

Program EVALPLOT
(Version 2017-1)

by

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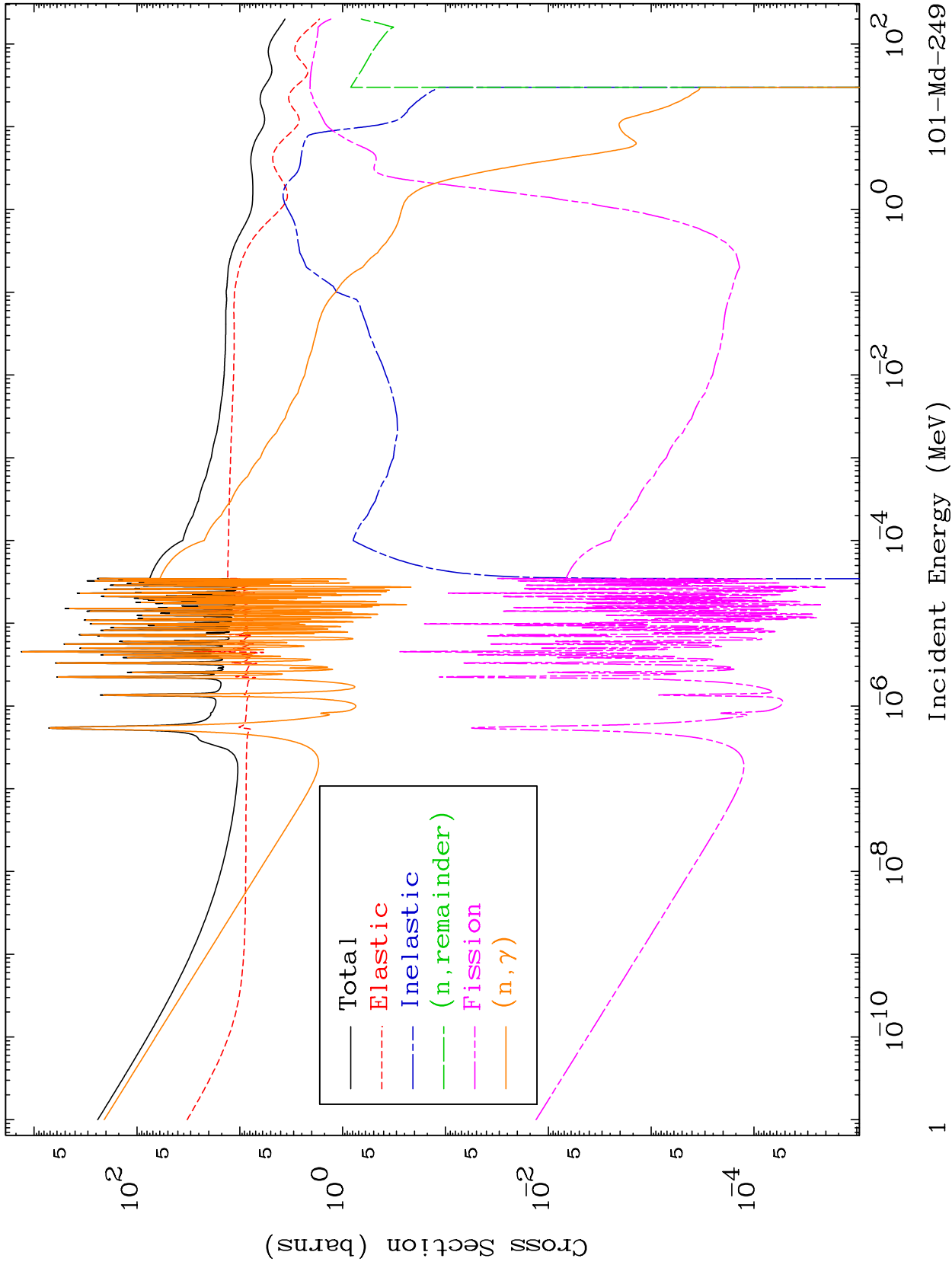
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Press Mouse Button to Start

MAT 9955

Major
293 Kelvin Cross Sections

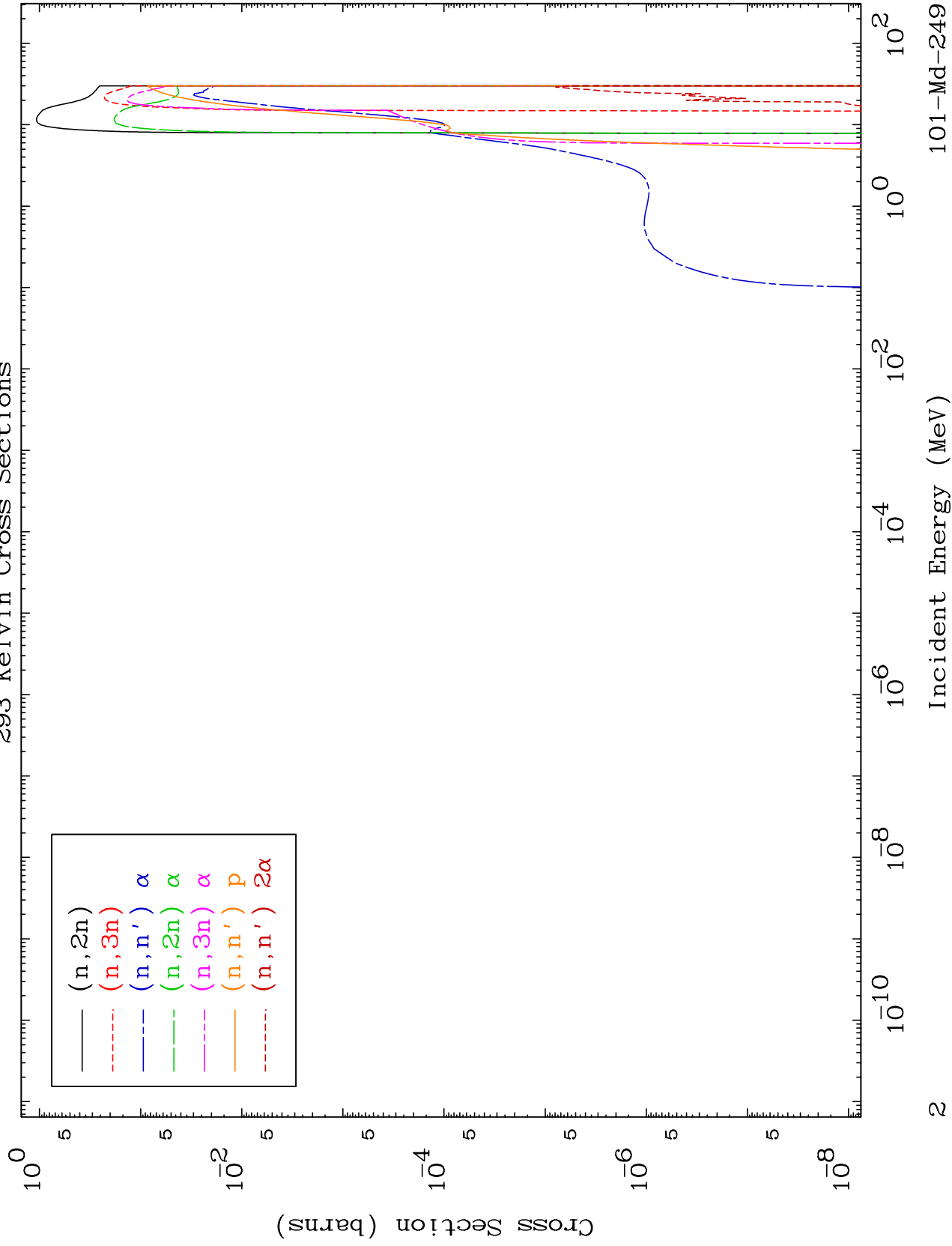
101-Md-249



MAT 9955

Neutron Production
293 Kelvin Cross Sections

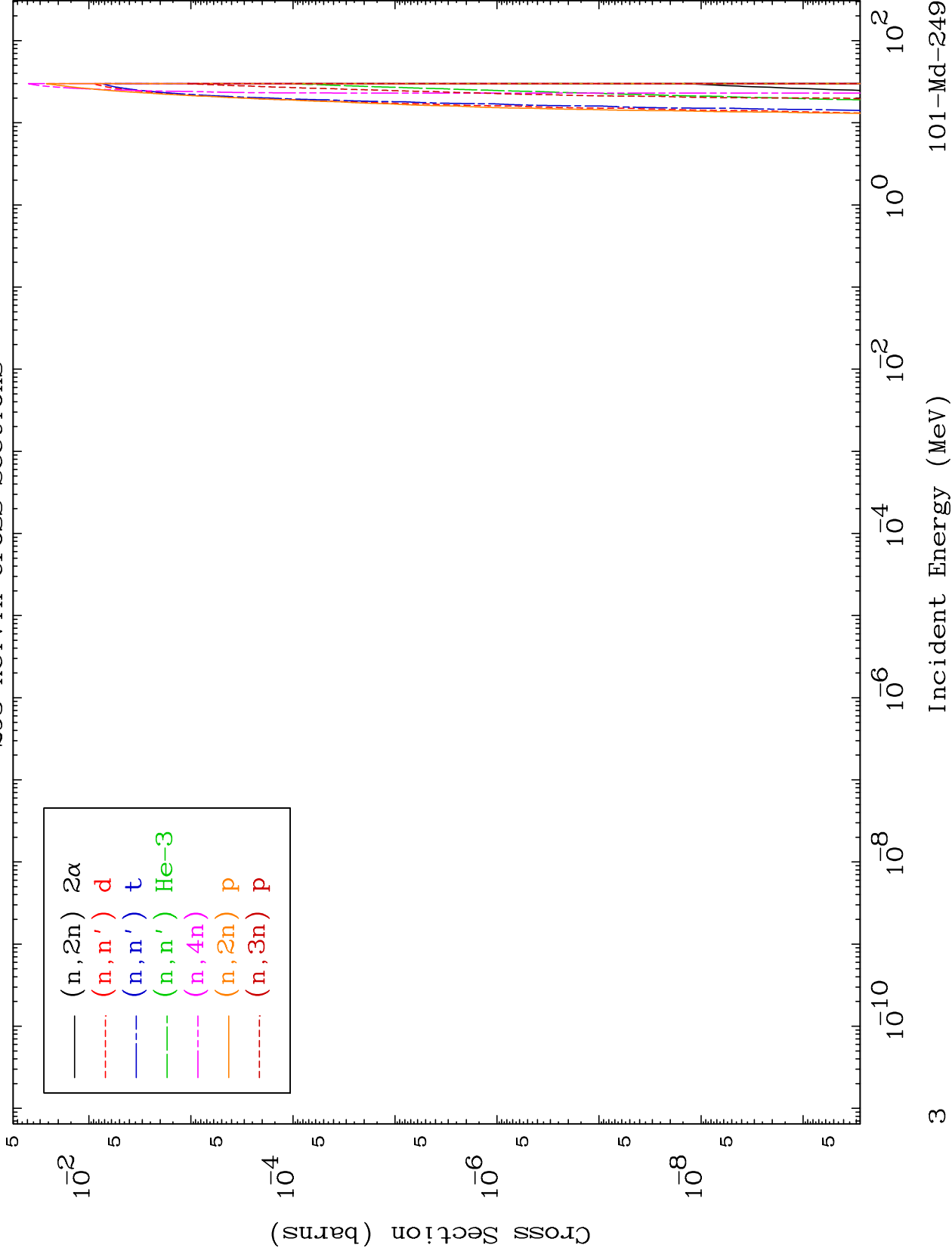
101-Md-249



MAT 9955

Neutron Production
293 Kelvin Cross Sections

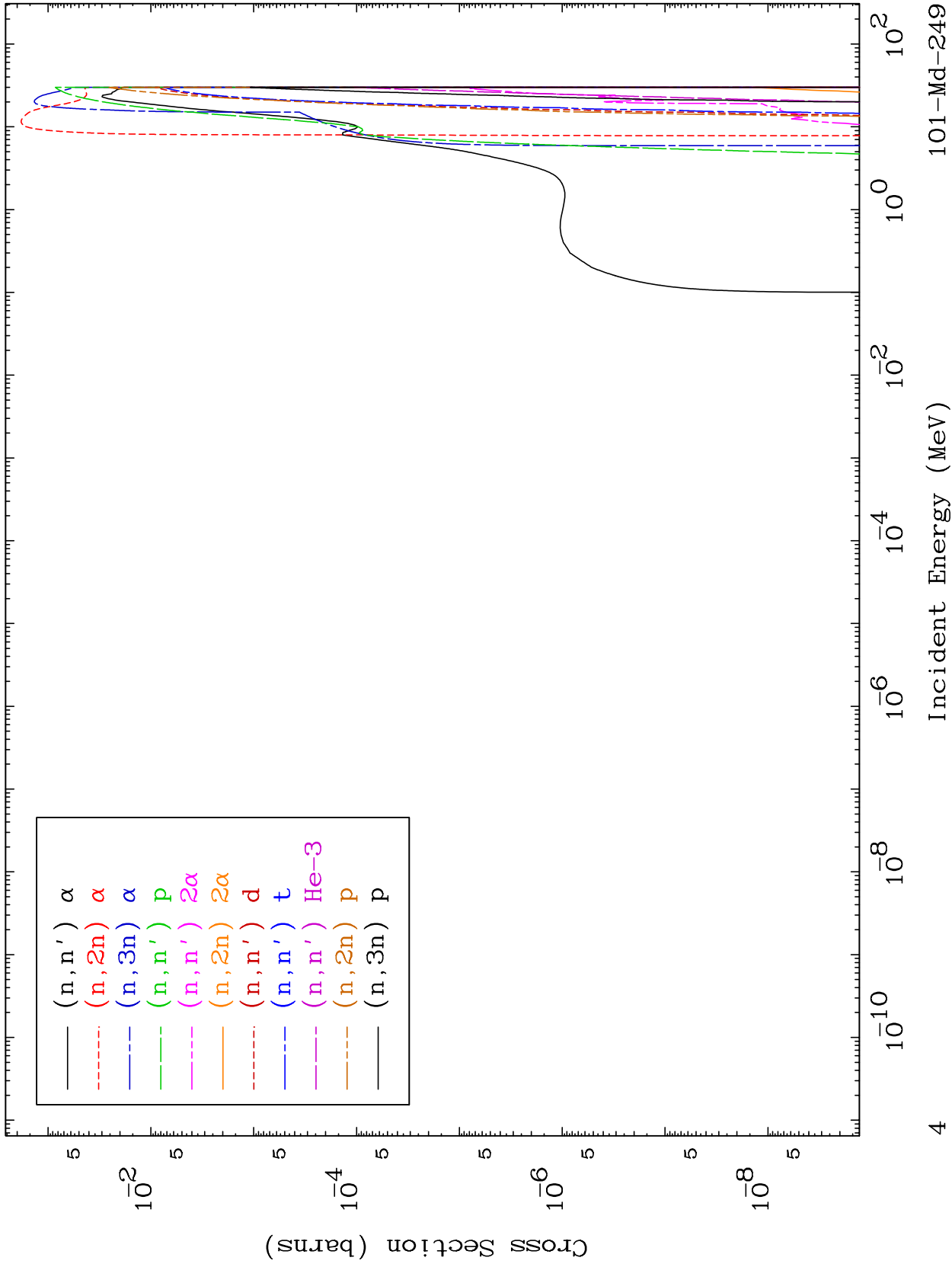
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MAT 9955

