

Program EVALPLOT
(Version 2017-1)

by

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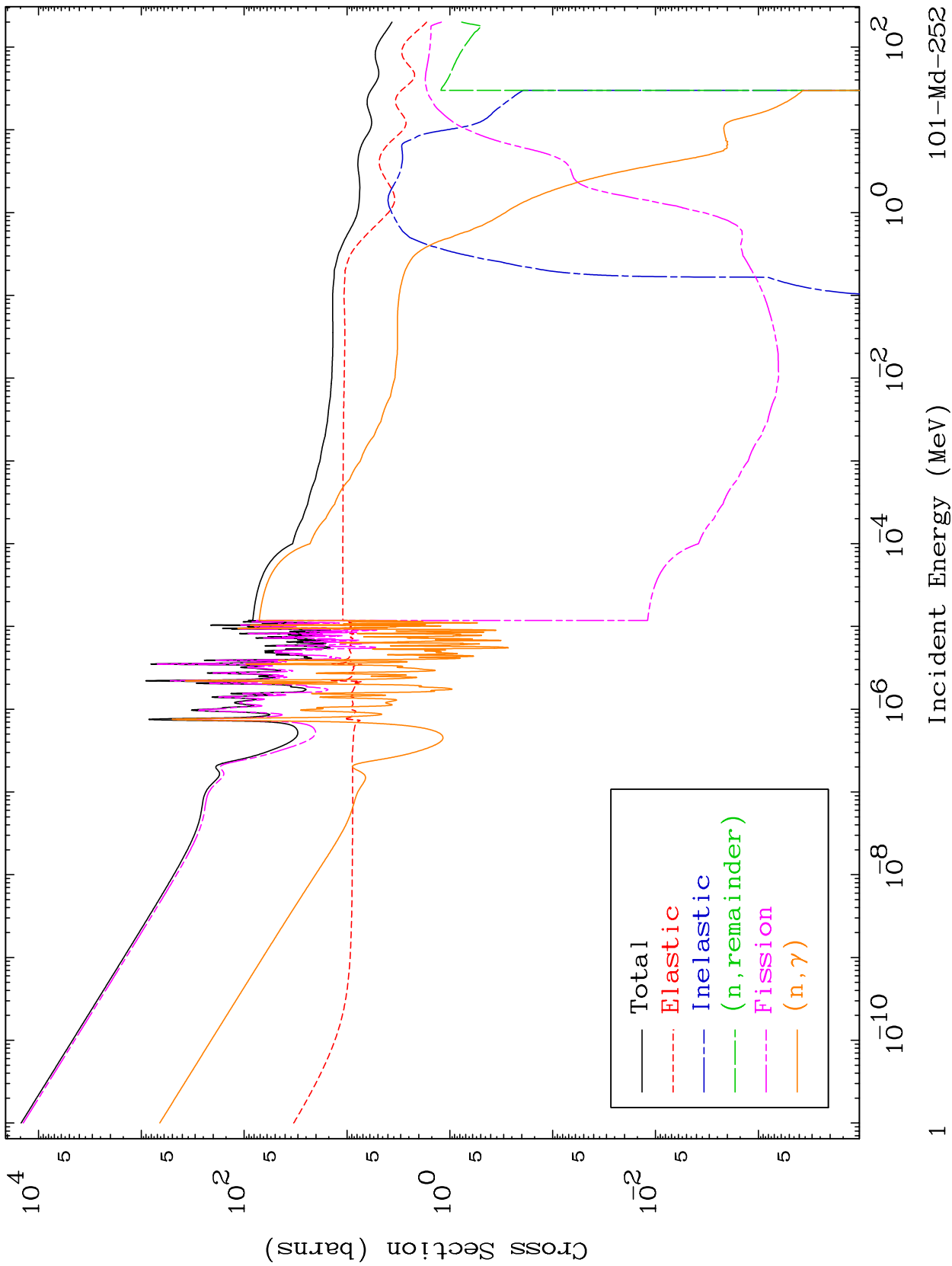
Press Mouse Button to Start

