

Program EVALPLOT
(Version 2017-1)

by

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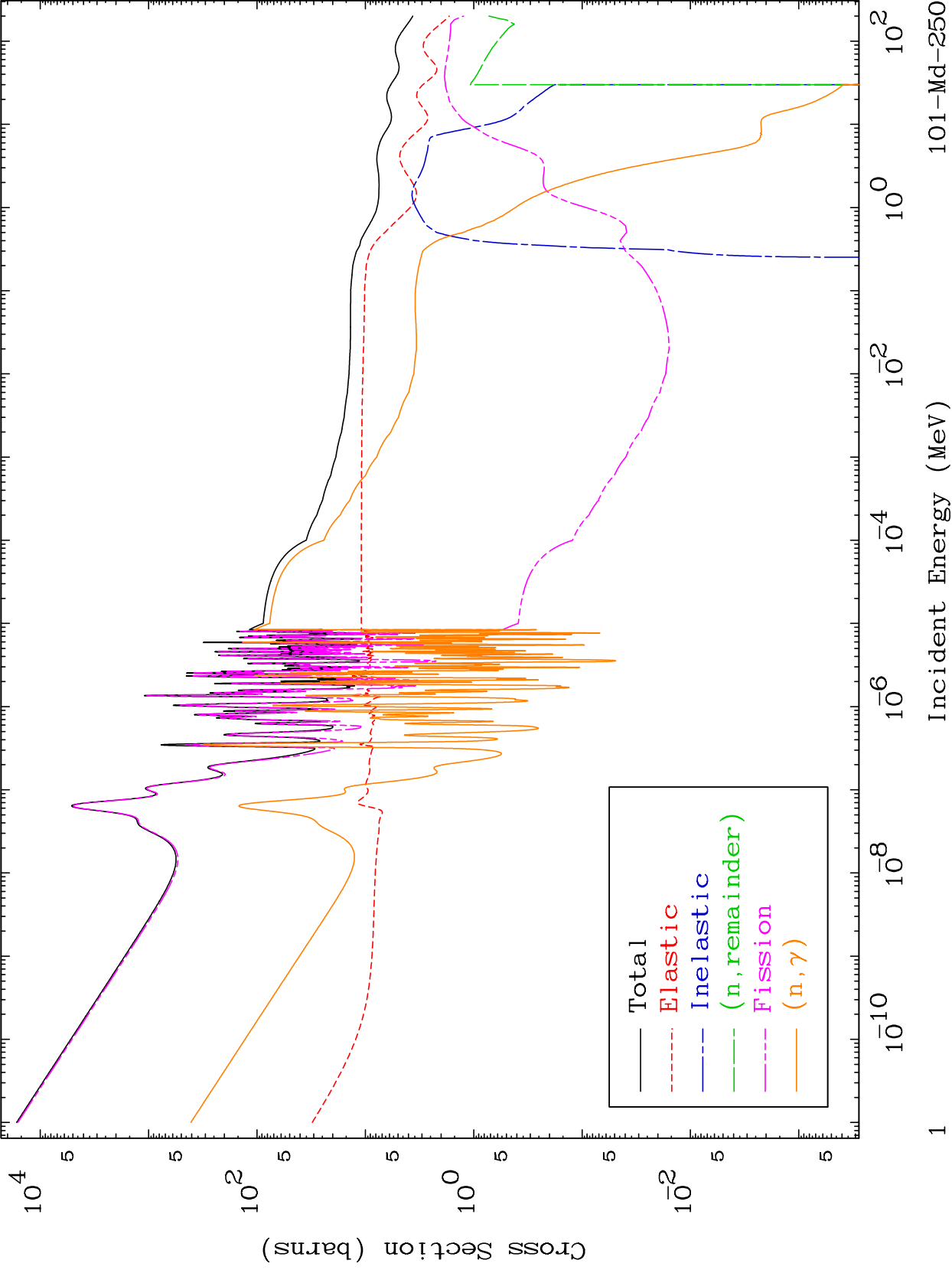
Press Mouse Button to Start