Charged Particle
293 Kelvin Cross Sections

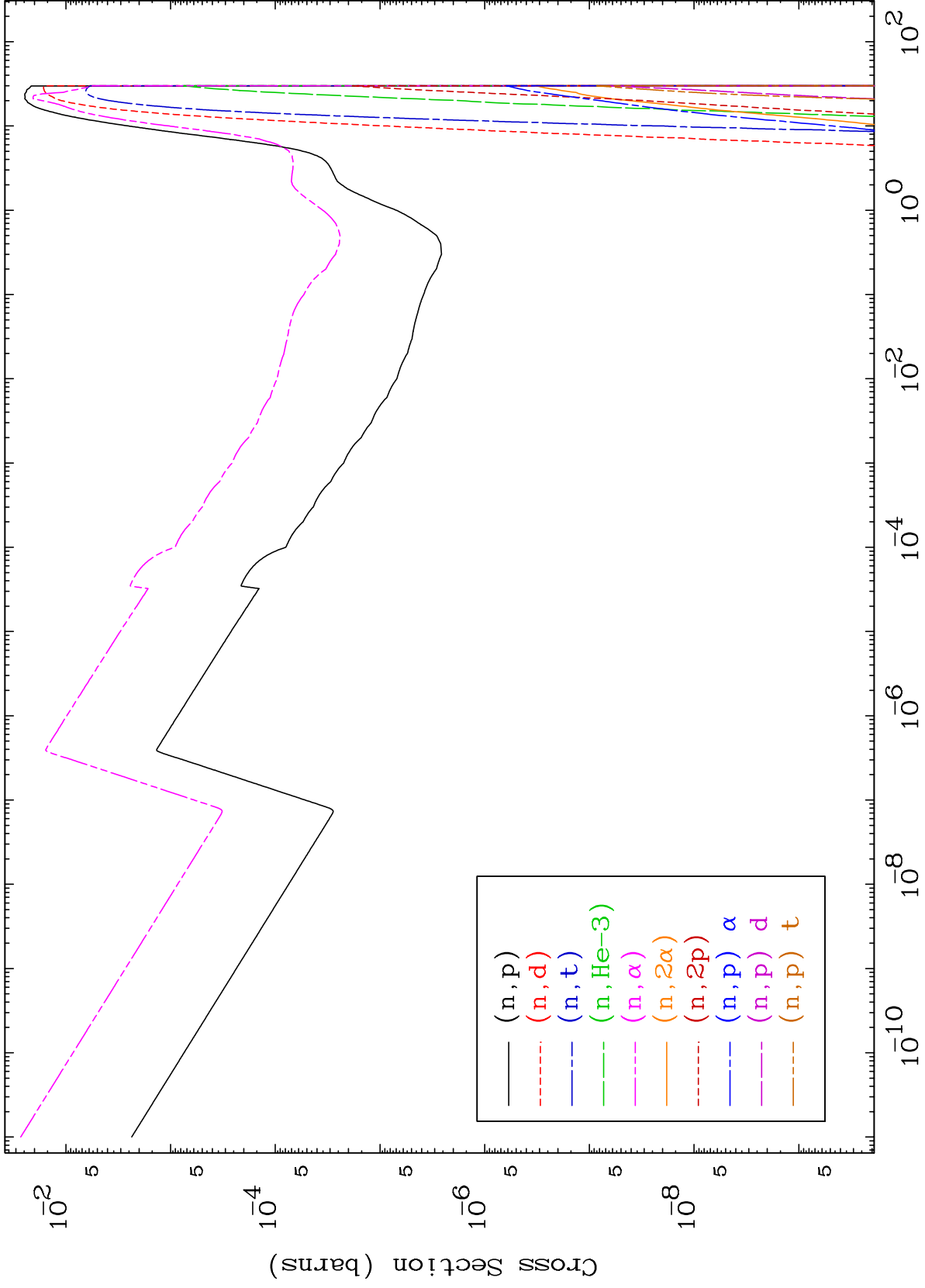
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Charged Particle
293 Kelvin Cross Sections

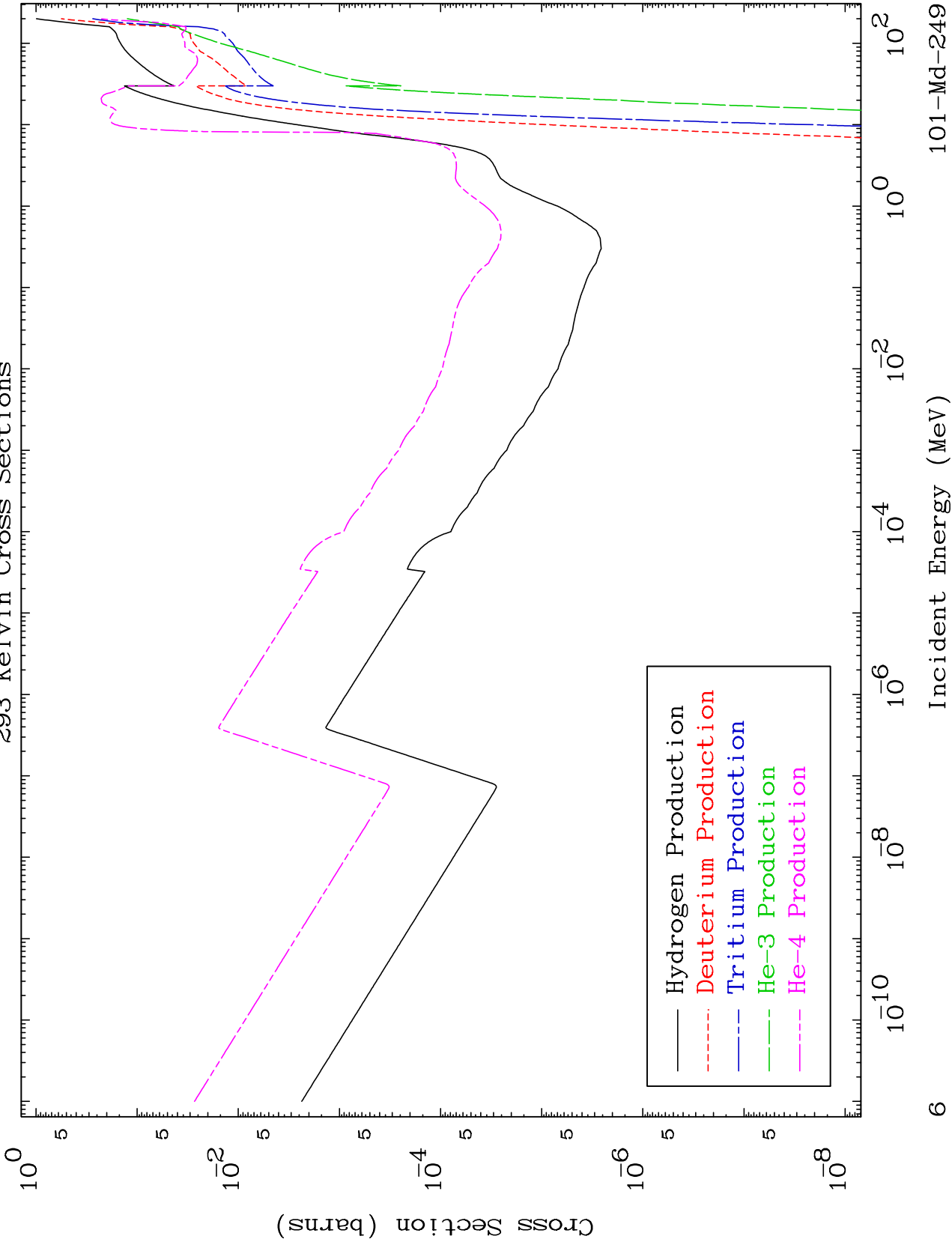
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Particle Production
293 Kelvin Cross Sections

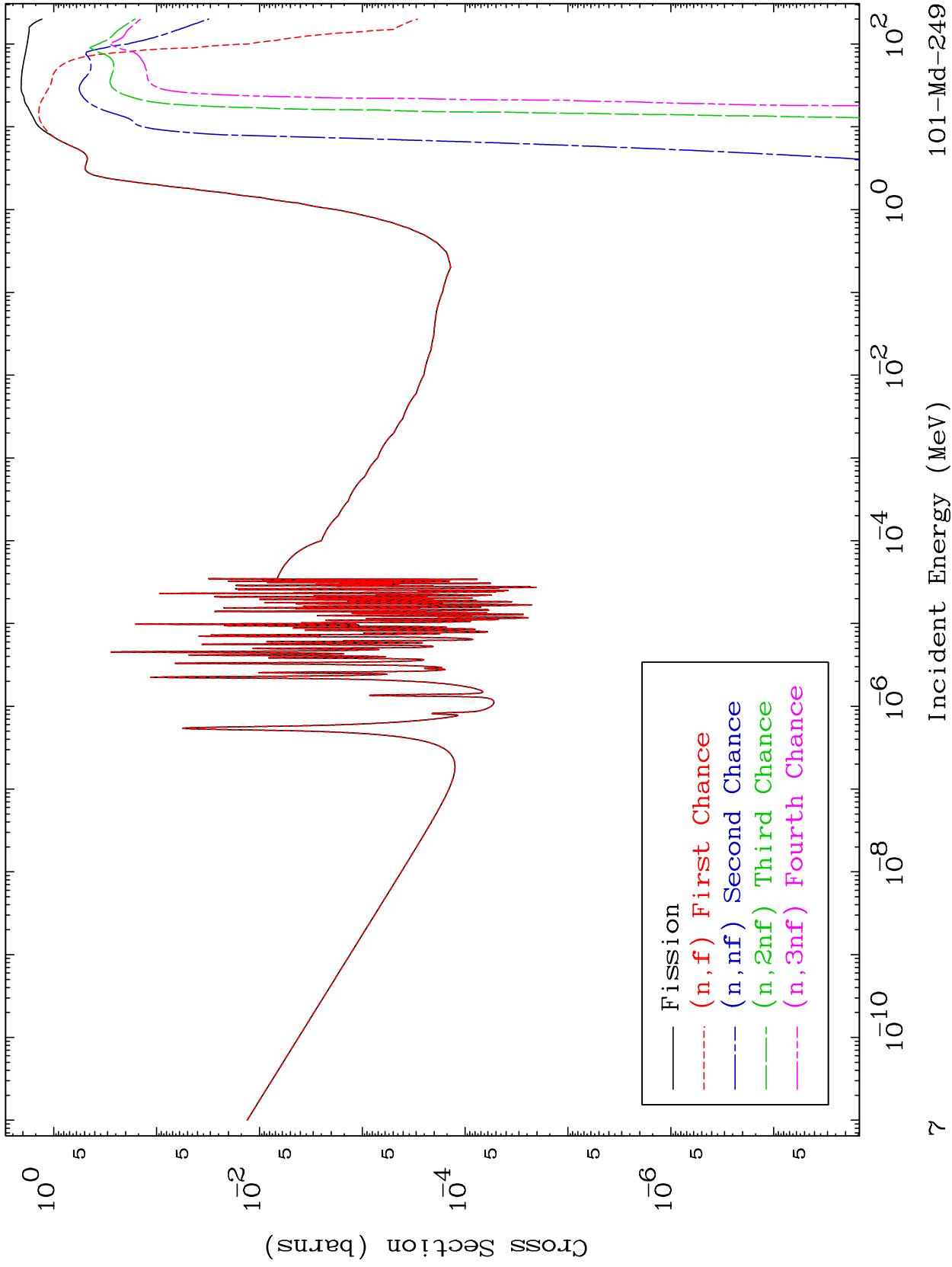
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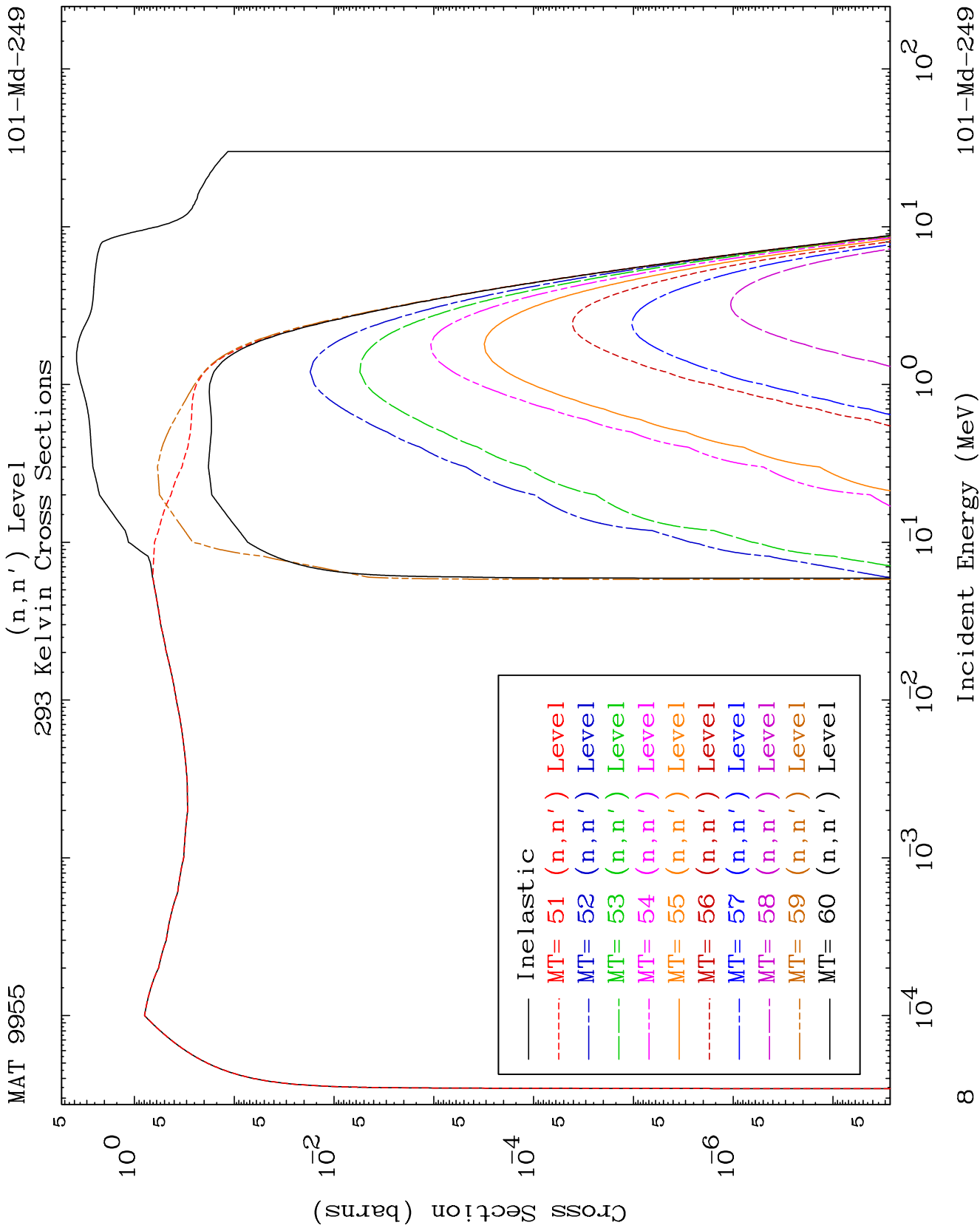