MAT 9953

Major

293 Kelvin Cross Sections

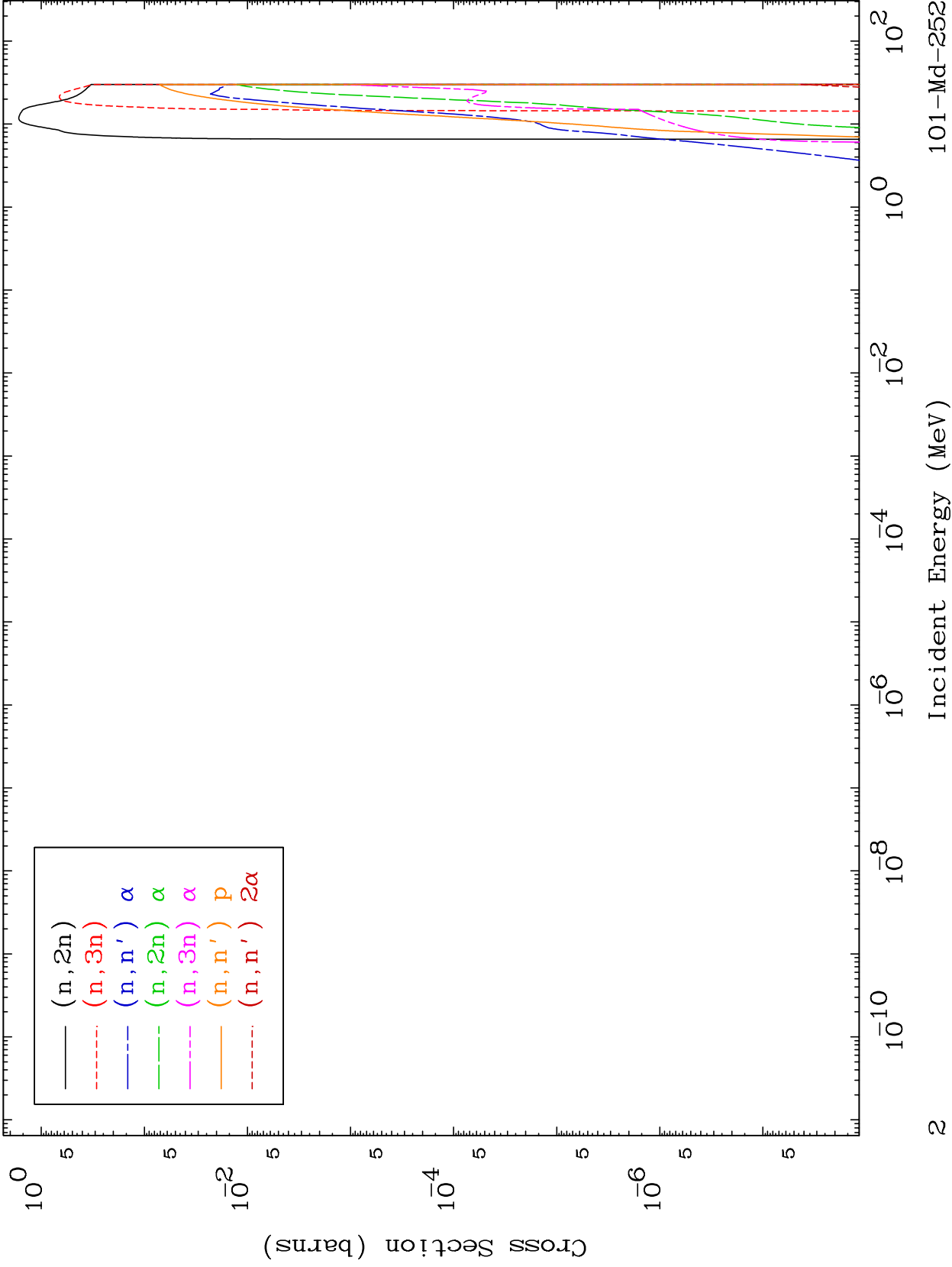
101-Md-252



MAT 9953

Neutron Production
293 Kelvin Cross Sections

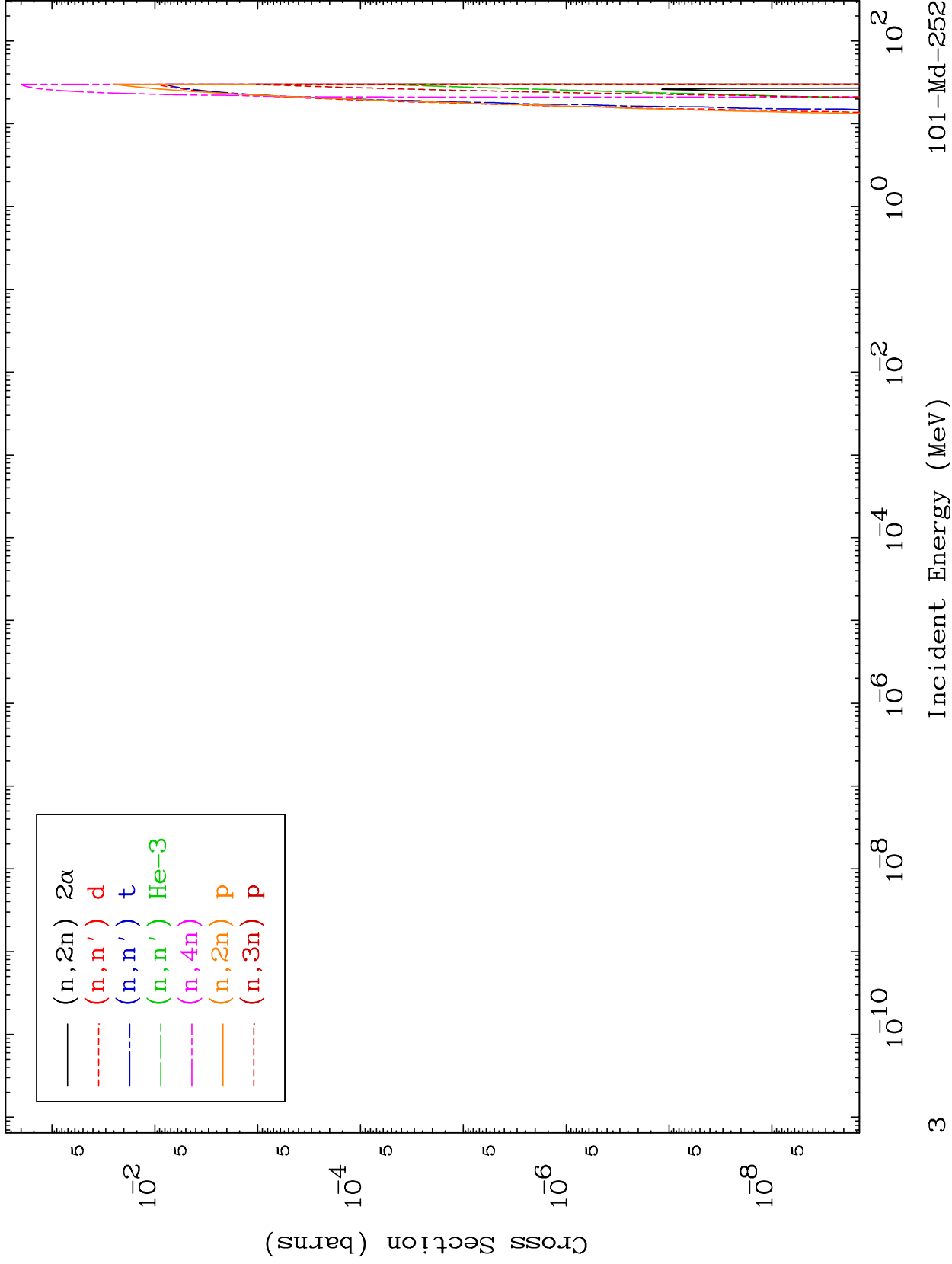
101-Md-252



MAT 9953

Neutron Production
293 Kelvin Cross Sections

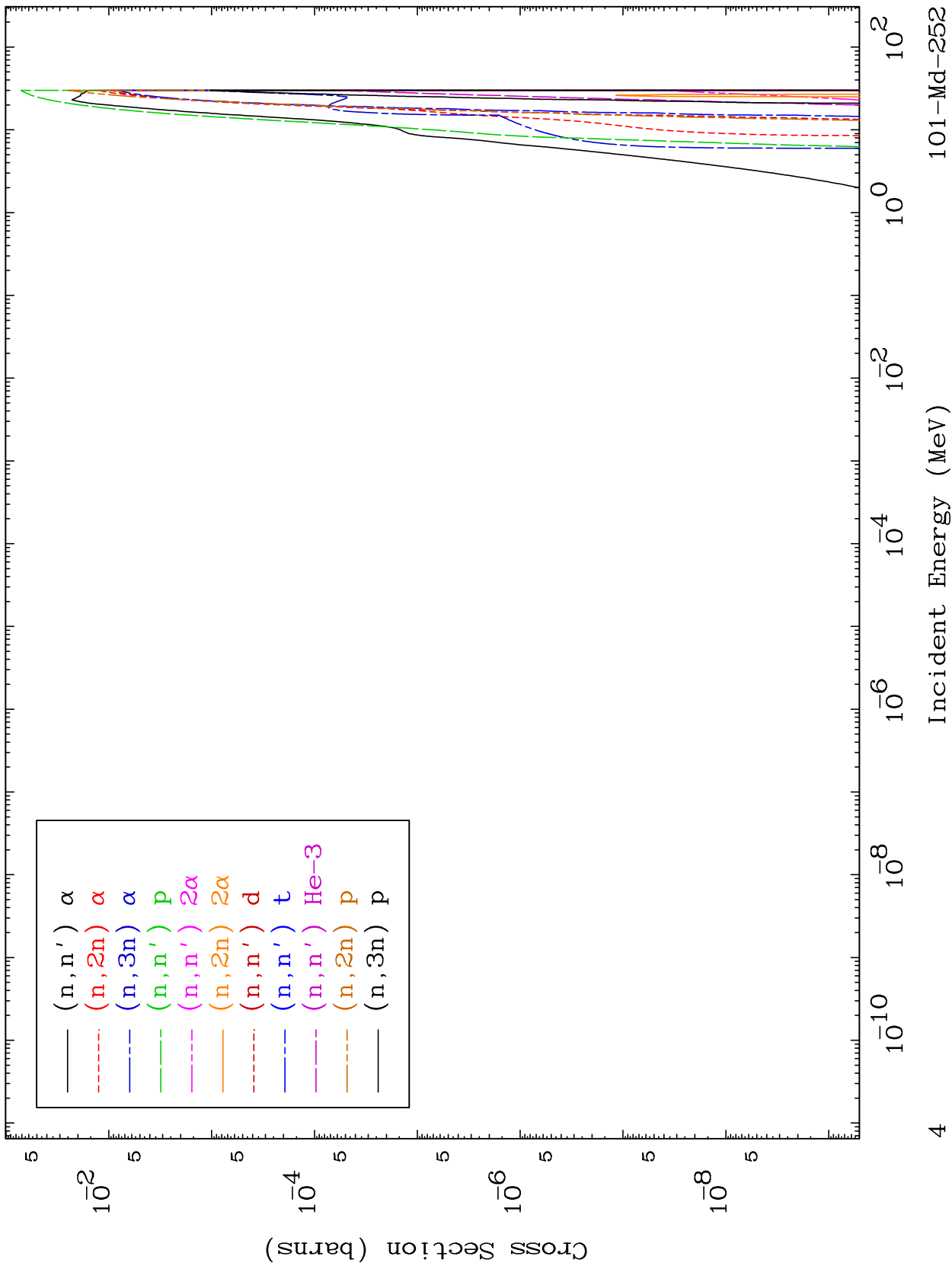
101-Md-252



MAT 9953

Charged Particle
293 Kelvin Cross Sections

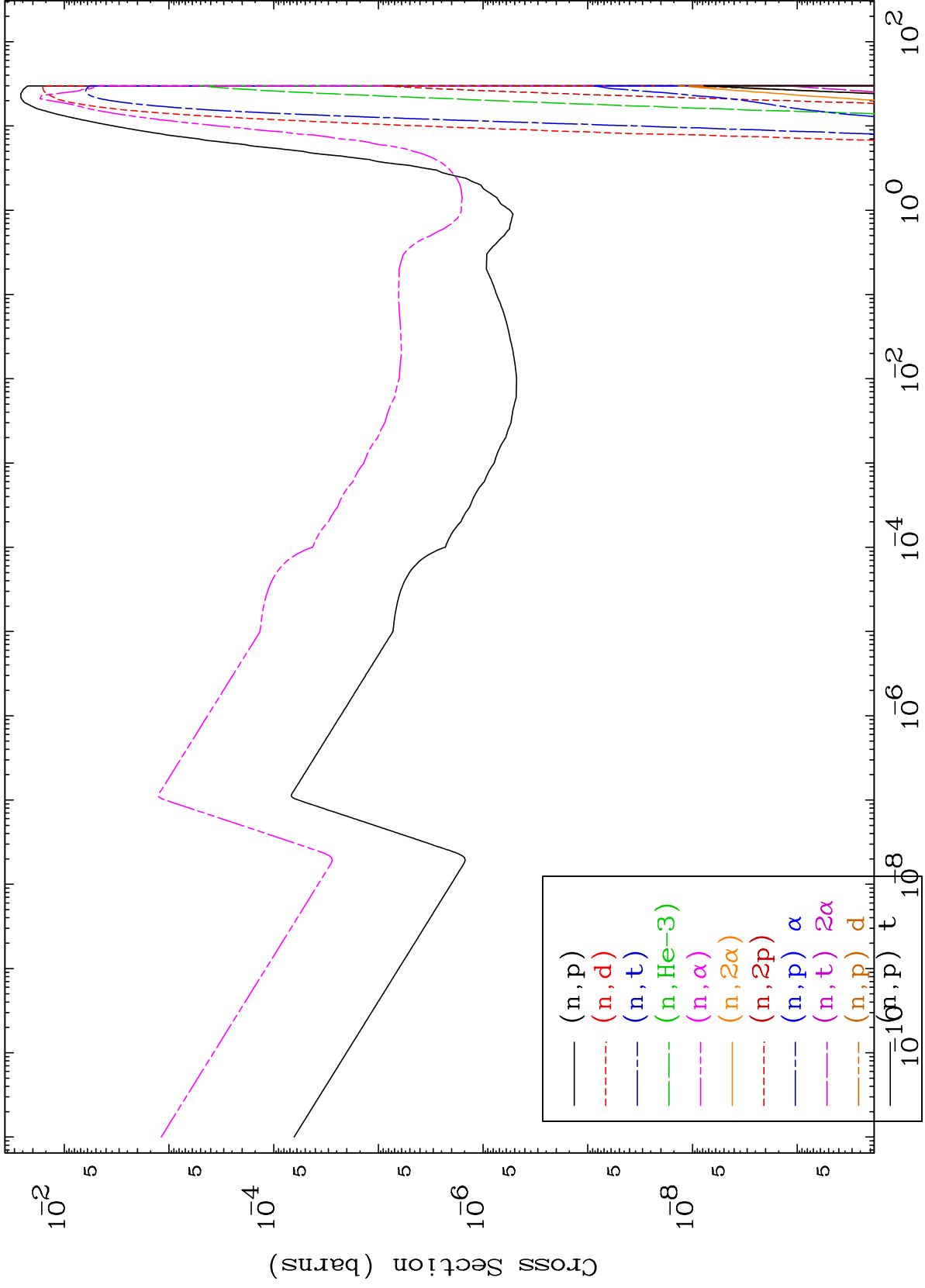
101-Md-252



MAT 9953

Charged Particle
293 Kelvin Cross Sections

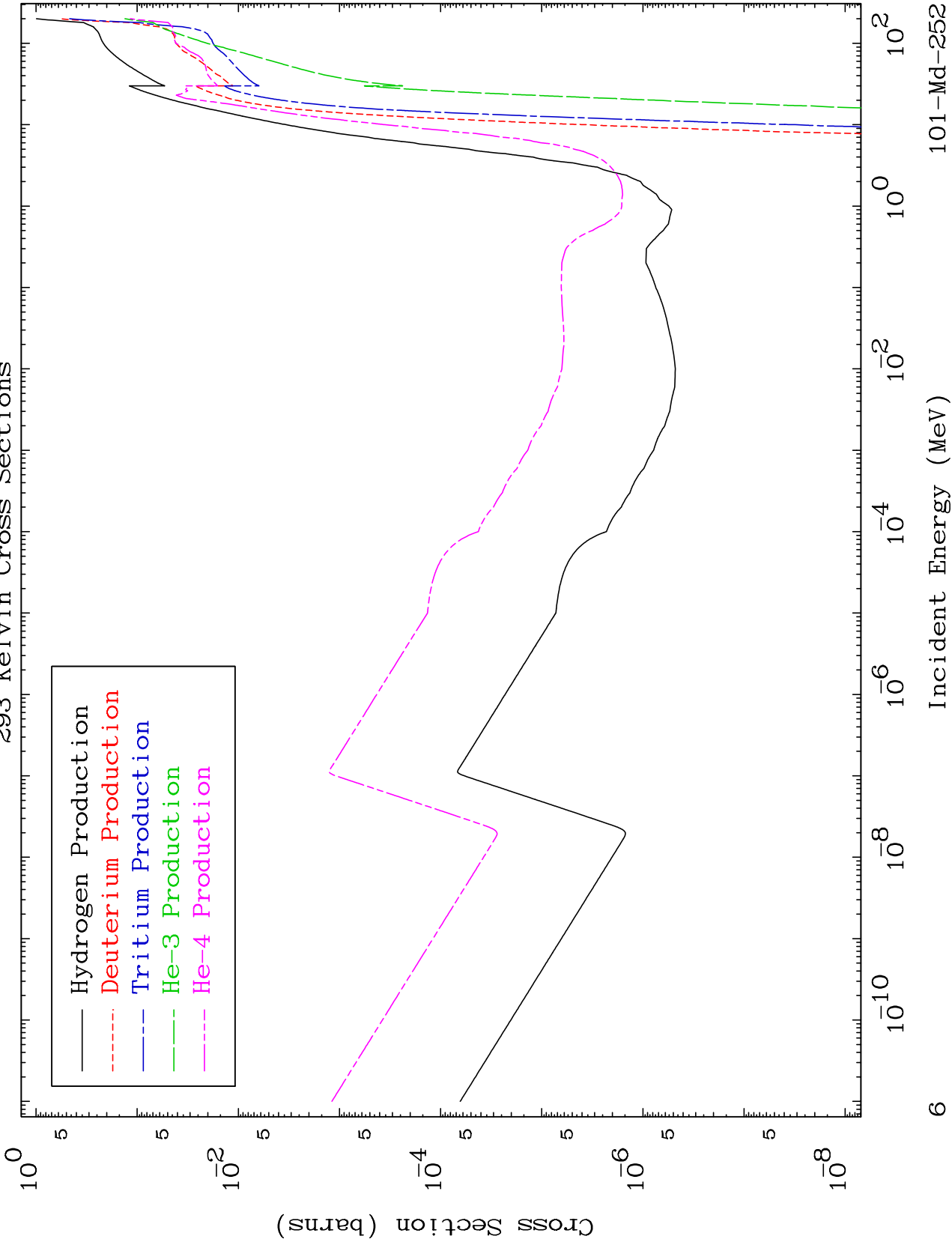
101-Md-252



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Particle Production
293 Kelvin Cross Sections

