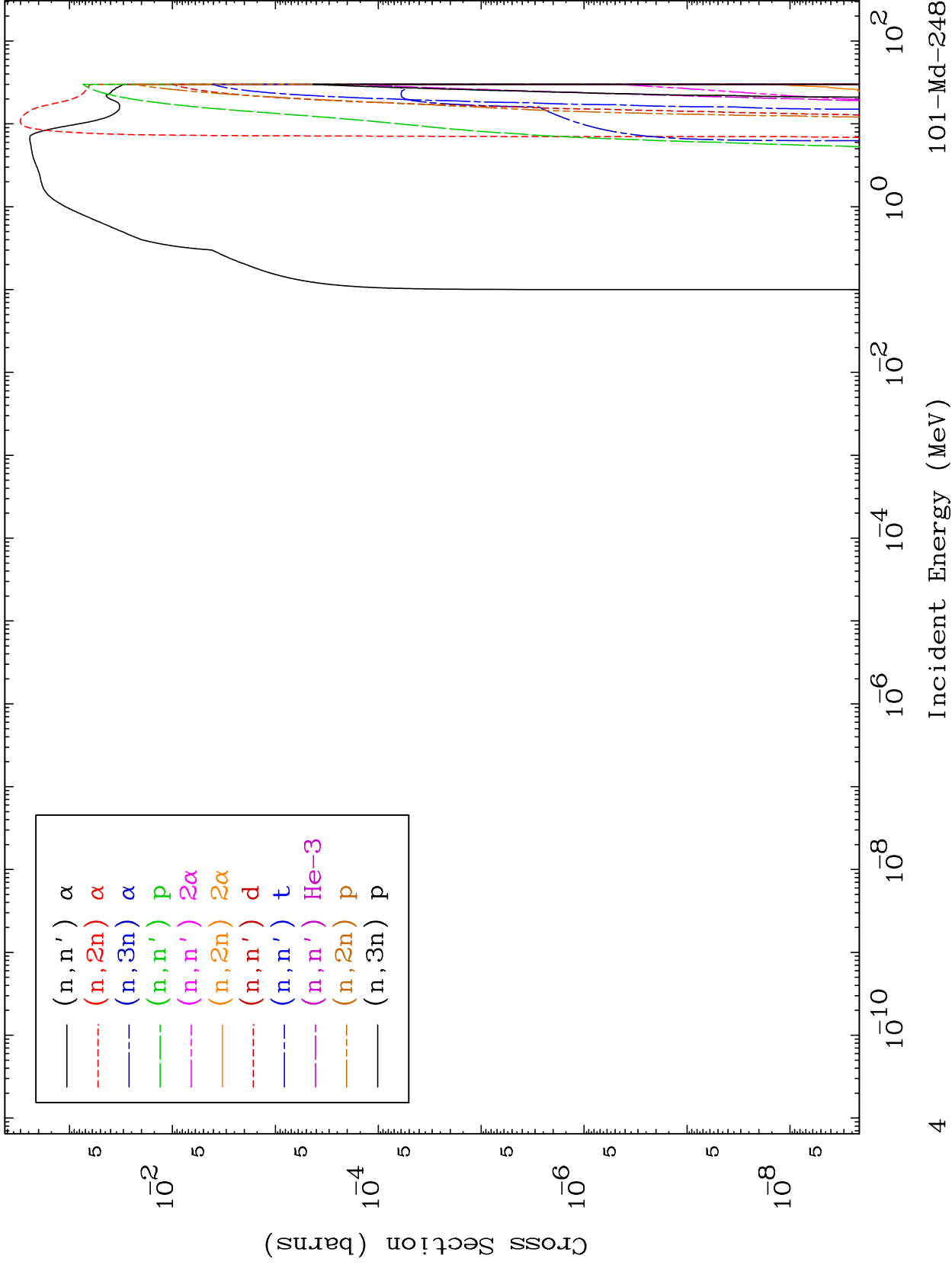
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

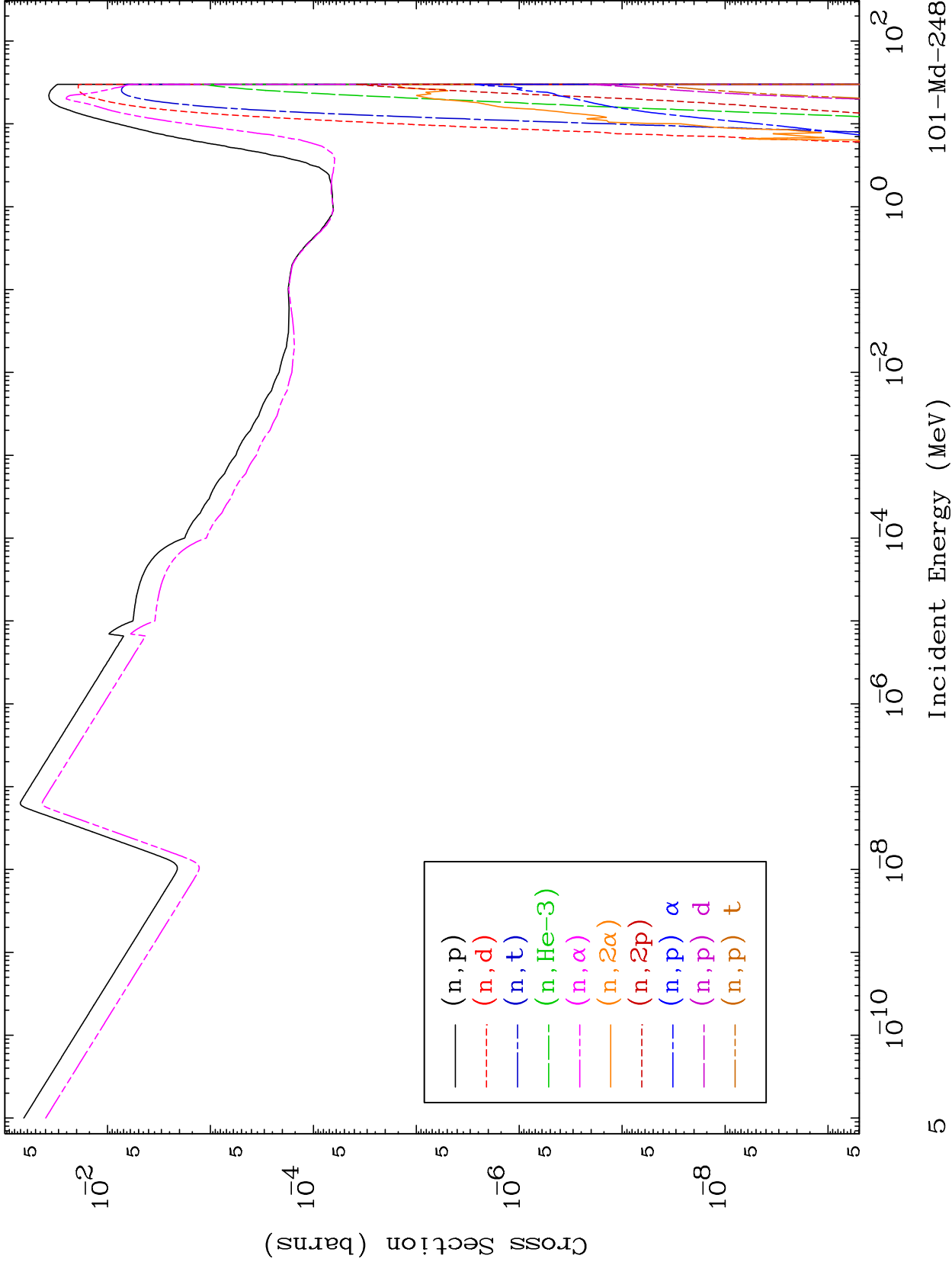
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

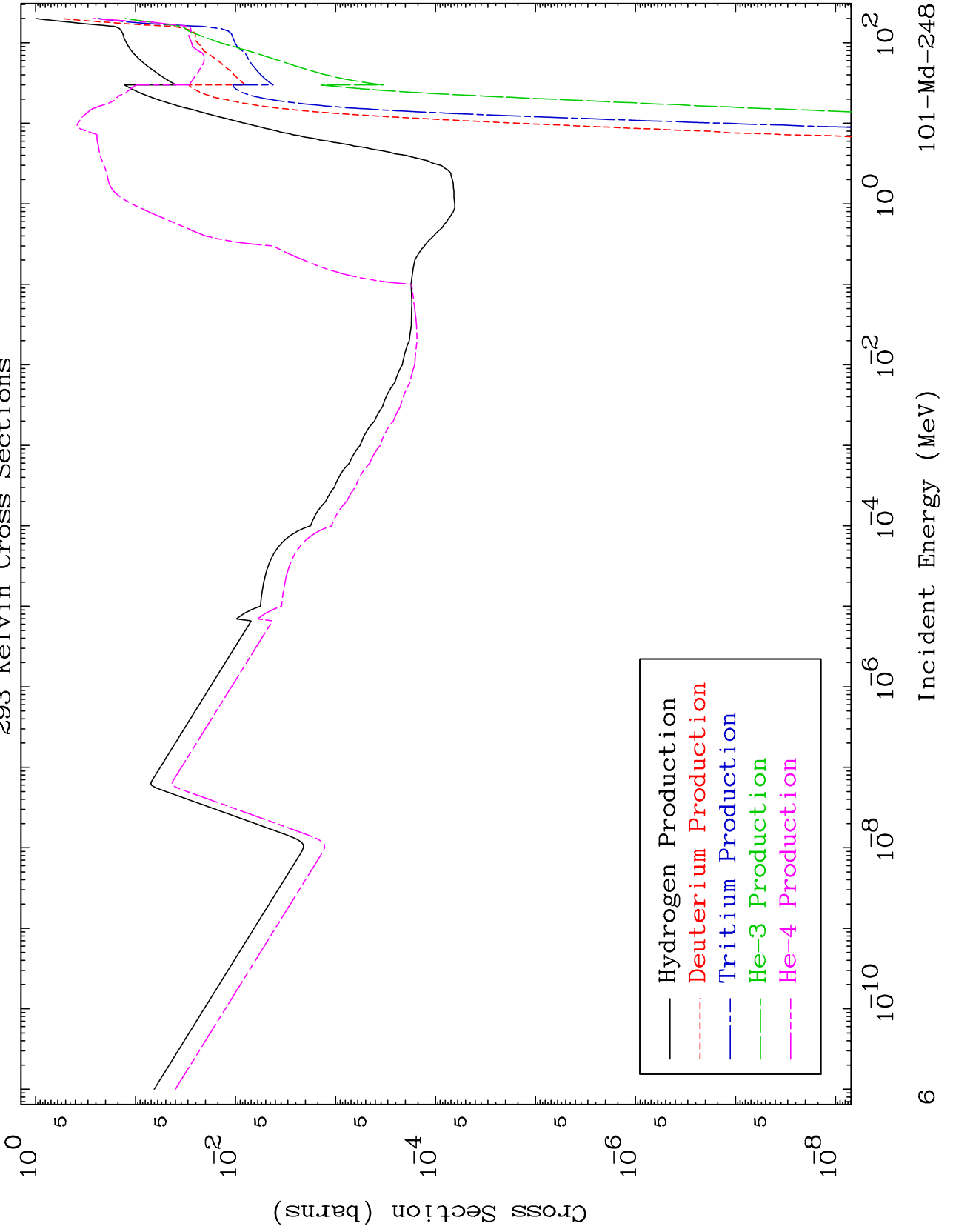
101-Md-248



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Particle Production
293 Kelvin Cross Sections