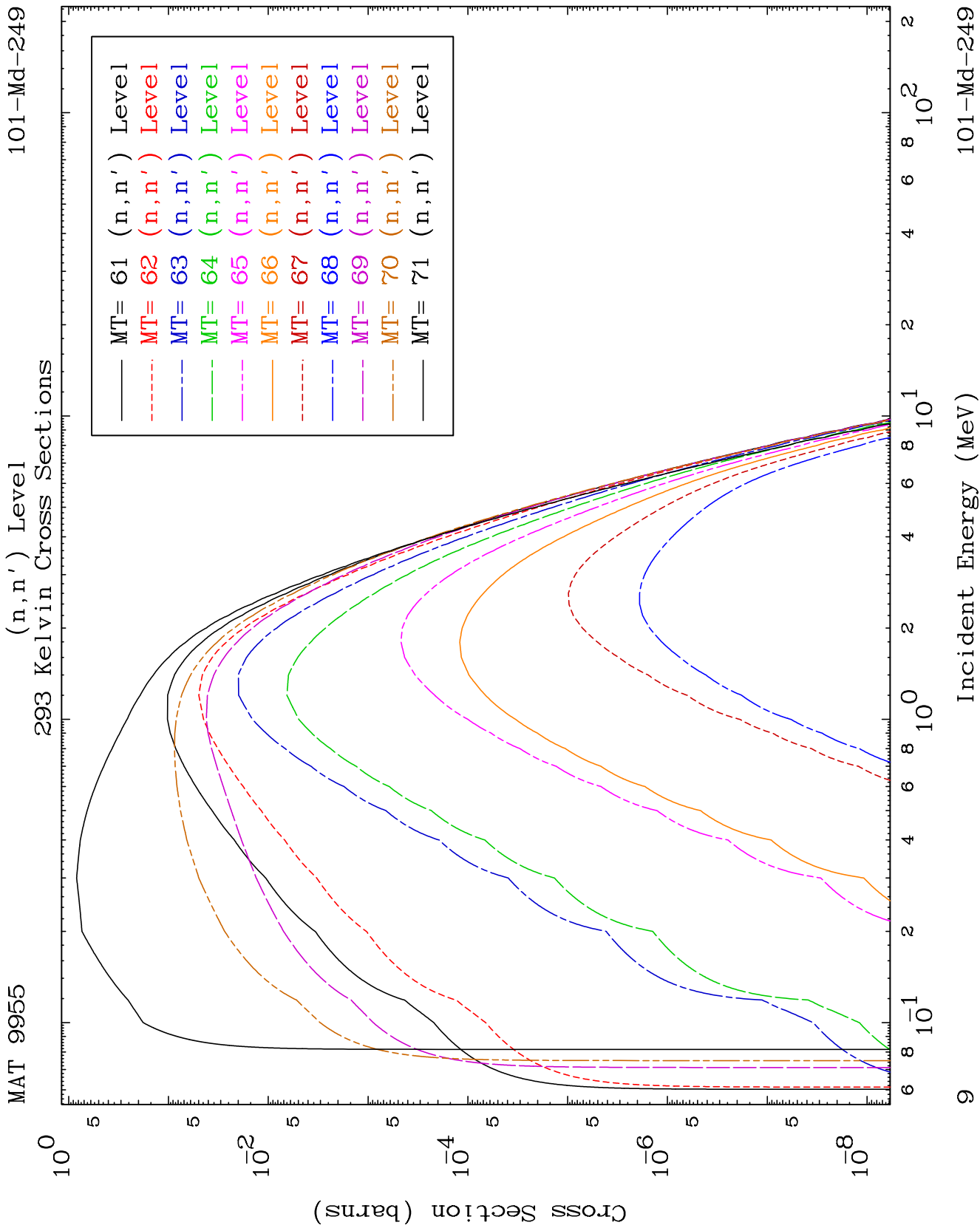
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Fission
293 Kelvin Cross Sections

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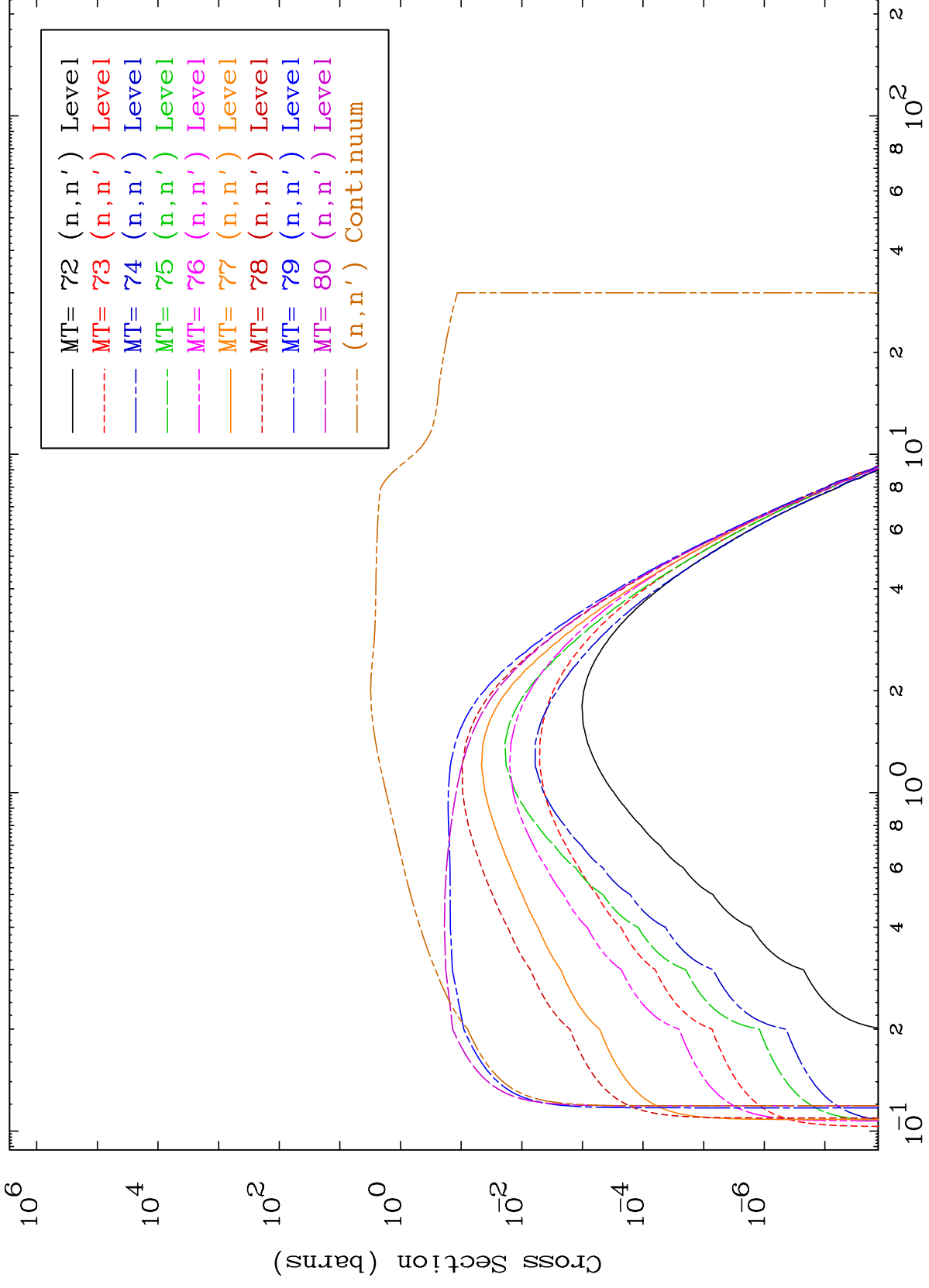


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(n,n') Level

101-Md-249

293 Kelvin Cross Sections



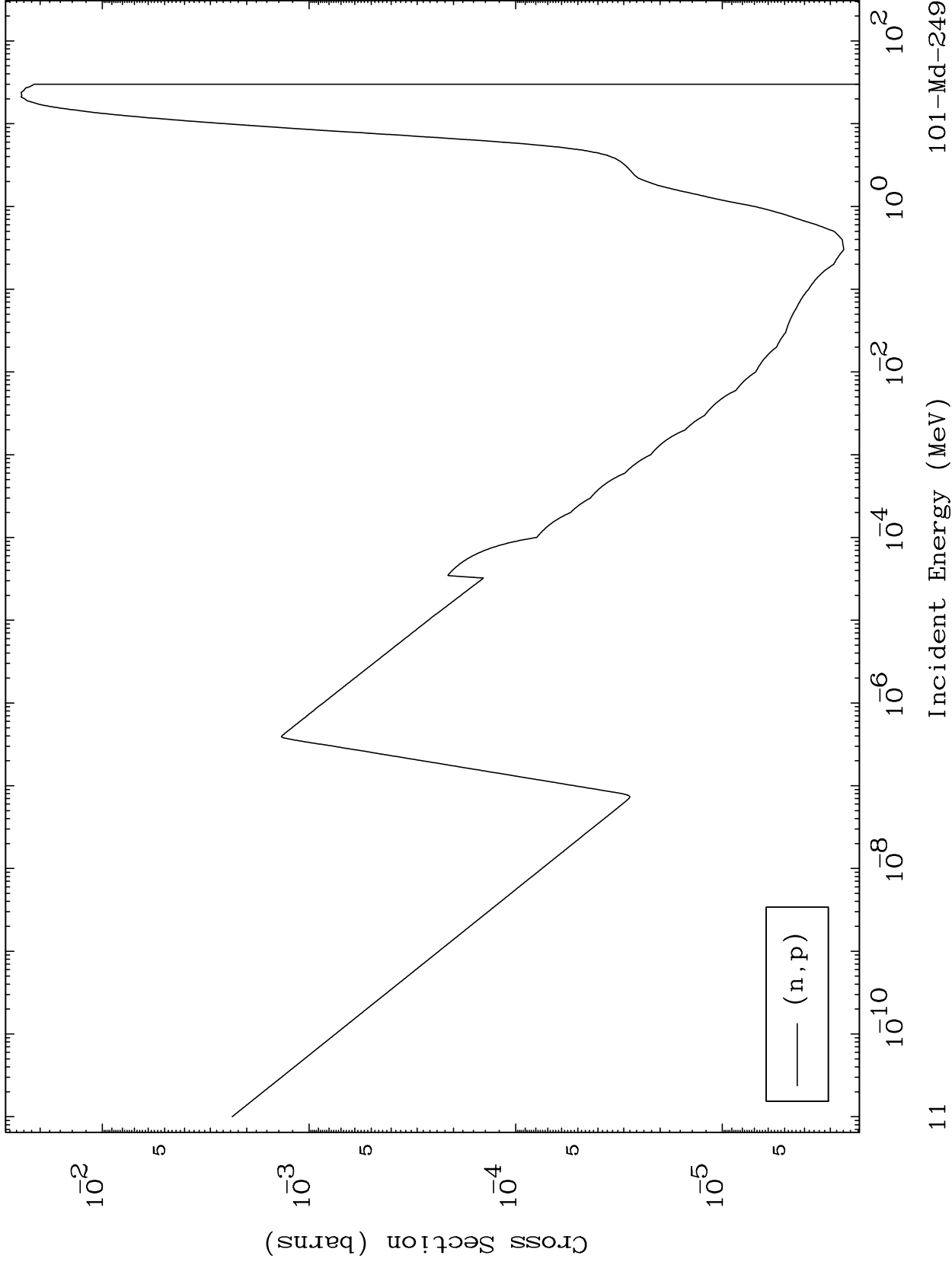
Incident Energy (MeV)