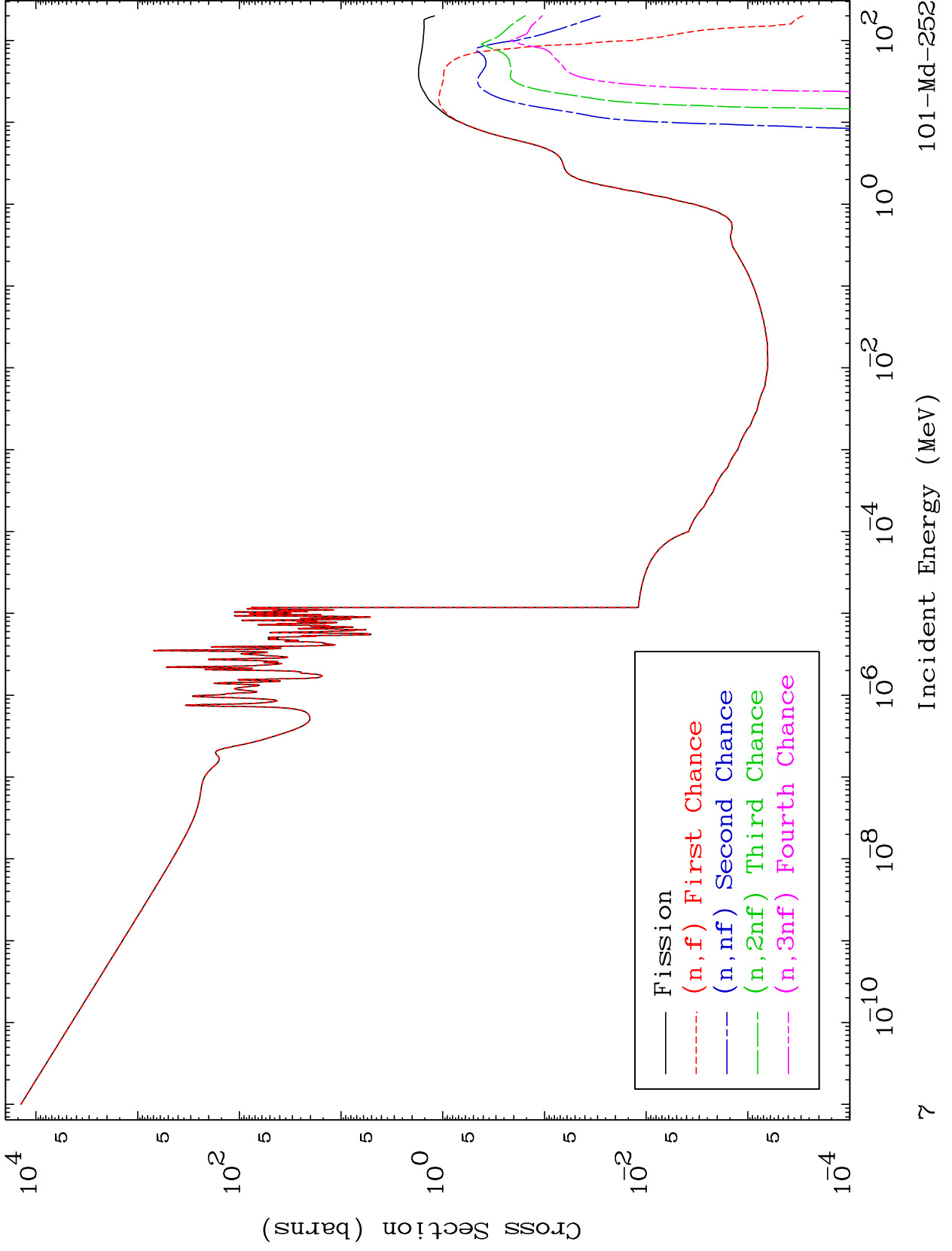
101-Md-252



MAT 9953

Fission
293 Kelvin Cross Sections

101-Md-252

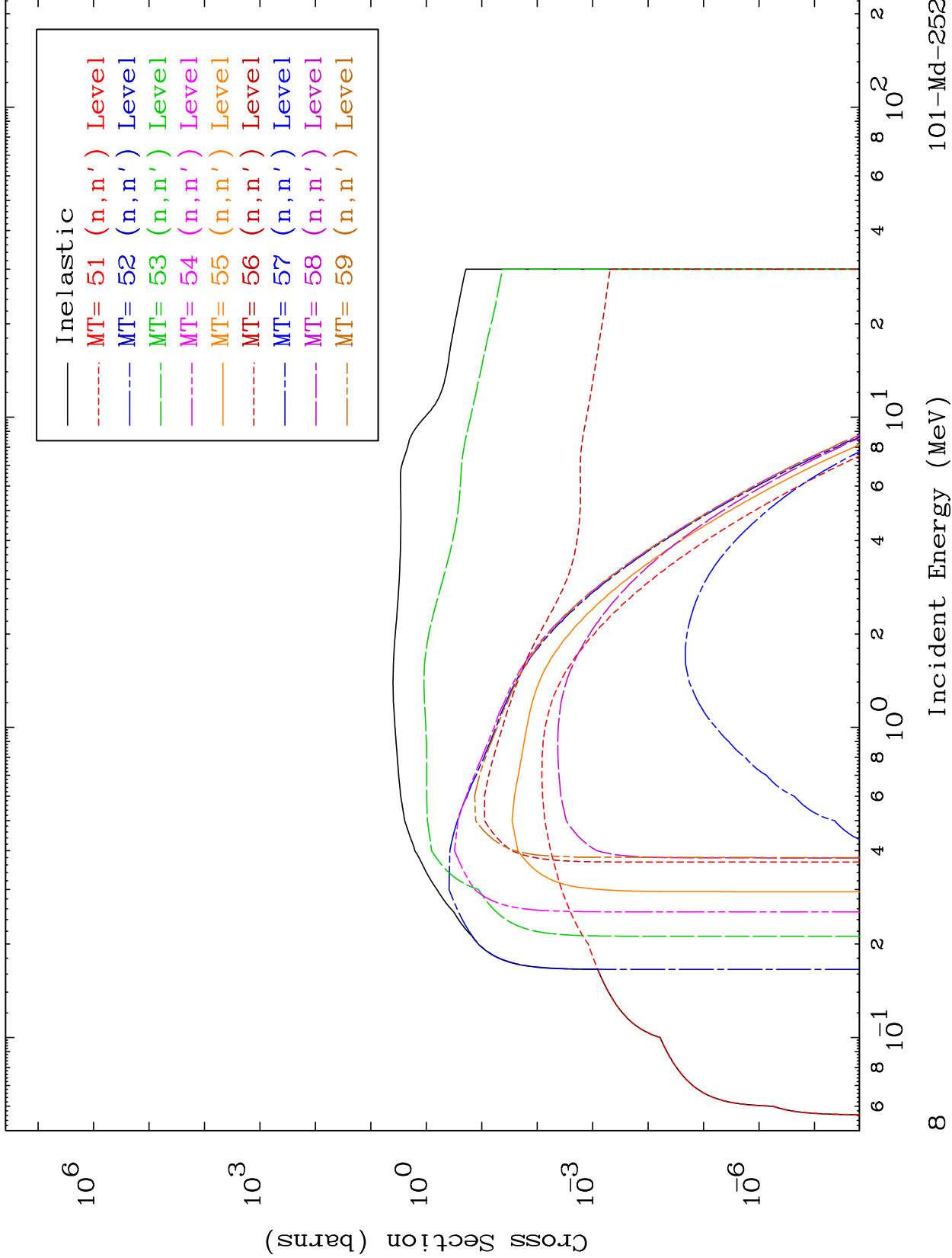


MAT 9953

(n,n') Level

101-Md-252

293 Kelvin Cross Sections

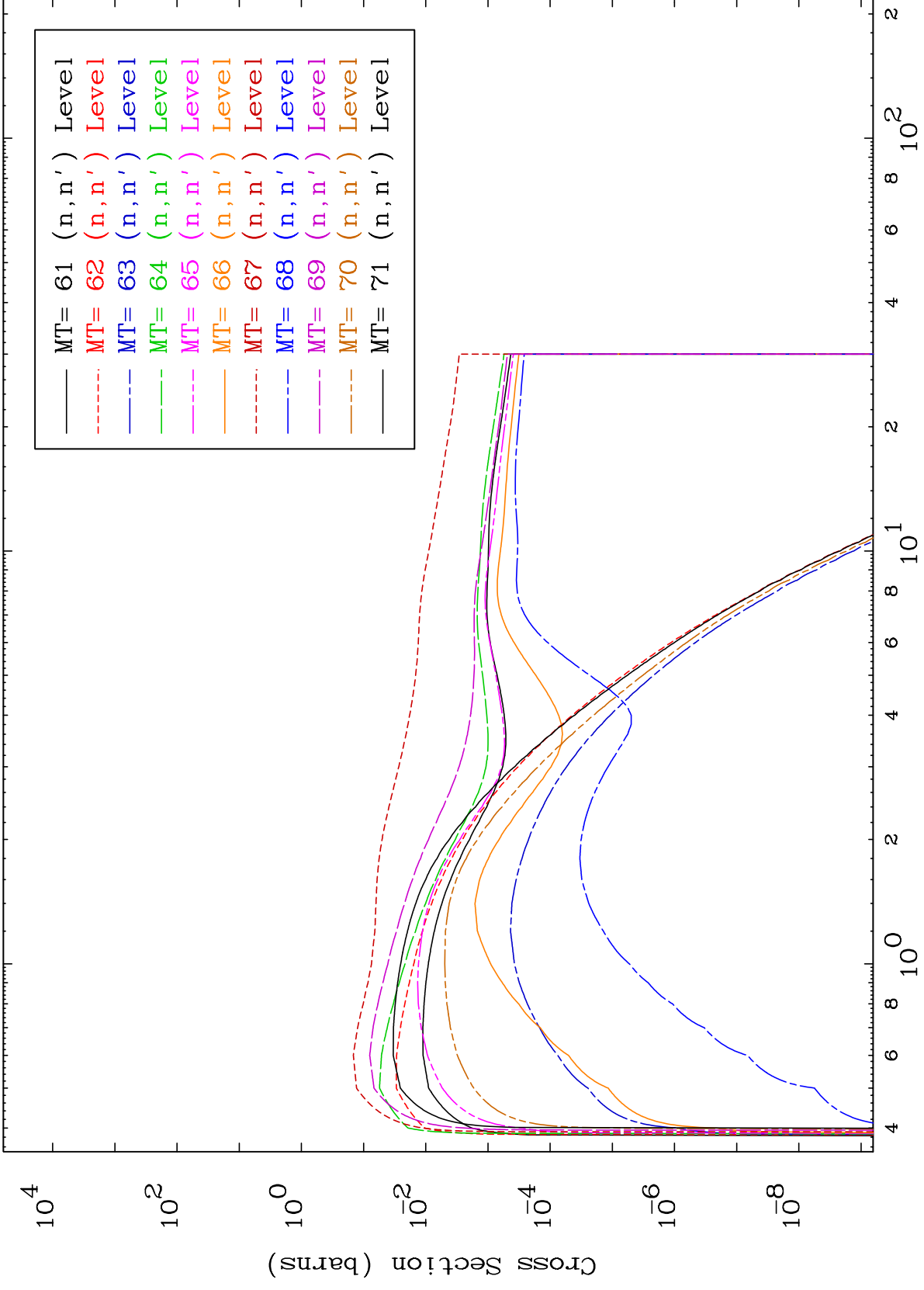


MAT 9953

(n,n') Level

101-Md-252

293 Kelvin Cross Sections



9

Incident Energy (MeV)

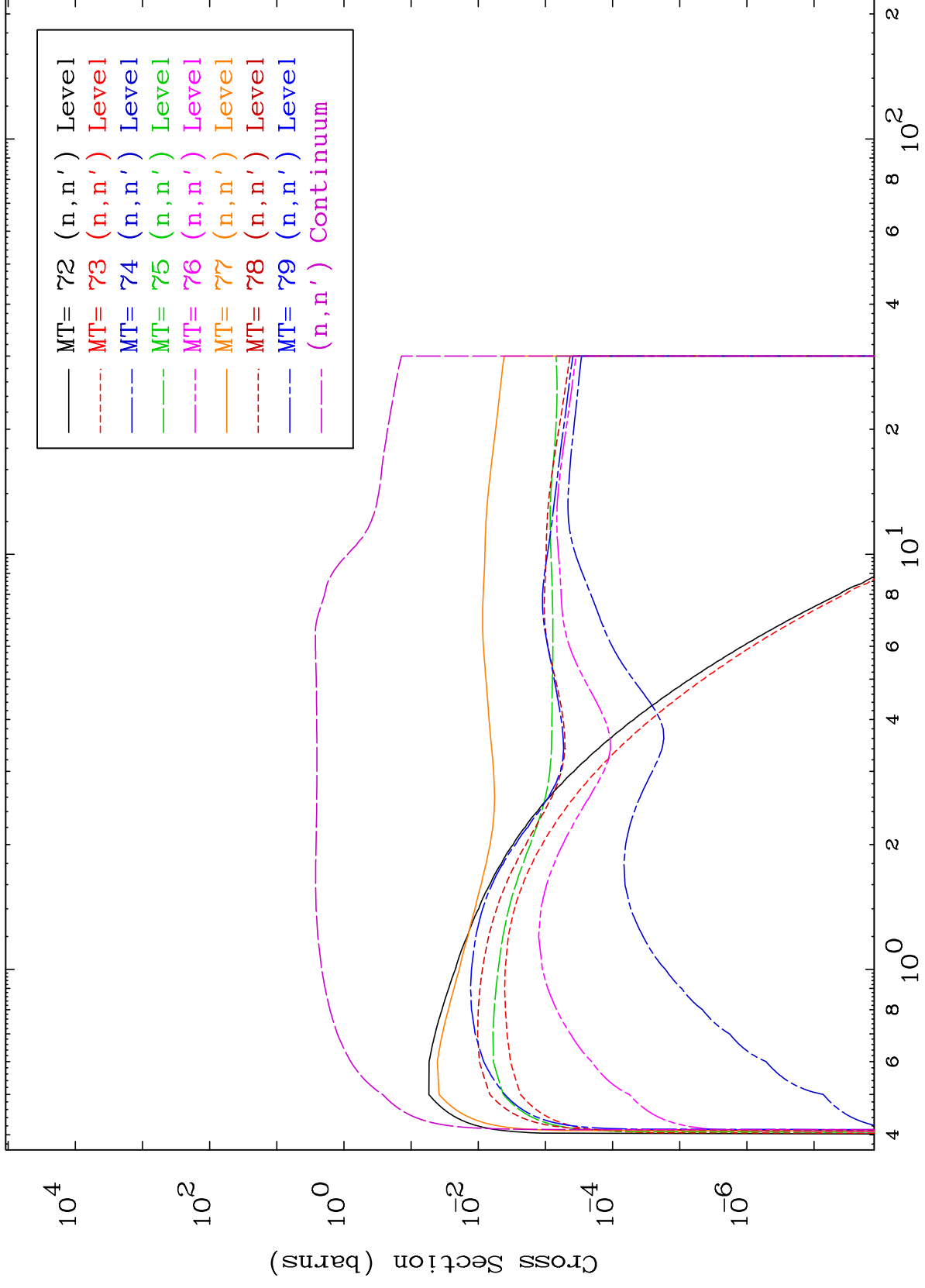
101-Md-252

MAT 9953

(n,n') Level

101-Md-252

293 Kelvin Cross Sections



10

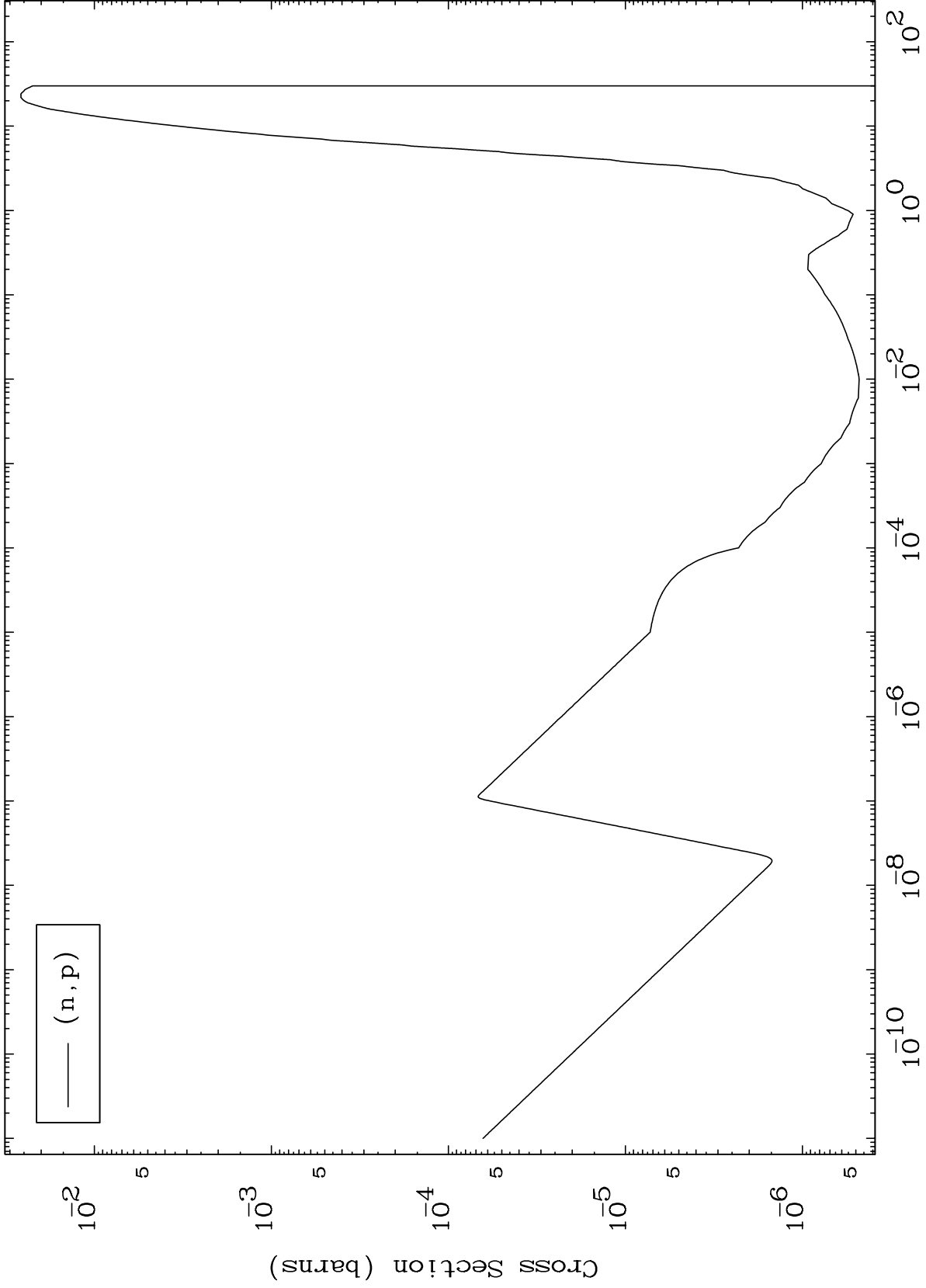
Incident Energy (MeV)

101-Md-252

MAT 9953

(n,p) Levels
293 Kelvin Cross Sections

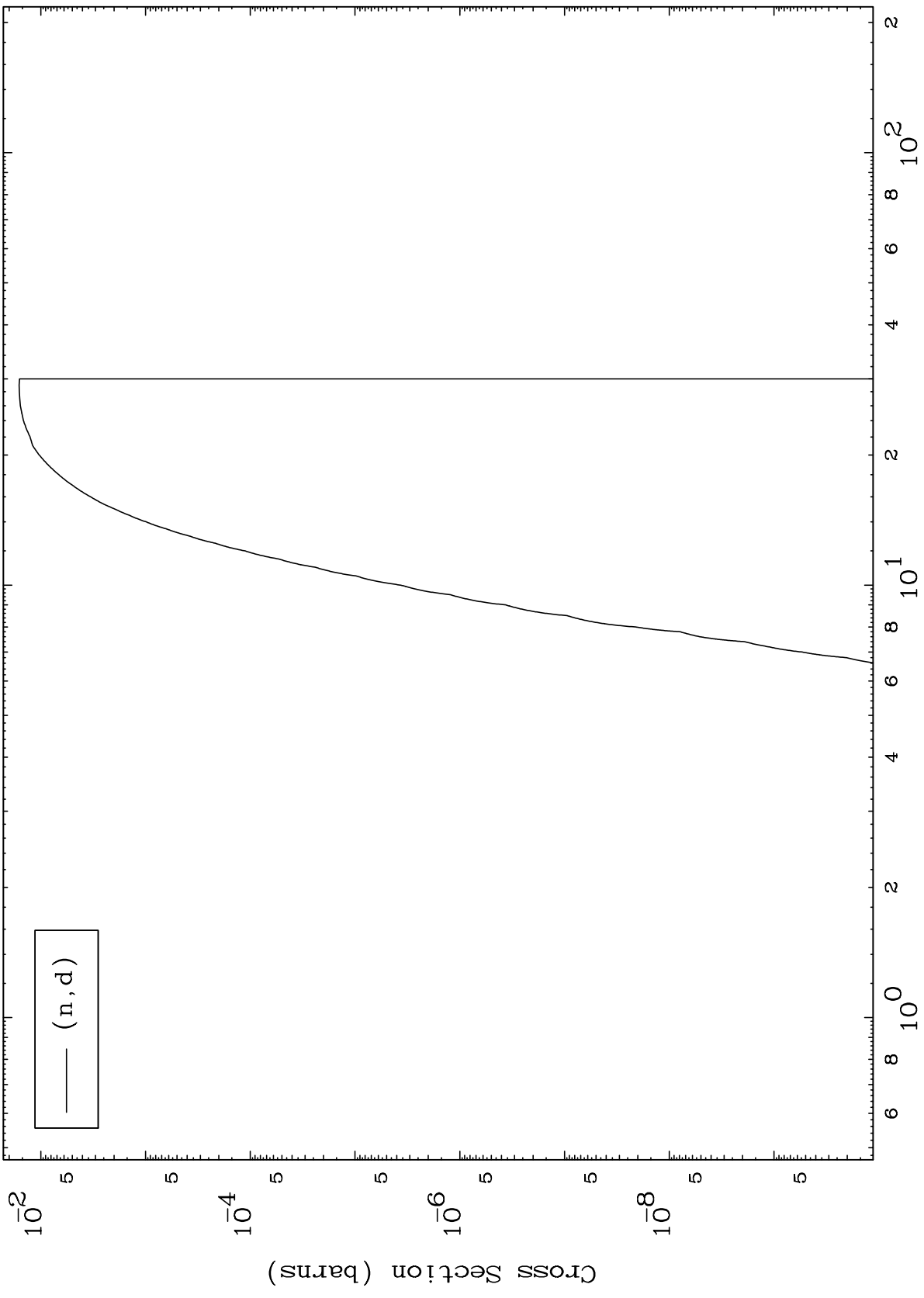
101-Md-252



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-252



12

Incident Energy (MeV)