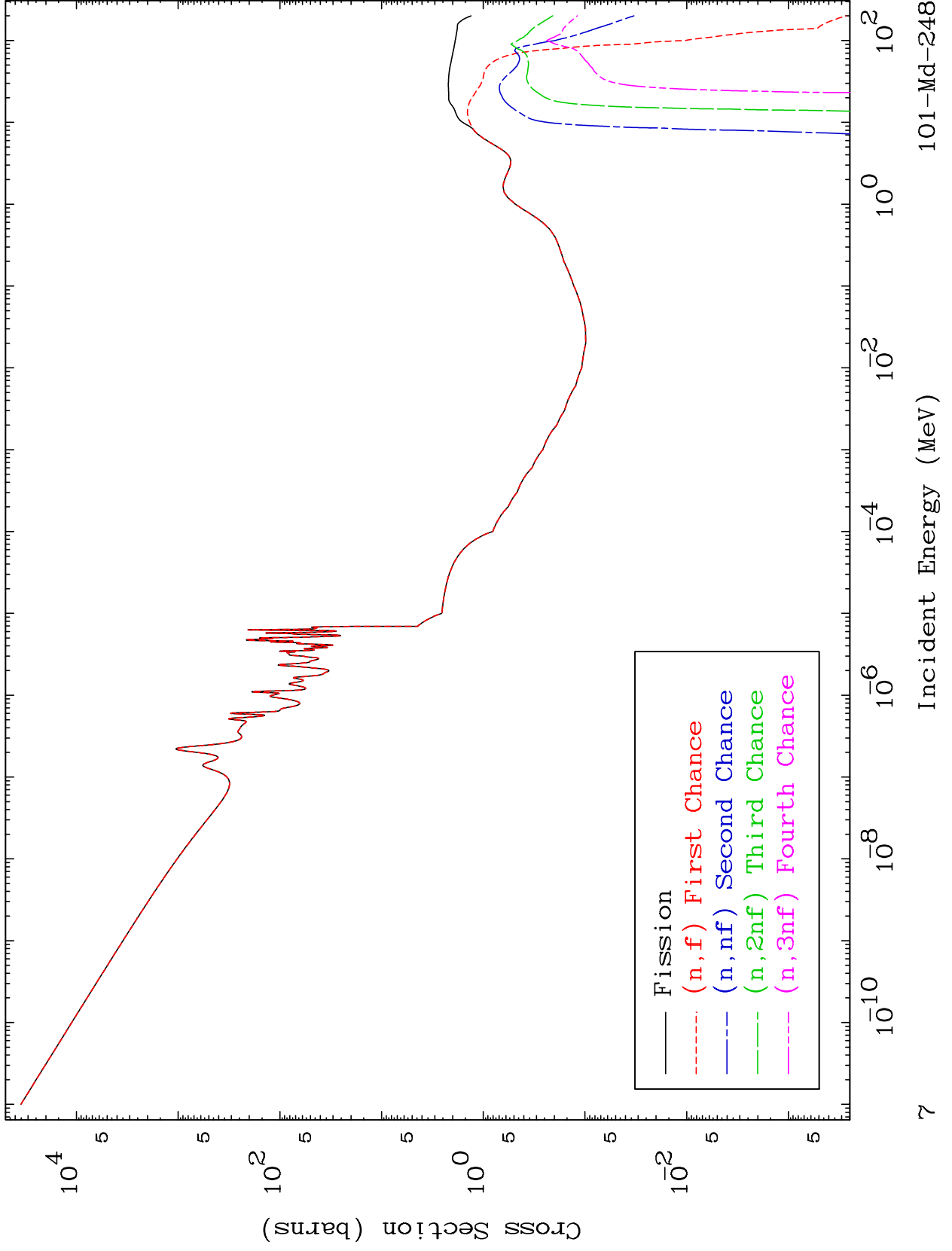
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MAT 9949

Fission
293 Kelvin Cross Sections

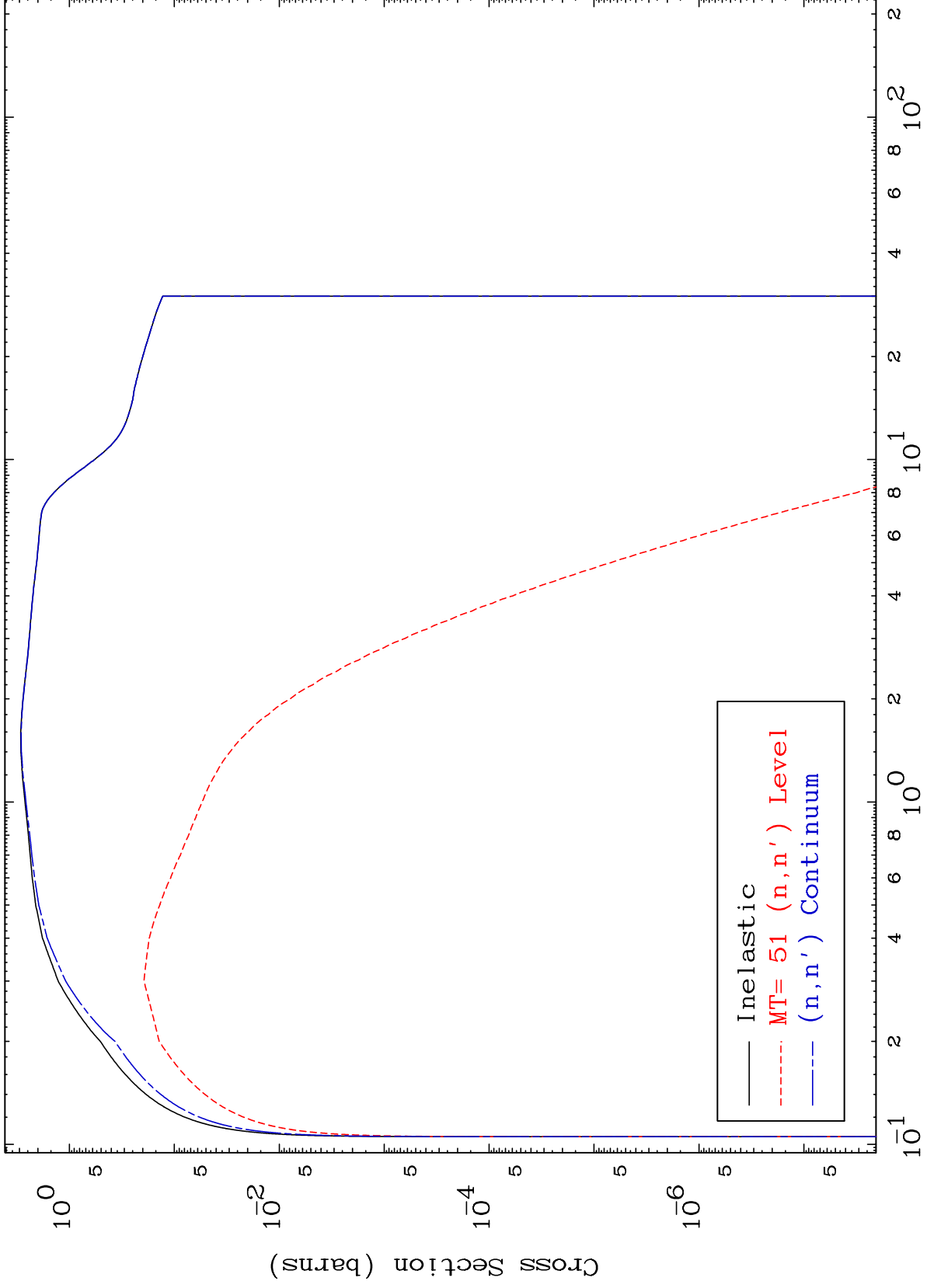
101-Md-248



MAT 9949

(n,n') Level
293 Kelvin Cross Sections

101-Md-248



101-Md-248

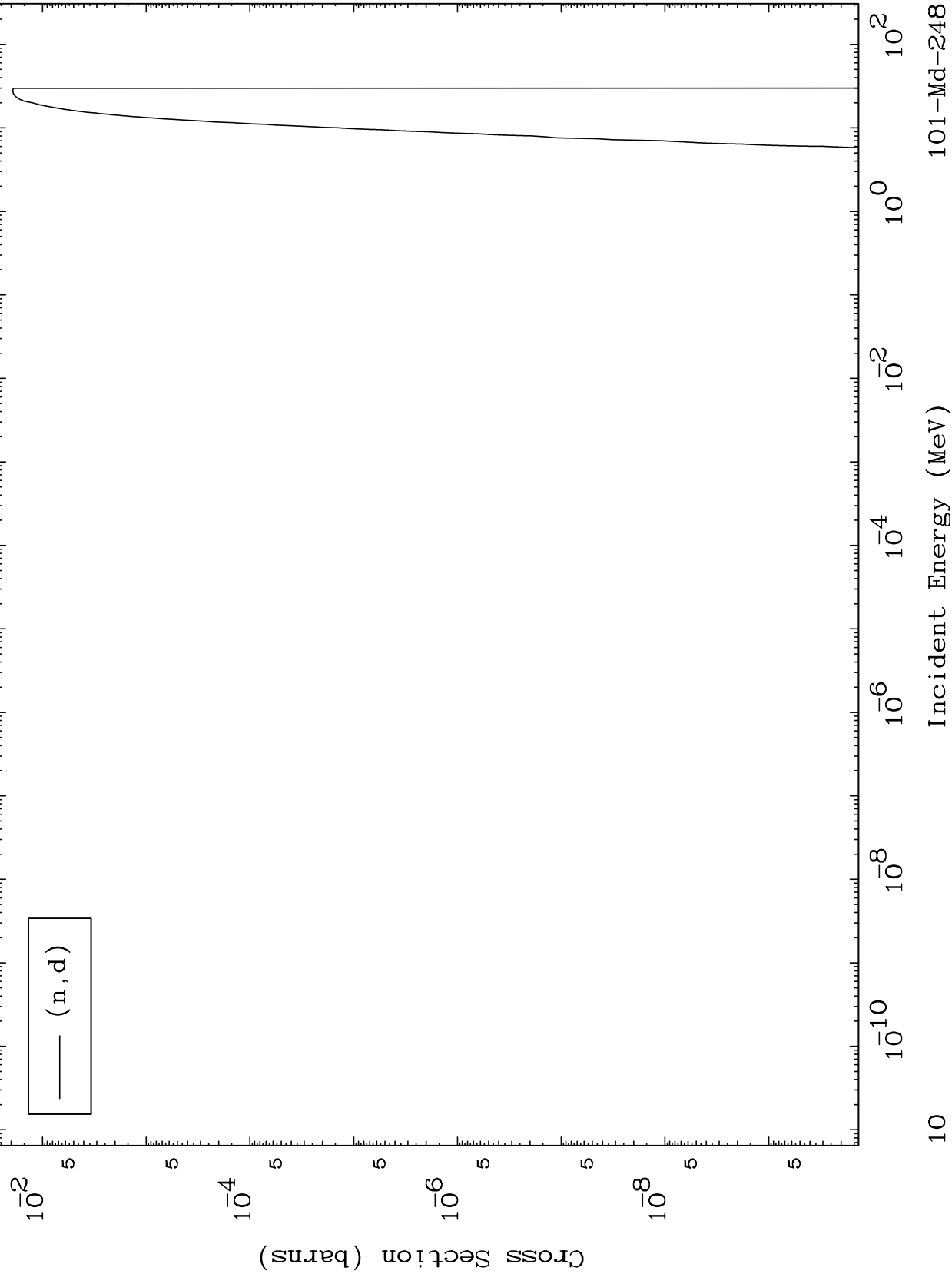
Incident Energy (MeV)