MAT 9951

Major

293 Kelvin Cross Sections

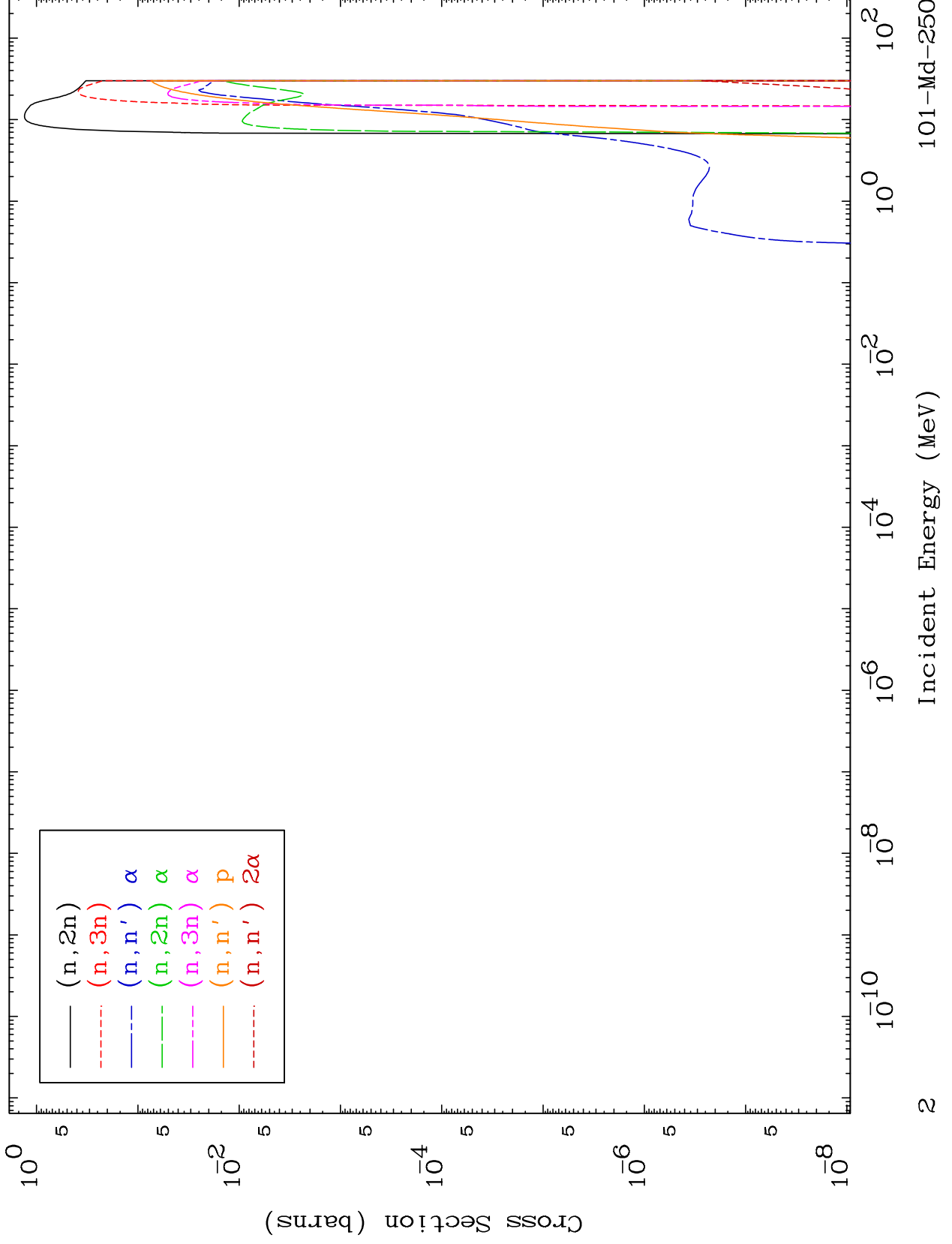
101-Md-250



MAT 9951

Neutron Production
293 Kelvin Cross Sections

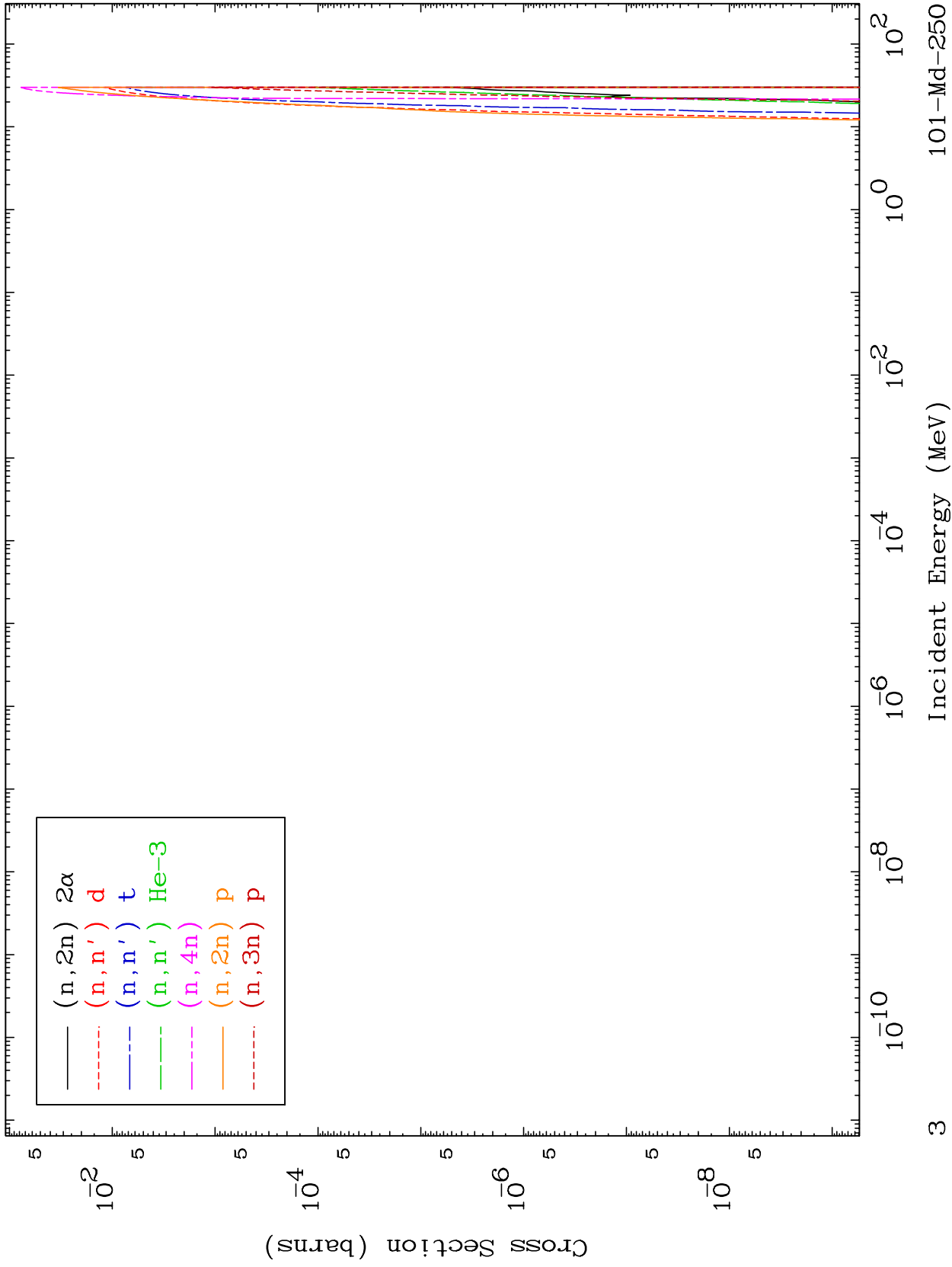
101-Md-250



MAT 9951

Neutron Production
293 Kelvin Cross Sections

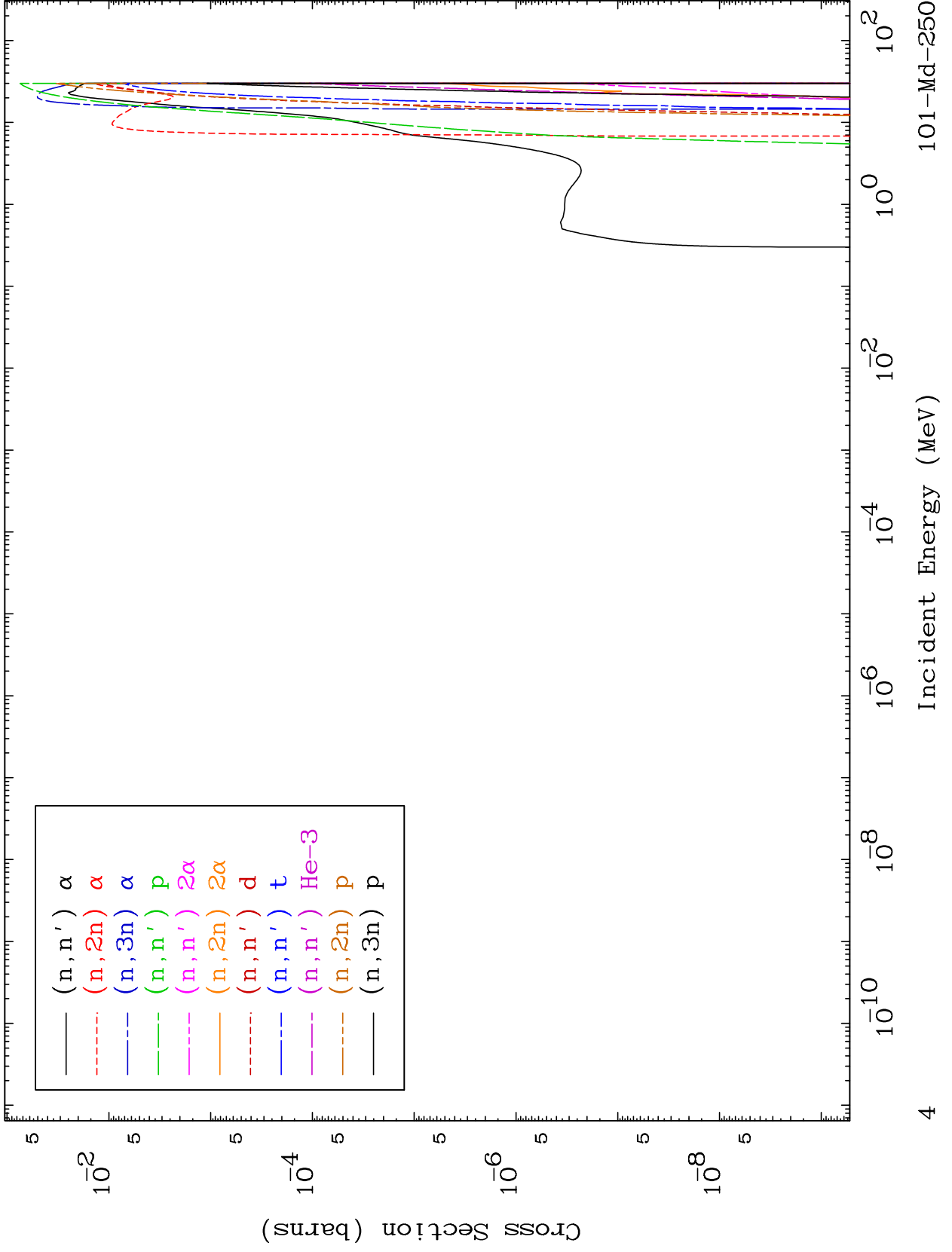
101-Md-250



MAT 9951

Charged Particle
293 Kelvin Cross Sections

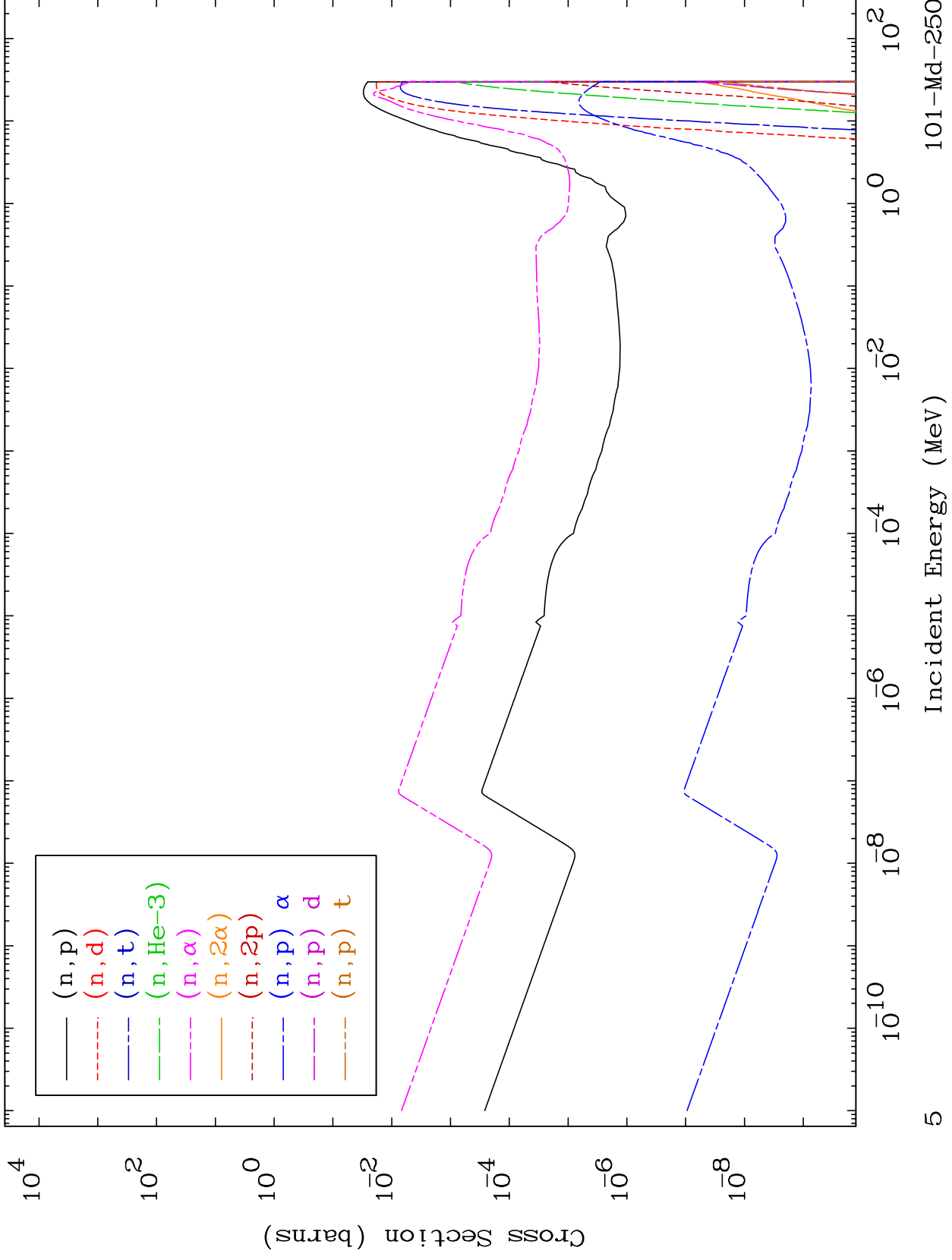
101-Md-250



MAT 9951

Charged Particle
293 Kelvin Cross Sections

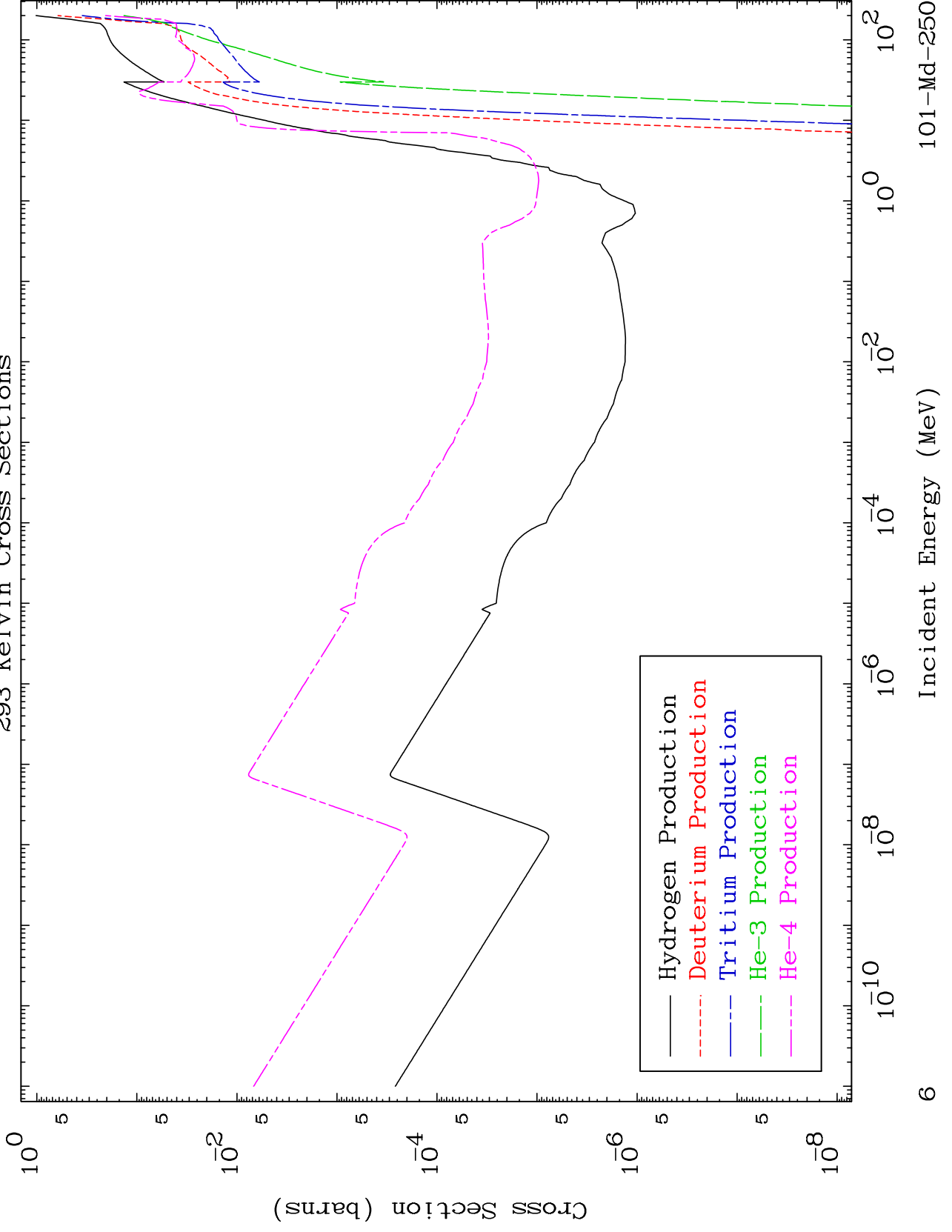
101-Md-250



MAT 9951

Particle Production
293 Kelvin Cross Sections

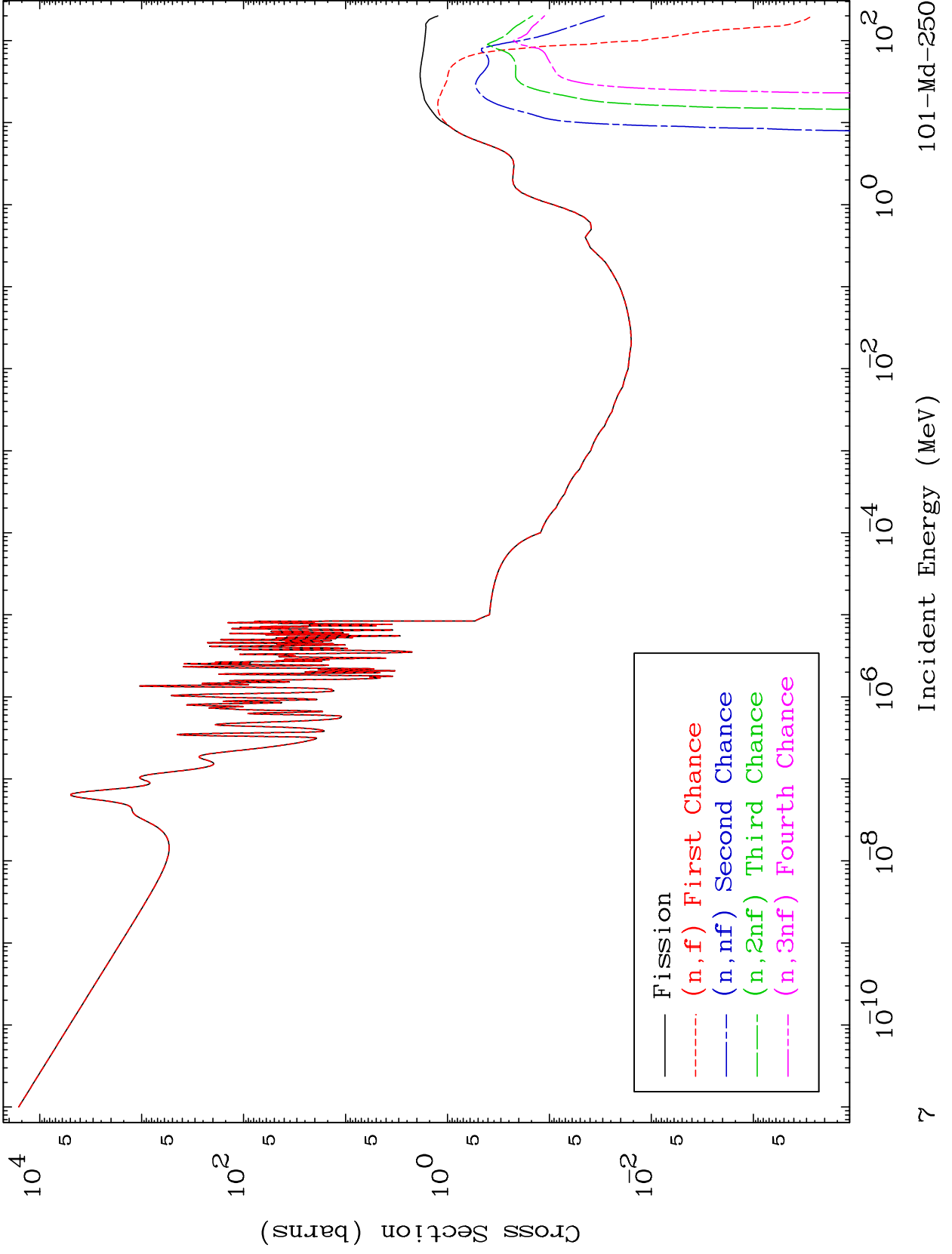
101-Md-250