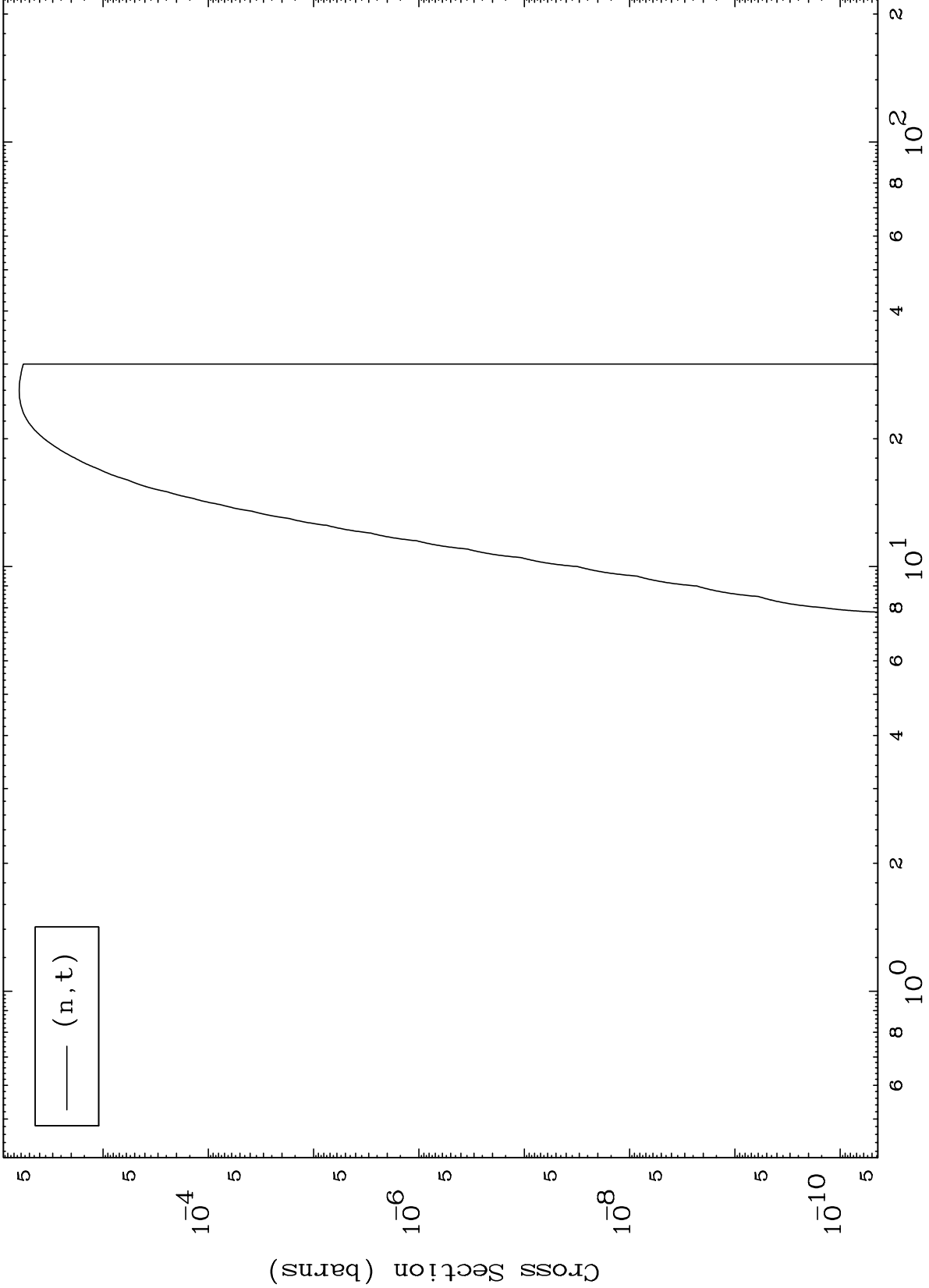
101-Md-252

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(n,t) Levels

101-Md-252

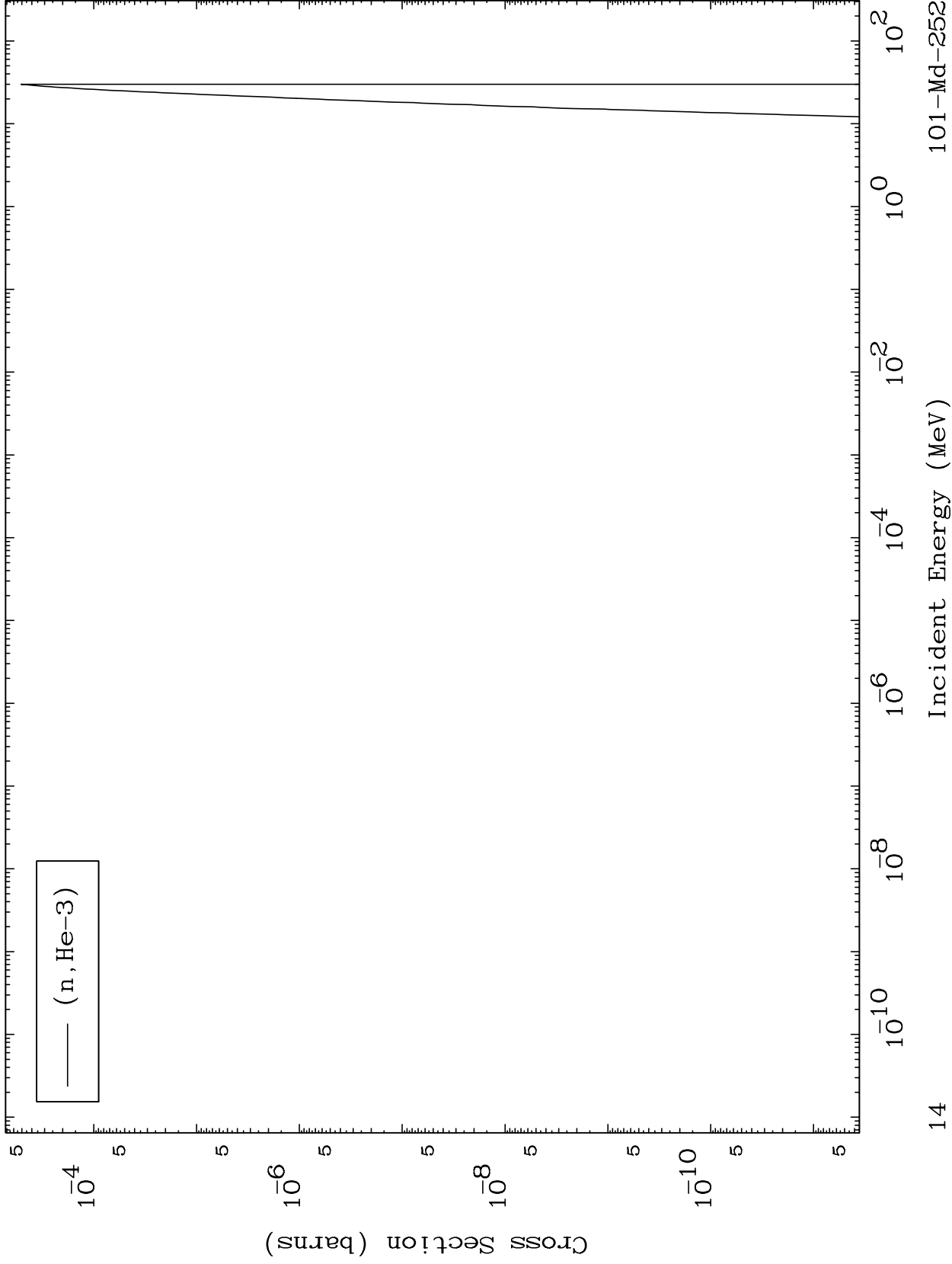
293 Kelvin Cross Sections



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(n,He3) Levels
293 Kelvin Cross Sections

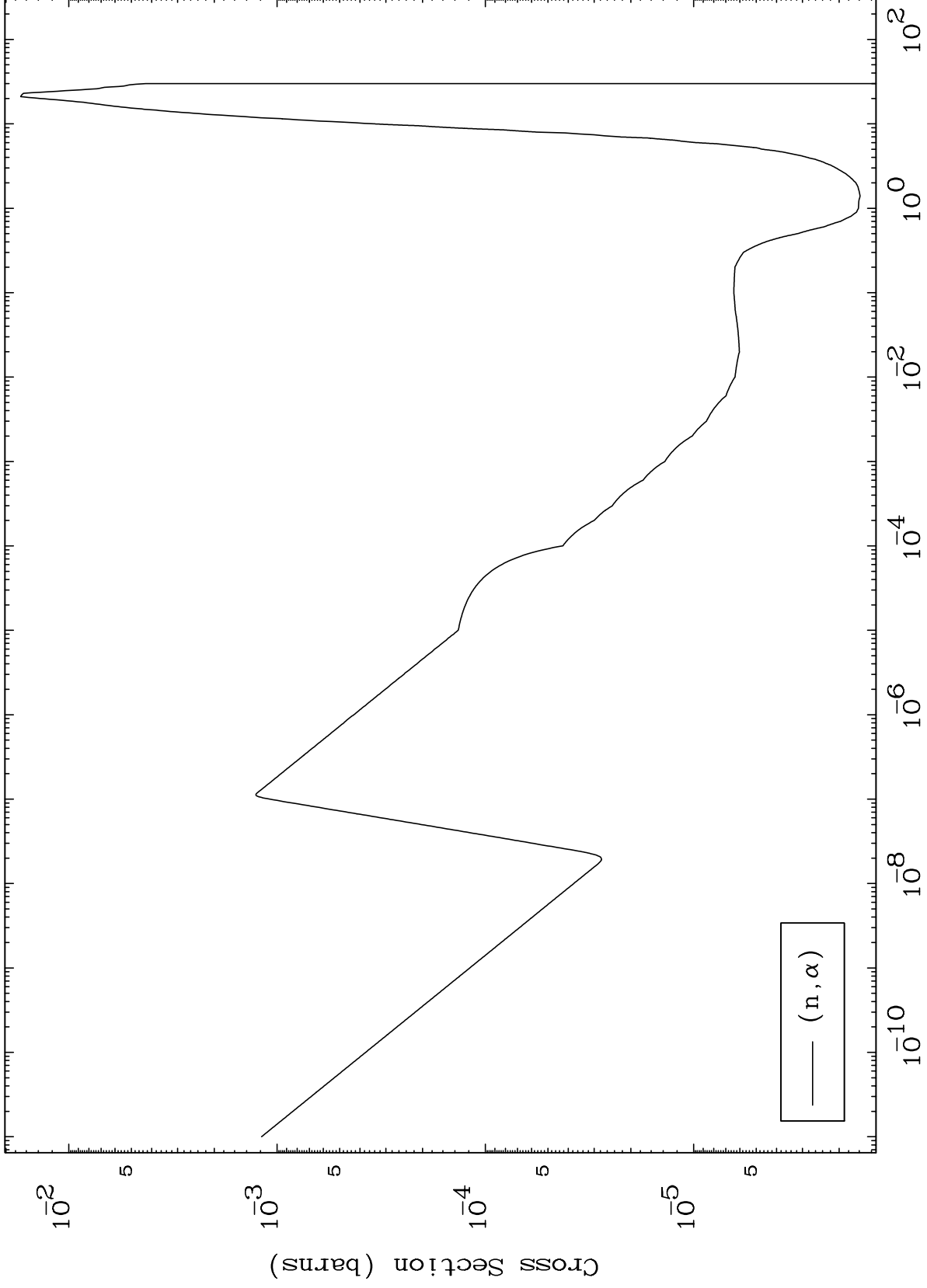
101-Md-252



MAT 9953

(n, α) Levels
293 Kelvin Cross Sections

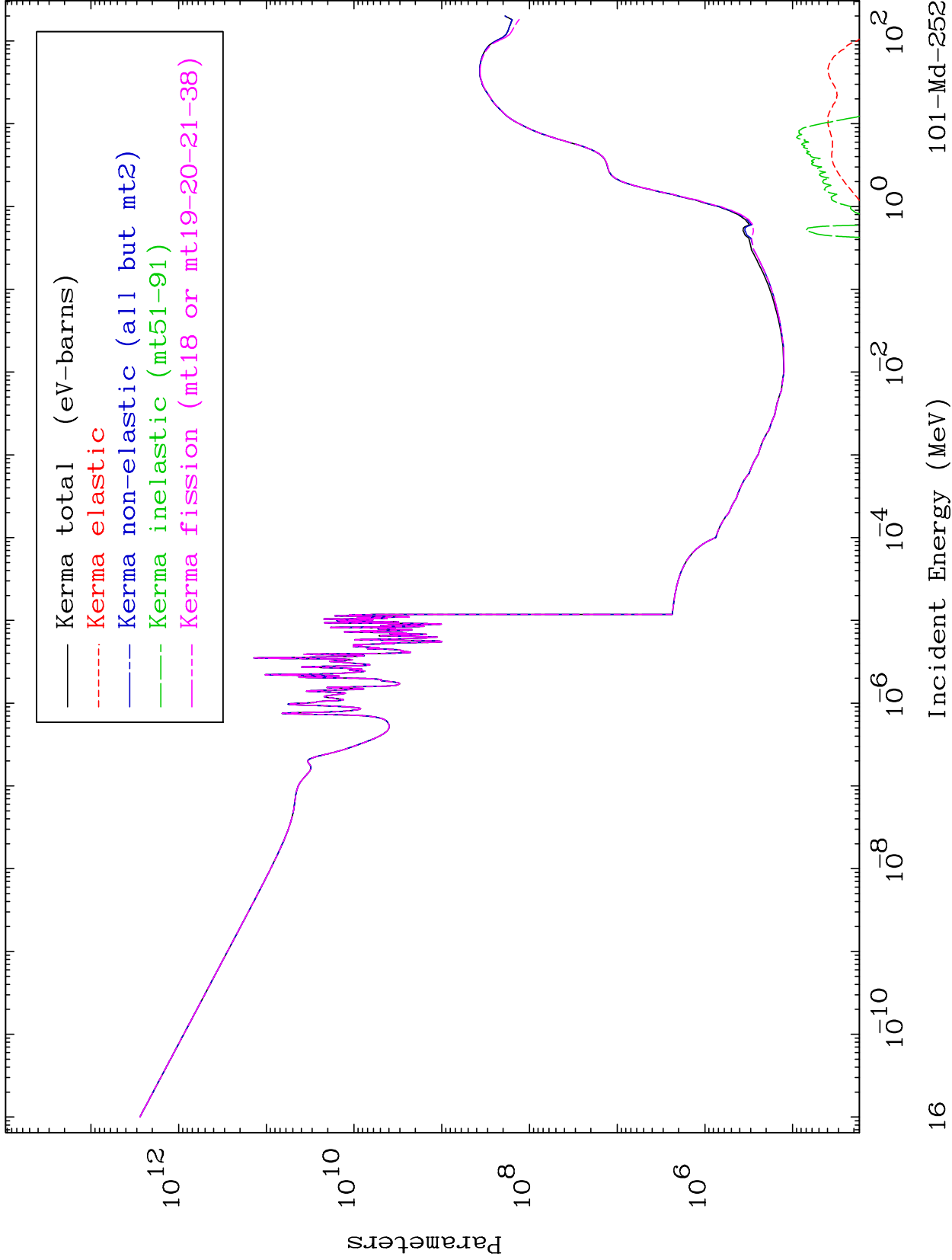
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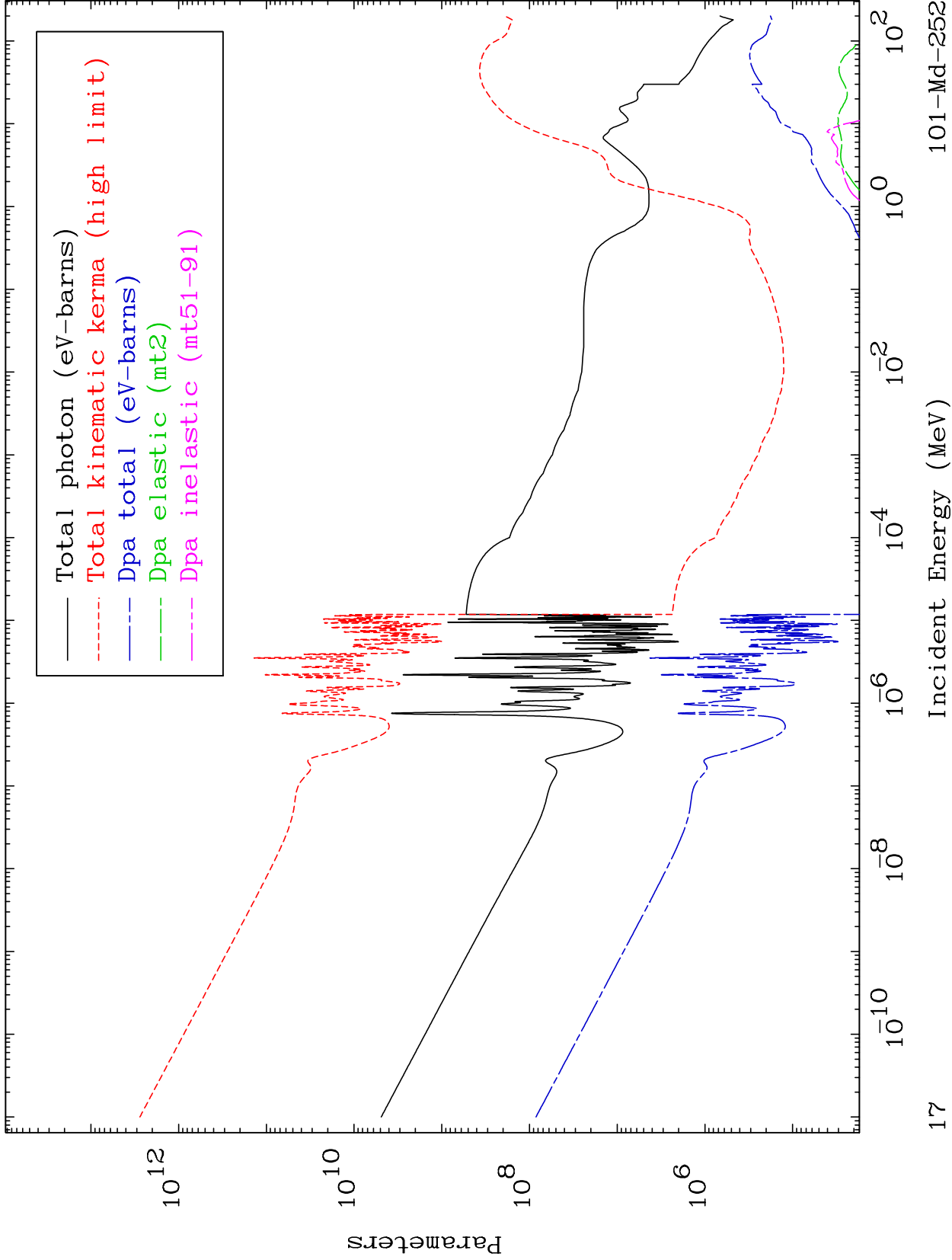