8

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(n,d) Levels
293 Kelvin Cross Sections

101-Md-248



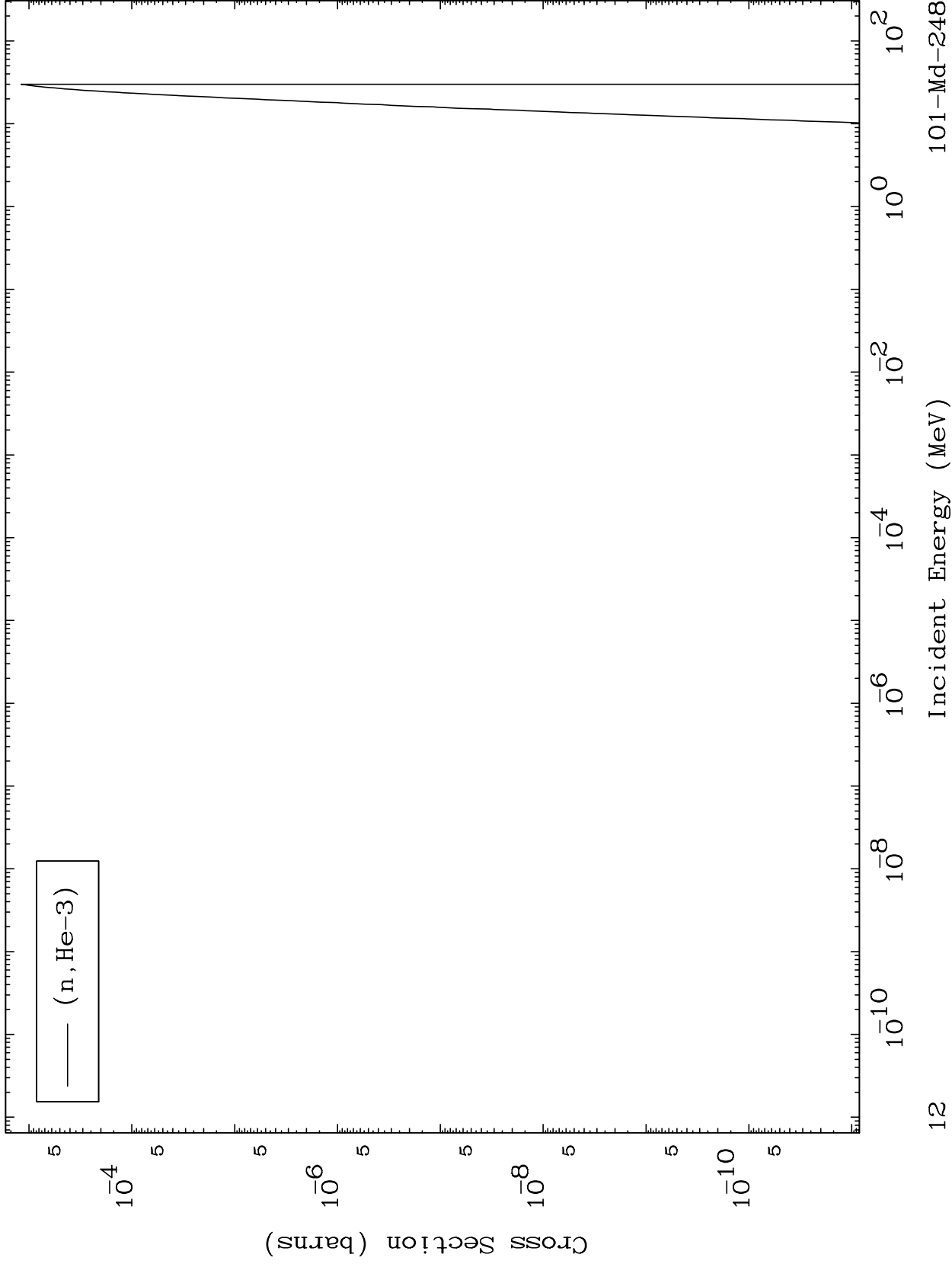
10

101-Md-248

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(n,He3) Levels
293 Kelvin Cross Sections

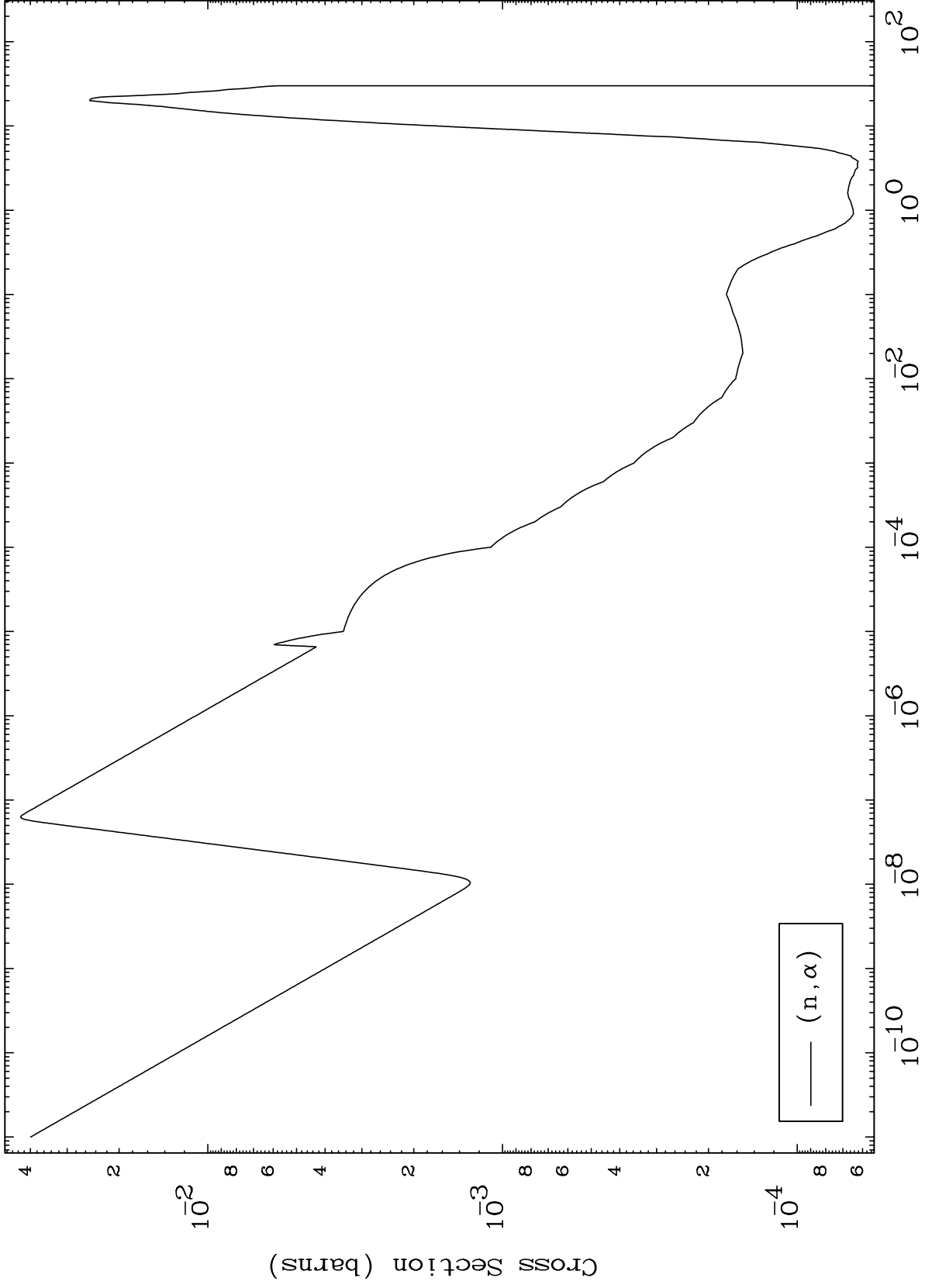
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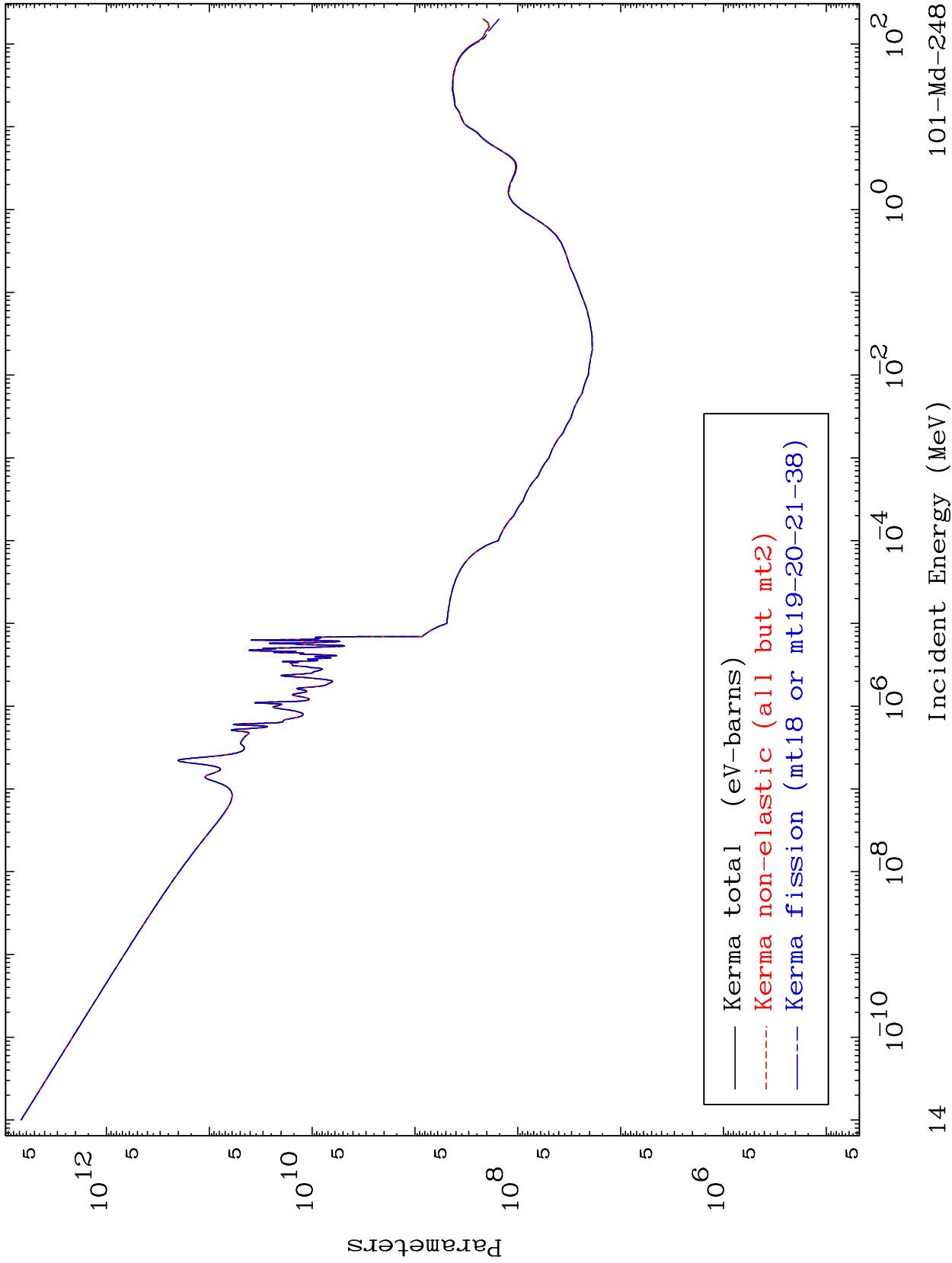