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(n,p) Levels
293 Kelvin Cross Sections

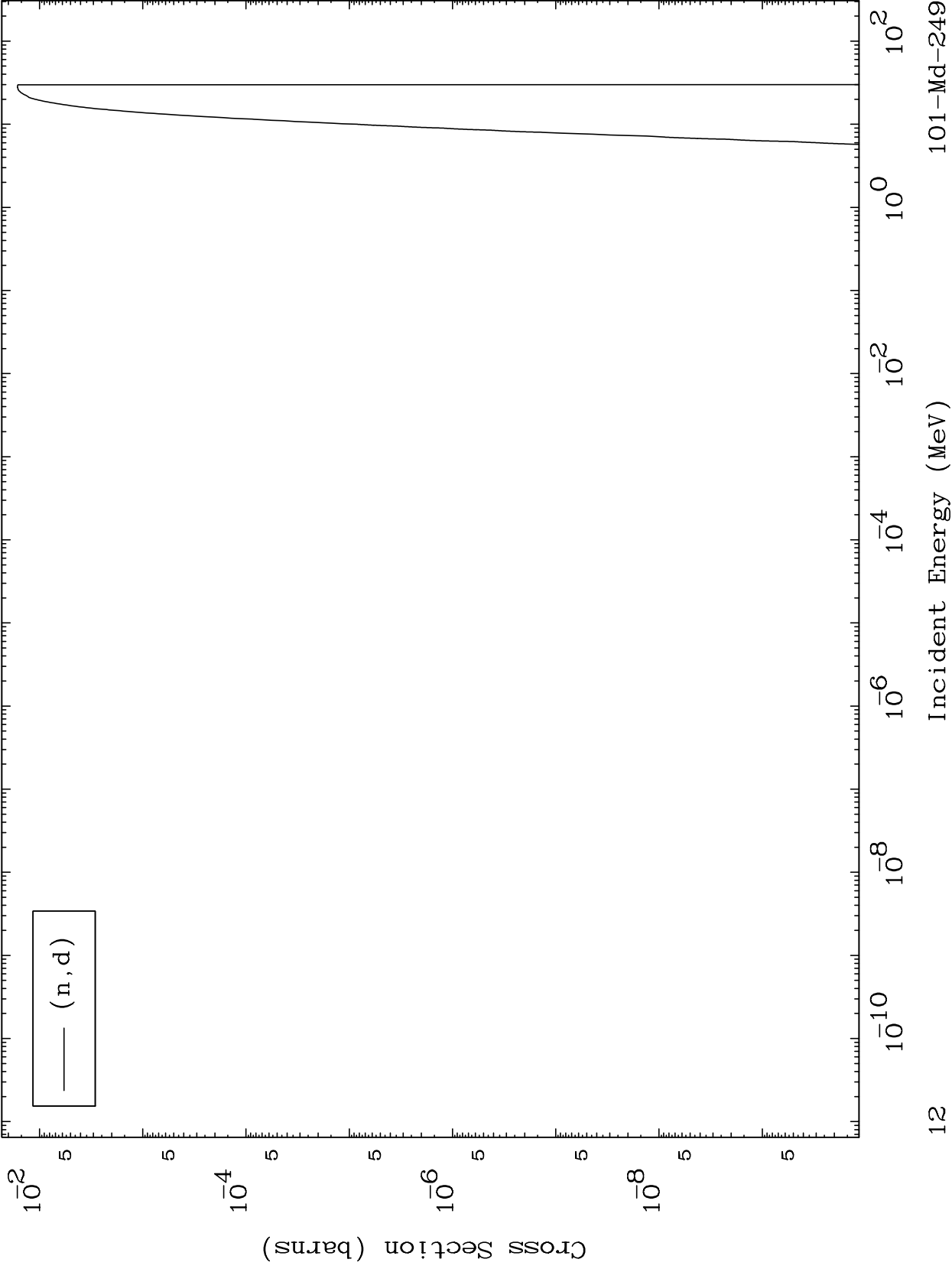
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(n,d) Levels
293 Kelvin Cross Sections

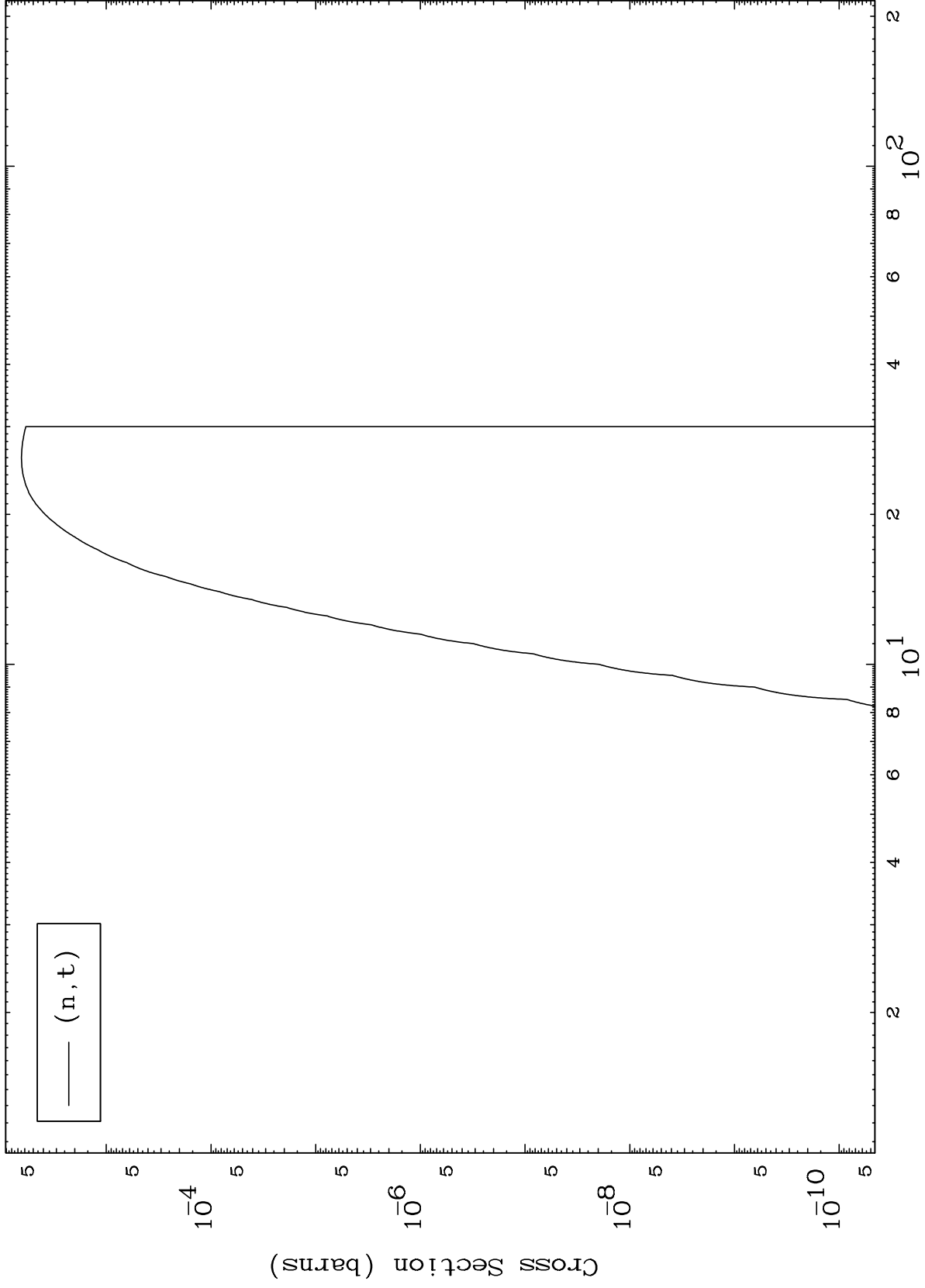
101-Md-249



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(n,t) Levels
293 Kelvin Cross Sections

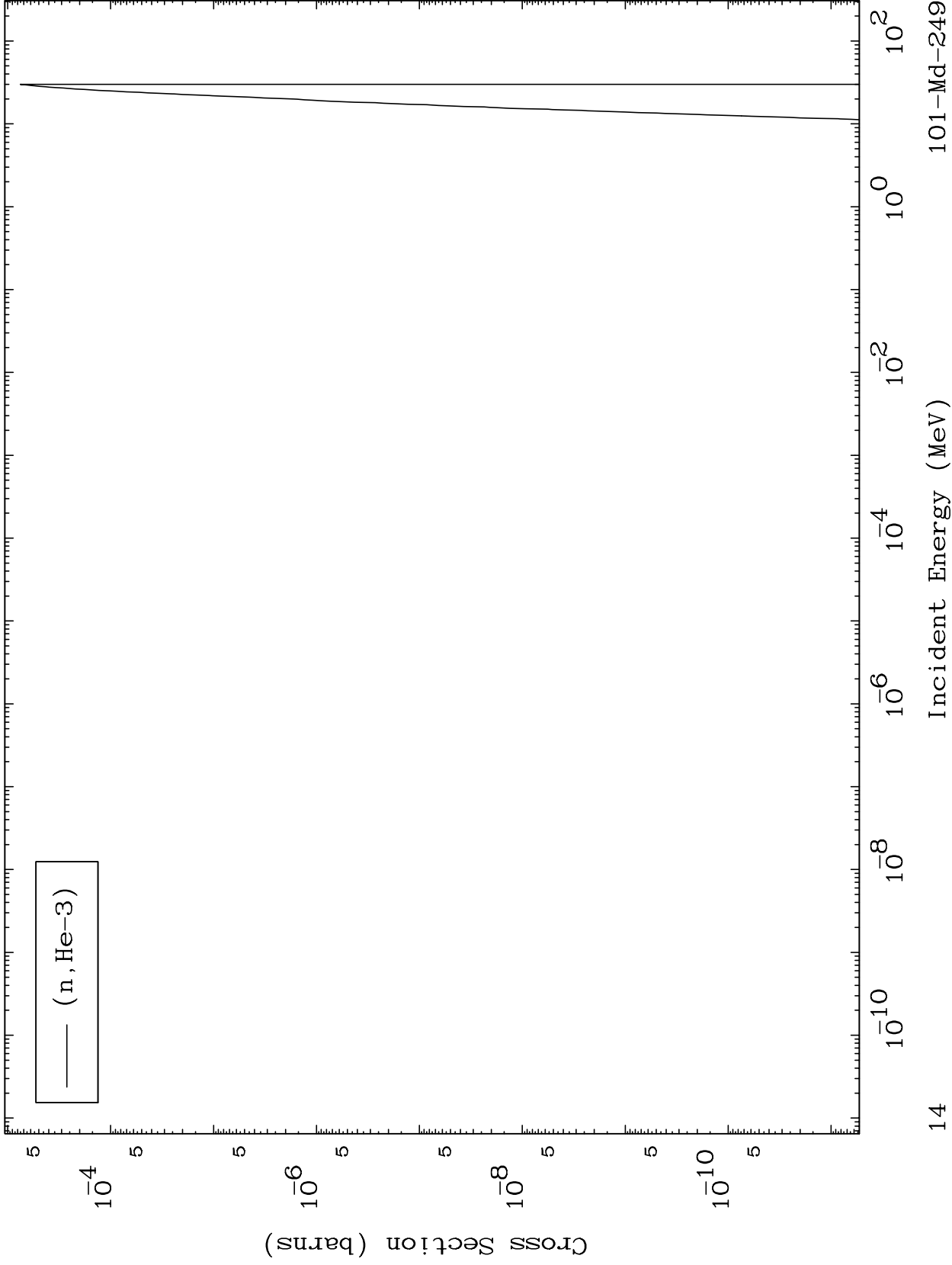
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(n,He3) Levels
293 Kelvin Cross Sections