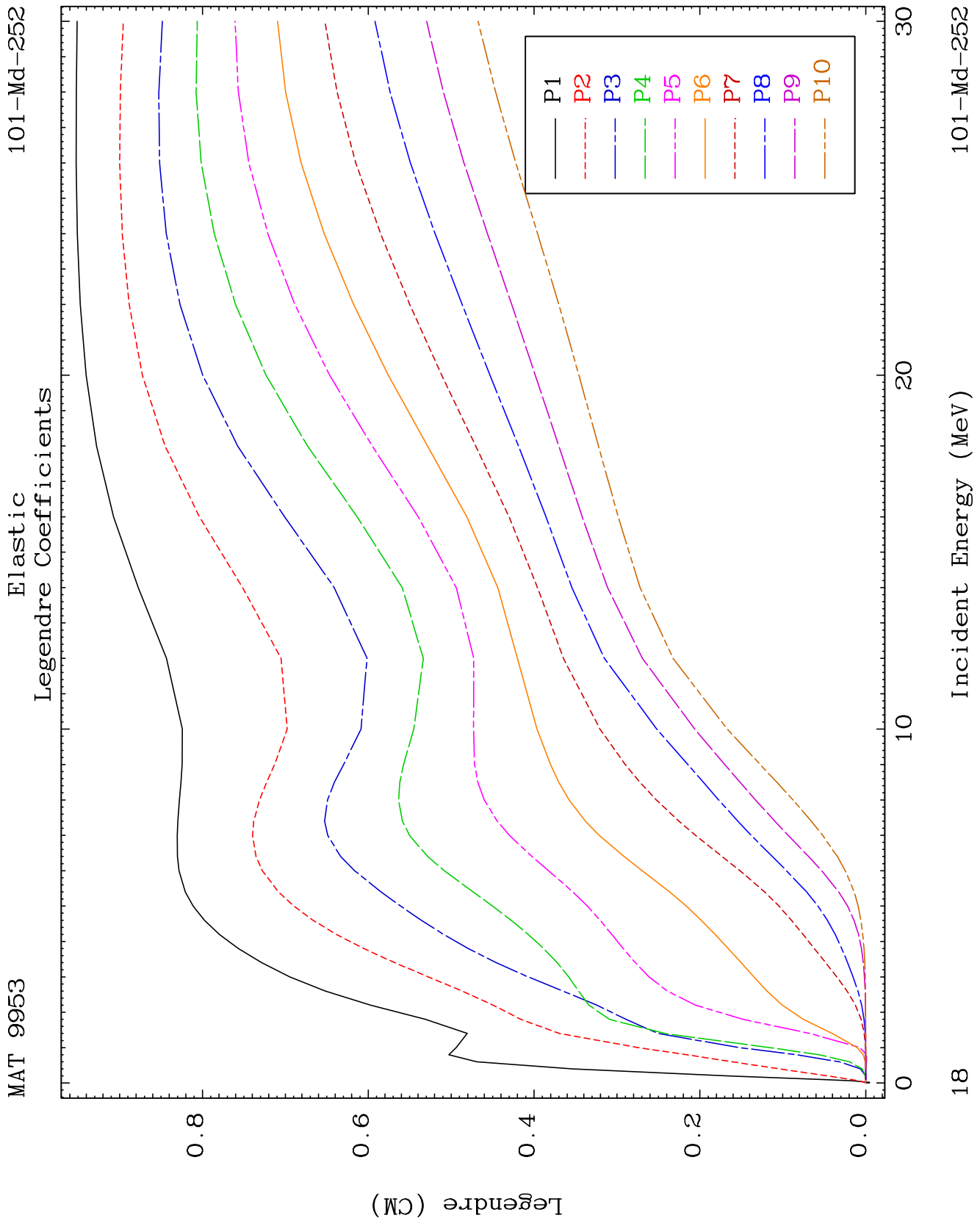
15

Incident Energy (MeV)

101-Md-252



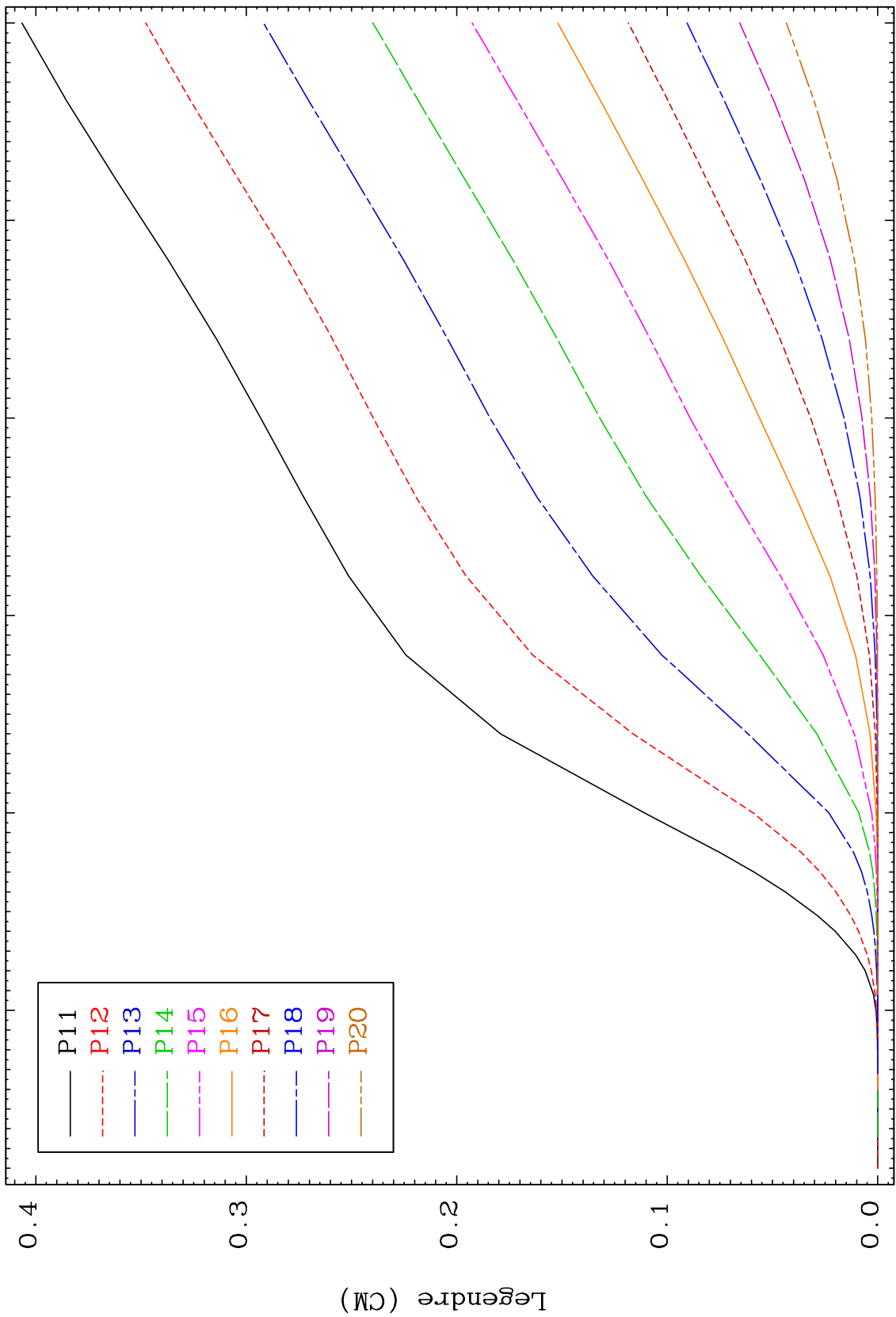




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Elastic Legendre Coefficients

101-Md-252



19

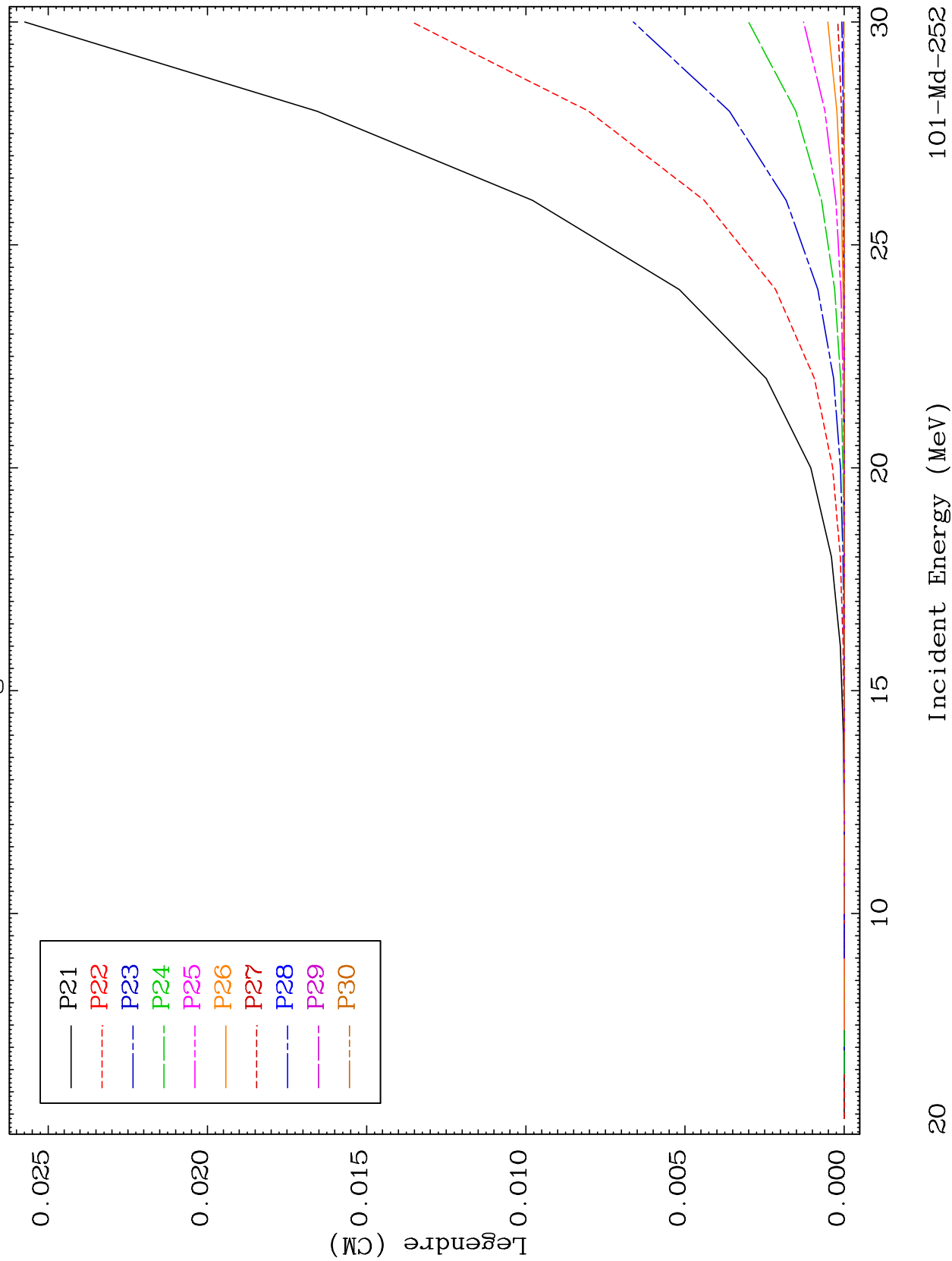
Incident Energy (MeV)

101-Md-252

MAT 9953

Elastic Legendre Coefficients

101-Md-252



20

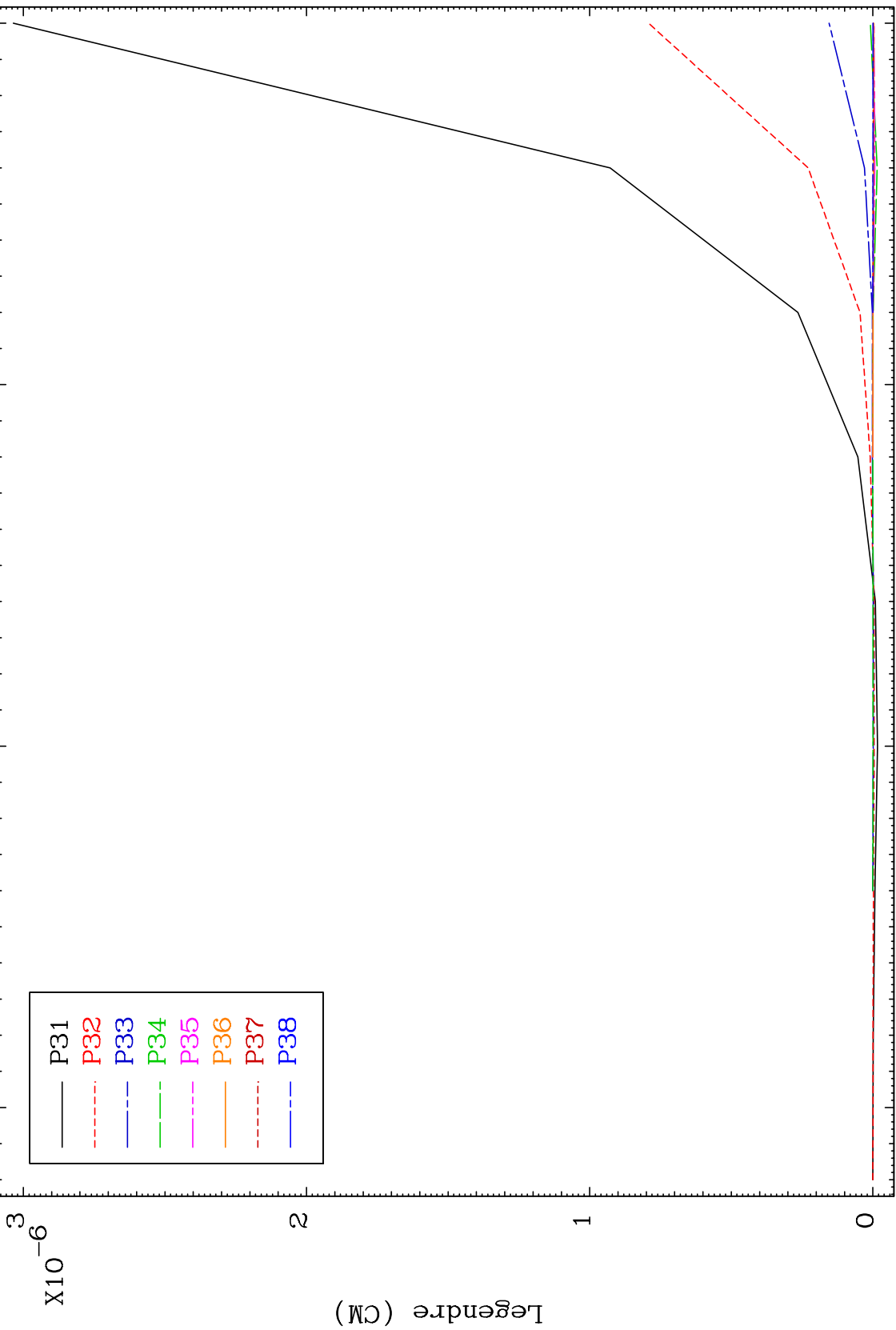
Incident Energy (MeV)