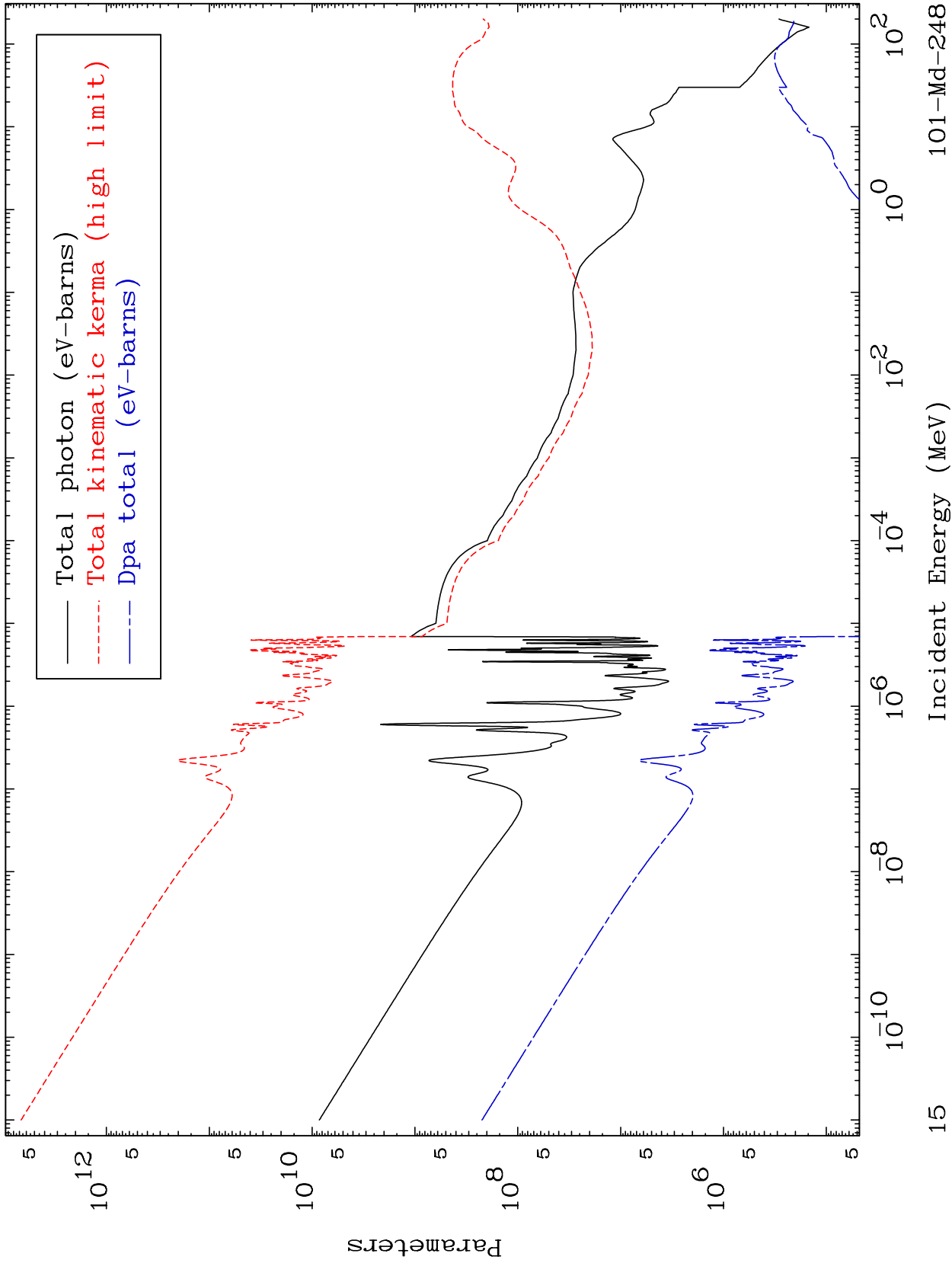
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(n,α) Levels
293 Kelvin Cross Sections

101-Md-248



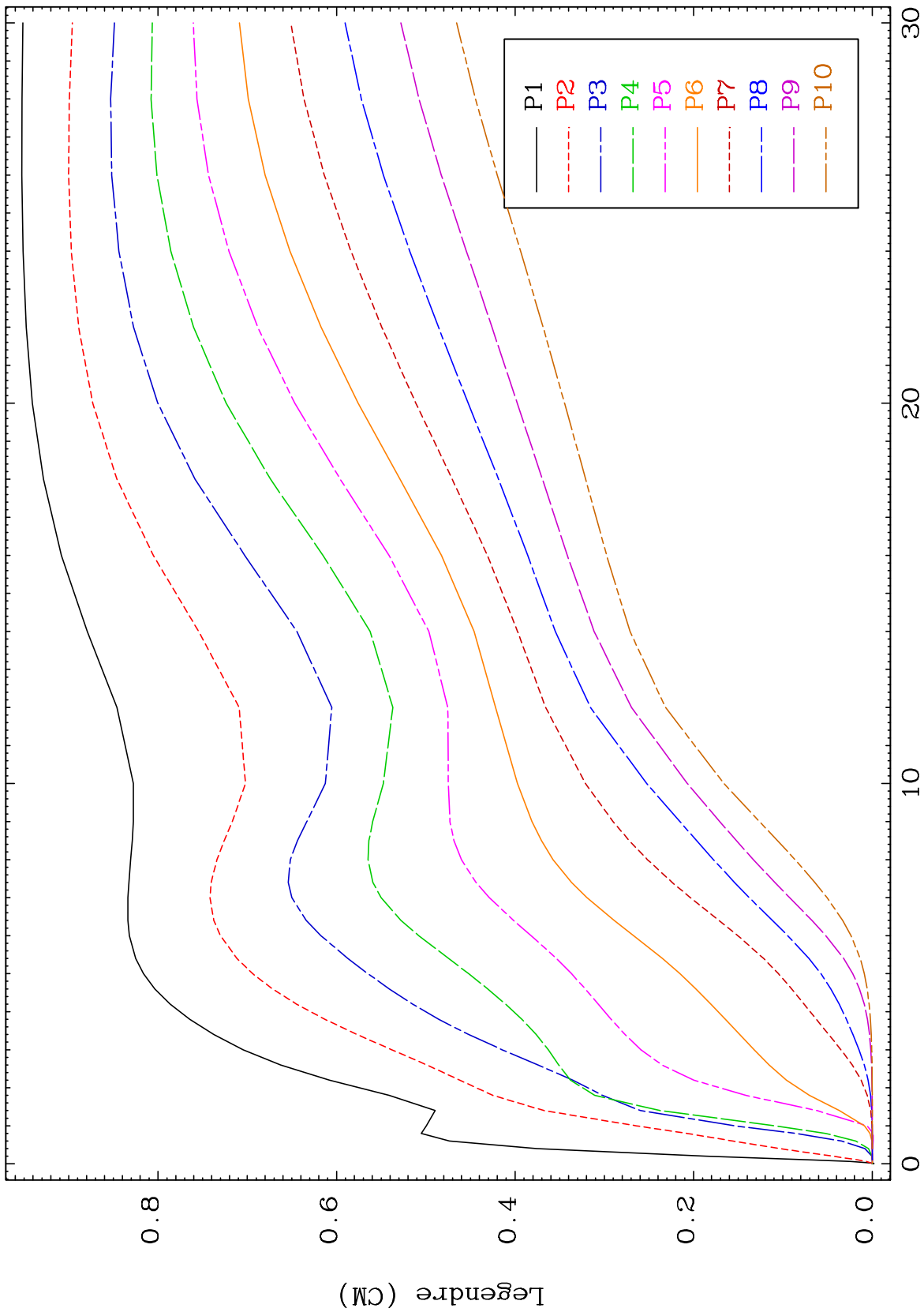




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Elastic Legendre Coefficients