MAT 9951

Fission
293 Kelvin Cross Sections

101-Md-250

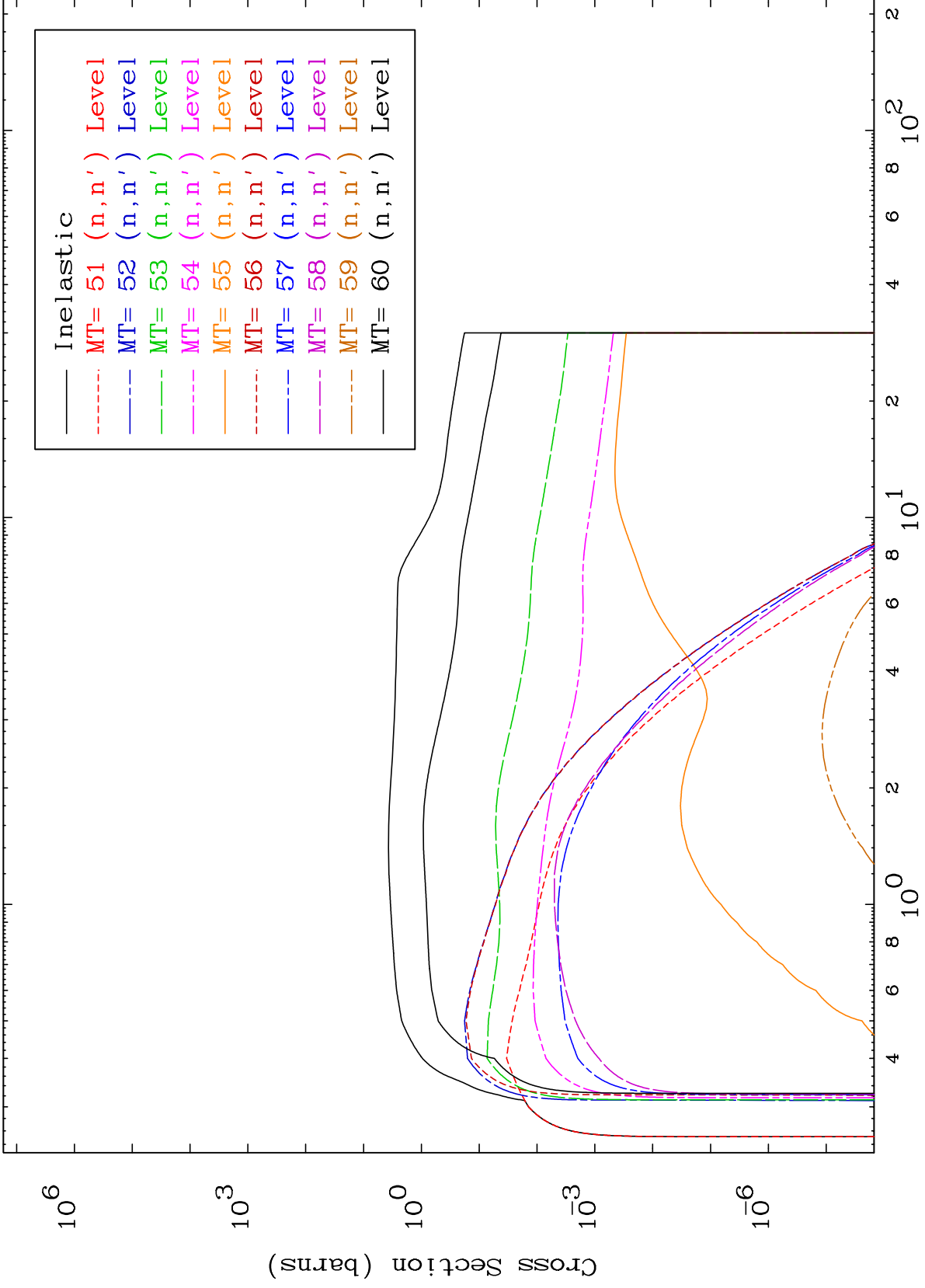


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(n,n') Level

101-Md-250

293 Kelvin Cross Sections

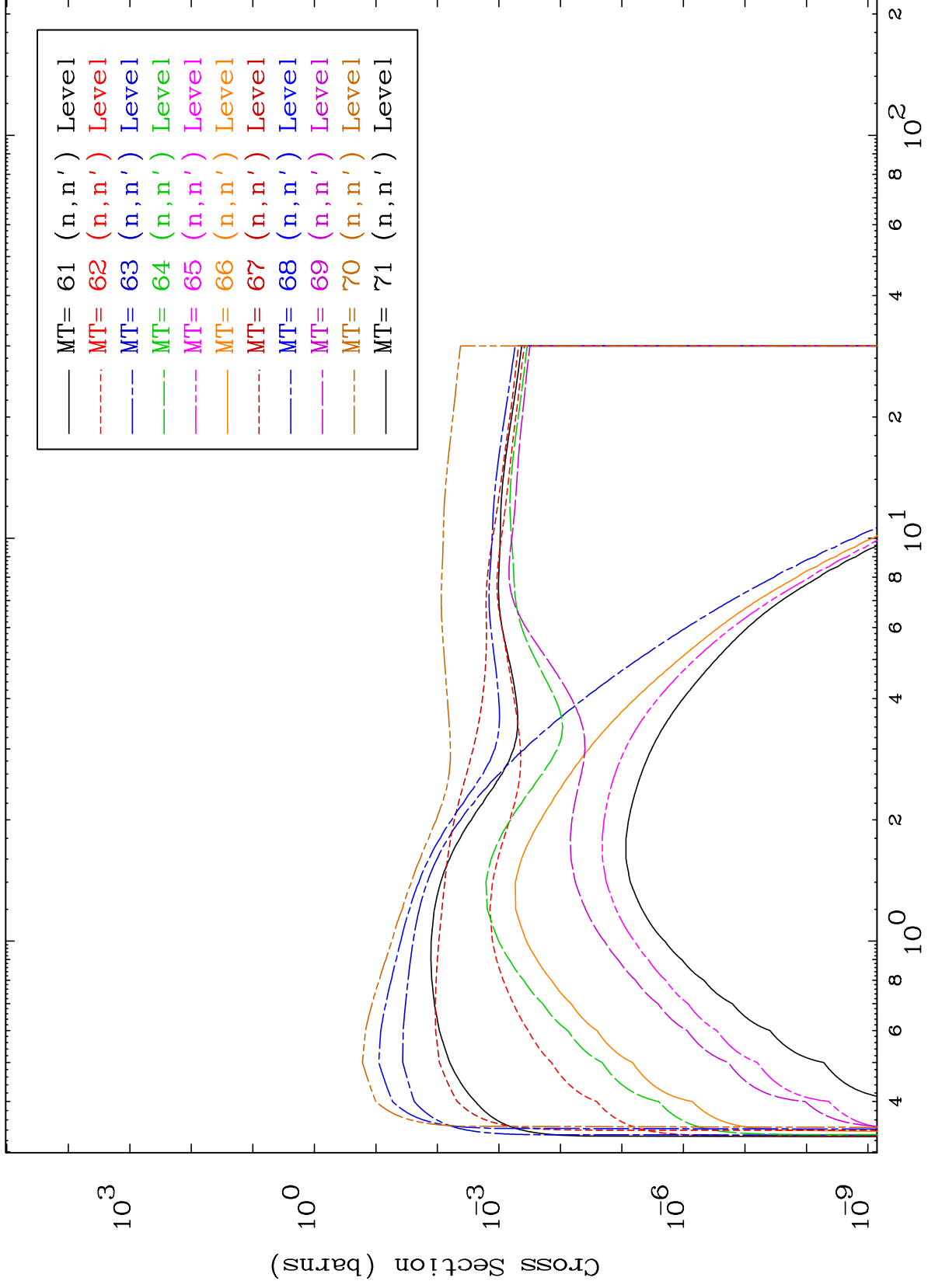


MAT 9951

(n,n') Level

101-Md-250

293 Kelvin Cross Sections



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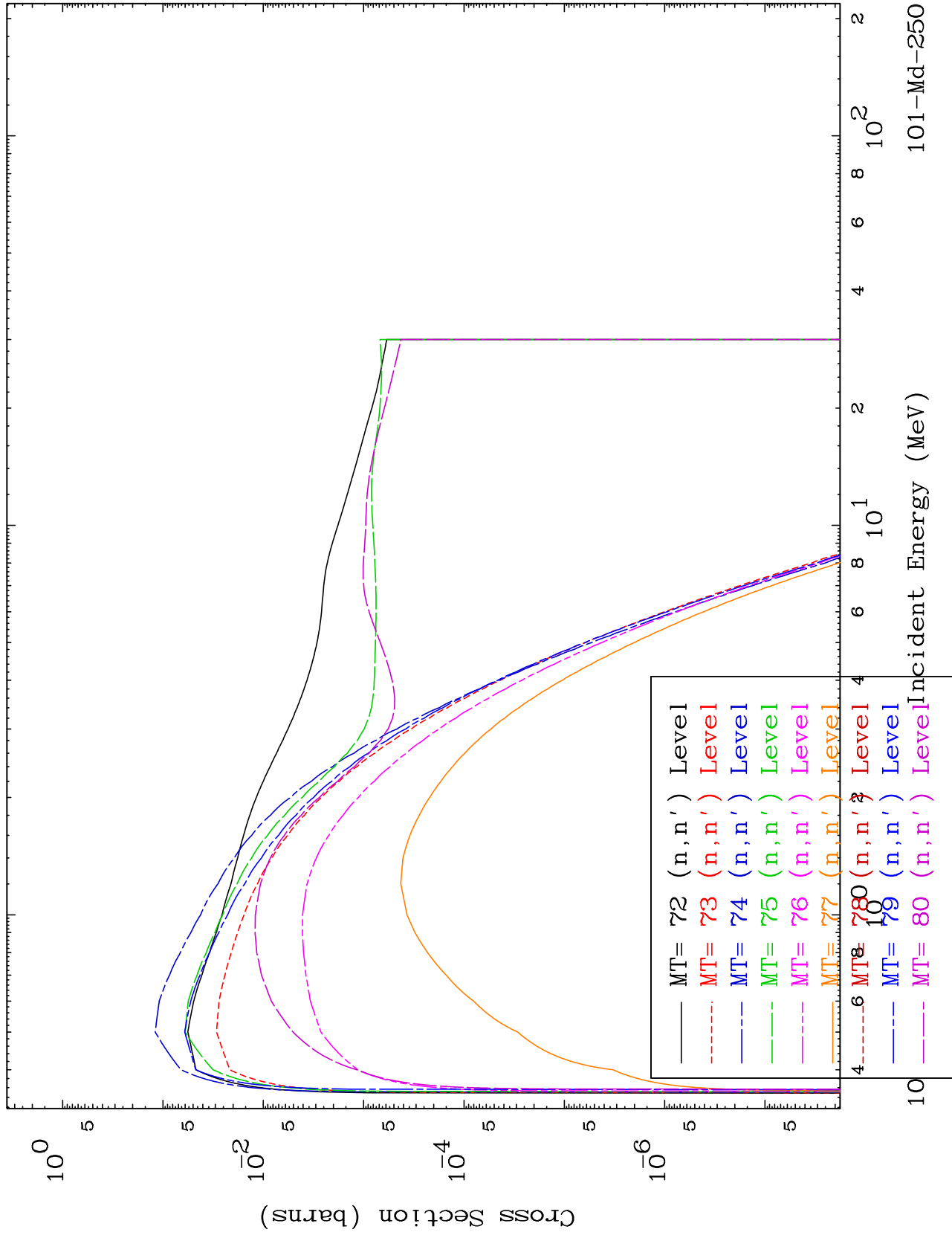
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,n') Level
293 Kelvin Cross Sections

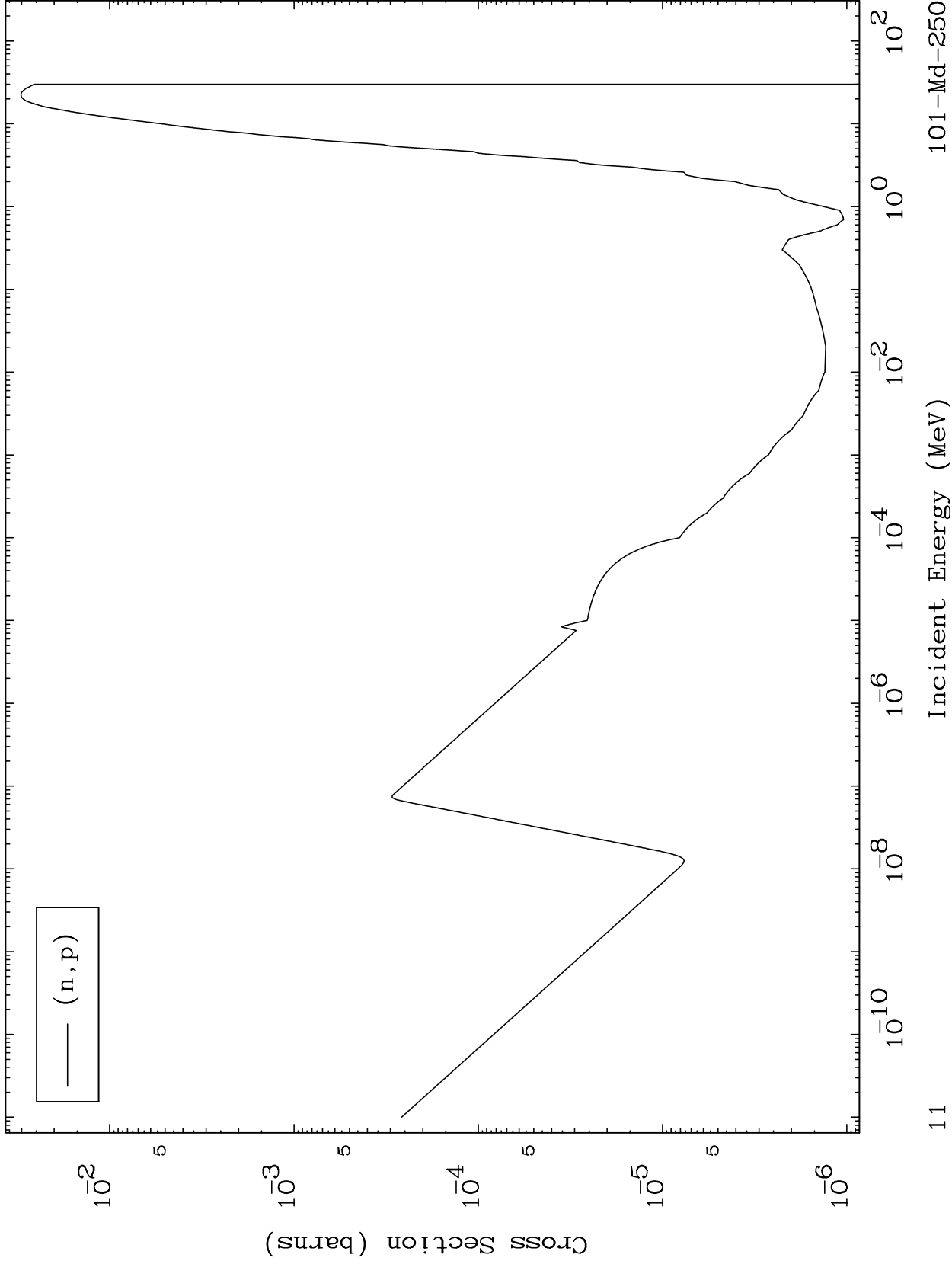
101-Md-250



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(n,p) Levels
293 Kelvin Cross Sections