101-Md-252

MAT 9953

Elastic
Legendre Coefficients

101-Md-252



21

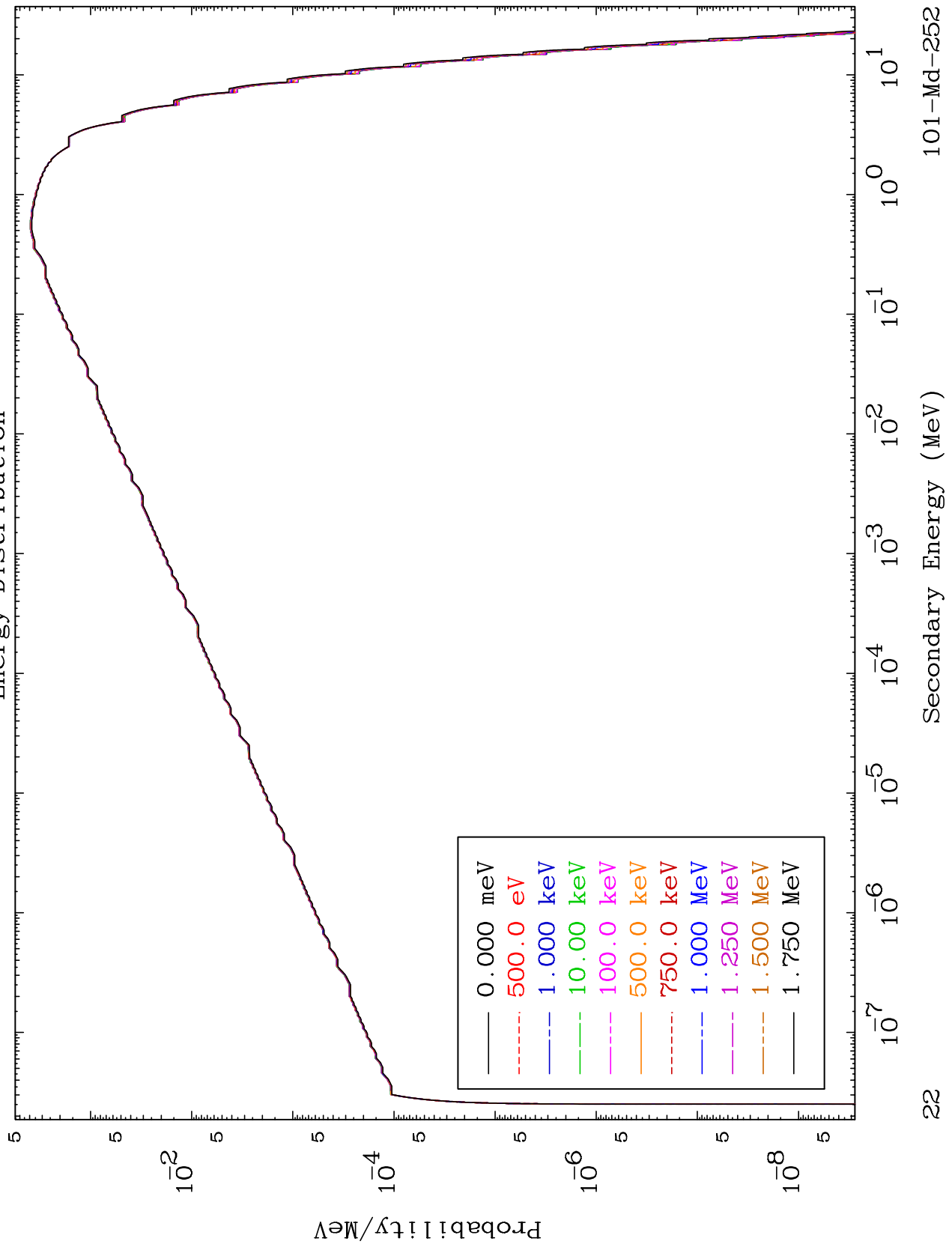
Incident Energy (MeV)

101-Md-252

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Fission Energy Distribution

101-Md-252

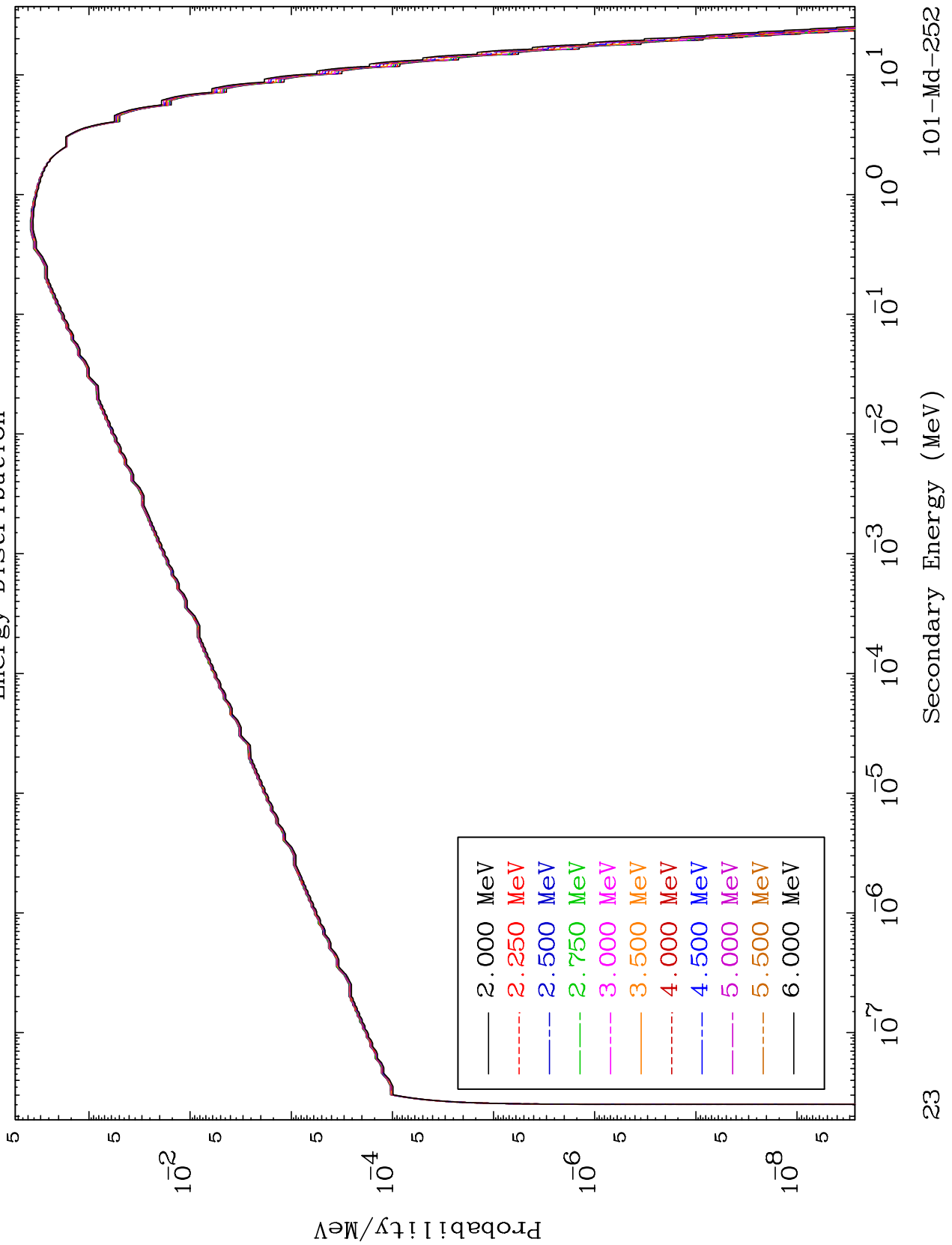


101-Md-252

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Fission Energy Distribution

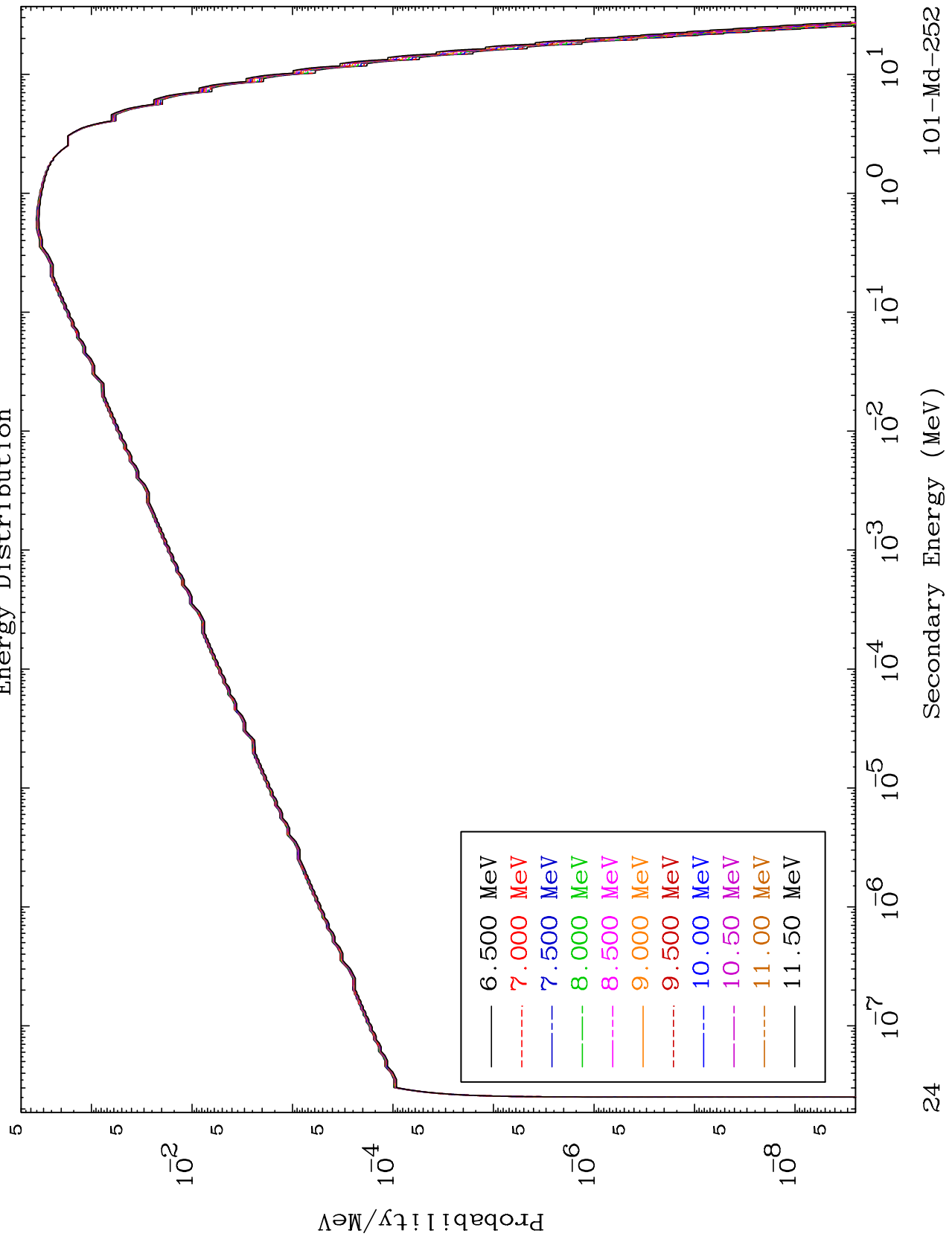
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Fission
Energy Distribution