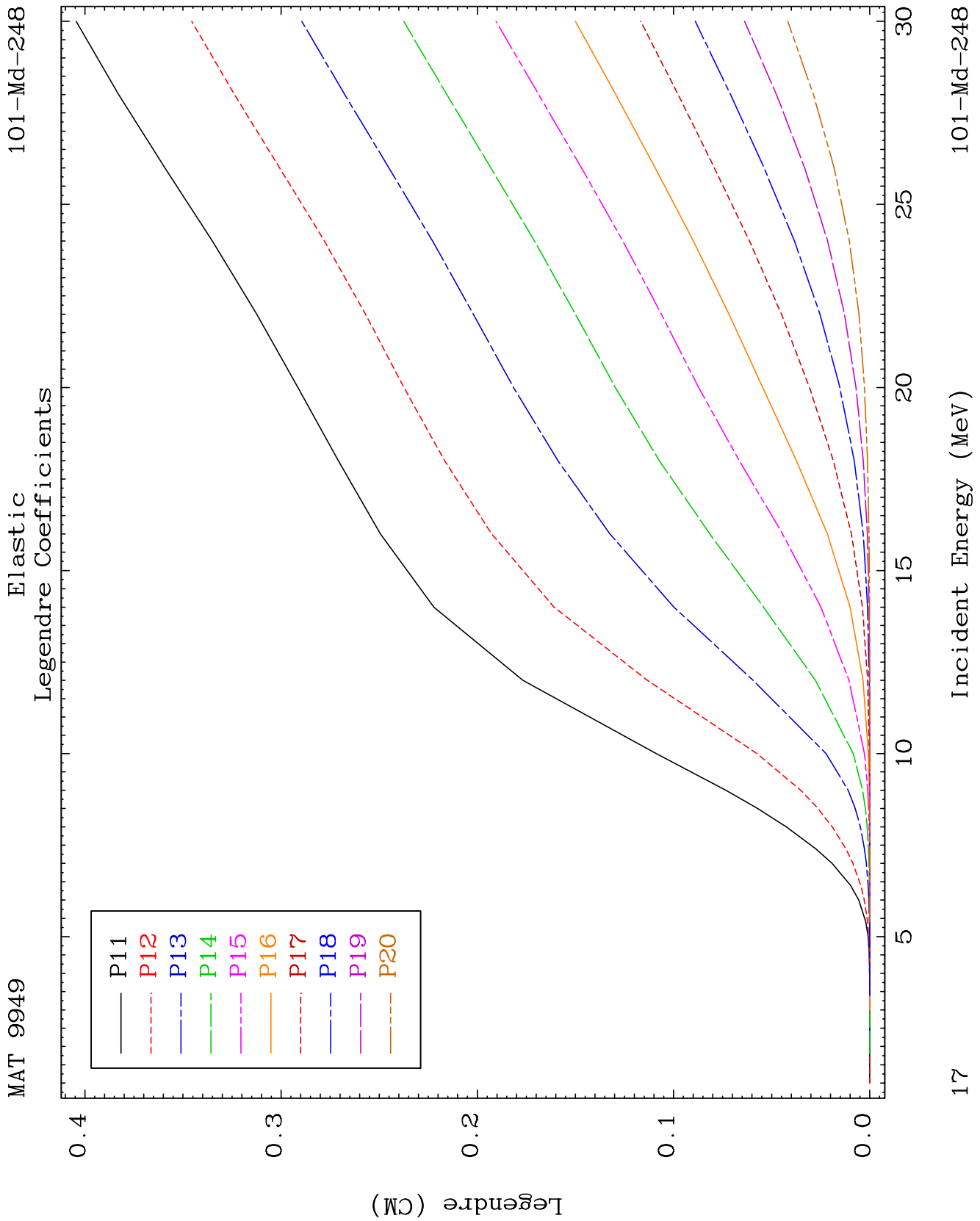
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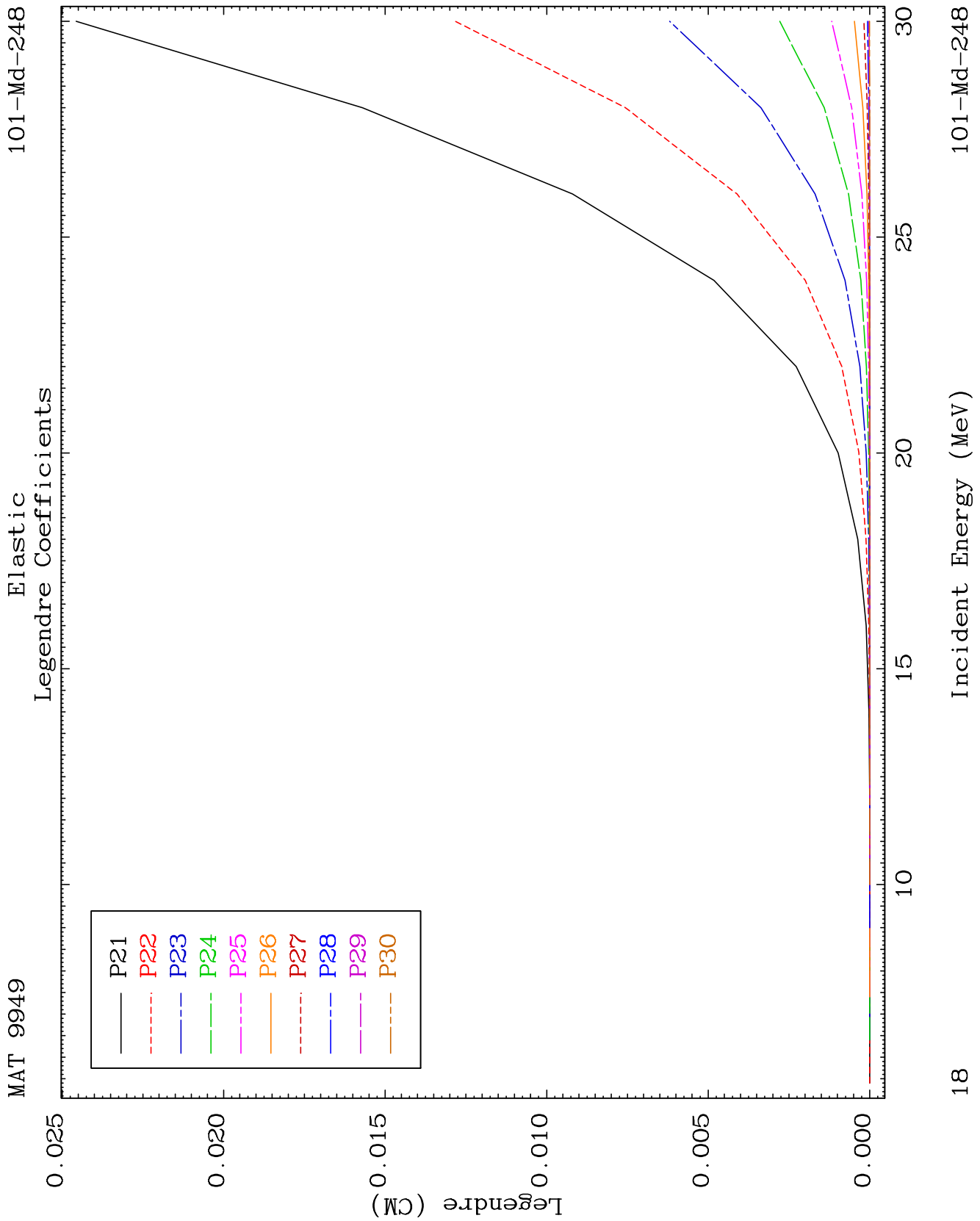


16

Incident Energy (MeV)

101-Md-248





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Elastic Legendre Coefficients

101-Md-248

18

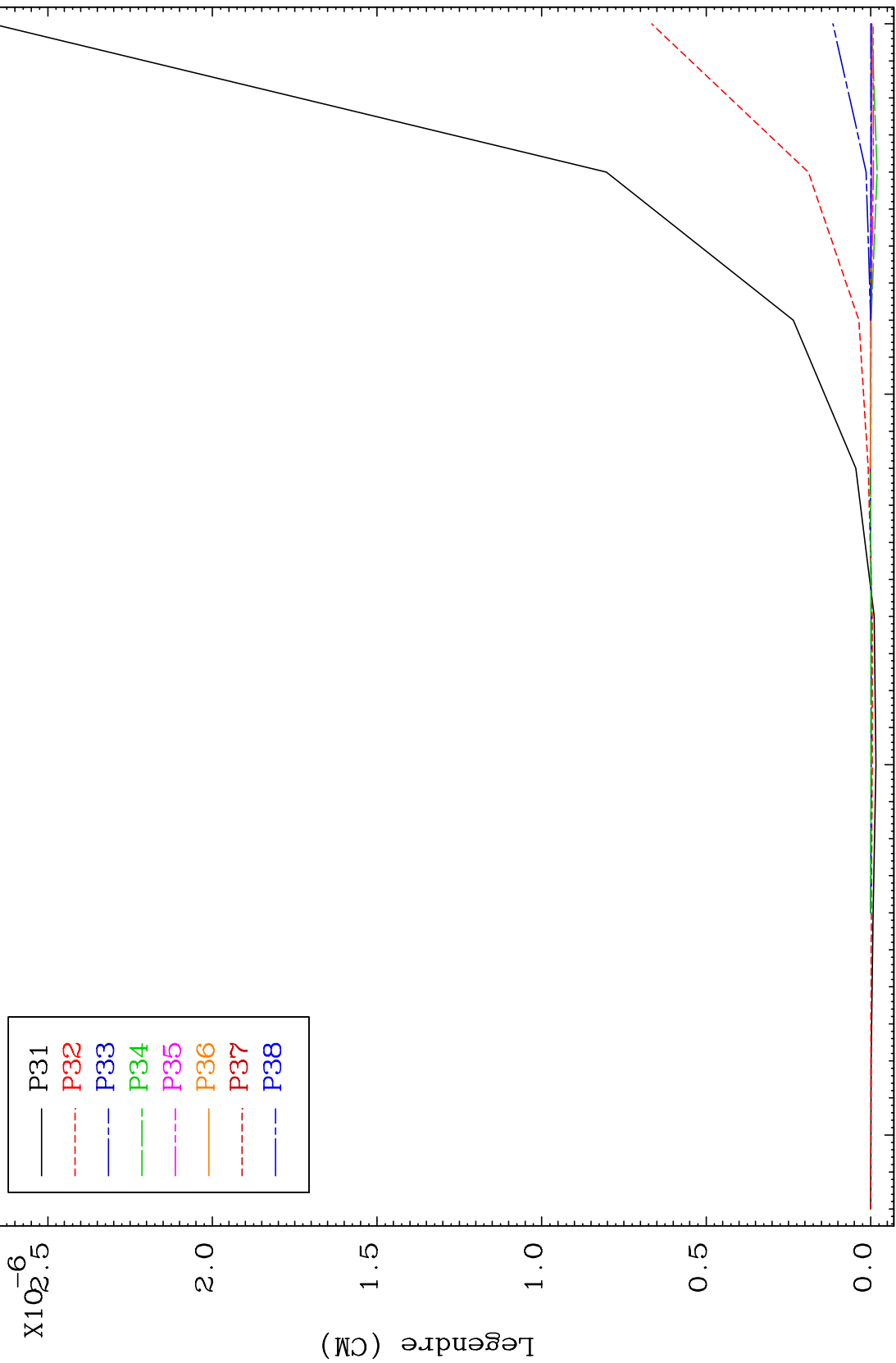
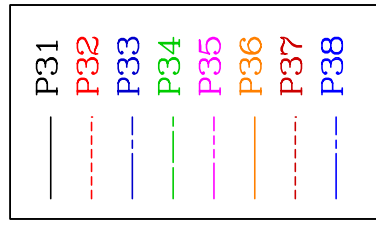
Incident Energy (MeV)