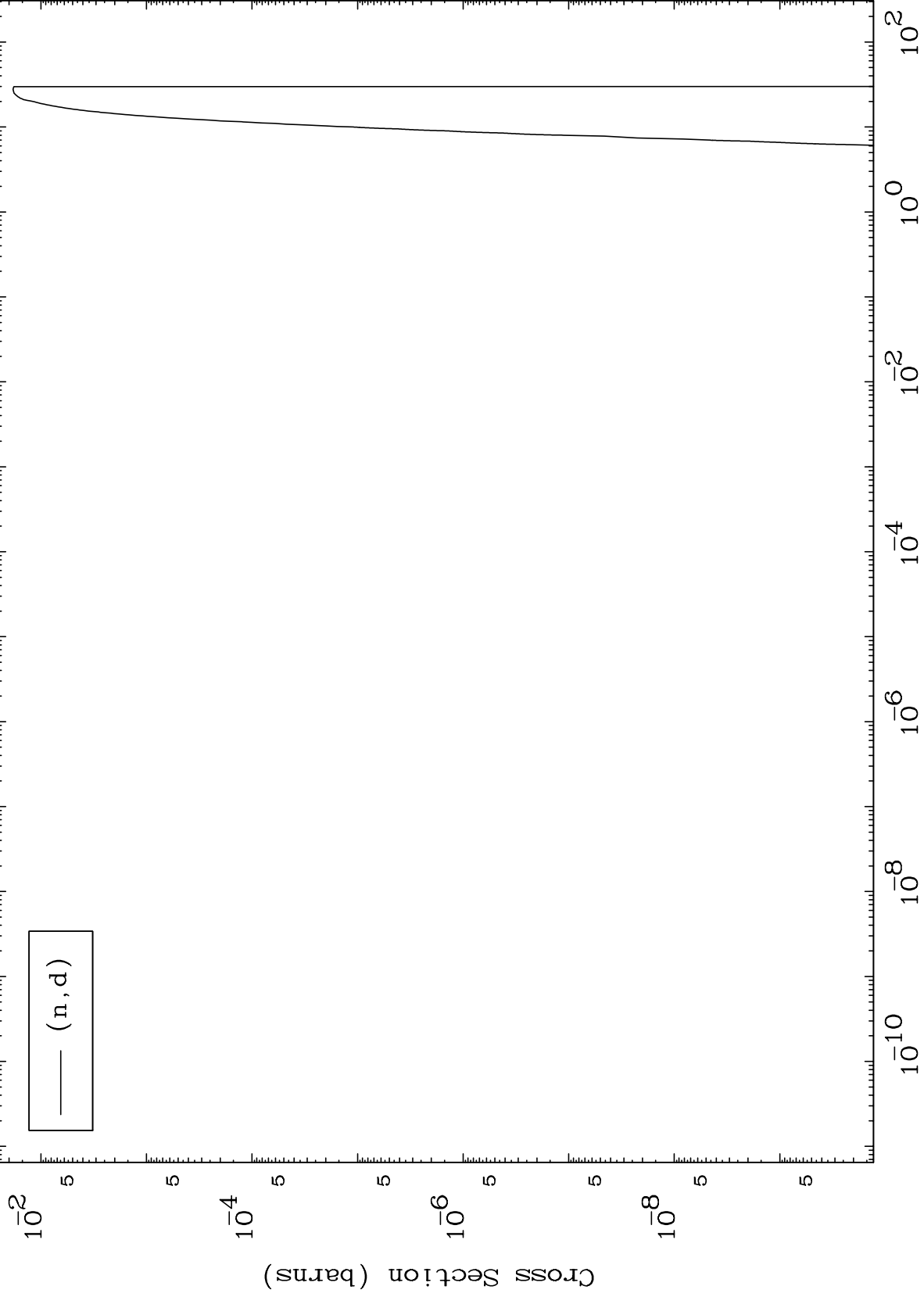
101-Md-250



MAT 9951

(n,d) Levels
293 Kelvin Cross Sections

101-Md-250

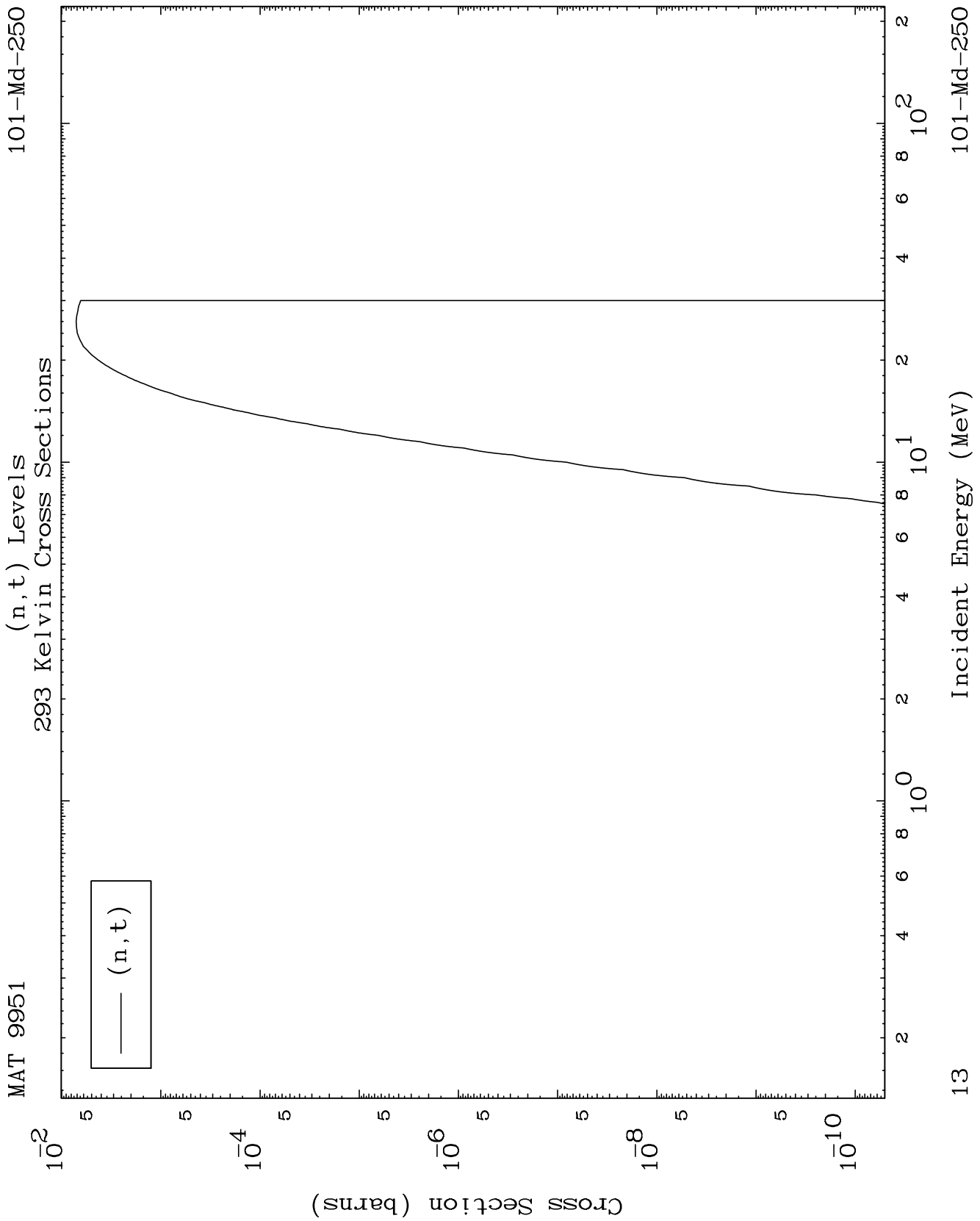


(n,d)

101-Md-250

Incident Energy (MeV)

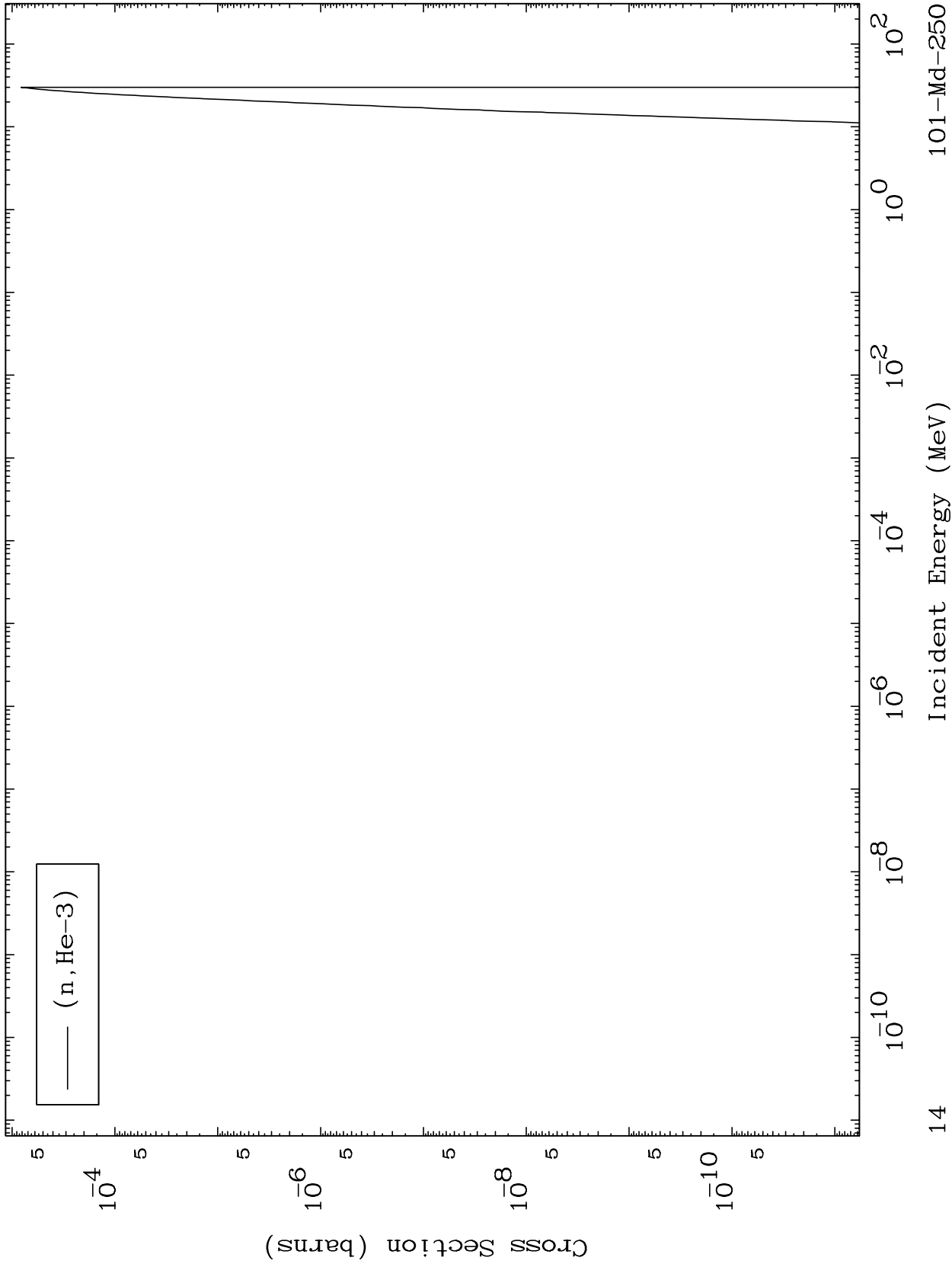
12



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(n,He3) Levels
293 Kelvin Cross Sections