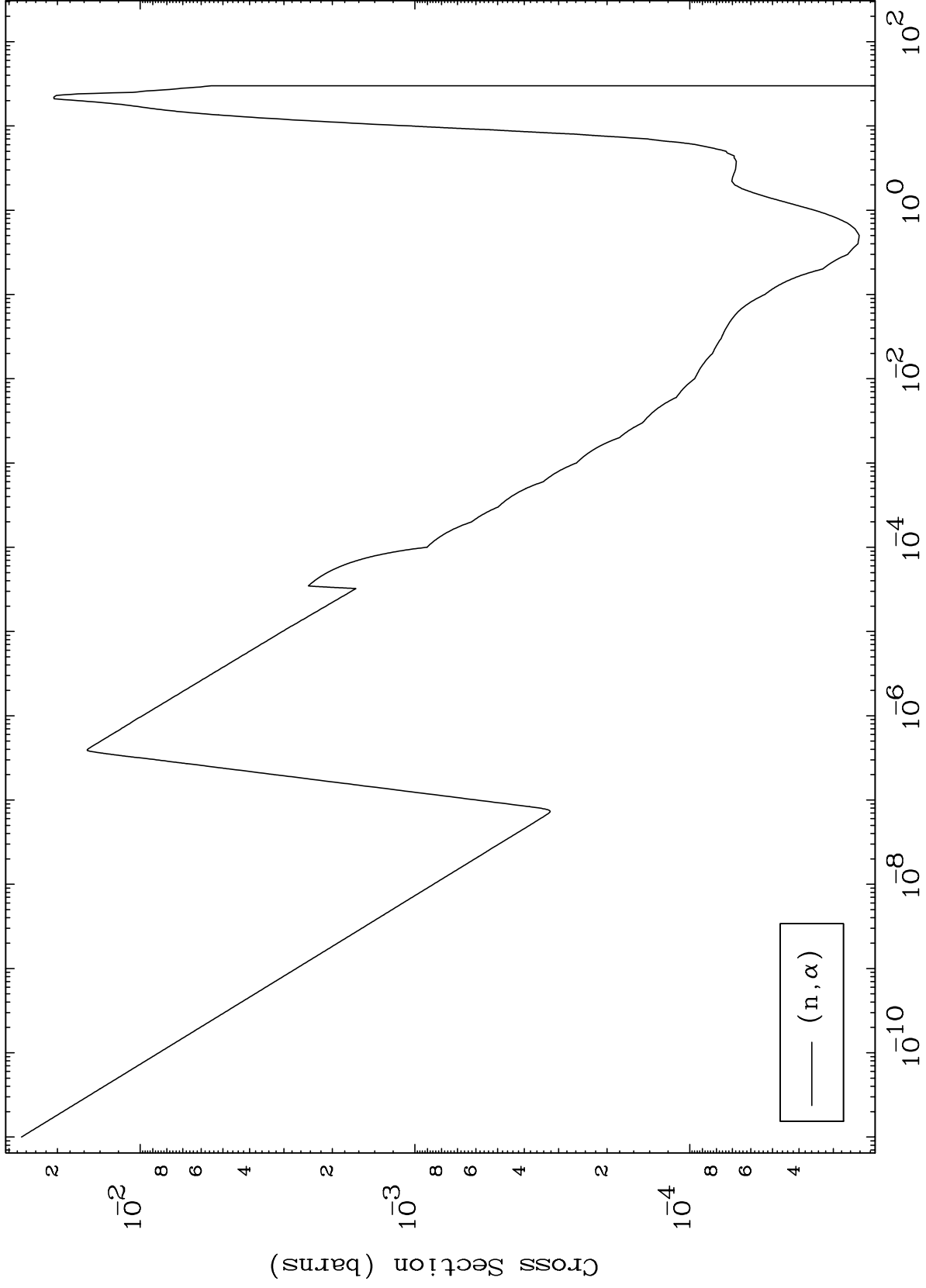
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(n,α) Levels
293 Kelvin Cross Sections

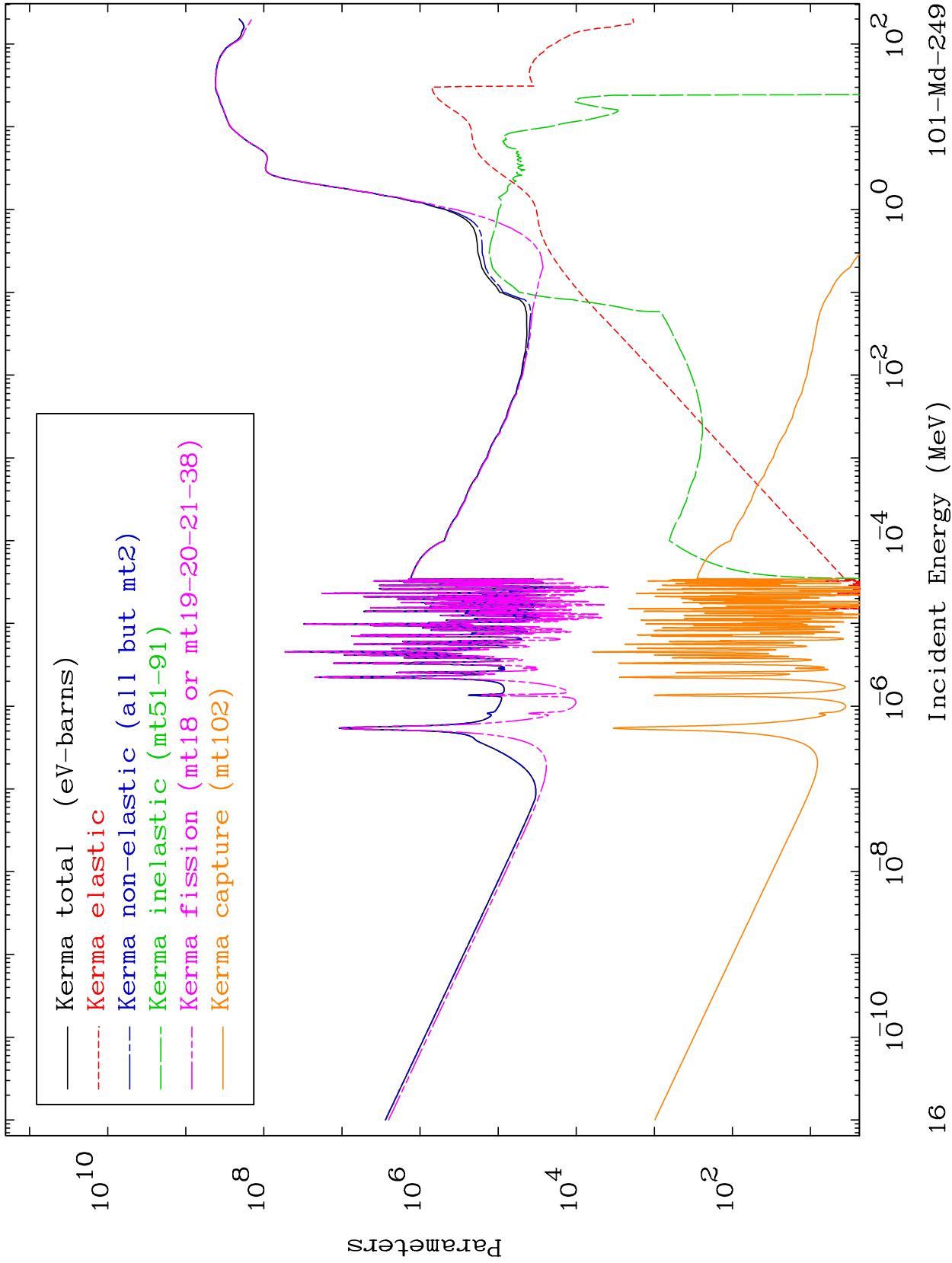
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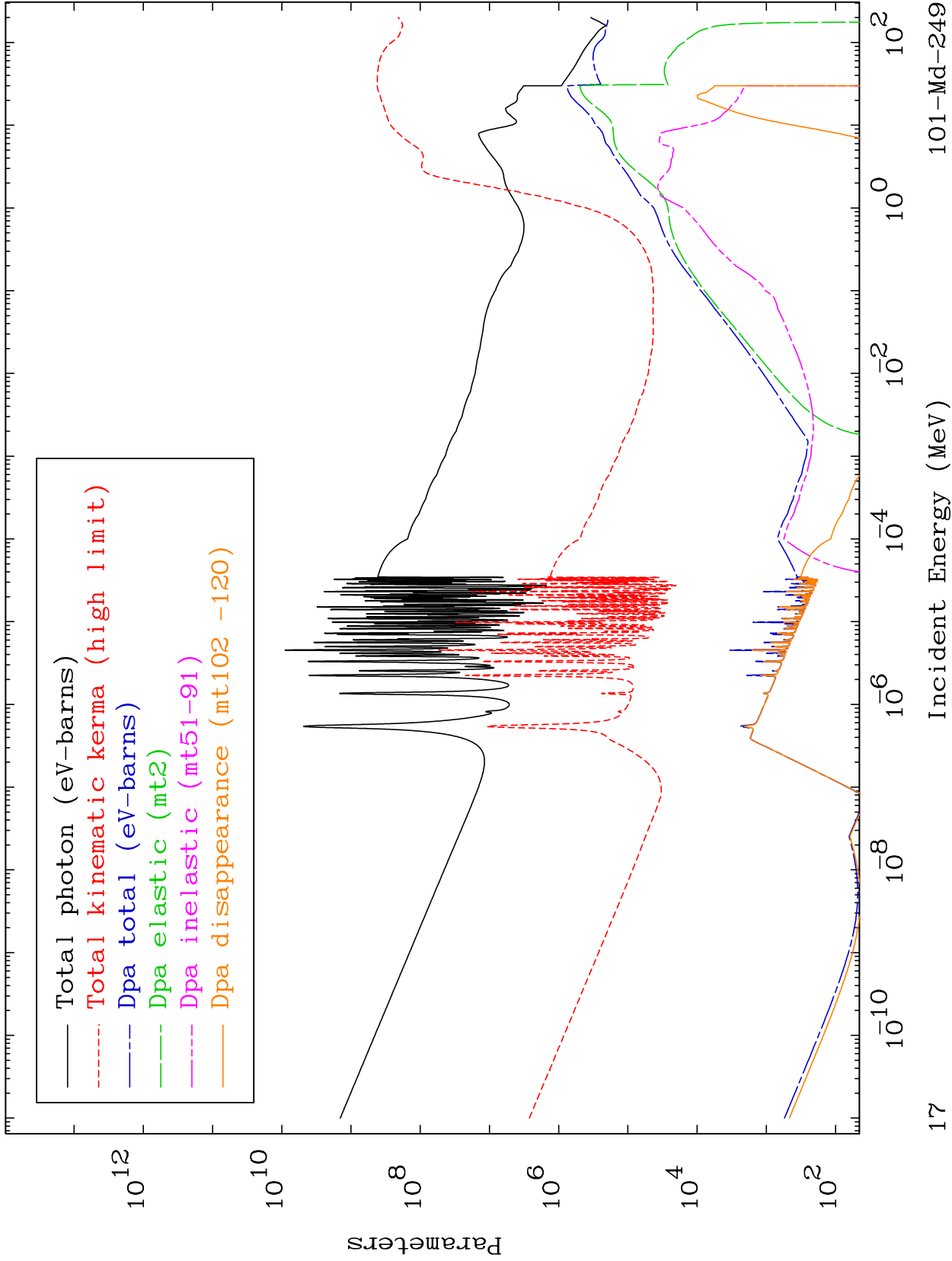


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Incident Energy (MeV)

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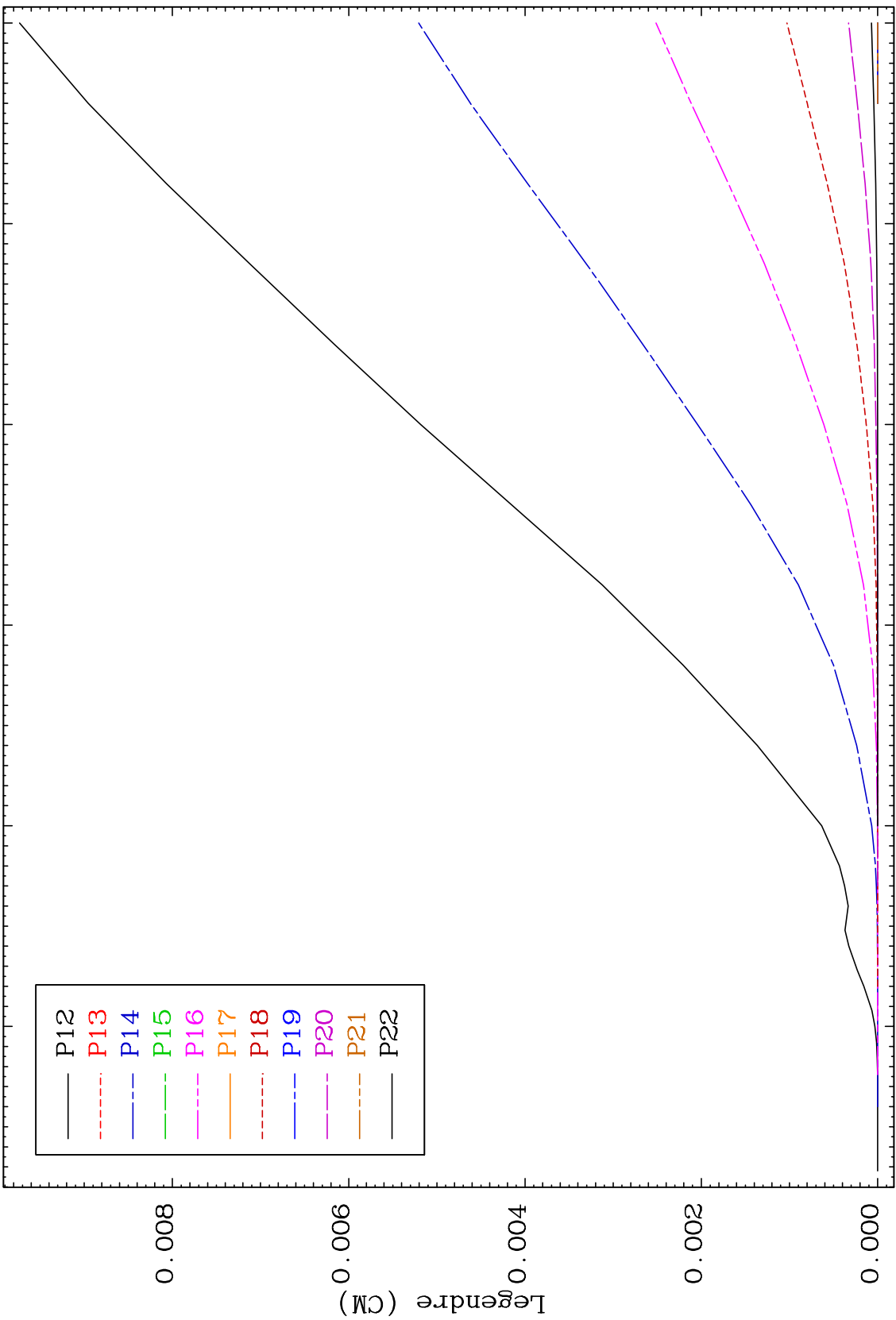