101-Md-248

MAT 9949

Elastic
Legendre Coefficients

101-Md-248



19

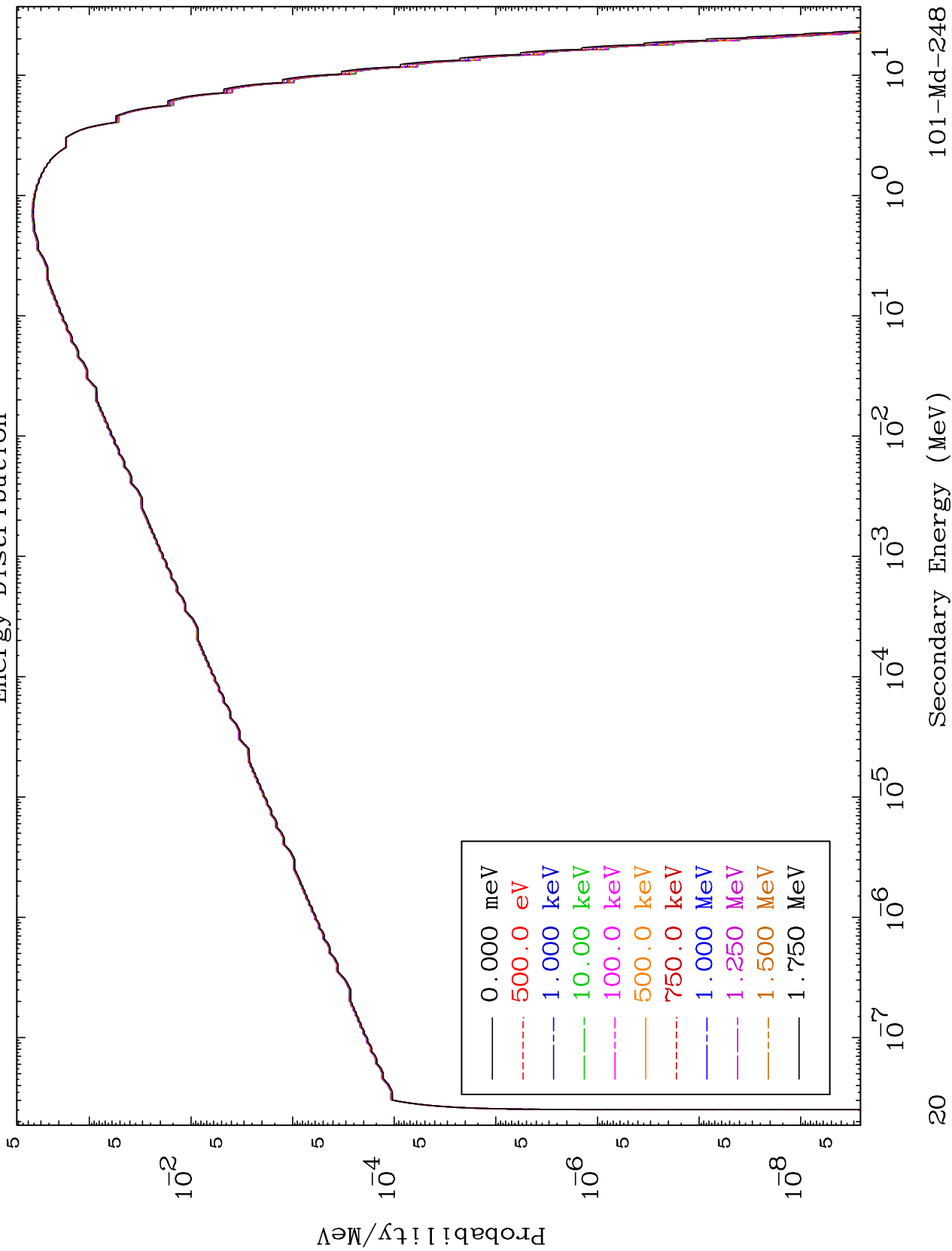
Incident Energy (MeV)