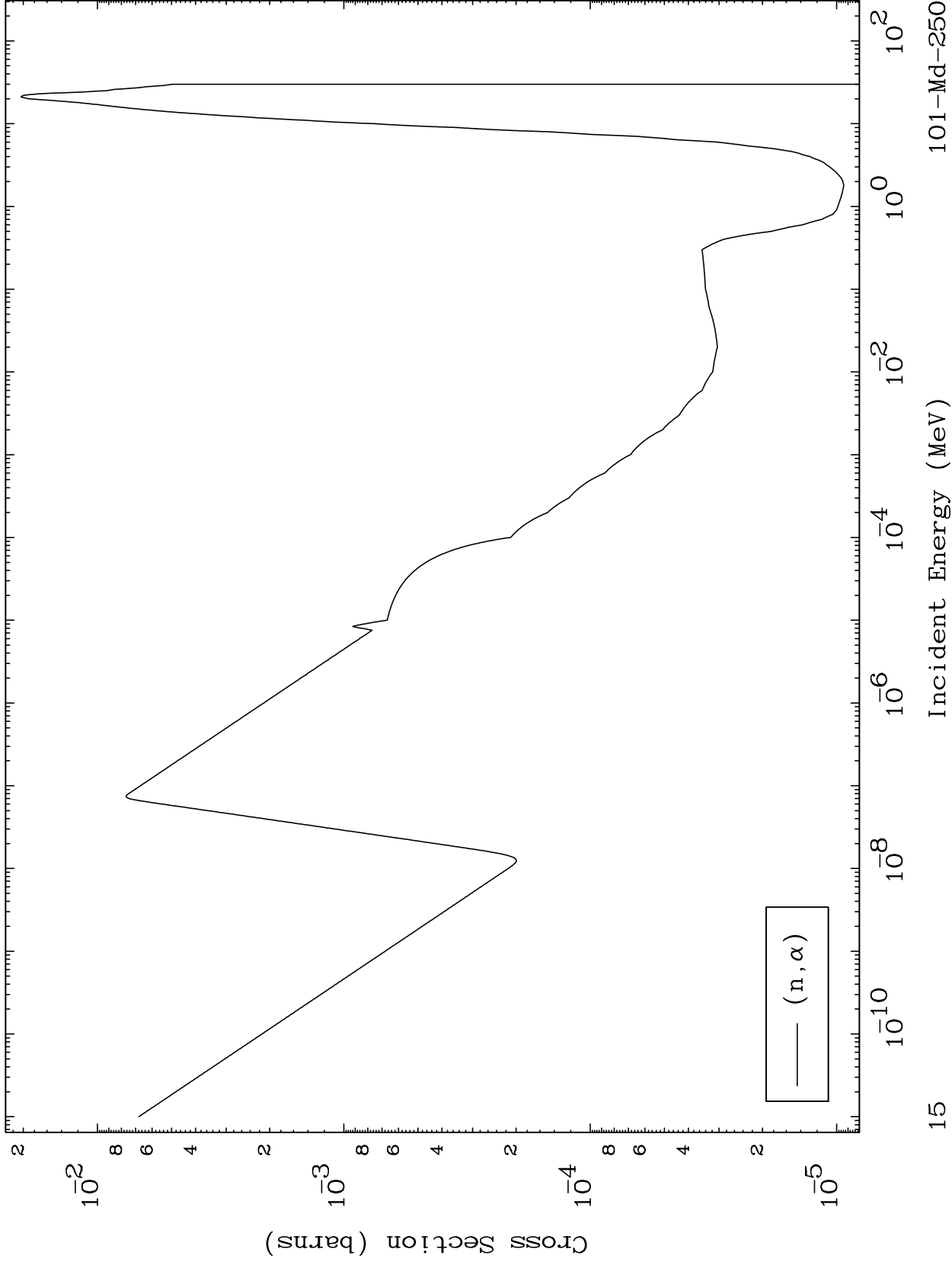
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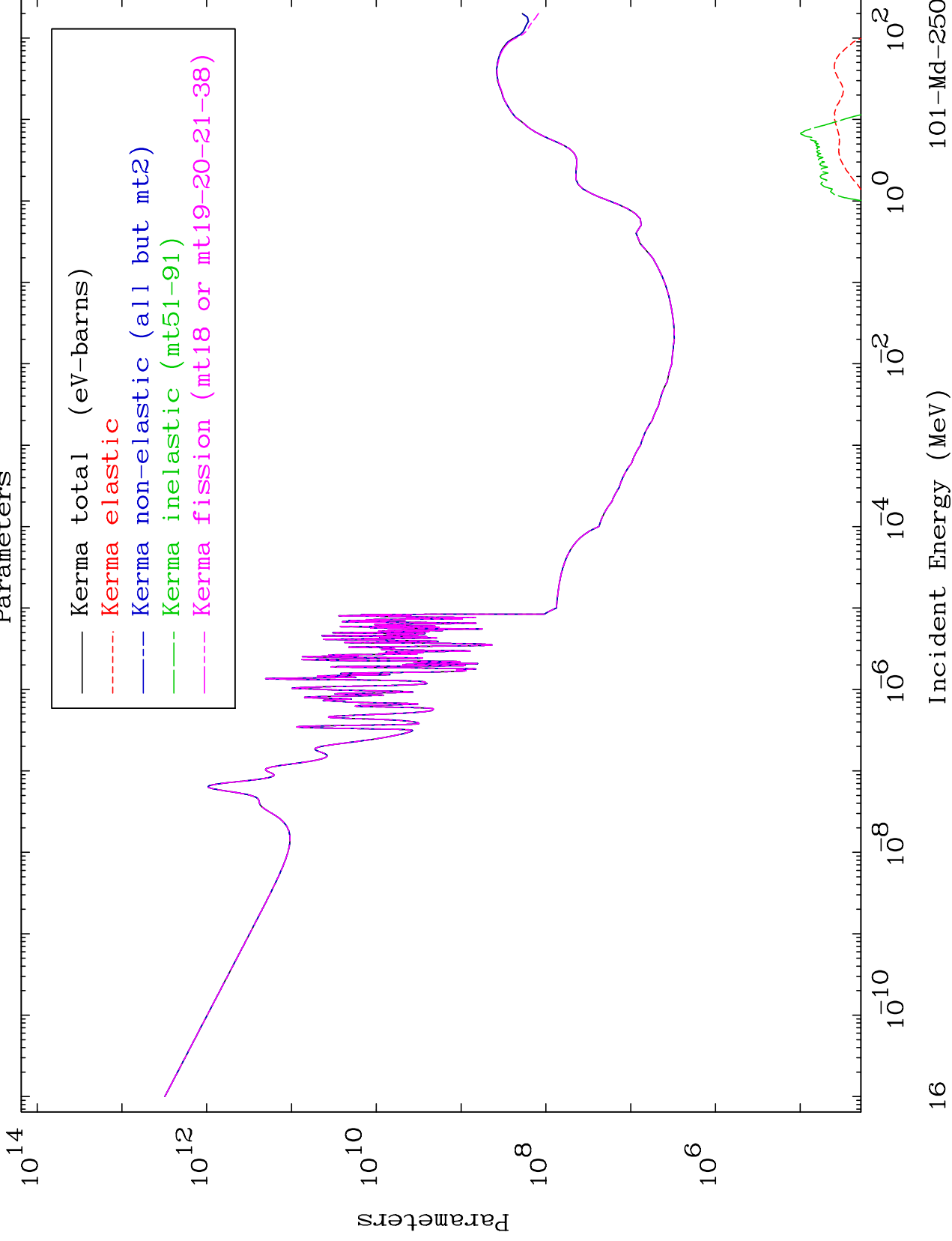