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Elastic Legendre Coefficients

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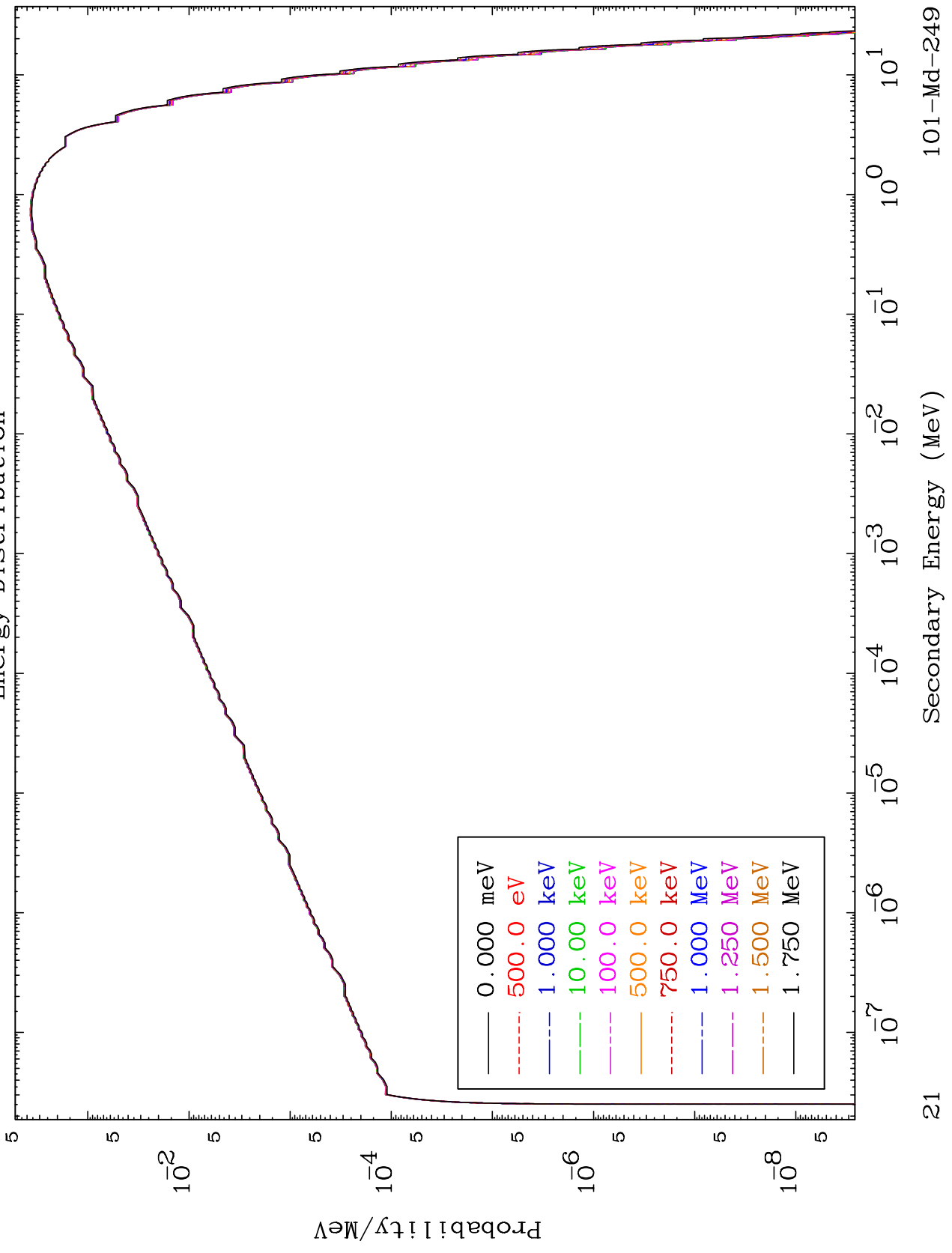
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Fission Energy Distribution

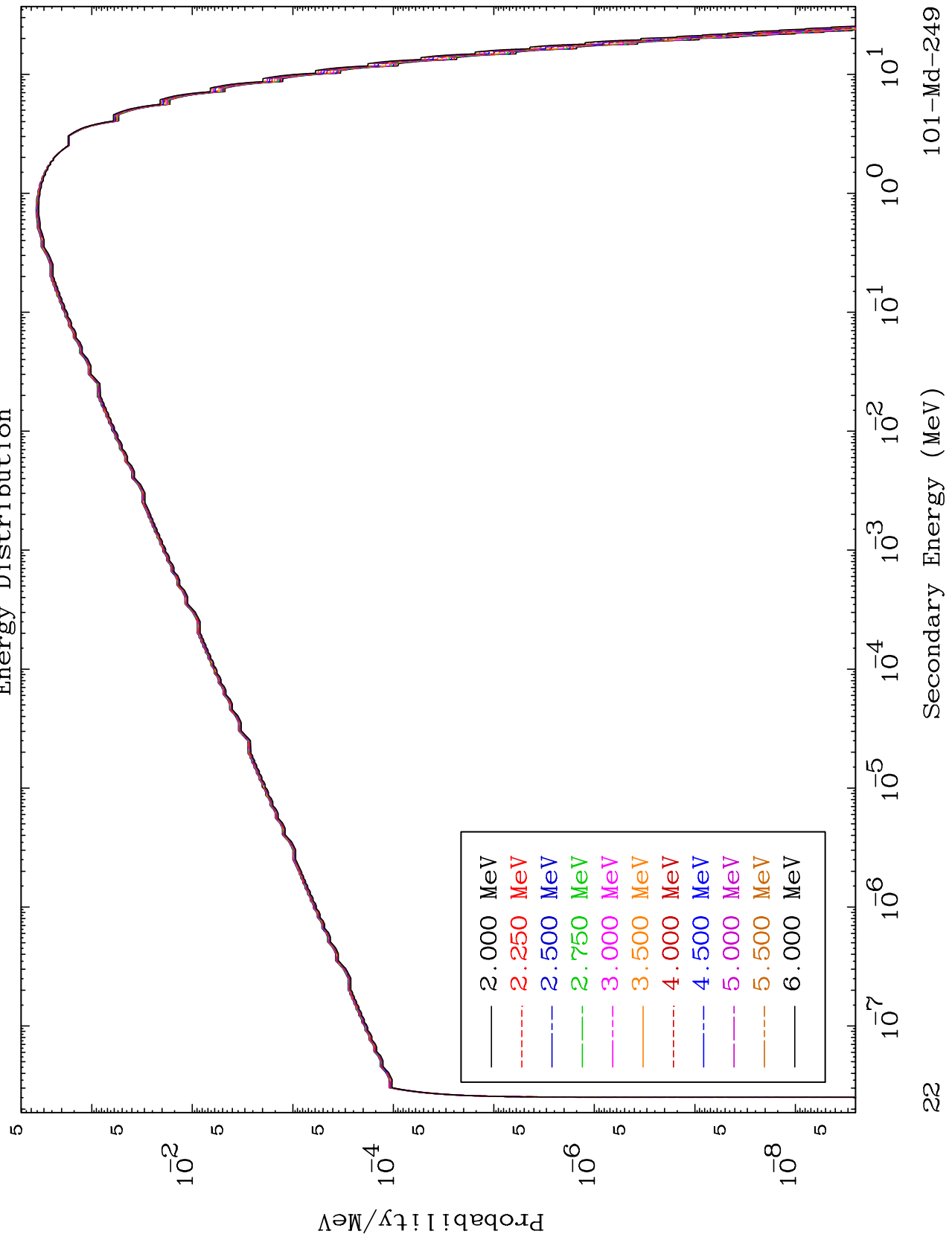
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Fission Energy Distribution

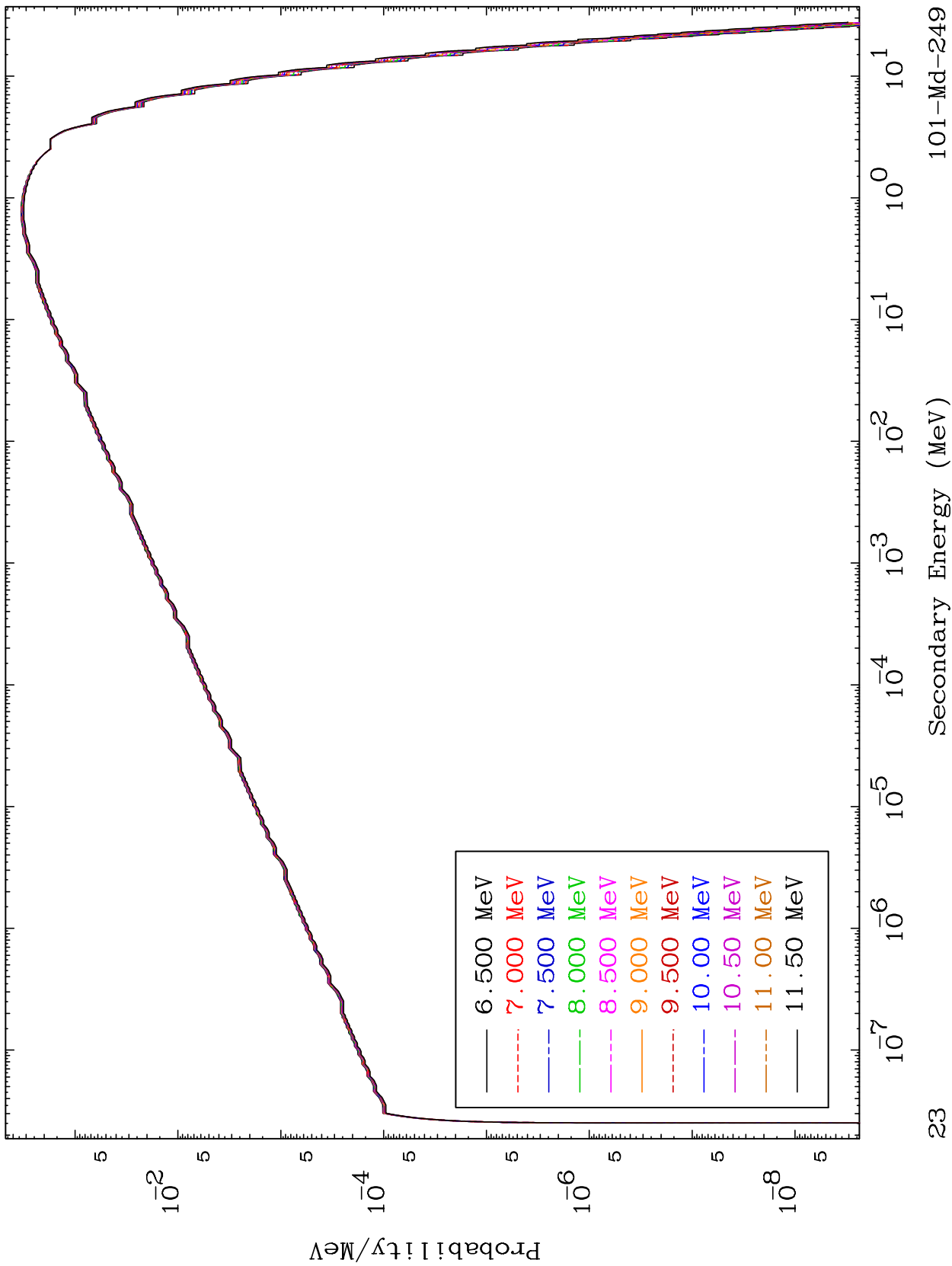
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Fission Energy Distribution