101-Md-248

MAT 9949

Fission
Energy Distribution

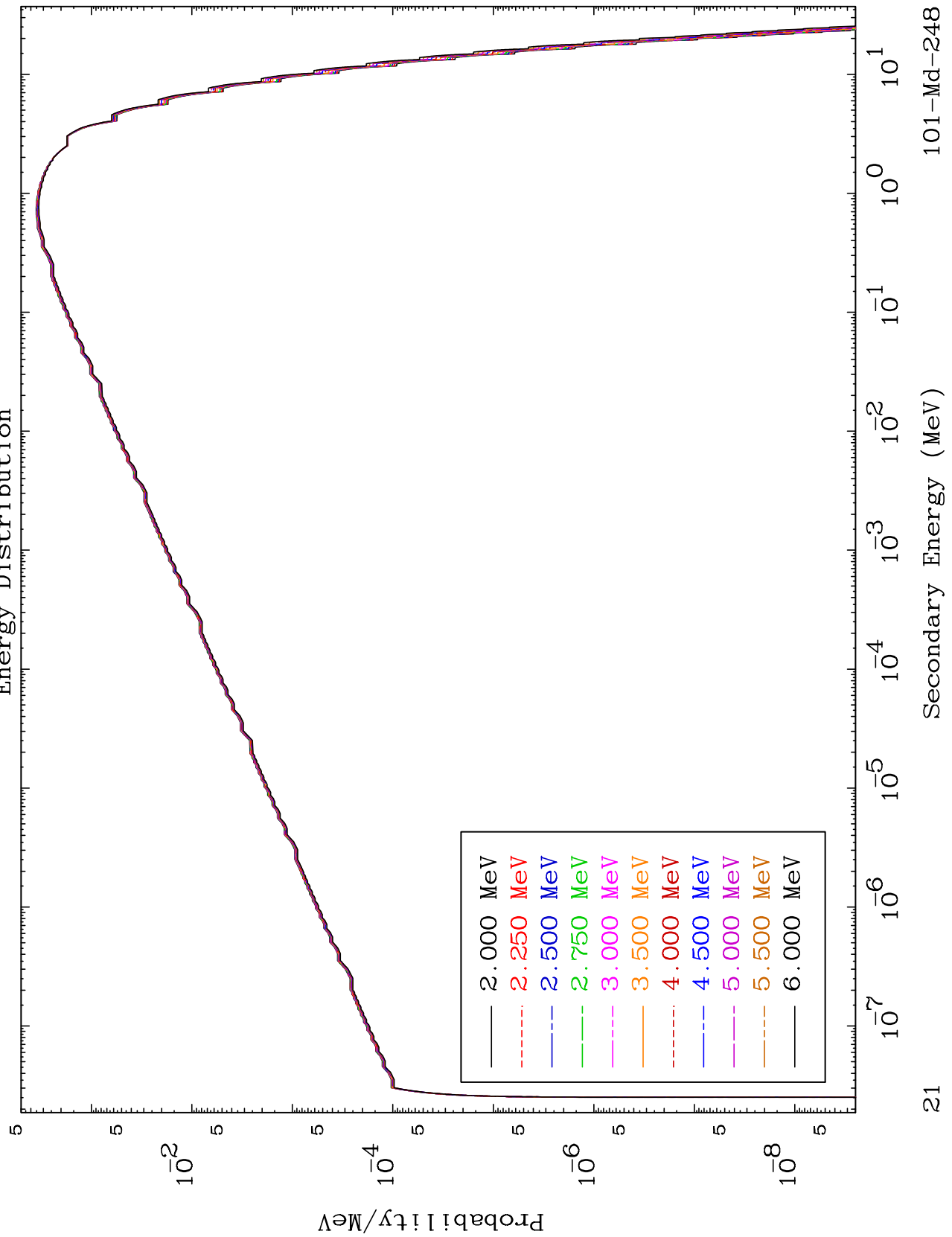
101-Md-248



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Fission
Energy Distribution

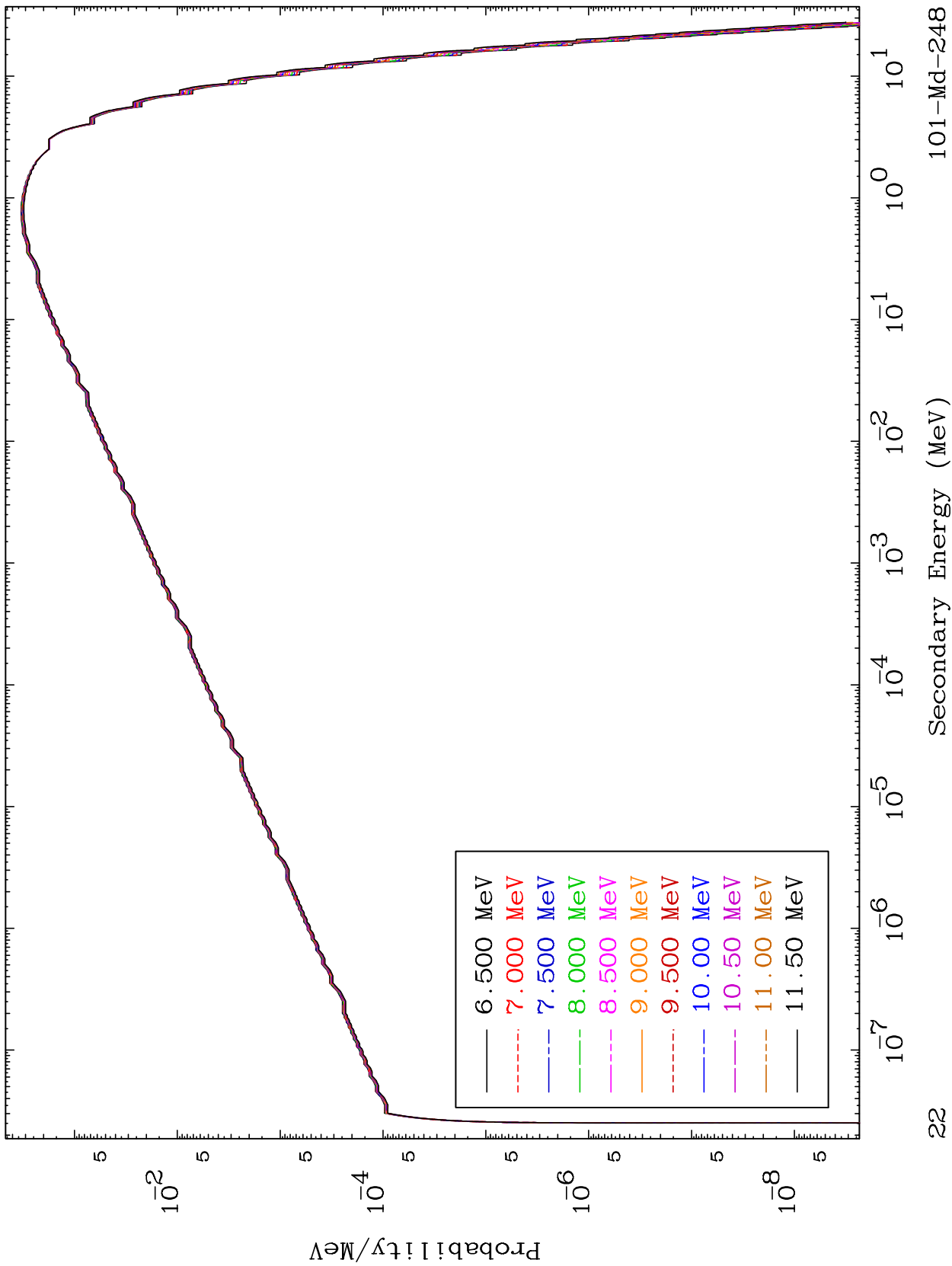
101-Md-248



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Fission Energy Distribution

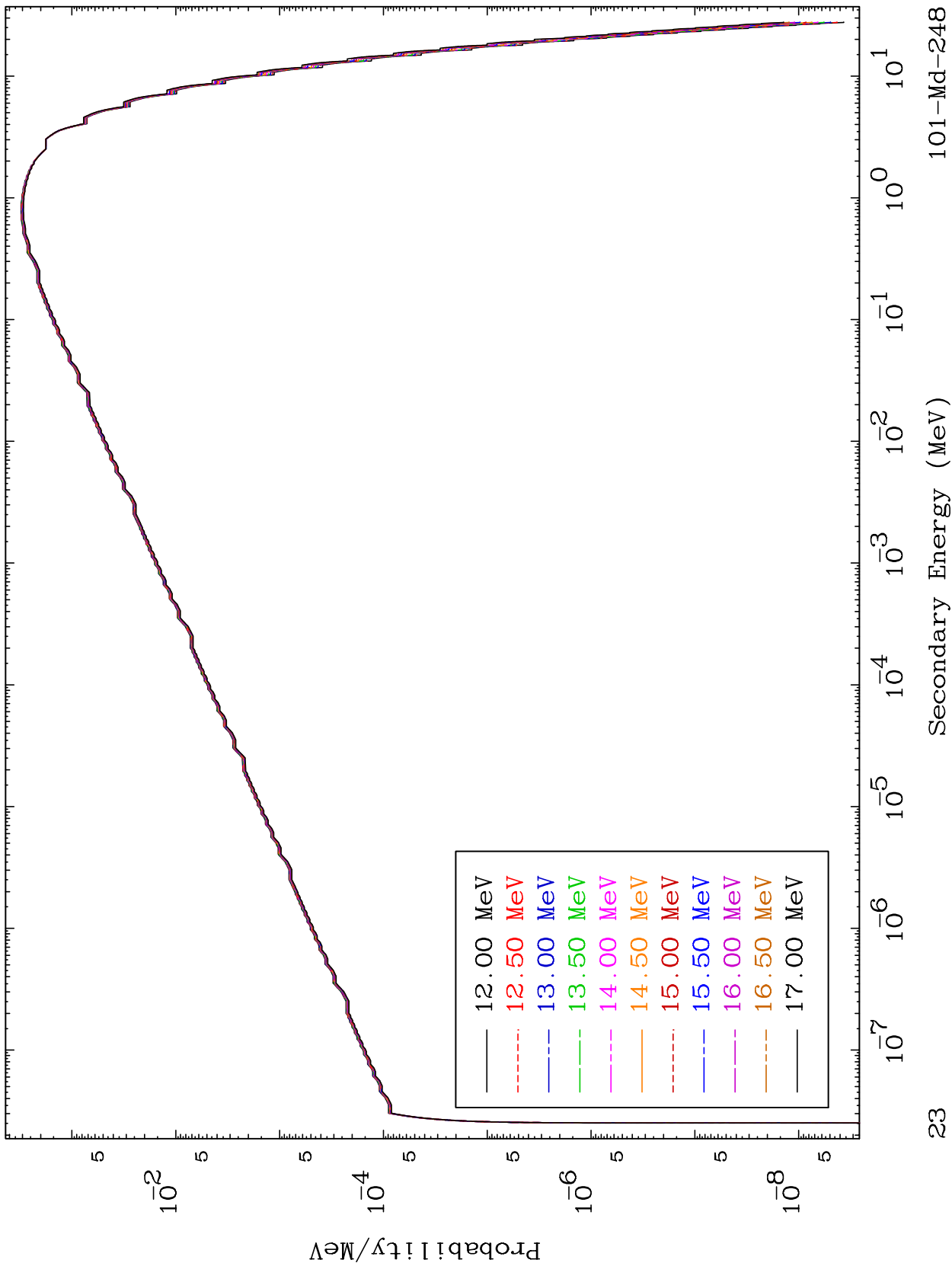
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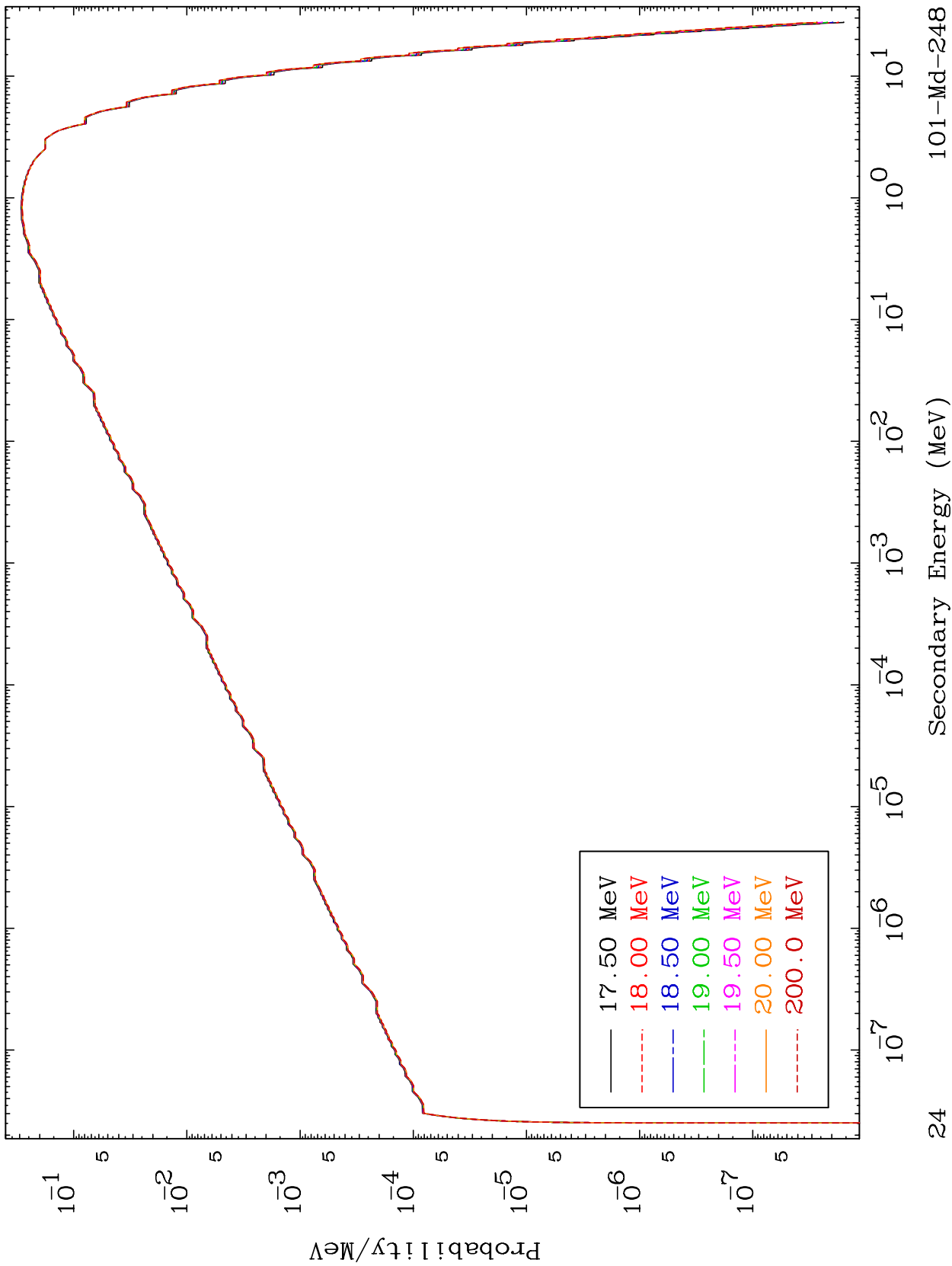
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Fission Energy Distribution