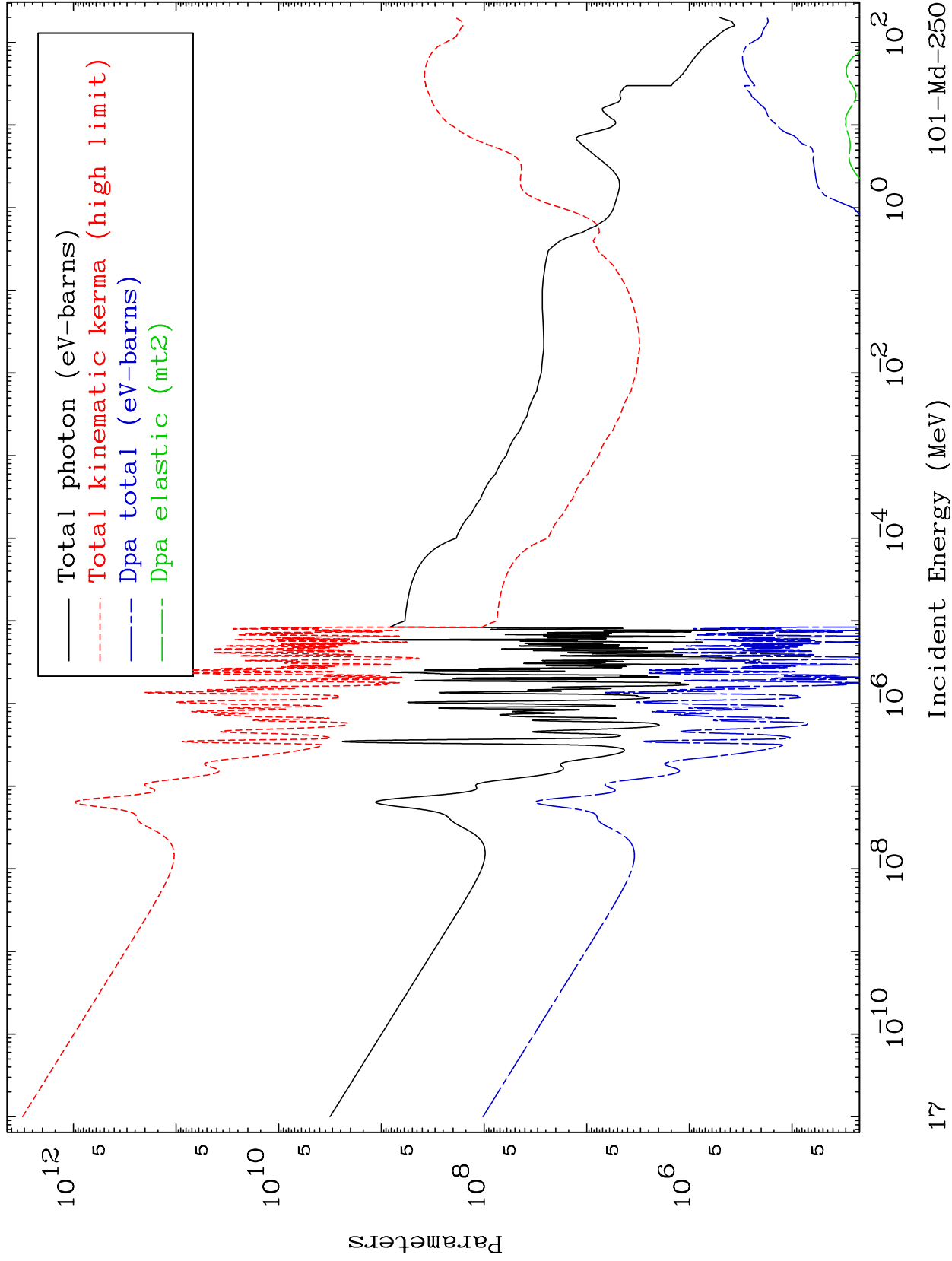
MAT 9951

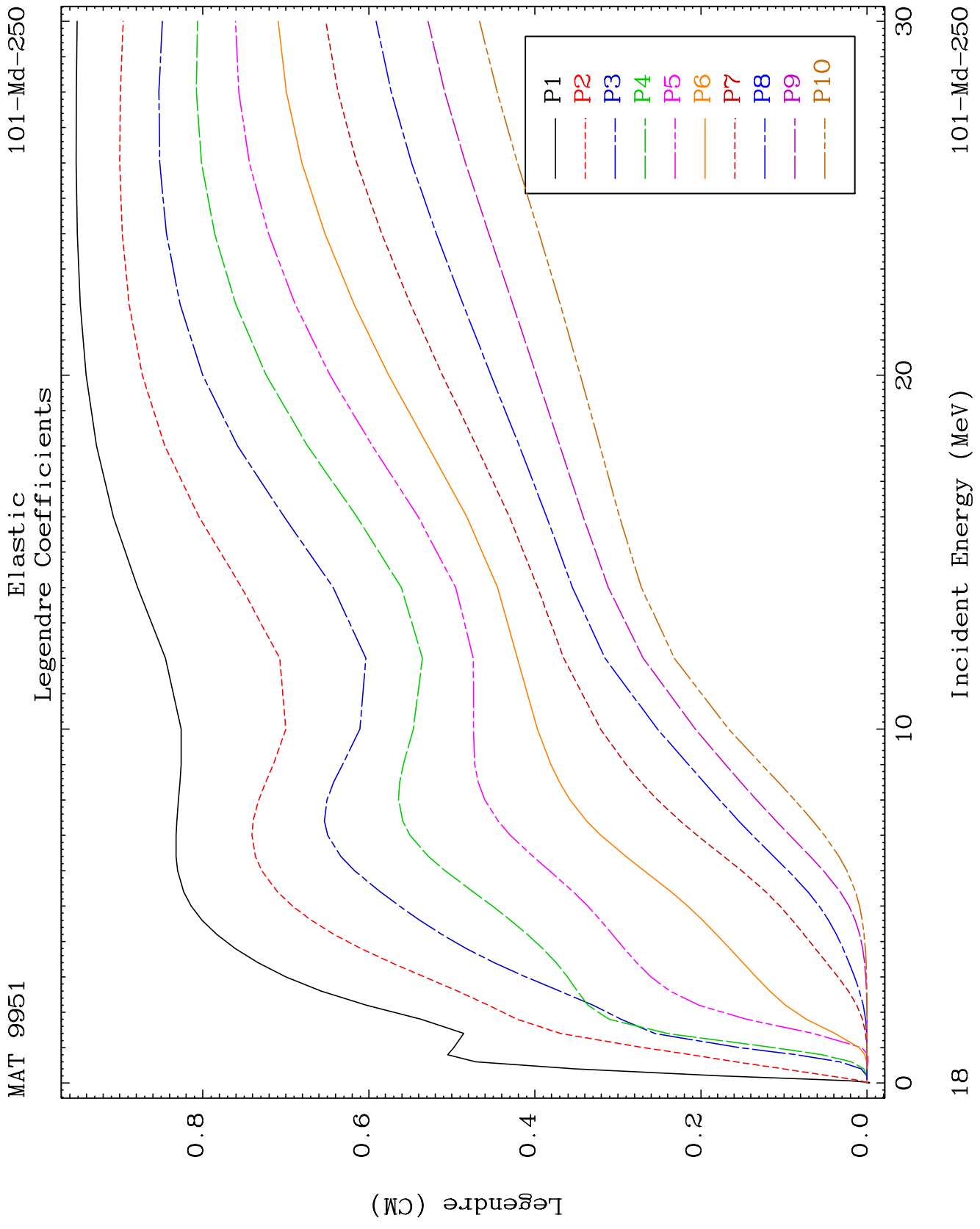
(n,α) Levels
293 Kelvin Cross Sections

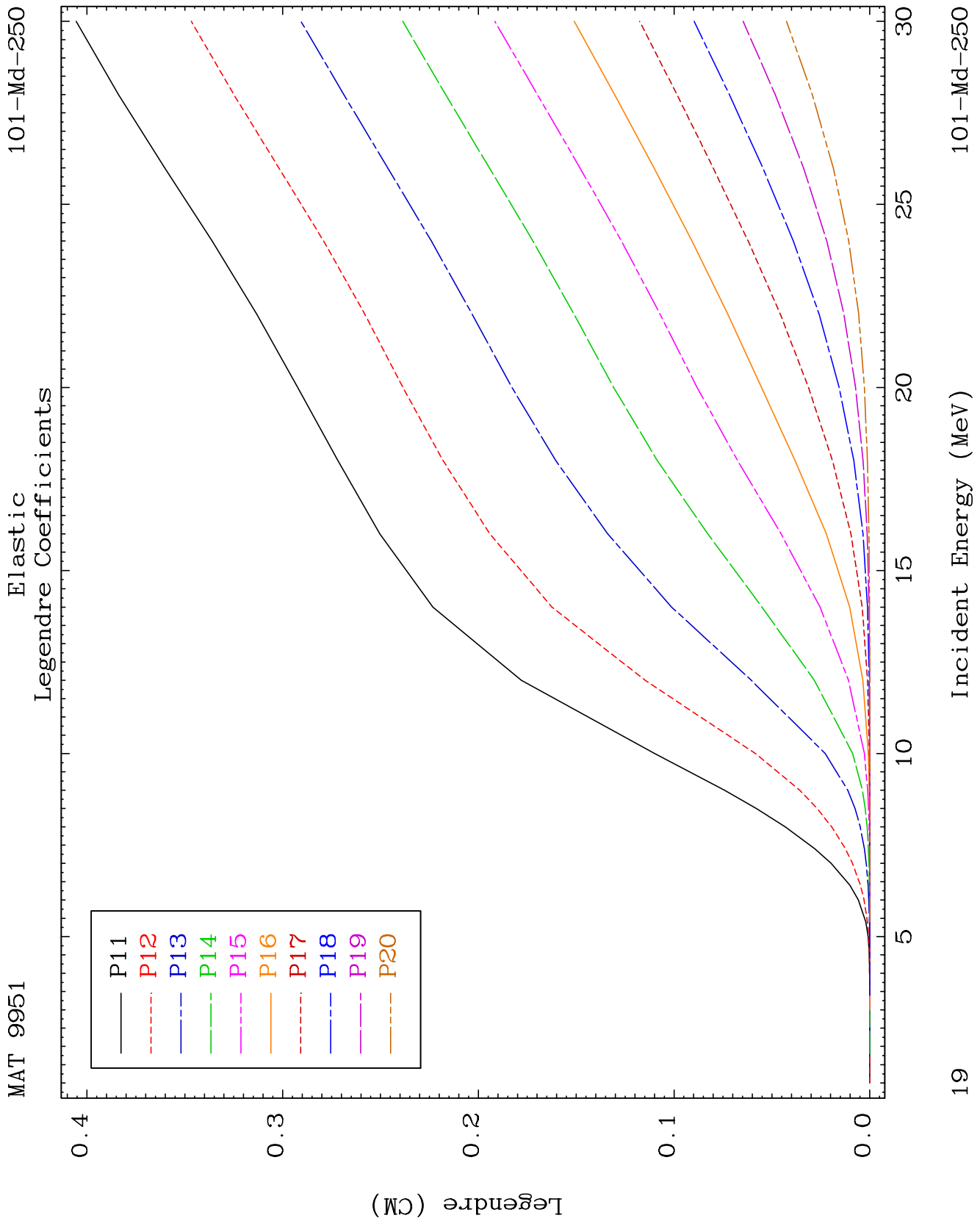
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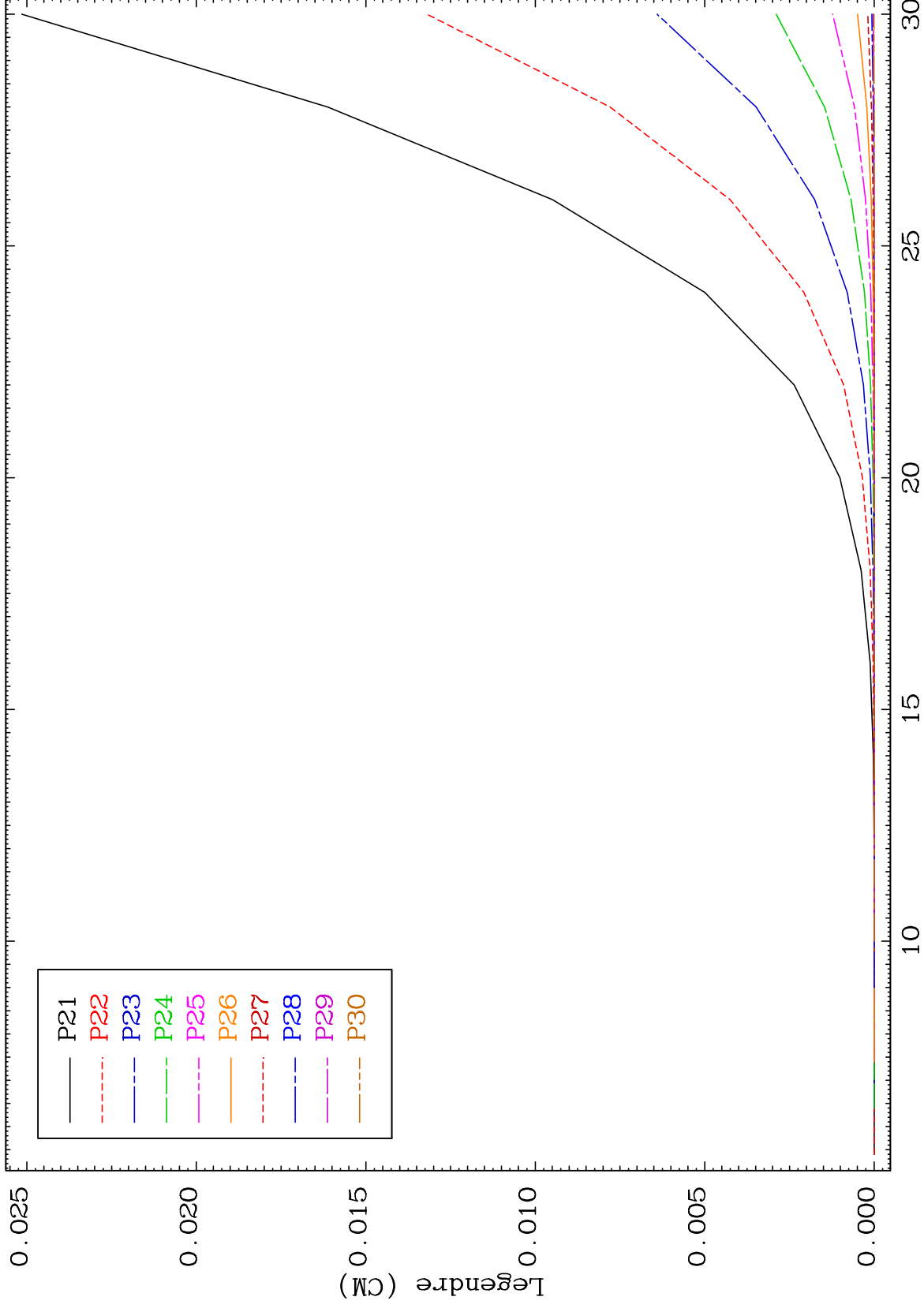




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Elastic
Legendre Coefficients

101-Md-250



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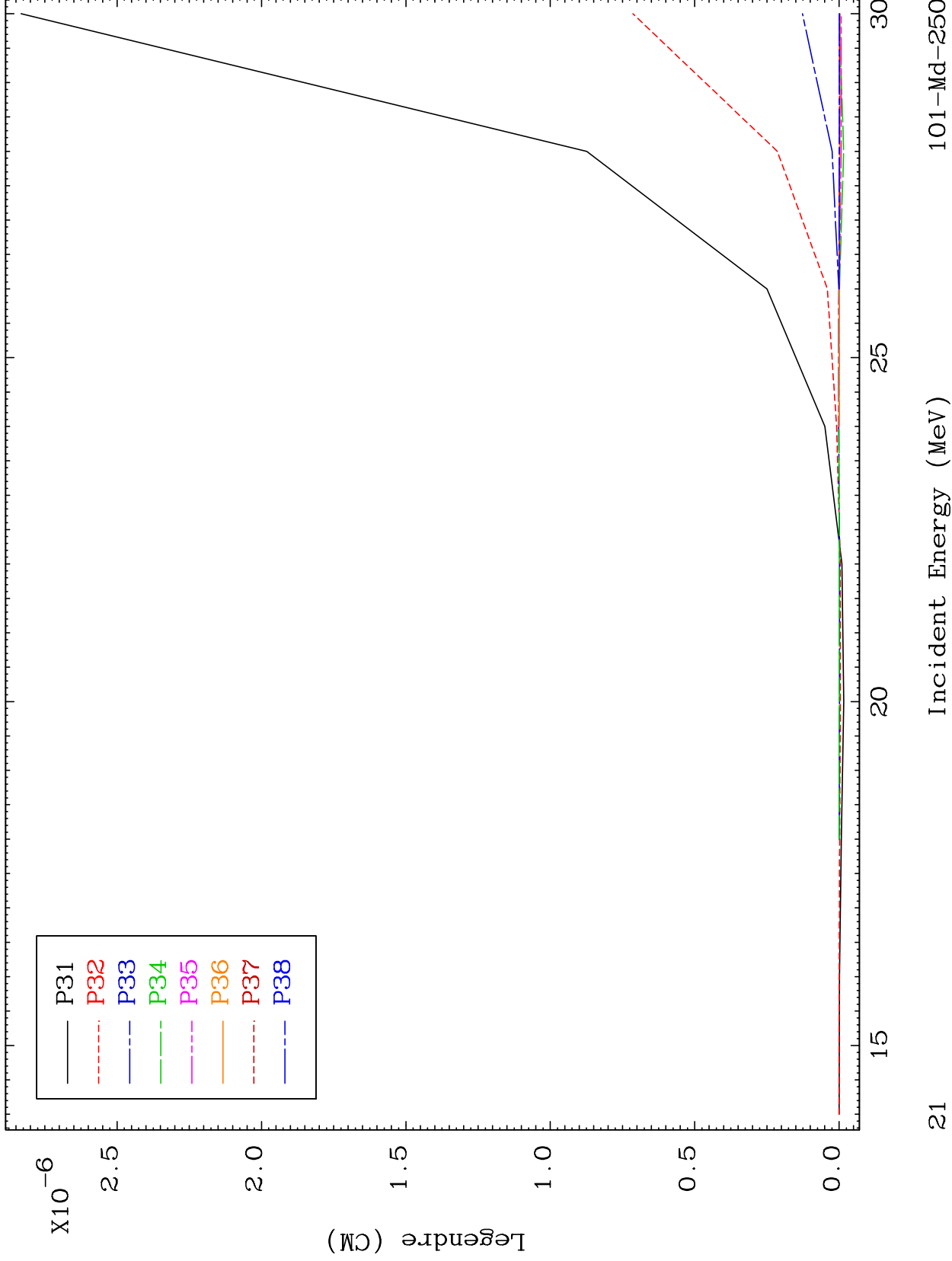
Incident Energy (MeV)