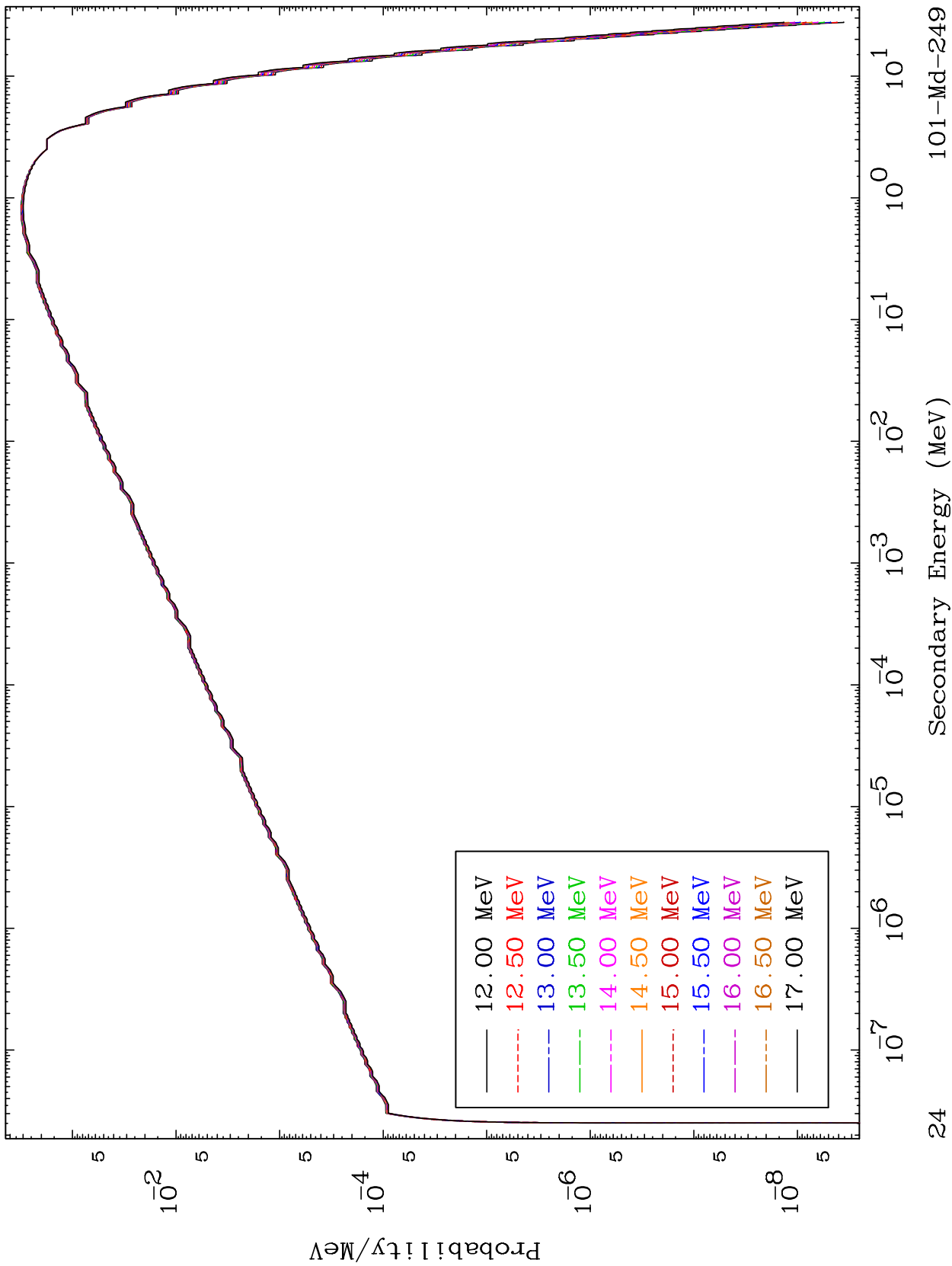
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Fission Energy Distribution

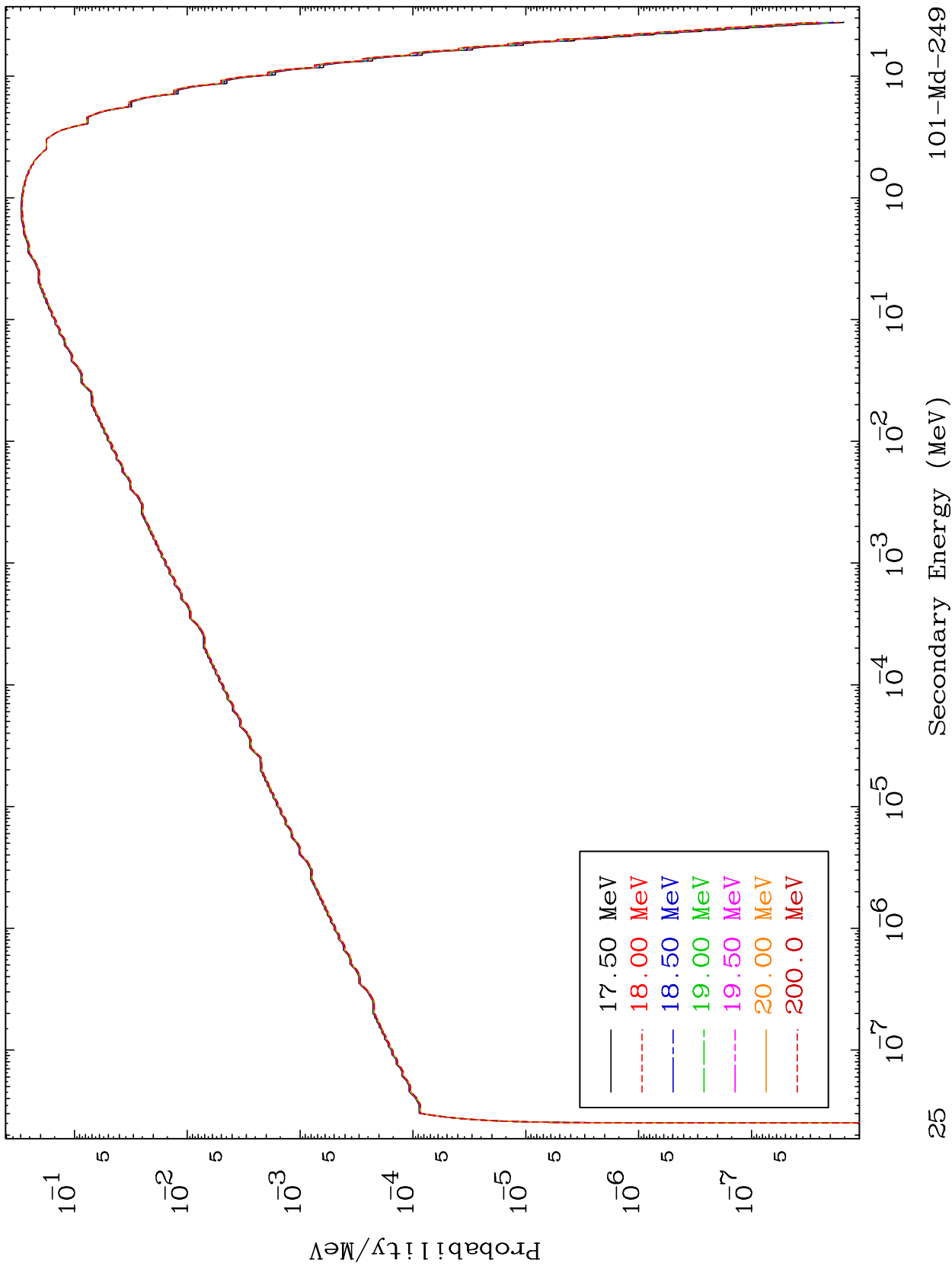
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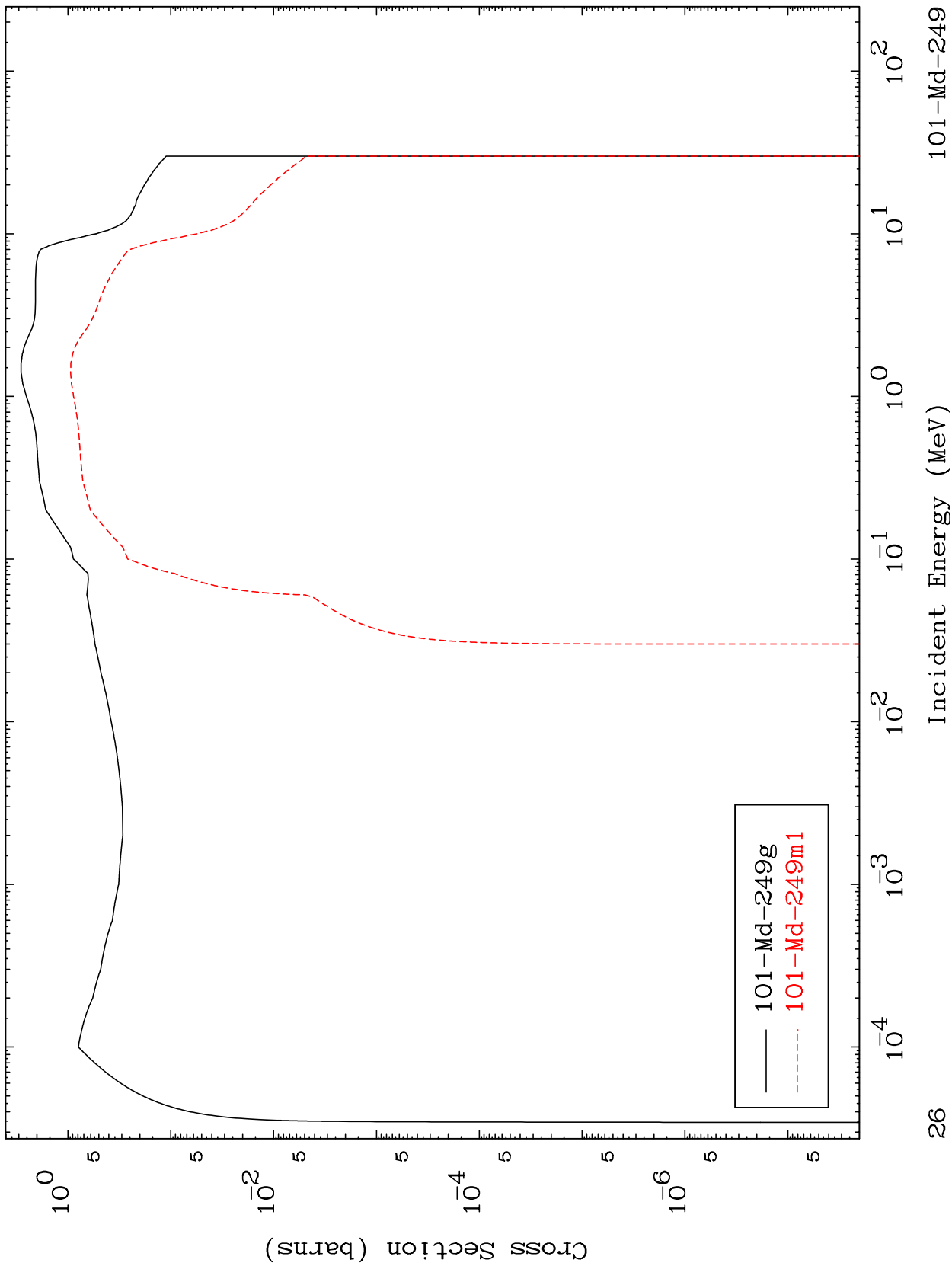
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Fission Energy Distribution

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Inelastic
Radionuclide Production Cross Section

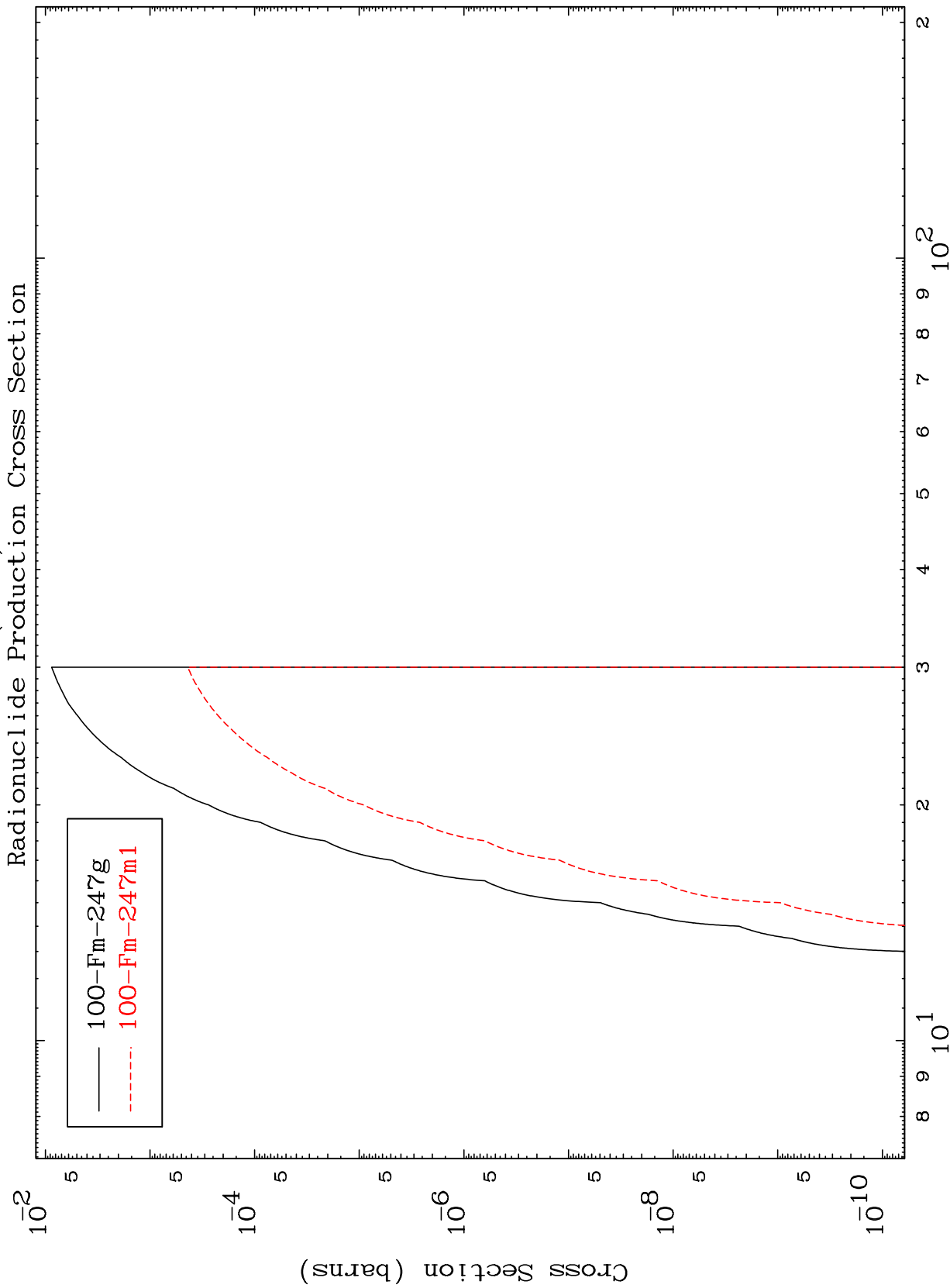


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(n,n') d

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Radionuclide Production Cross Section