101-Md-248



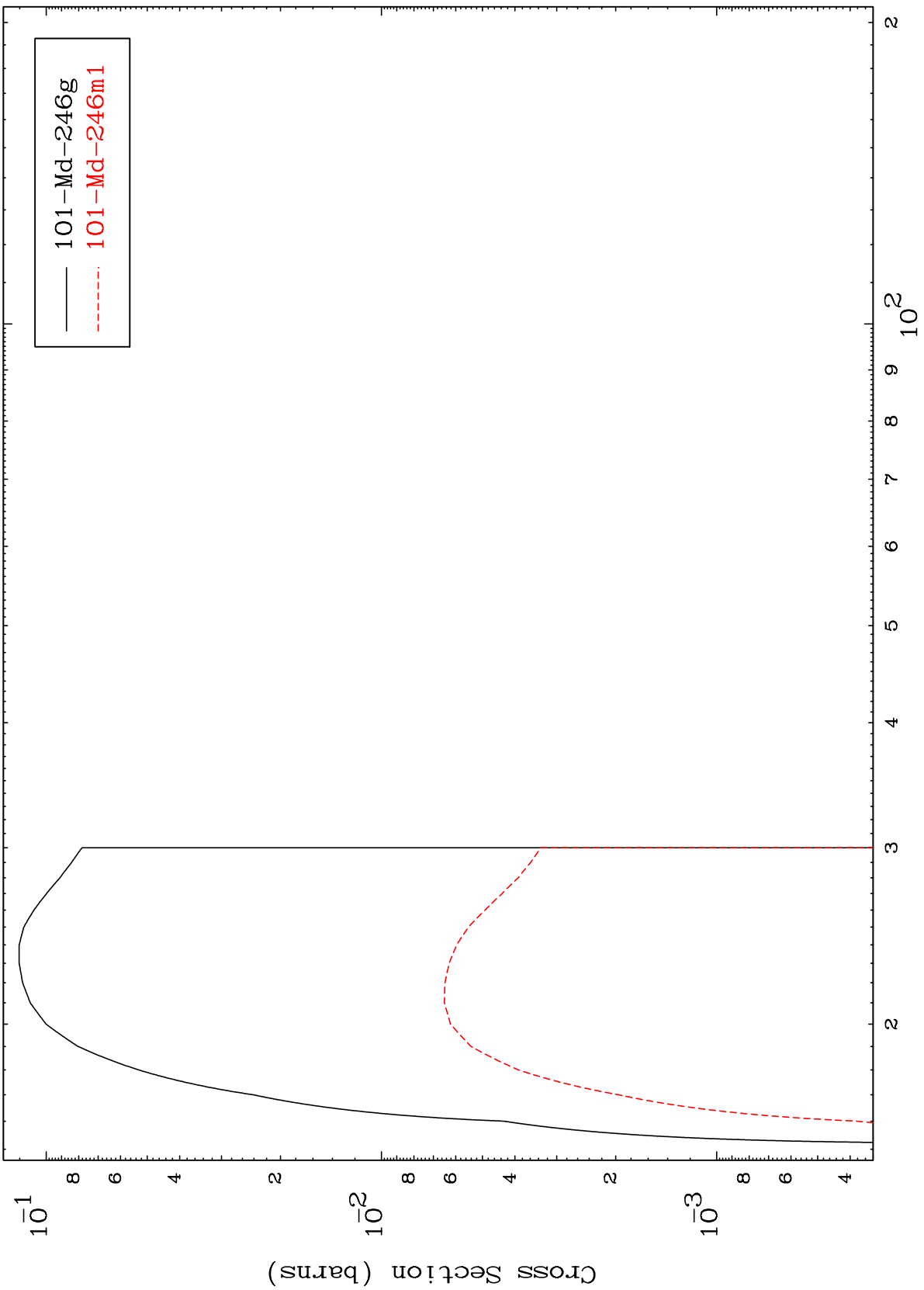
Fission
Energy Distribution



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101-Md-248

(n,3n)
Radionuclide Production Cross Section



25

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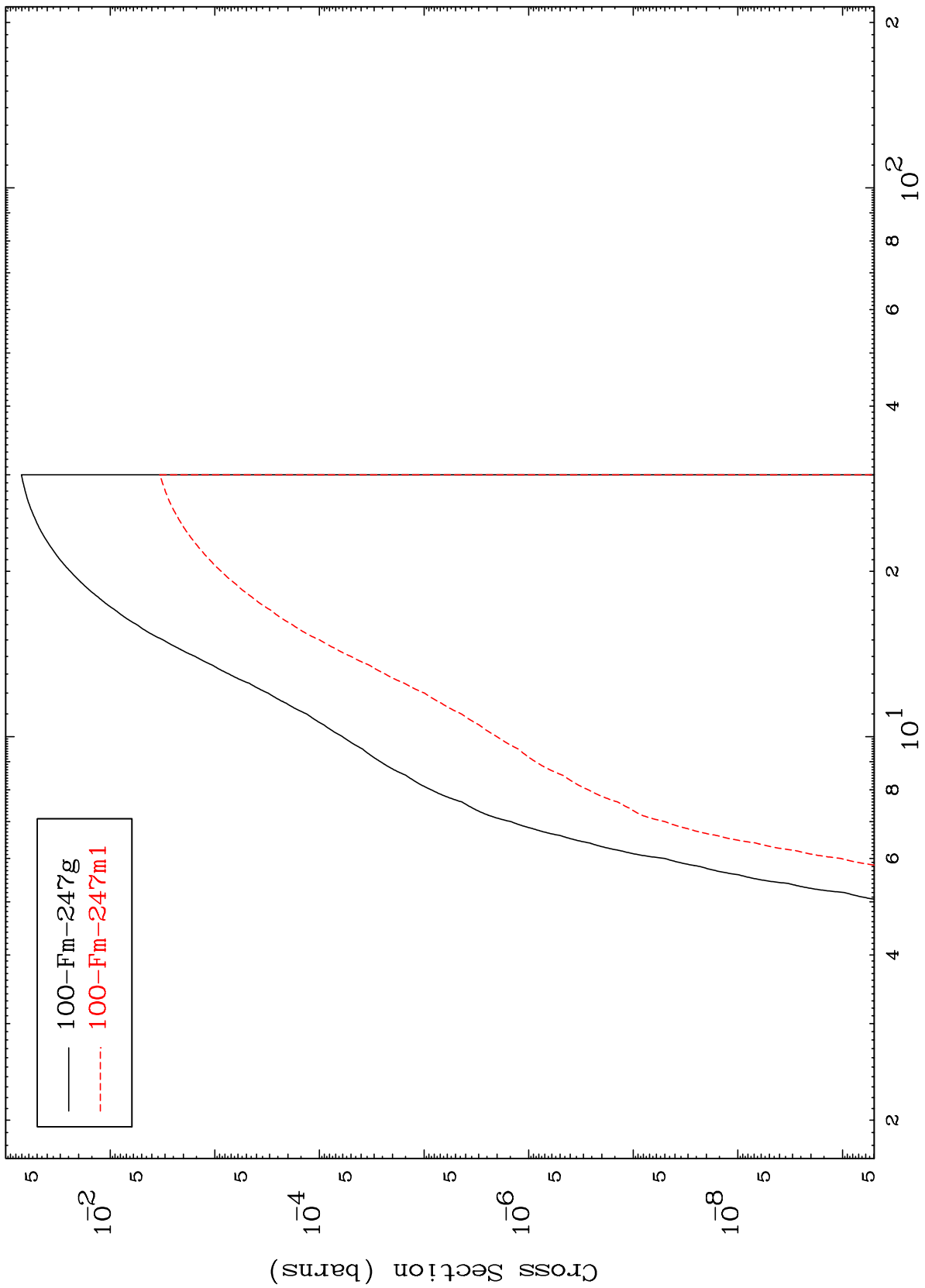
Incident Energy (MeV)

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(n,n') p

101-Md-248

Radionuclide Production Cross Section



— 100-Fm-247g
- - - 100-Fm-247m1

26

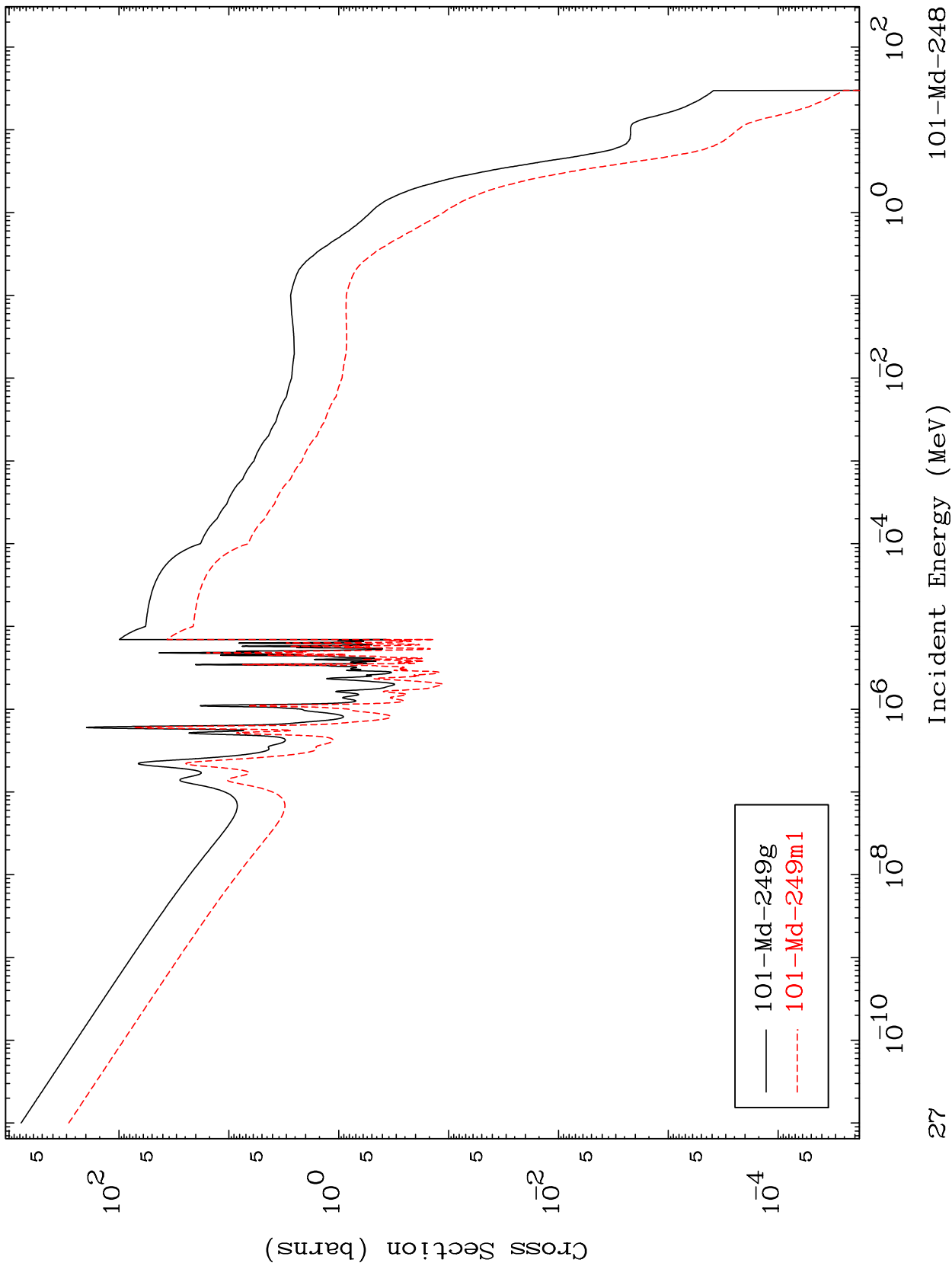
Incident Energy (MeV)

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101-Md-248

Radionuclide Production Cross Section
(n, γ)



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(n,d)

101-Md-248

Radionuclide Production Cross Section