101-Md-250

MAT 9951

Elastic
Legendre Coefficients

101-Md-250

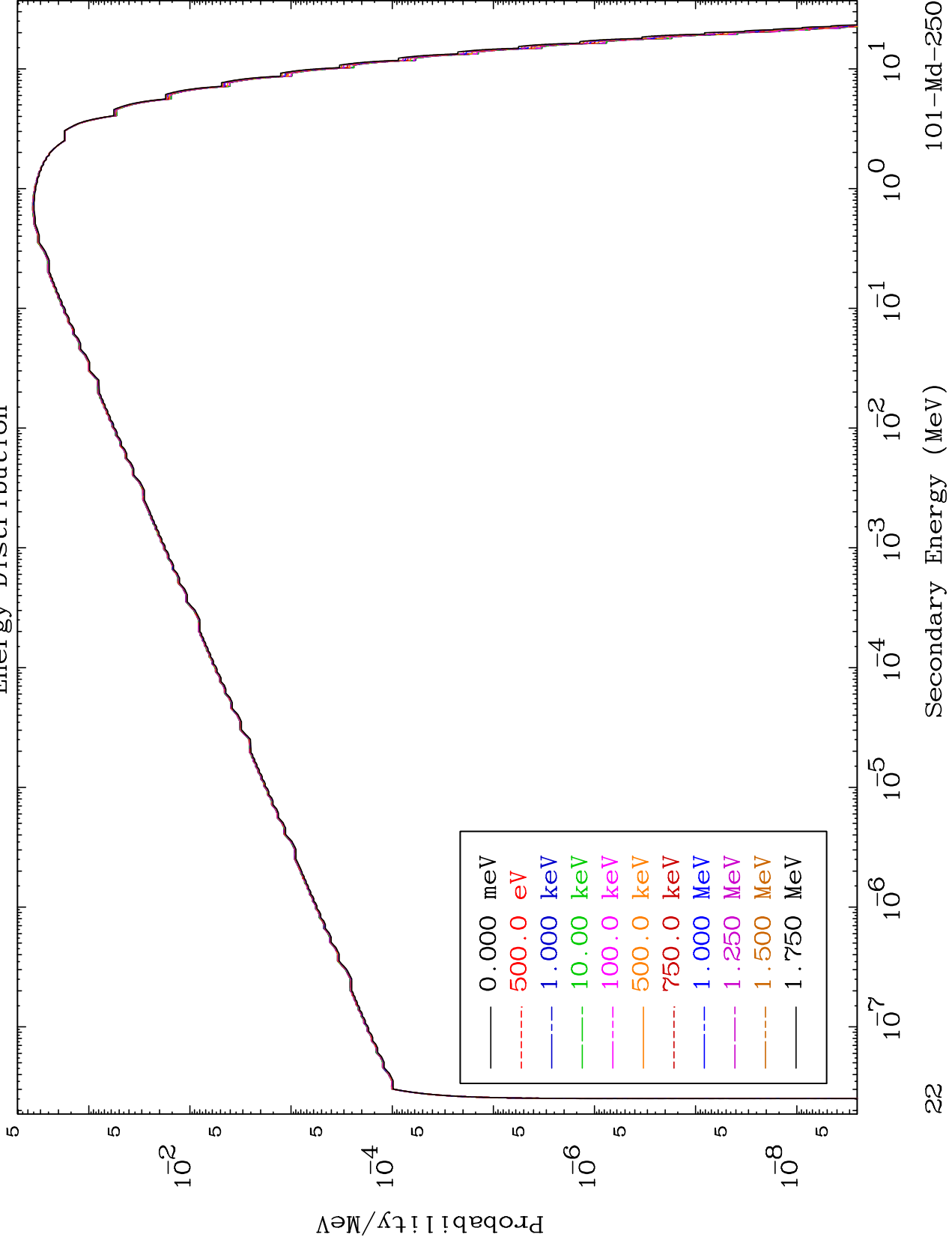


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Fission Energy Distribution

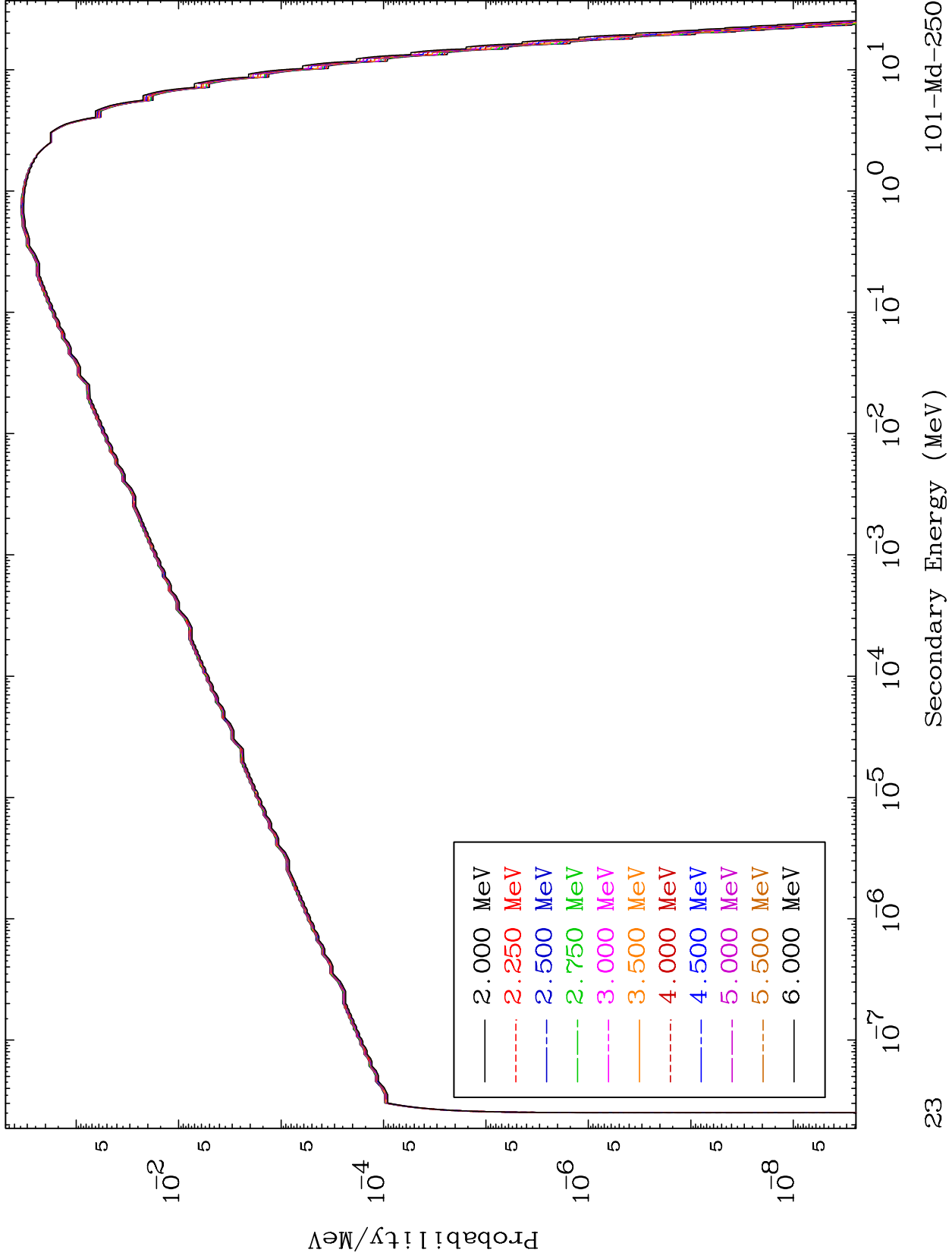
101-Md-250



MAT 9951

Fission Energy Distribution

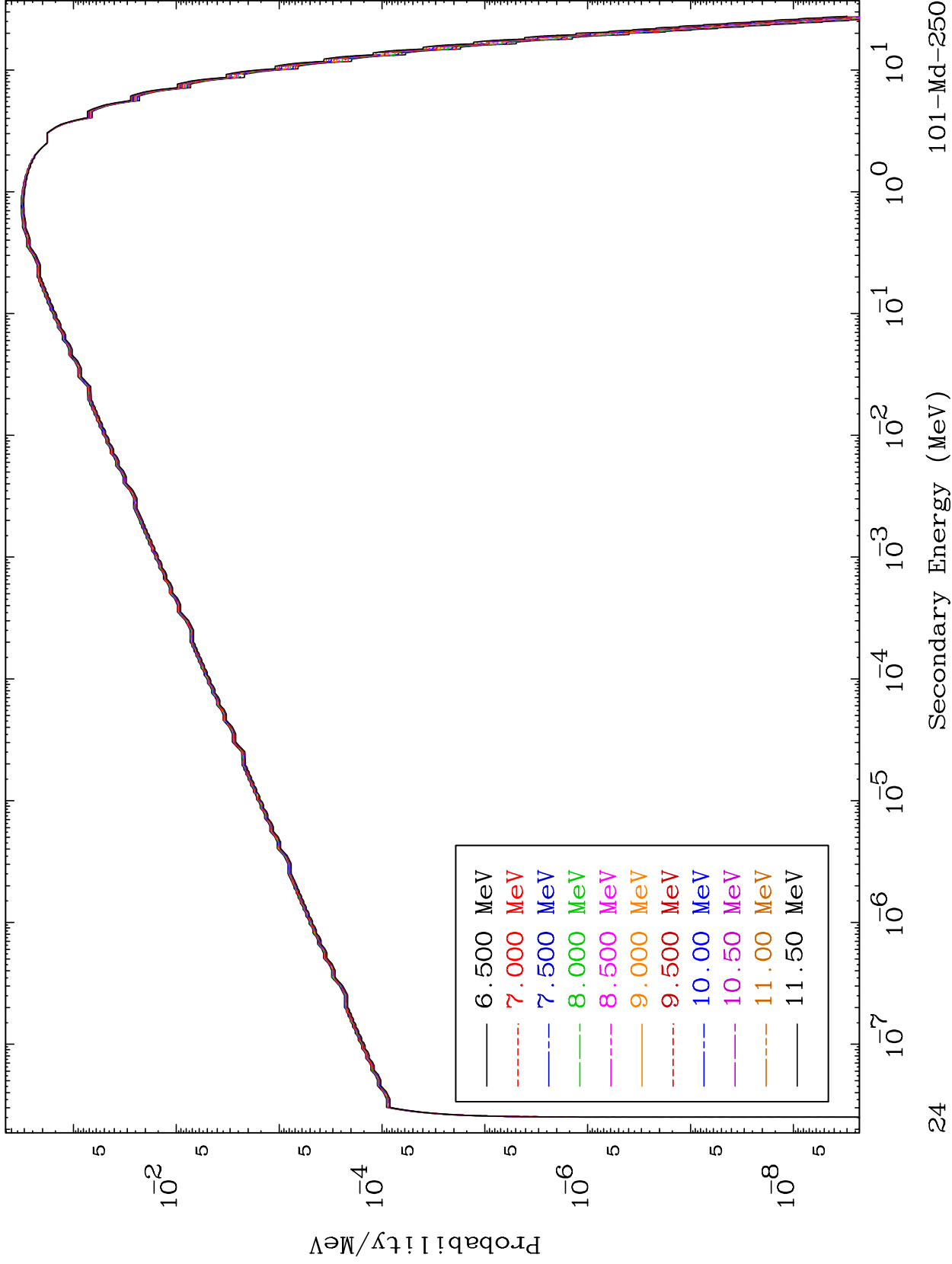
101-Md-250



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Fission Energy Distribution

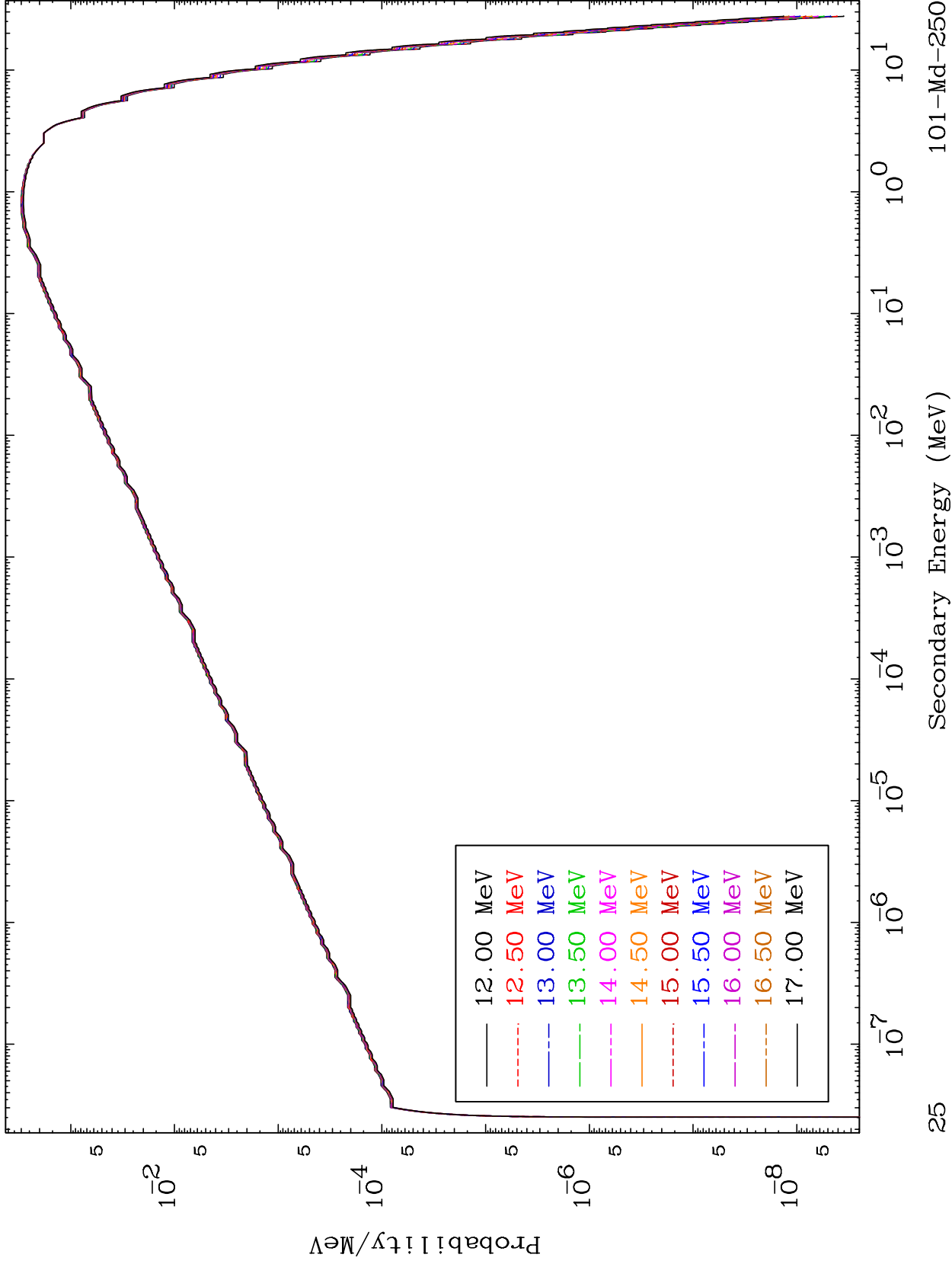
101-Md-250



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Fission Energy Distribution

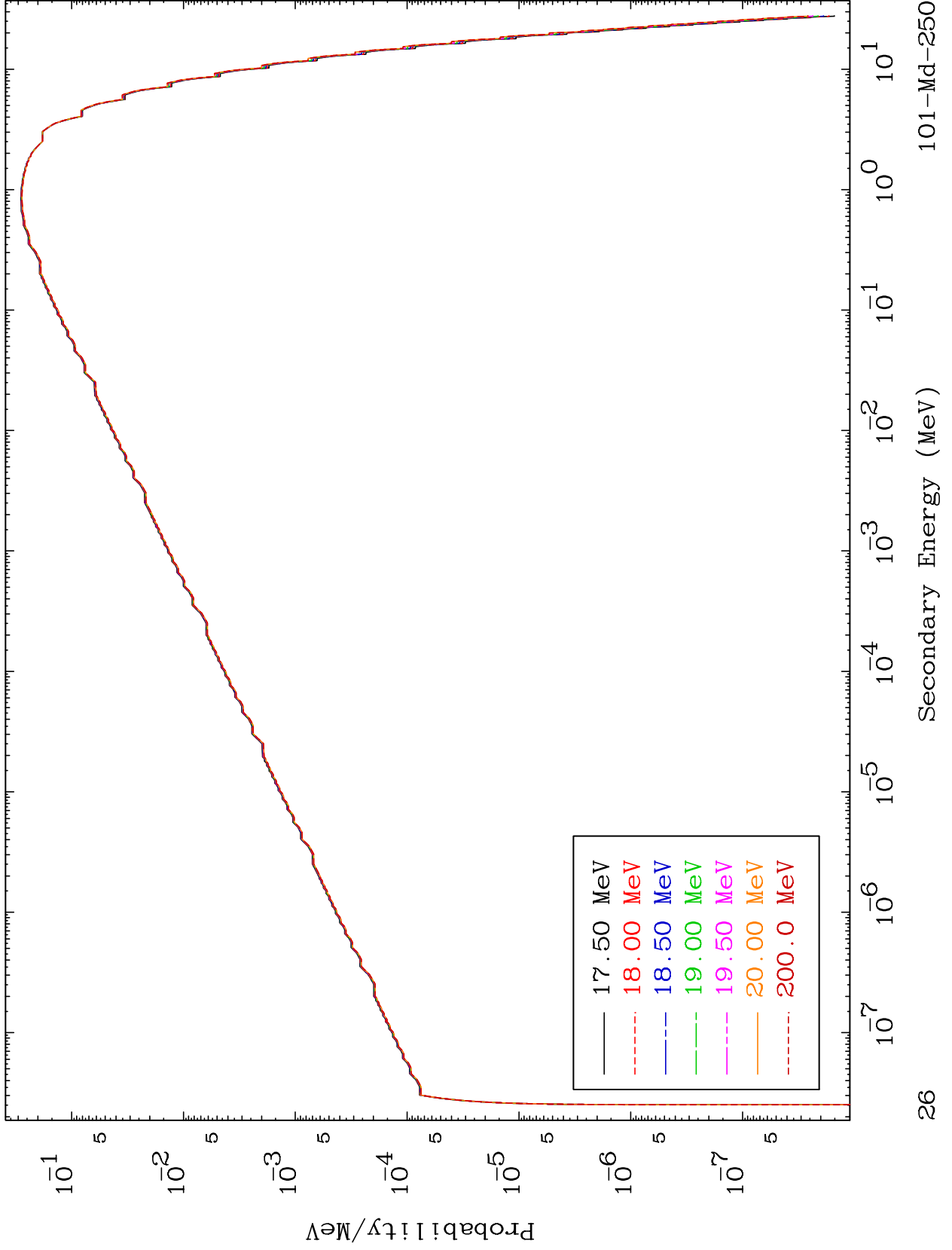
101-Md-250



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Fission Energy Distribution