27

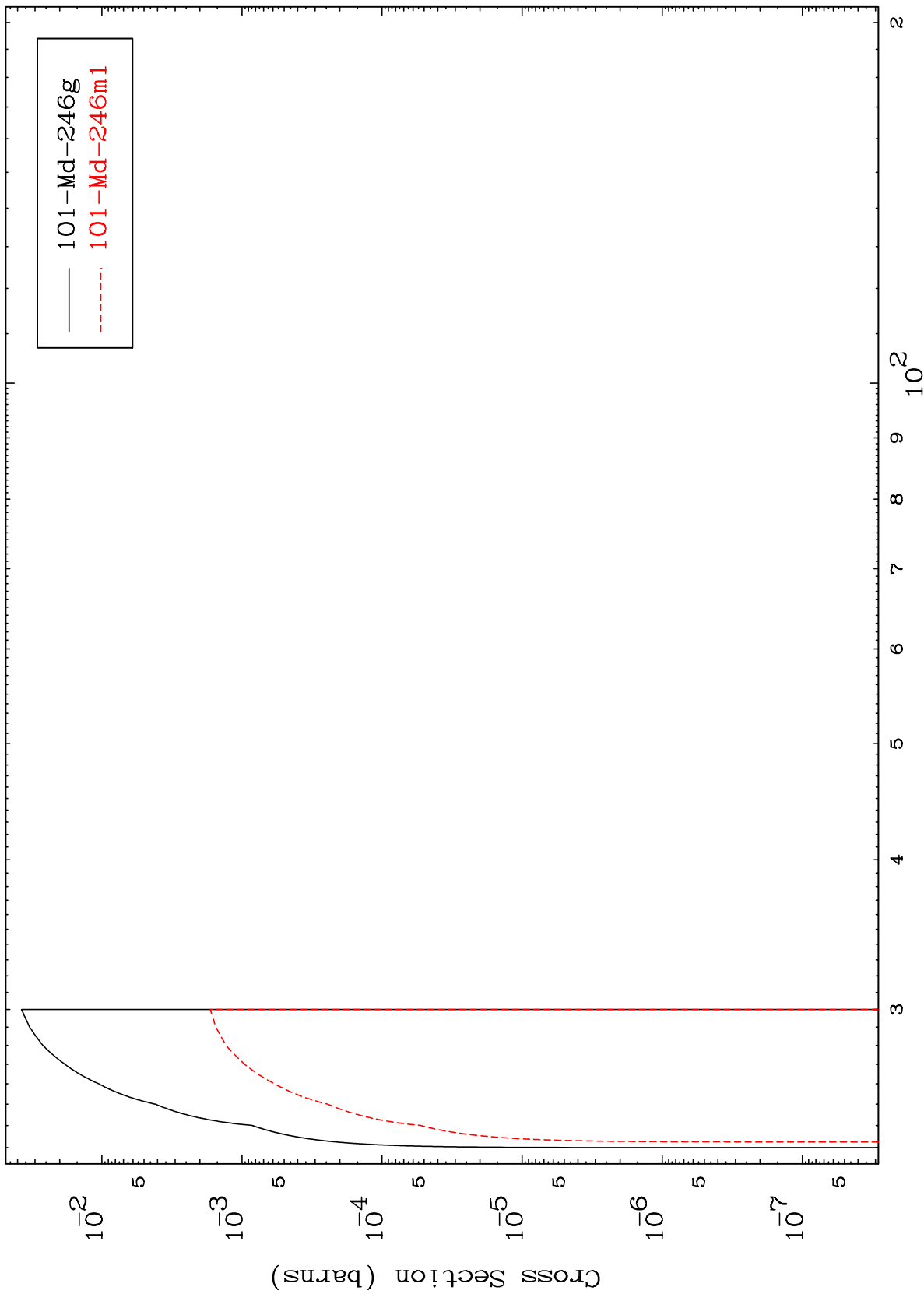
Incident Energy (MeV)

101-Md-249

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(n,4n)
Radionuclide Production Cross Section



101-Md-249

Incident Energy (MeV)

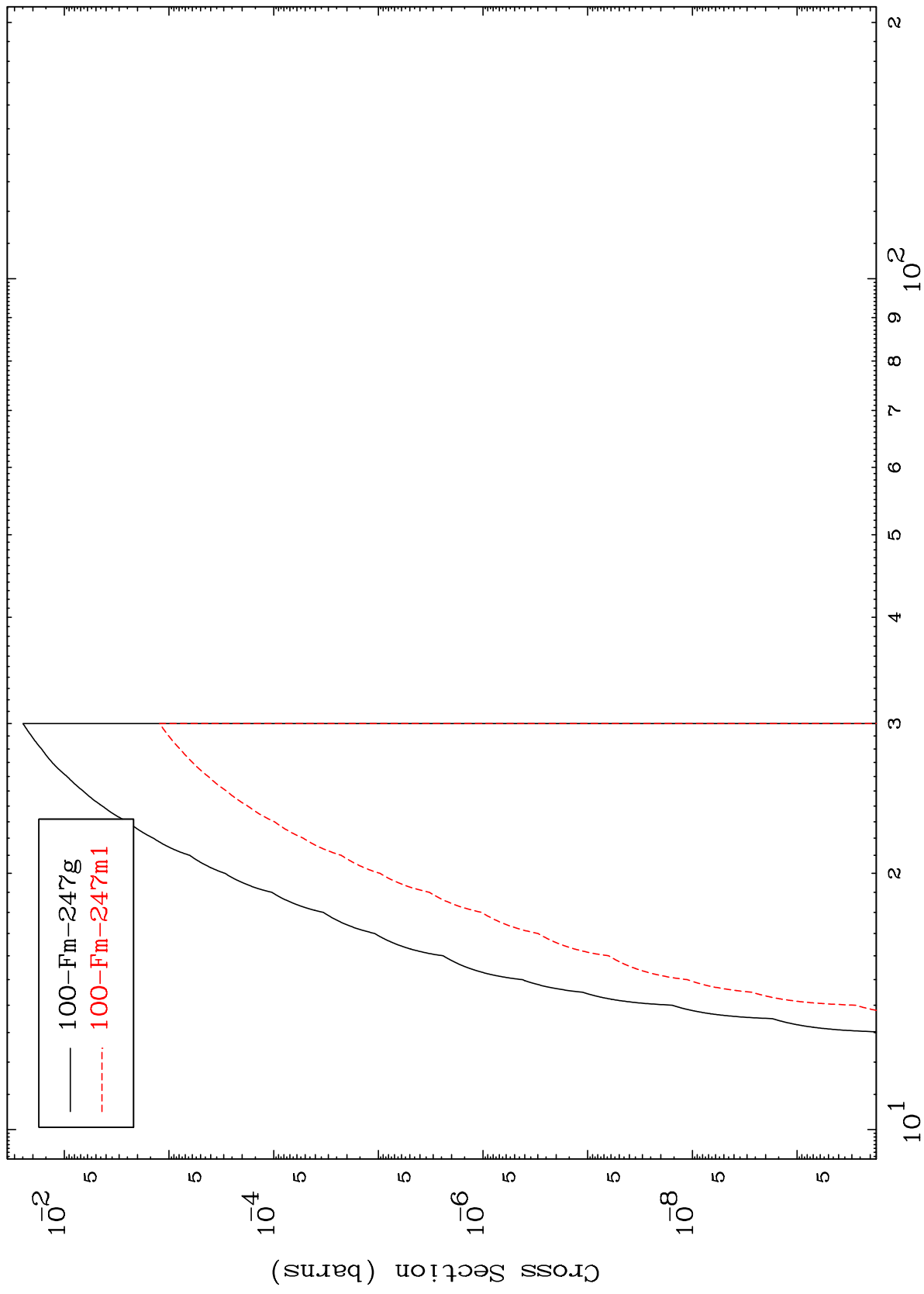
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(n,2n) p

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Radionuclide Production Cross Section



29

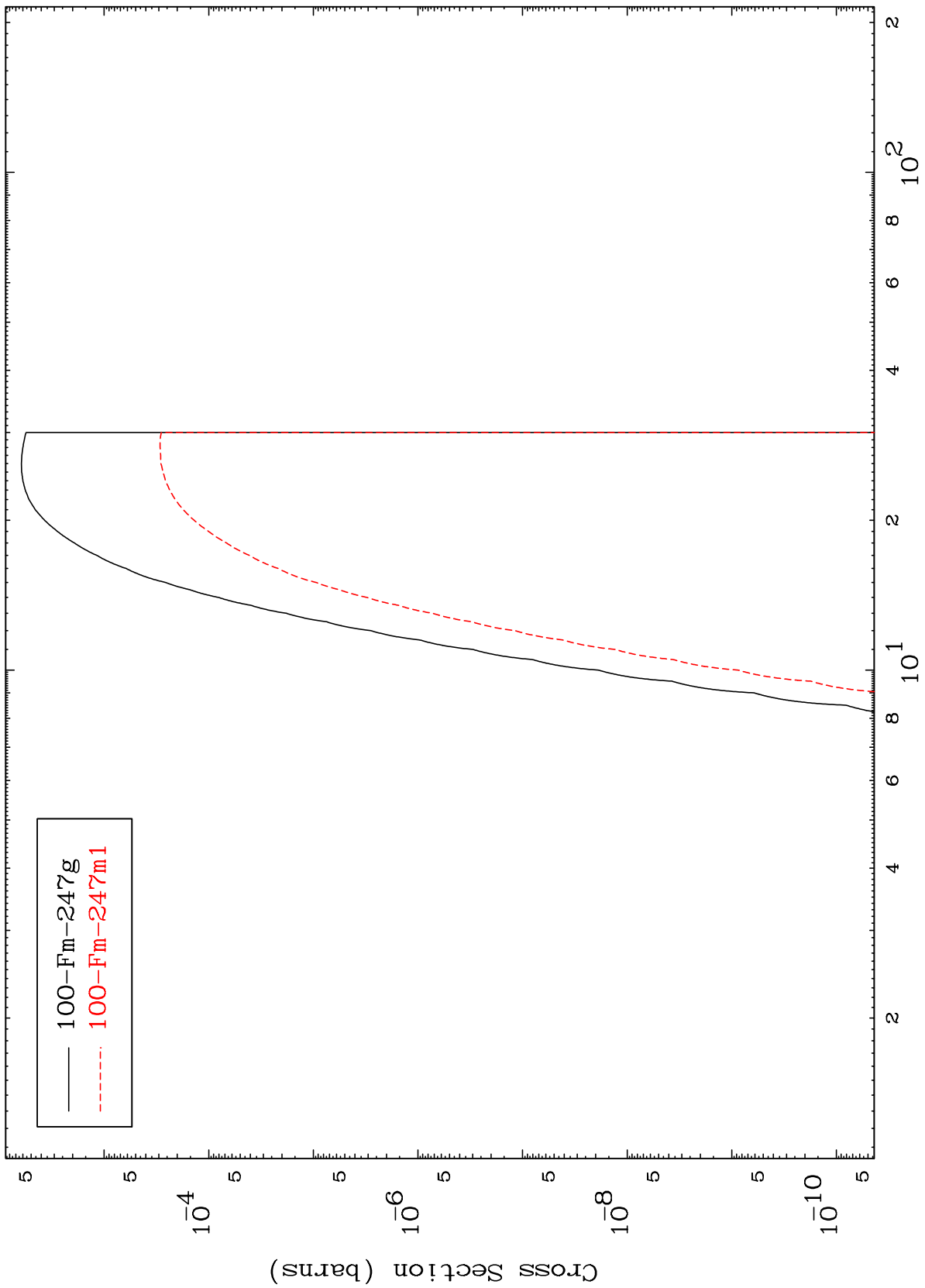
Incident Energy (MeV)

101-Md-249

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101-Md-249

(n, t)
Radionuclide Production Cross Section



30

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Incident Energy (MeV)