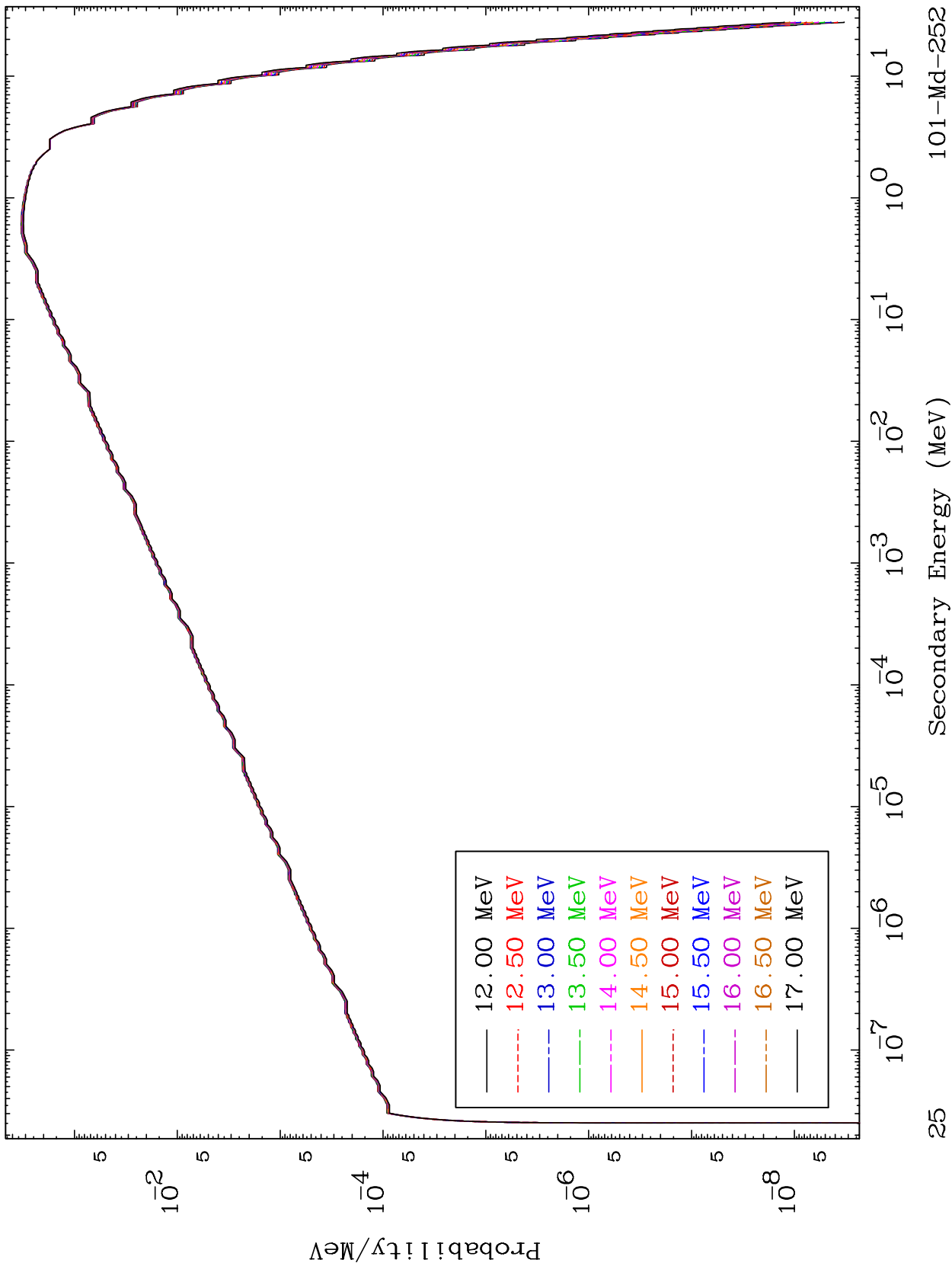
101-Md-252



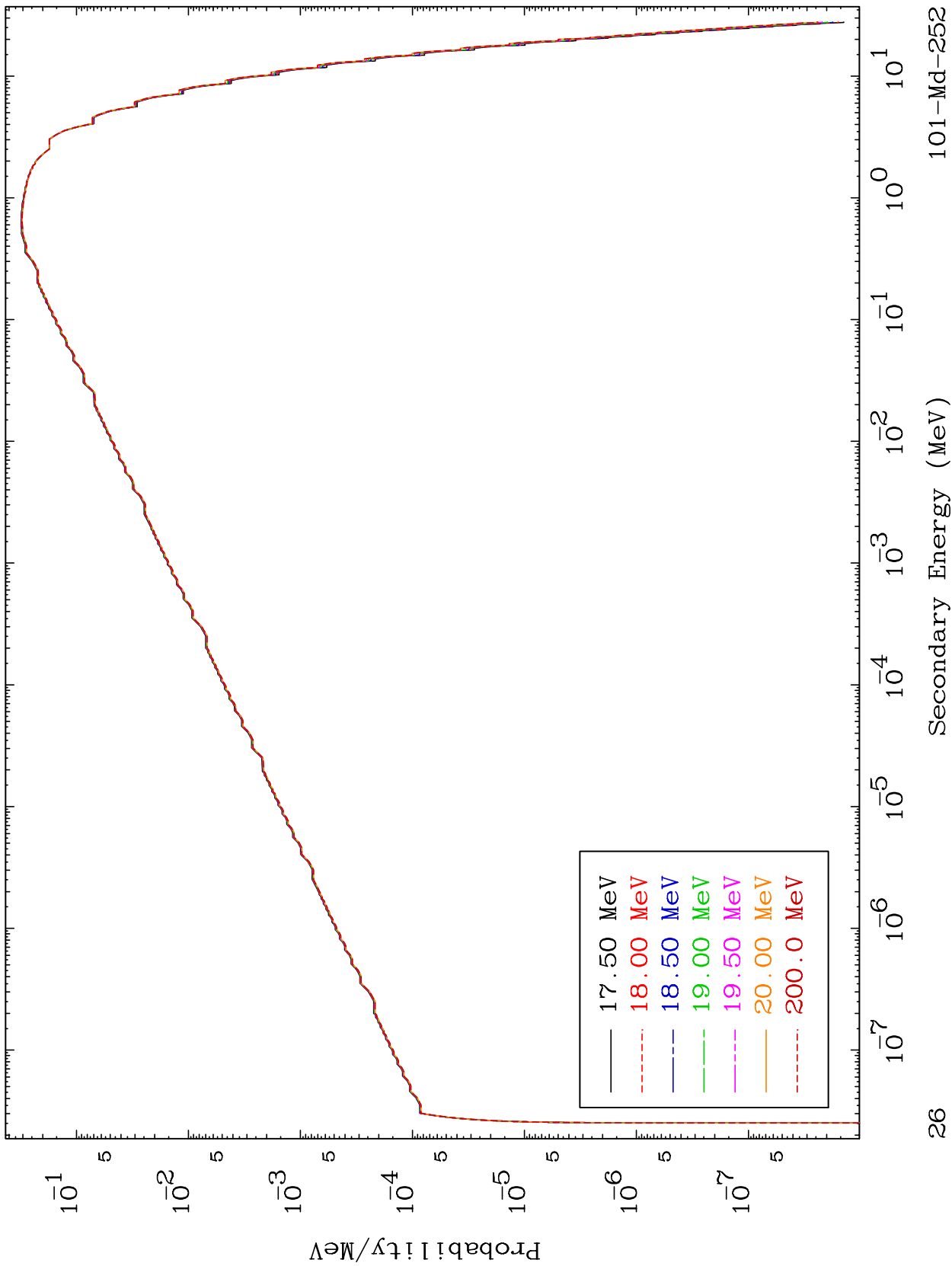
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Fission Energy Distribution

101-Md-252



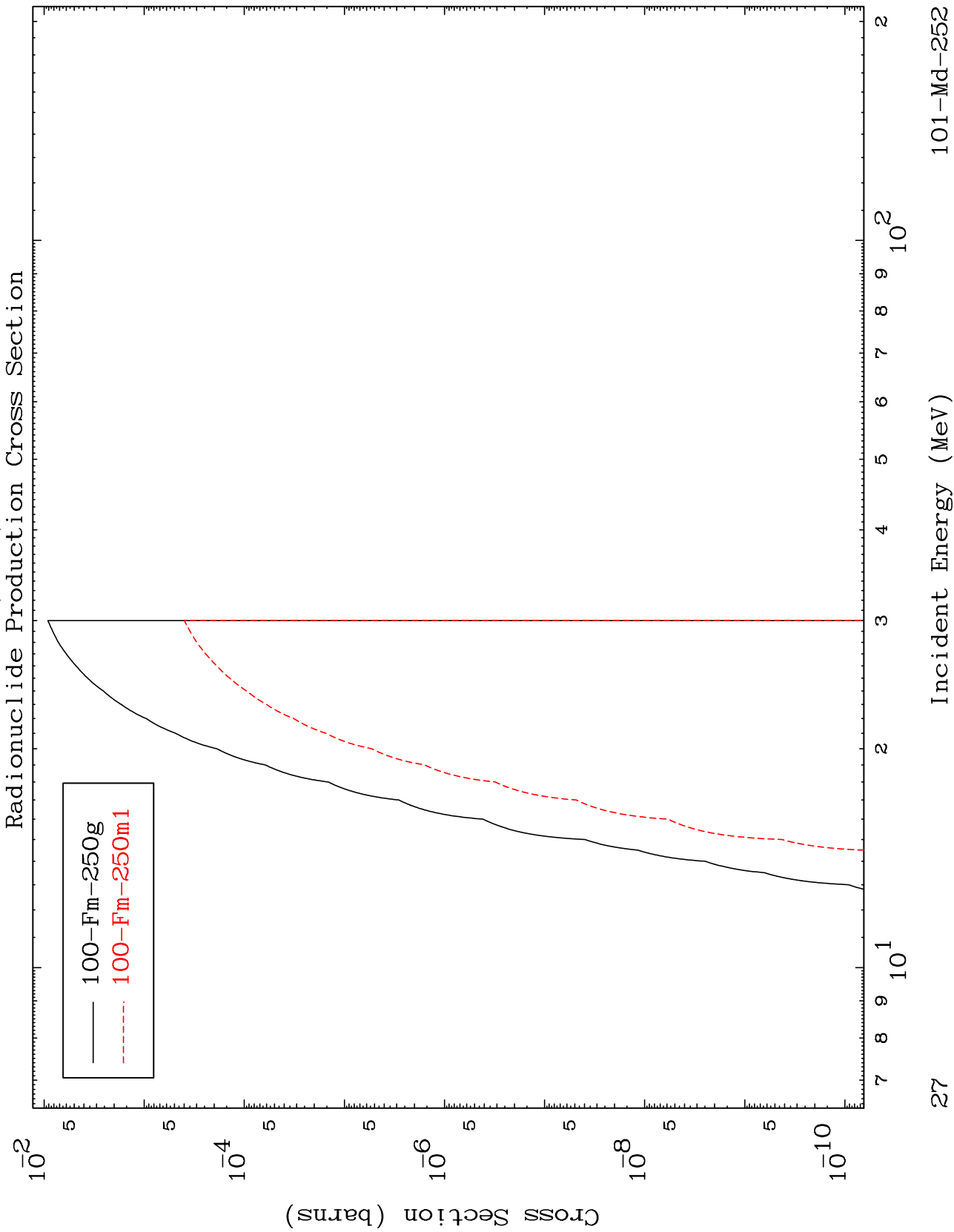
Fission Energy Distribution



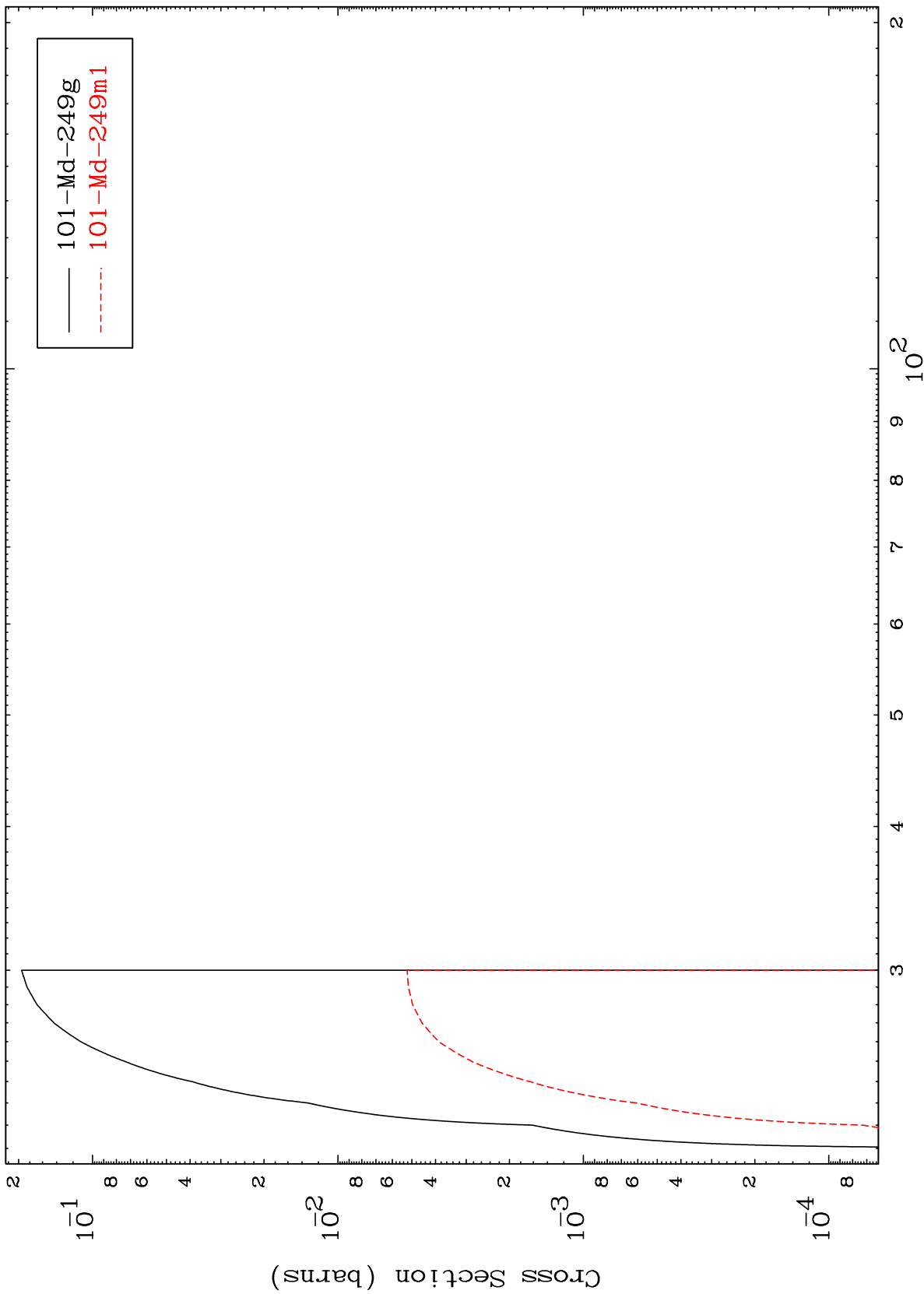
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(n,n') d

101-Md-252



(n,4n)
Radionuclide Production Cross Section

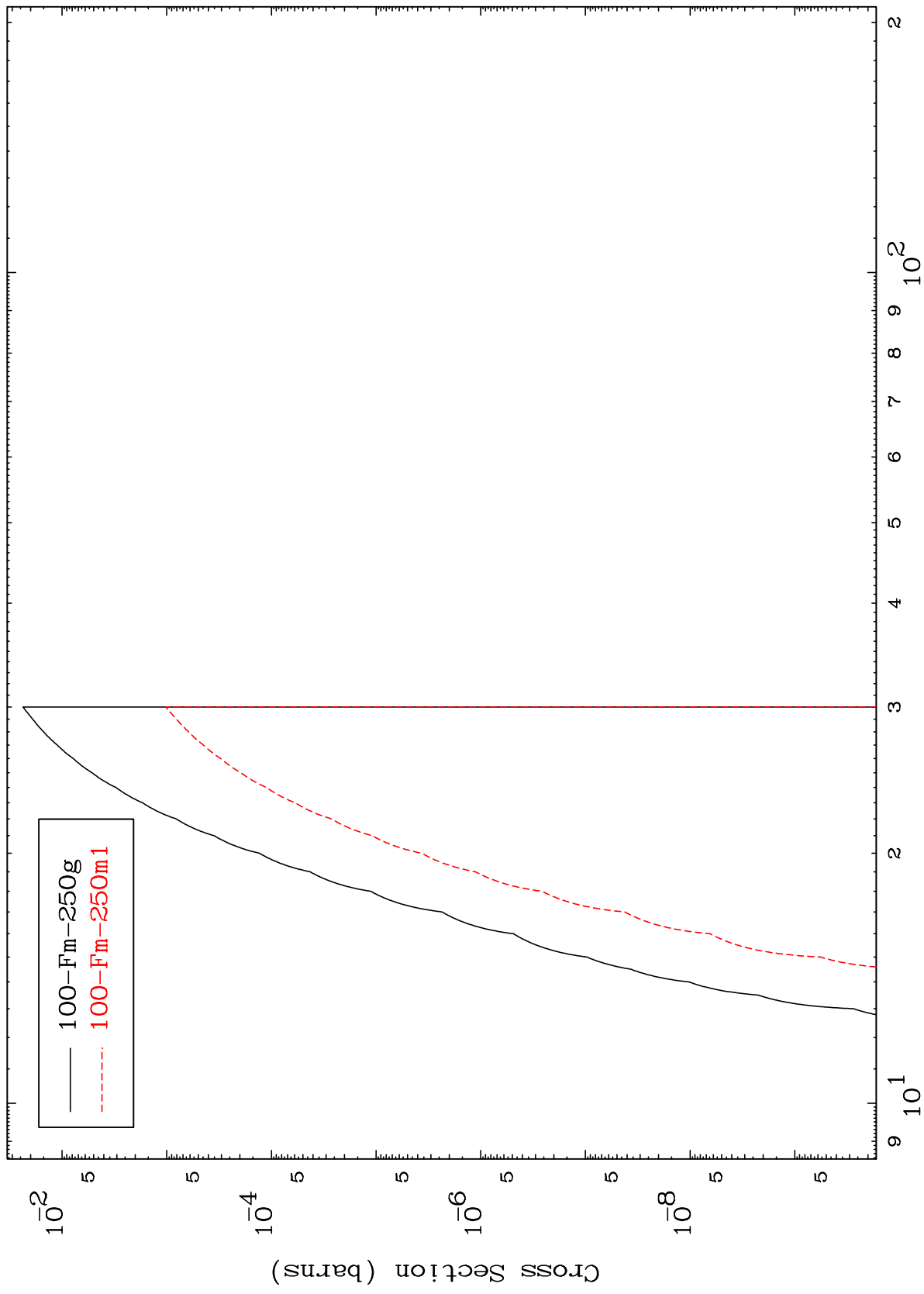


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(n,2n) p

101-Md-252

Radionuclide Production Cross Section



29

Incident Energy (MeV)