101-Md-250

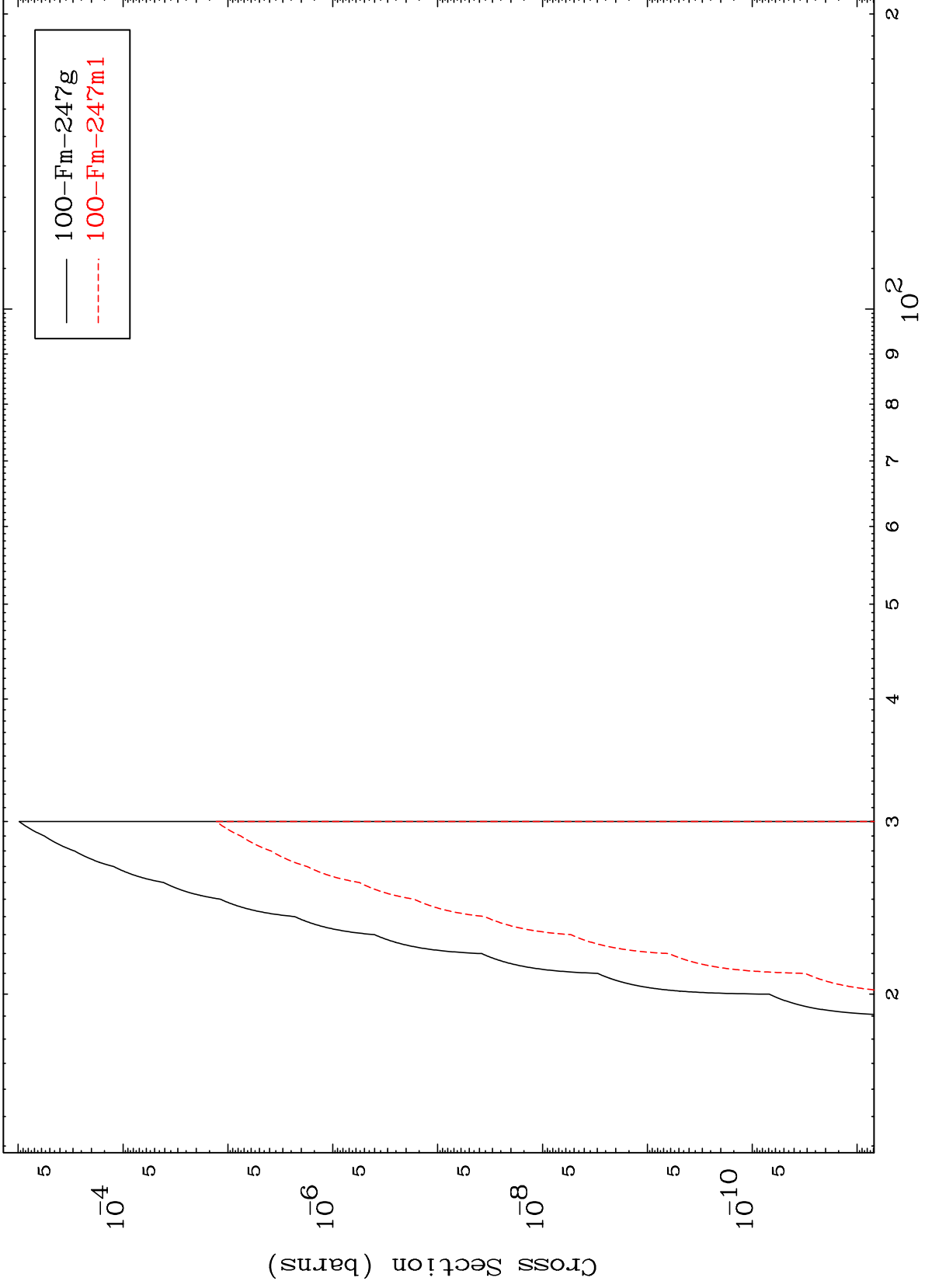


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(n,2n) d

101-Md-250

Radionuclide Production Cross Section



27

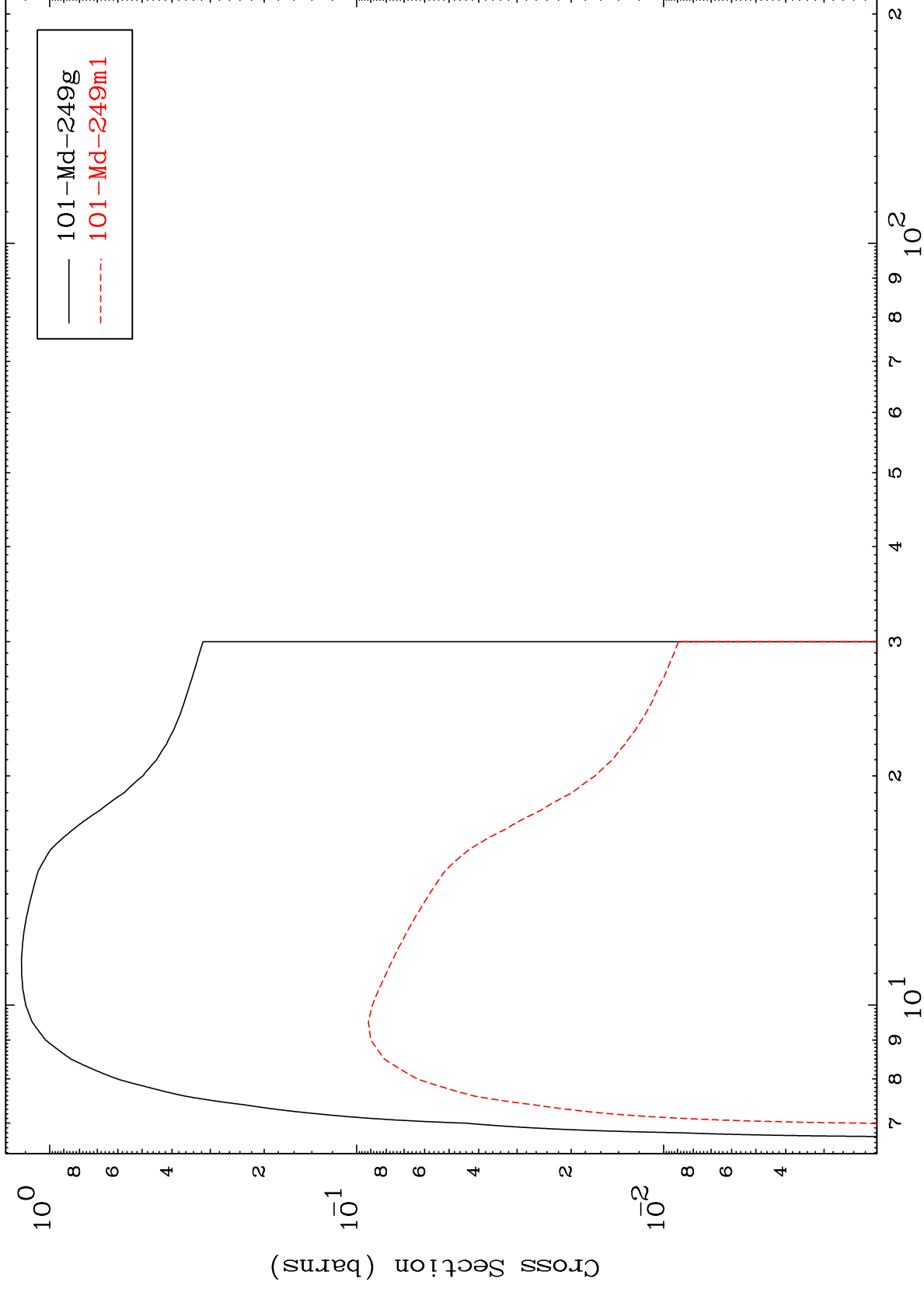
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

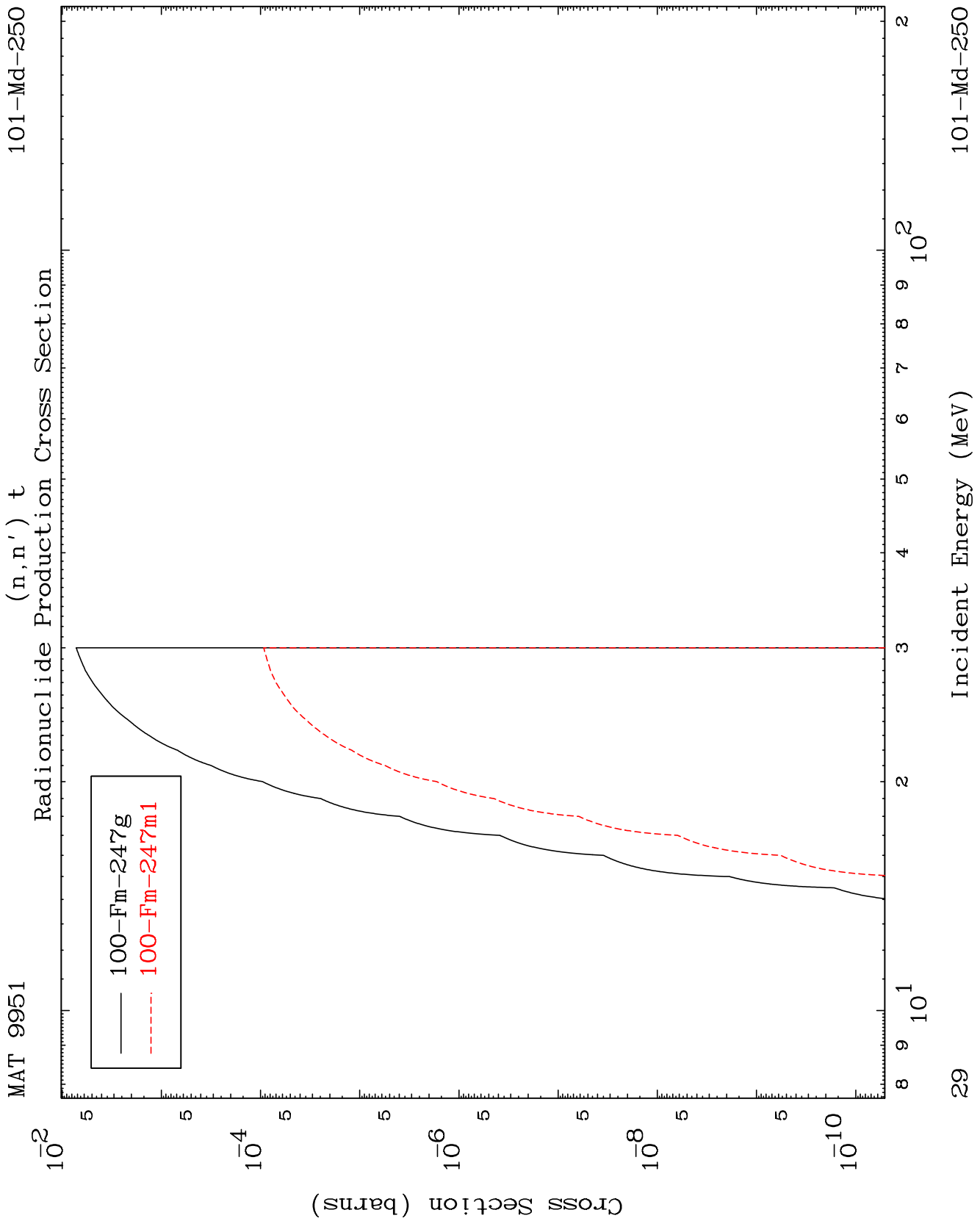
(n,2n)
Radionuclide Production Cross Section



28

Incident Energy (MeV)

101-Md-250

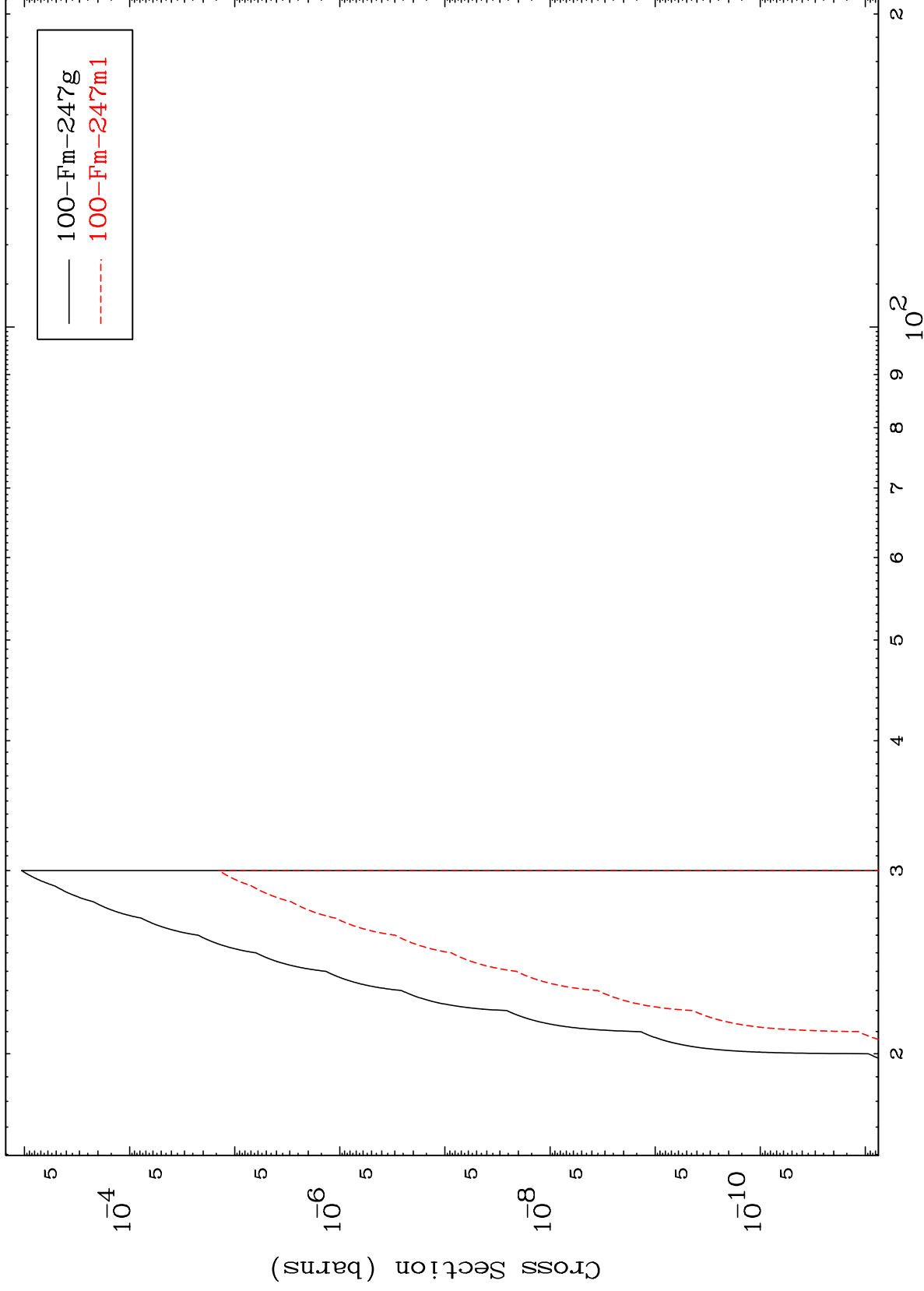


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(n,3n) p

101-Md-250

Radionuclide Production Cross Section



30

Incident Energy (MeV)

101-Md-250

MAT 9951

(n,p)
Radionuclide Production Cross Section

101-Md-250