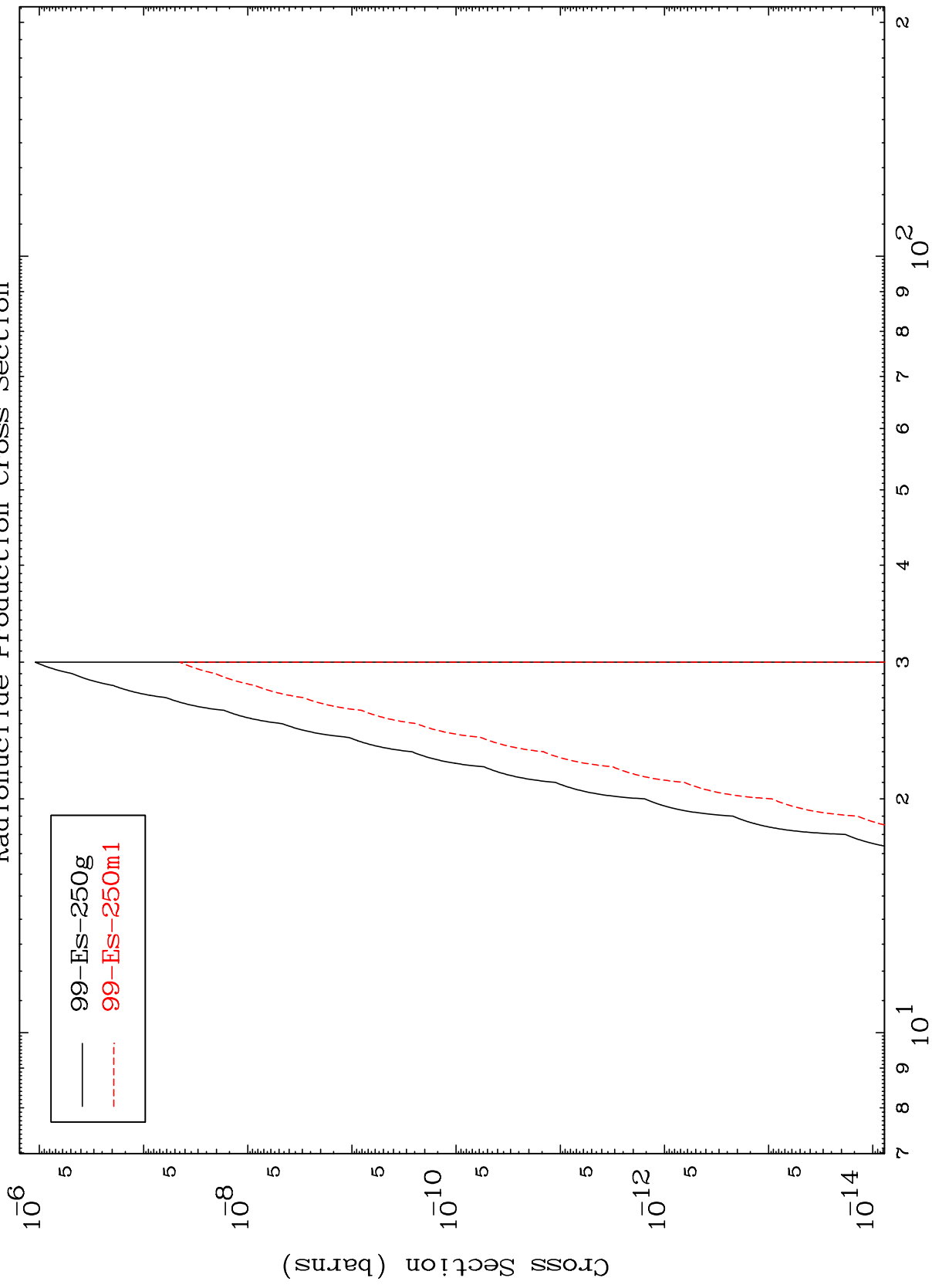
101-Md-252

MAT 9953

(n,2n) p

101-Md-252

Radionuclide Production Cross Section



30

Incident Energy (MeV)

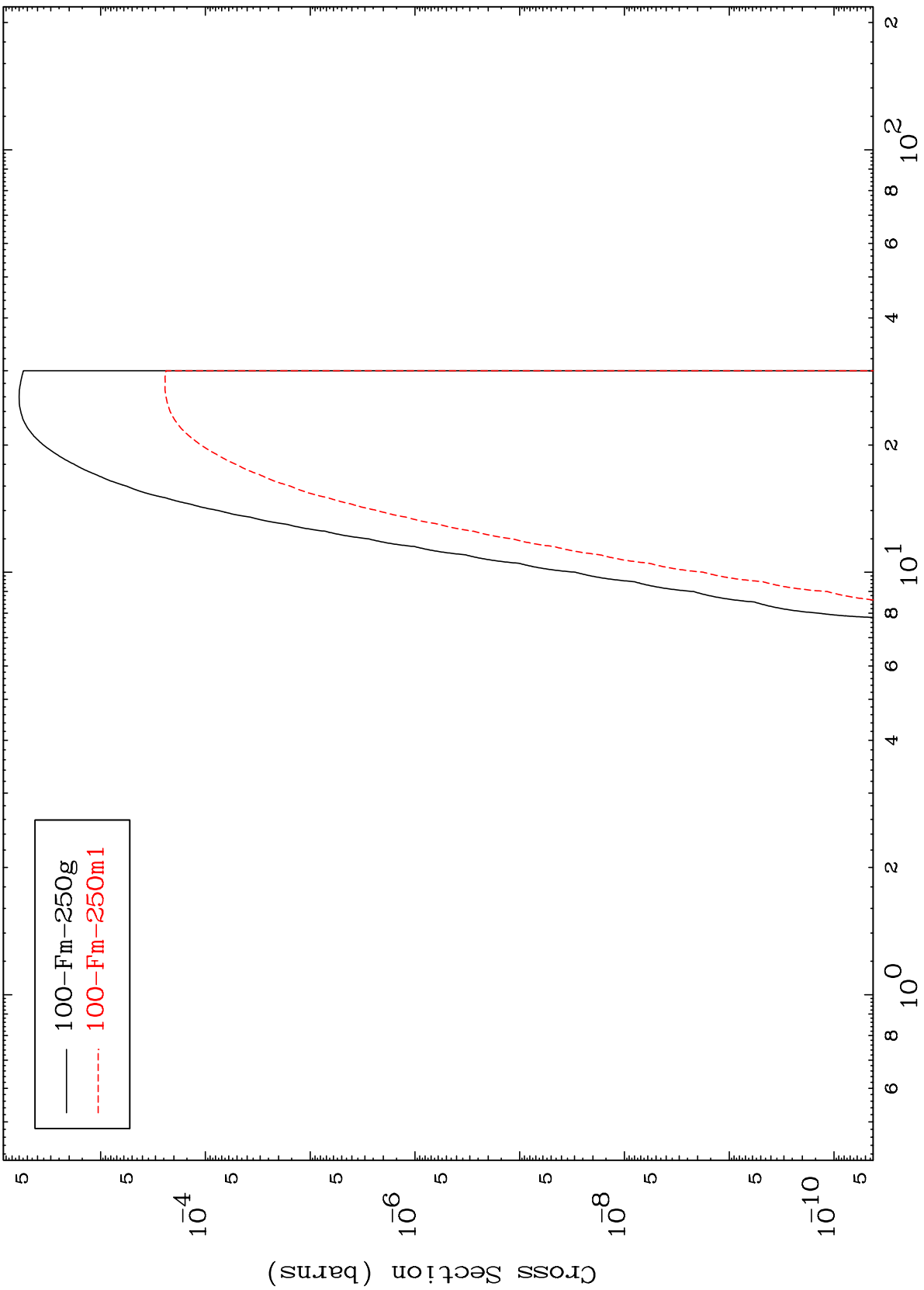
101-Md-252

MAT 9953

(n, t)

101-Md-252

Radionuclide Production Cross Section

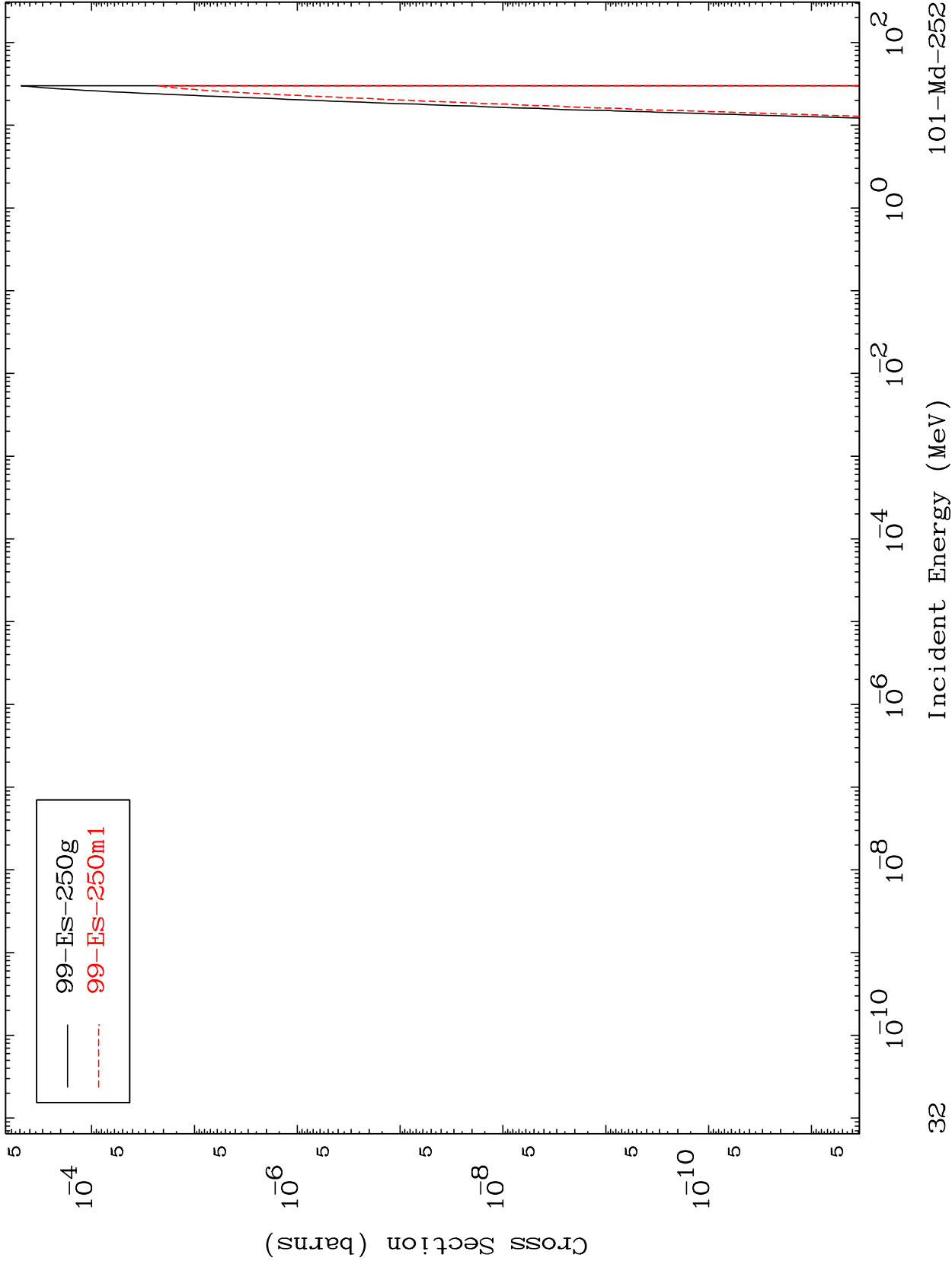


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(n,He-3)

101-Md-252

Radionuclide Production Cross Section



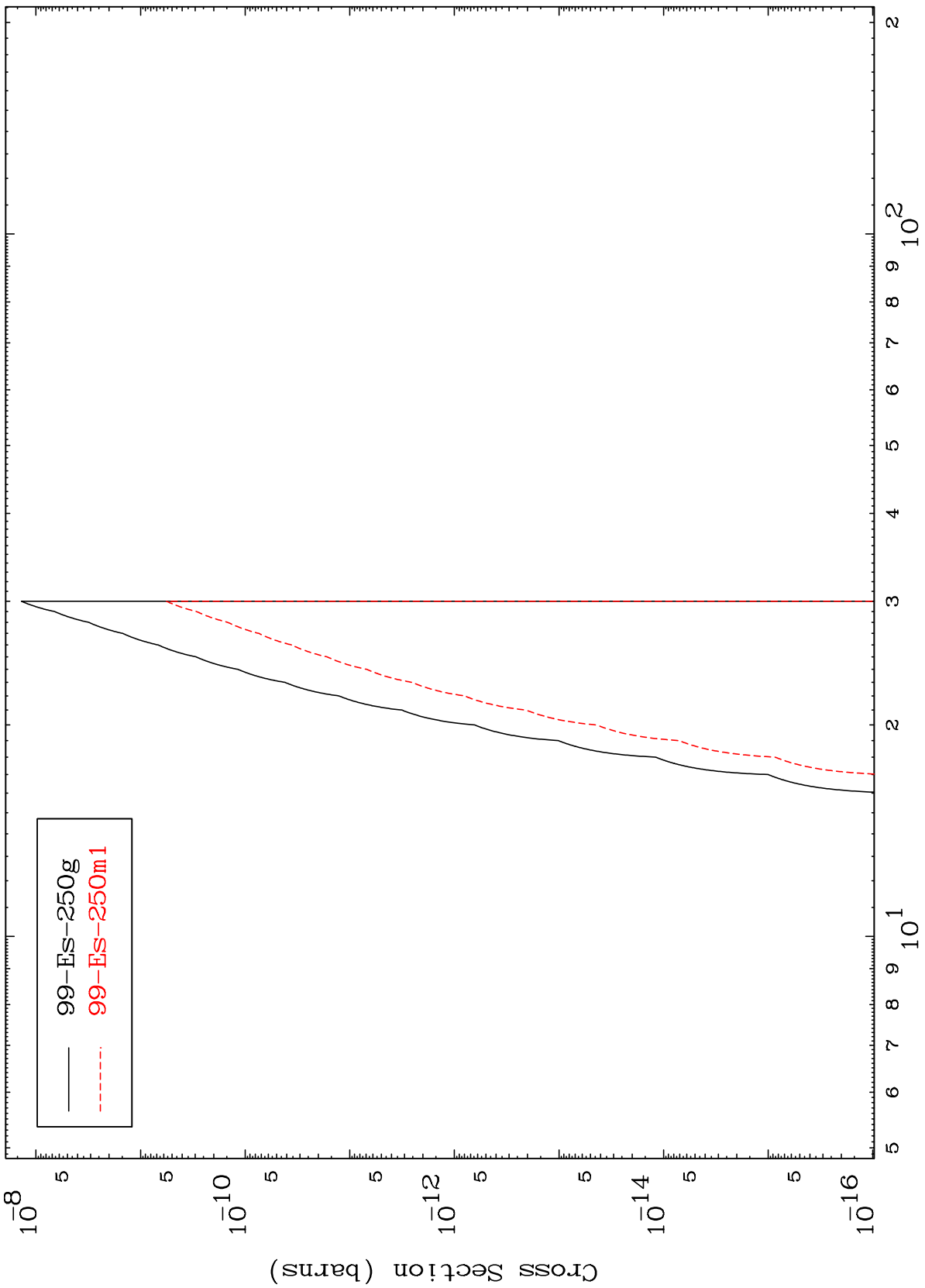
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- - - 99-Es-250m1

MAT 9953

(n,p) d

101-Md-252

Radionuclide Production Cross Section



99-Es-250g
99-Es-250m1